

Program EVALPLOT
(Version 2021-1)

by

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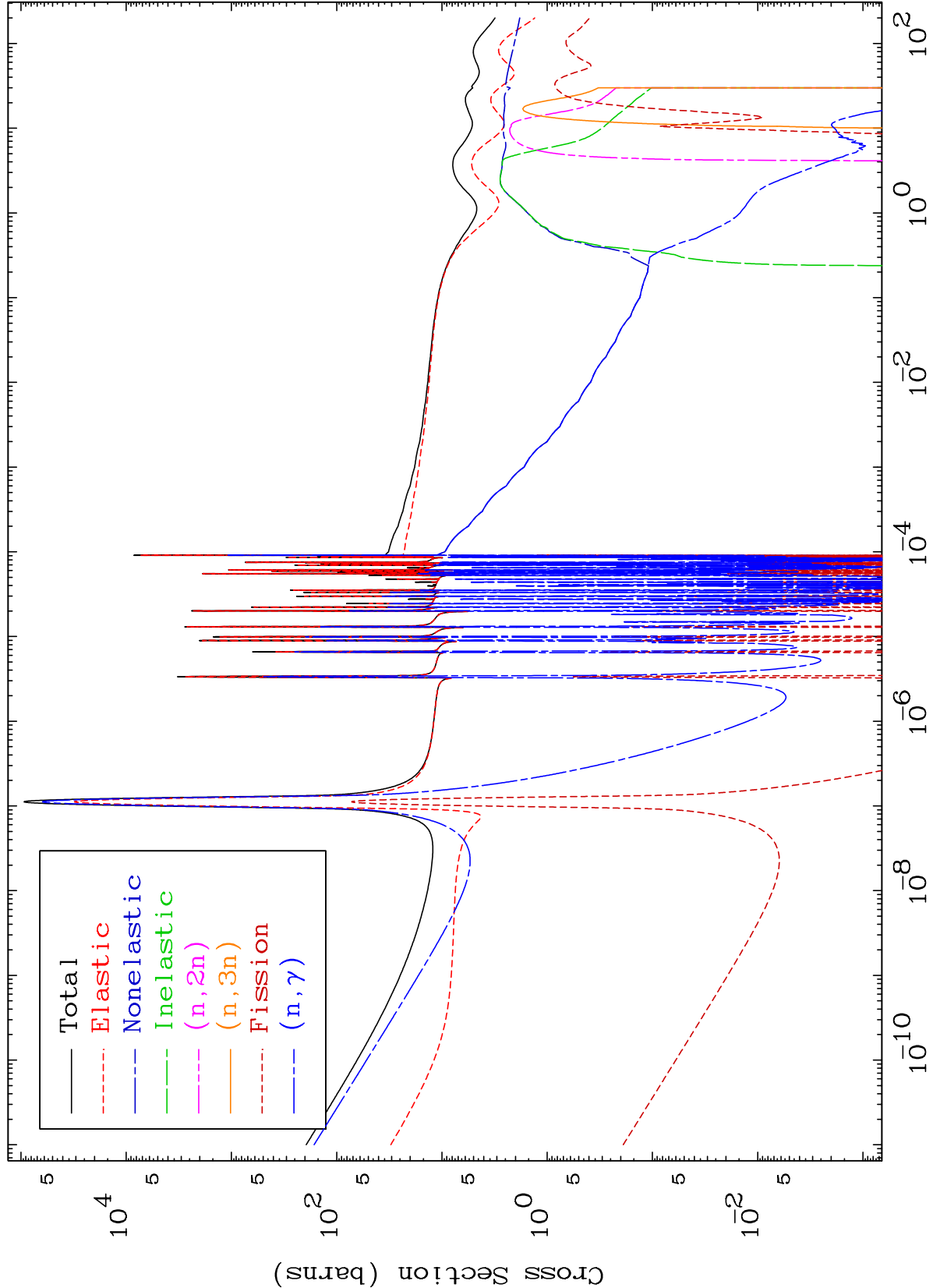
Press Mouse Button to Start

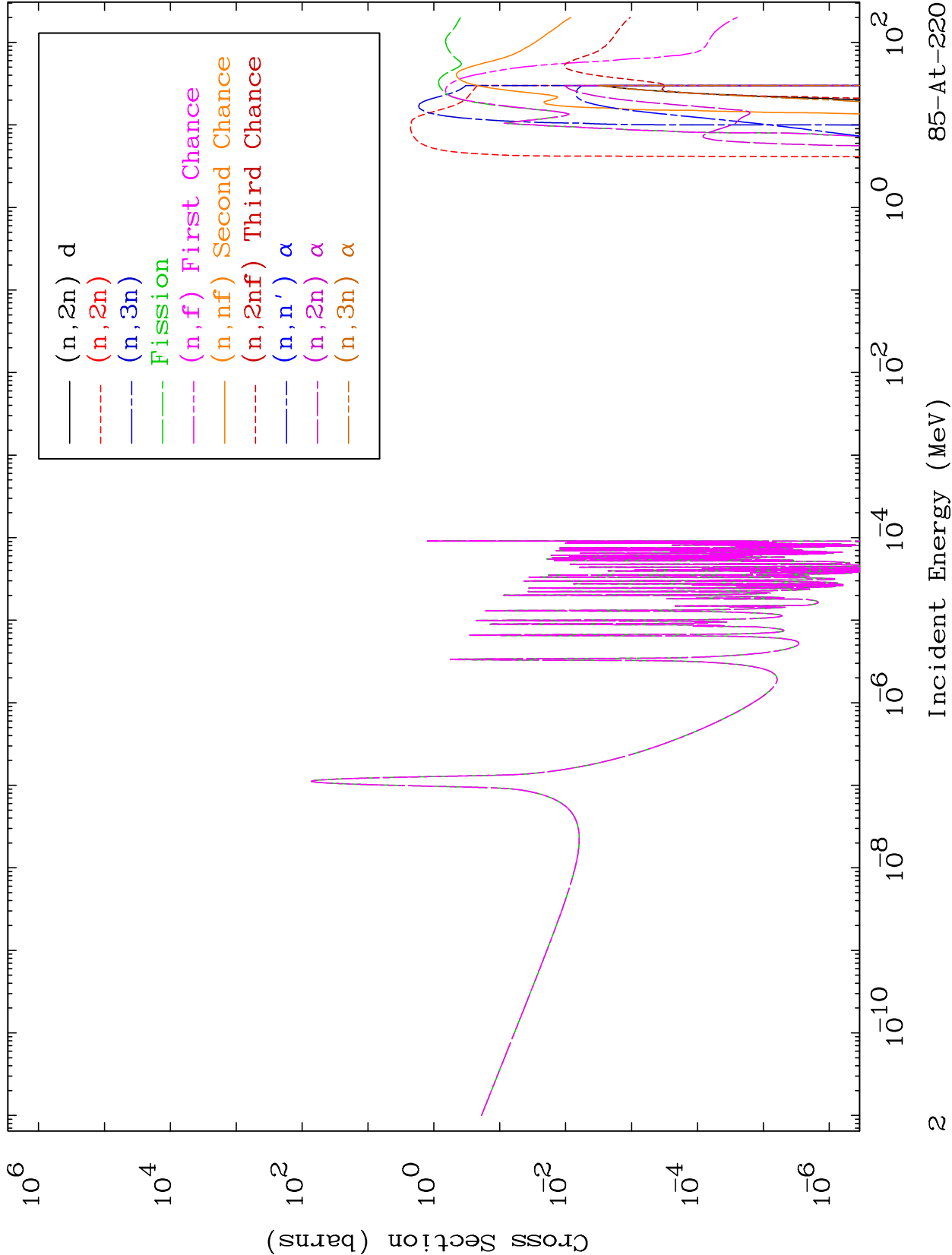
MAT 8576

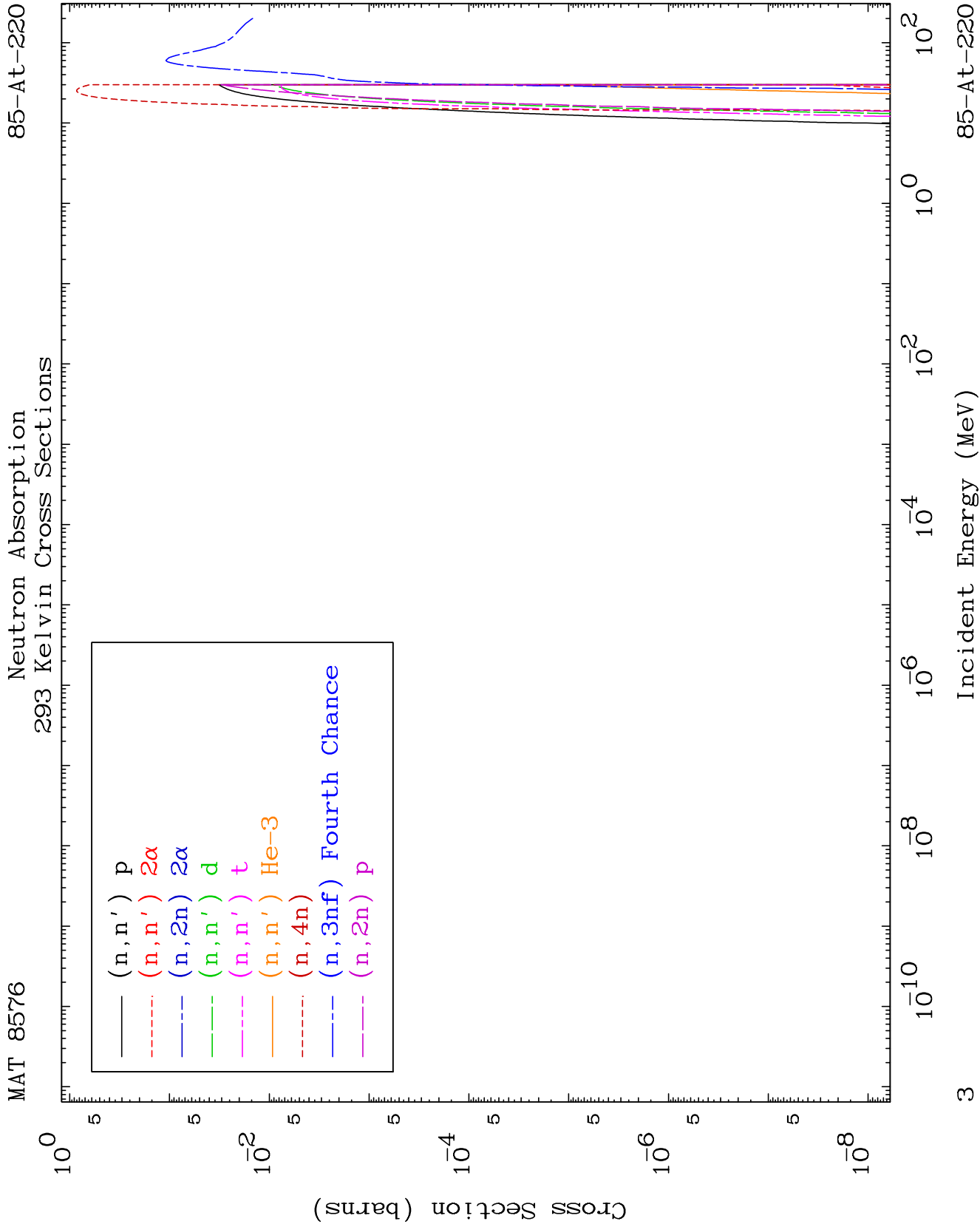
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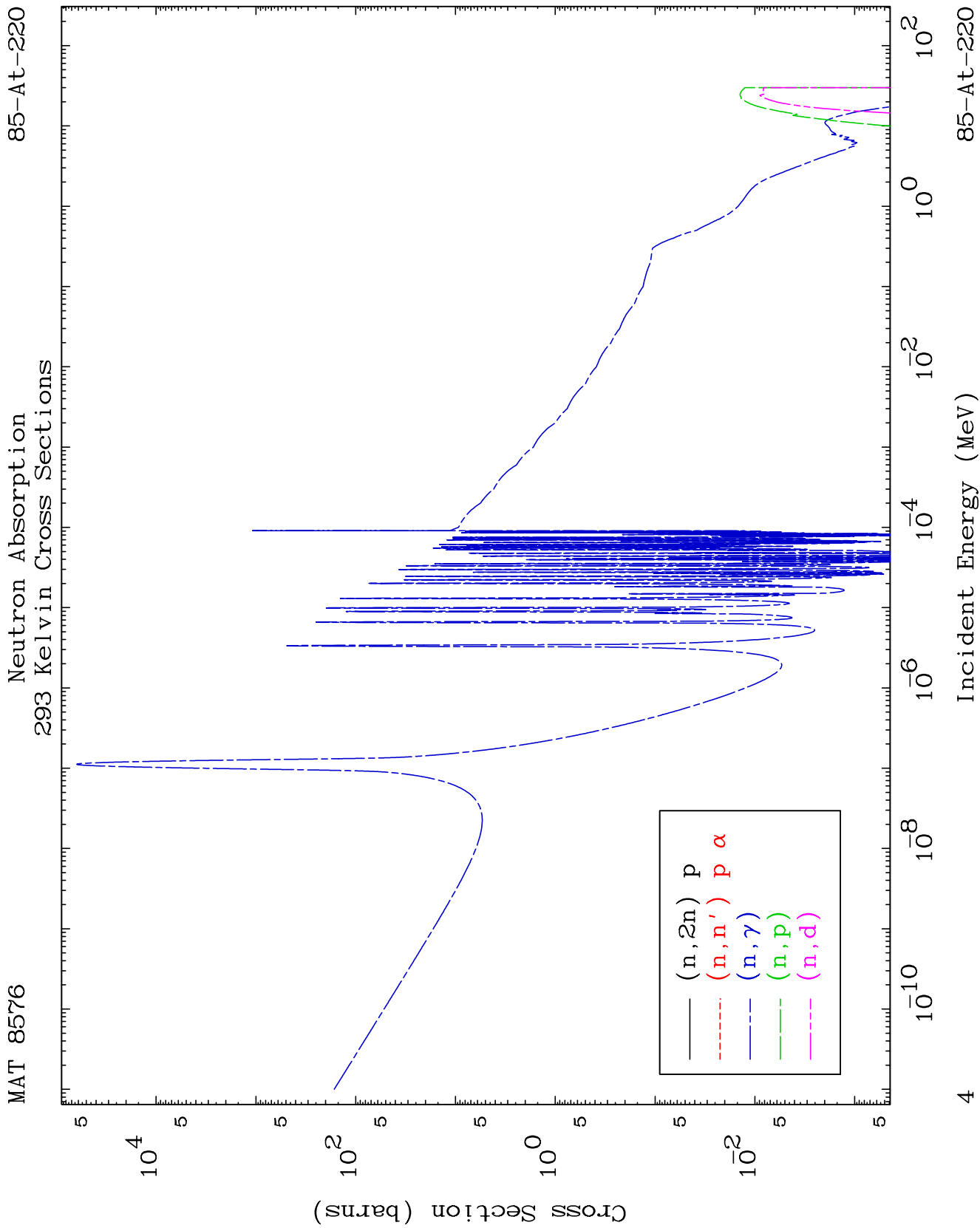
293 Kelvin Cross Sections

