

Program EVALPLOT
(Version 2021-1)

by

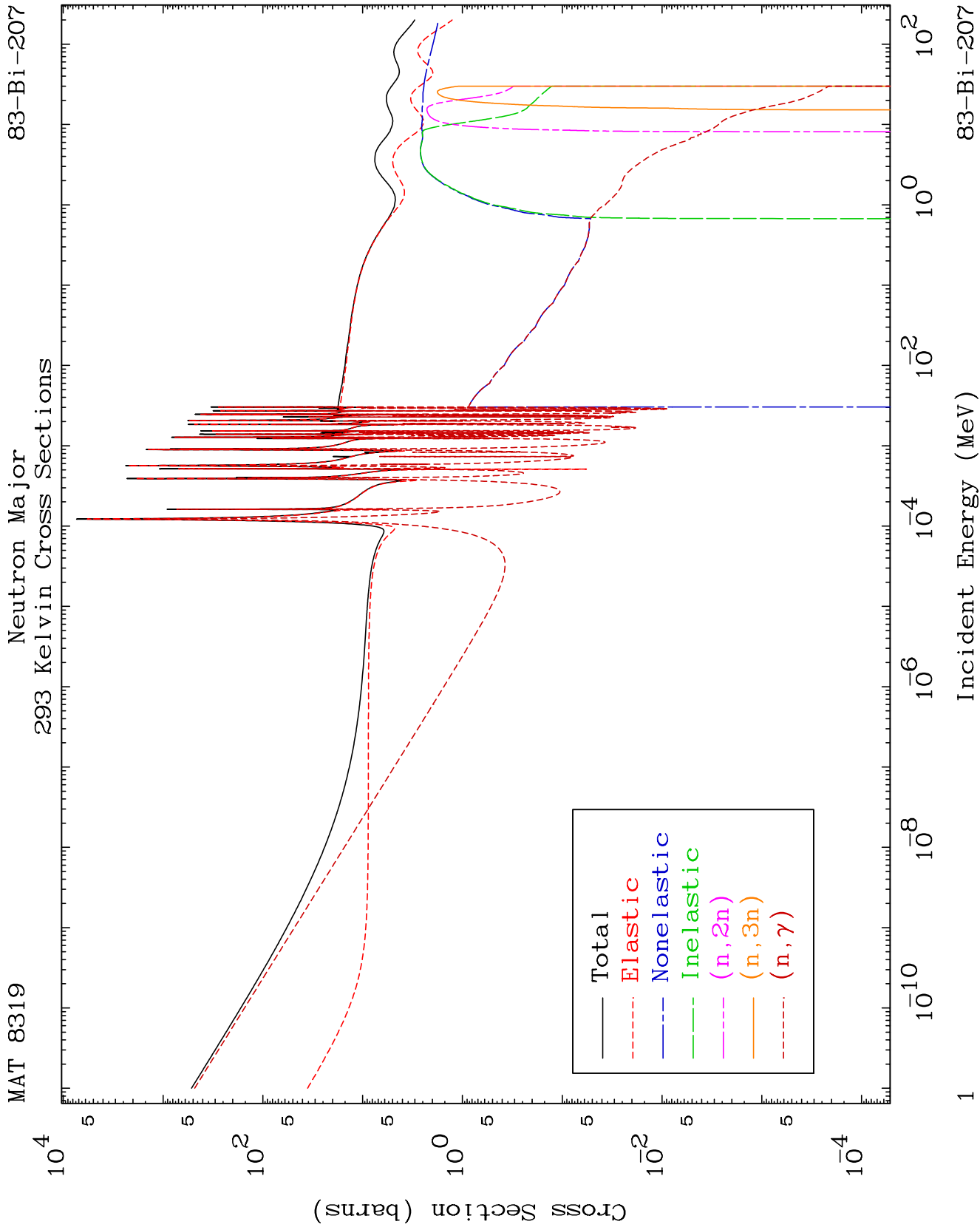
Dermott E. Cullen
(Present Contact Information)

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Livermore, CA 94550
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E.Mail: redcullen1@comcast.net
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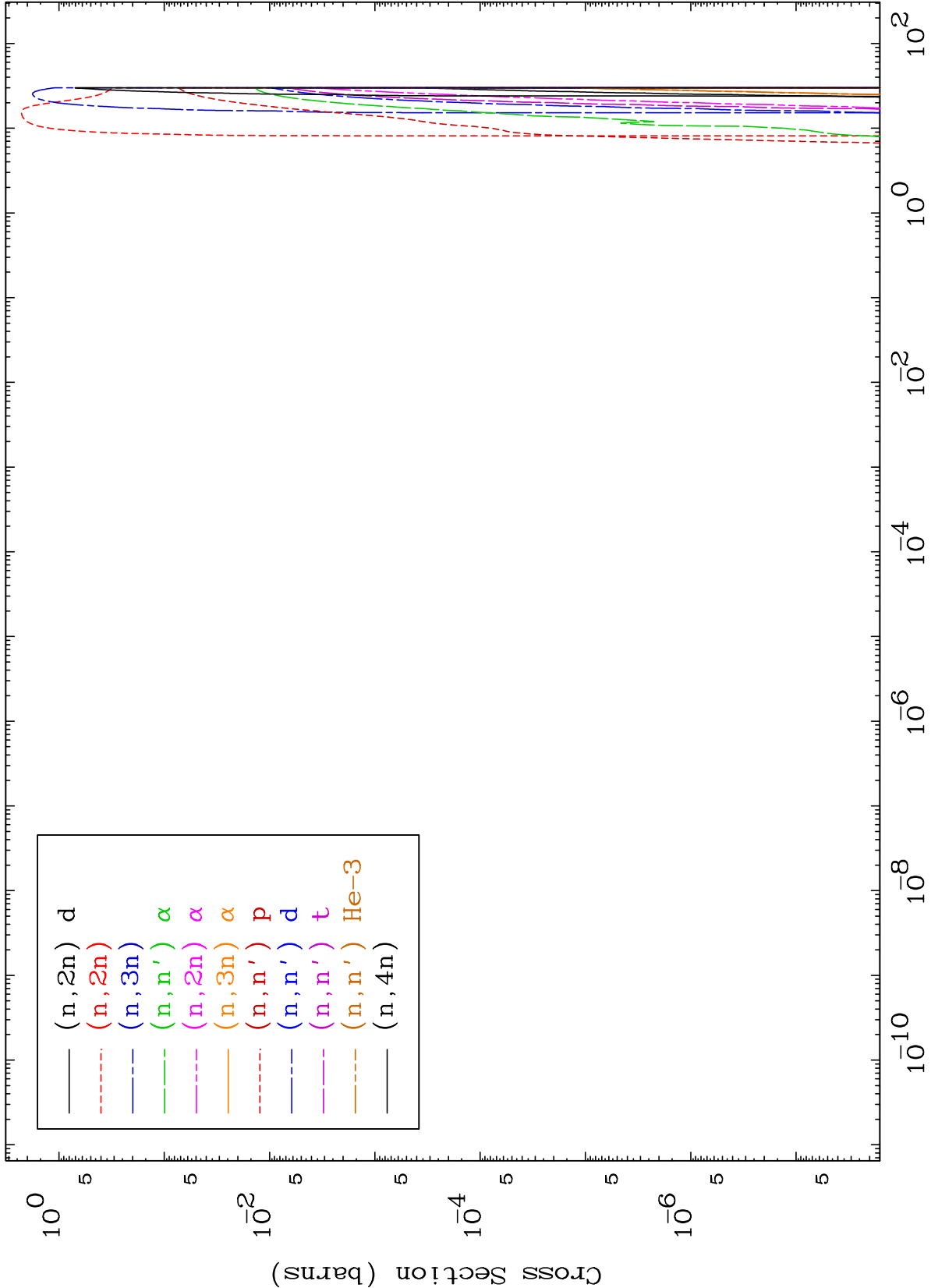
Press Mouse Button to Start

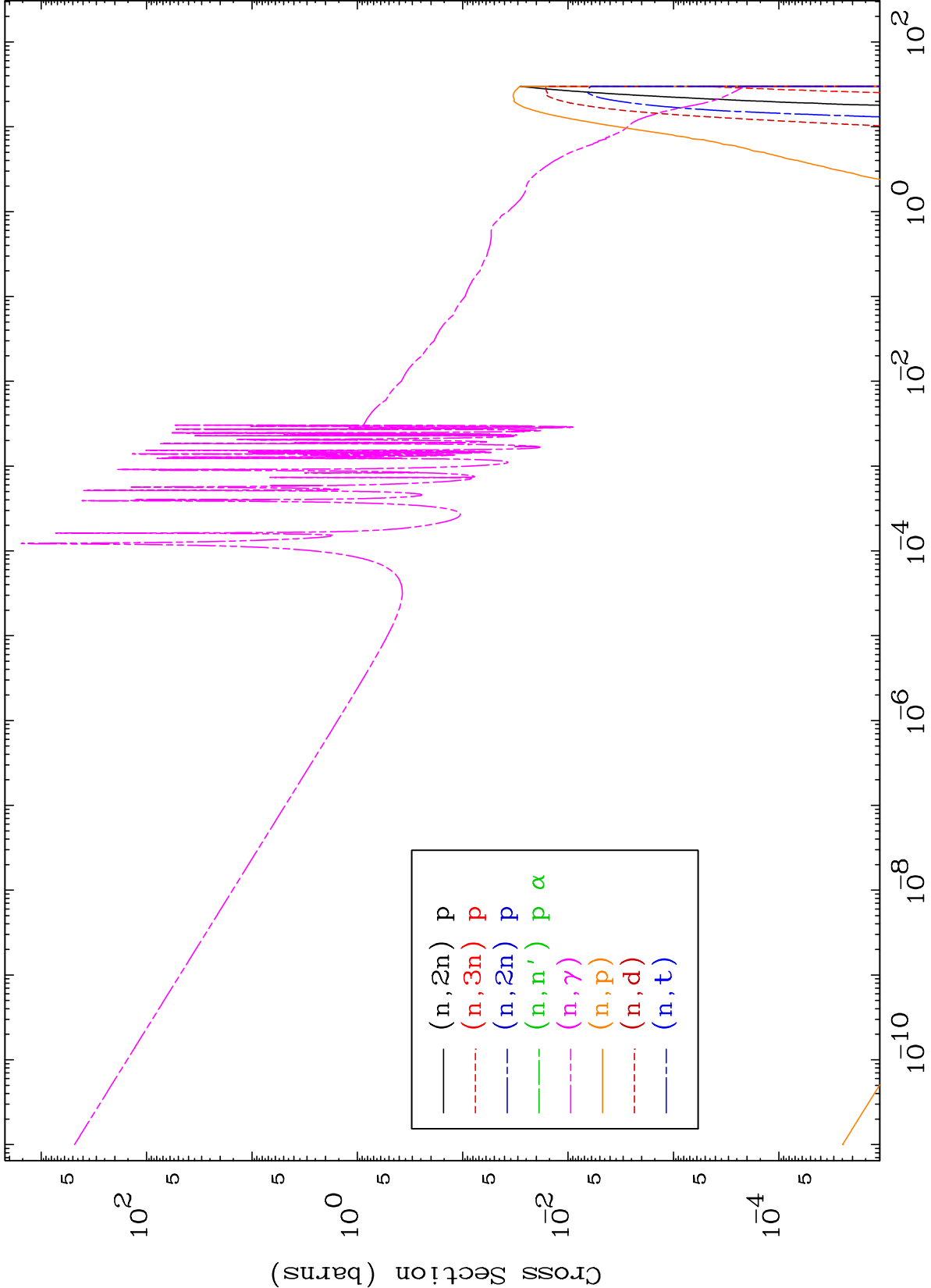


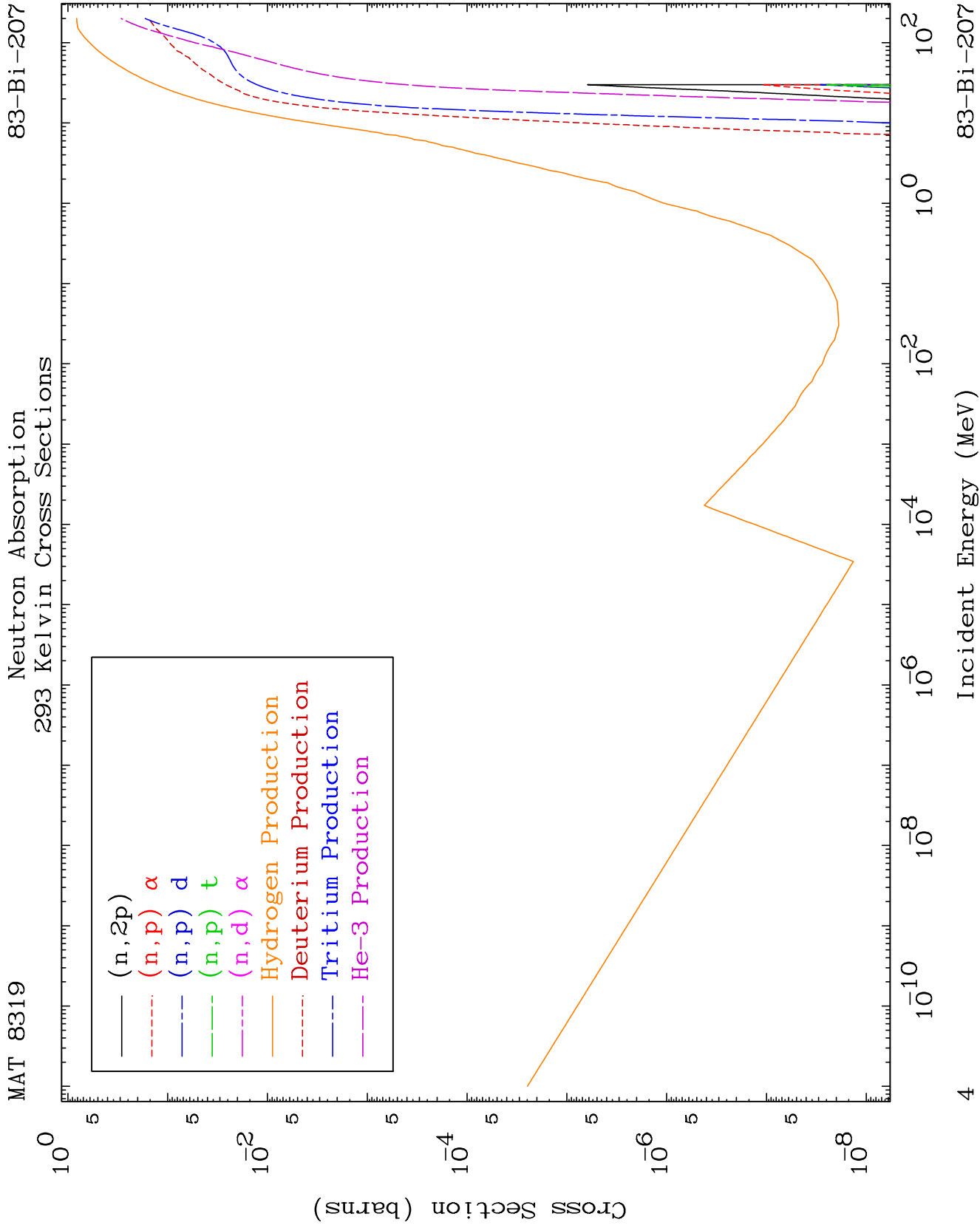
MAT 8319

Neutron Absorption
293 Kelvin Cross Sections

83-Bi-207



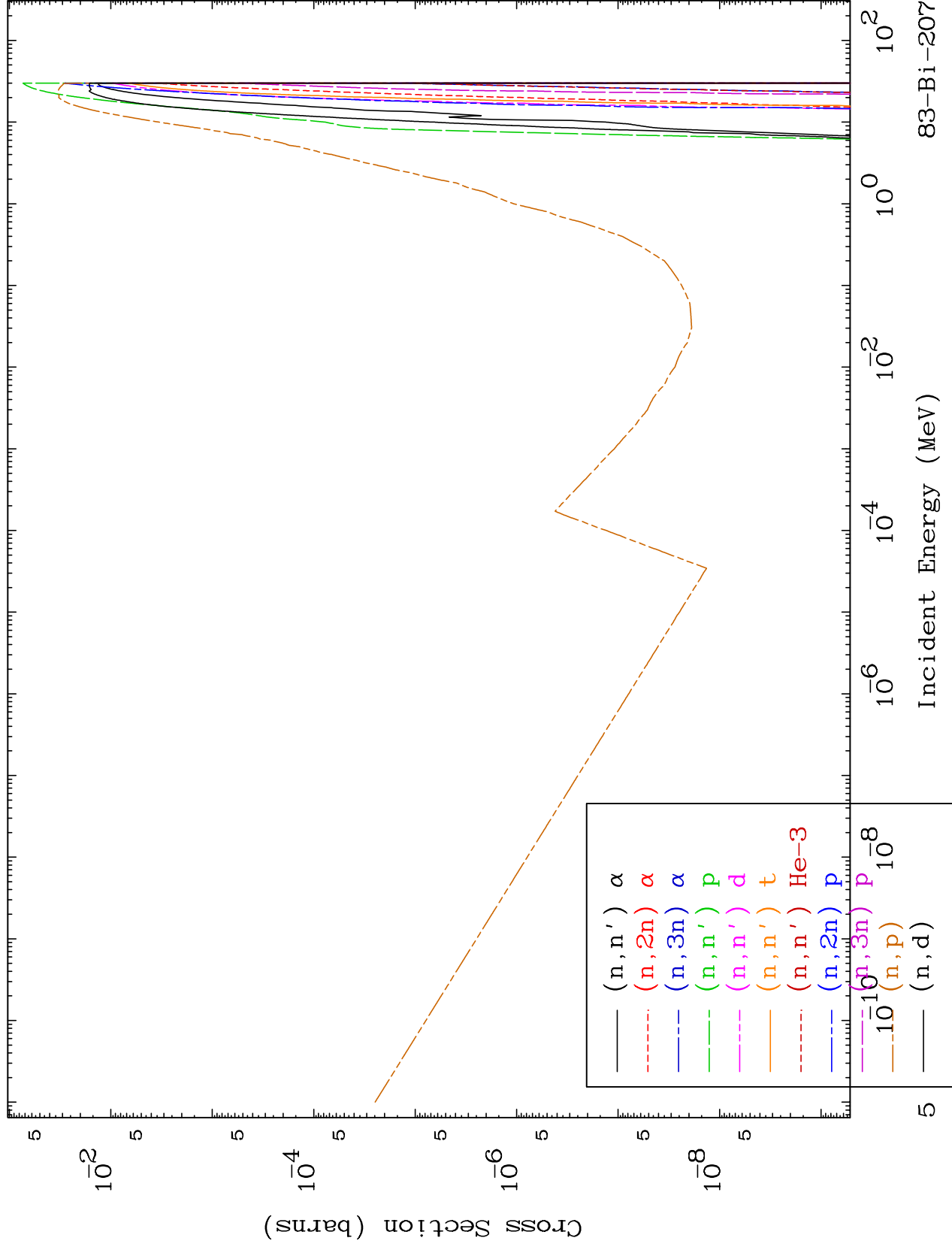


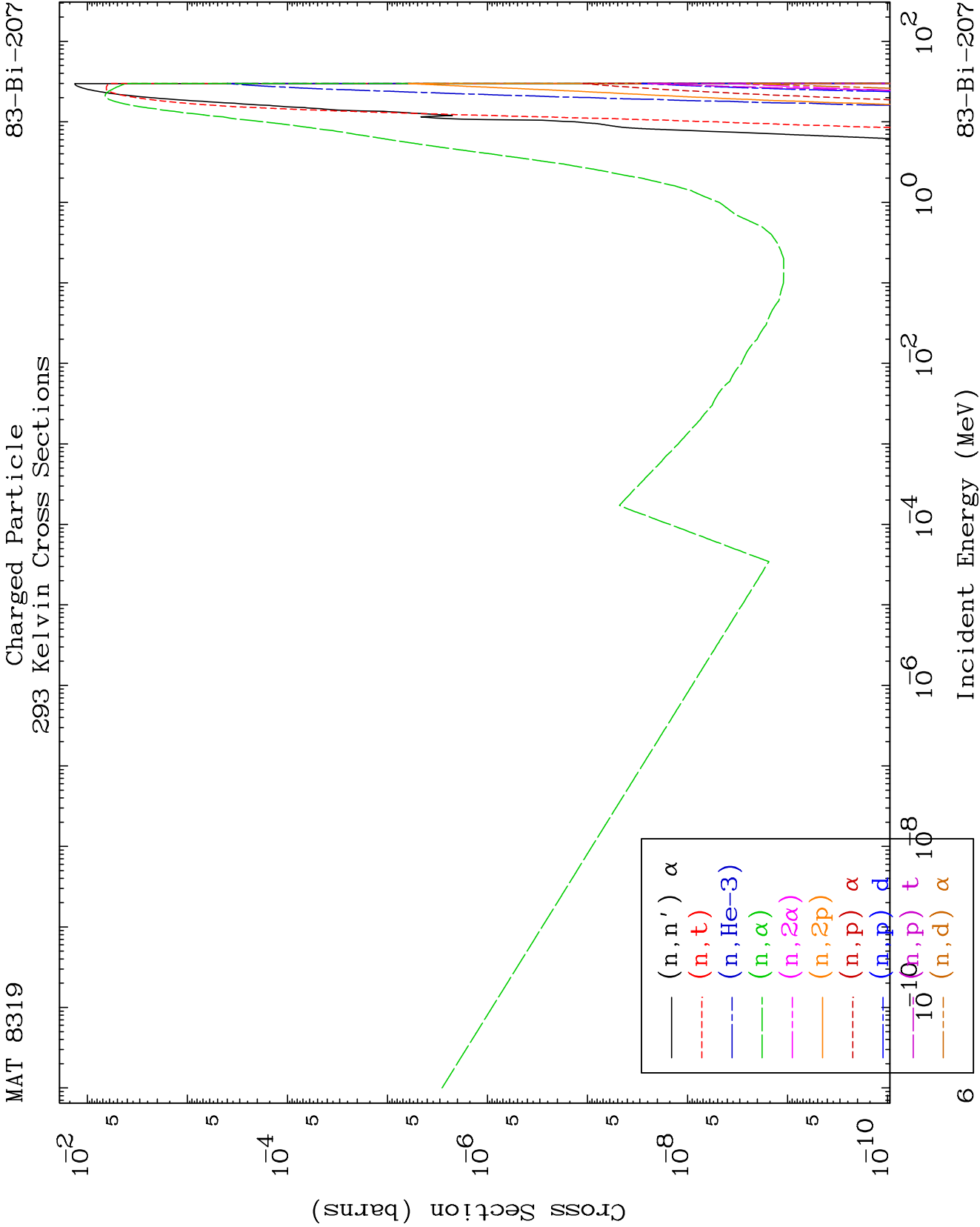


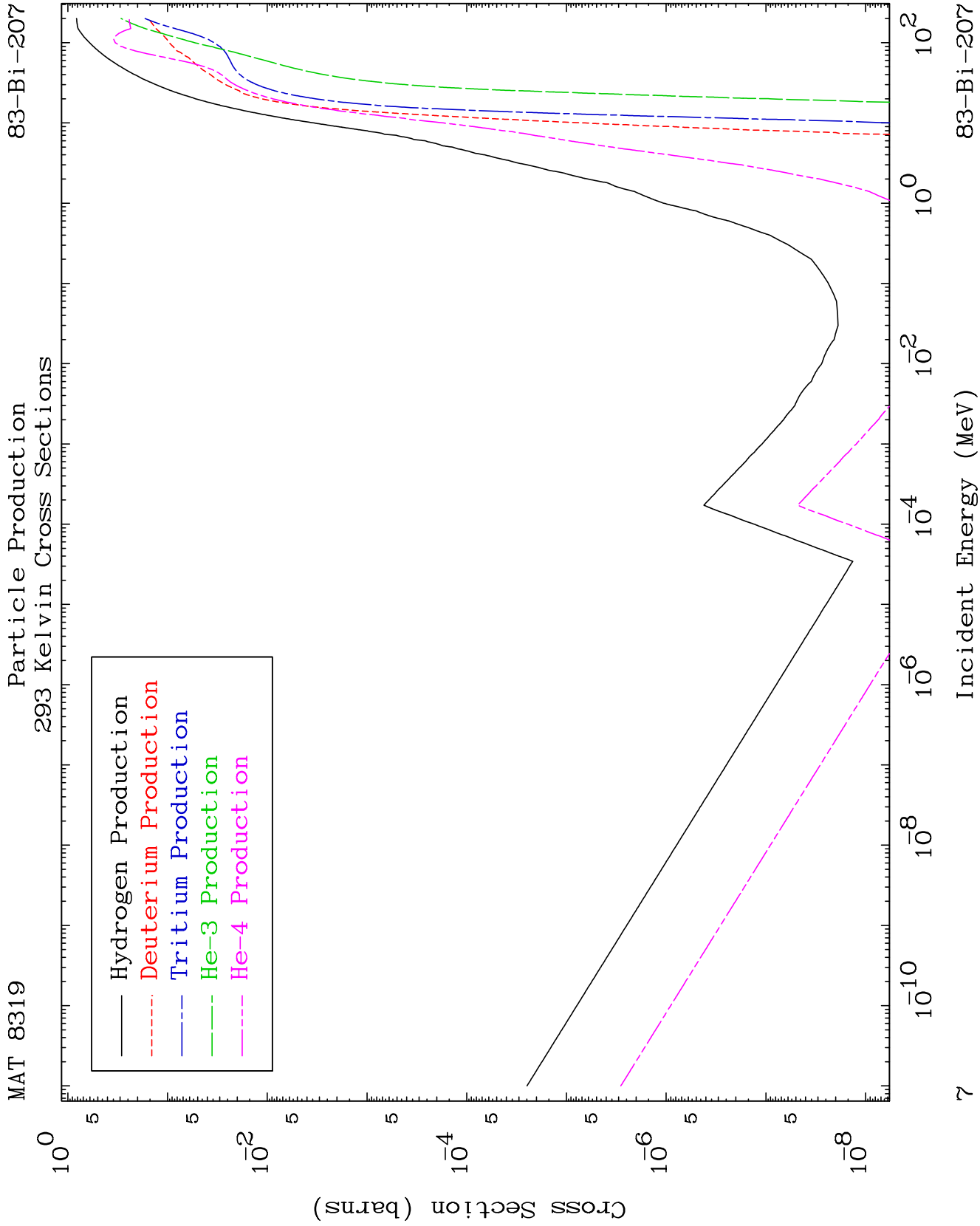
MAT 8319

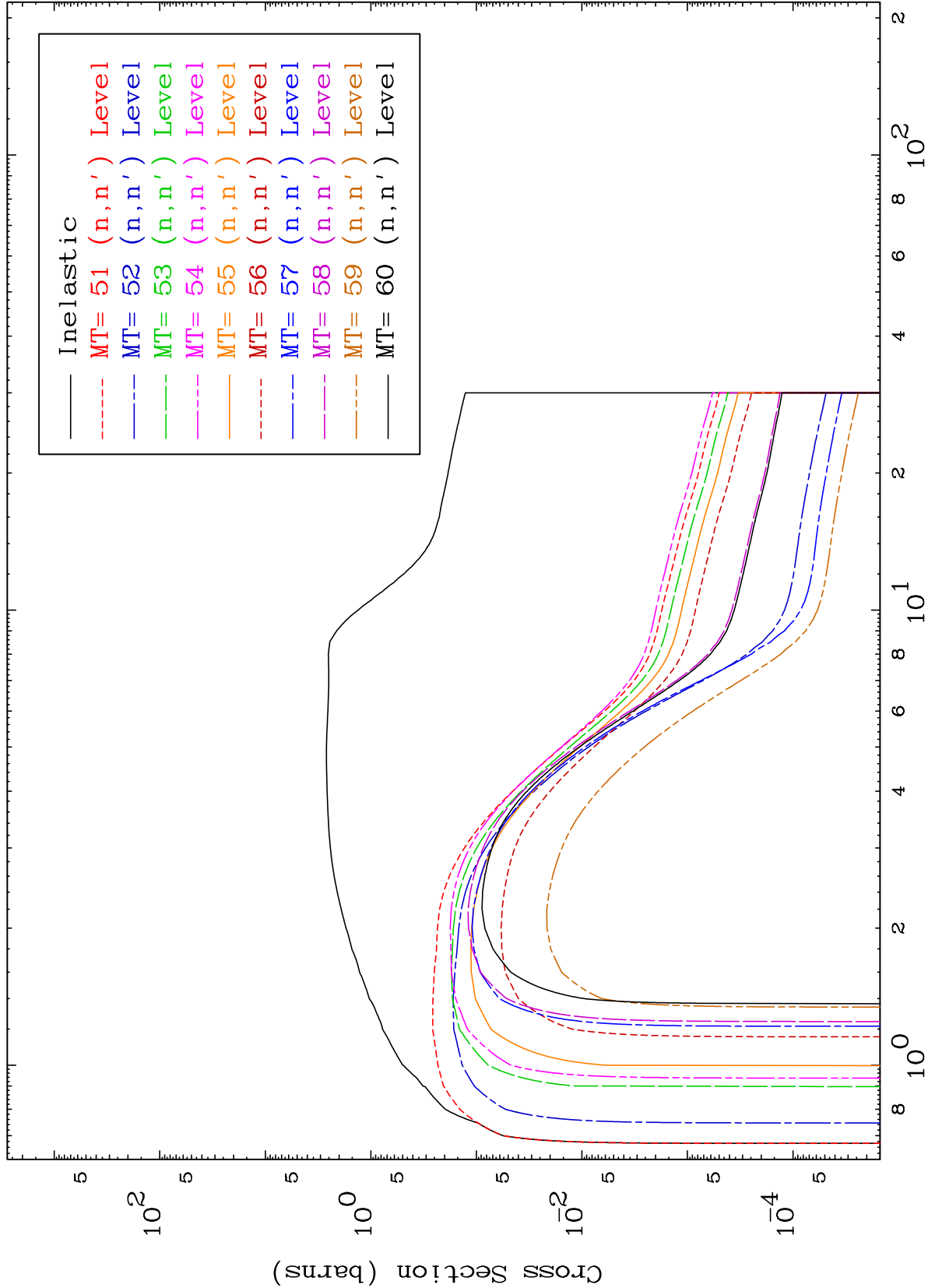
Charged Particle
293 Kelvin Cross Sections

