

Program EVALPLOT
(Version 2021-1)

by

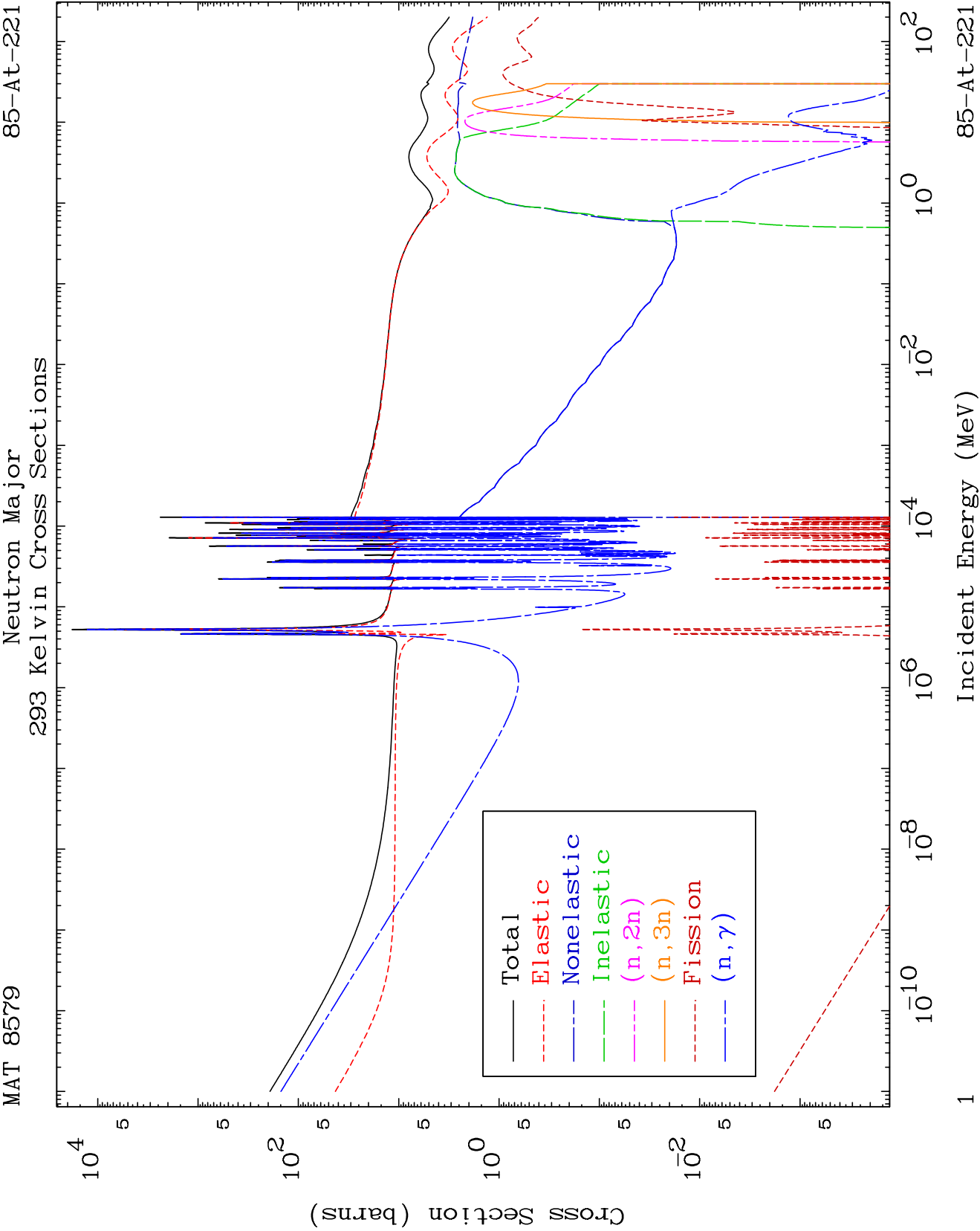
Dermott E. Cullen
(Present Contact Information)

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Livermore, CA 94550
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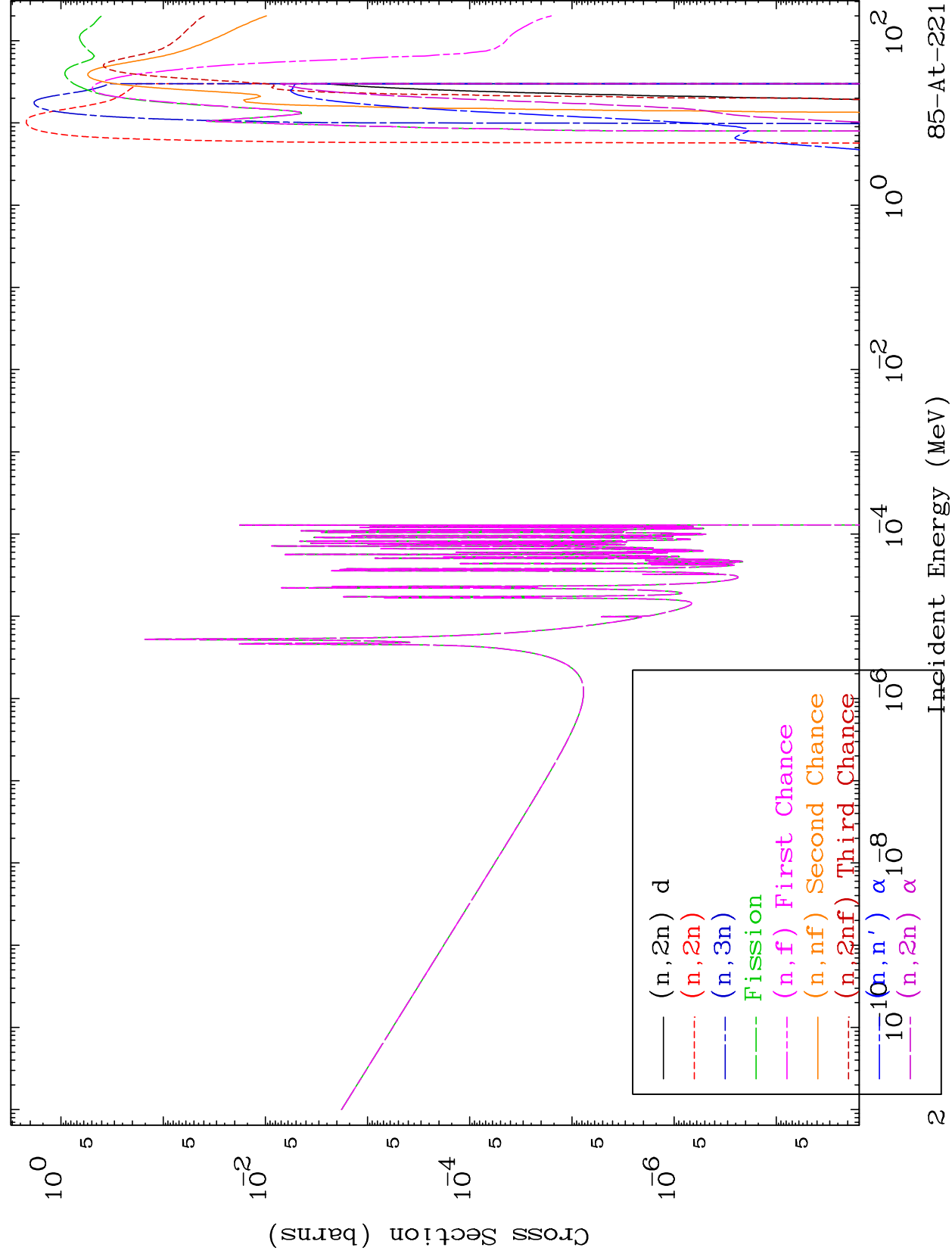
Press Mouse Button to Start