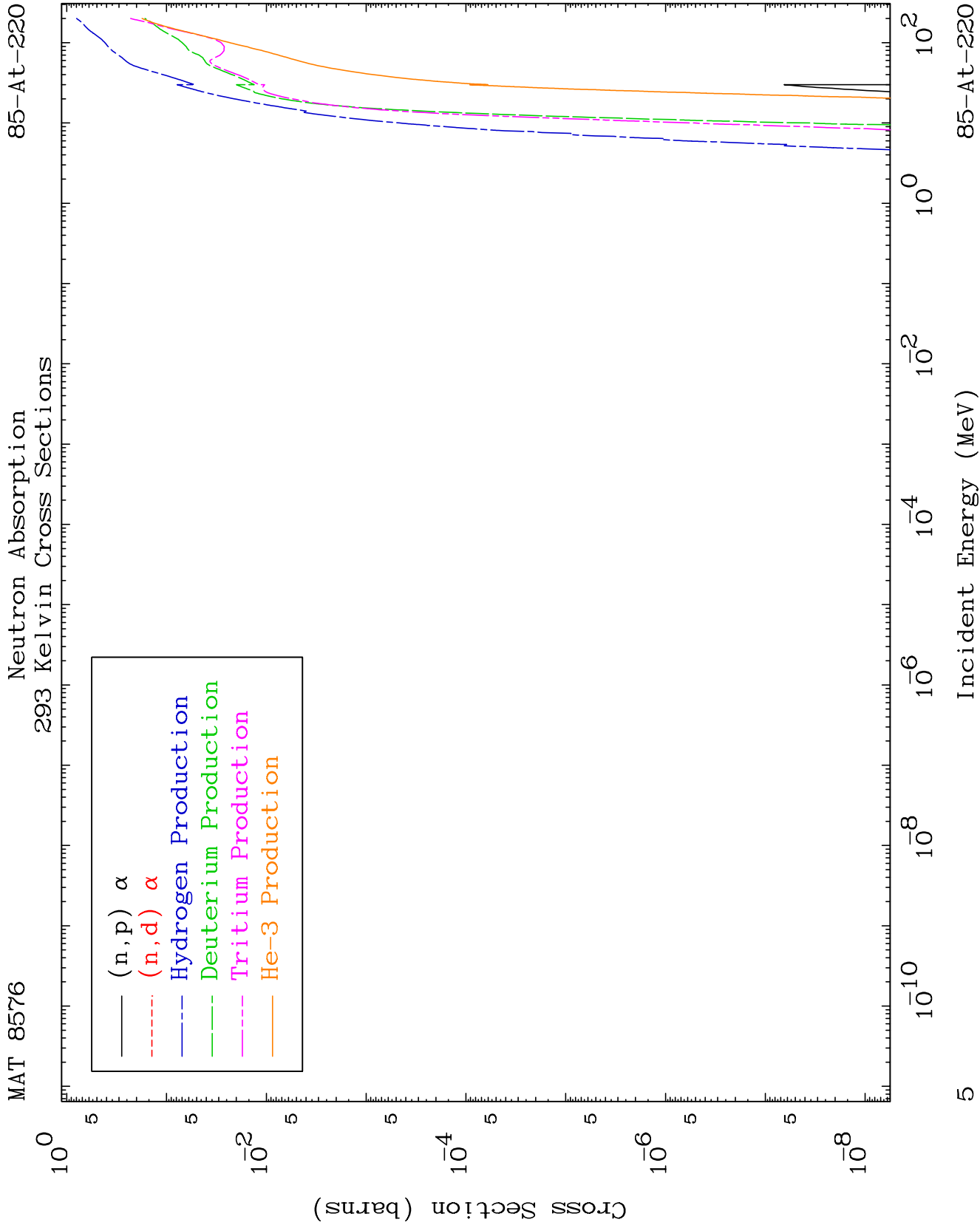
85-At-220

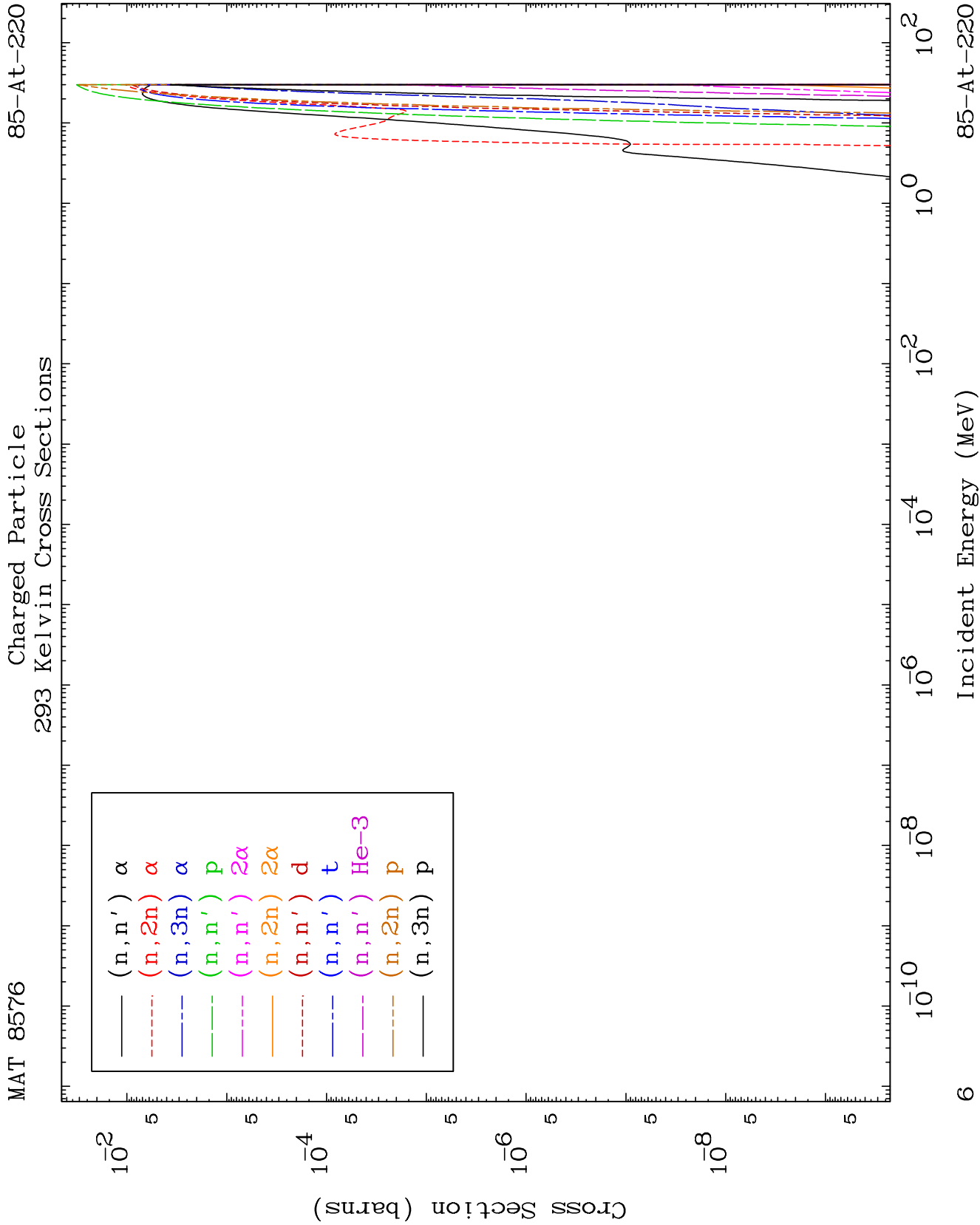


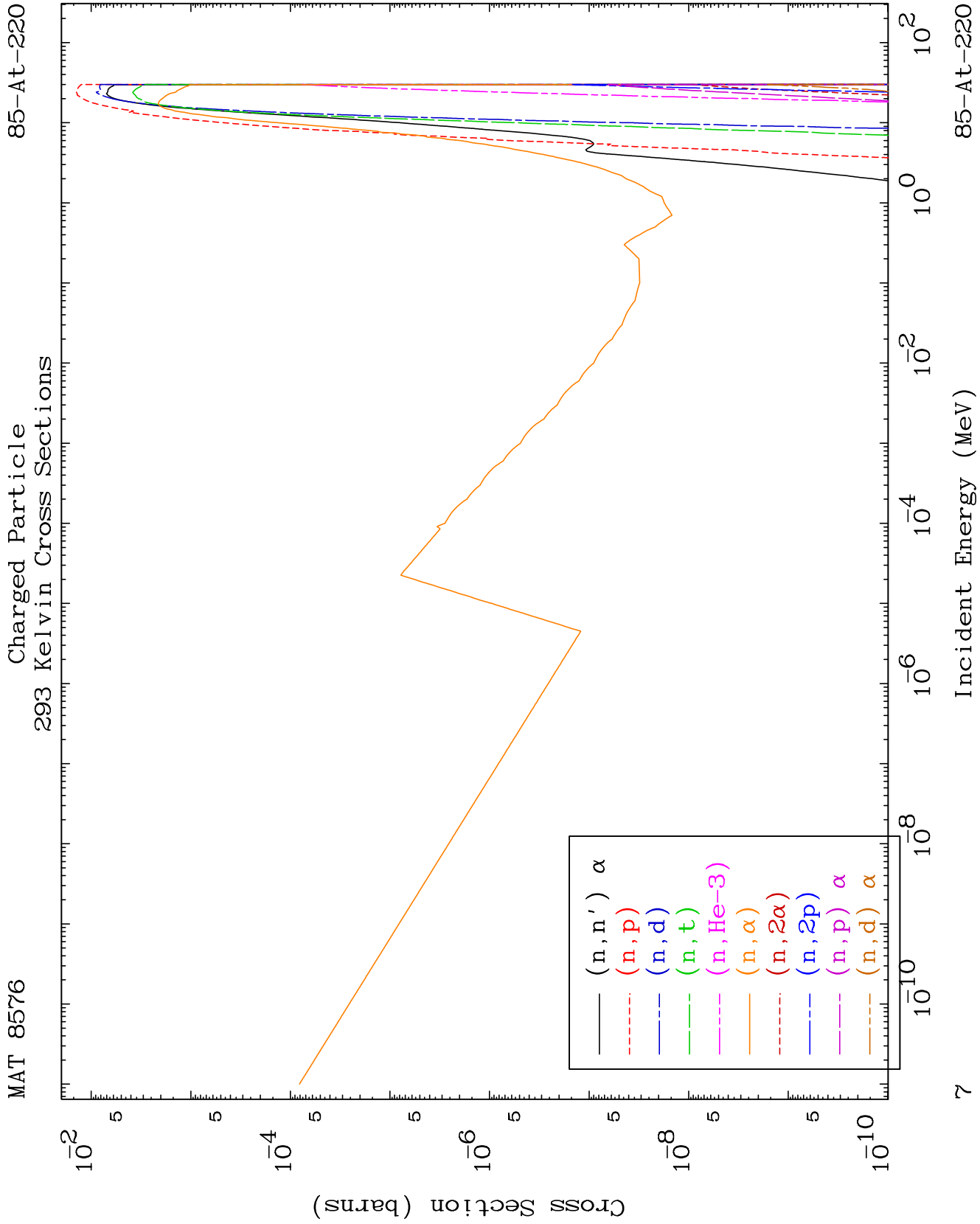


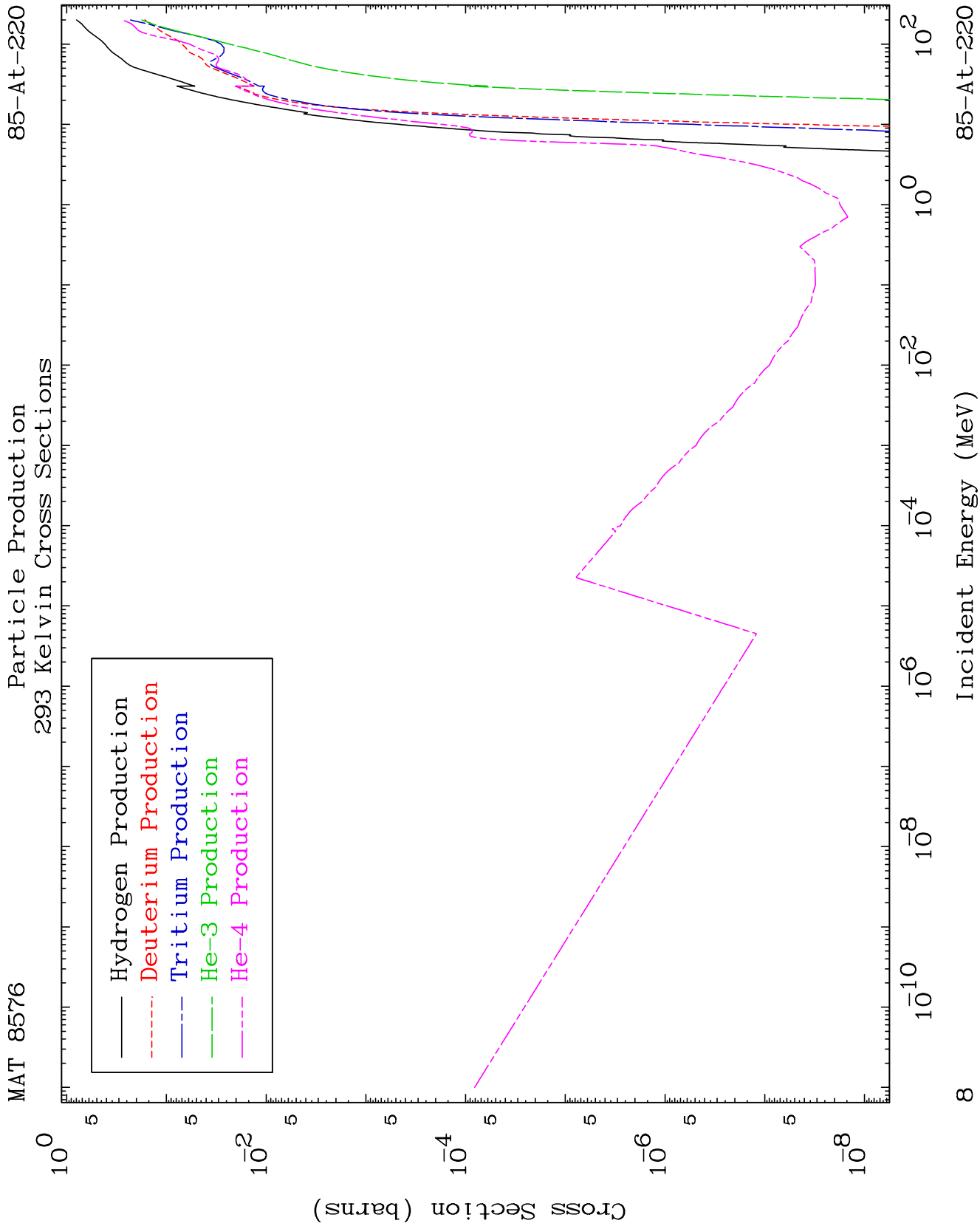


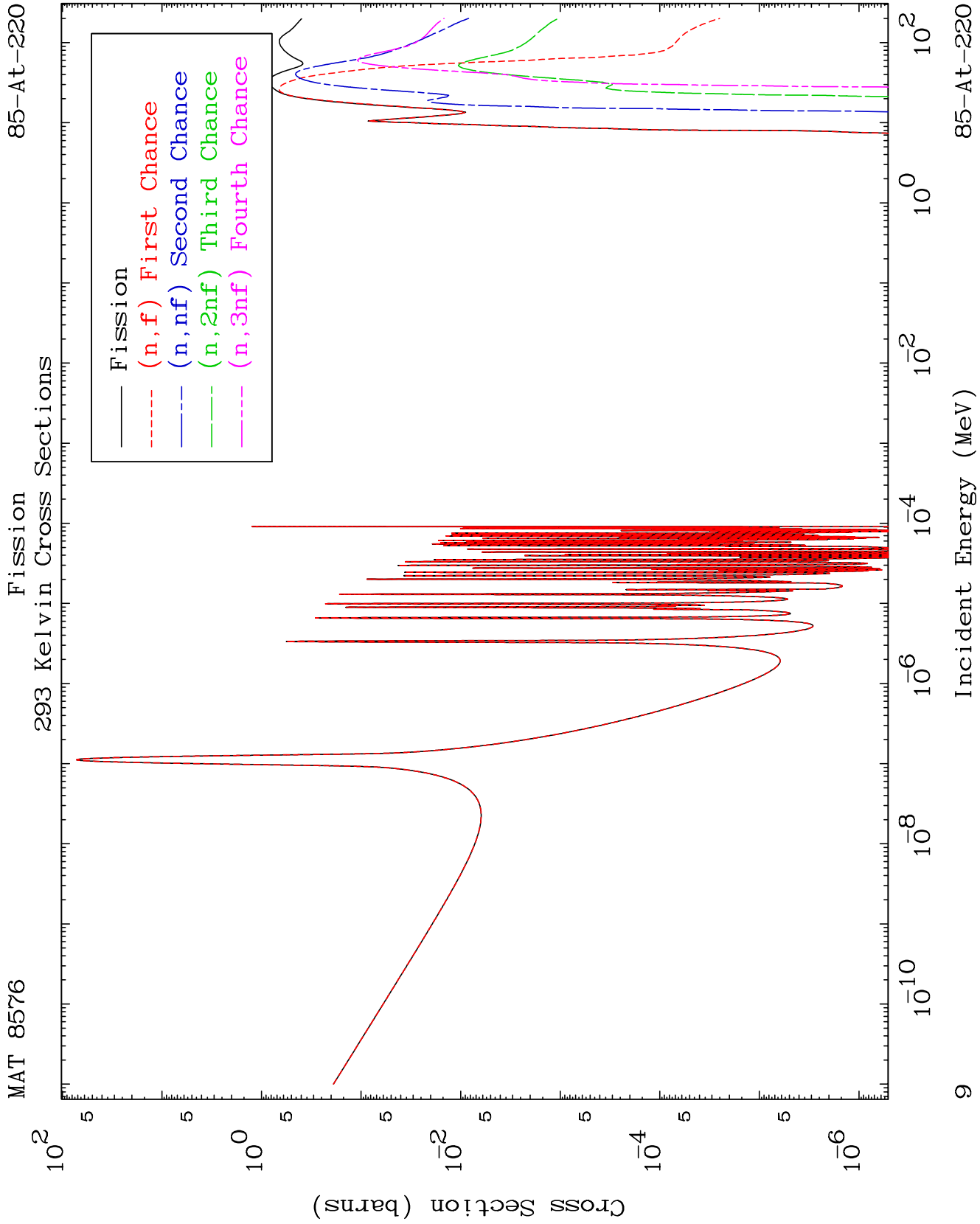