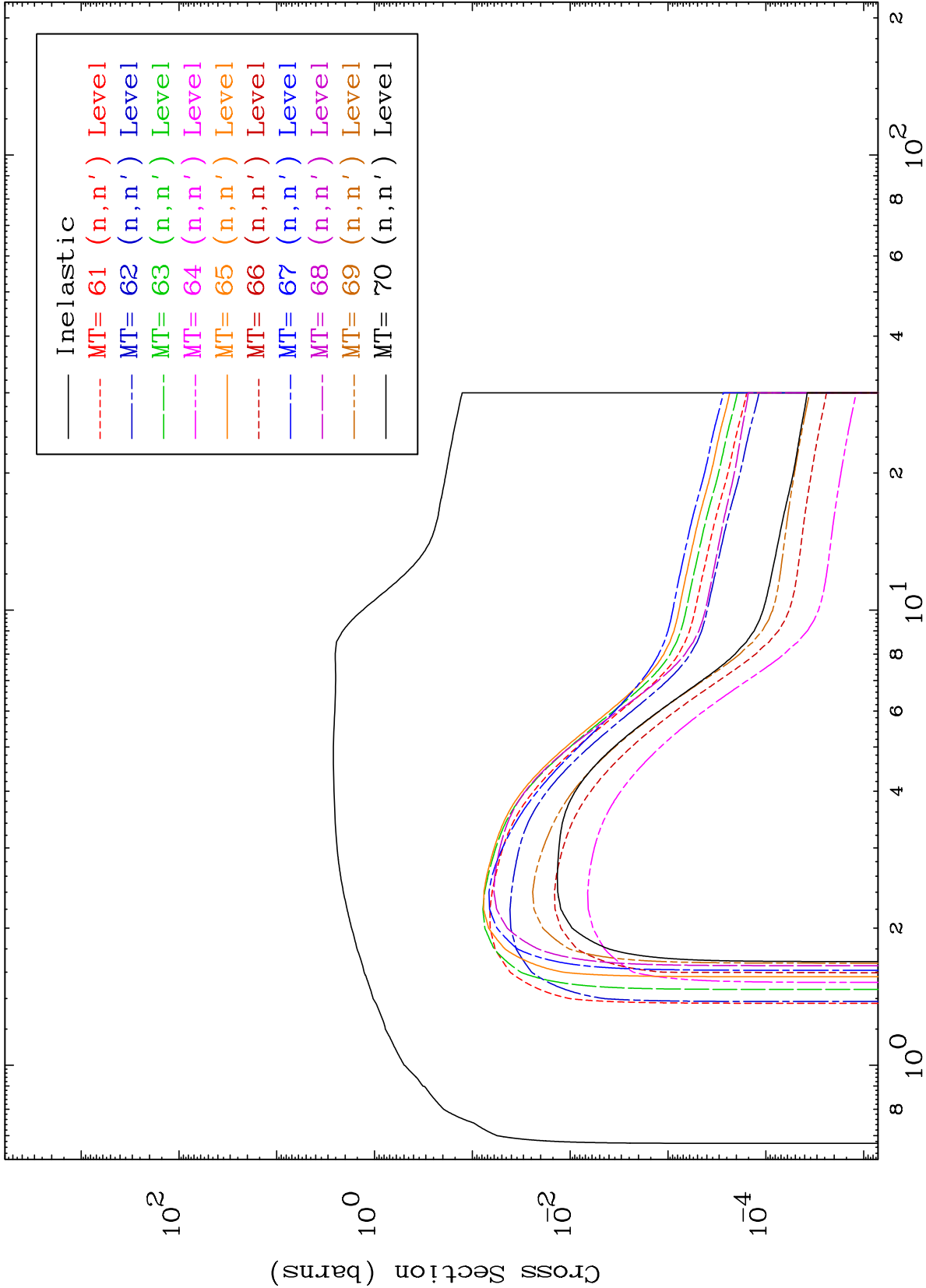
83-Bi-207









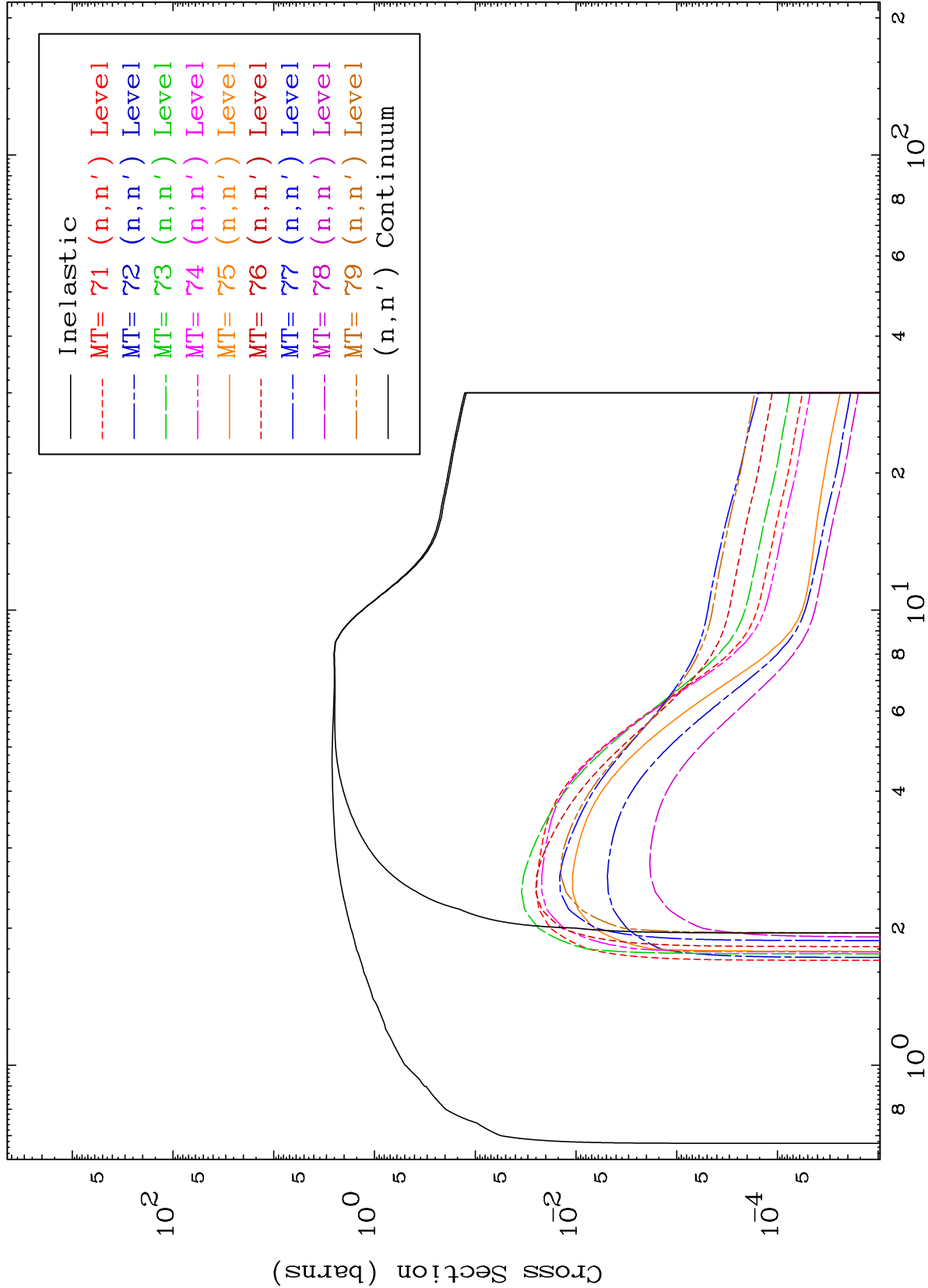


MAT 8319

(n,n') Levels

293 Kelvin Cross Sections

83-Bi-207



10

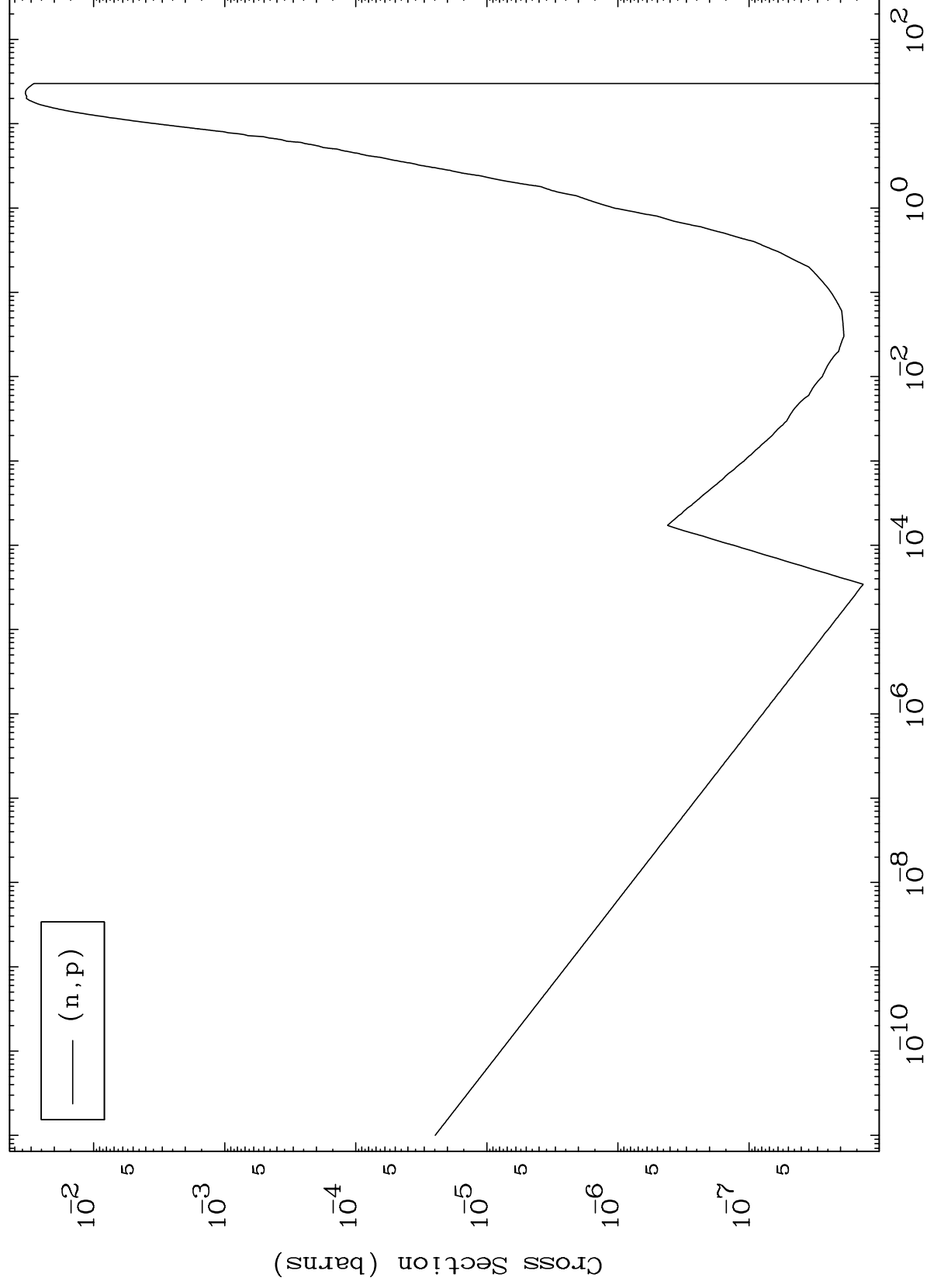
Incident Energy (MeV)

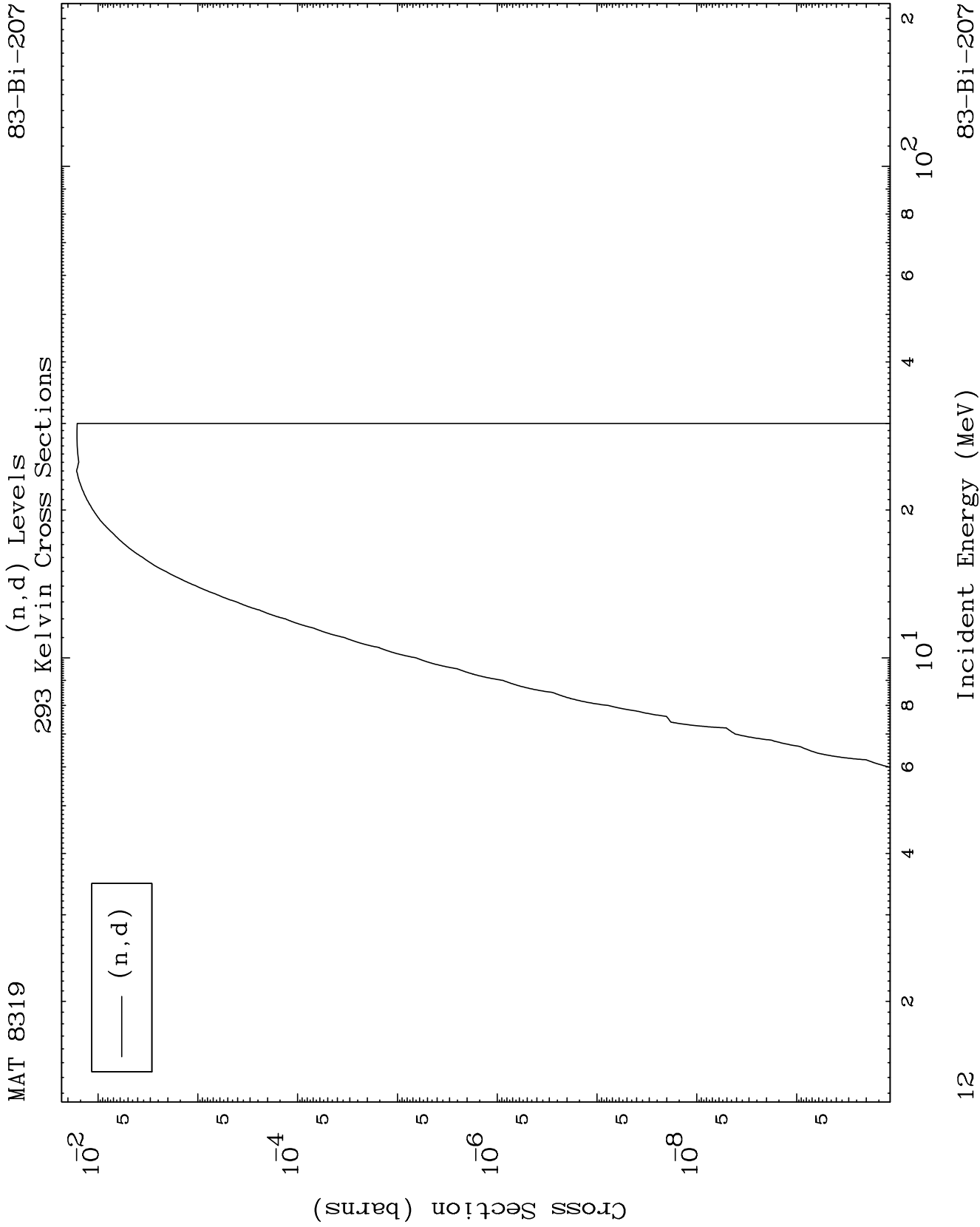
83-Bi-207

MAT 8319

(n,p) Levels
293 Kelvin Cross Sections

83-Bi-207

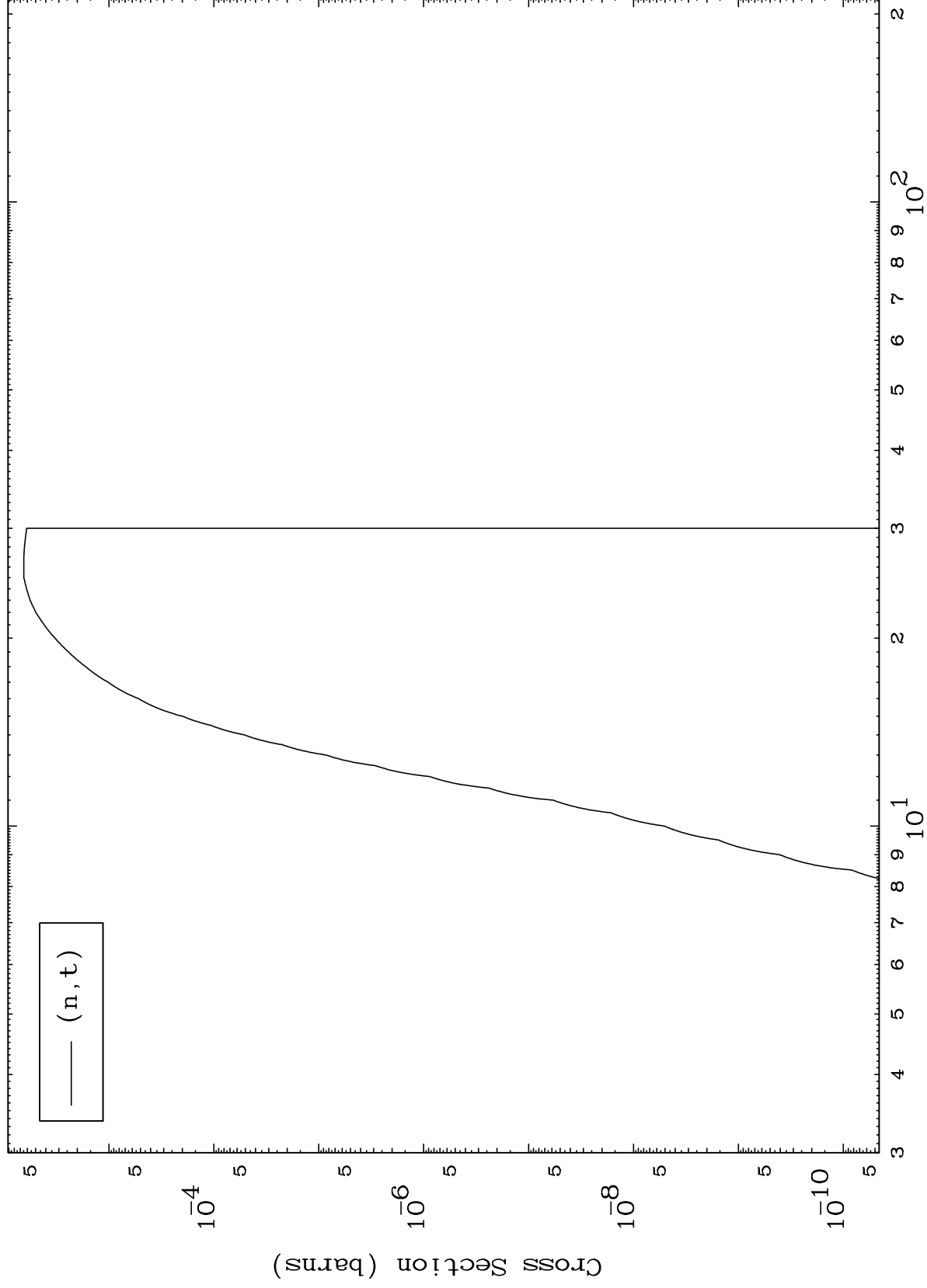




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83-Bi-207

(n,t) Levels
293 Kelvin Cross Sections



83-Bi-207

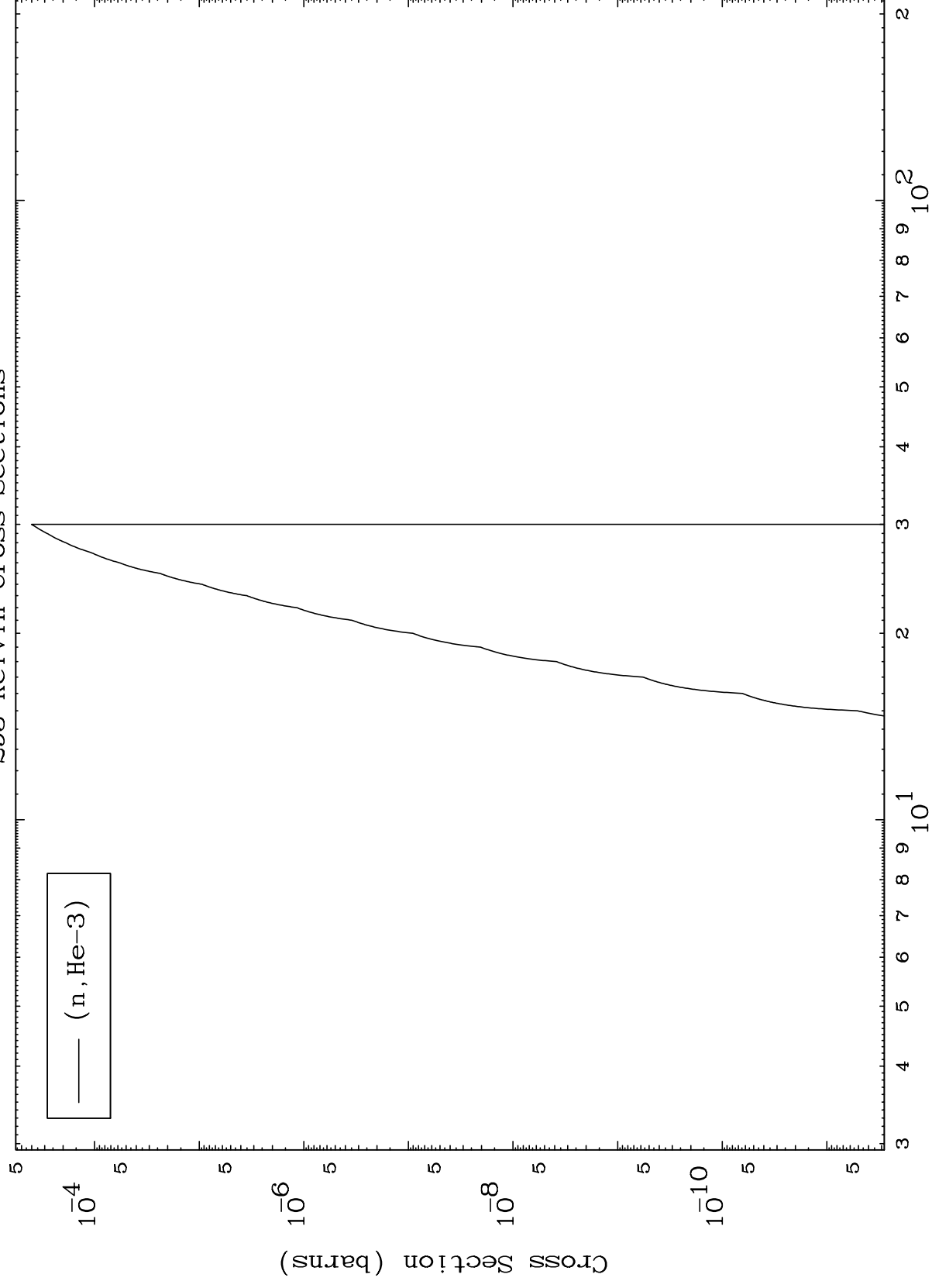
Incident Energy (MeV)

13

MAT 8319

83-Bi-207

(n, He3) Levels
293 Kelvin Cross Sections



83-Bi-207

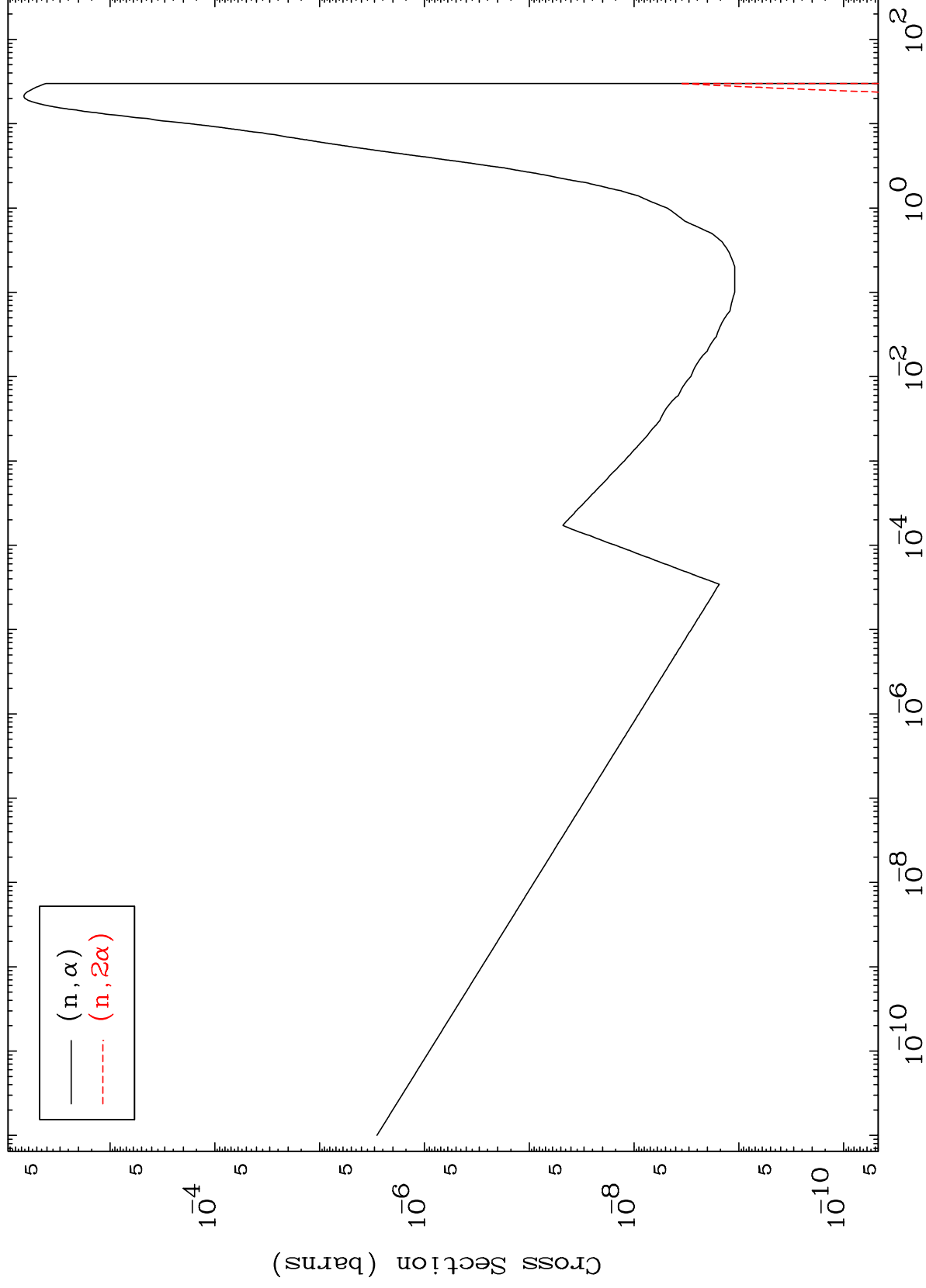
Incident Energy (MeV)