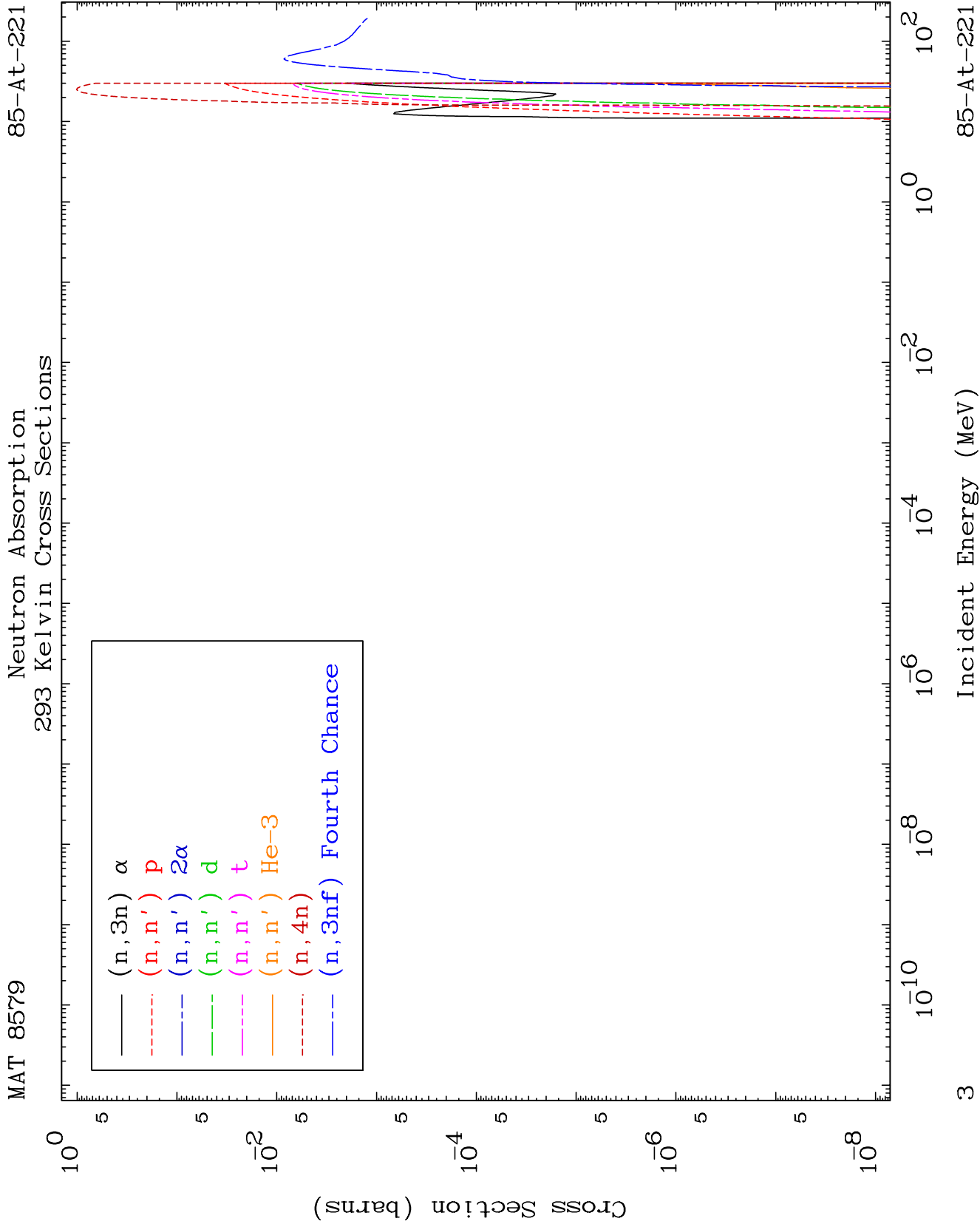


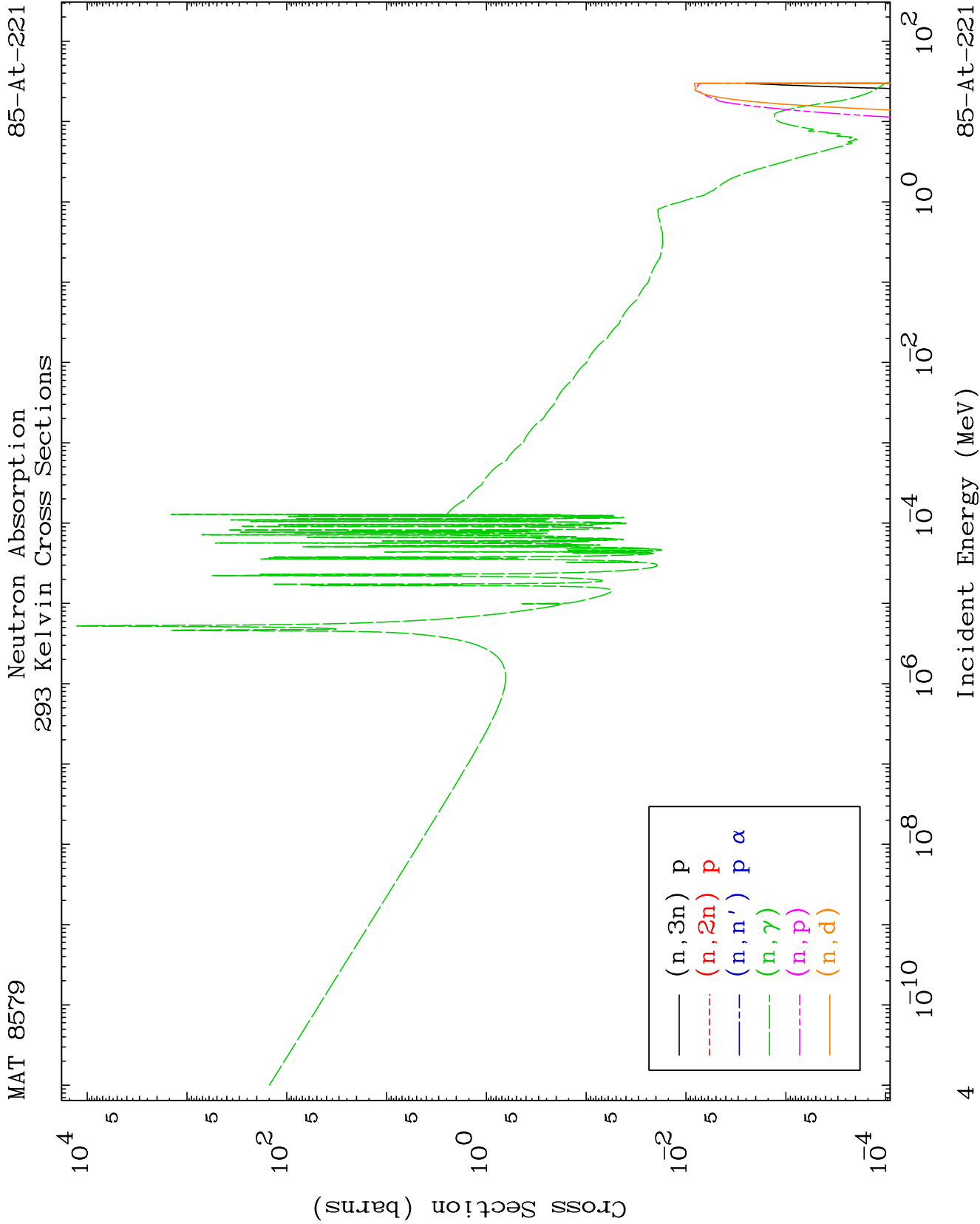
MAT 8579

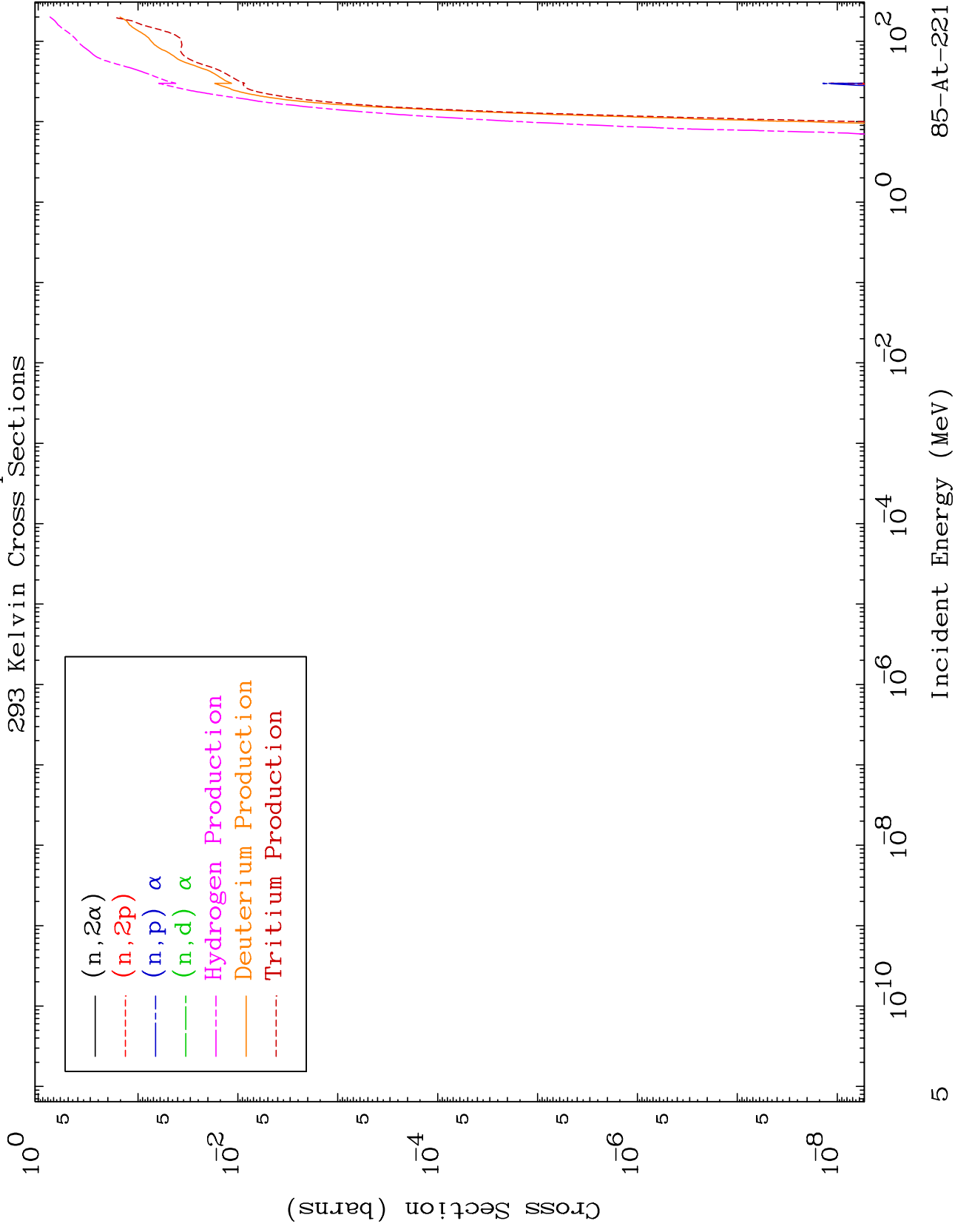
Neutron Absorption
293 Kelvin Cross Sections