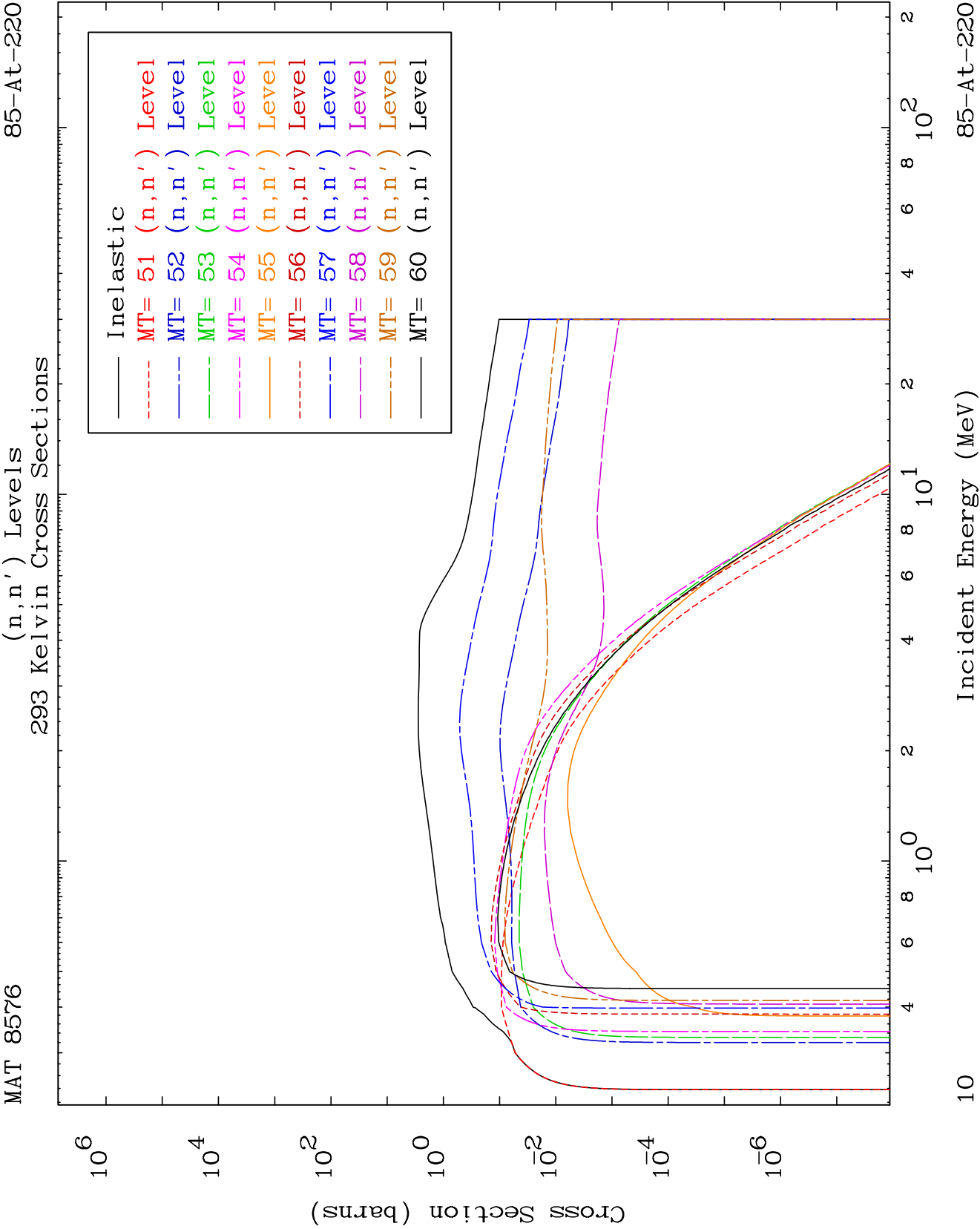


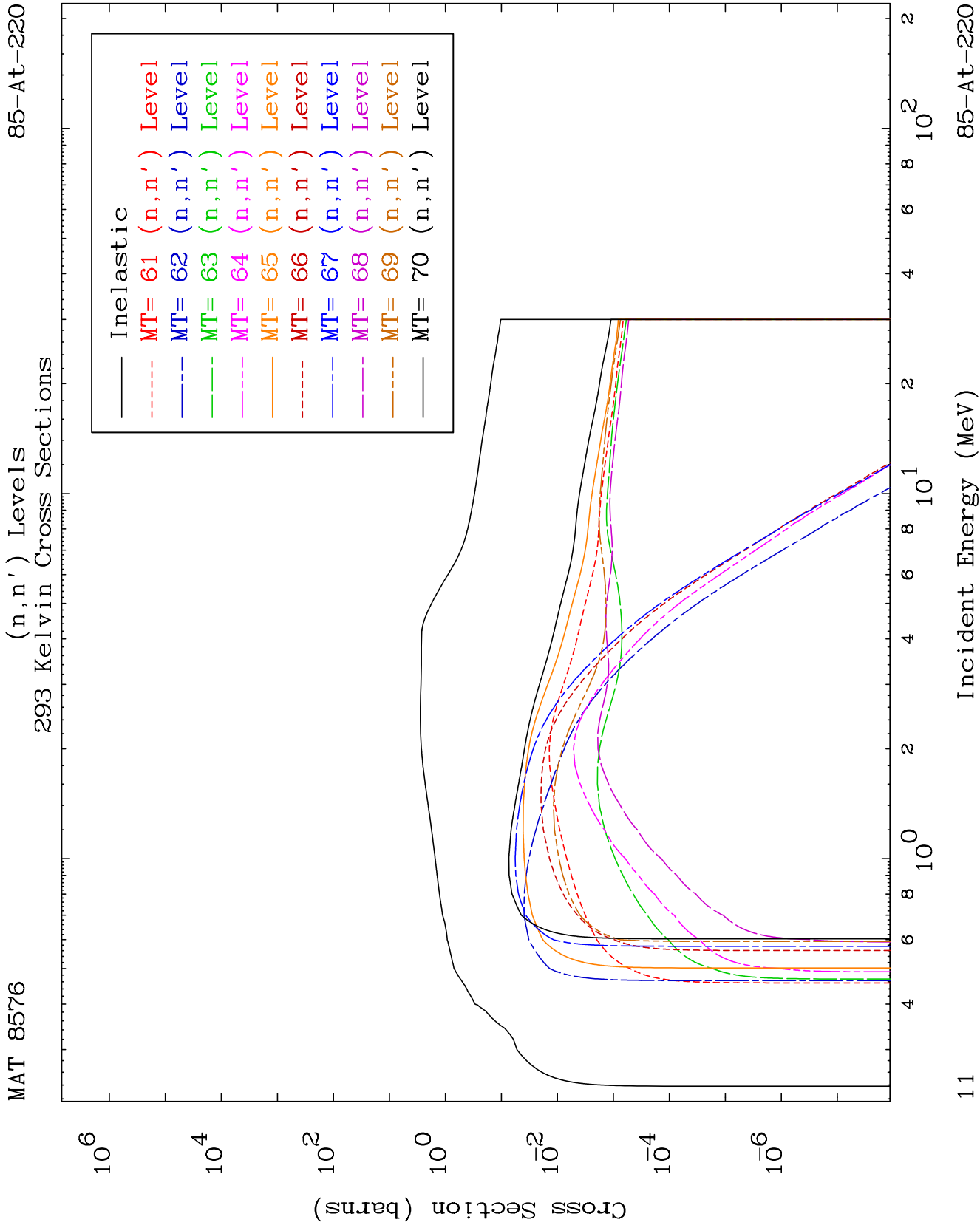


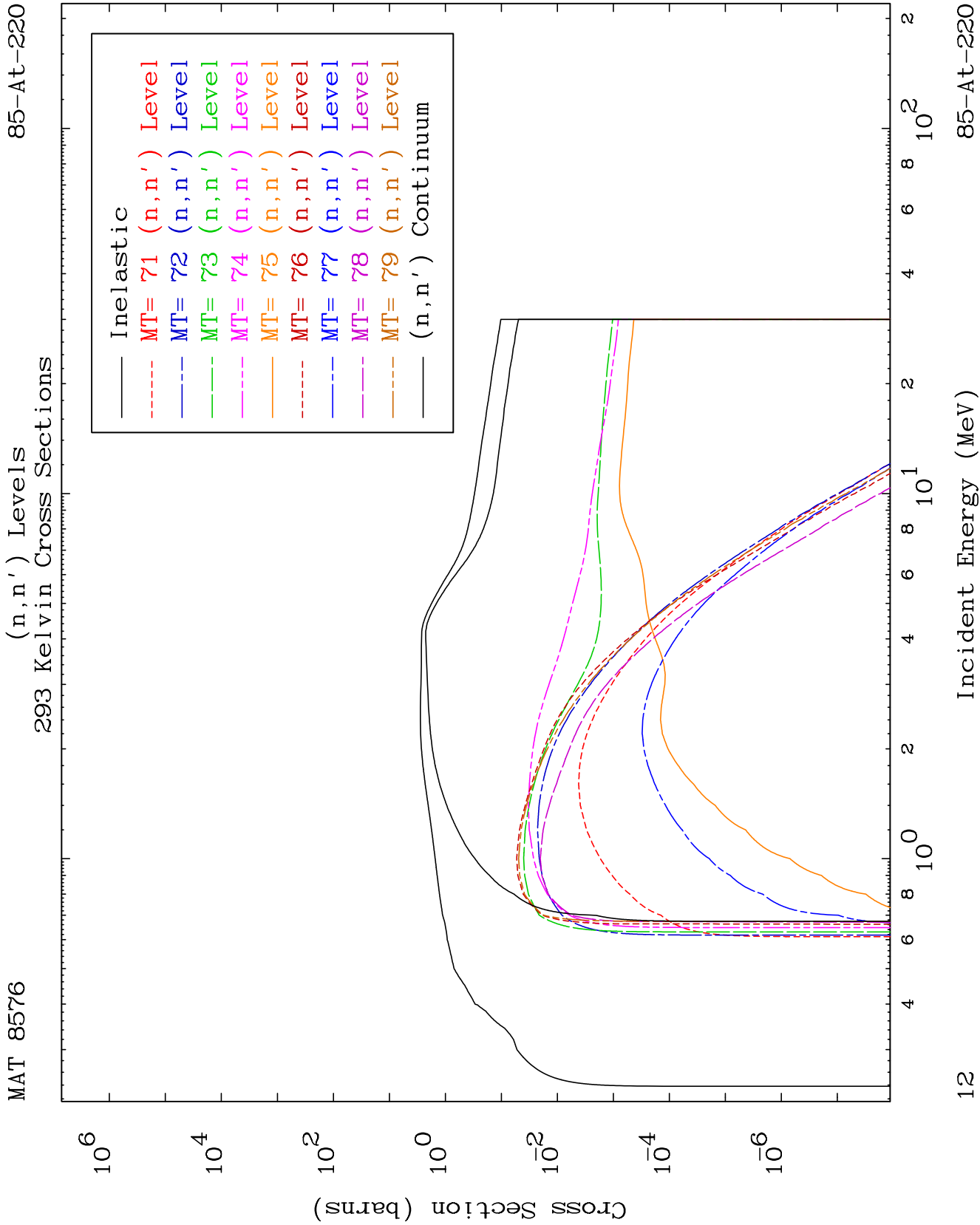


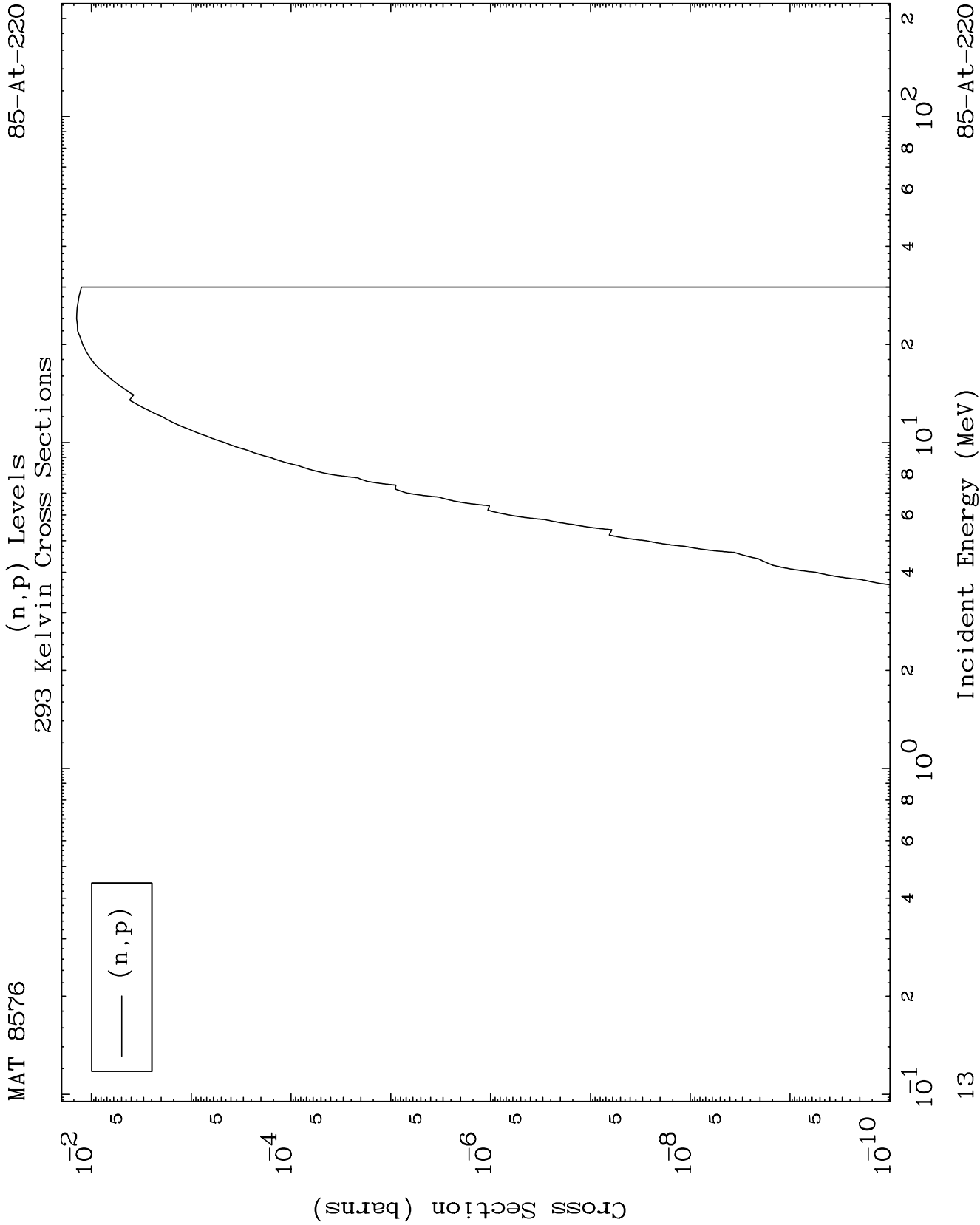


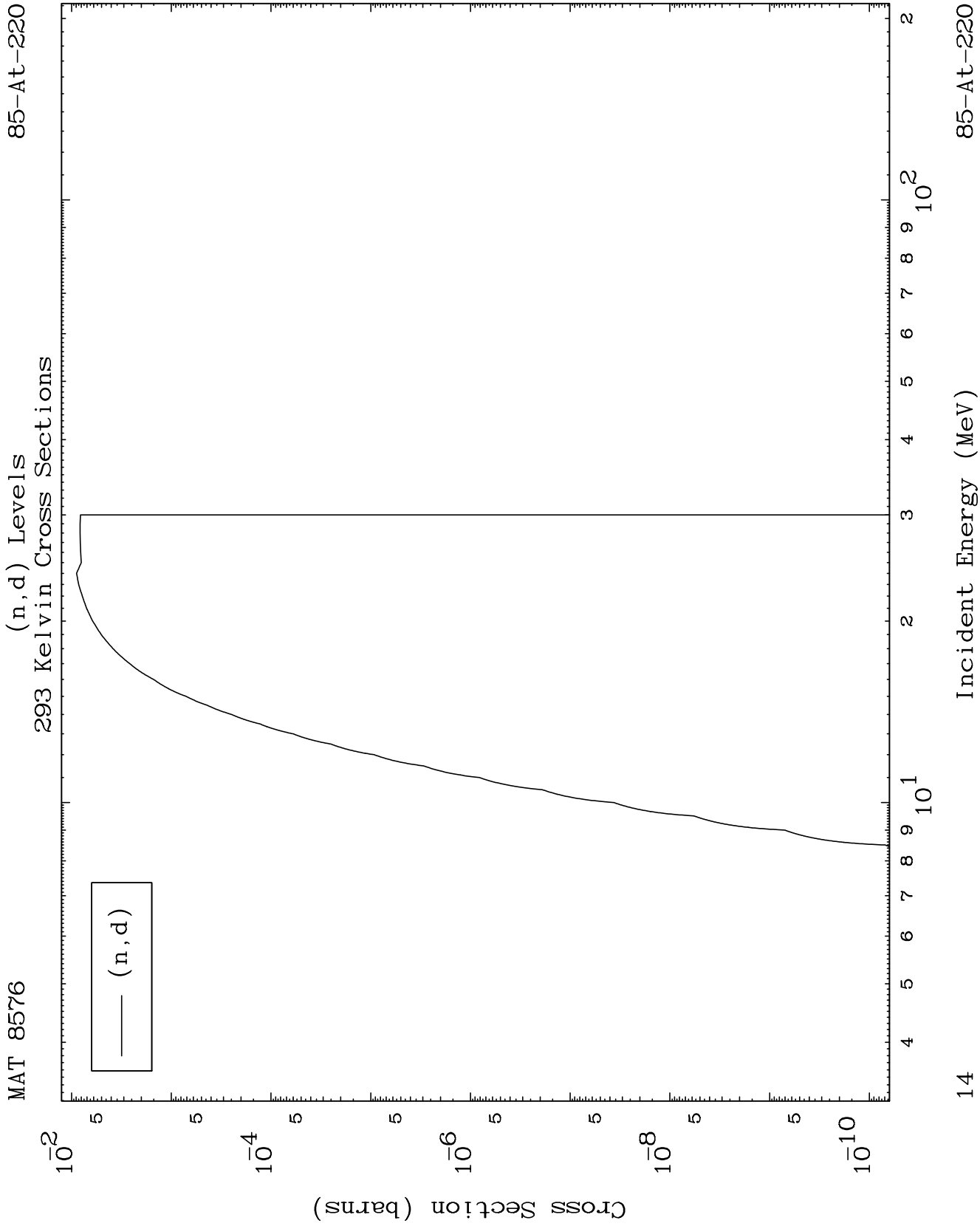