14

MAT 8319

(n, α) Levels
293 Kelvin Cross Sections

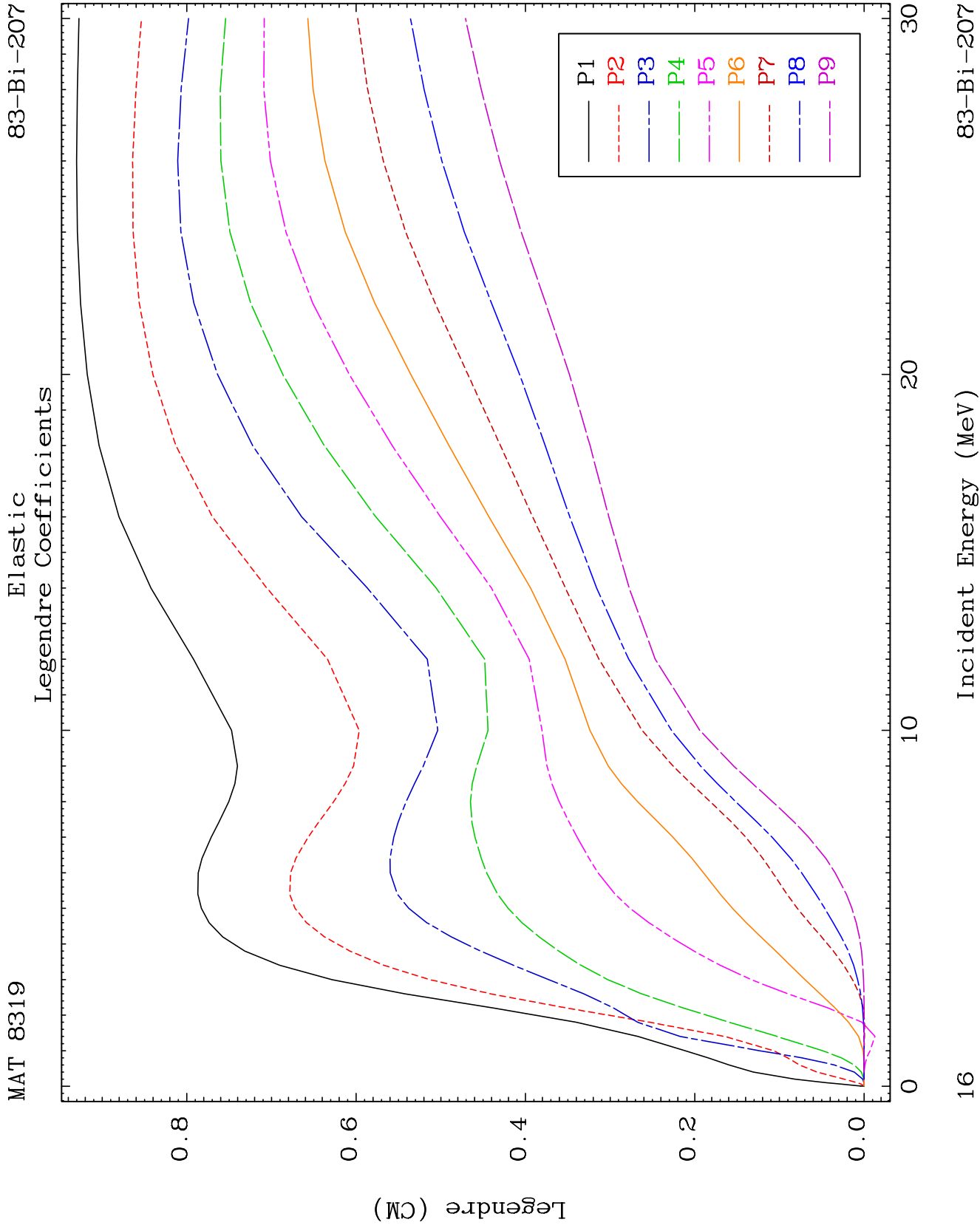
83-Bi-207

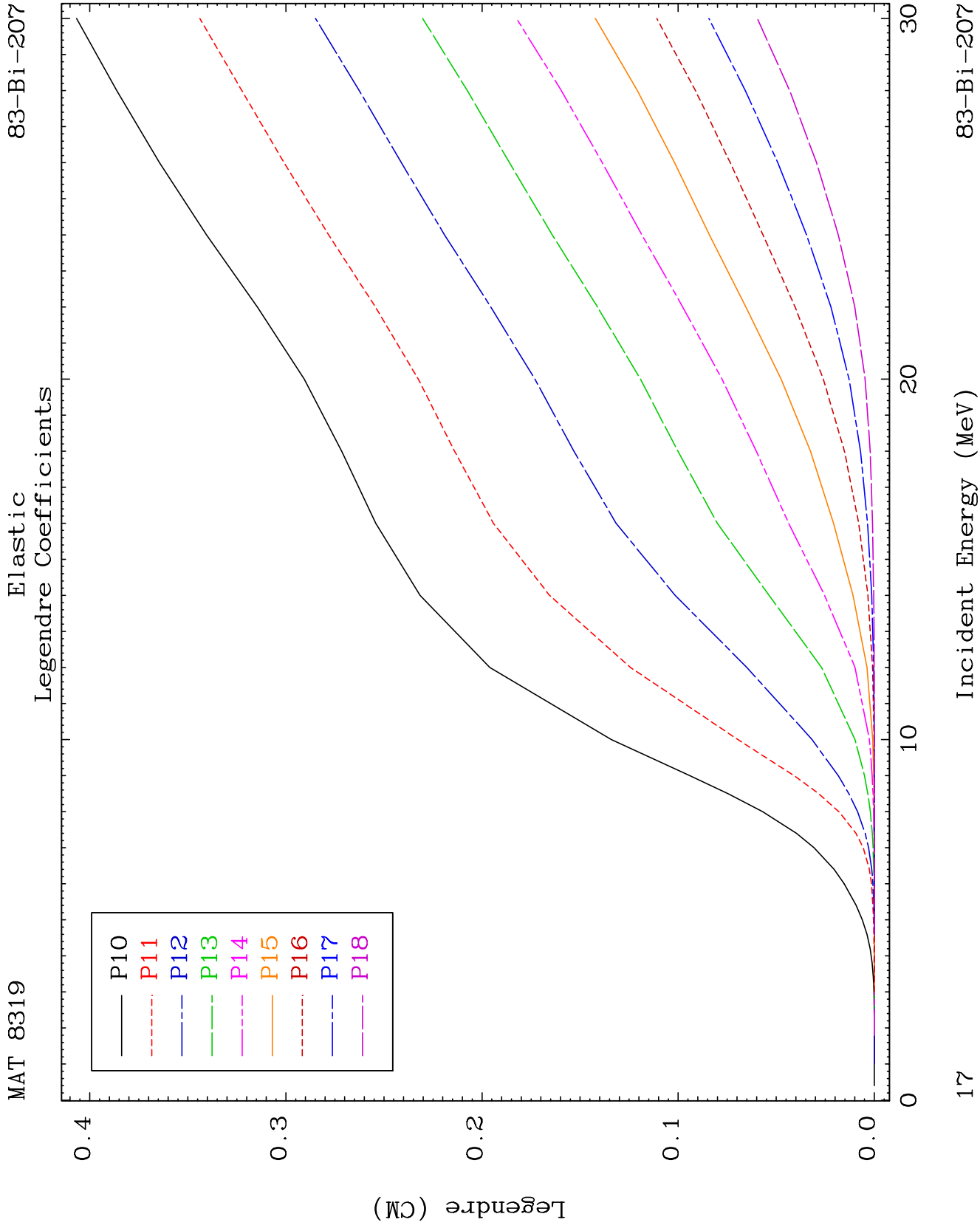


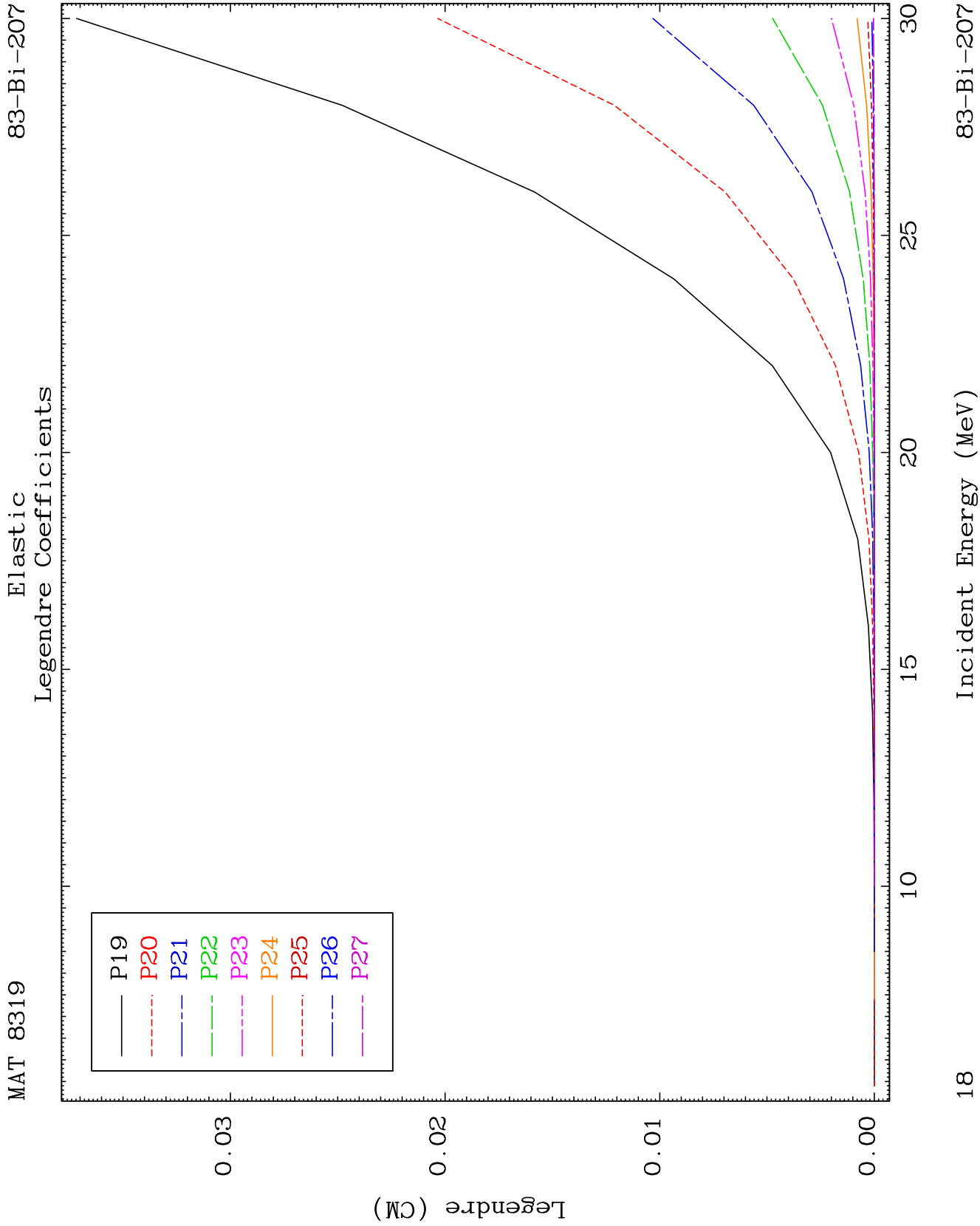
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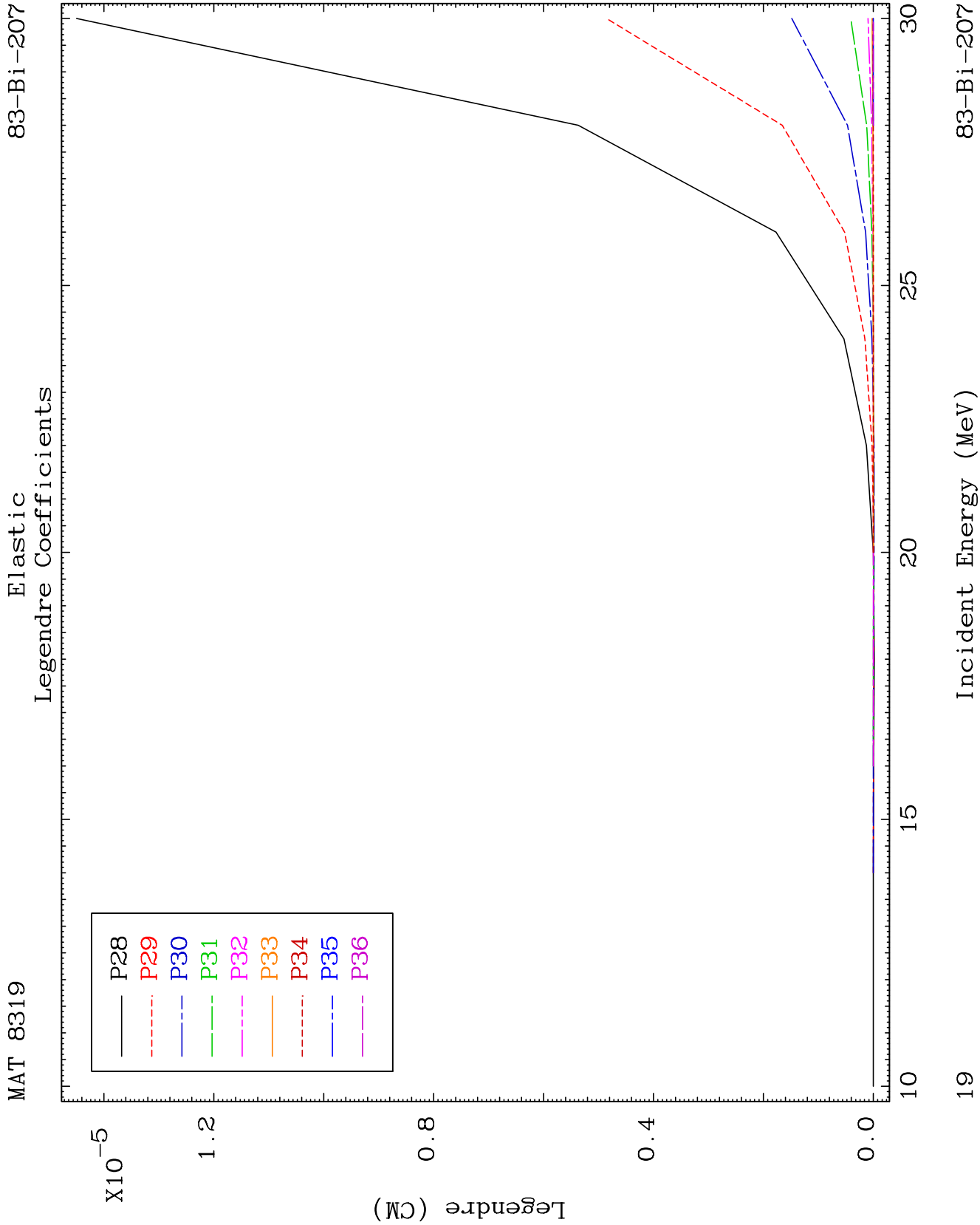
Incident Energy (MeV)

83-Bi-207





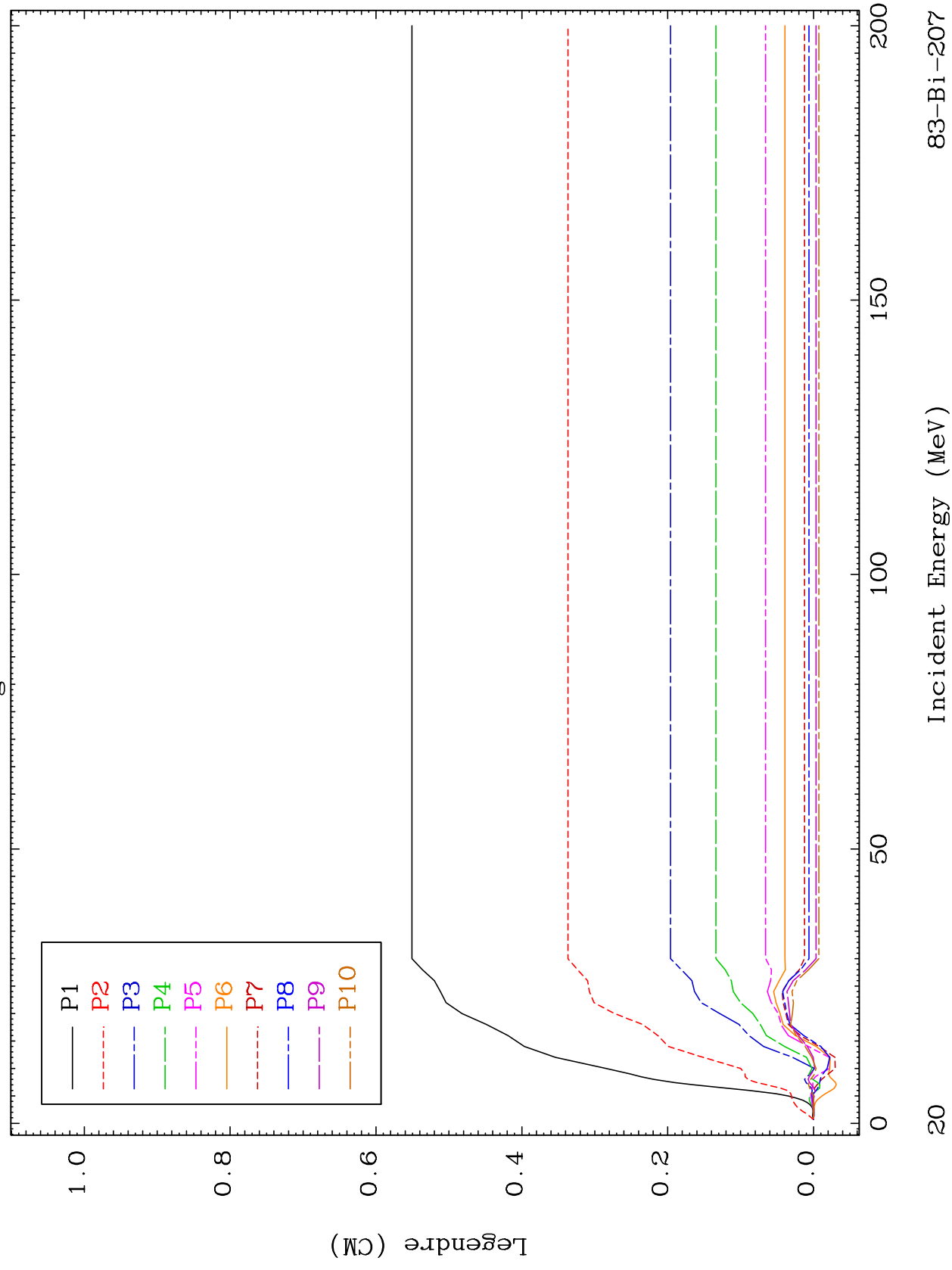


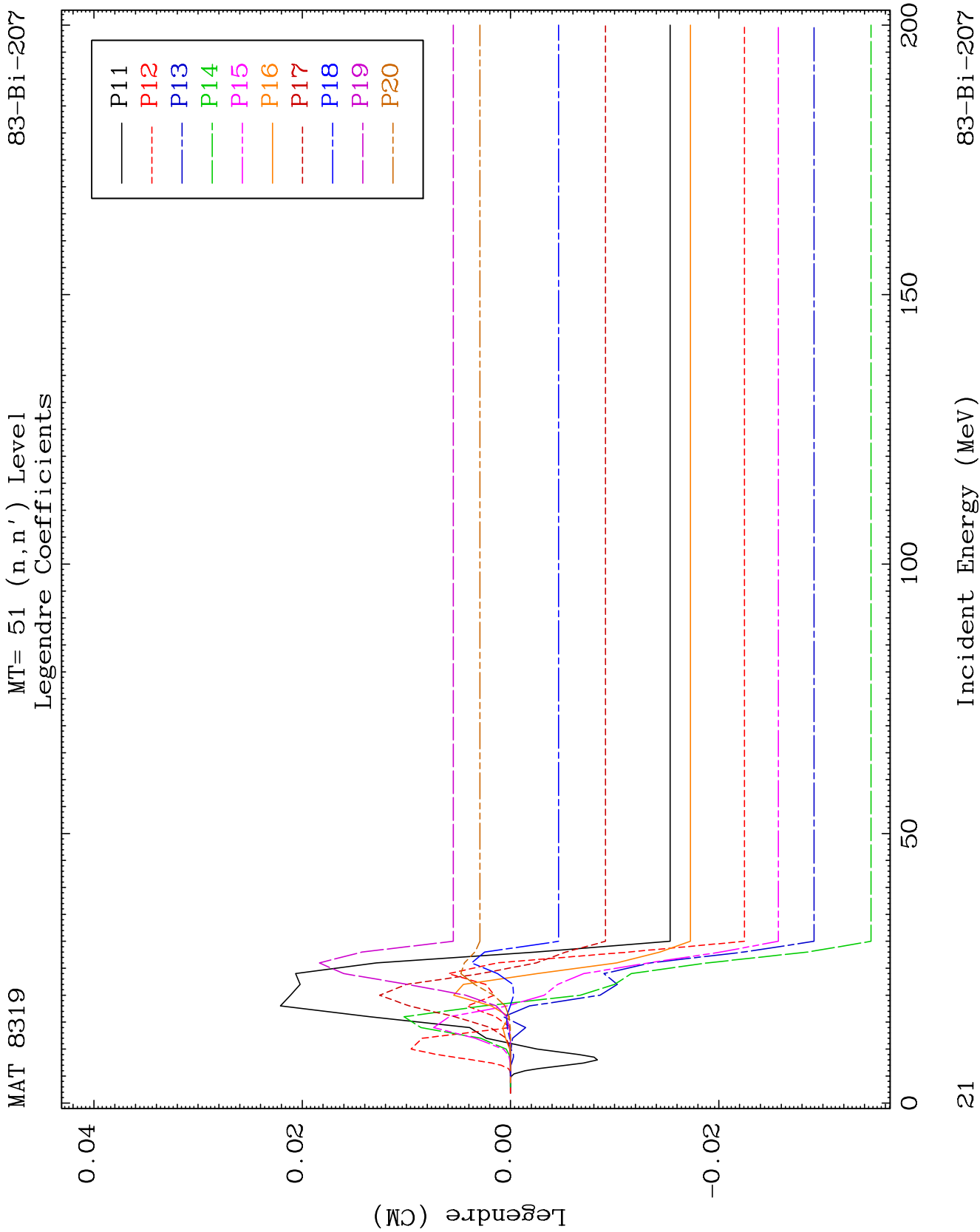


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MT= 51 (n, n') Level
Legendre Coefficients

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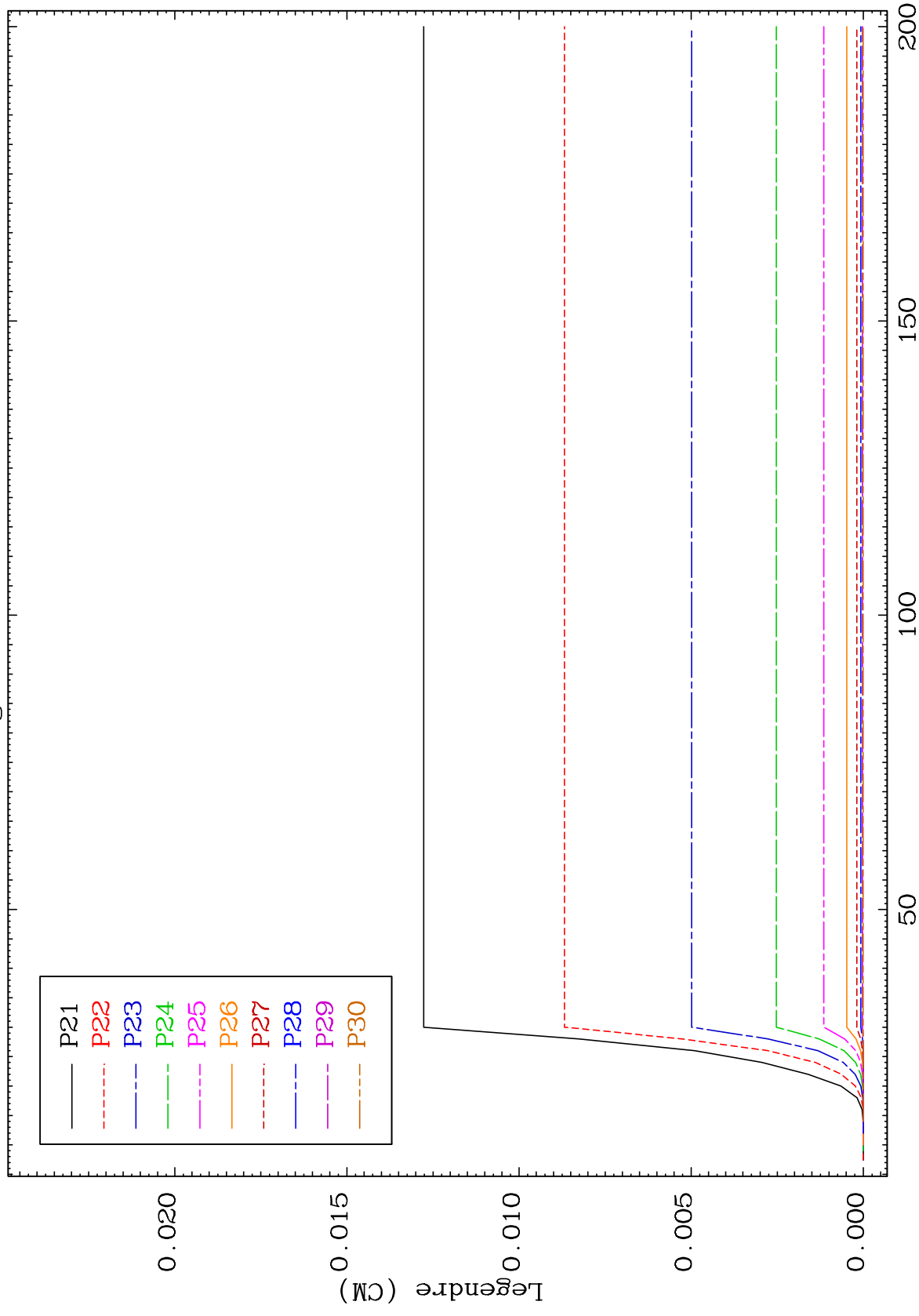




MAT 8319

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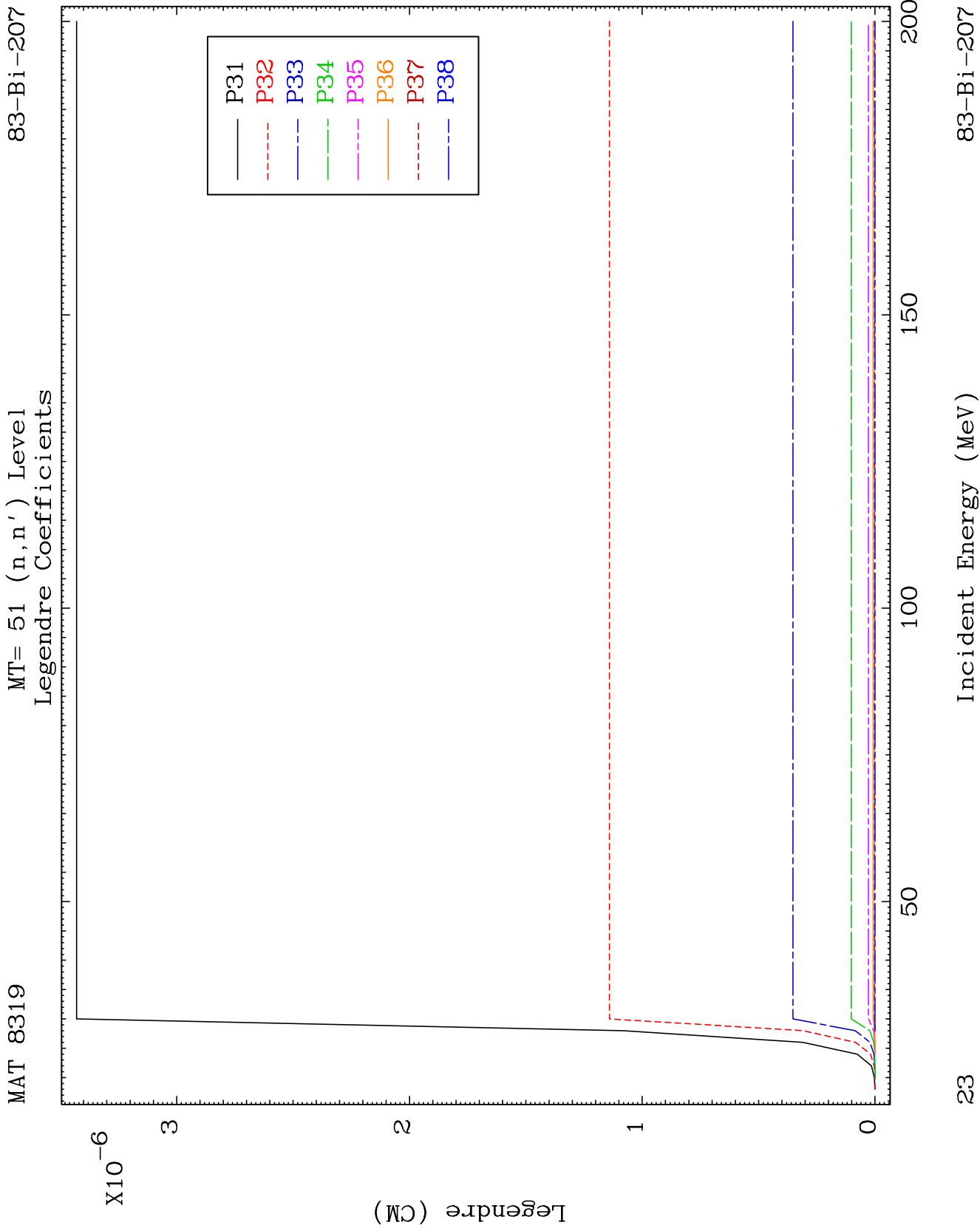
MT= 51 (n, n') Level
Legendre Coefficients