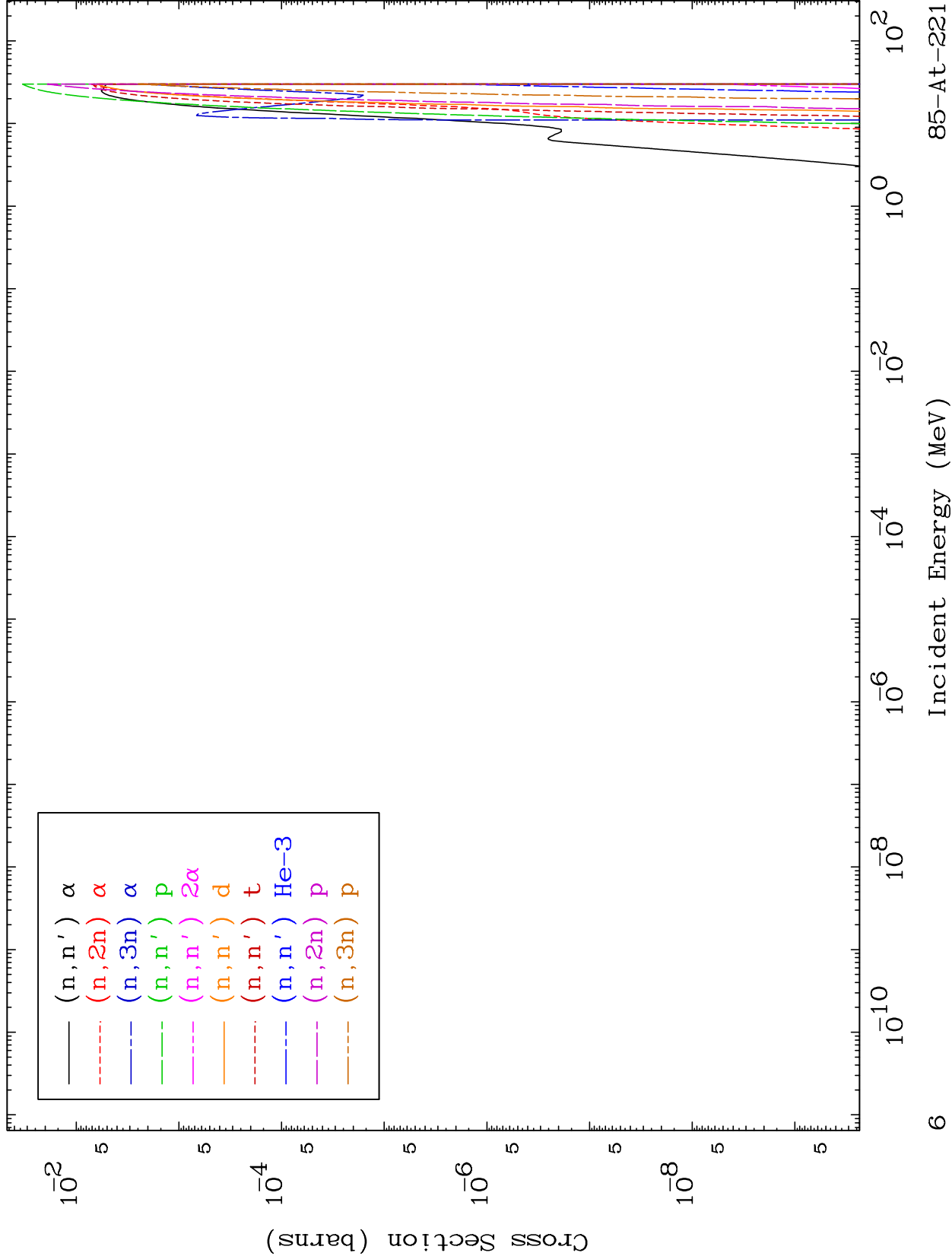
85-At-221

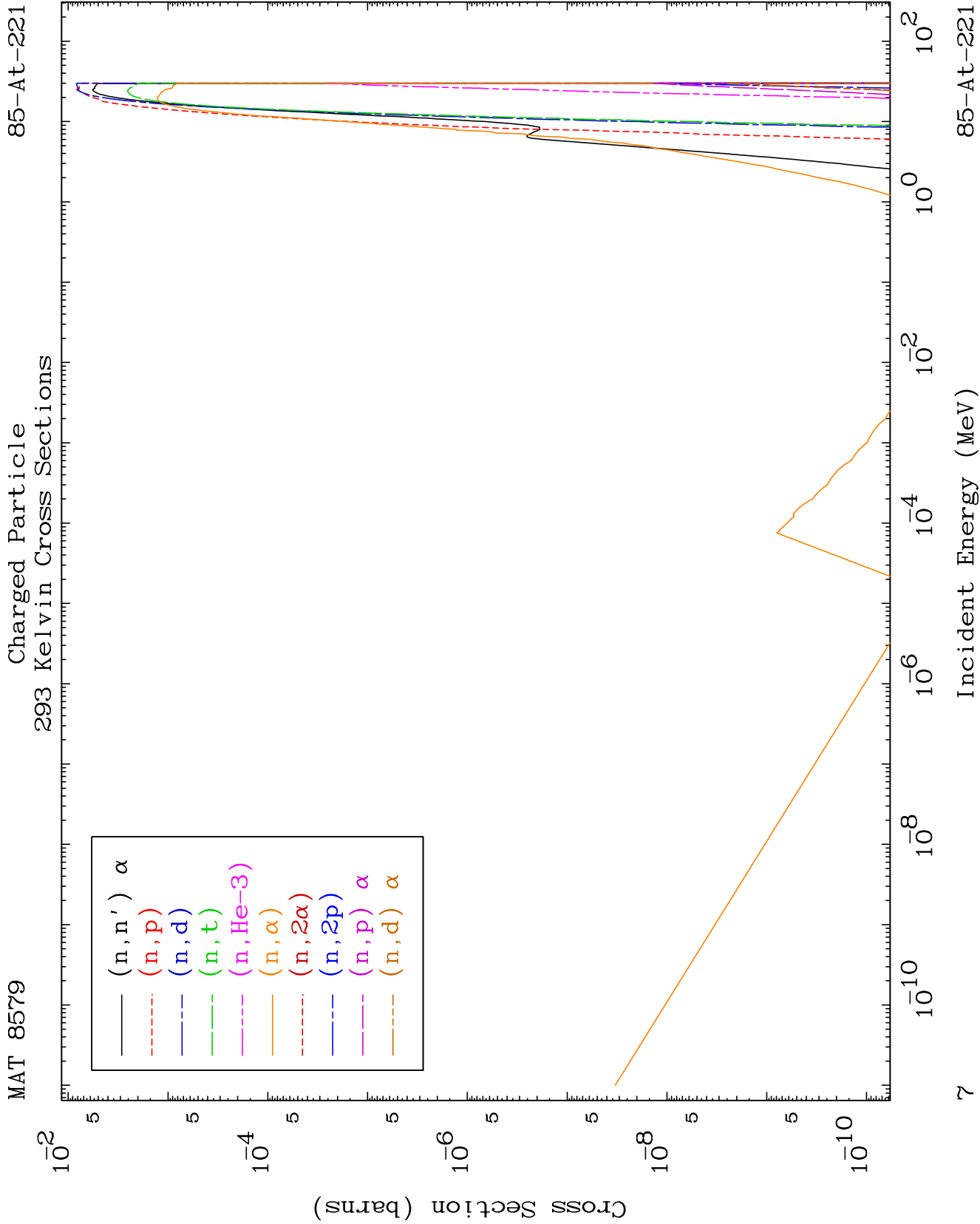


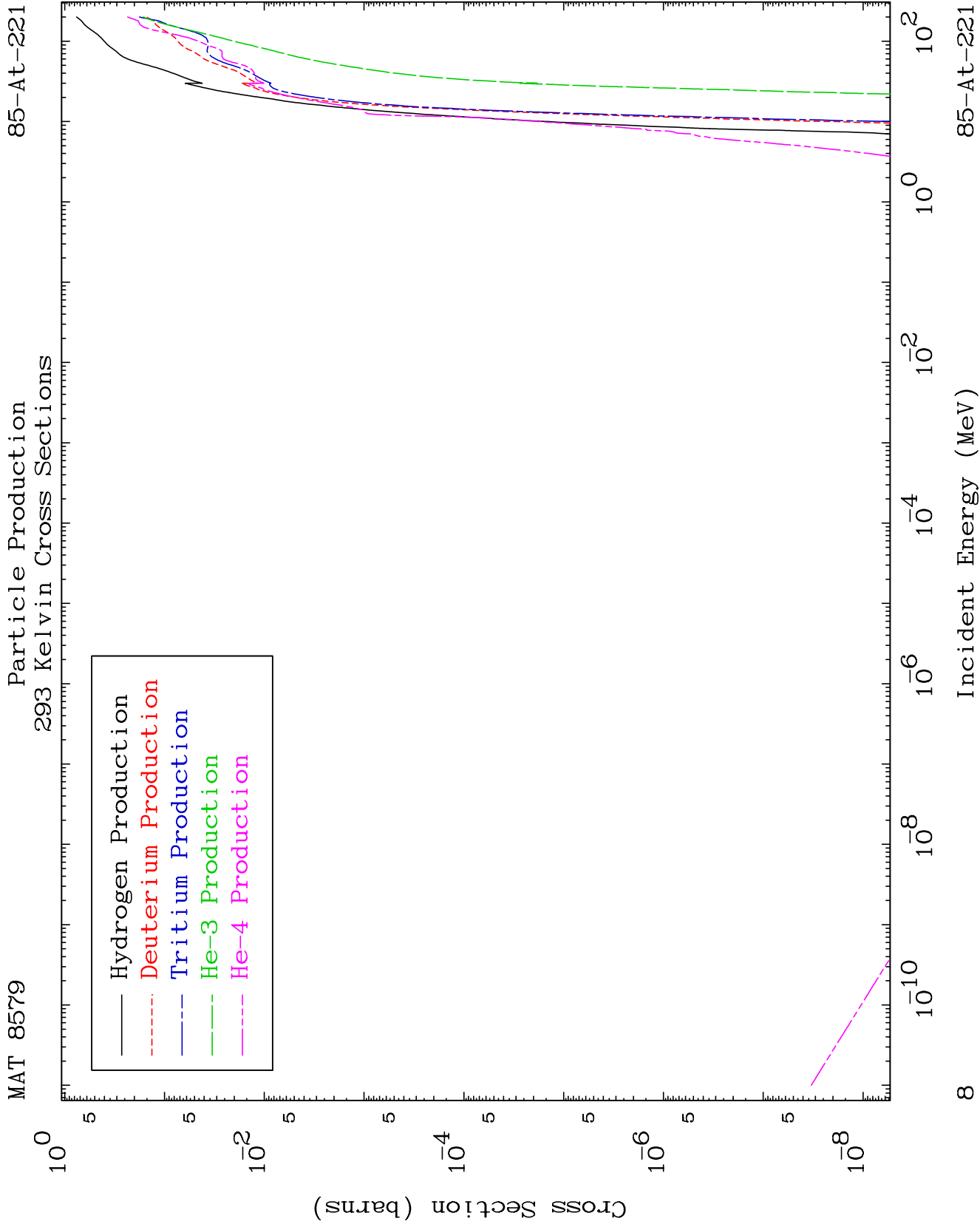


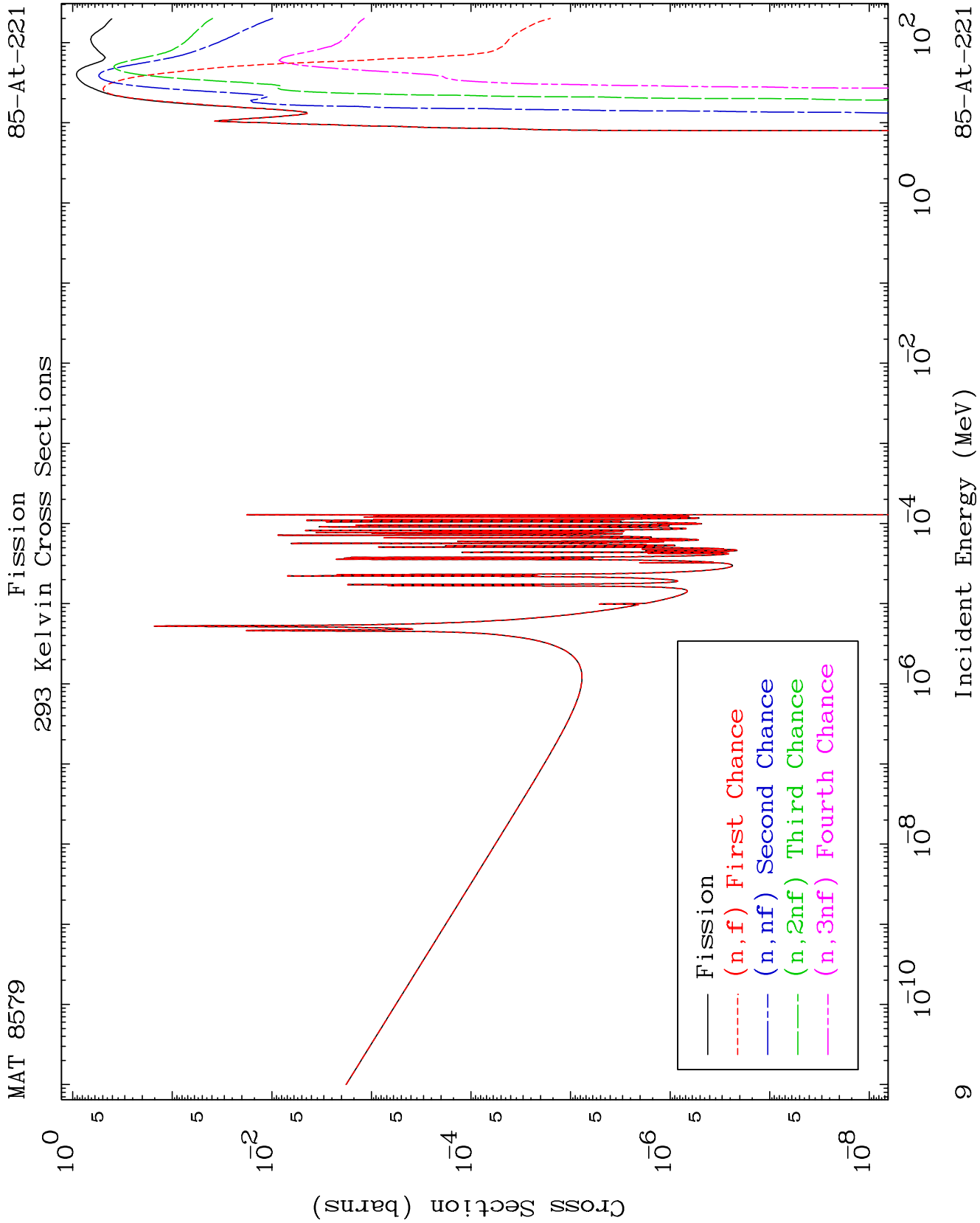


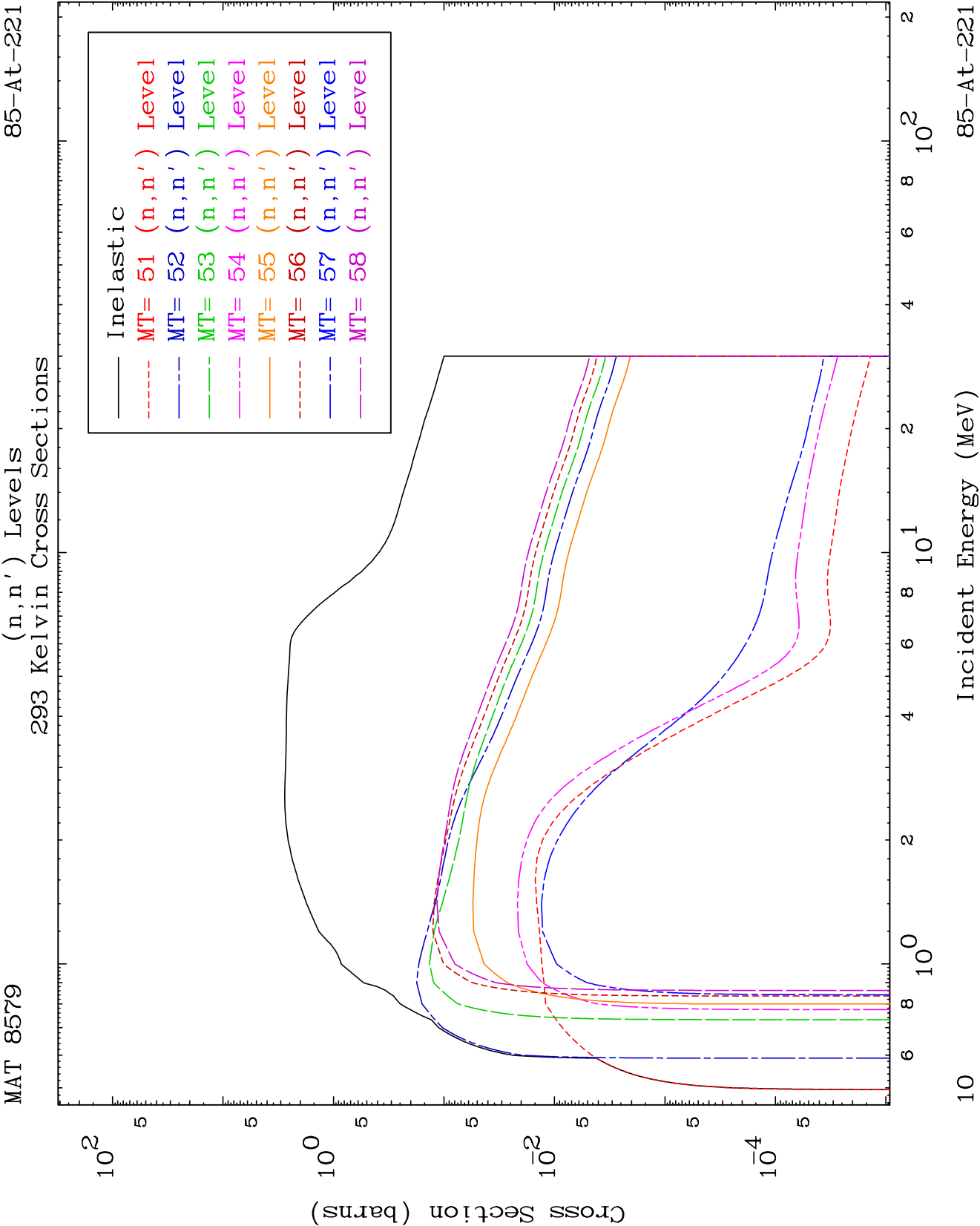




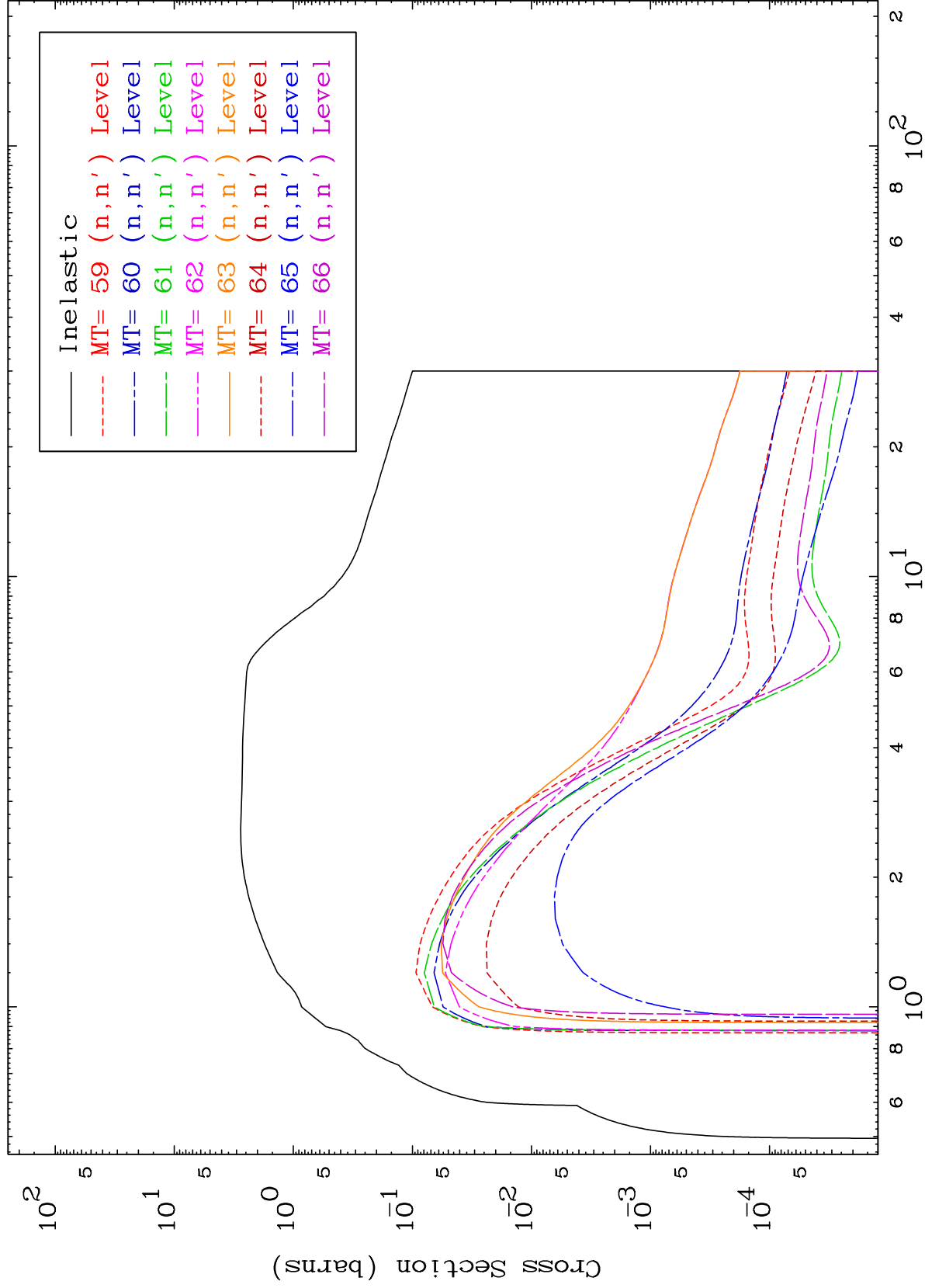


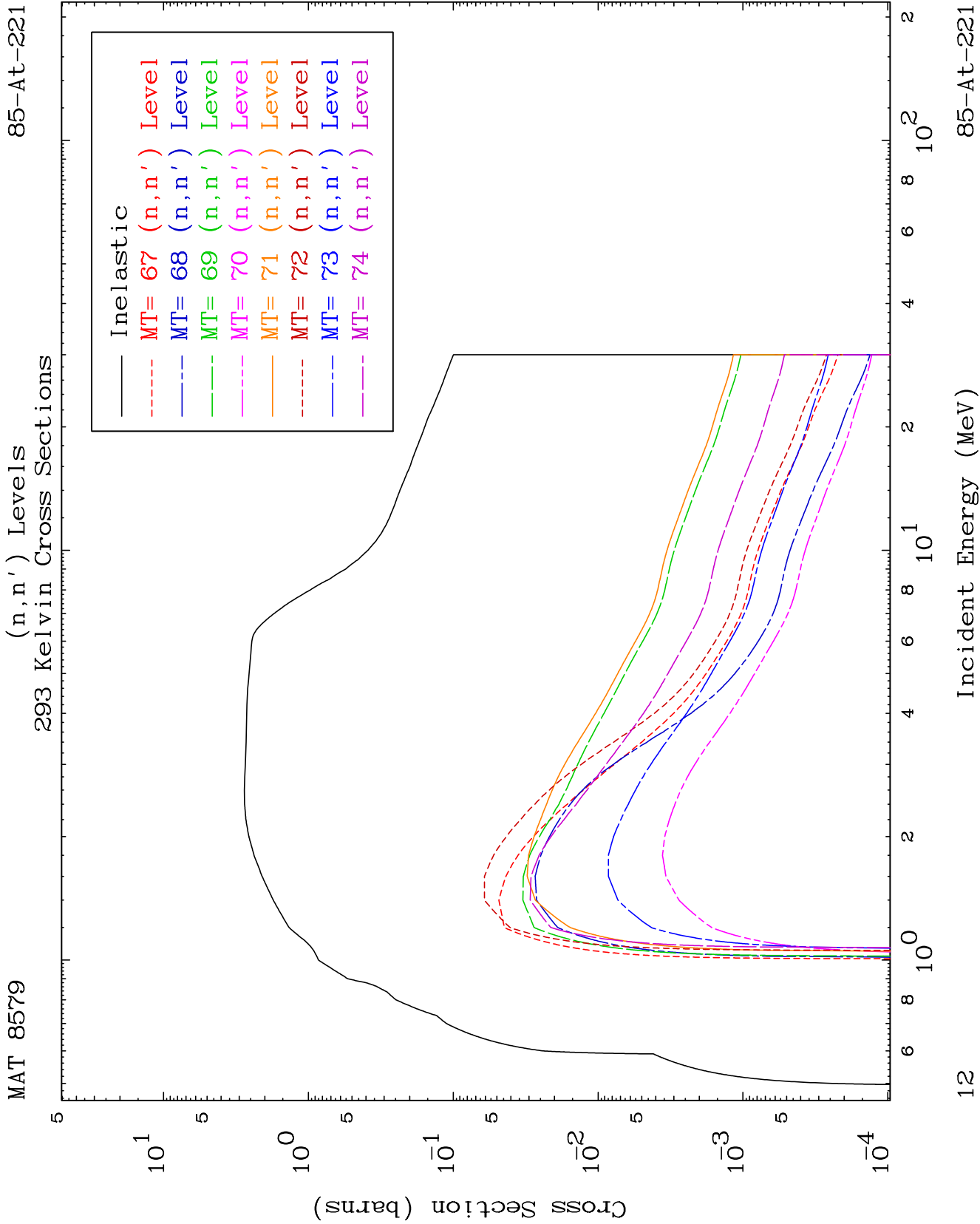