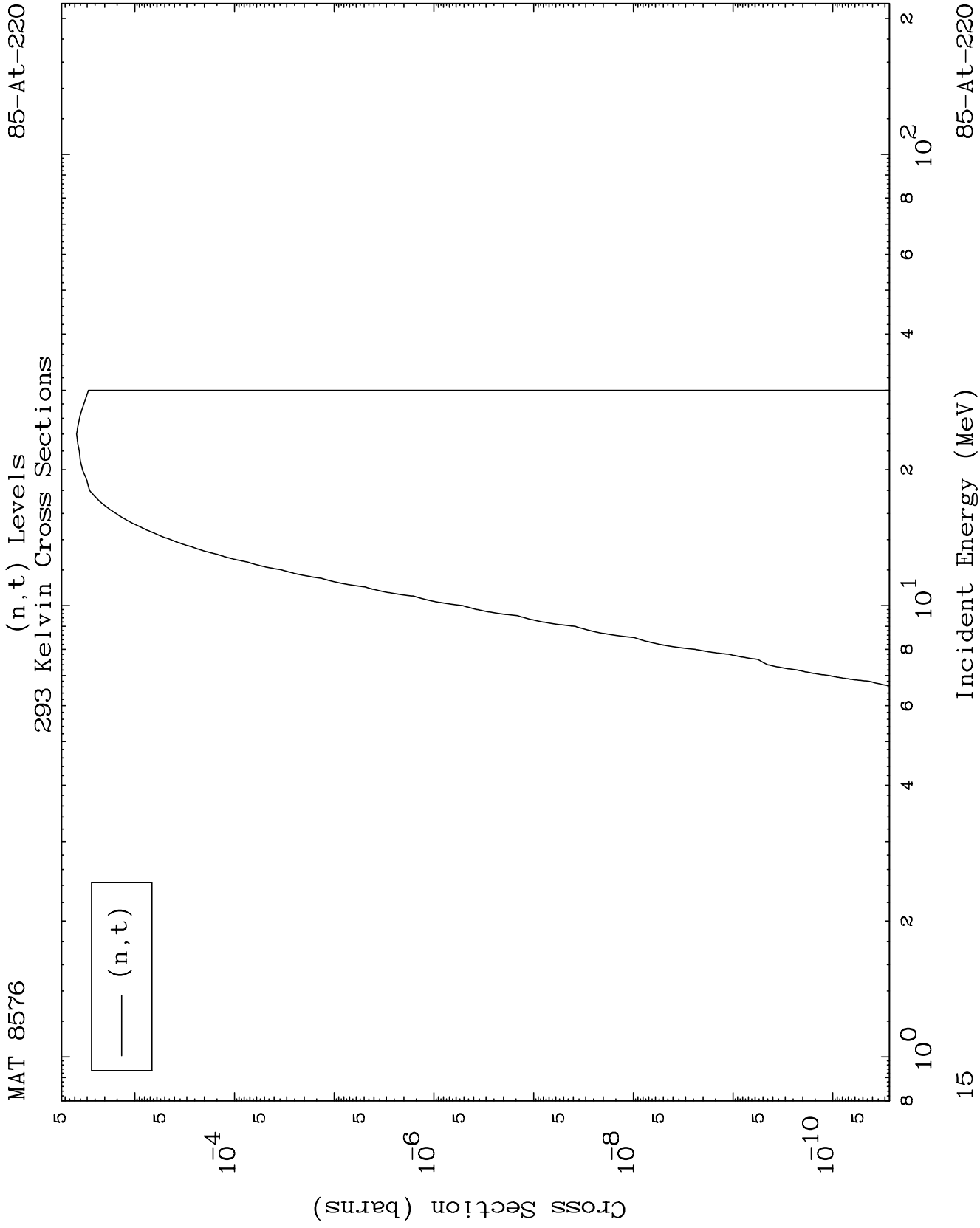


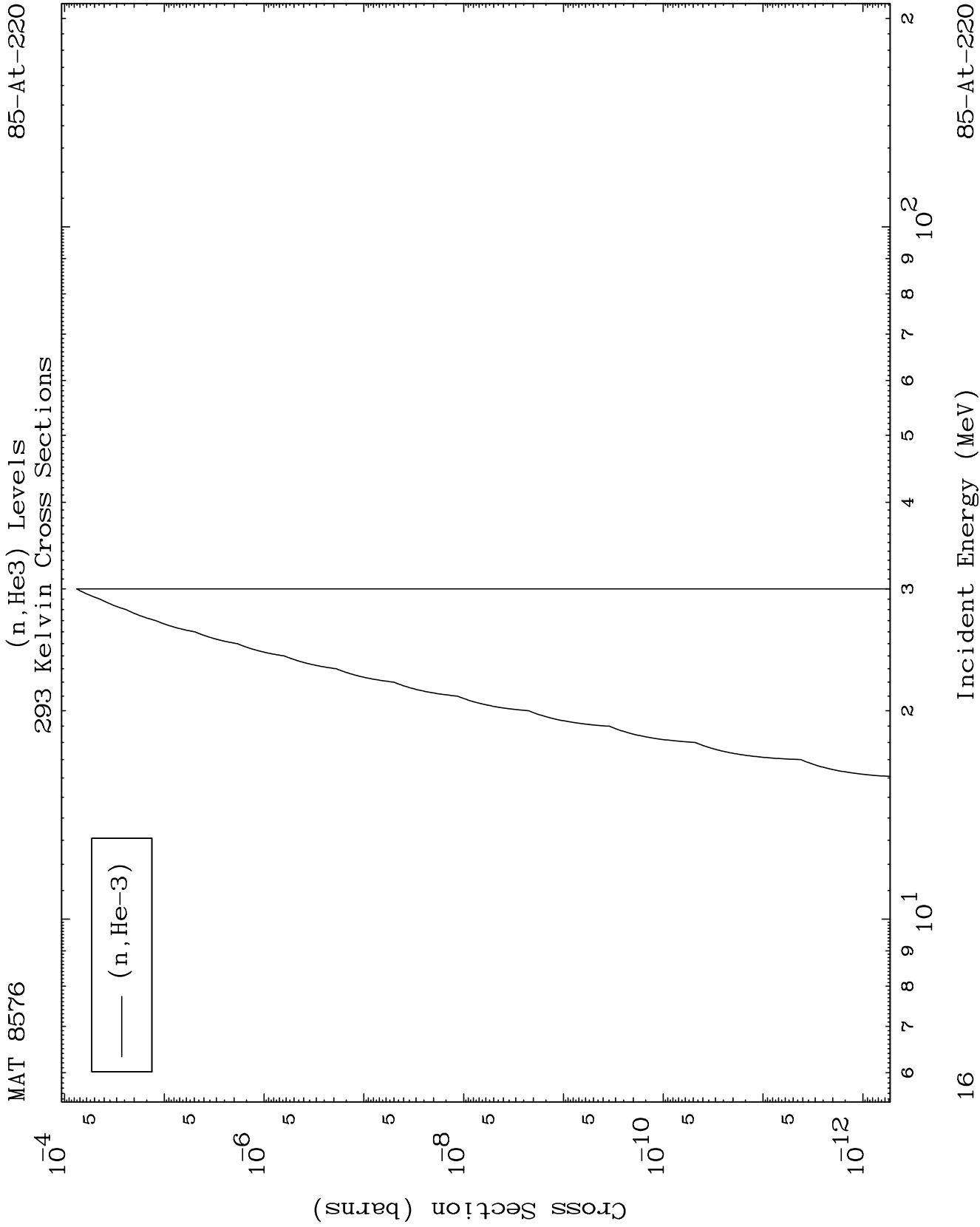








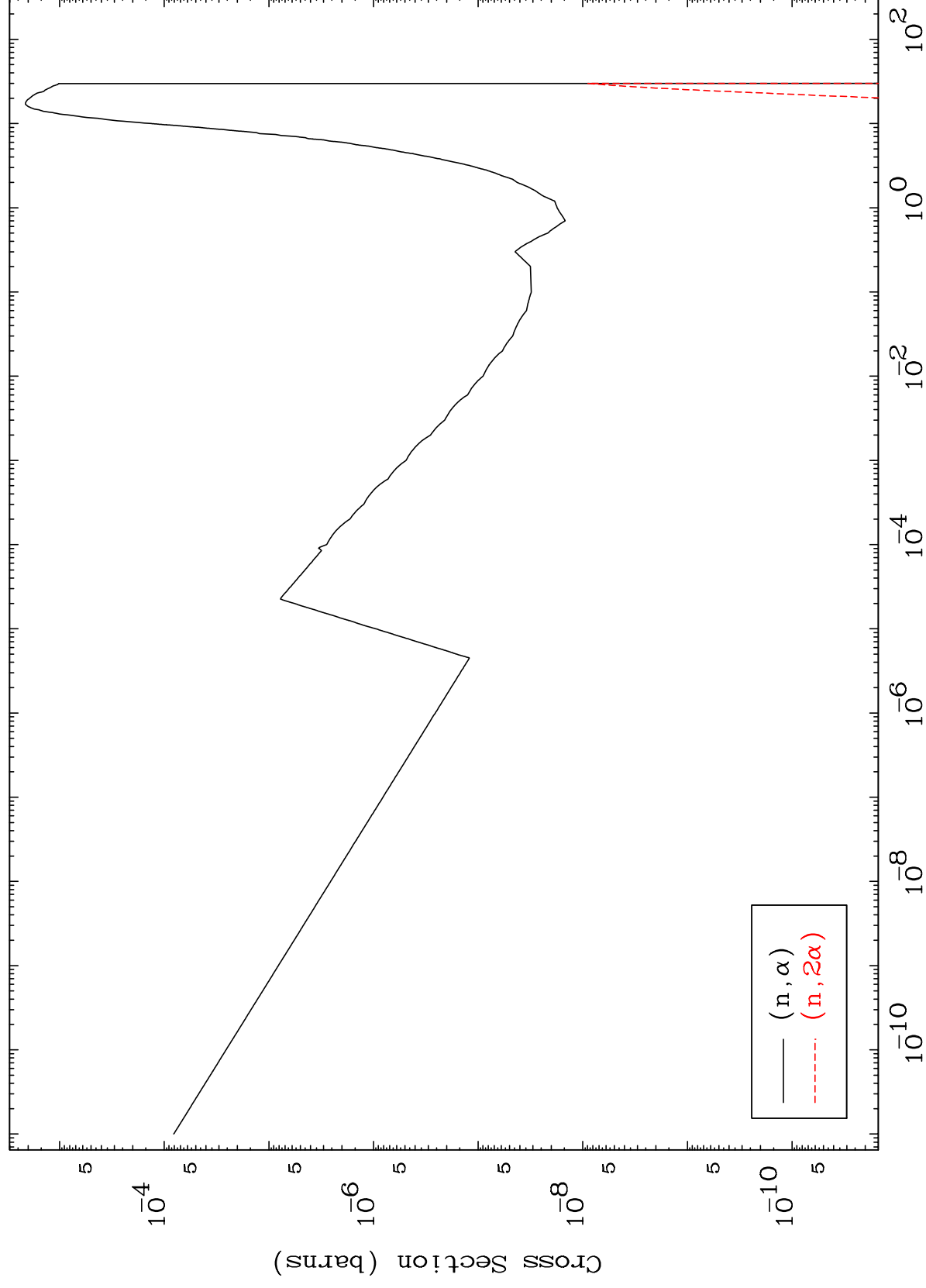




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(n, α) Levels
293 Kelvin Cross Sections