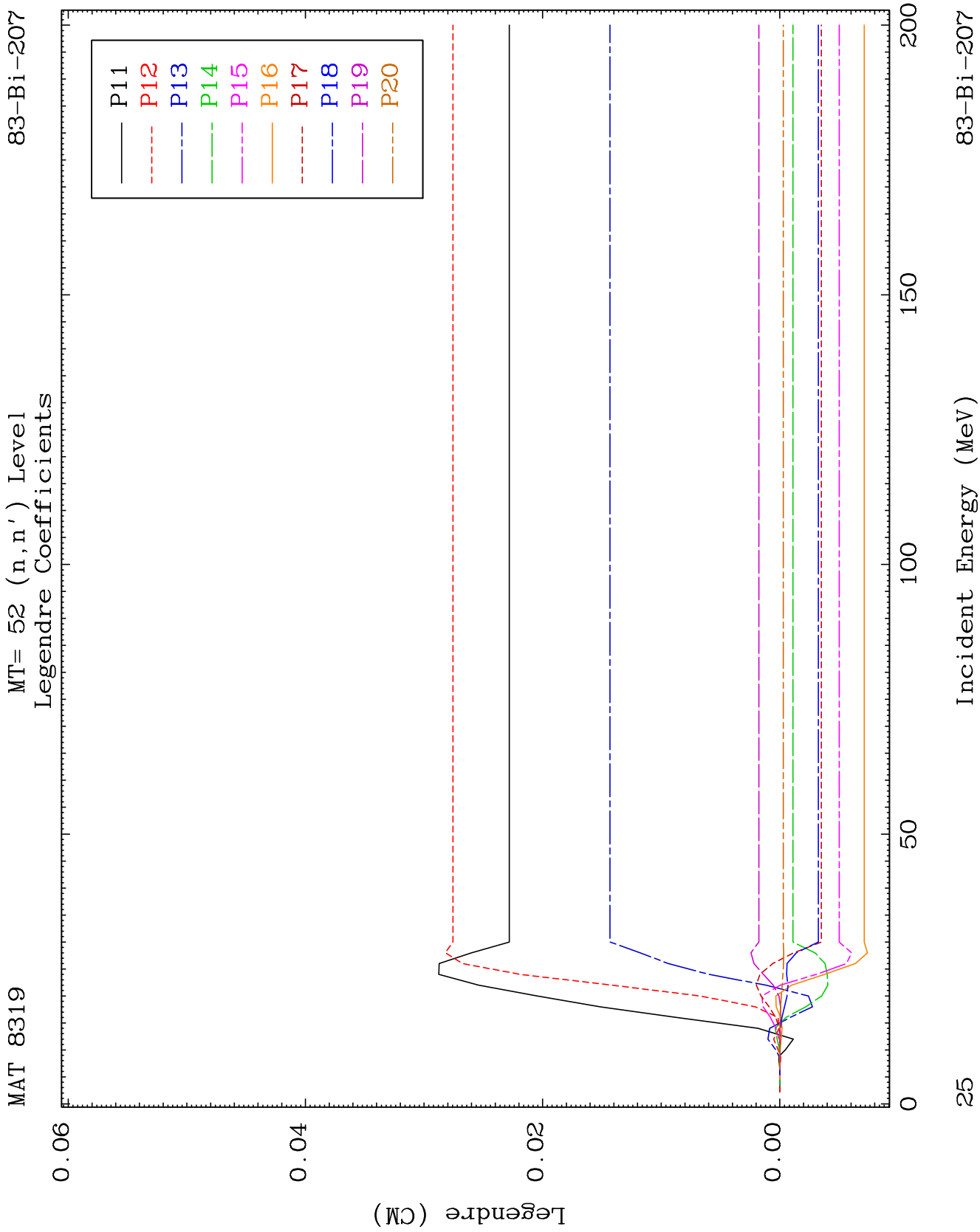


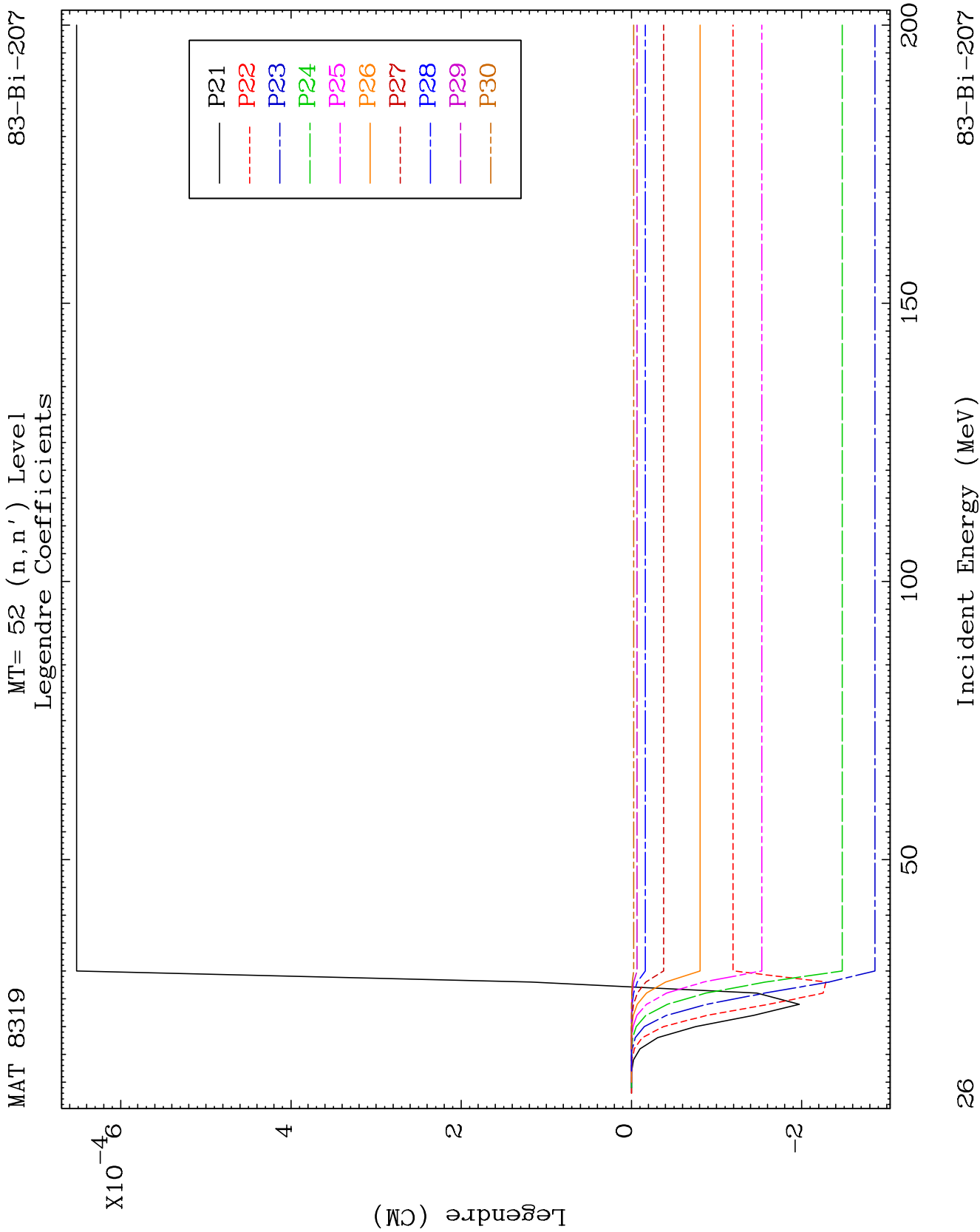
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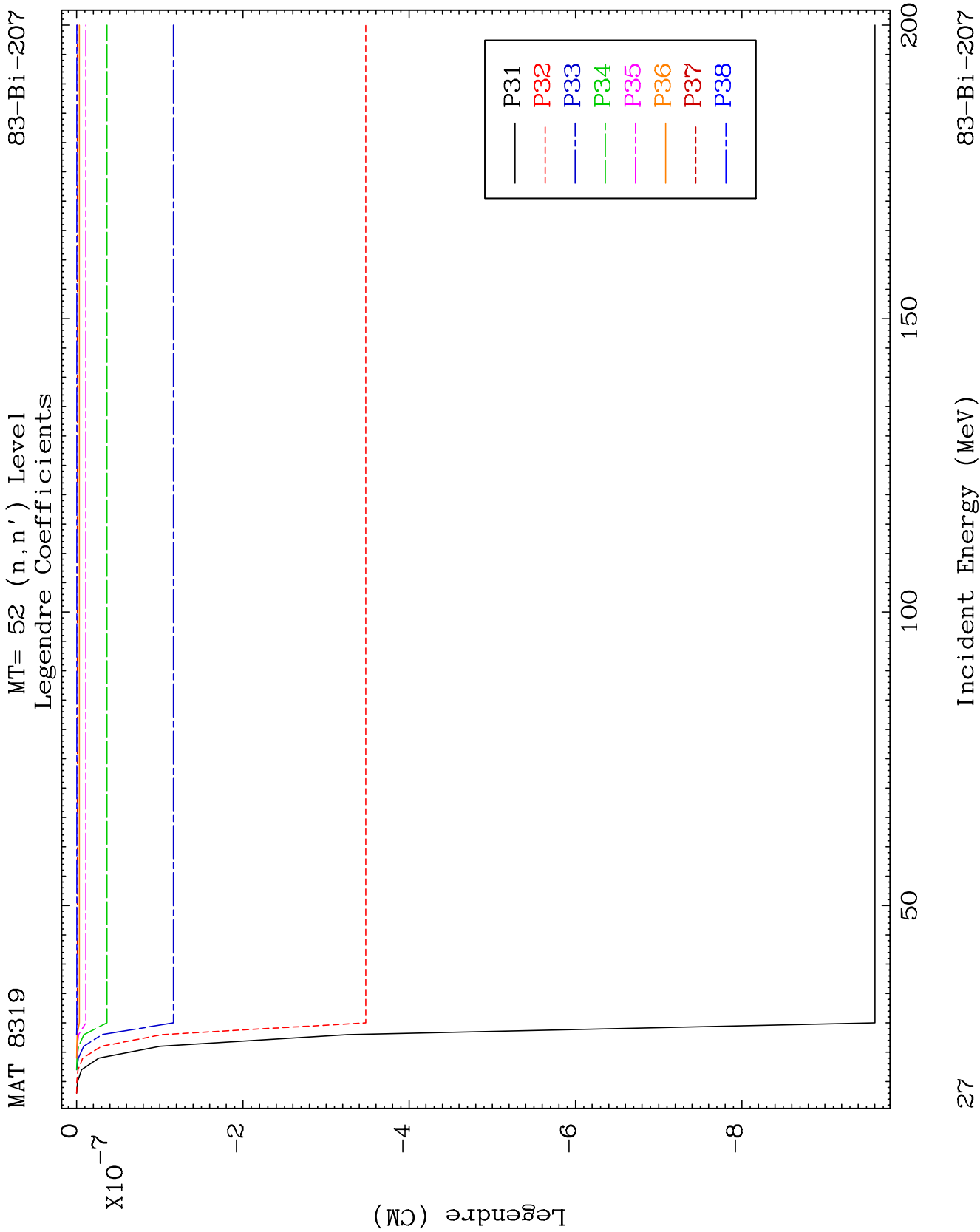
Incident Energy (MeV)

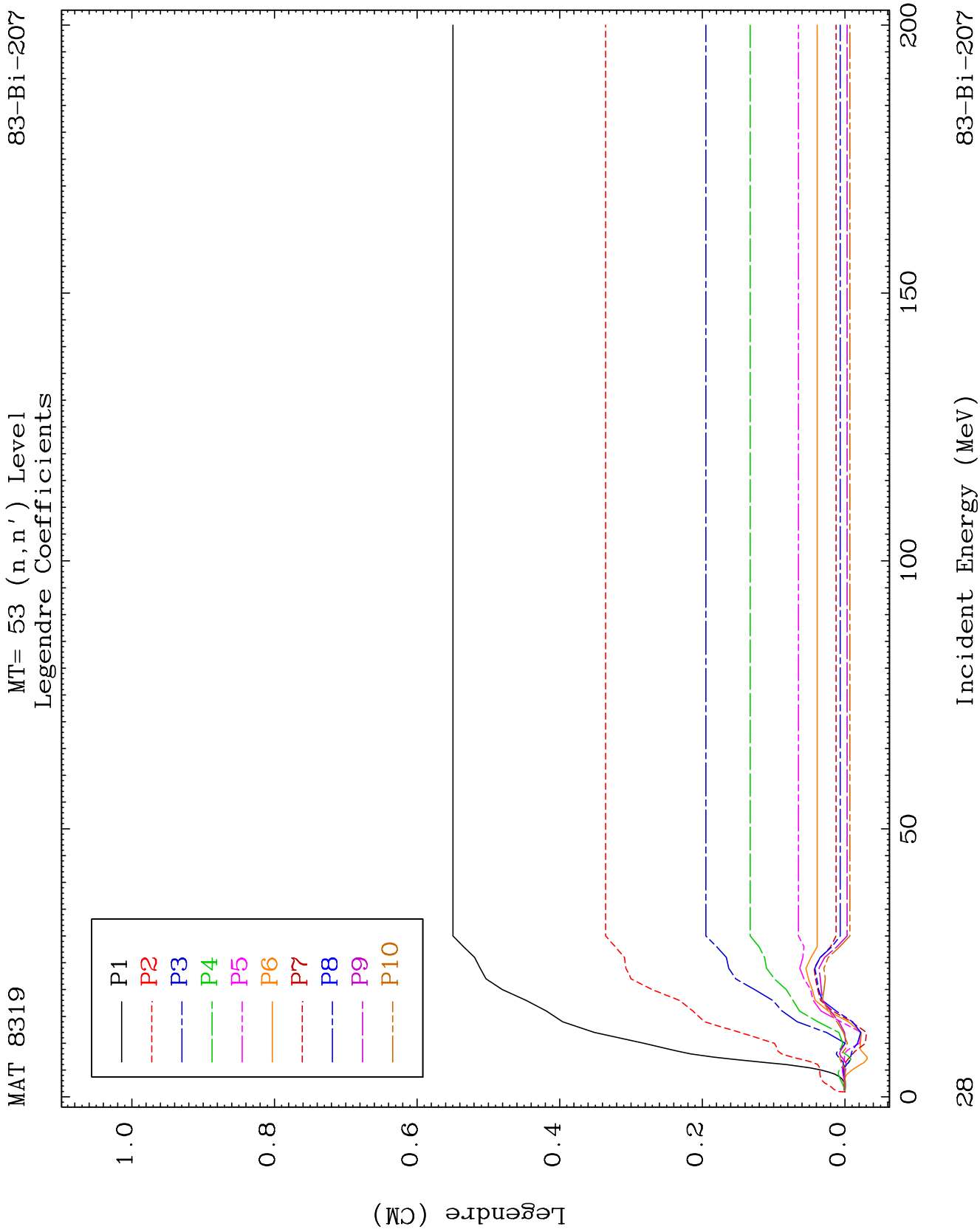
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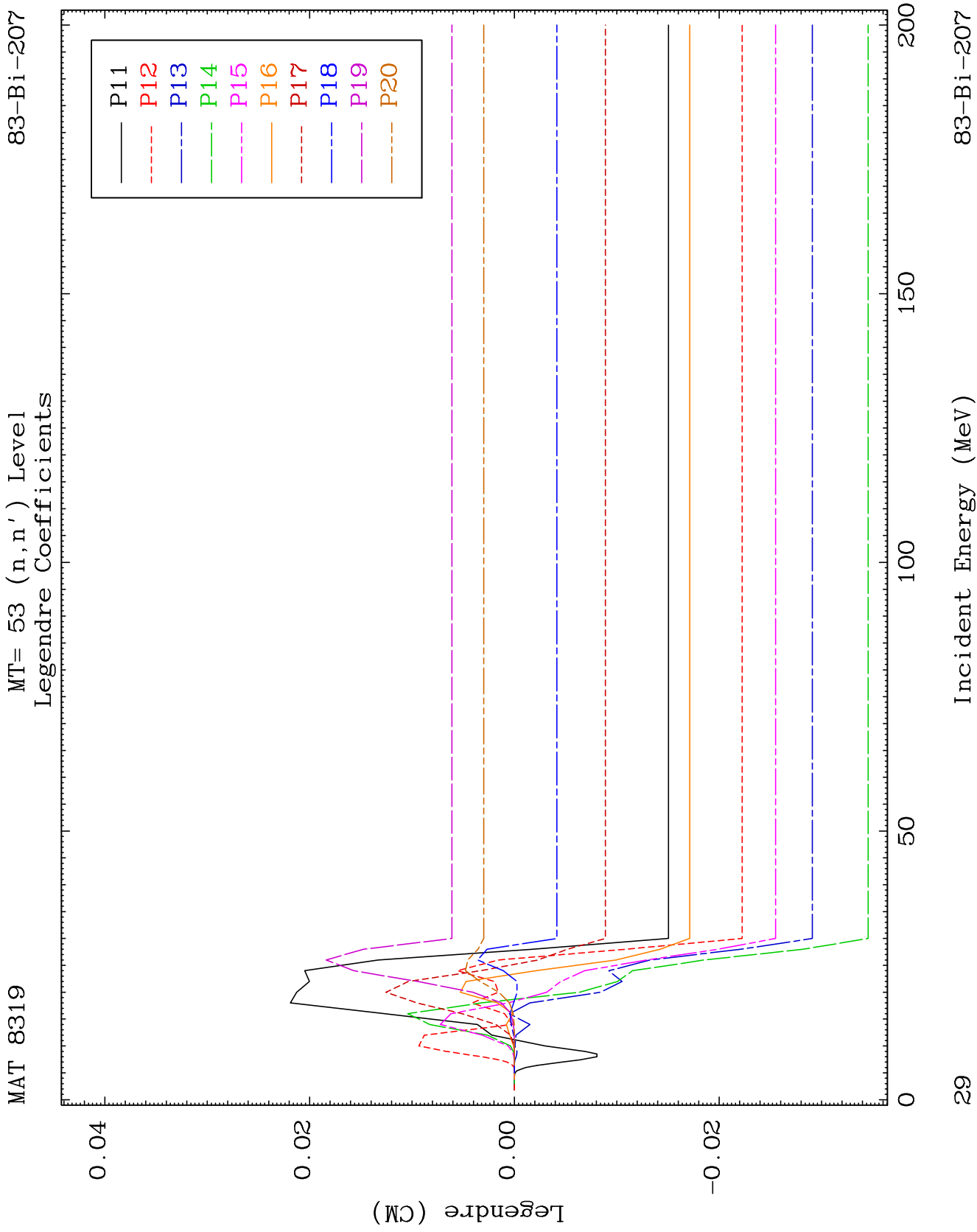








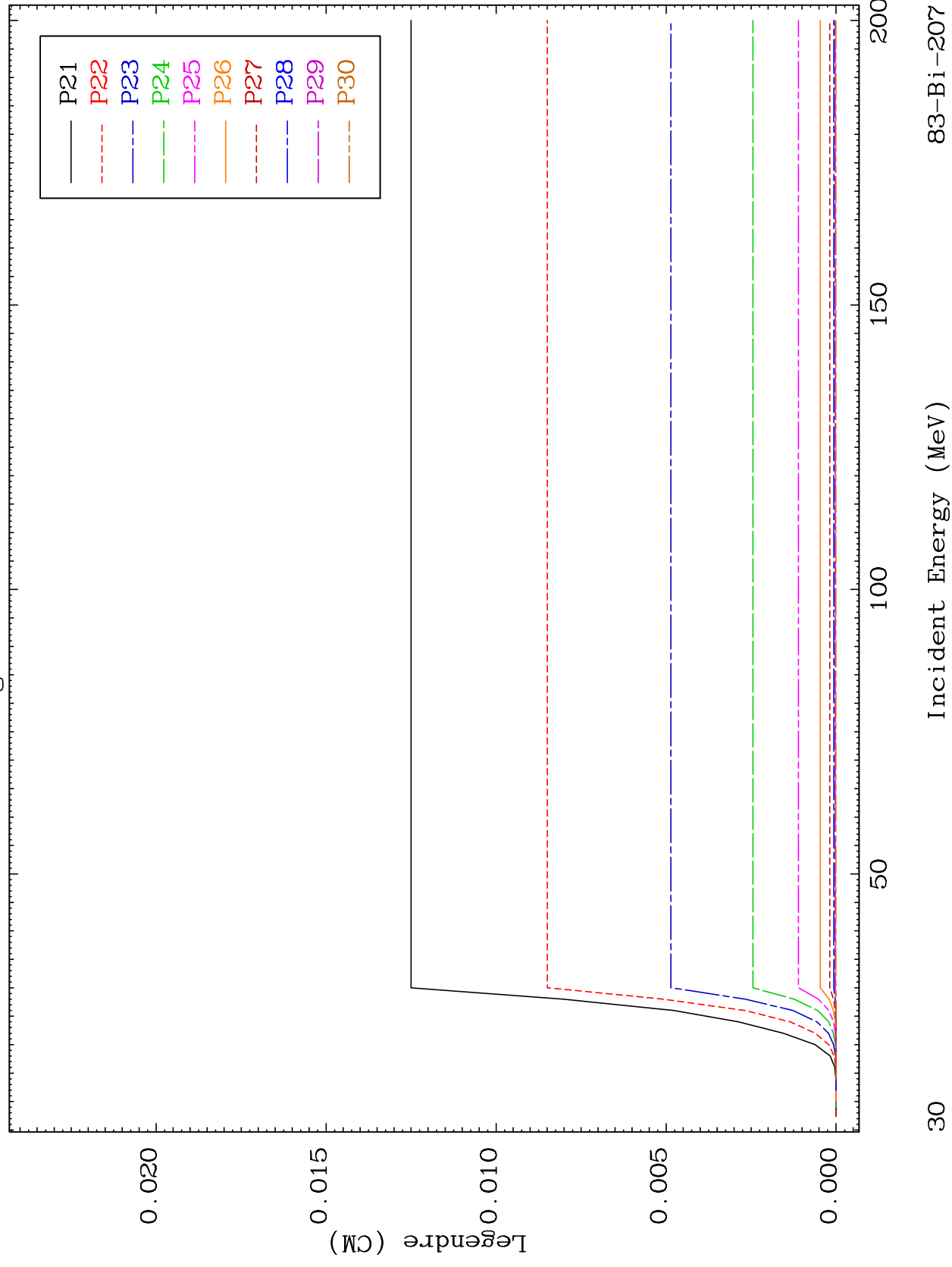


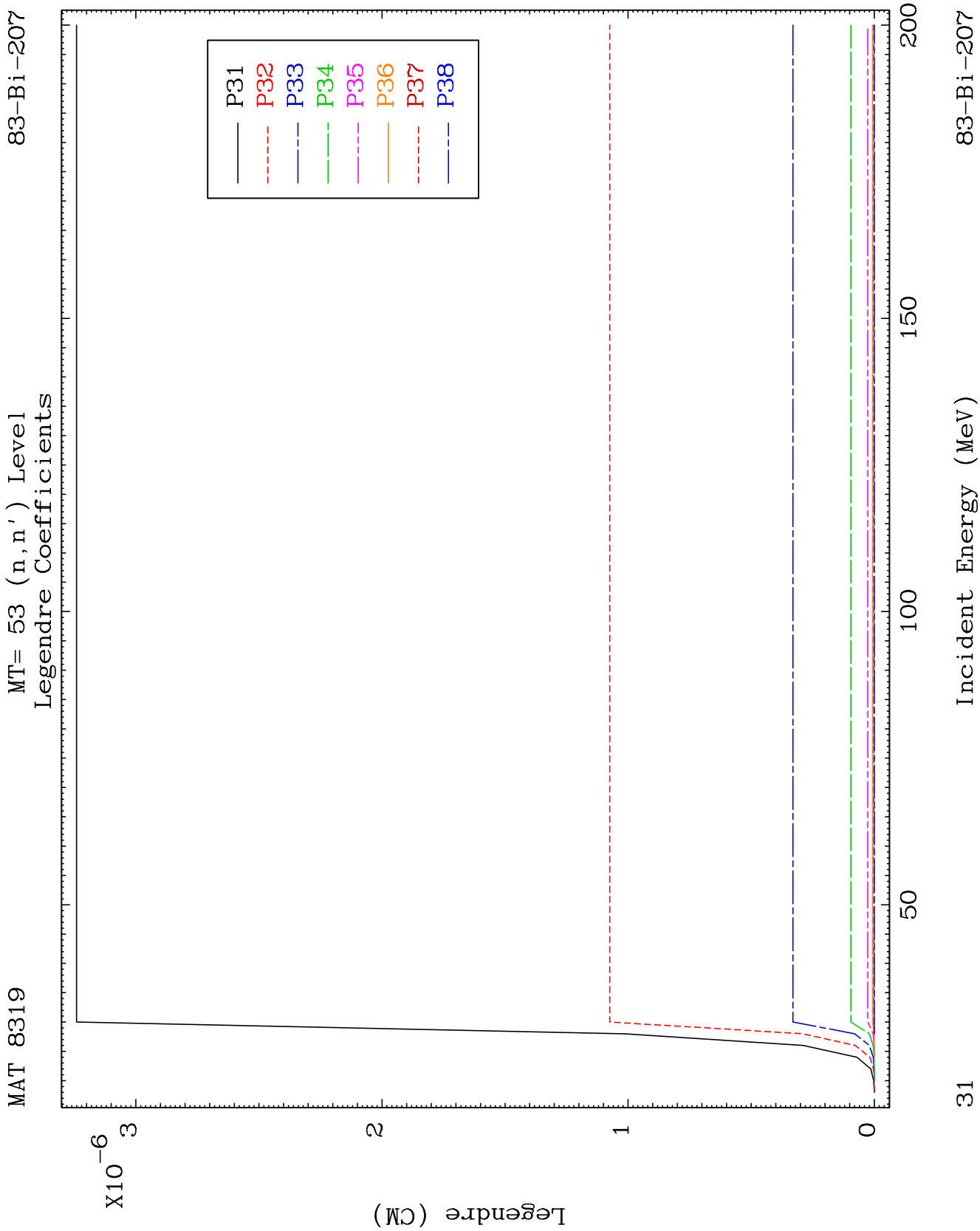


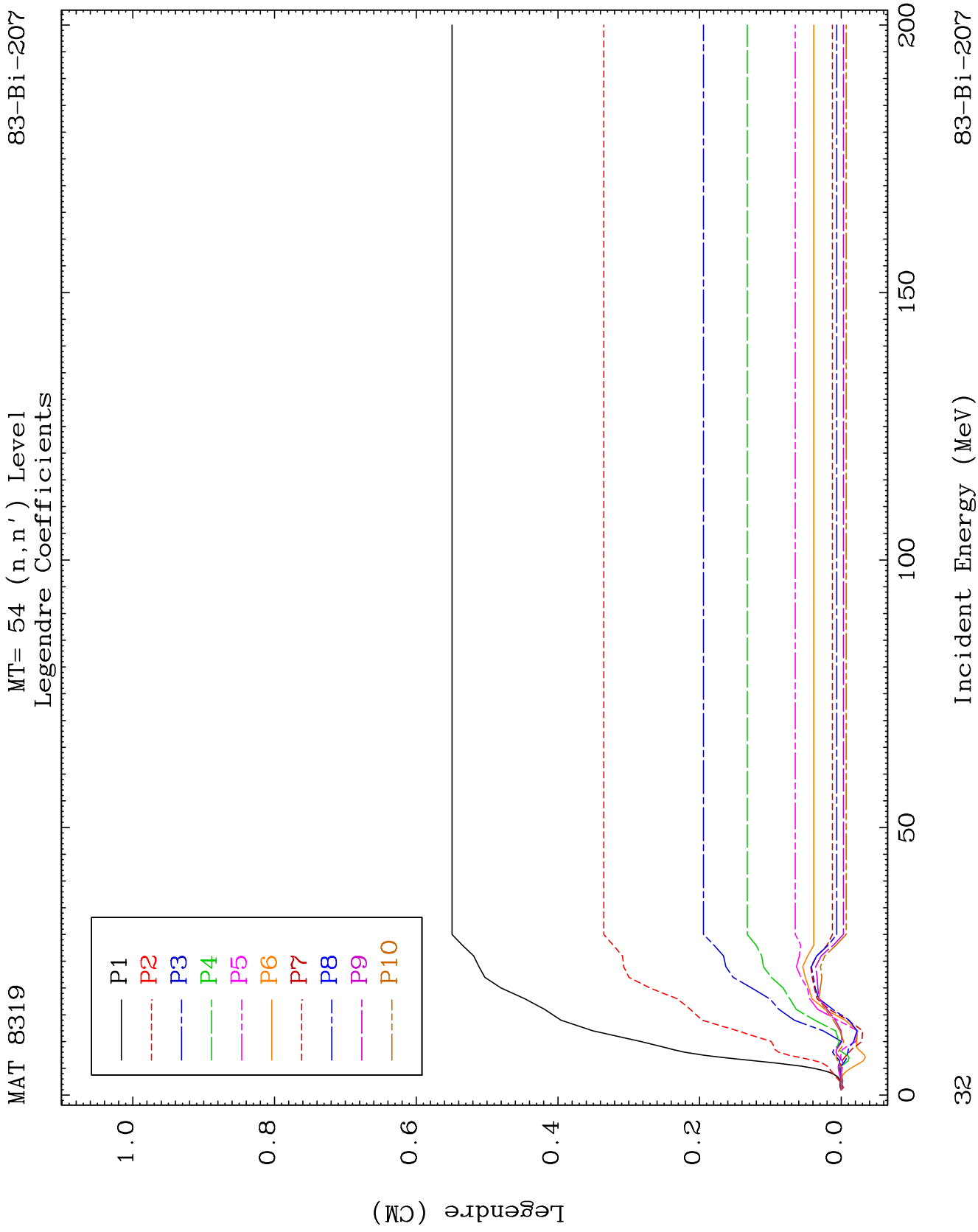
MAT 8319

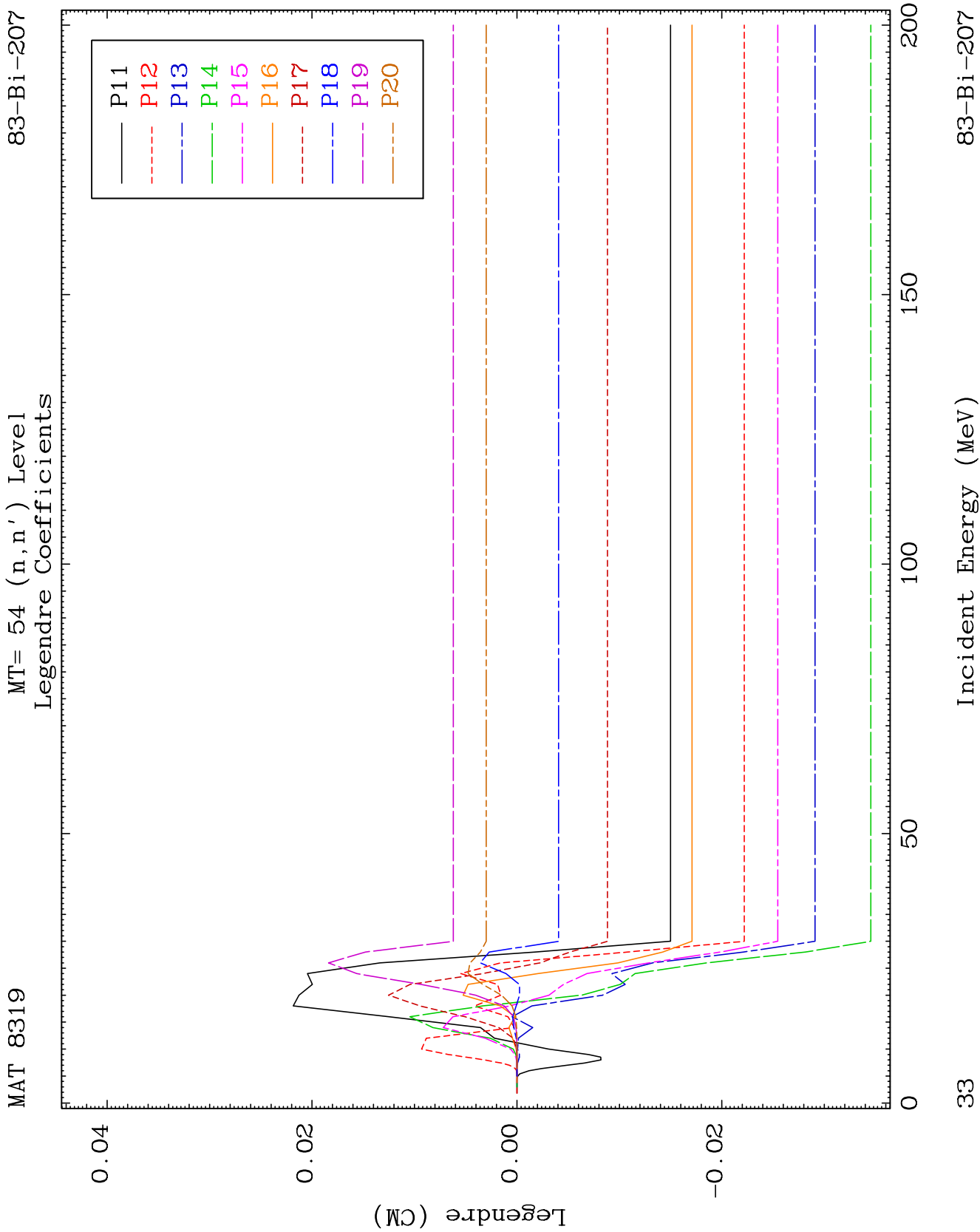
MT= 53 (n, n') Level
Legendre Coefficients