293 Kelvin Cross Sections



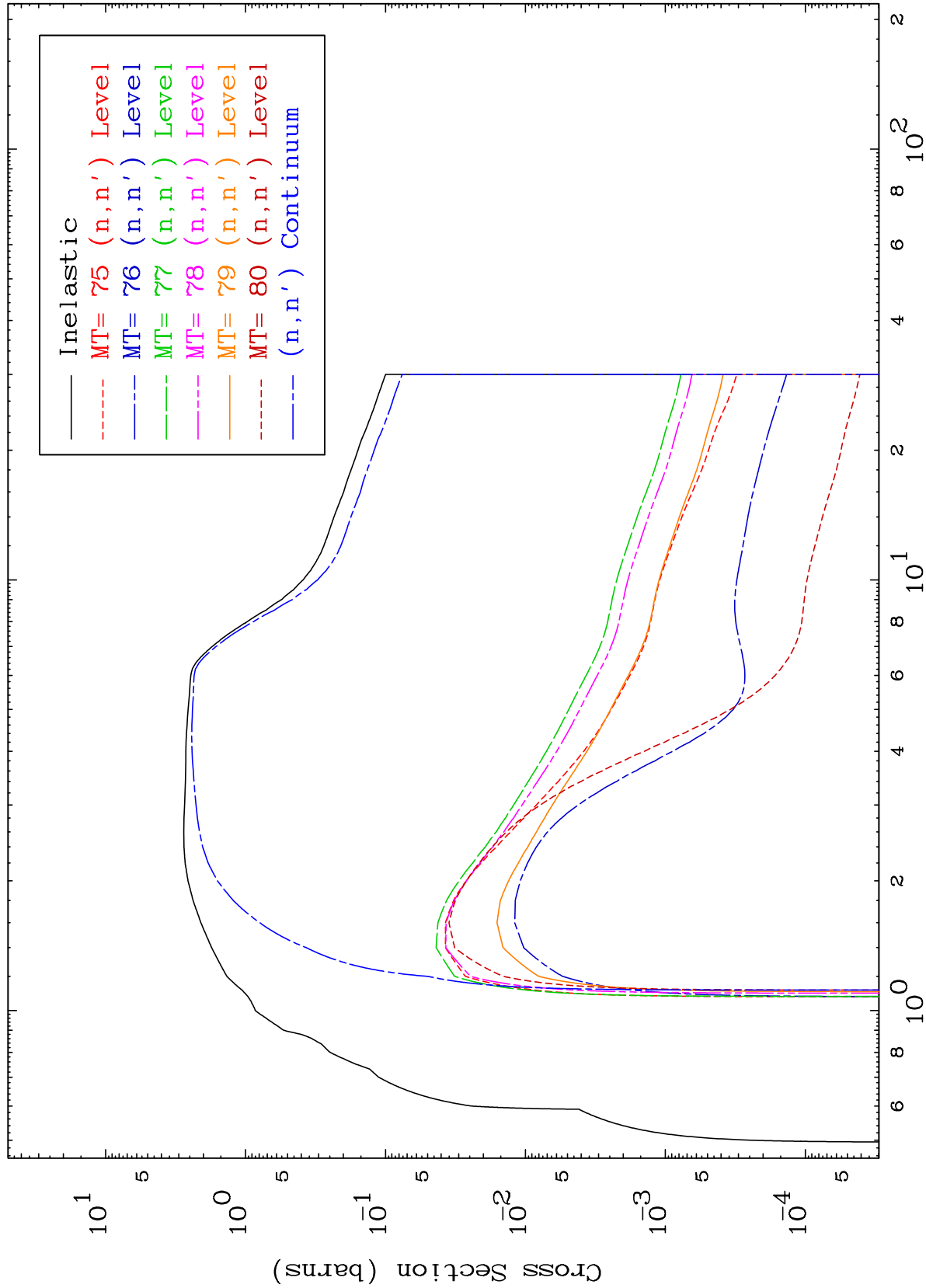


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(n,n') Levels

85-At-221

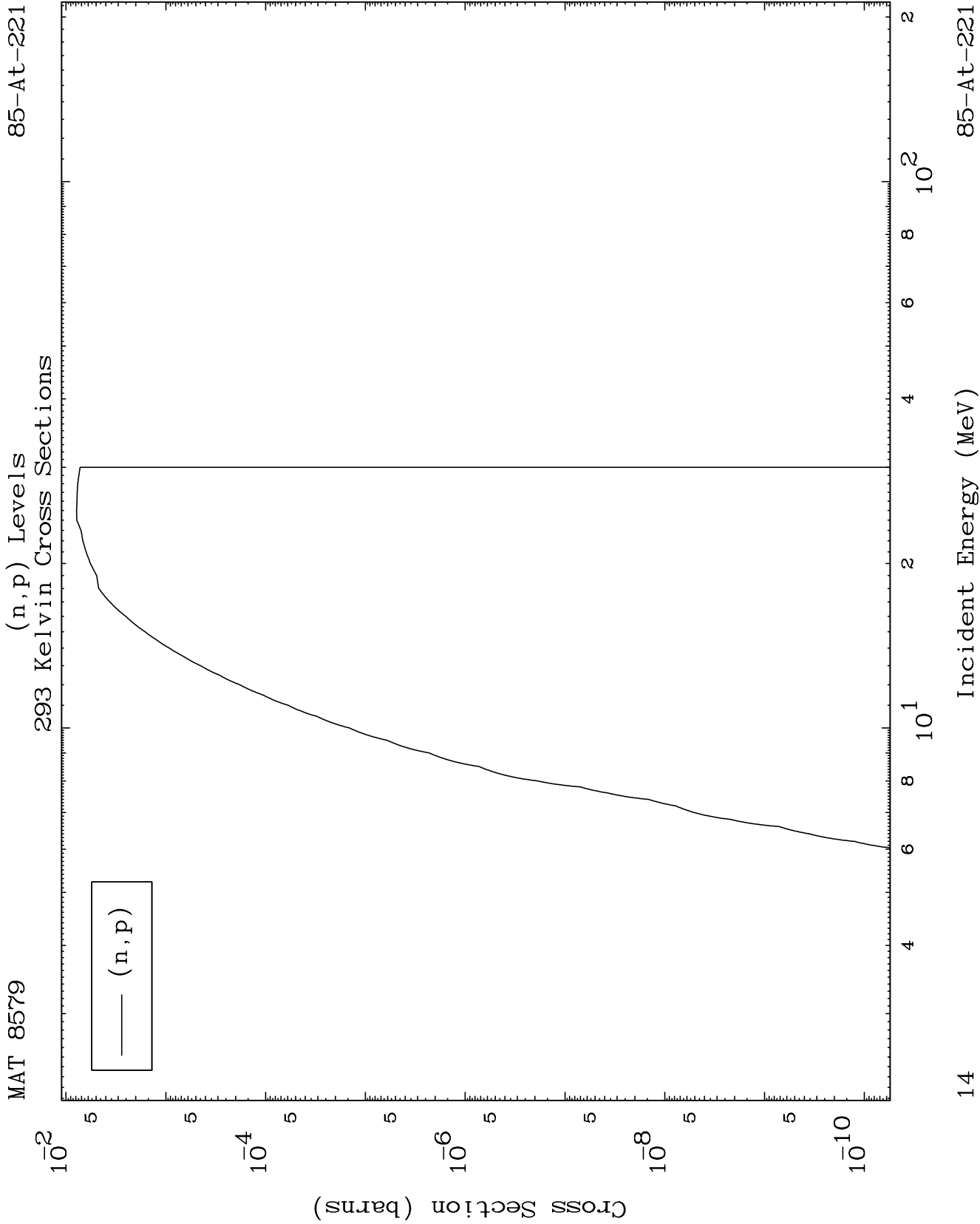
293 Kelvin Cross Sections

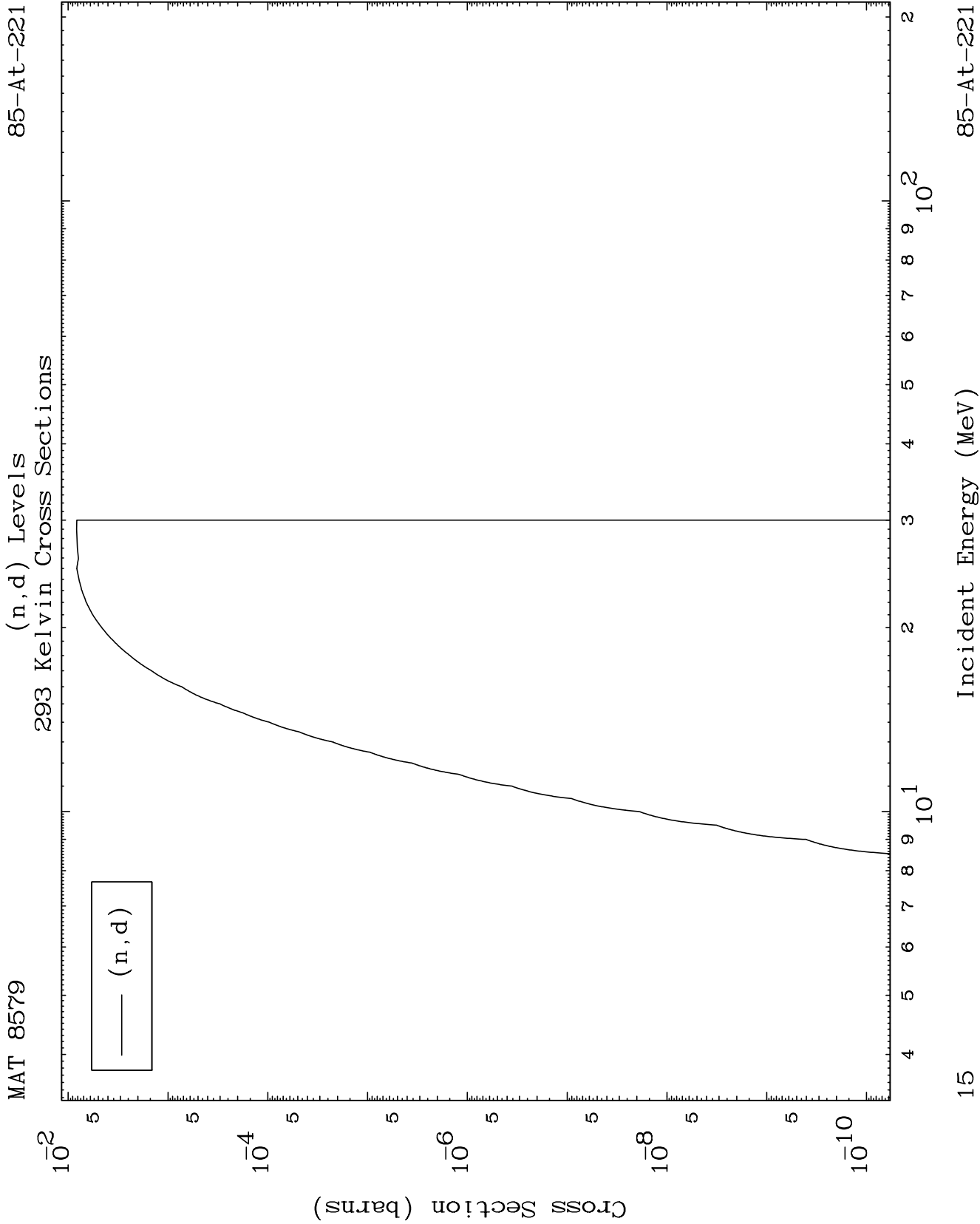


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Incident Energy (MeV)