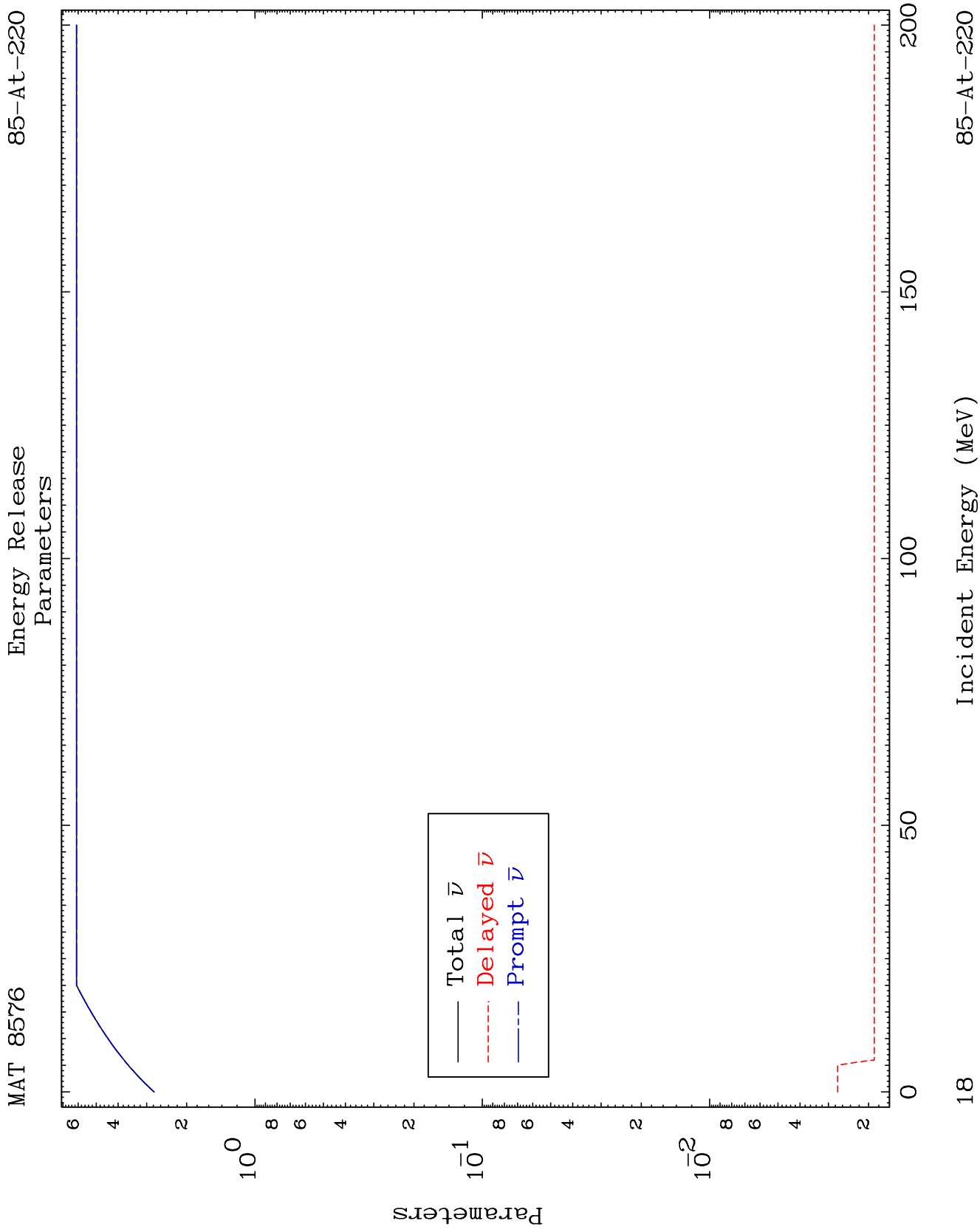
85-At-220

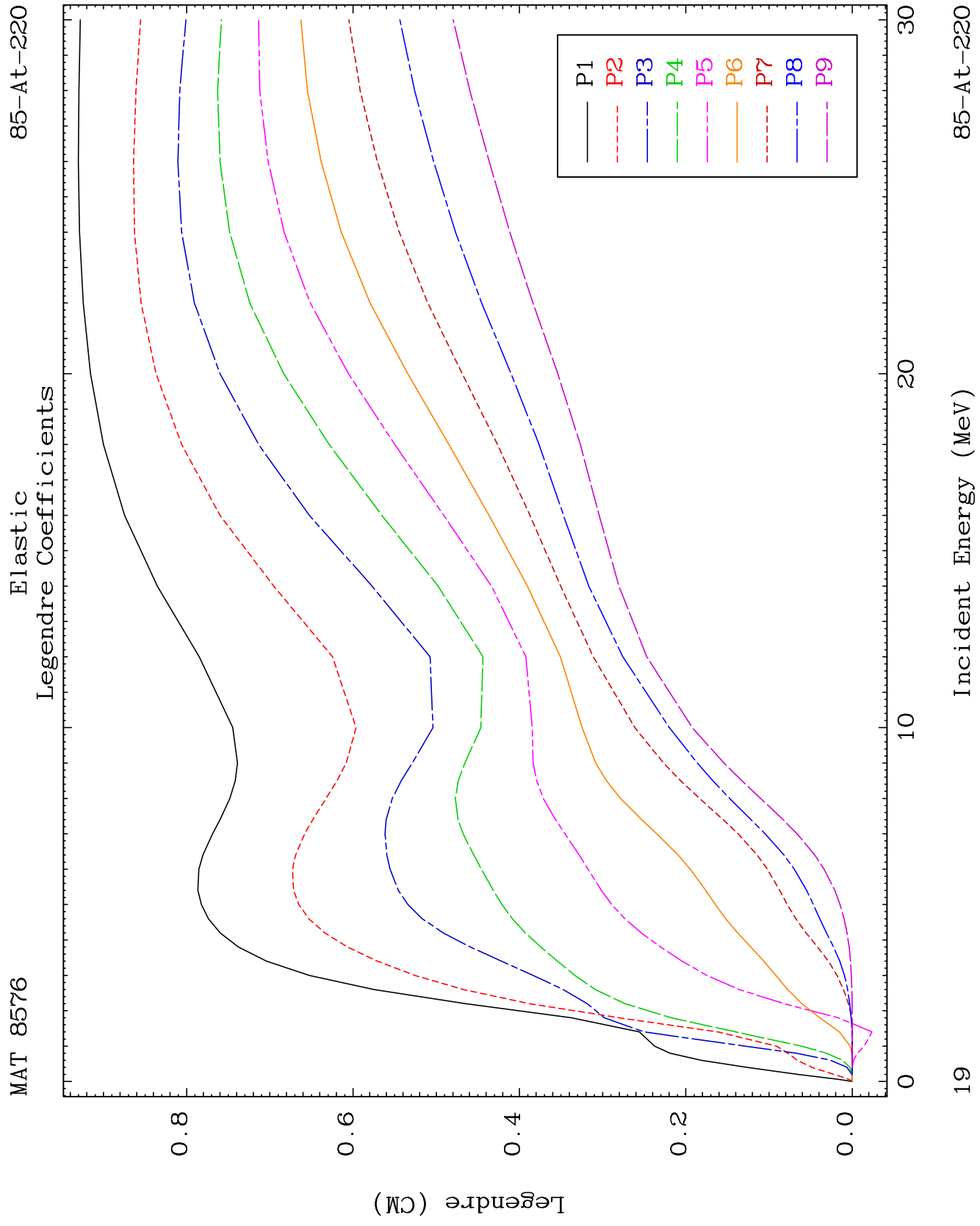


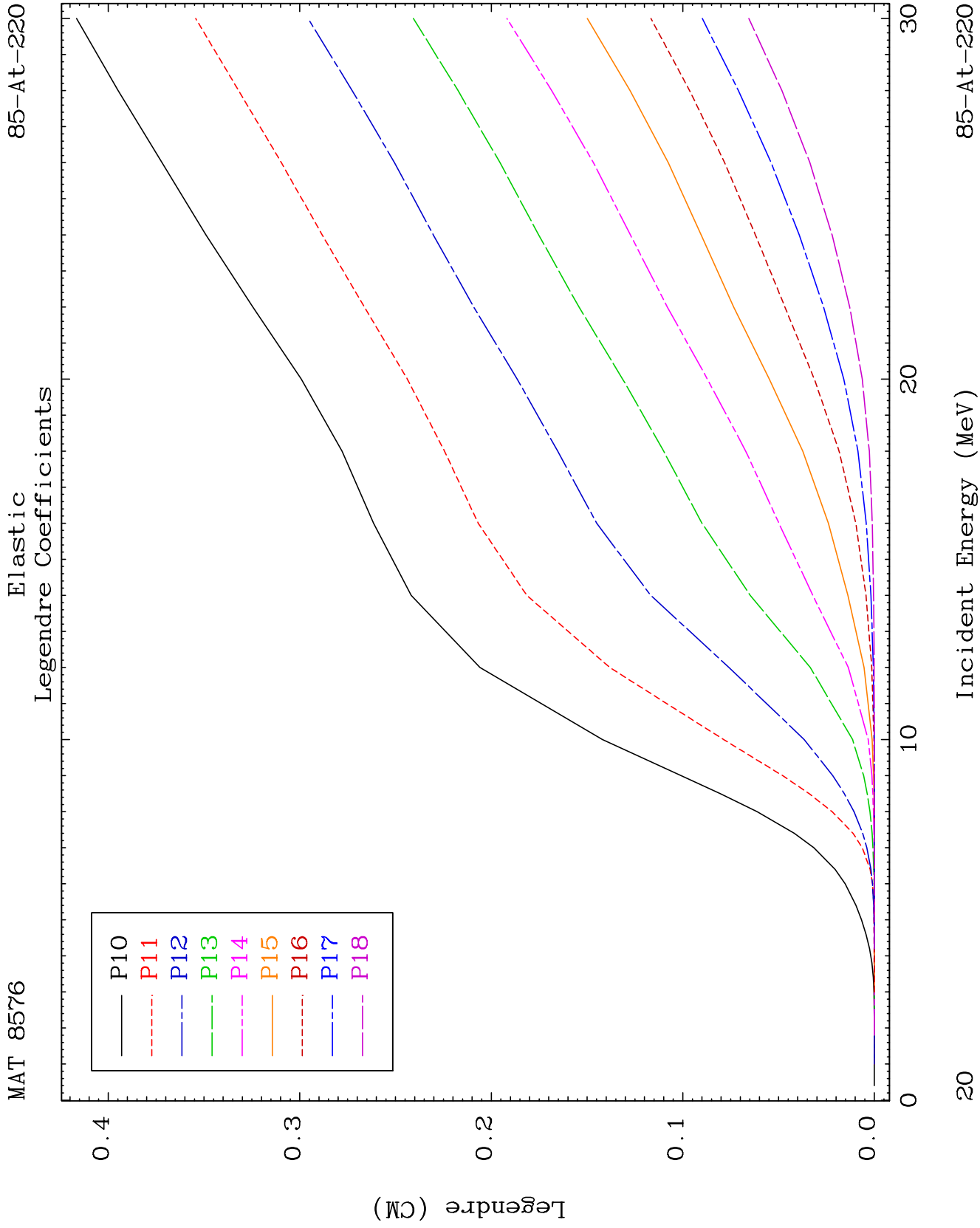
17

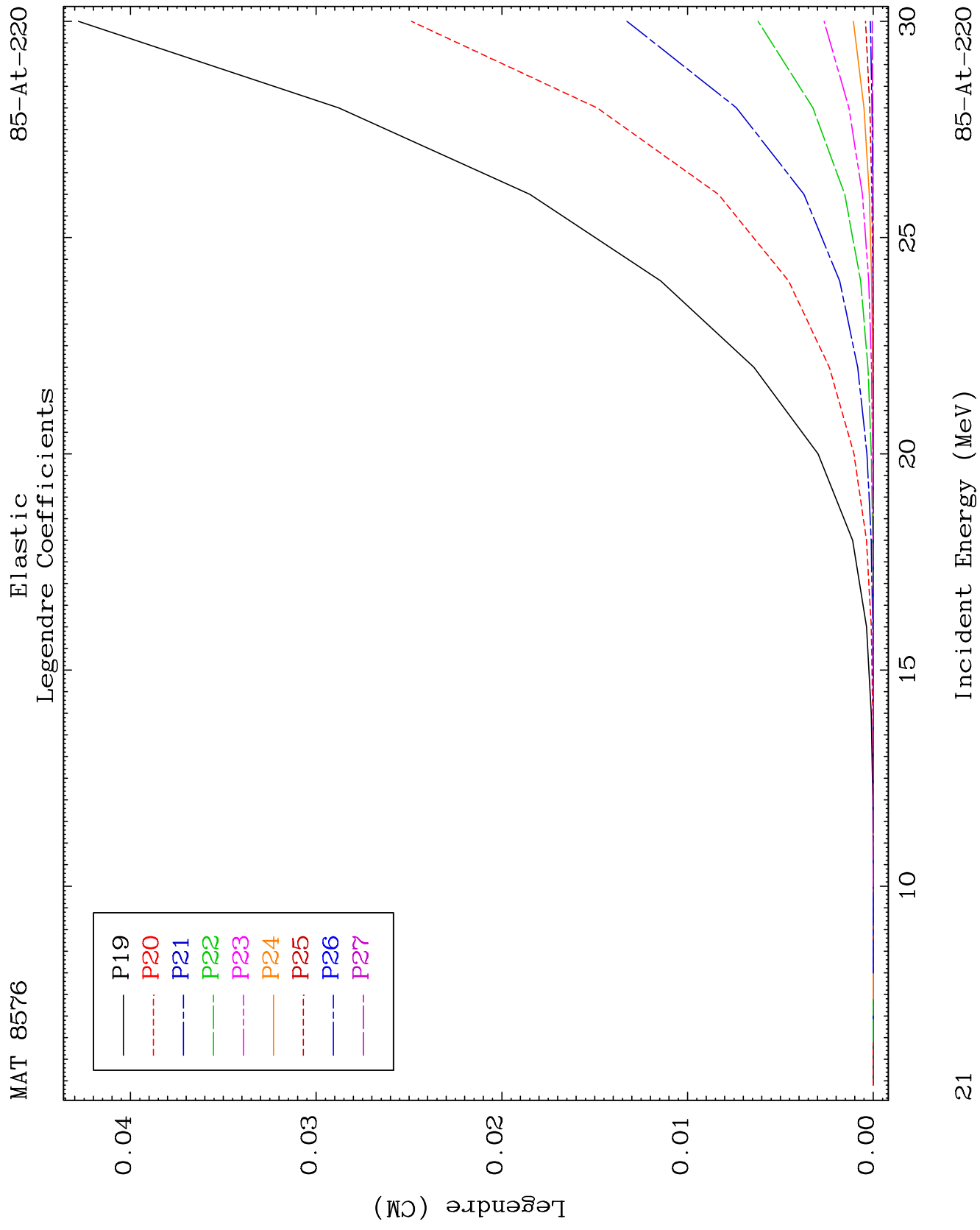
Incident Energy (MeV)