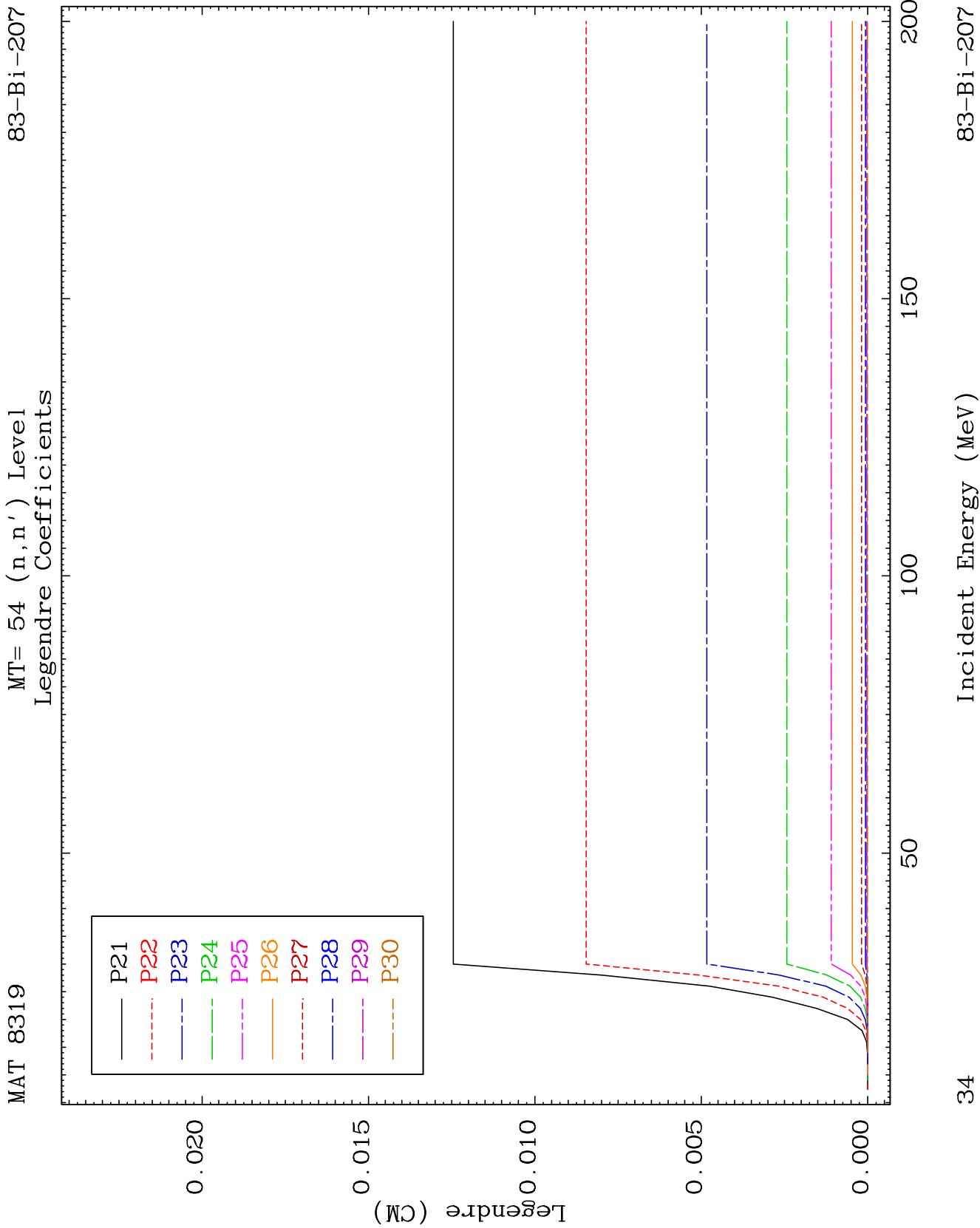
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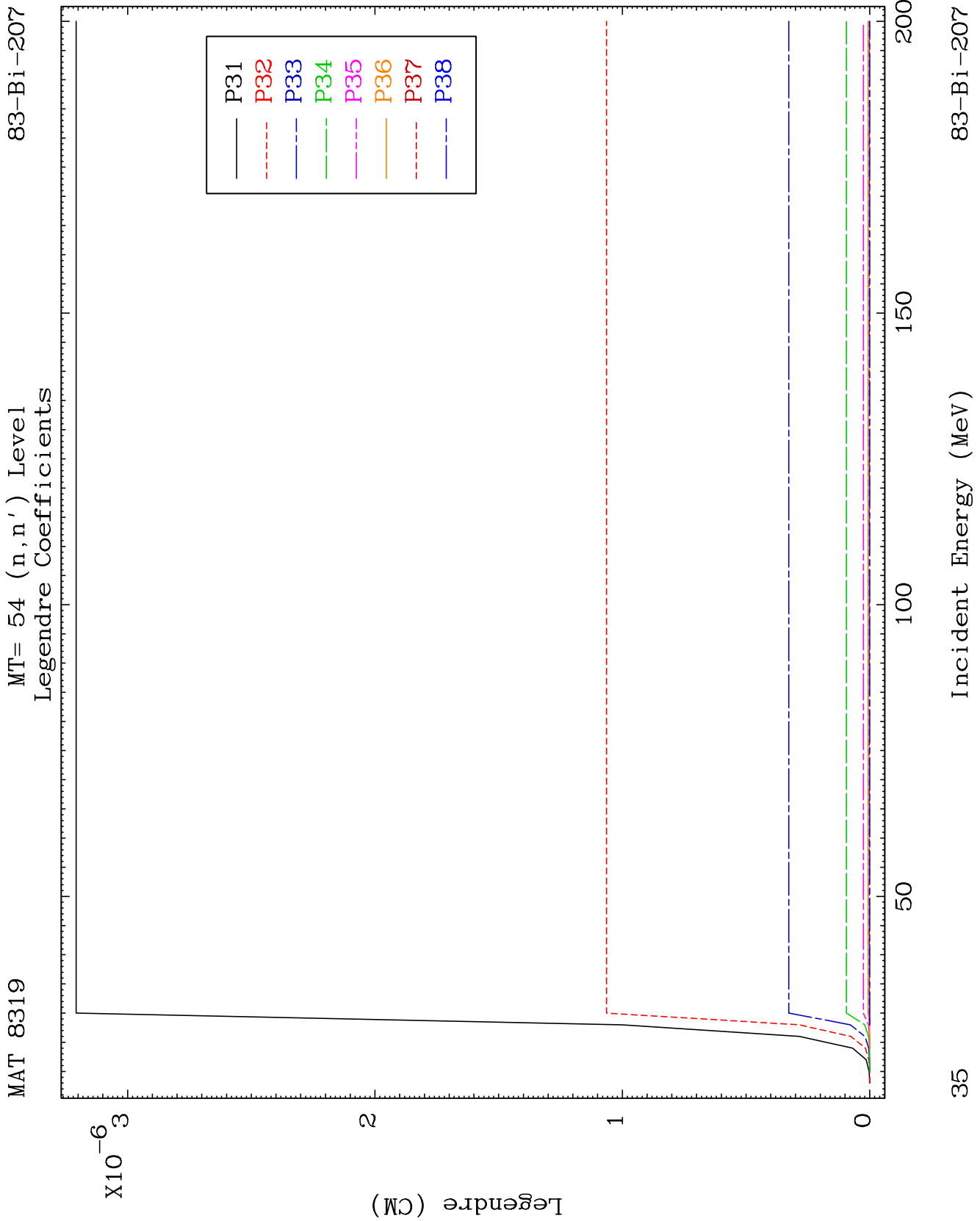








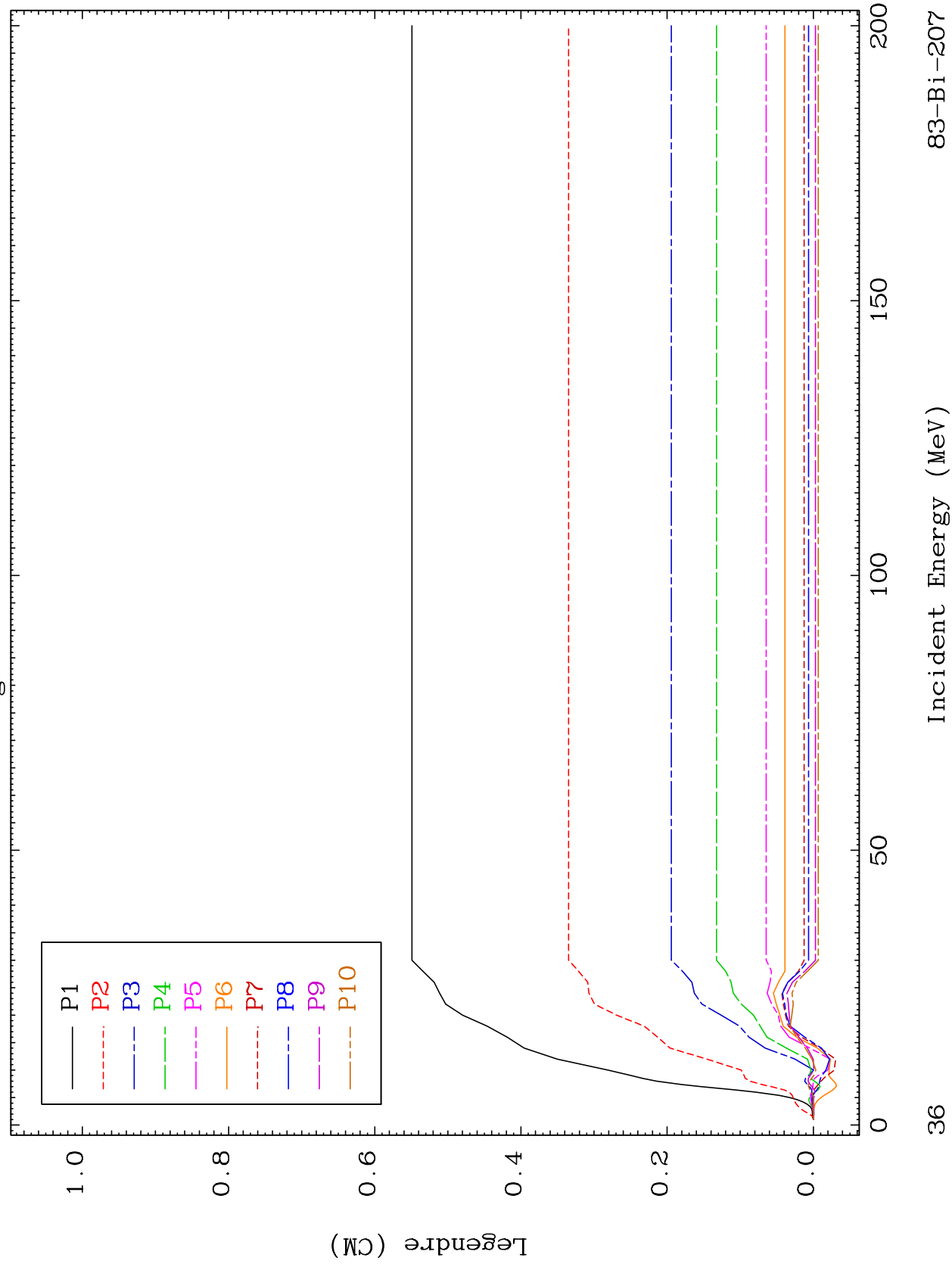


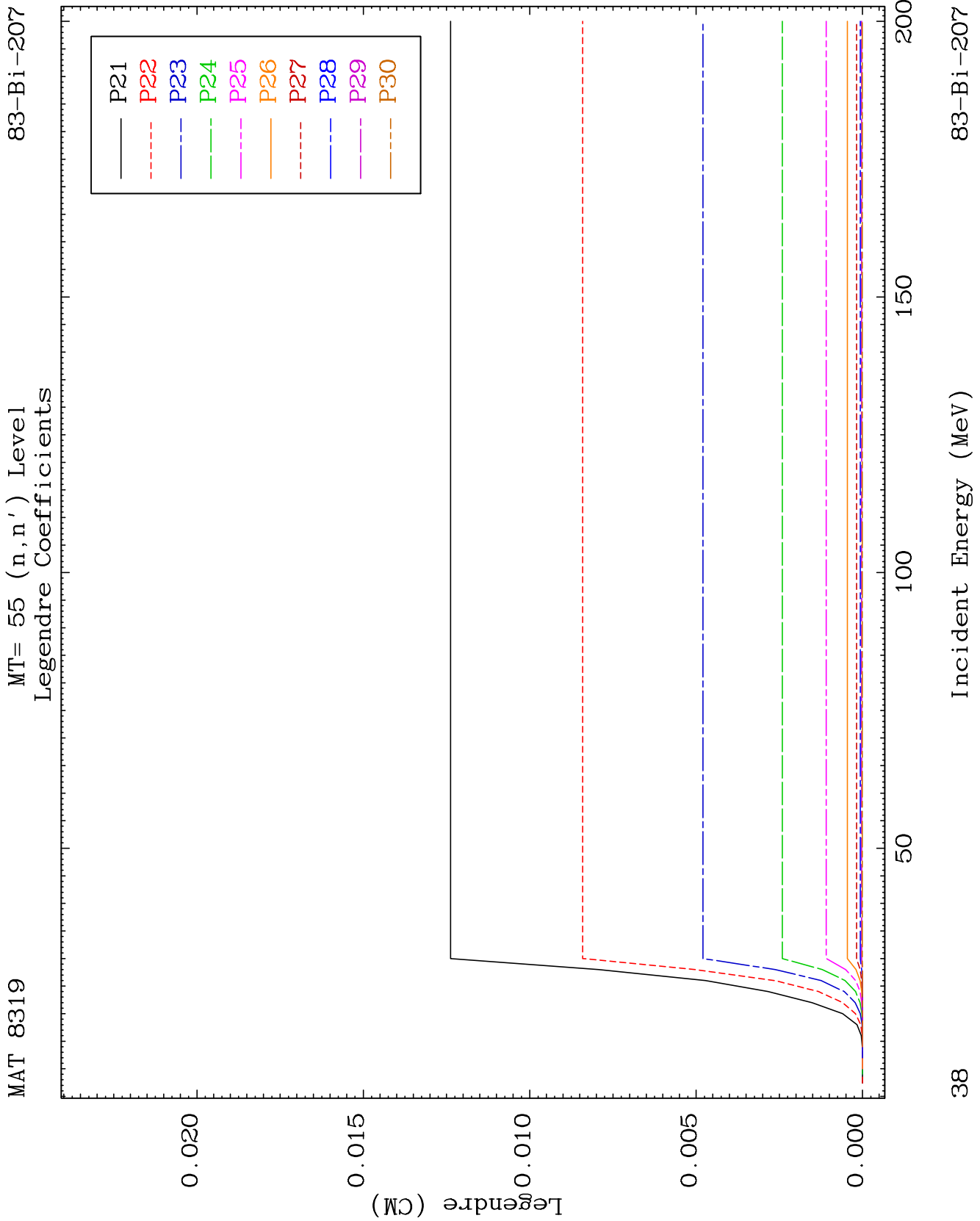


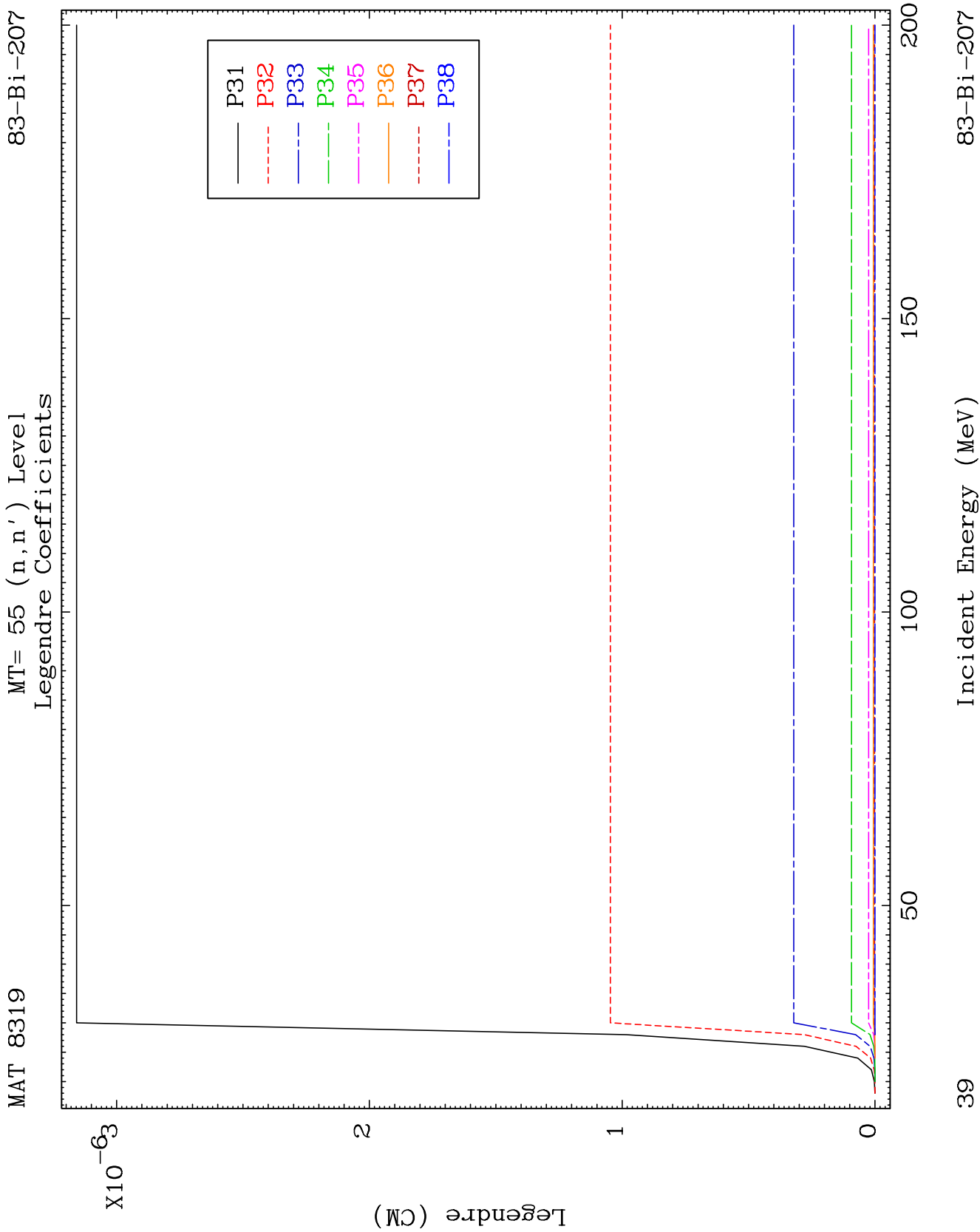
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MT= 55 (n, n') Level
Legendre Coefficients

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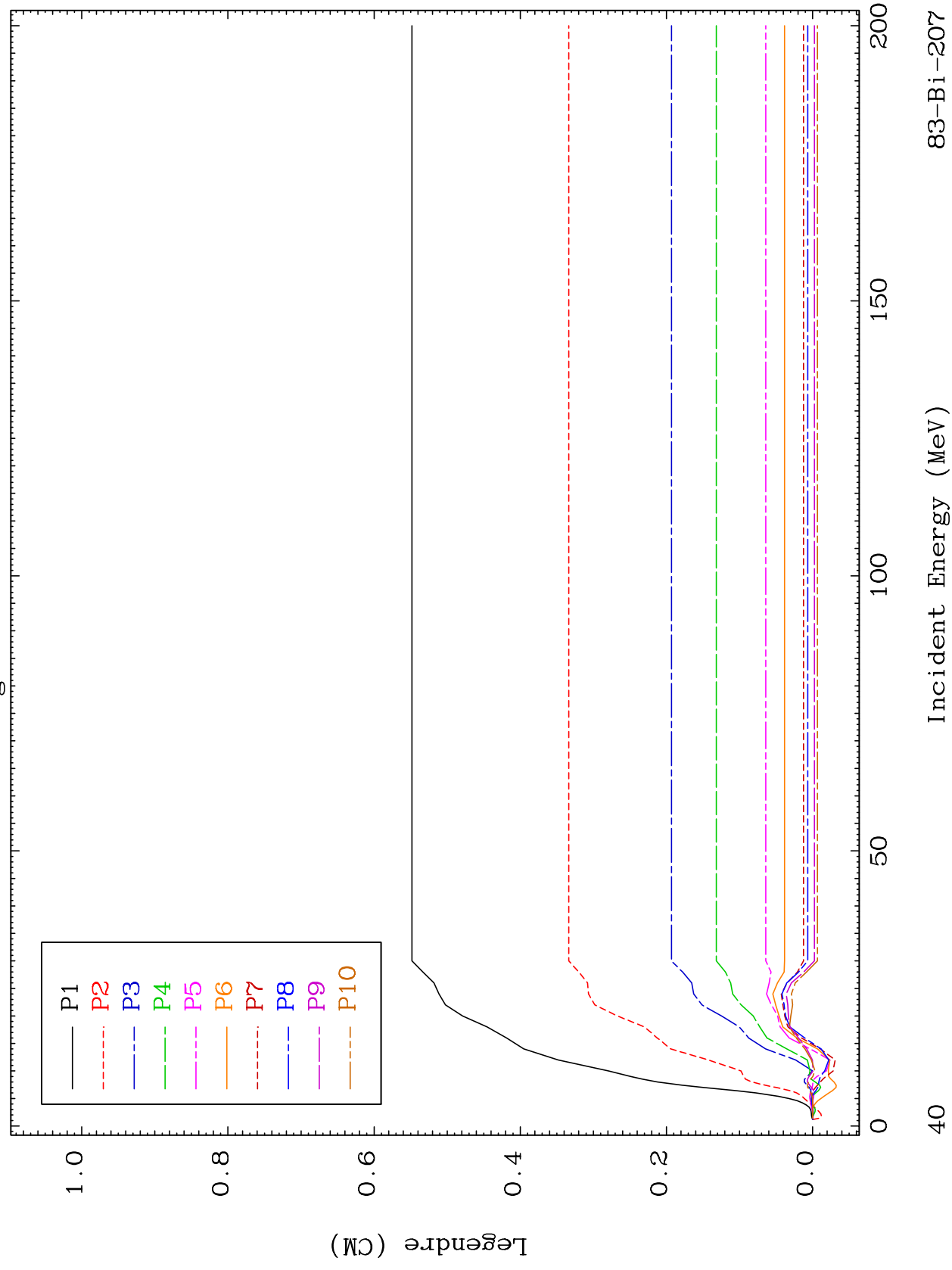