85-At-221

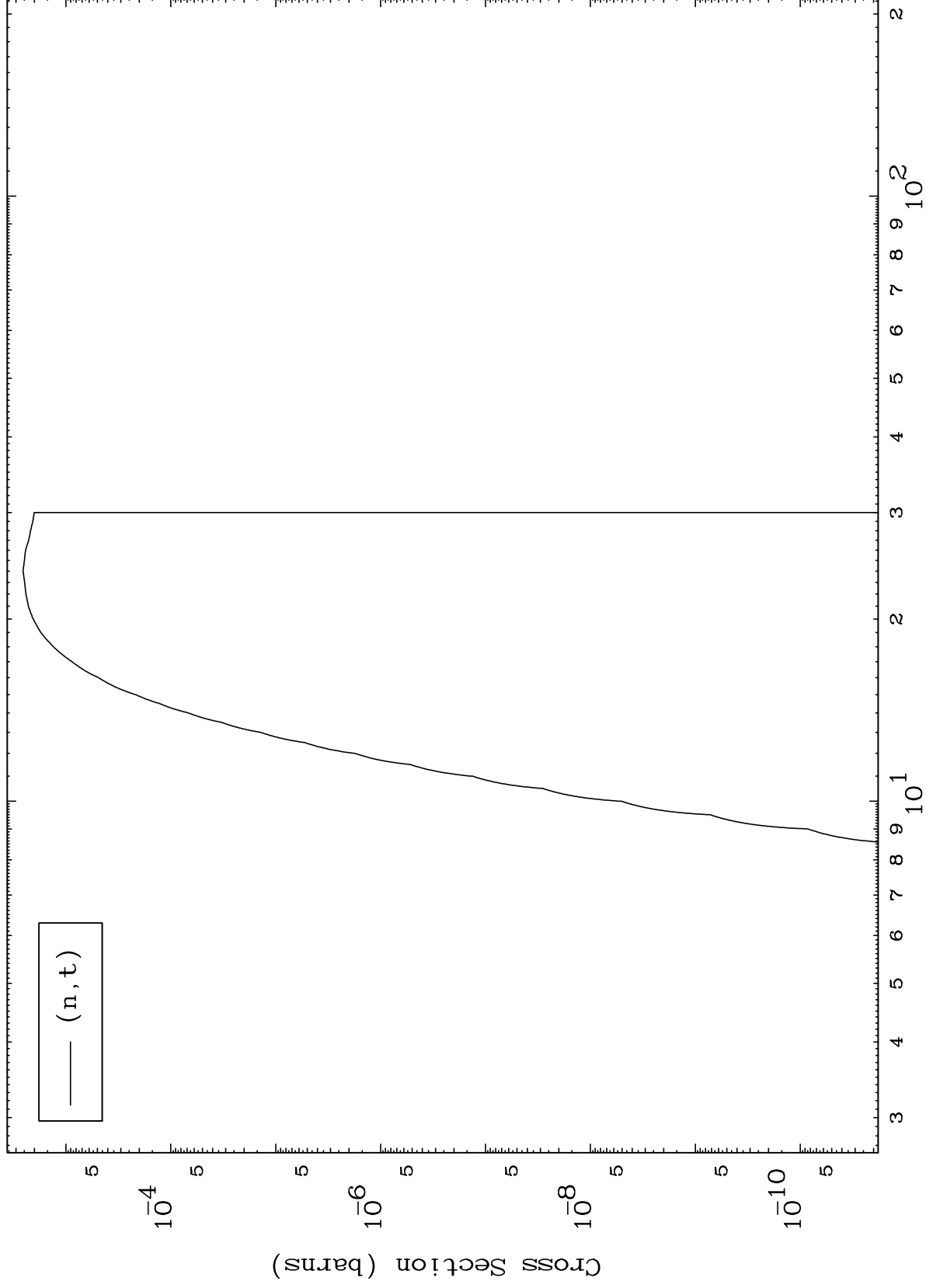




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(n,t) Levels
293 Kelvin Cross Sections

85-At-221



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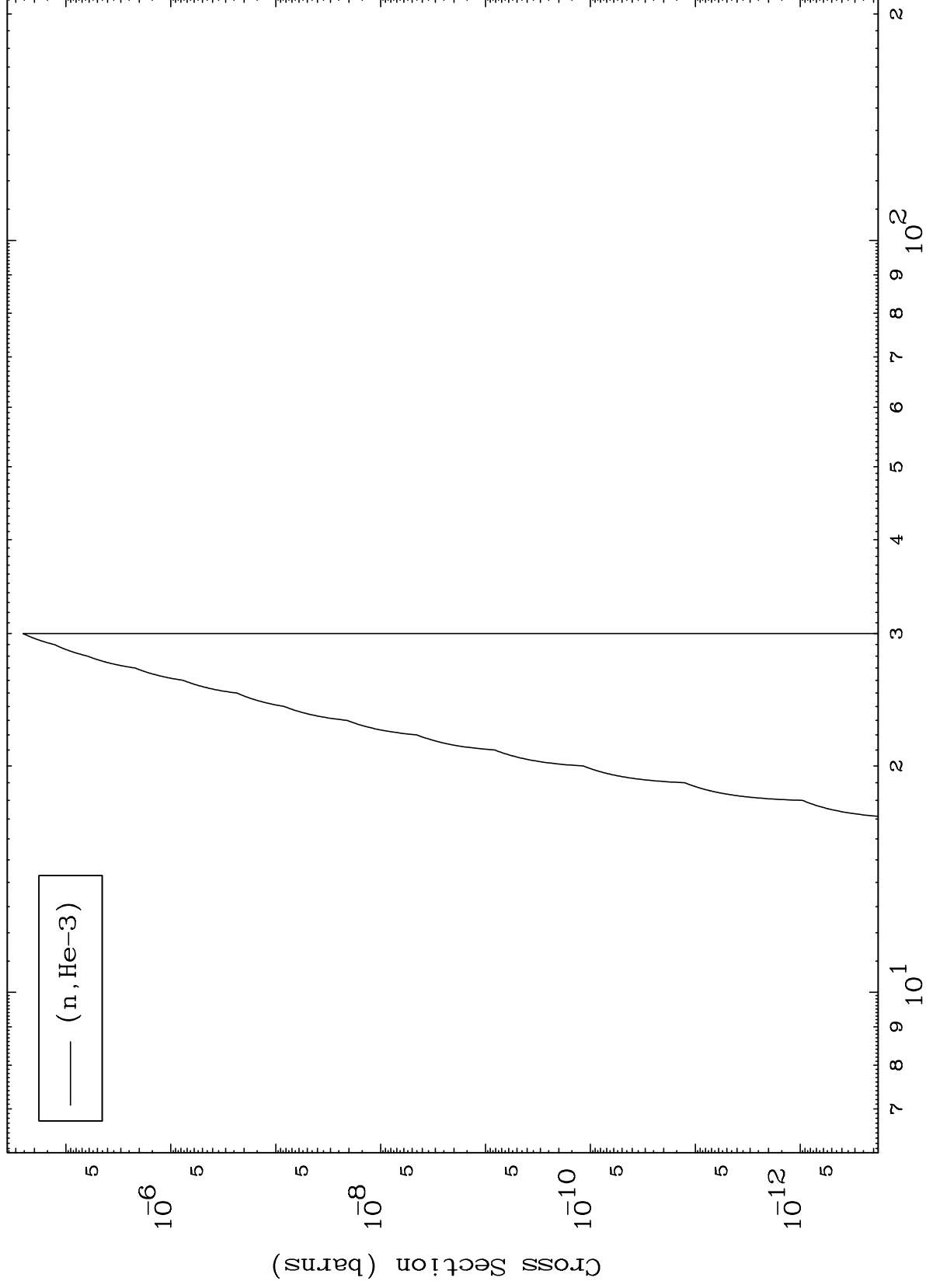
Incident Energy (MeV)

85-At-221

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85-At-221

(n,He3) Levels
293 Kelvin Cross Sections



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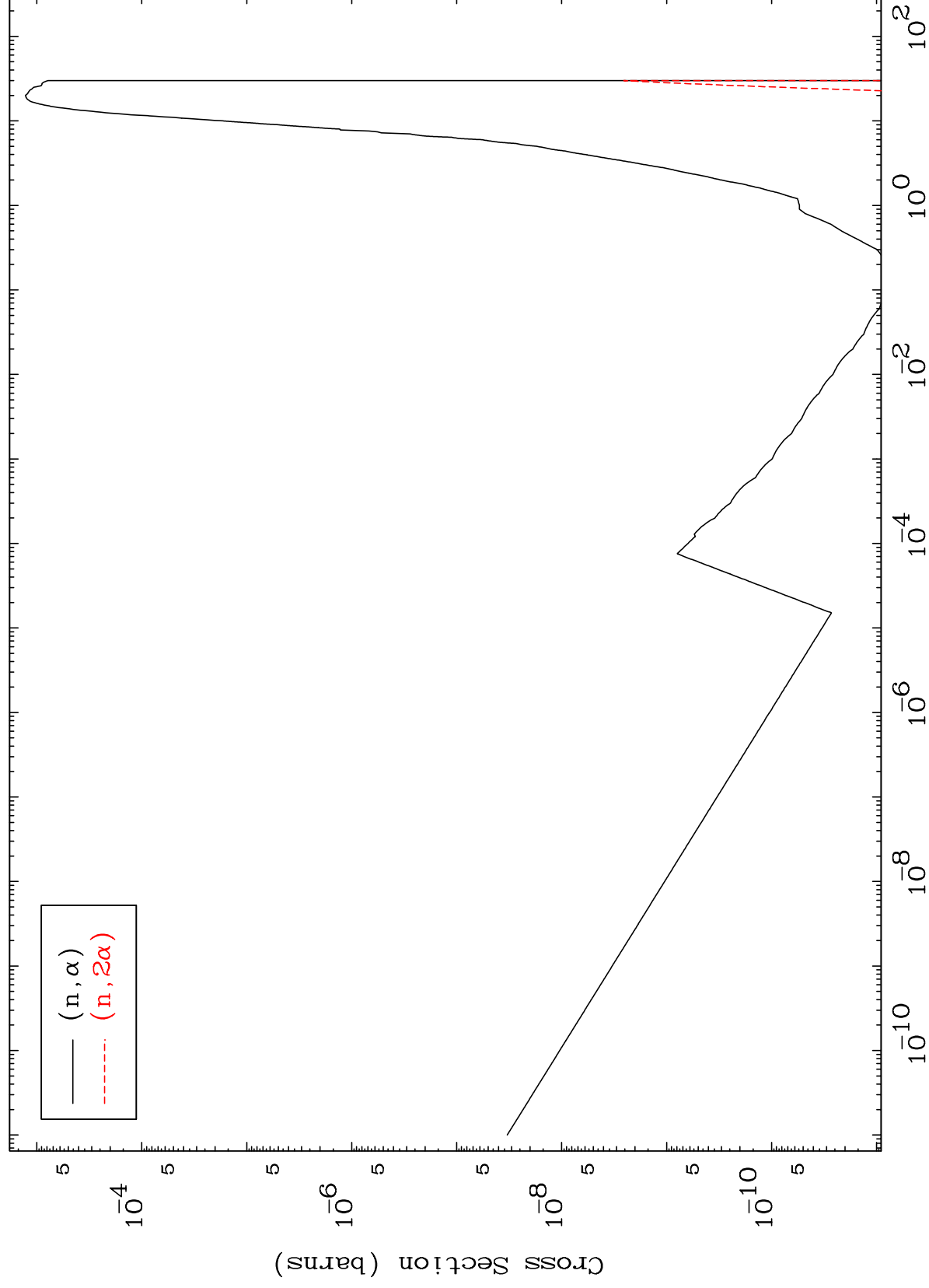
85-At-221

Incident Energy (MeV)

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(n, α) Levels
293 Kelvin Cross Sections

85-At-221



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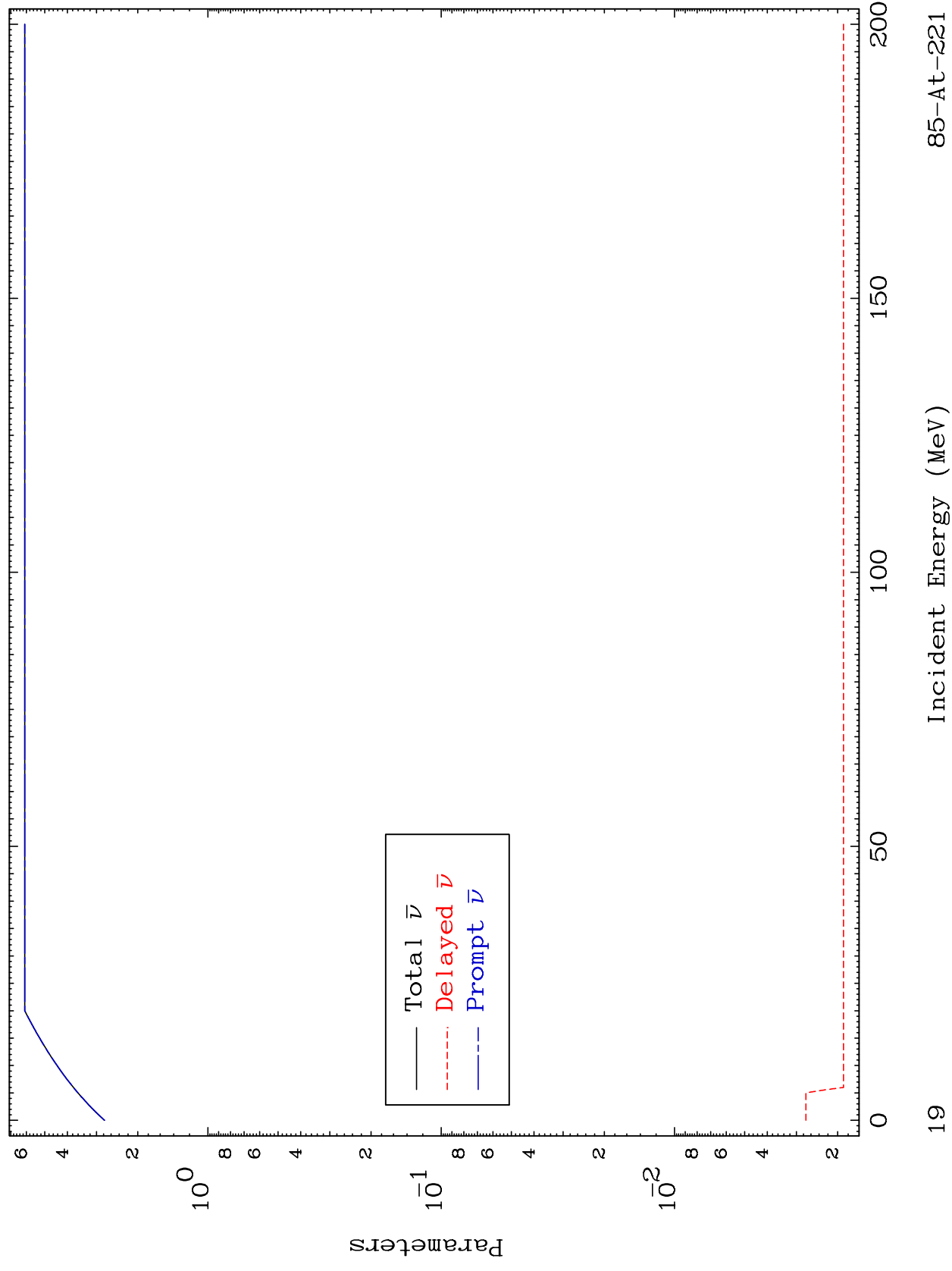
Incident Energy (MeV)