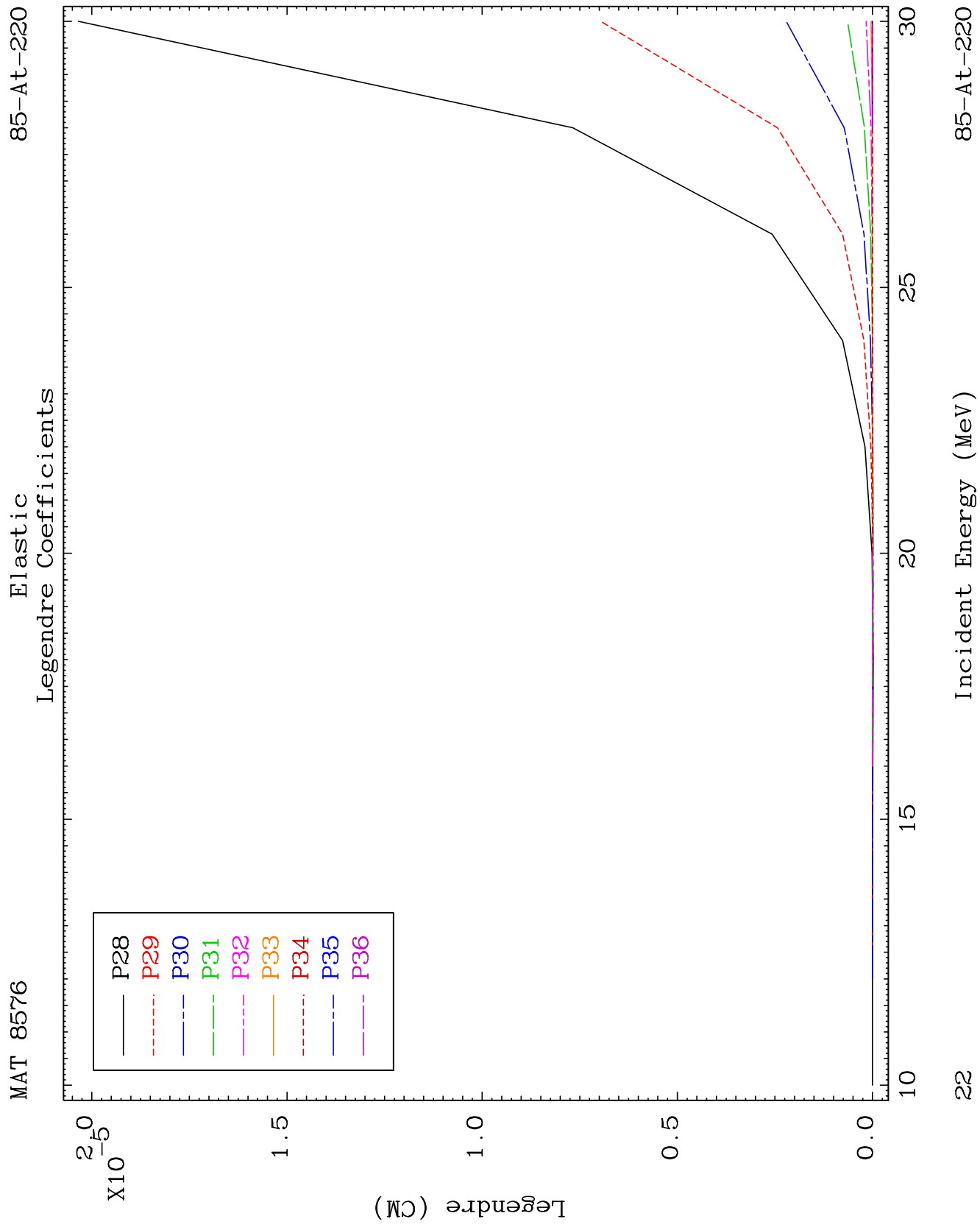
85-At-220







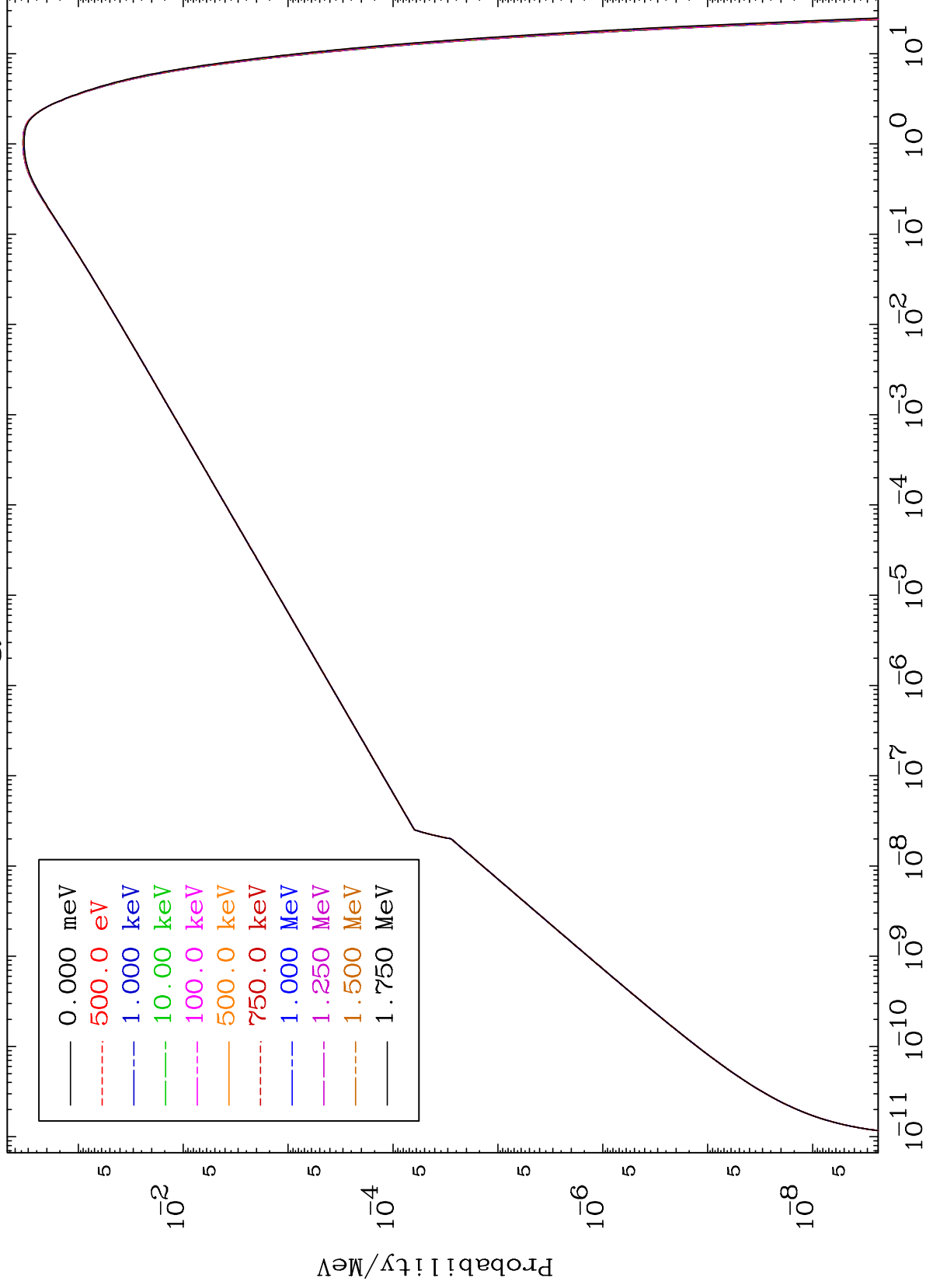




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Fission
Energy Distributions