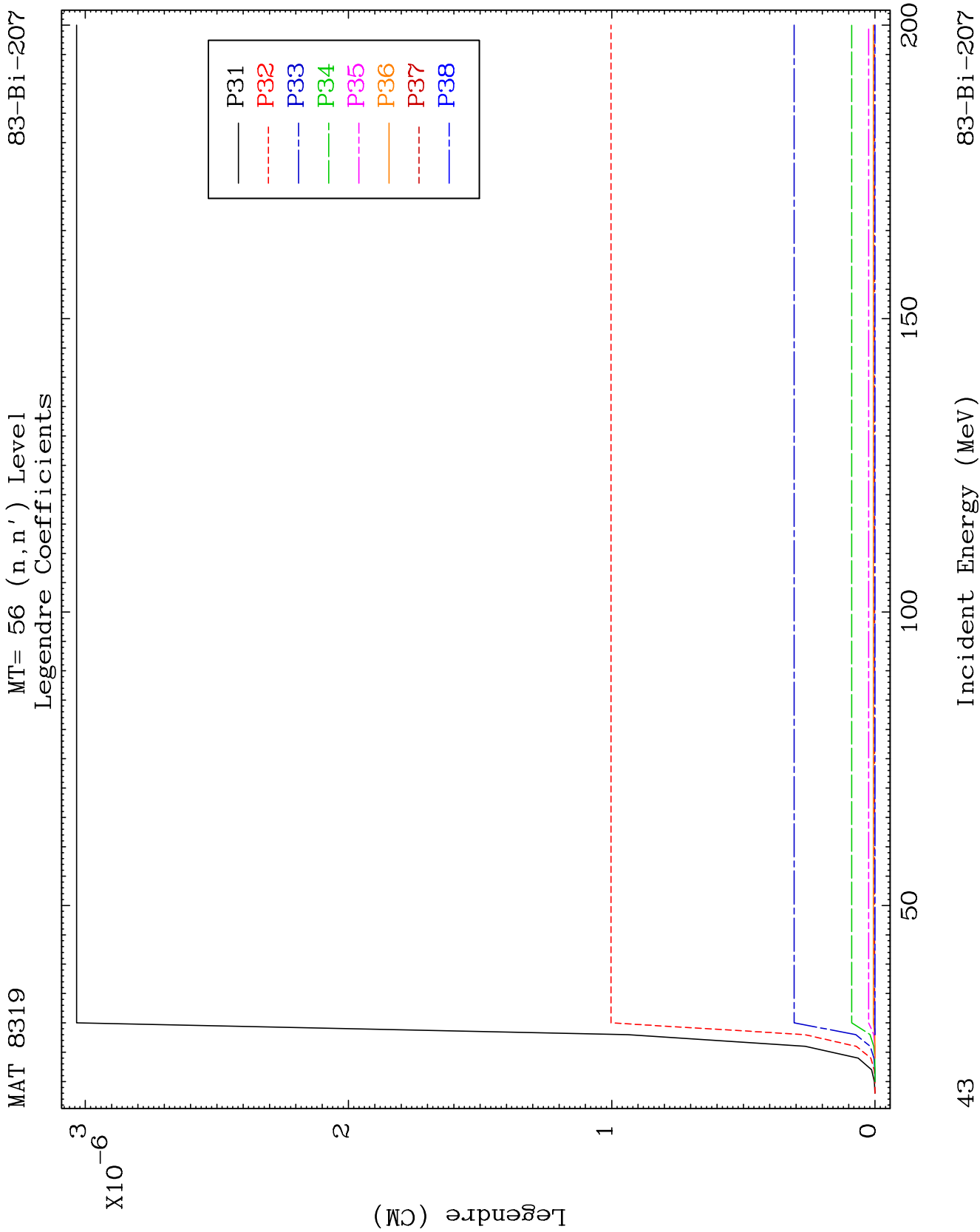


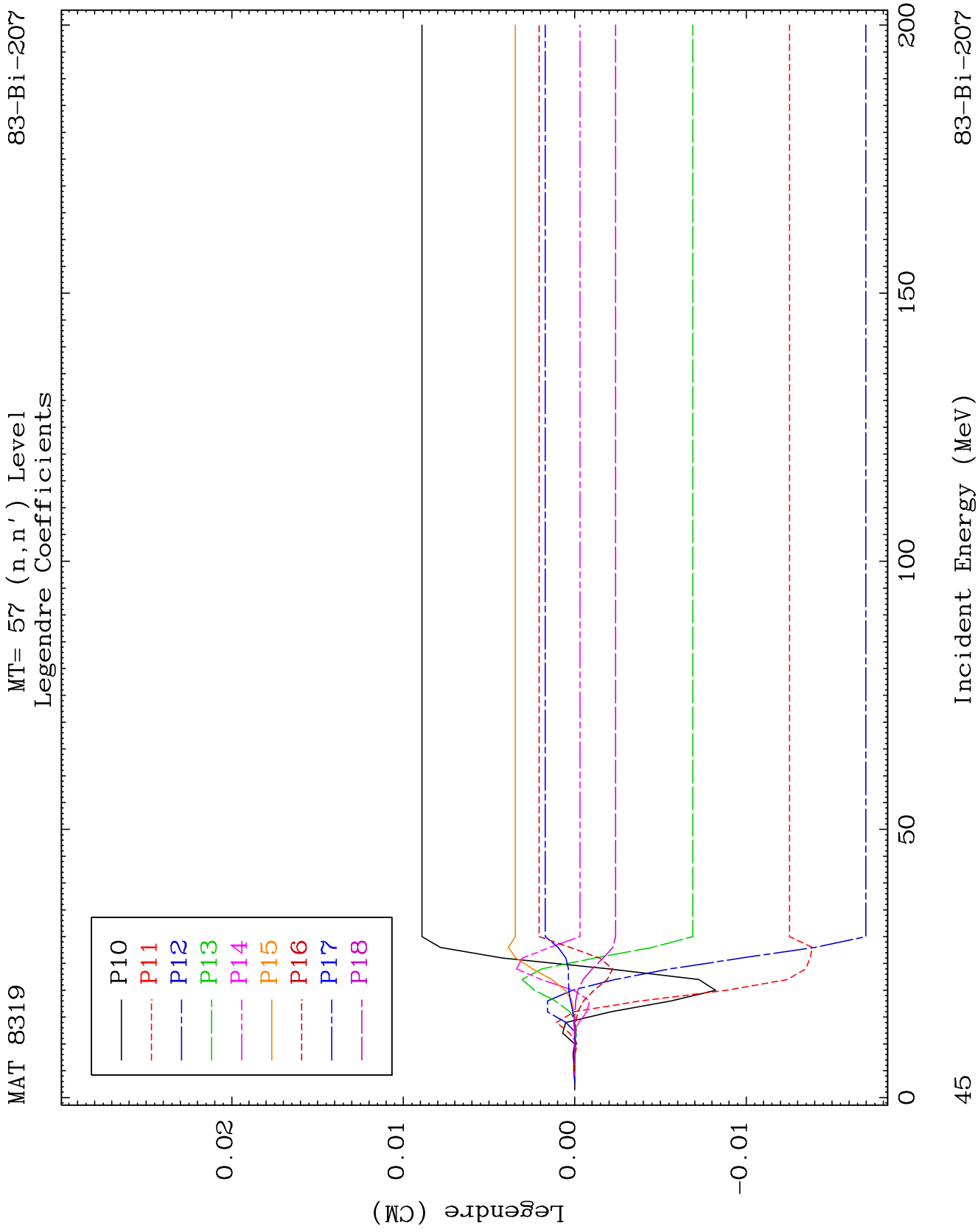
MAT 8319

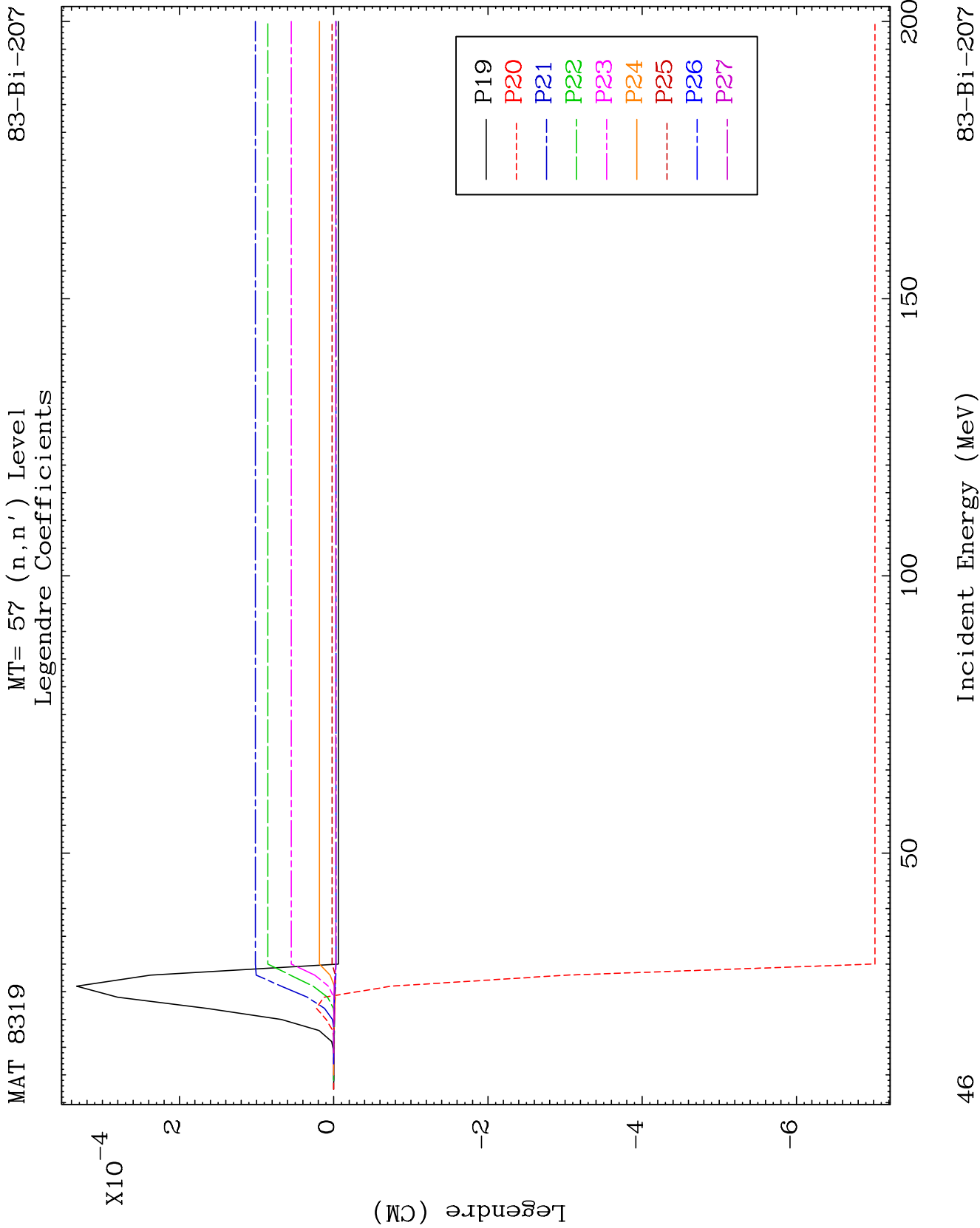
MT= 56 (n, n') Level
Legendre Coefficients

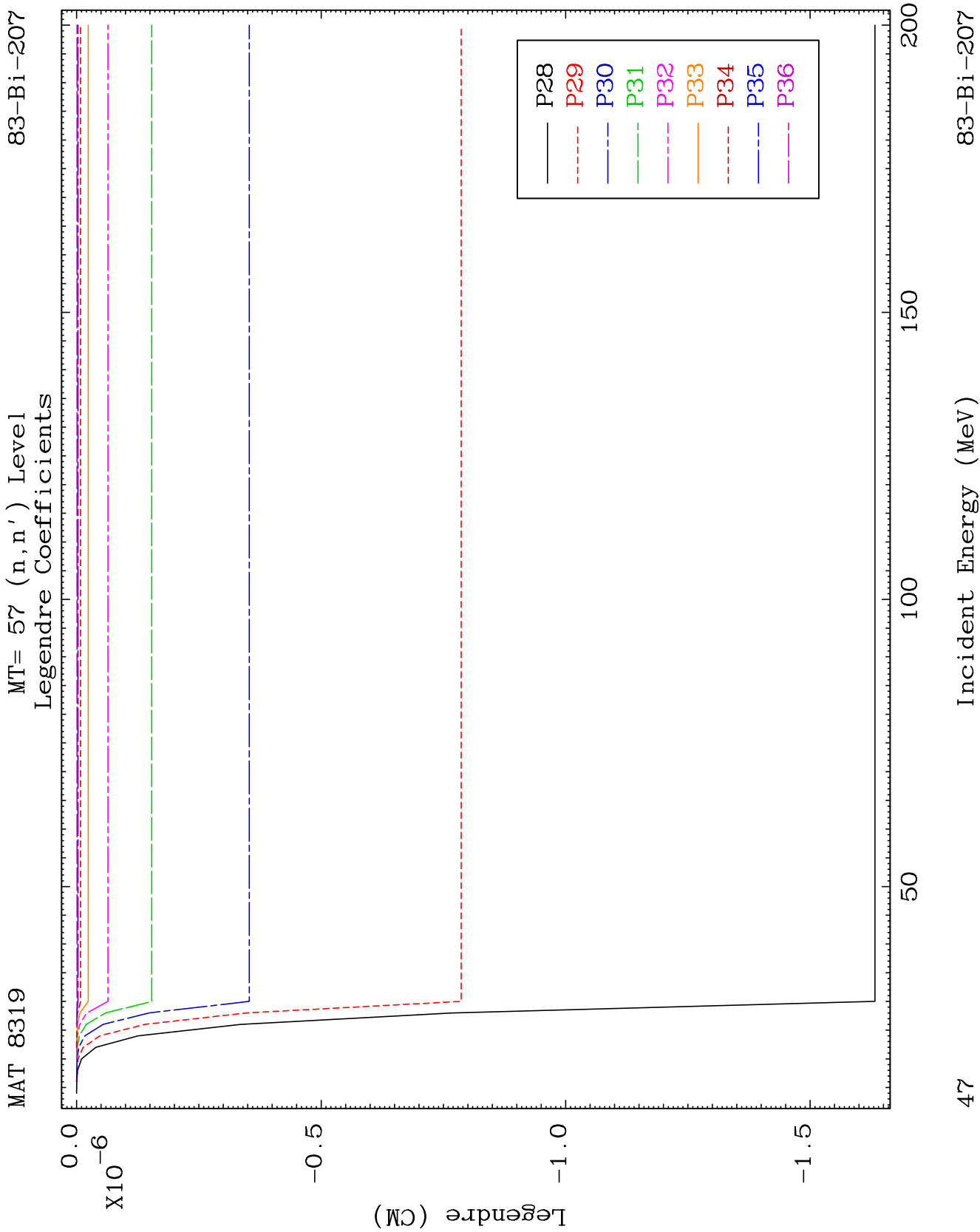
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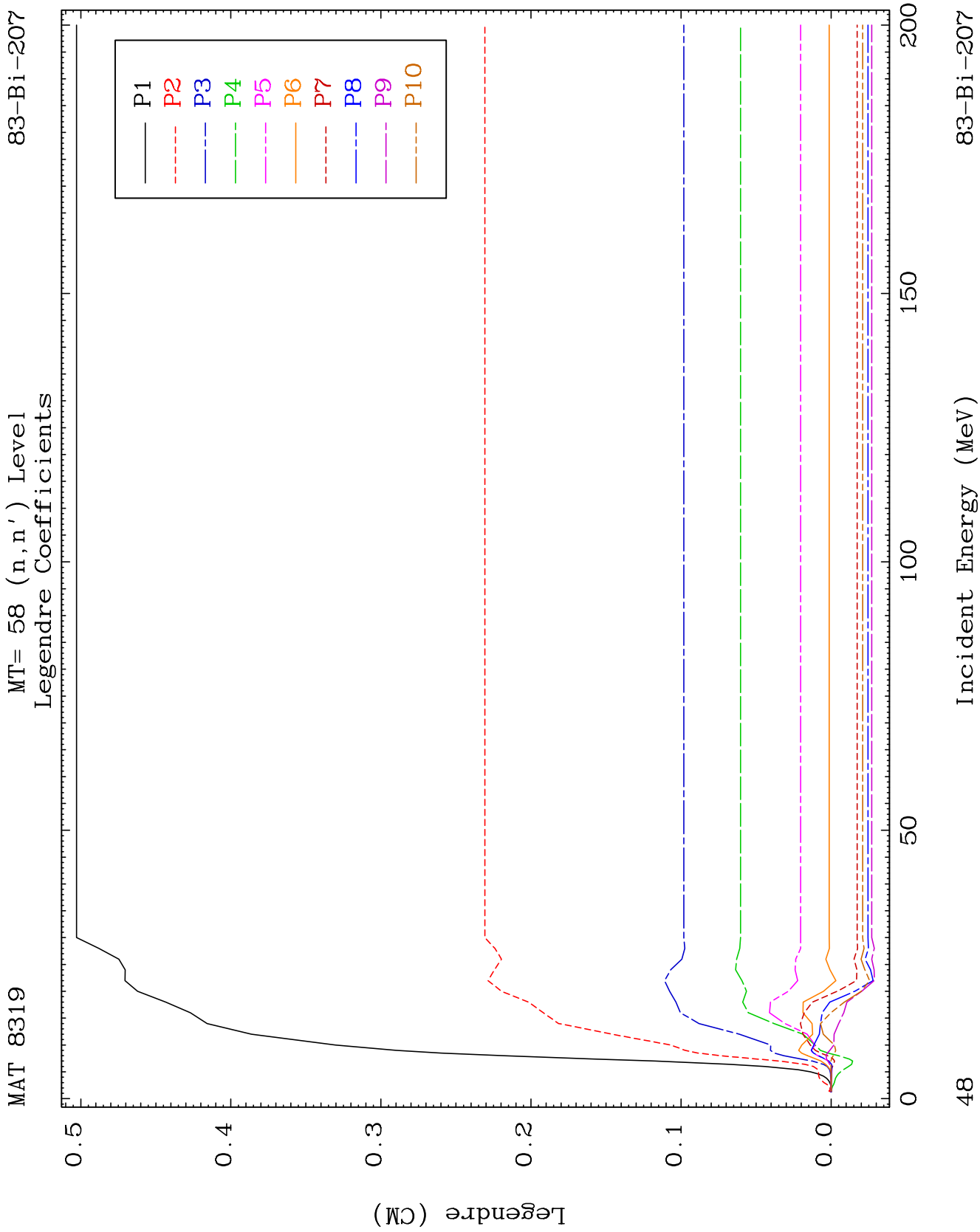