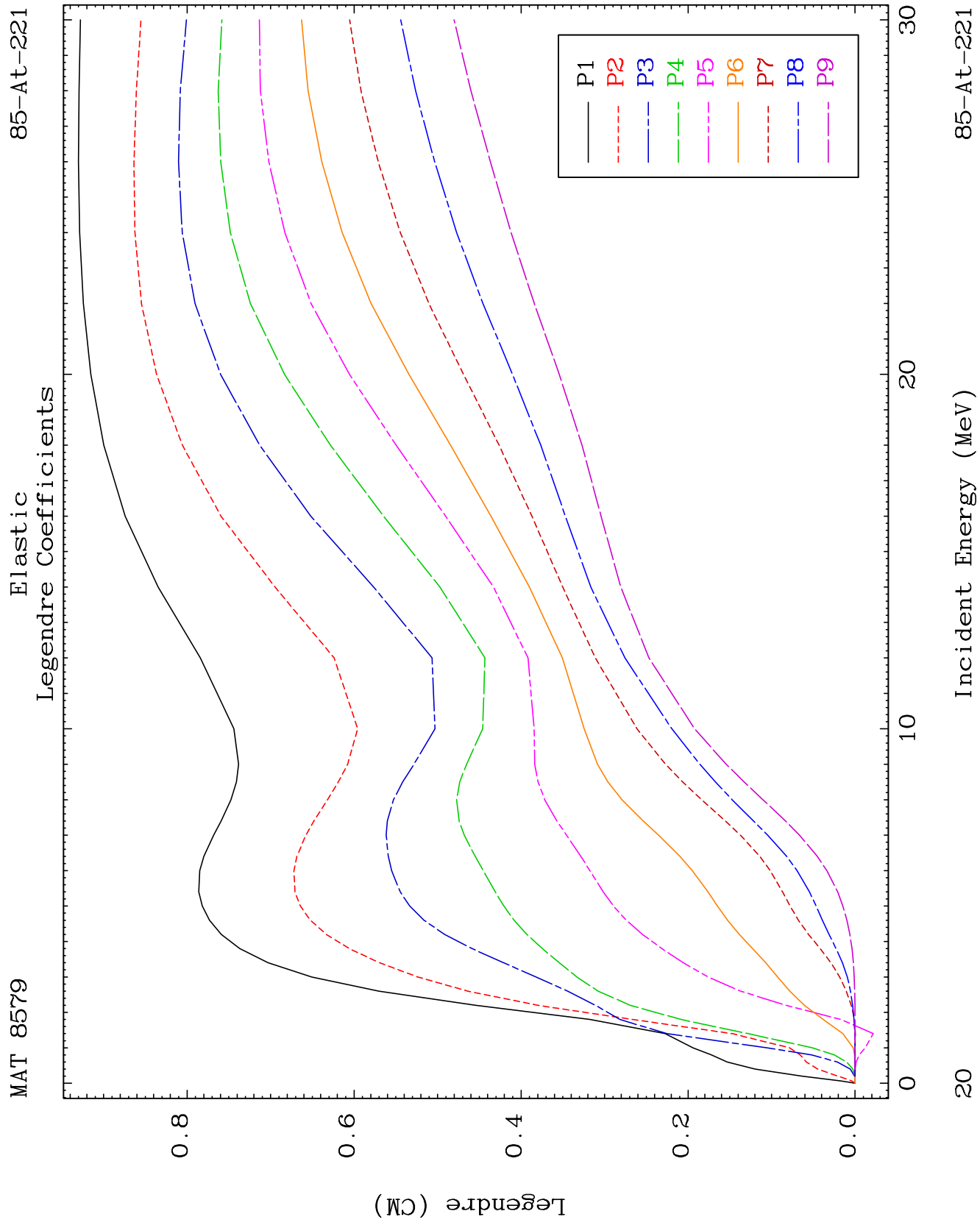
85-At-221

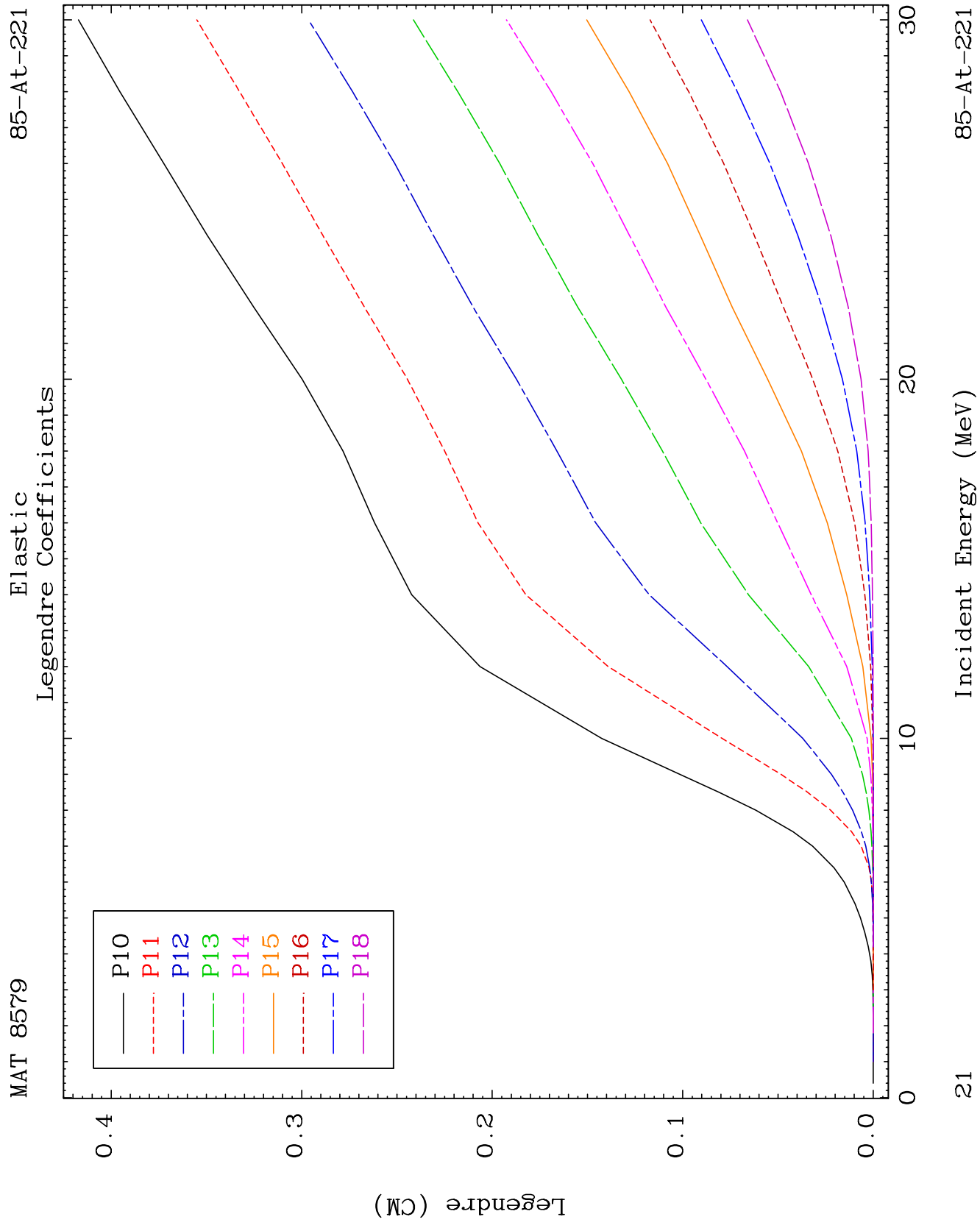
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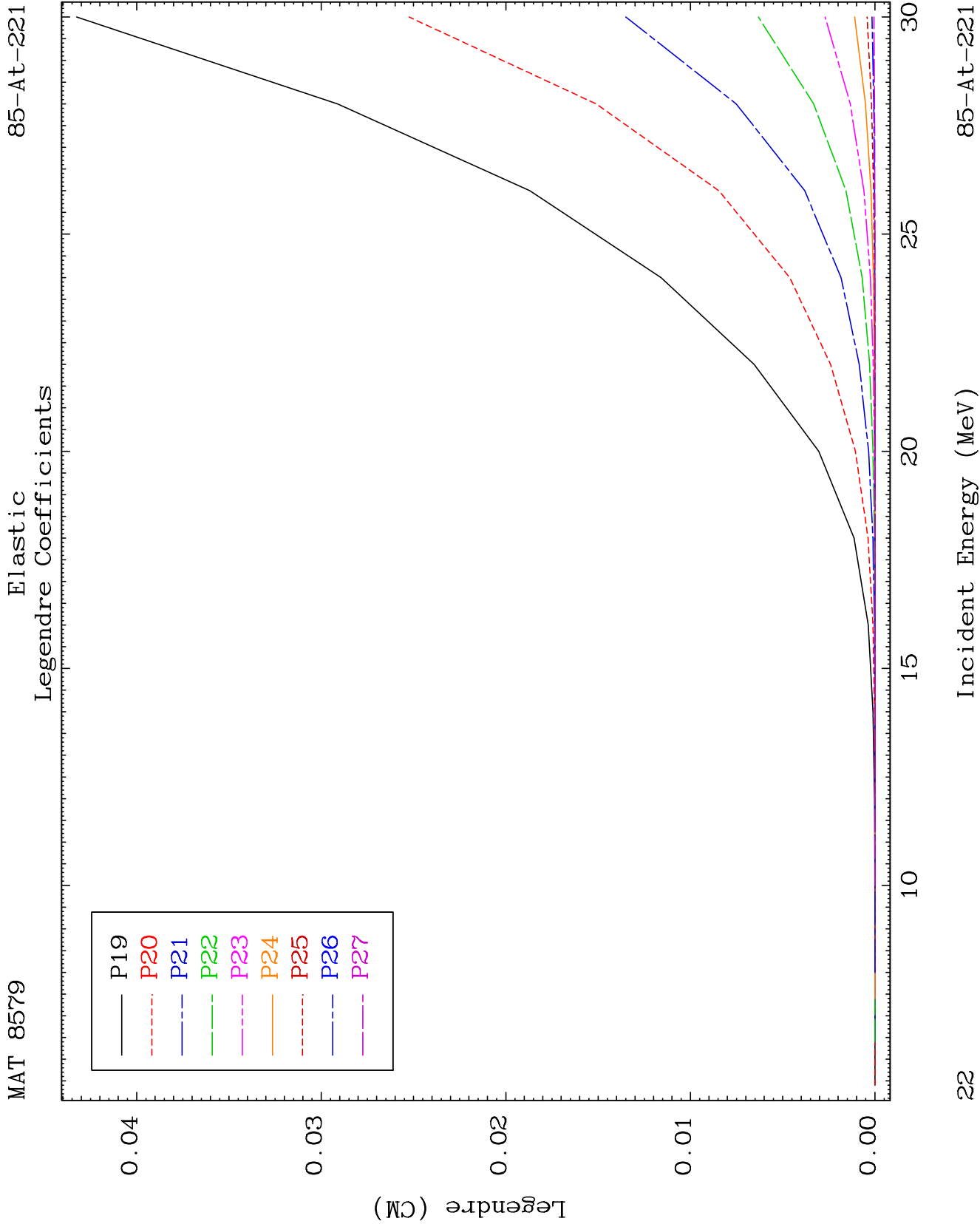
Energy Release Parameters