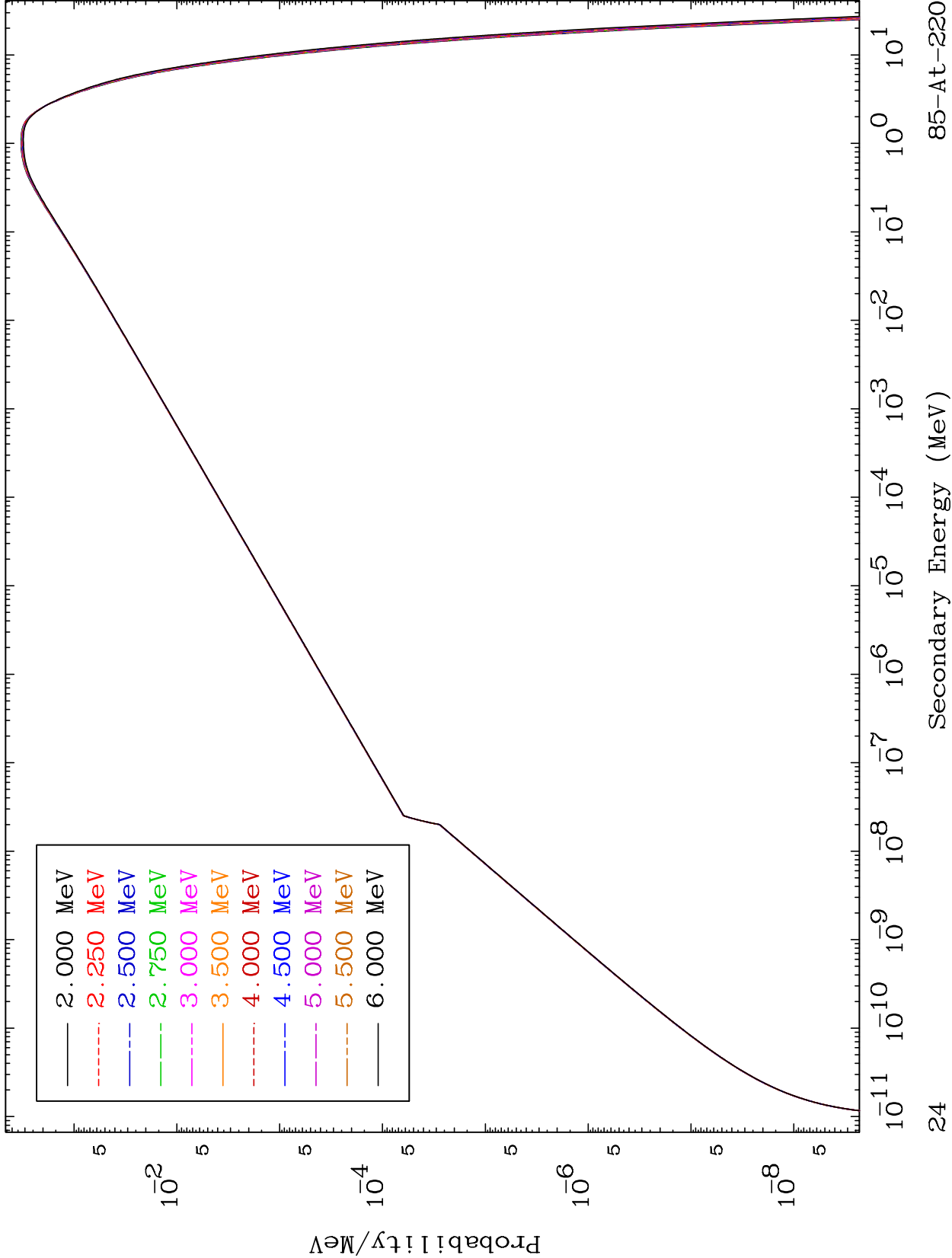
85-At-220



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Secondary Energy (MeV)

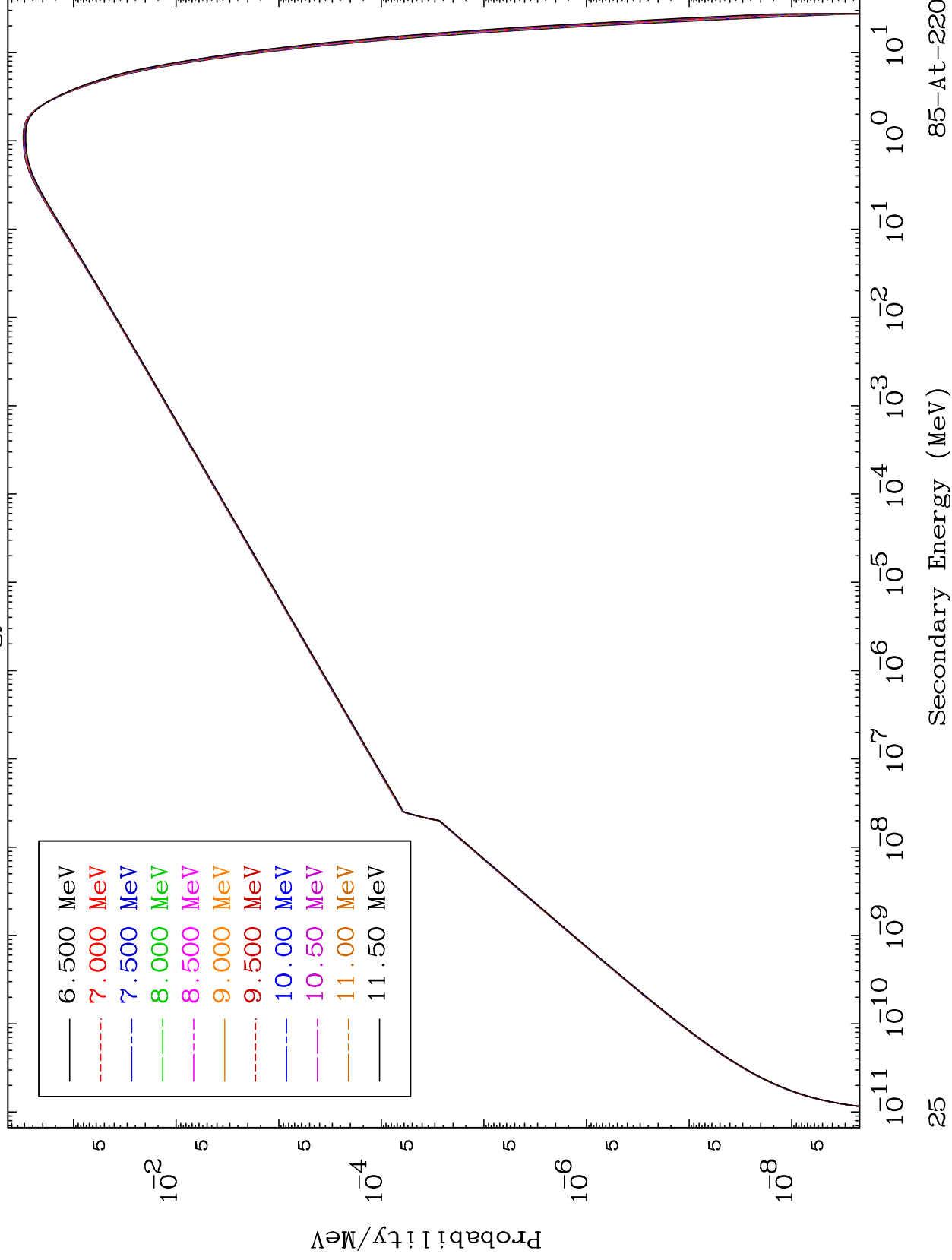
85-At-220



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Fission
Energy Distributions

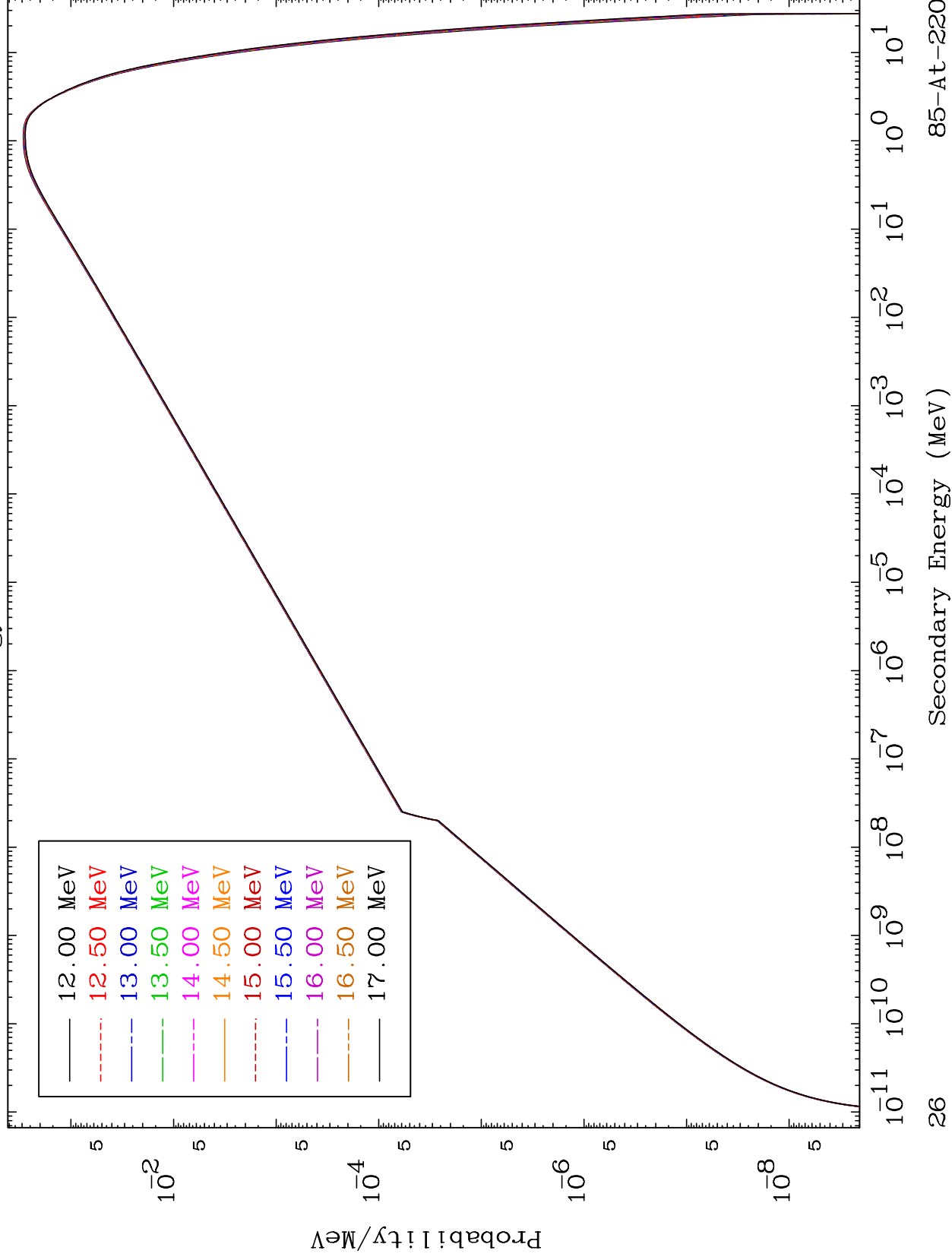
85-At-220



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Fission
Energy Distributions