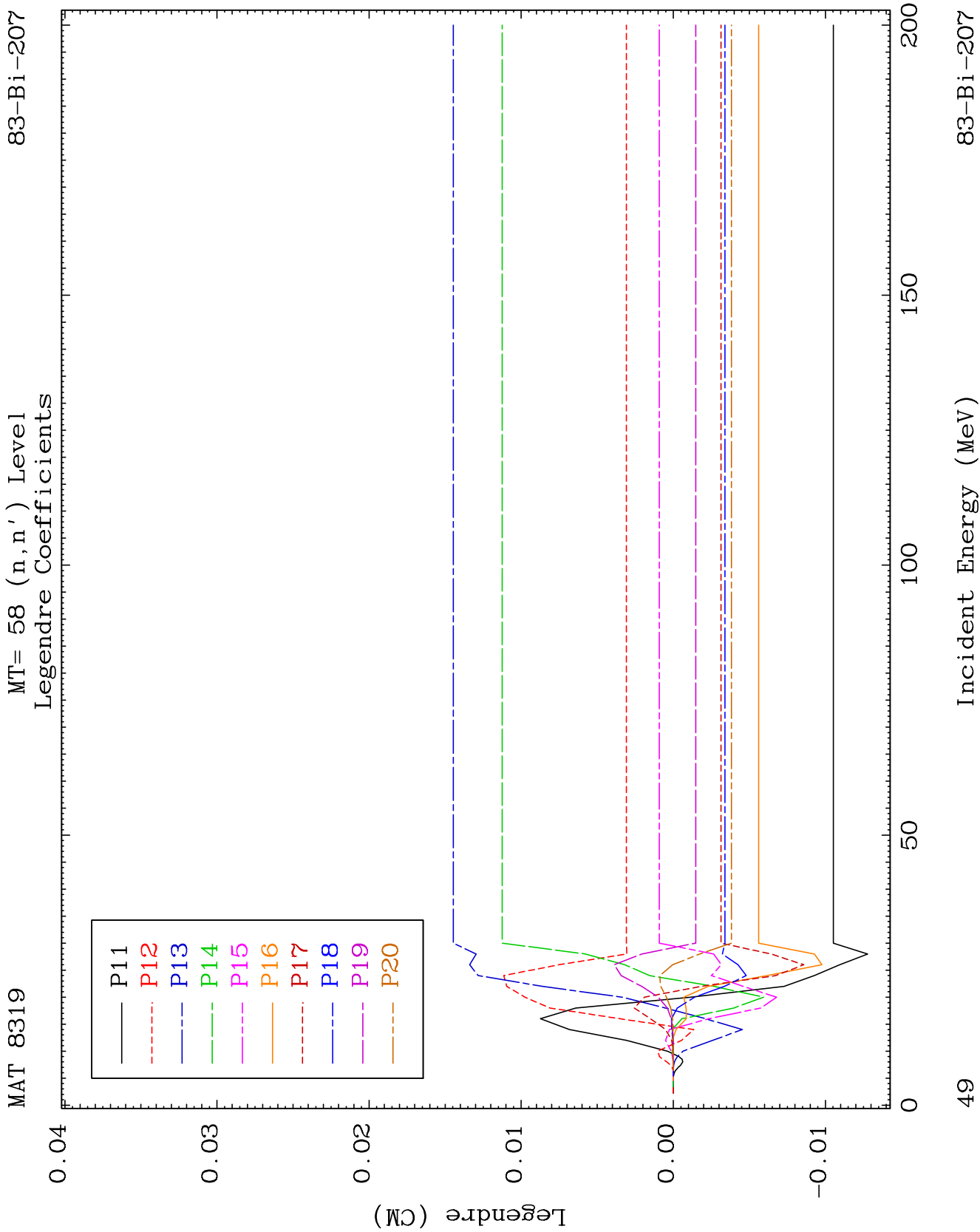


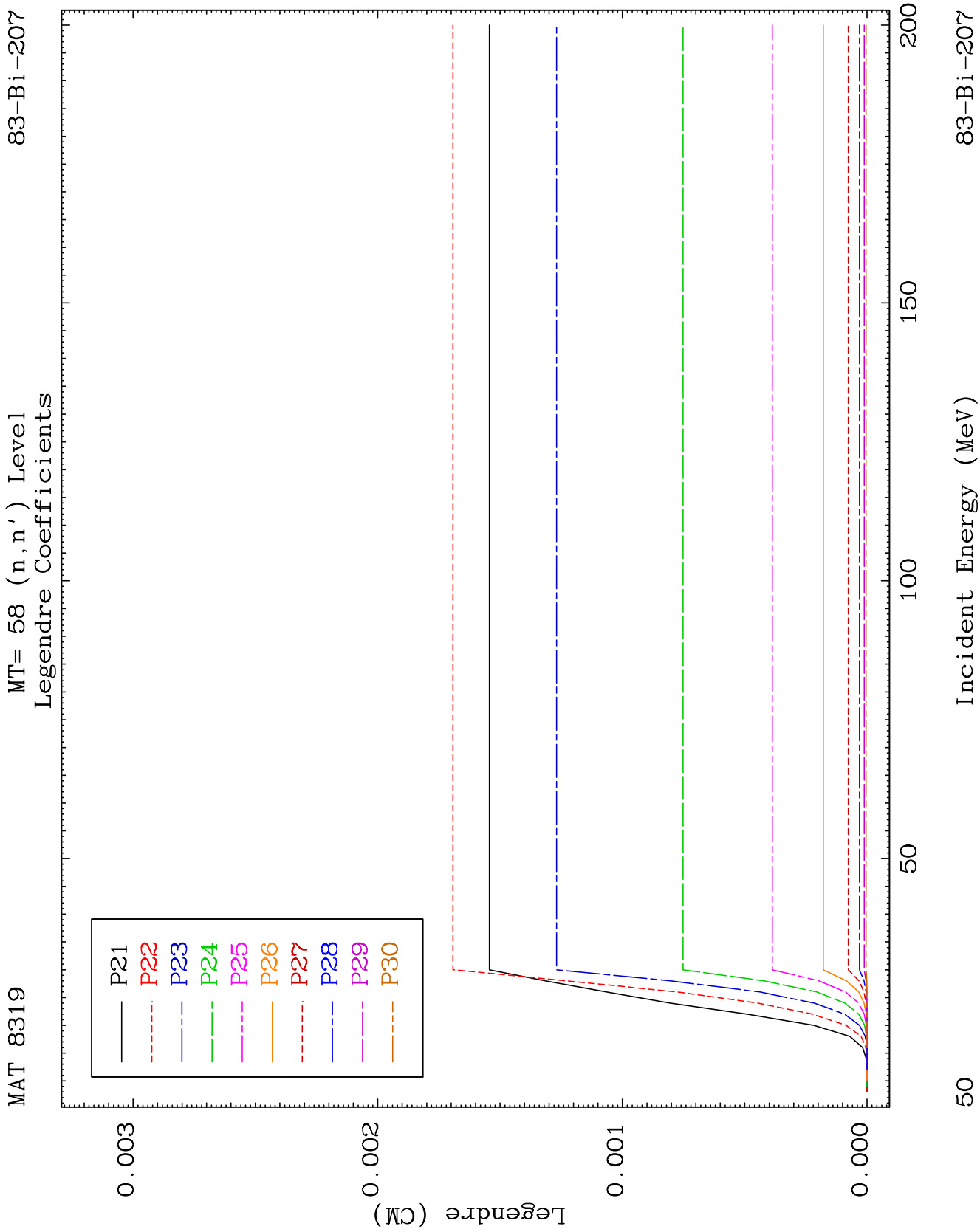


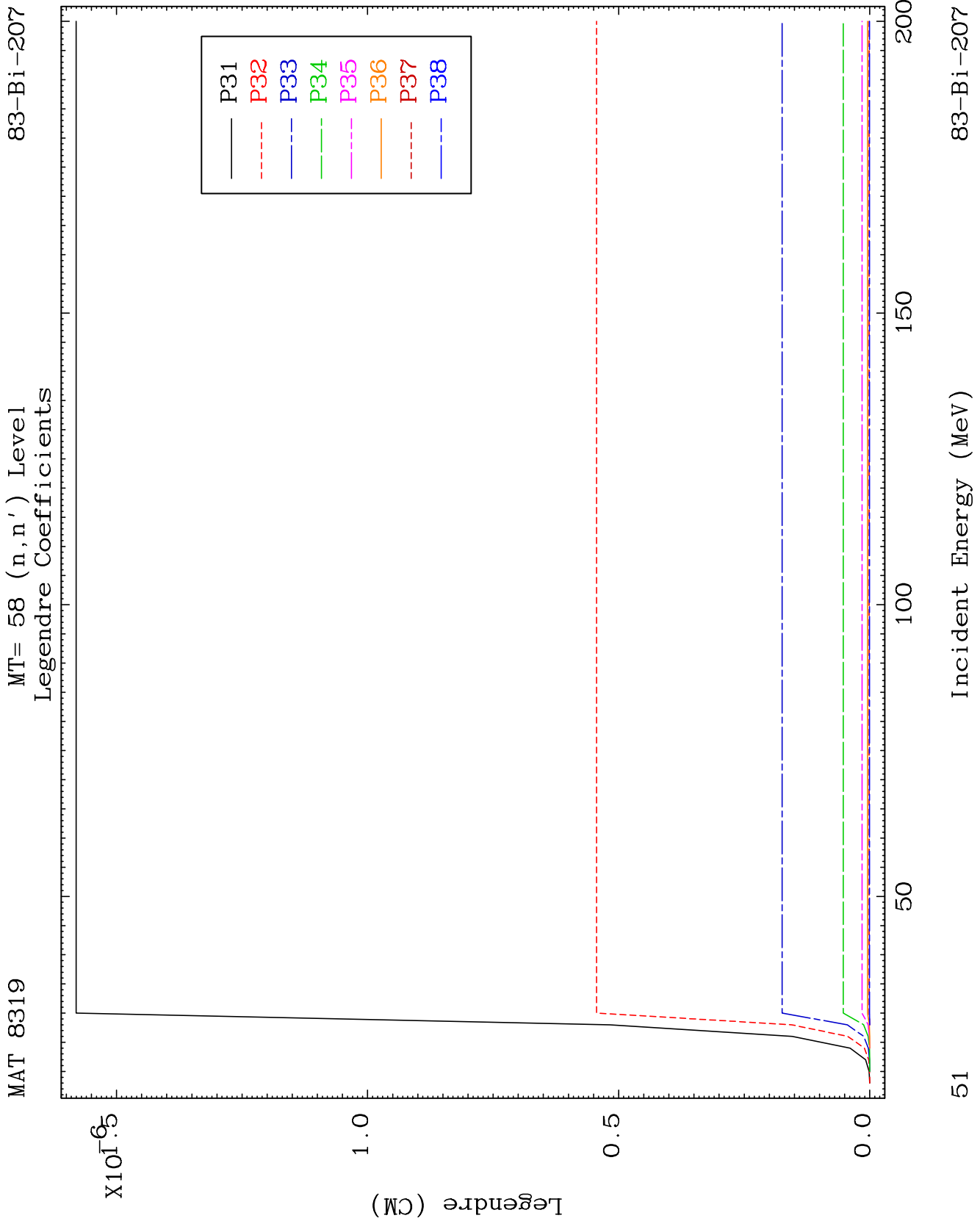








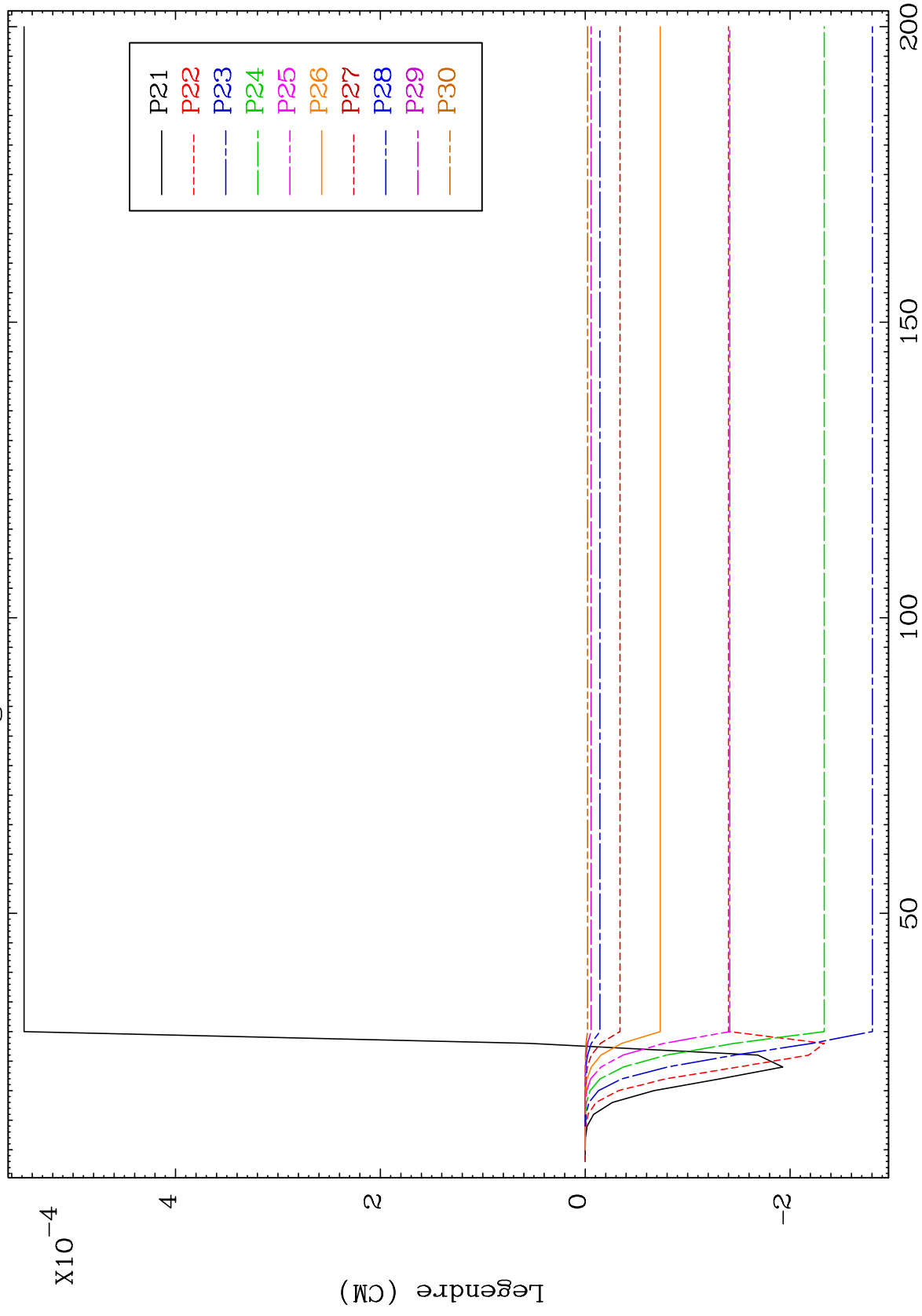




MAT 8319

MT= 59 (n,n') Level
Legendre Coefficients

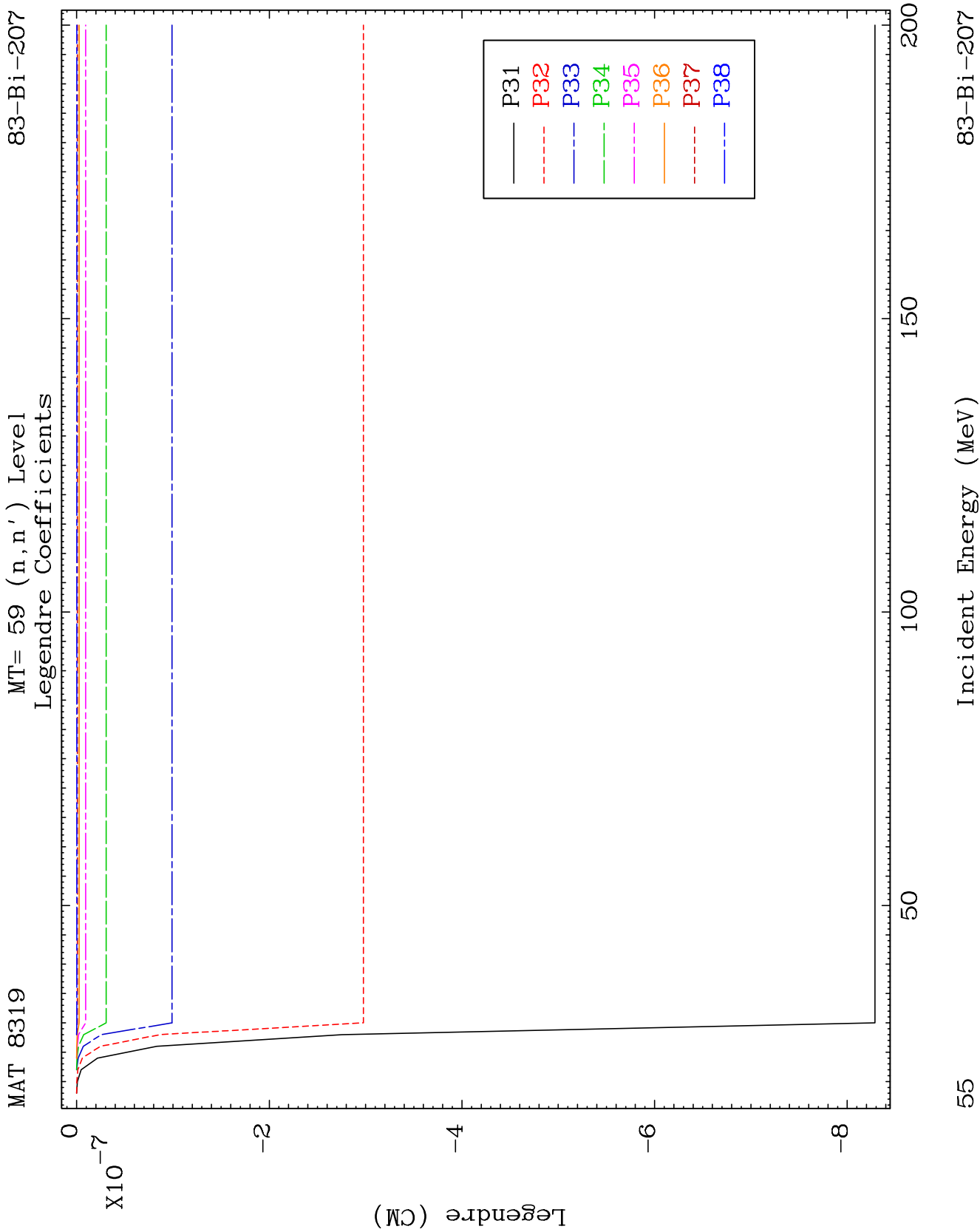
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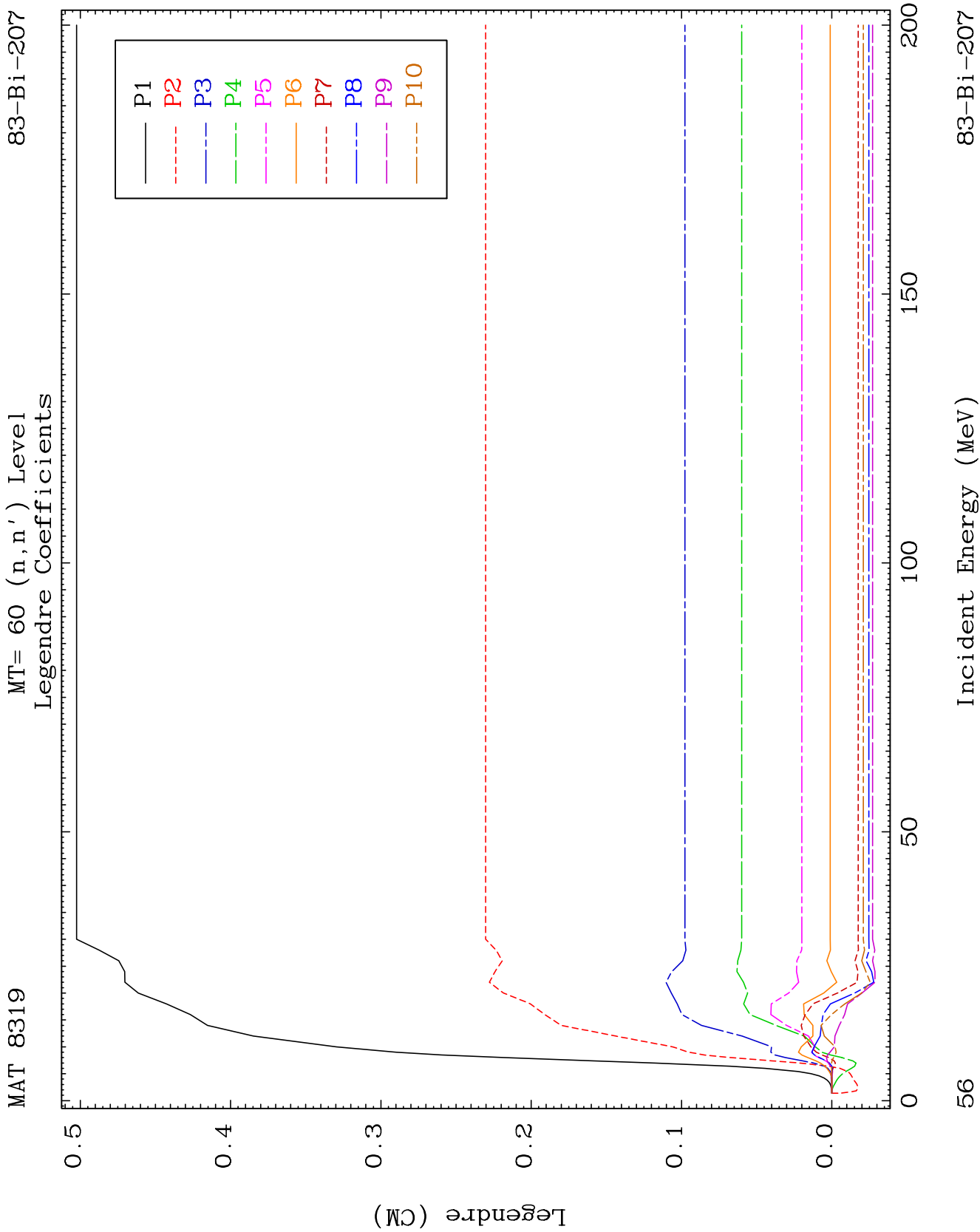


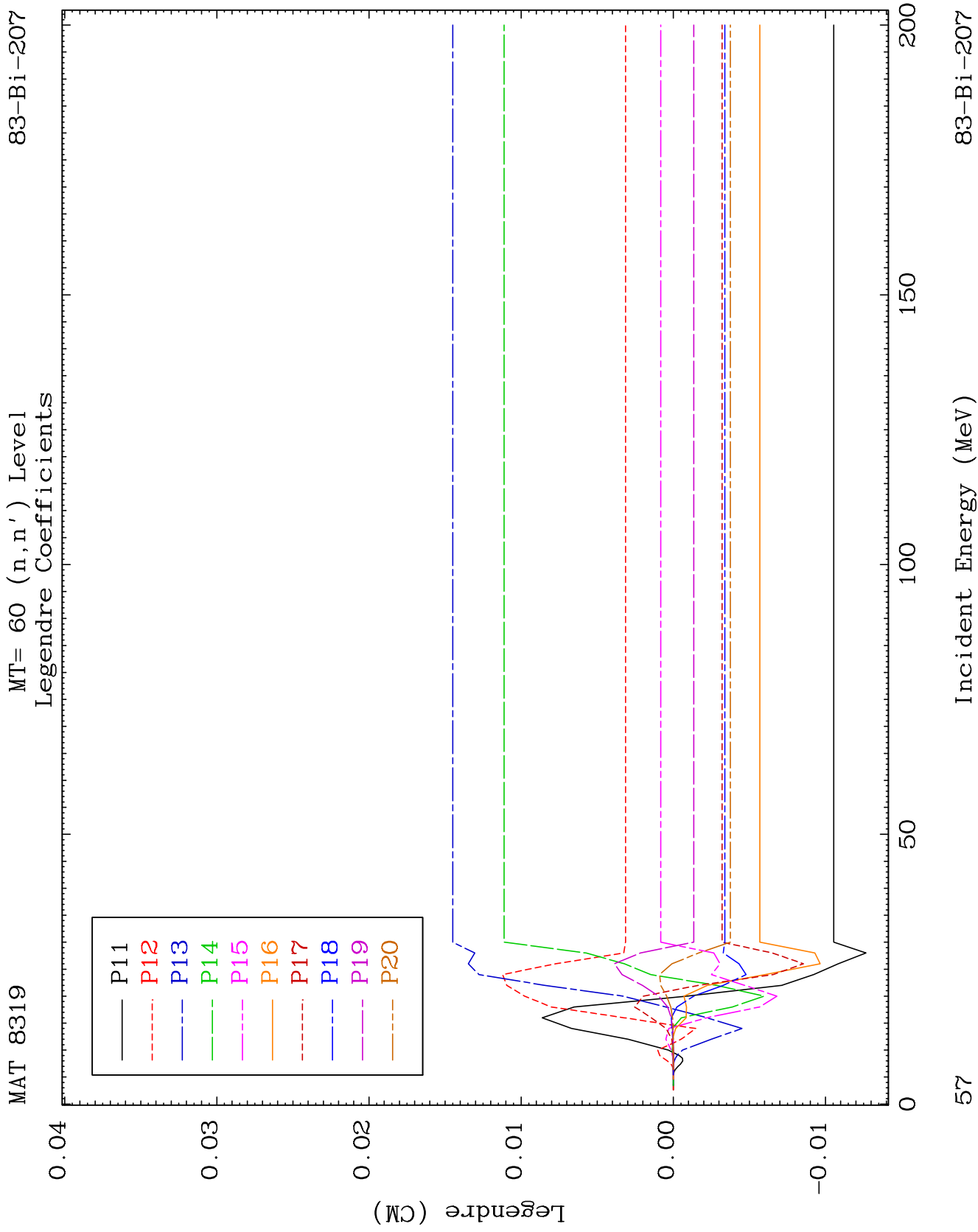
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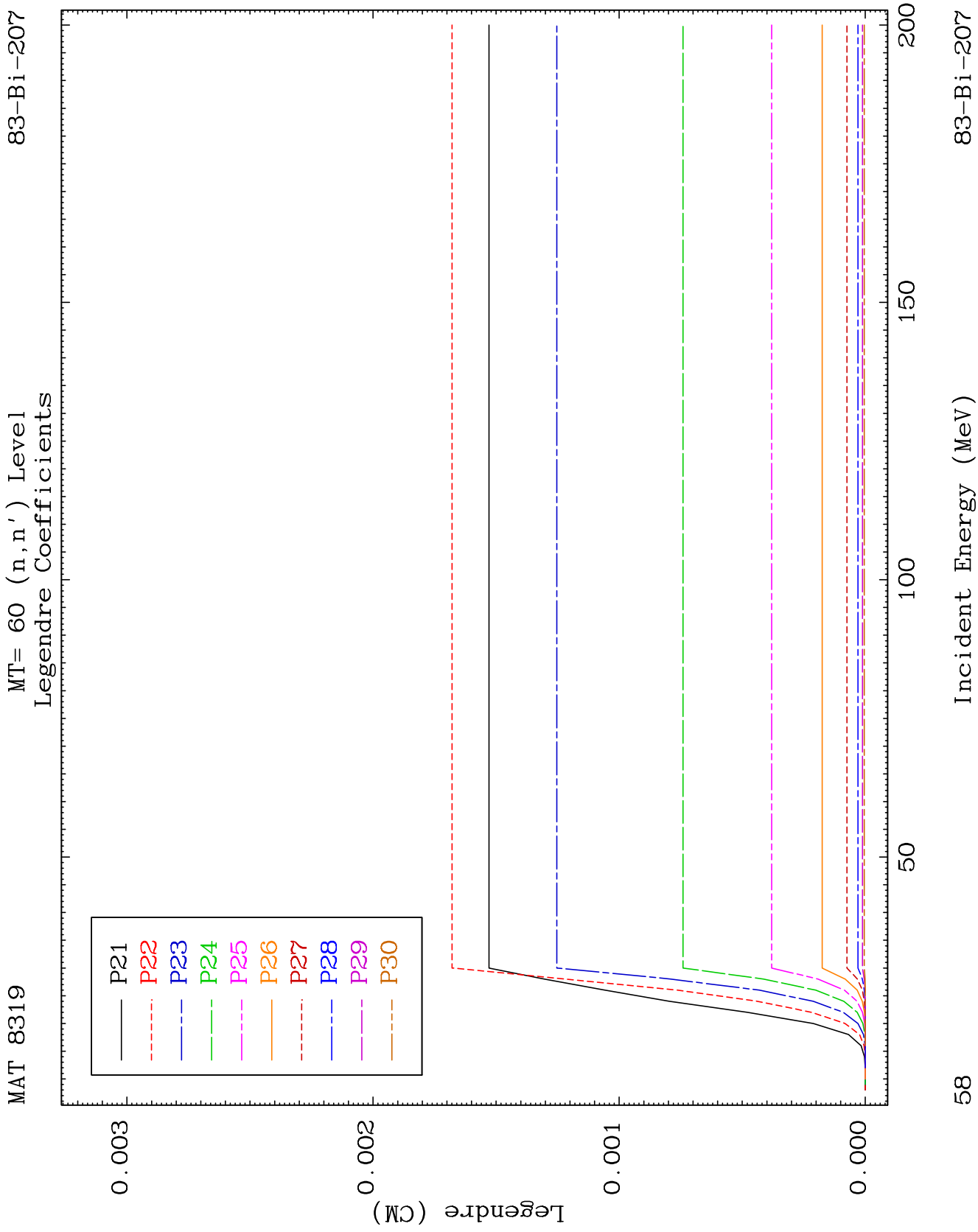
Incident Energy (MeV)

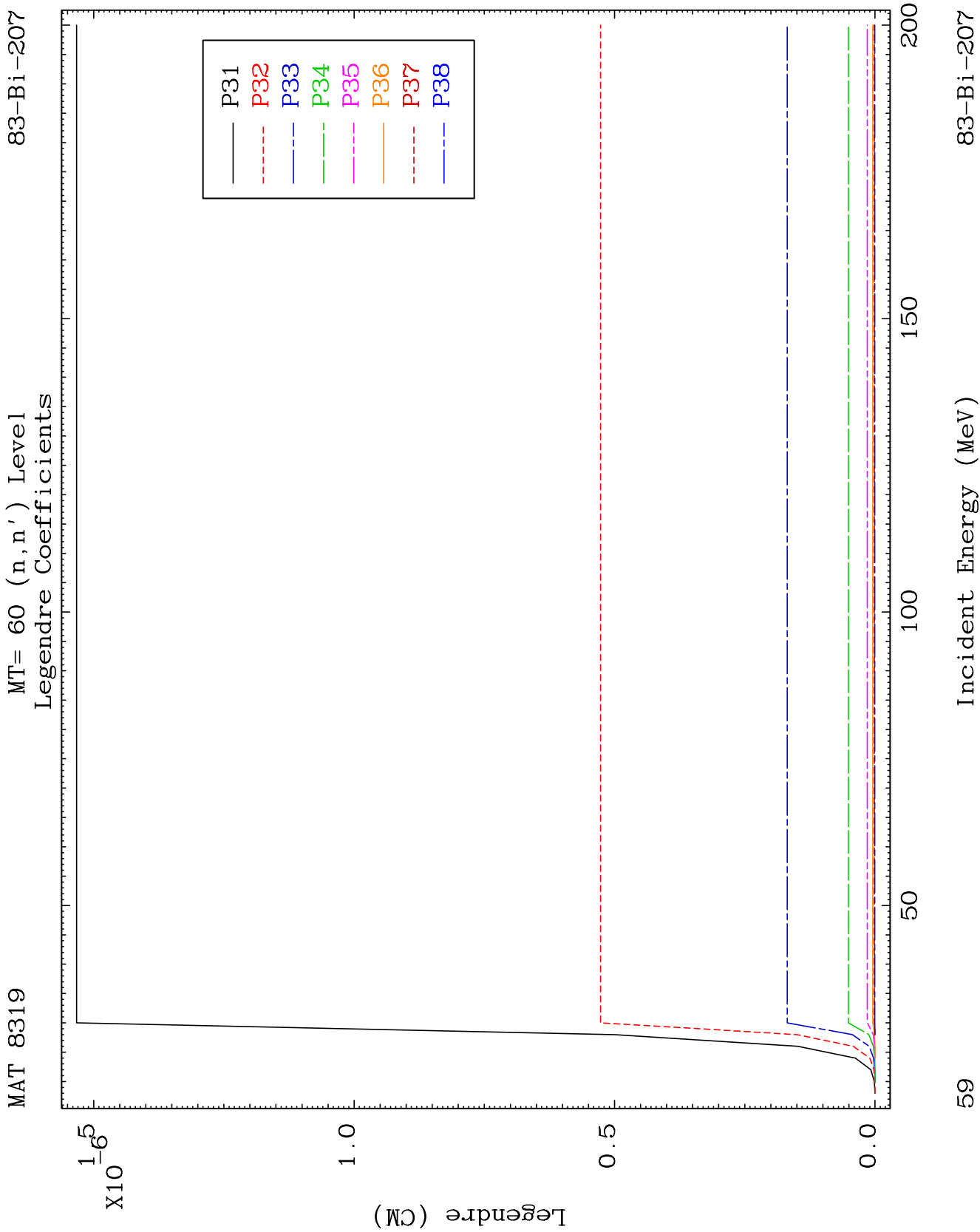
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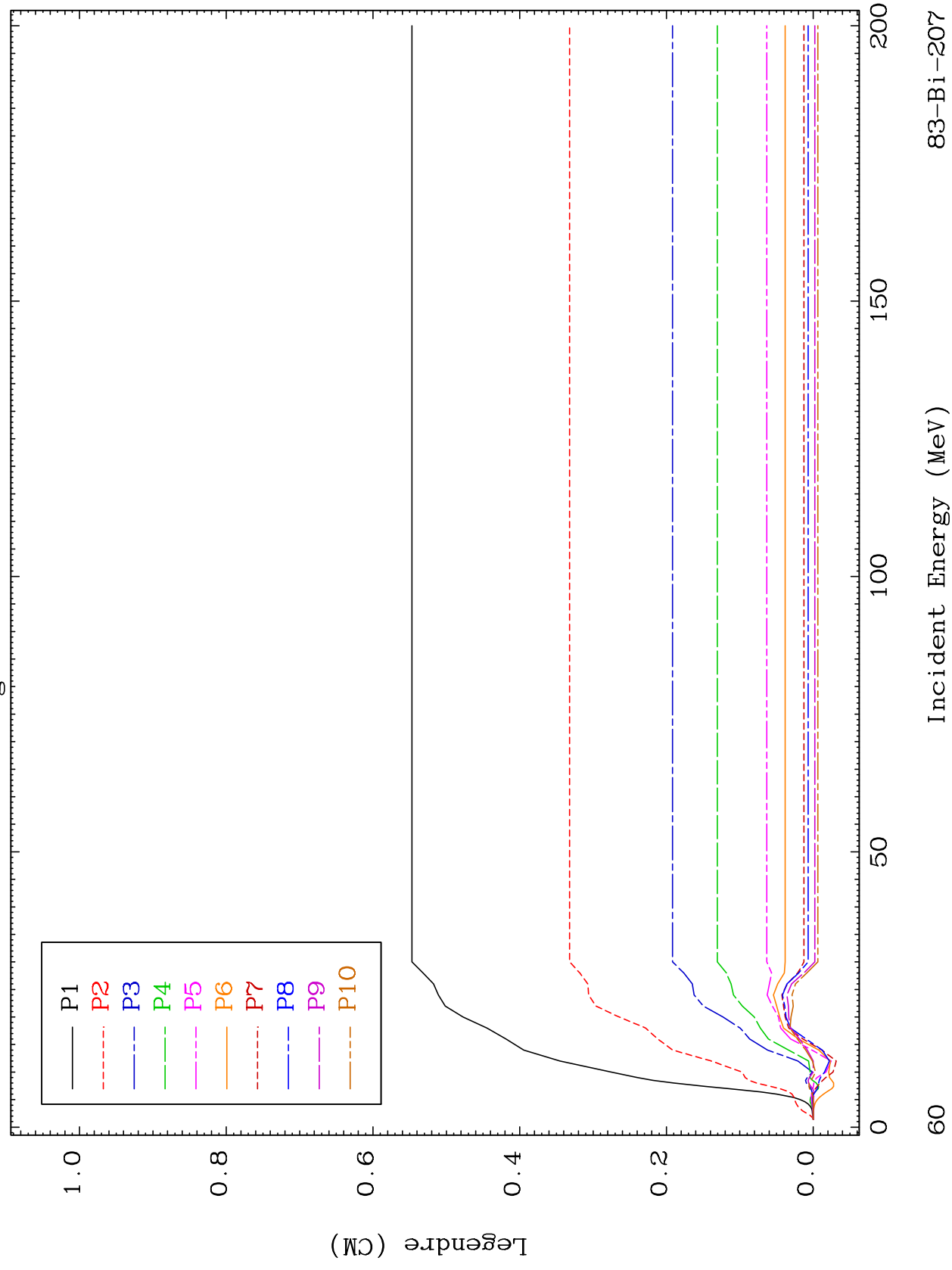