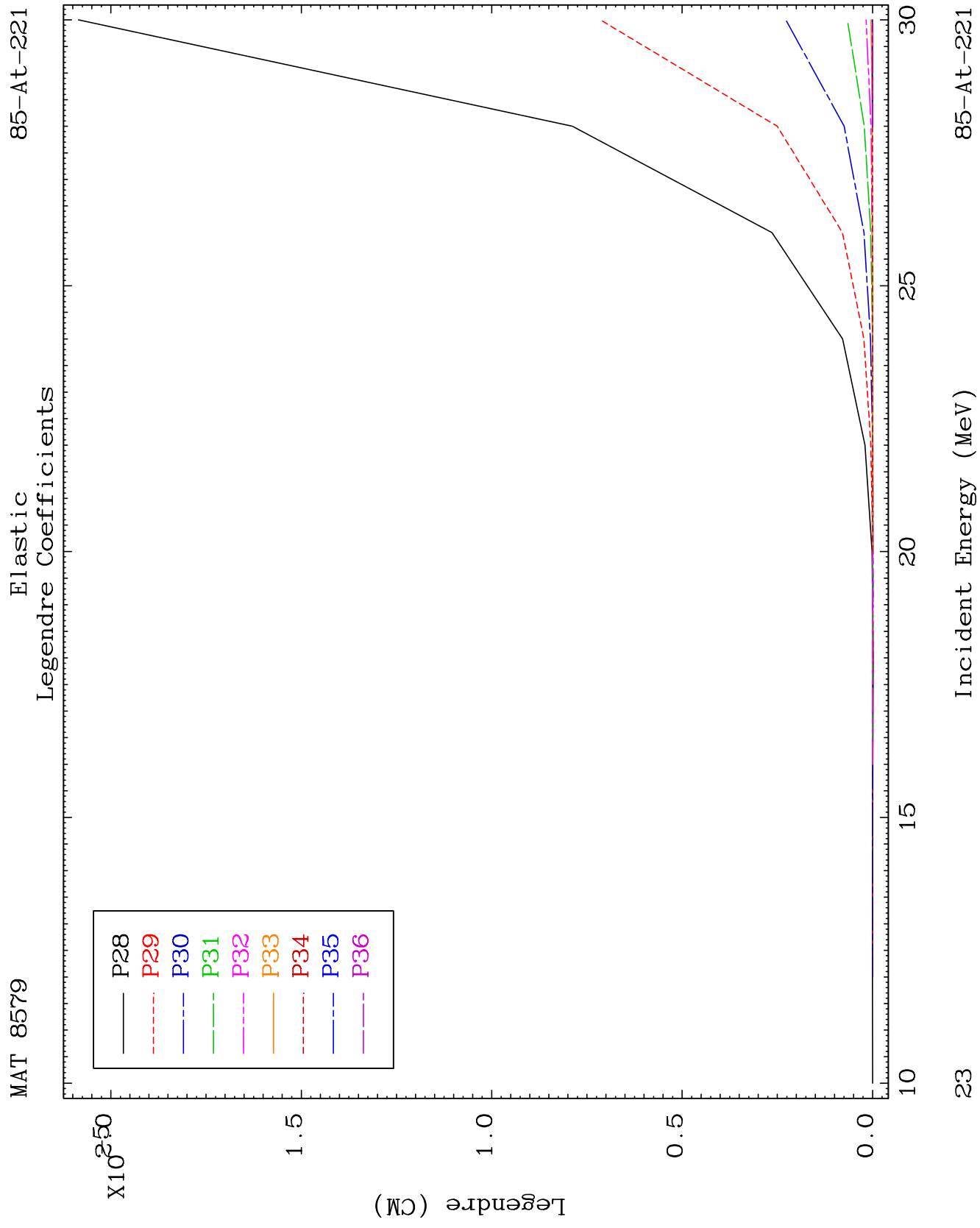
85-At-221

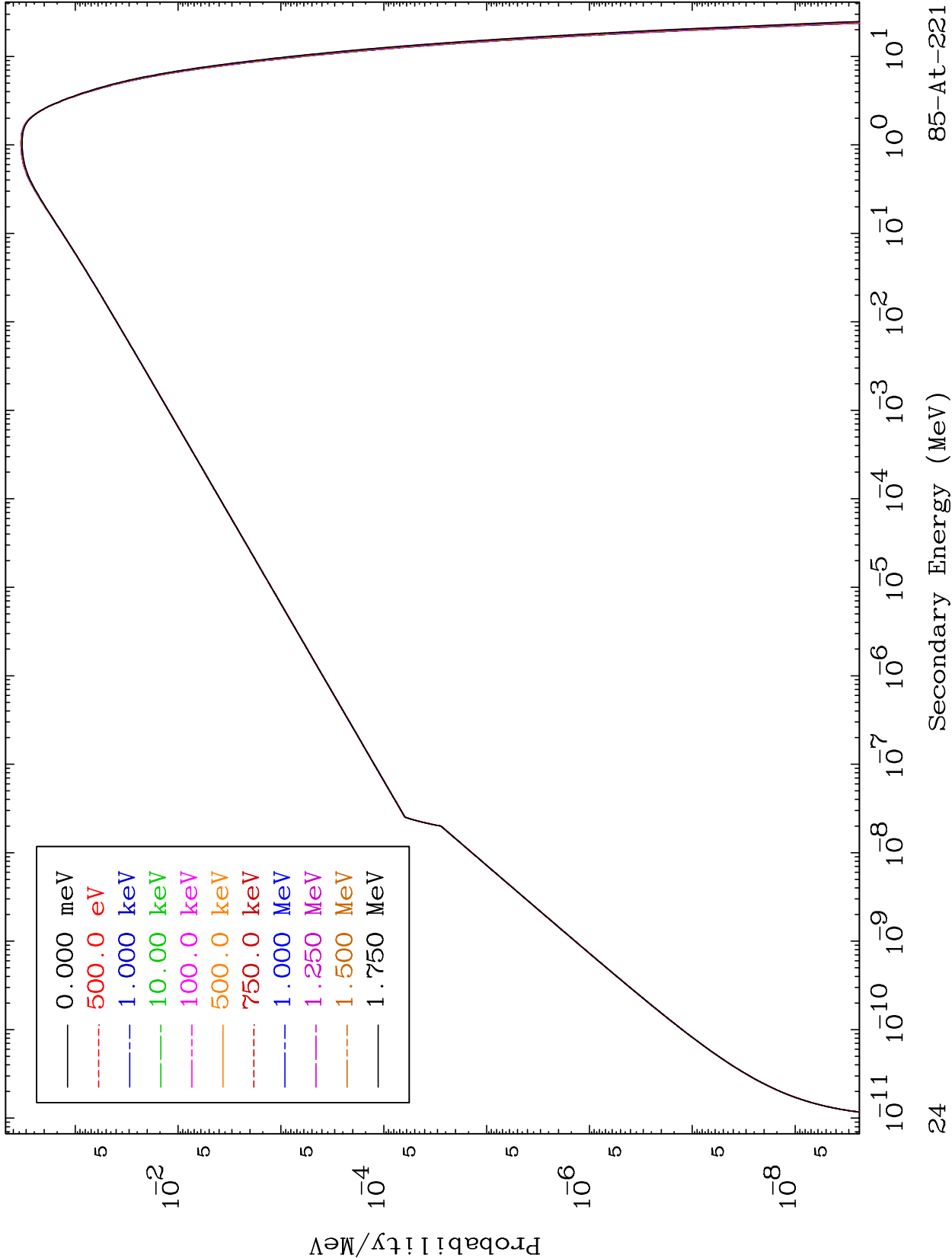








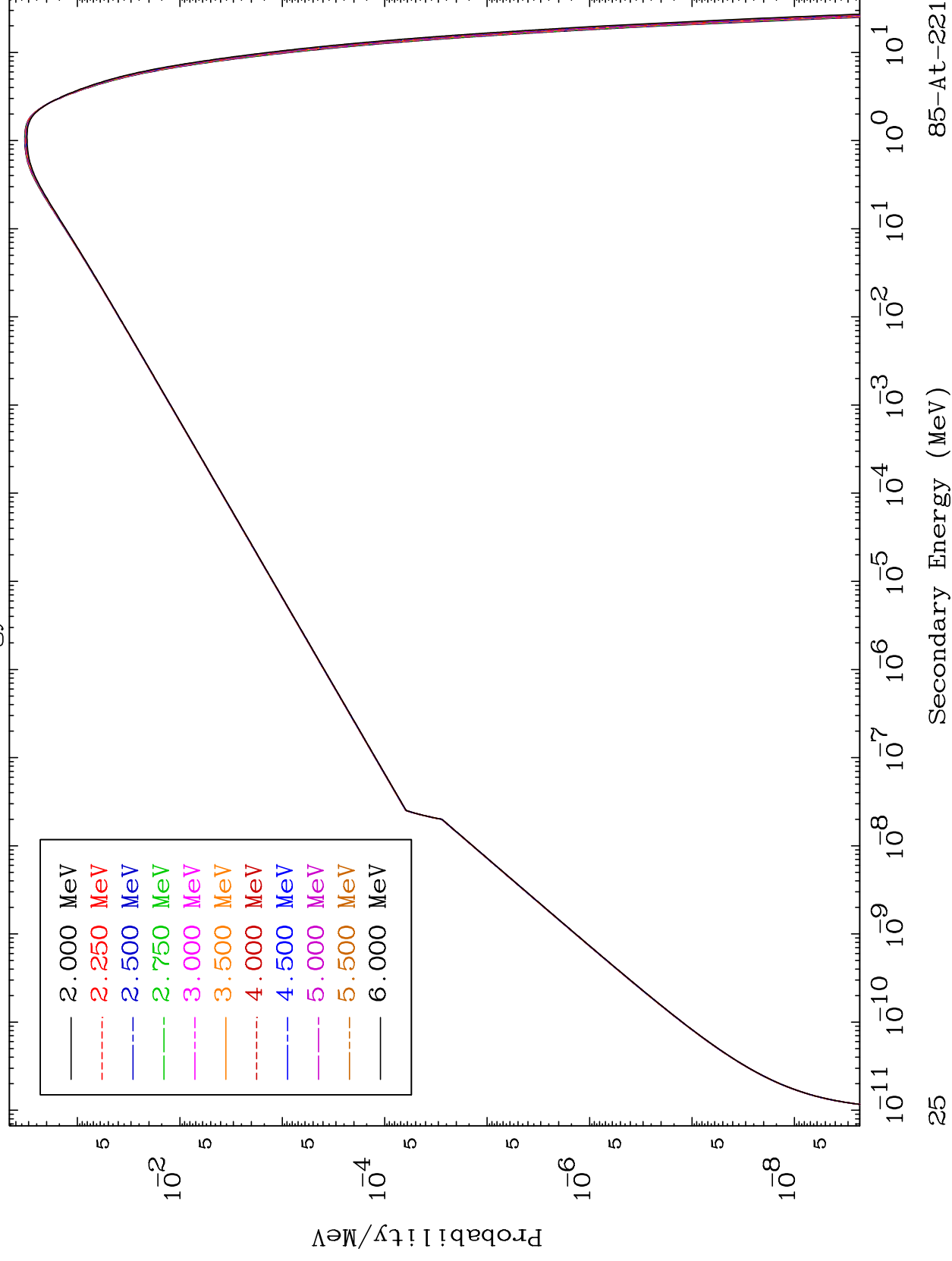


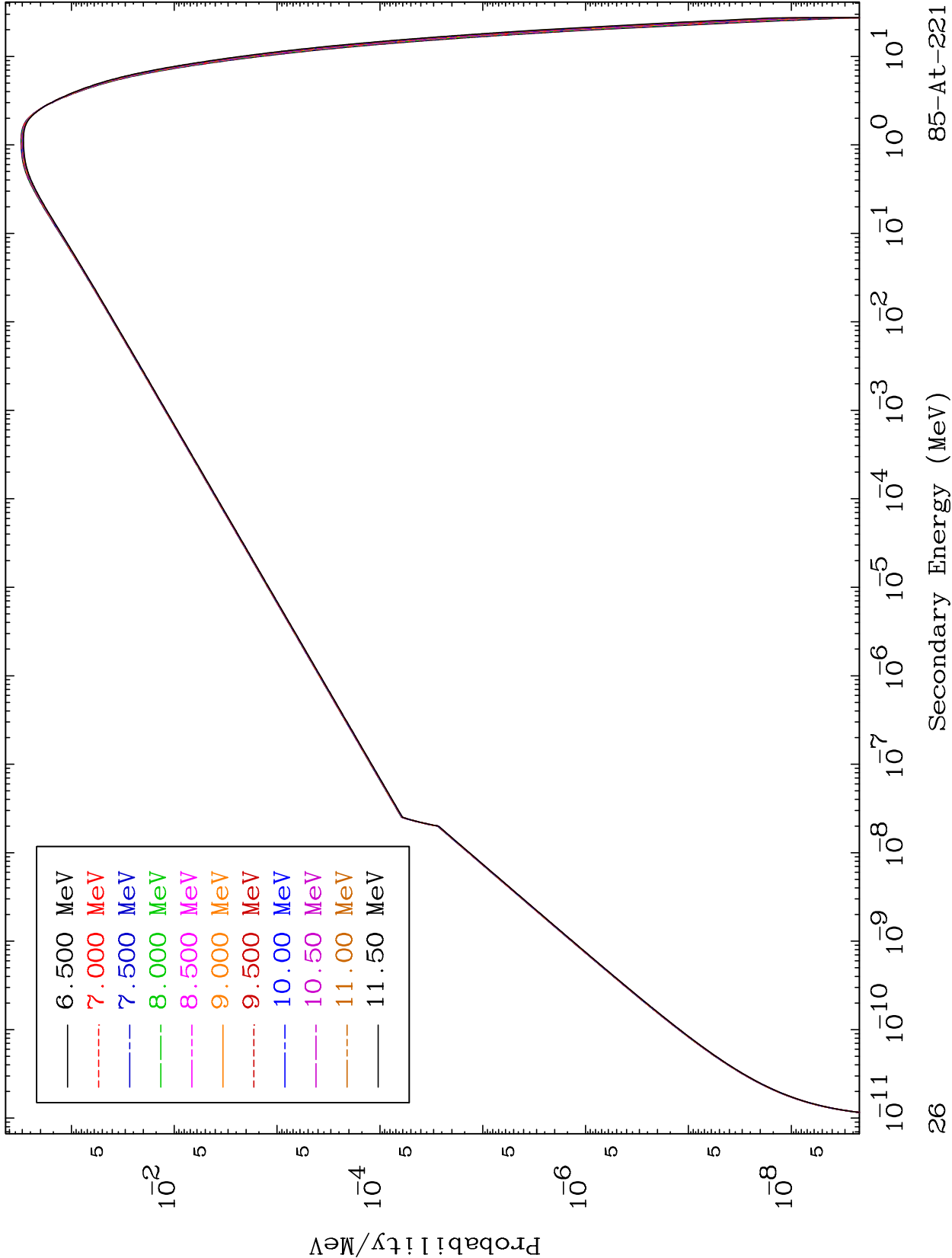


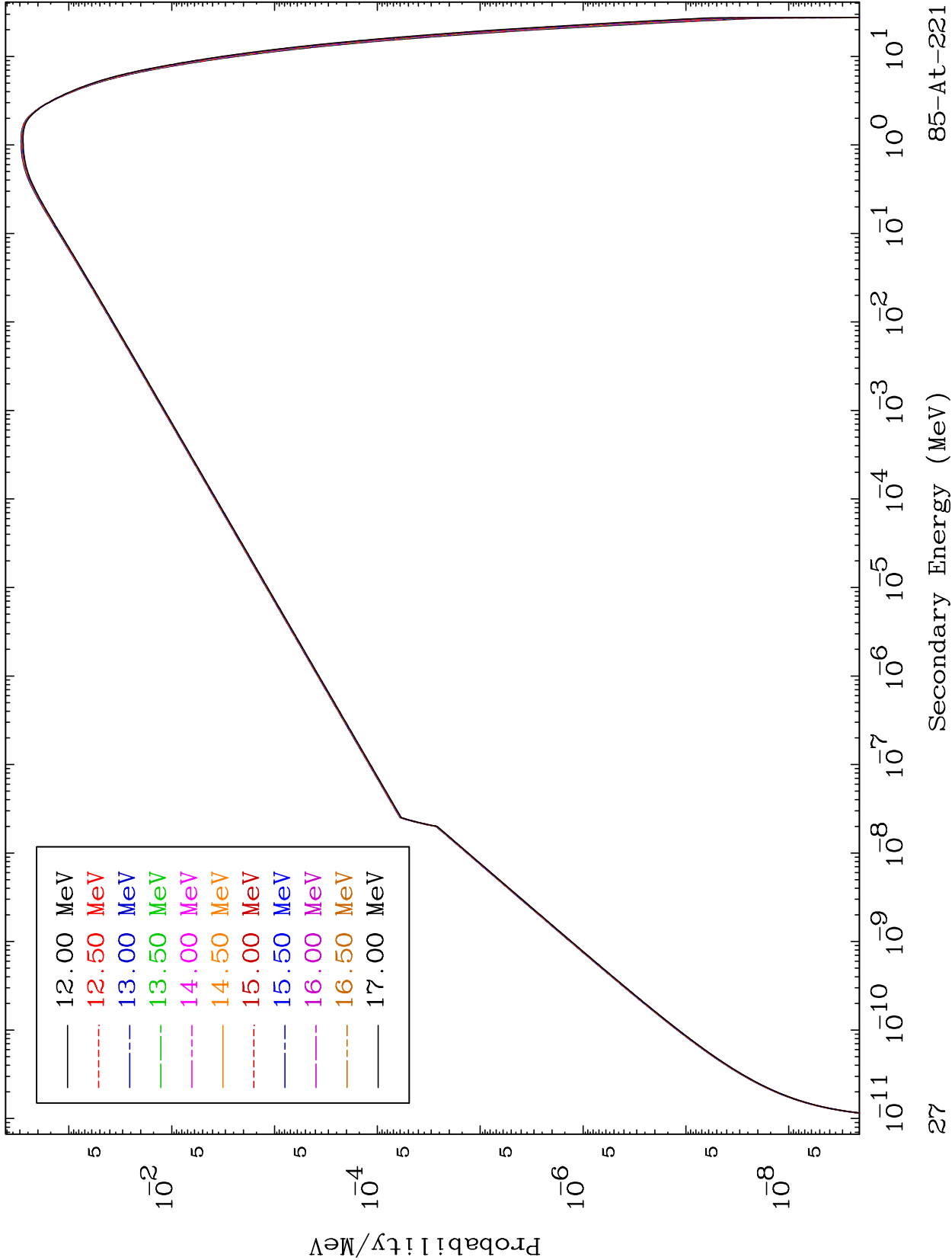
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Fission Energy Distributions



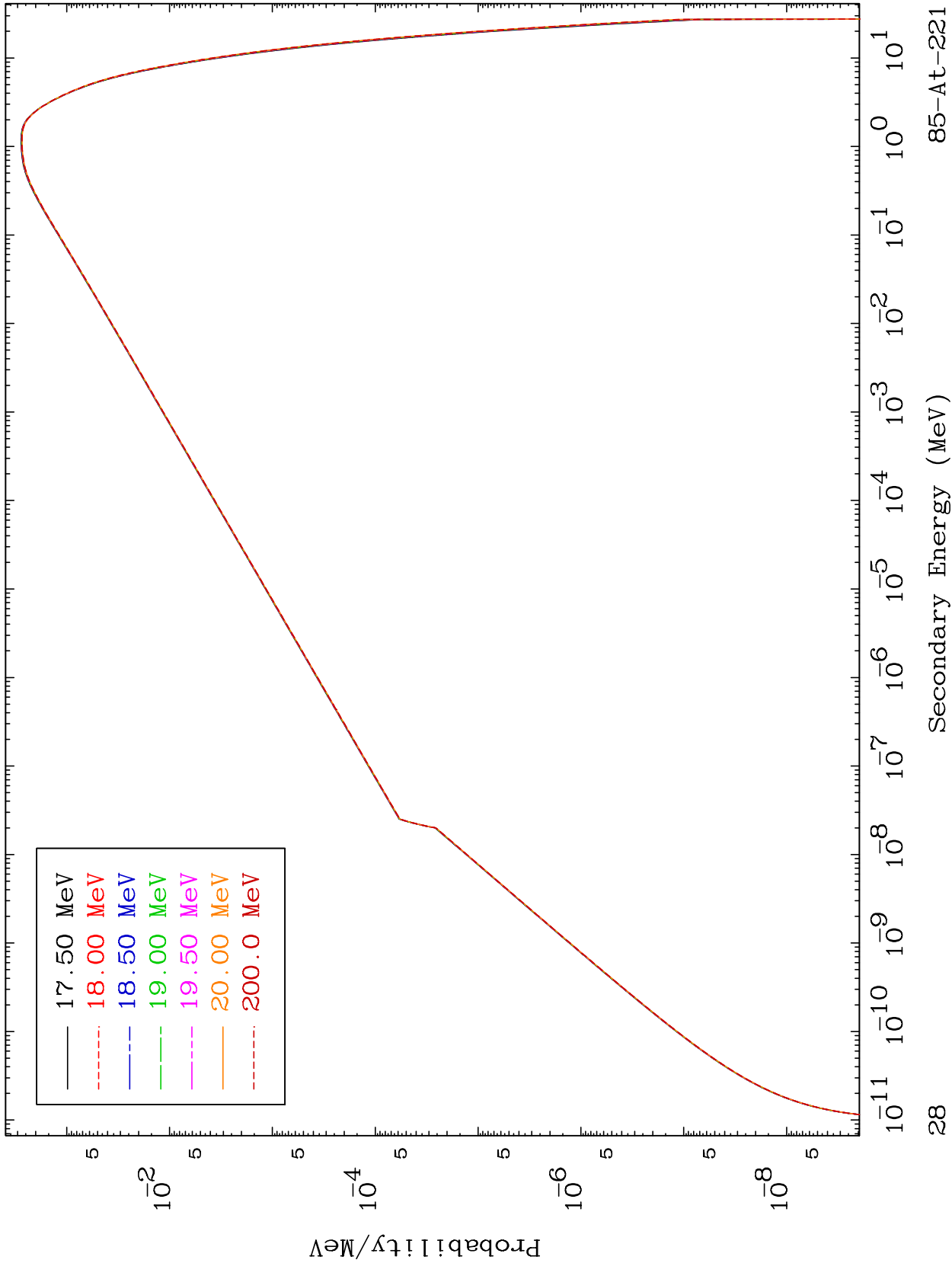




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Fission
Energy Distributions

85-At-221

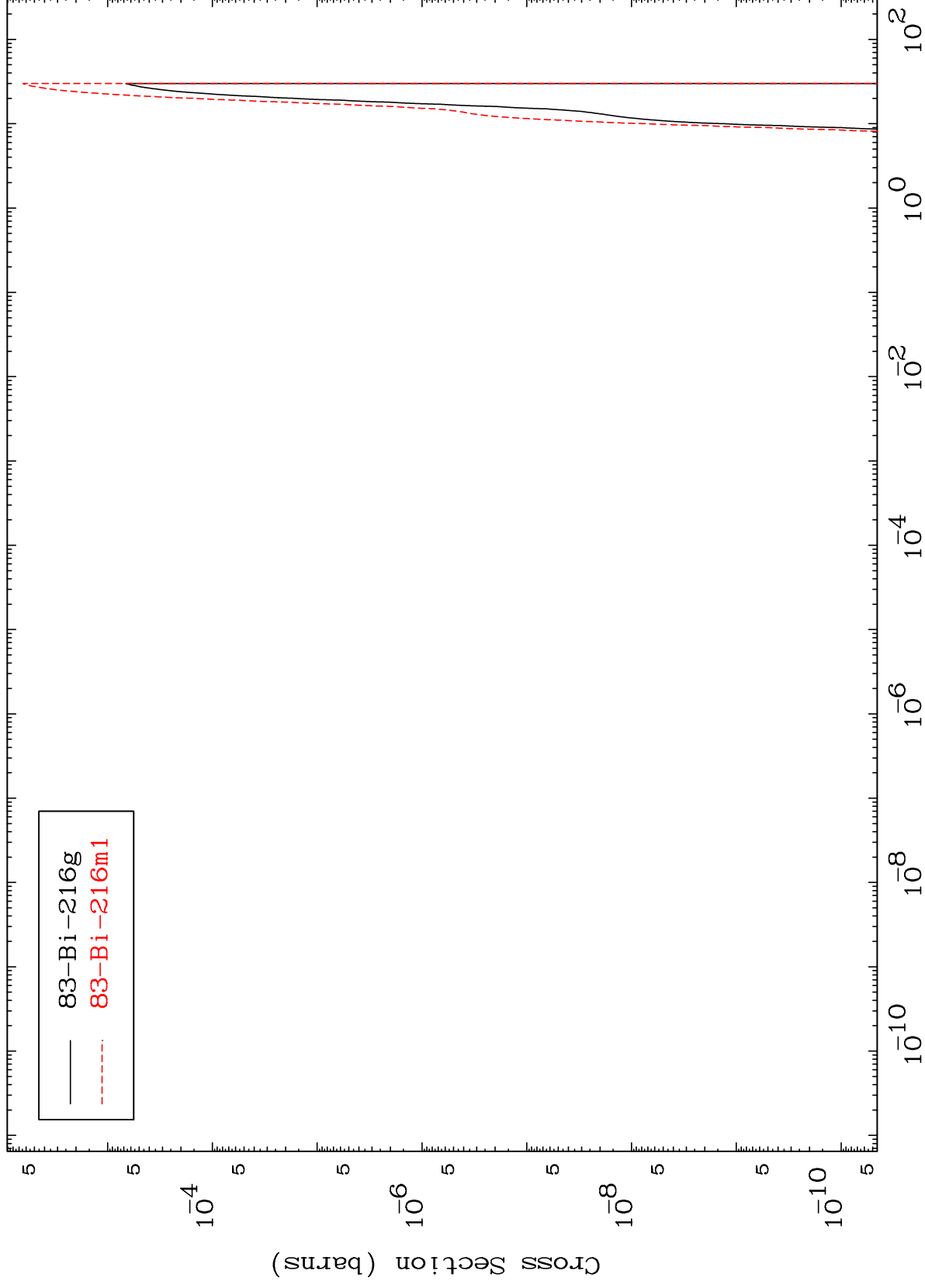