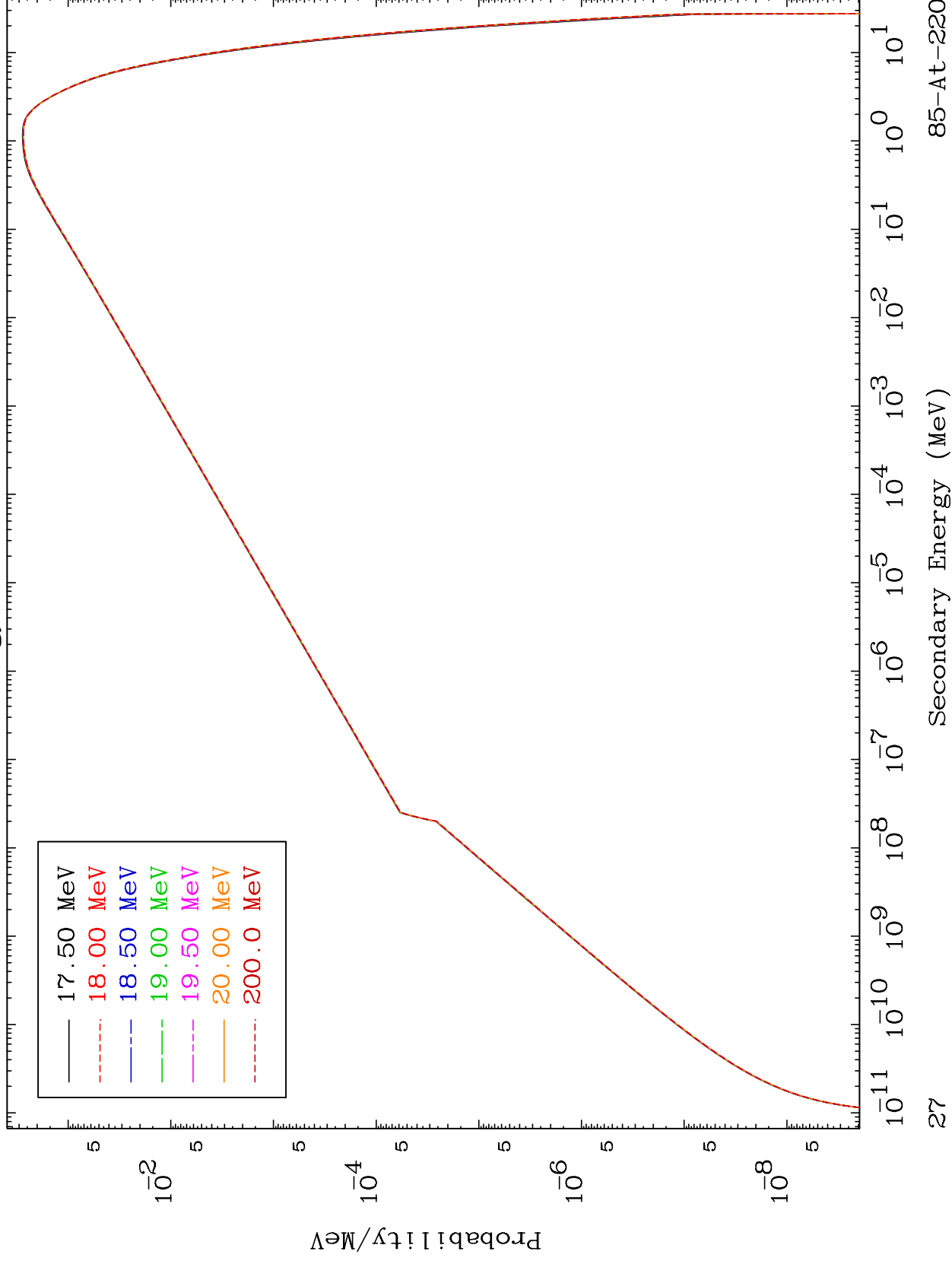
85-At-220



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Fission
Energy Distributions

85-At-220

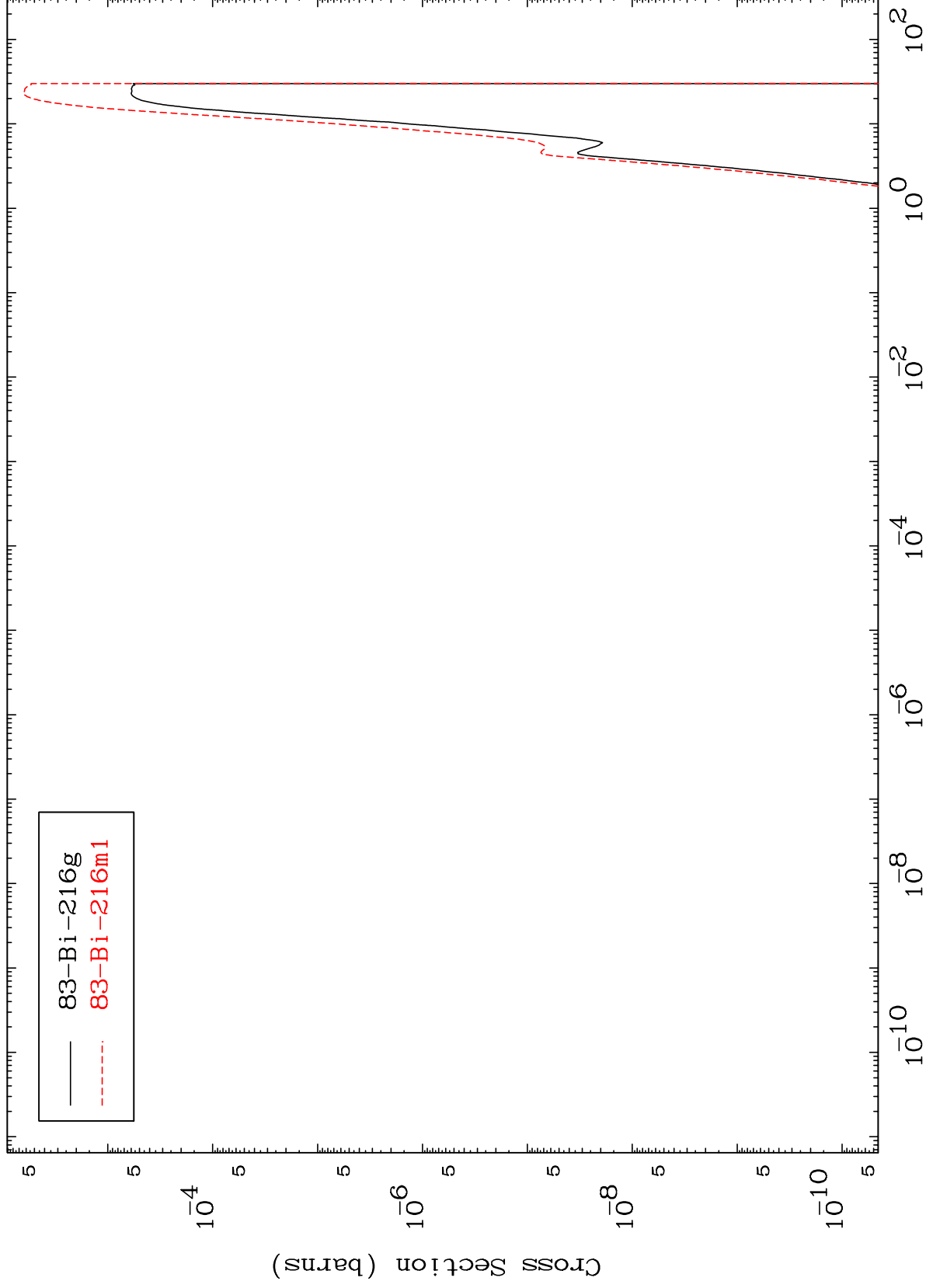


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$(n,n') \alpha$

85-At-220

Radionuclide Production Cross Section

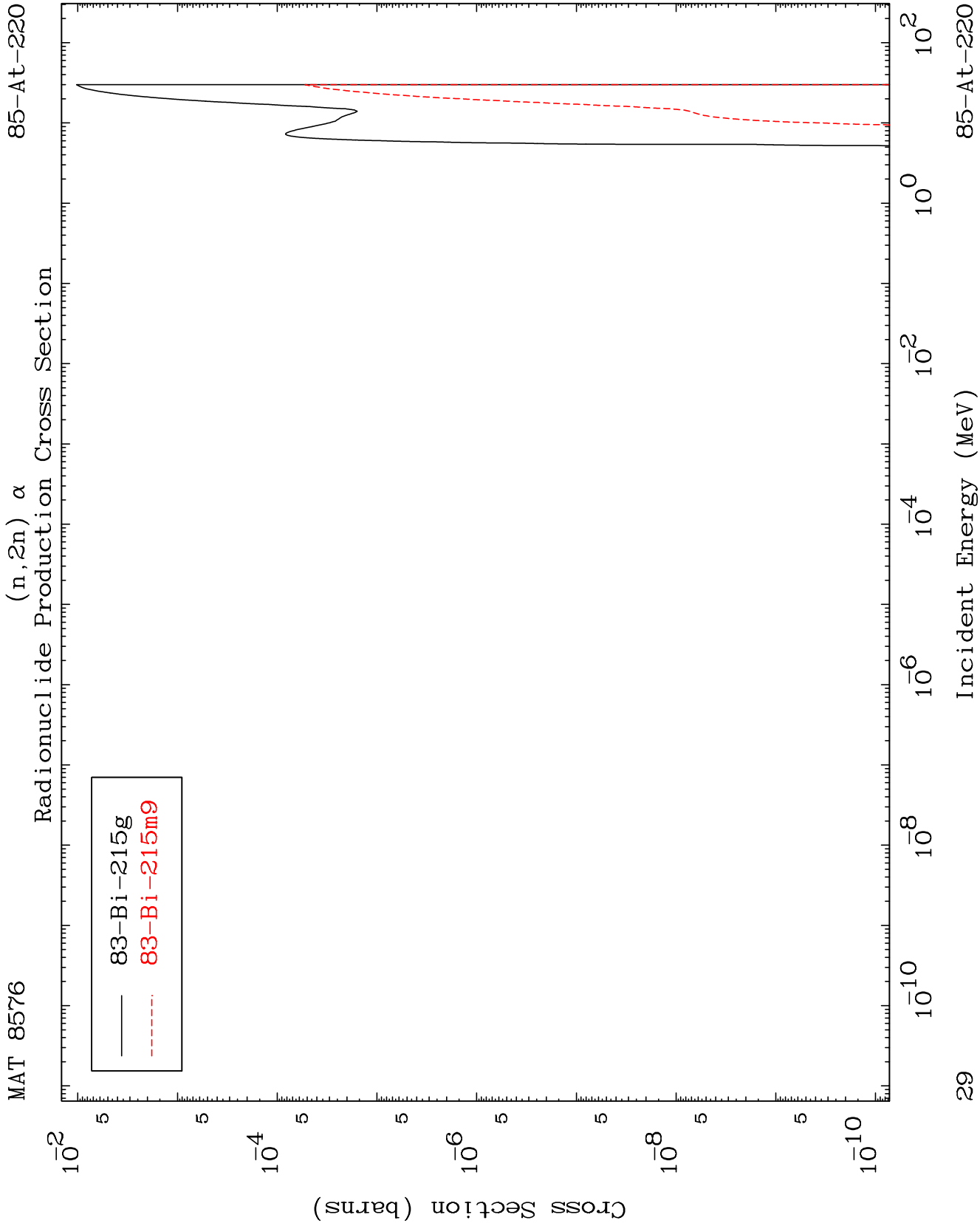


83-Bi-216g
83-Bi-216m1

28

Incident Energy (MeV)

85-At-220

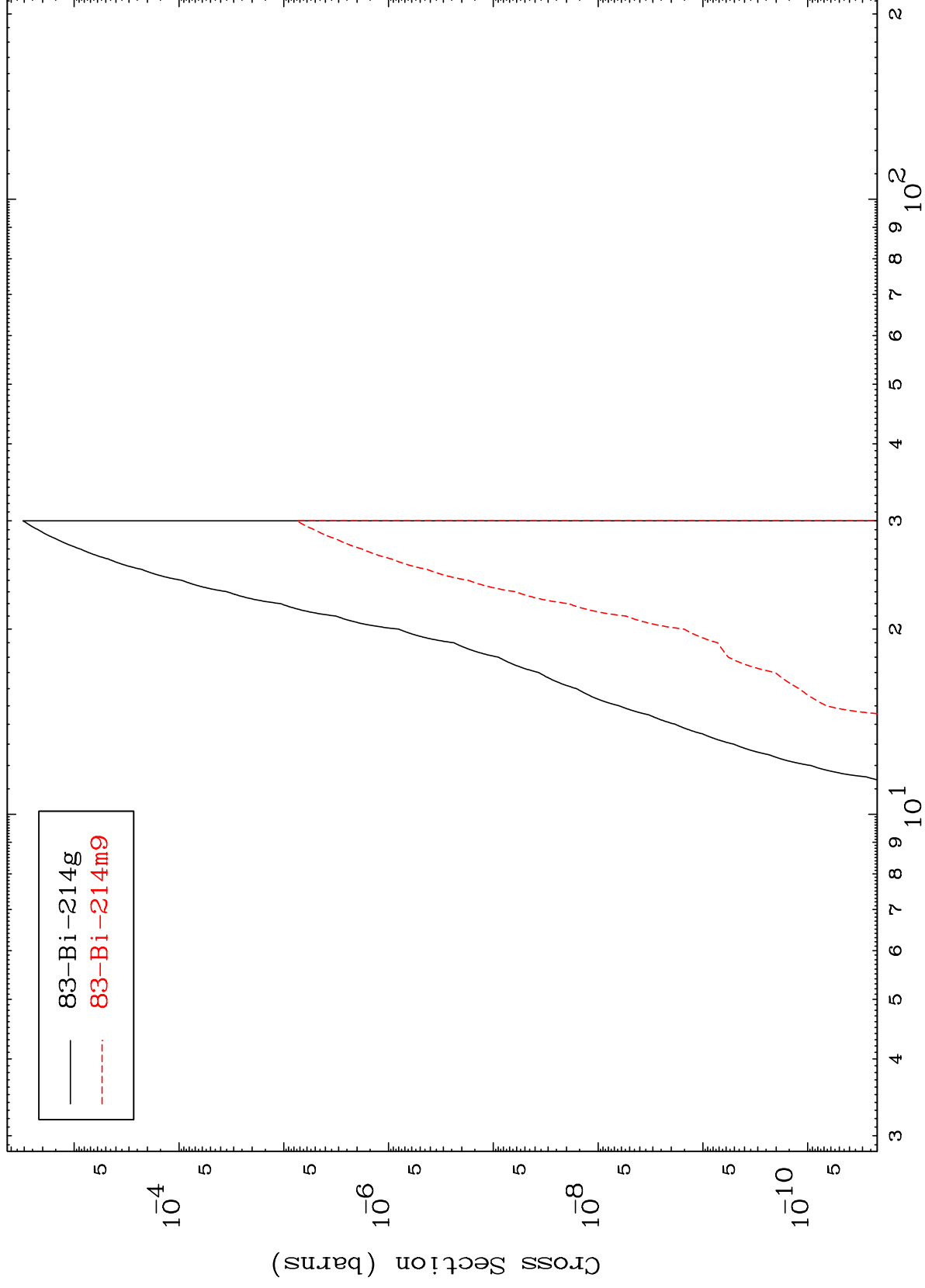


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(n,3n) α

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Radionuclide Production Cross Section



Incident Energy (MeV)

85-At-220

30