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(n,2n) α

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Radionuclide Production Cross Section



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Incident Energy (MeV)

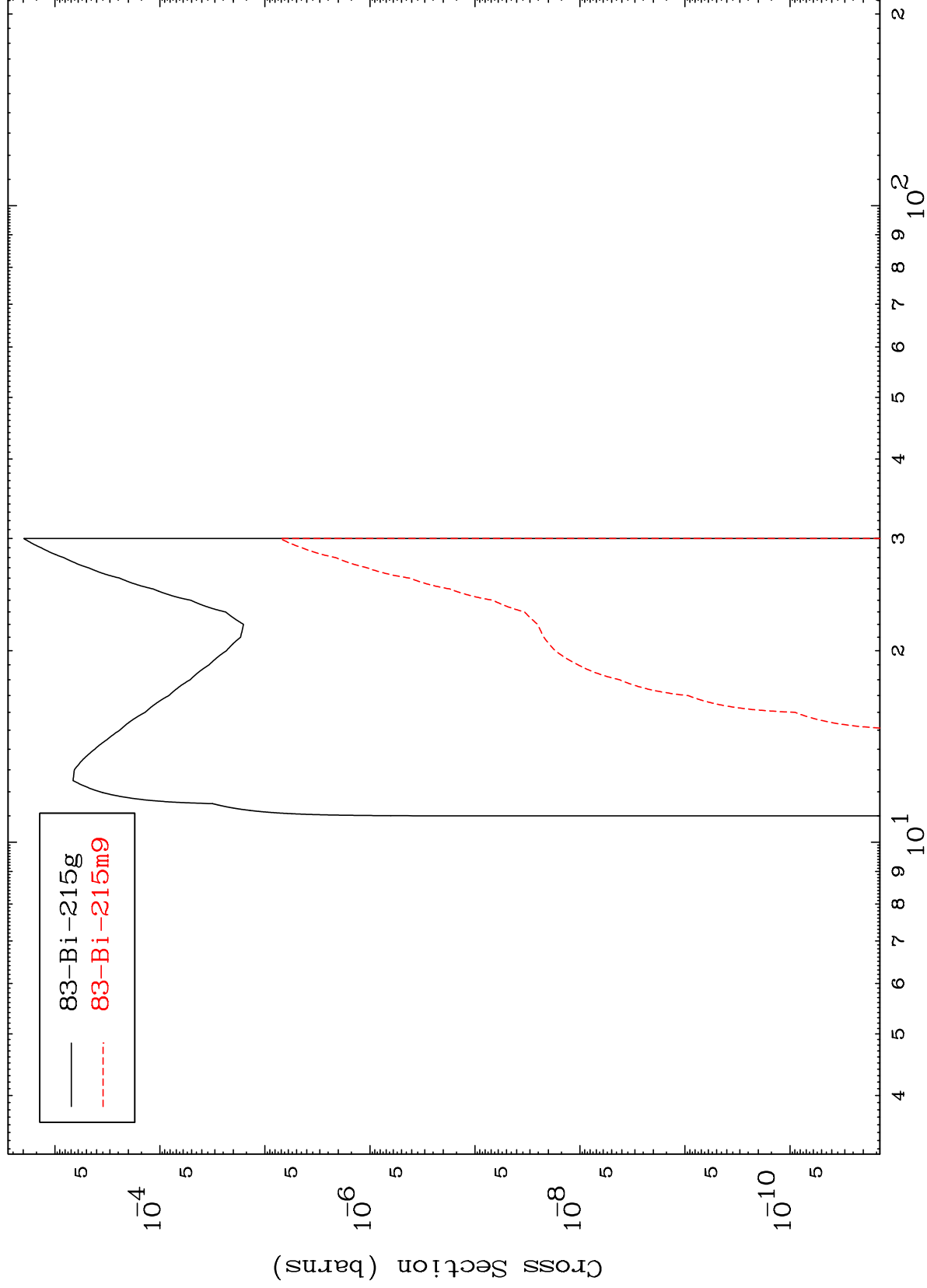
85-At-221

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(n,3n) α

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Radionuclide Production Cross Section



30

Incident Energy (MeV)

85-At-221