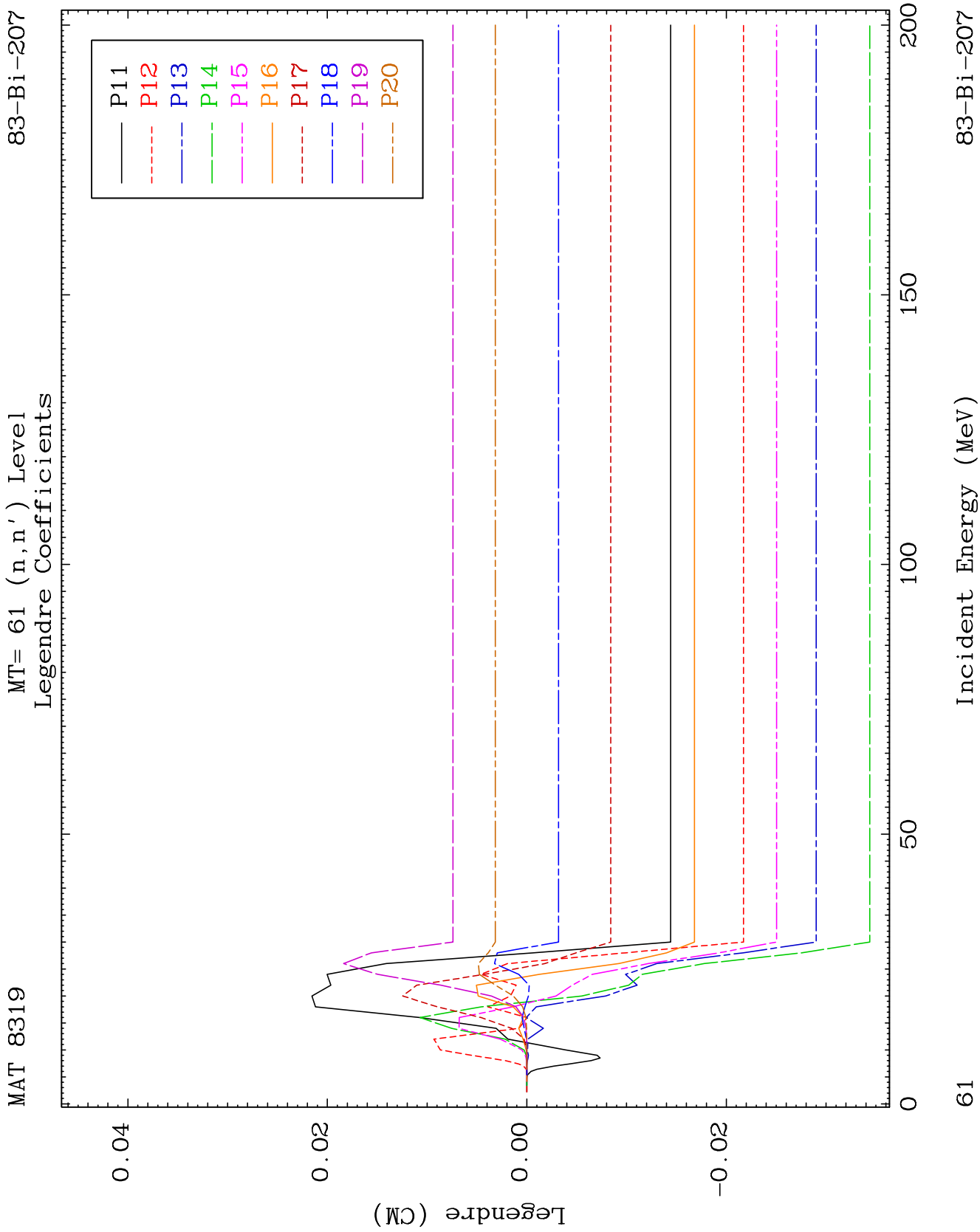


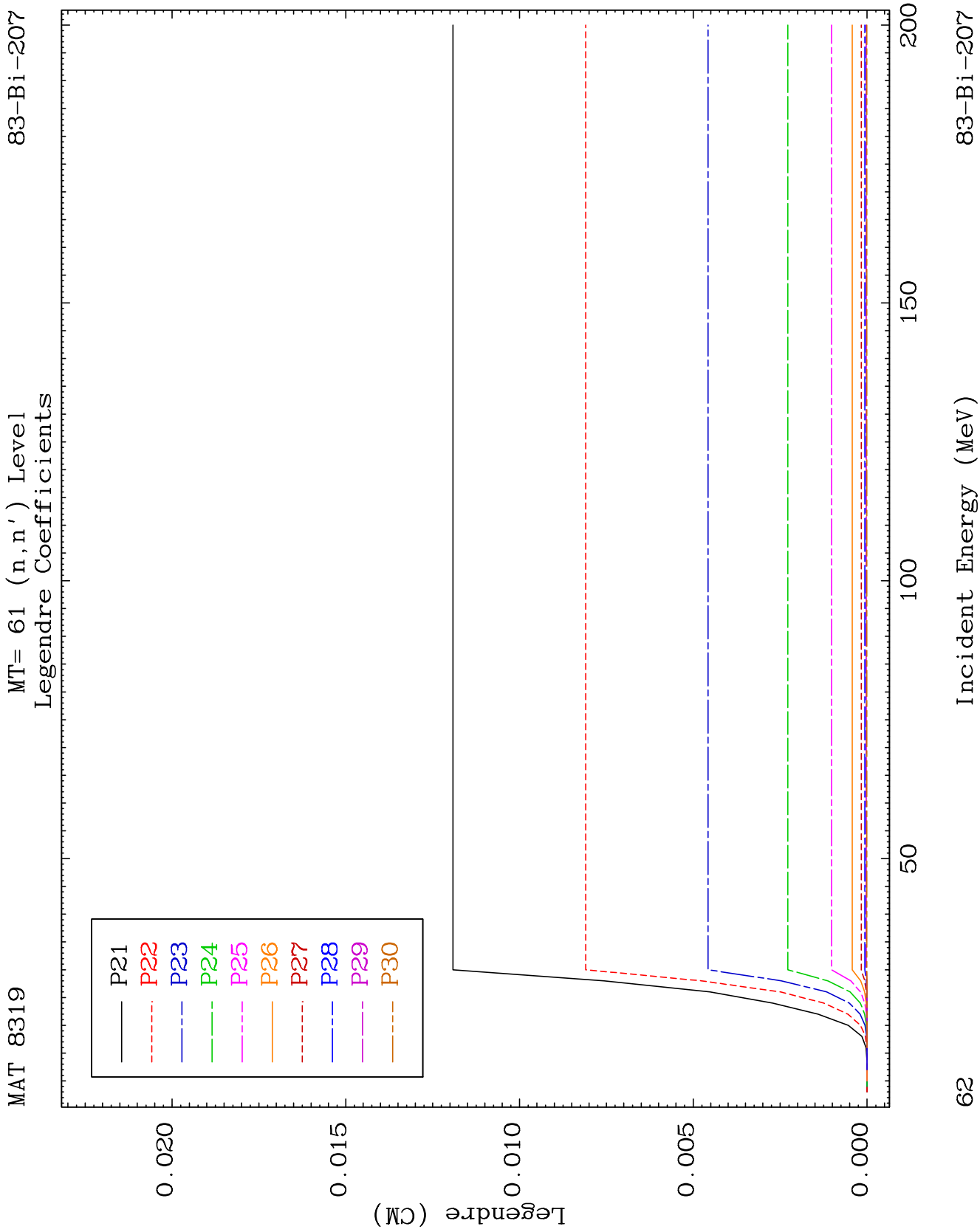
MAT 8319

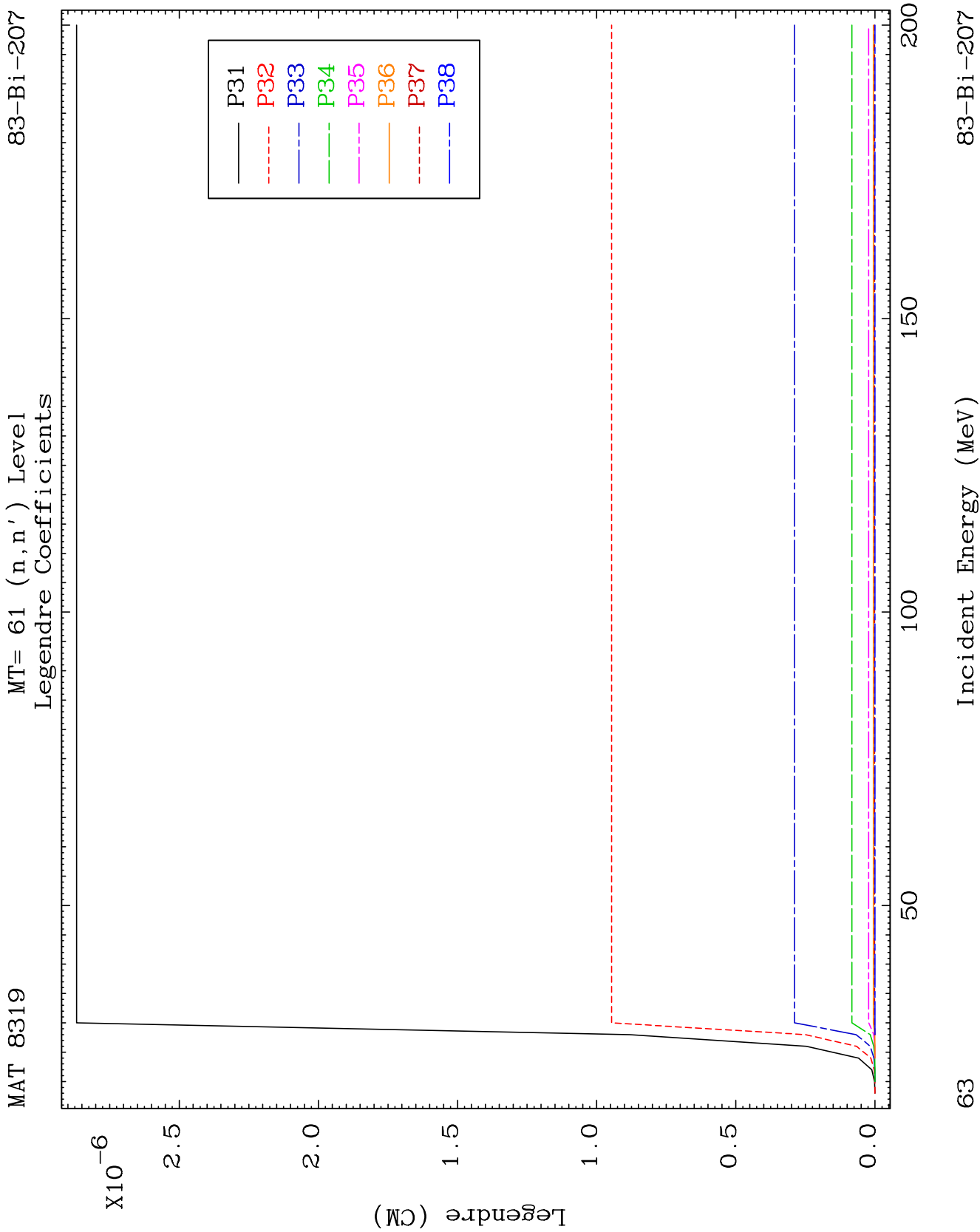
MT= 61 (n, n') Level
Legendre Coefficients

83-Bi-207





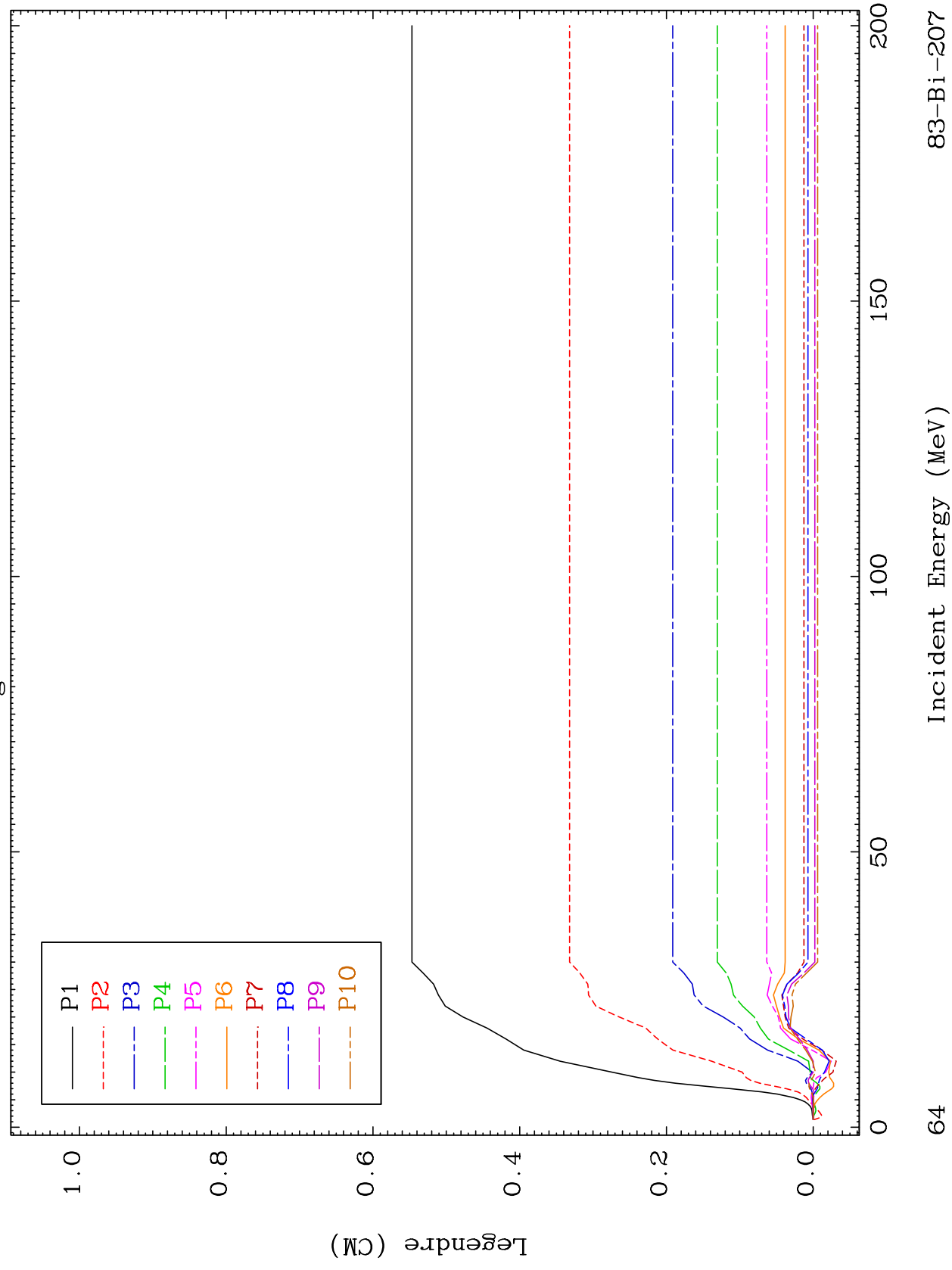


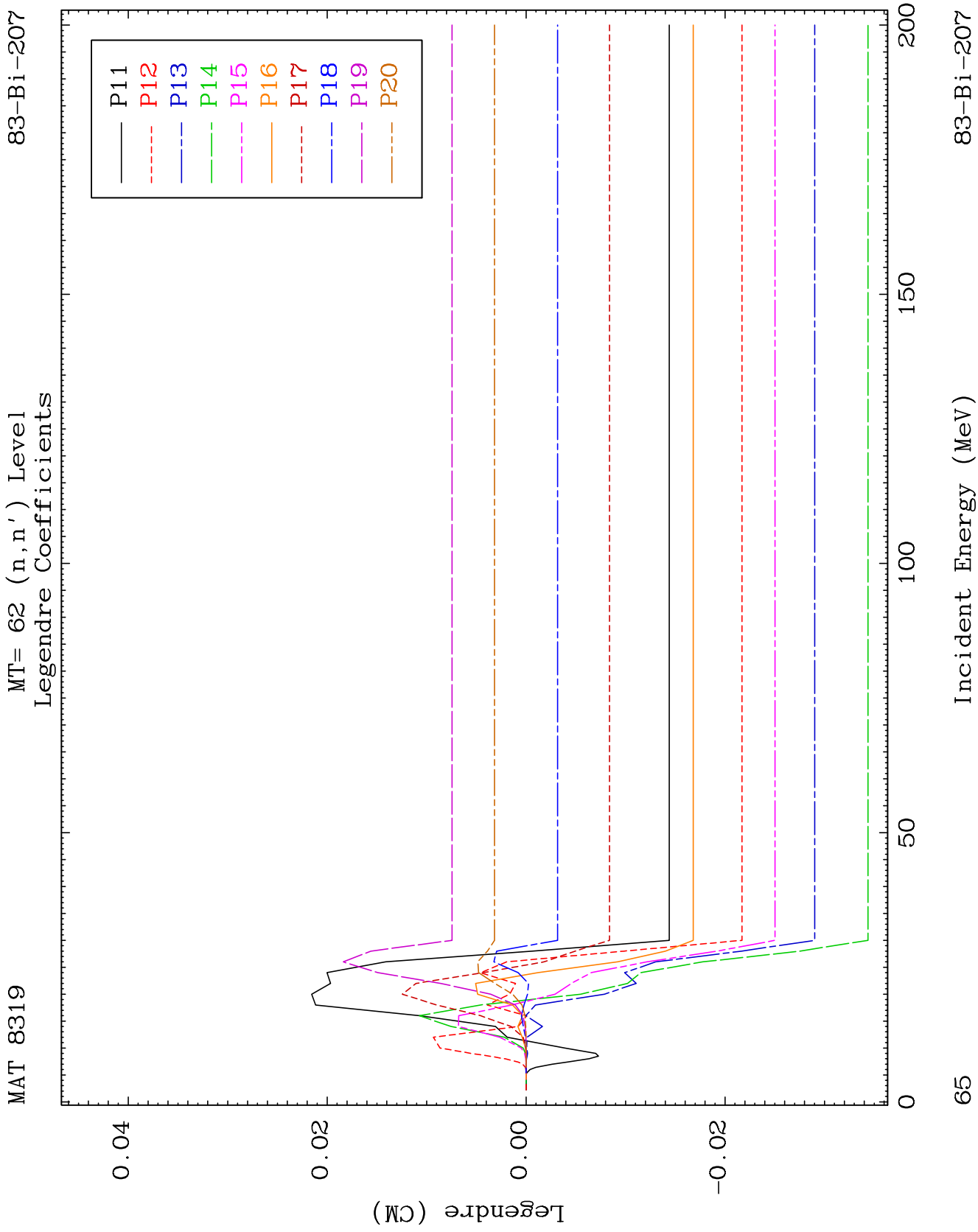


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MT= 62 (n, n') Level
Legendre Coefficients

83-Bi-207

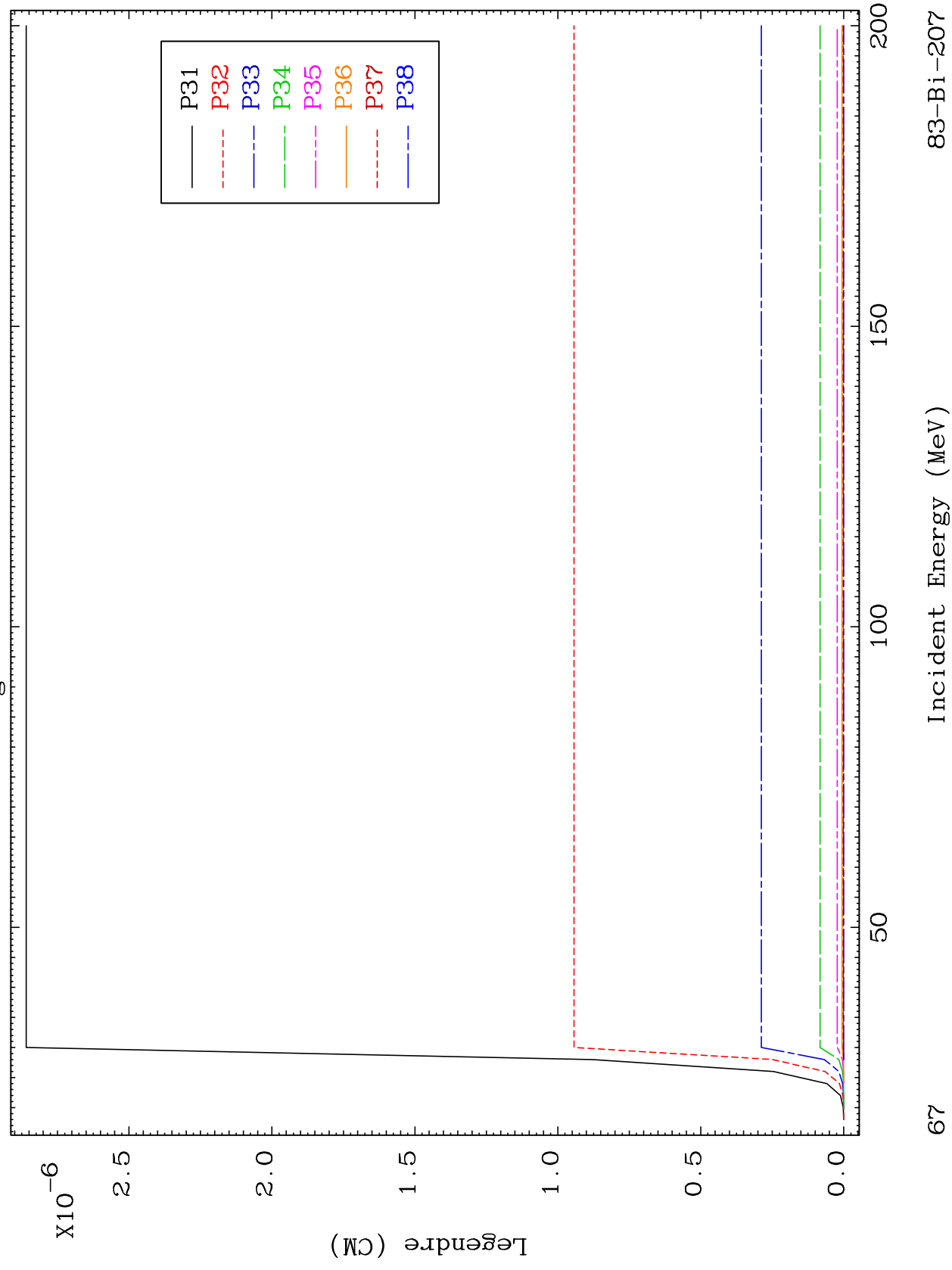




MAT 8319

MT= 62 (n, n') Level
Legendre Coefficients

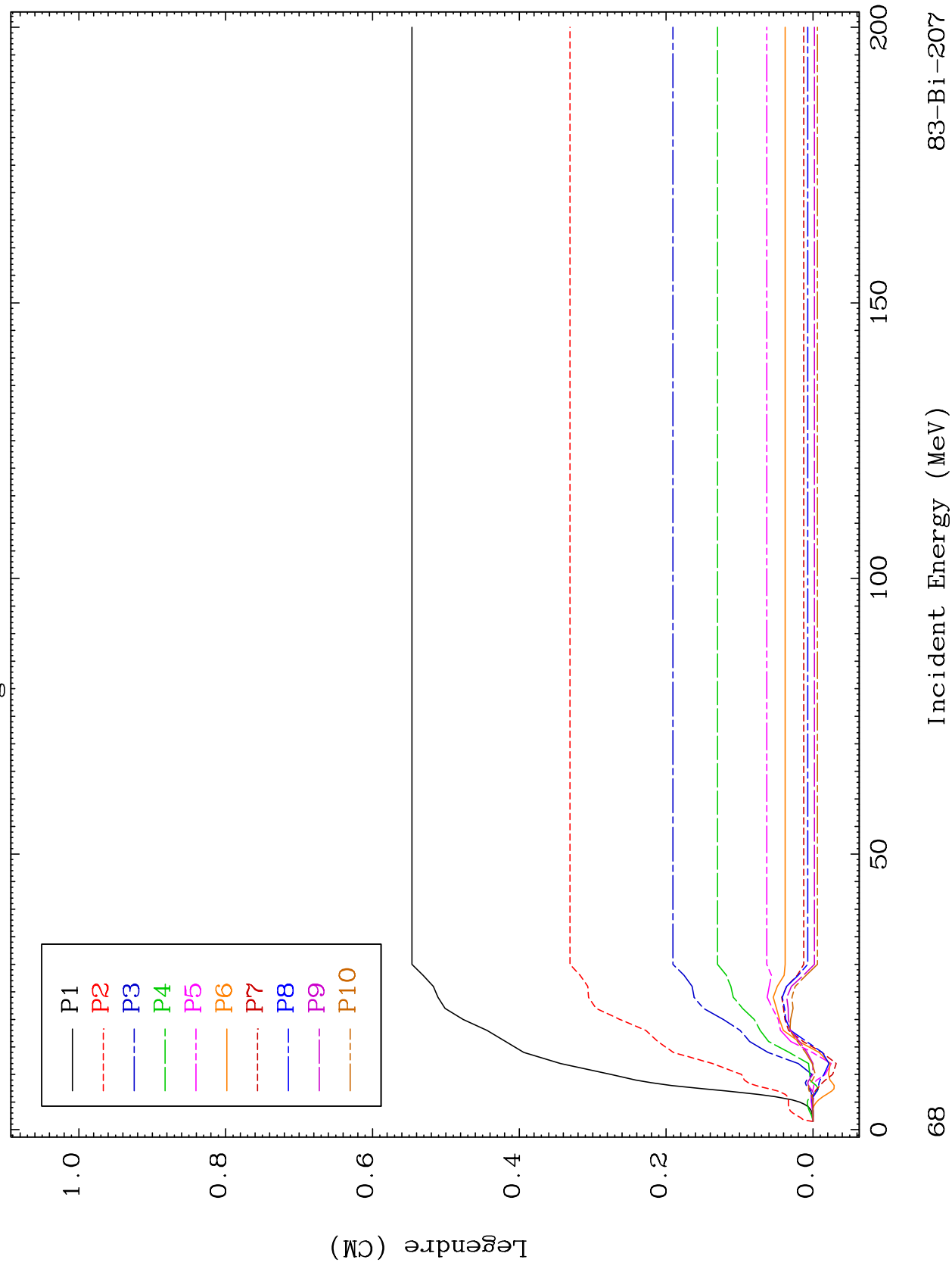
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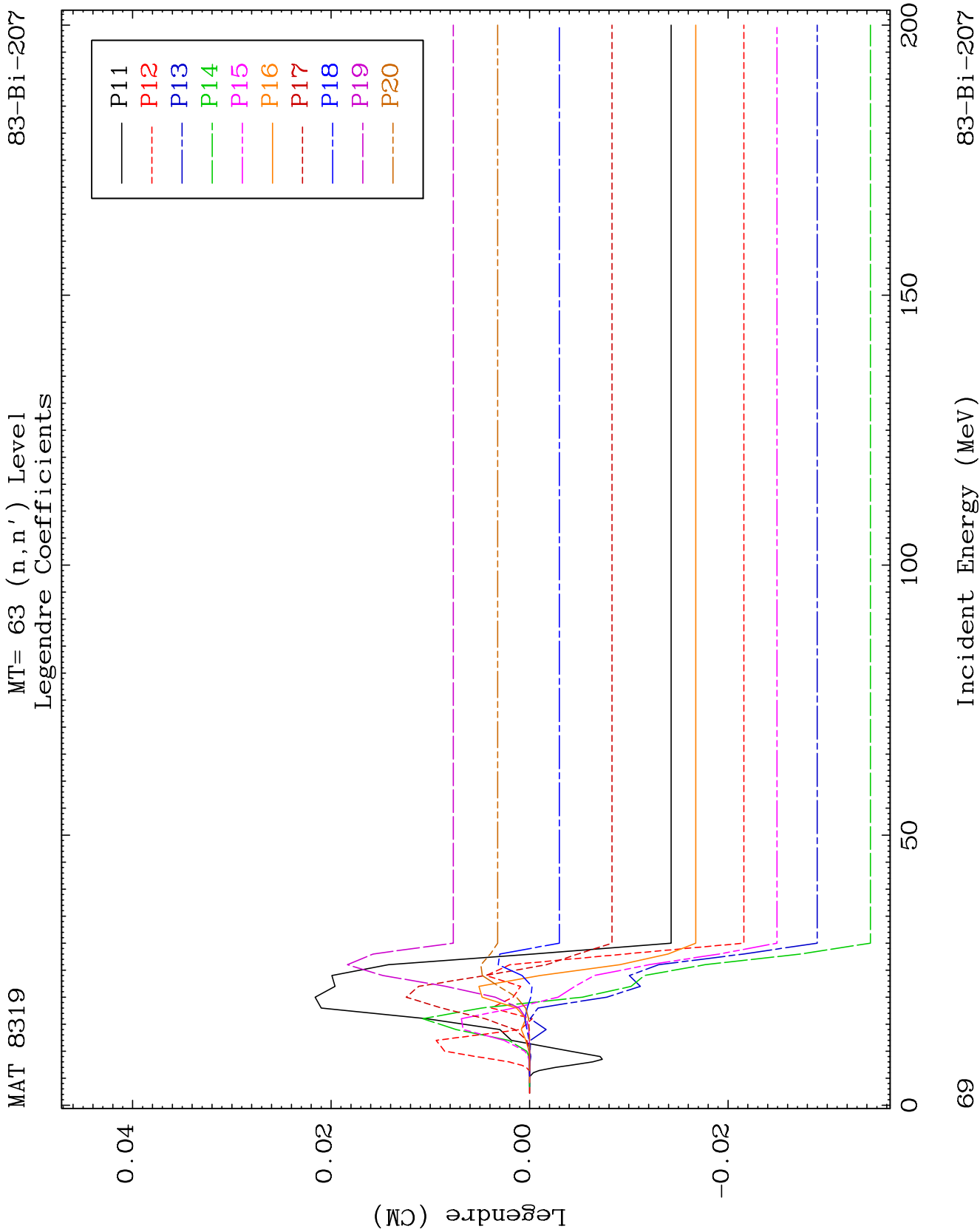


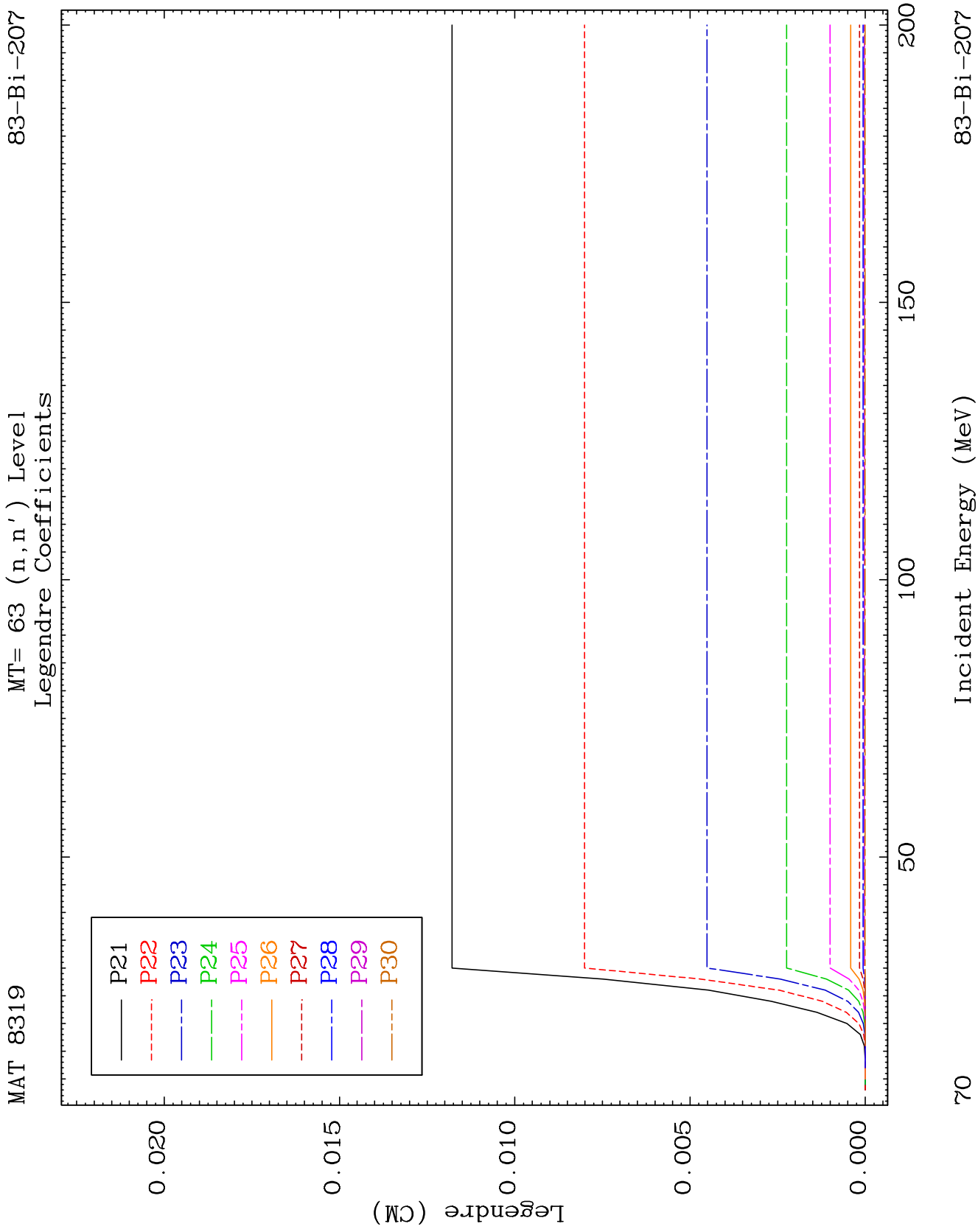
MAT 8319

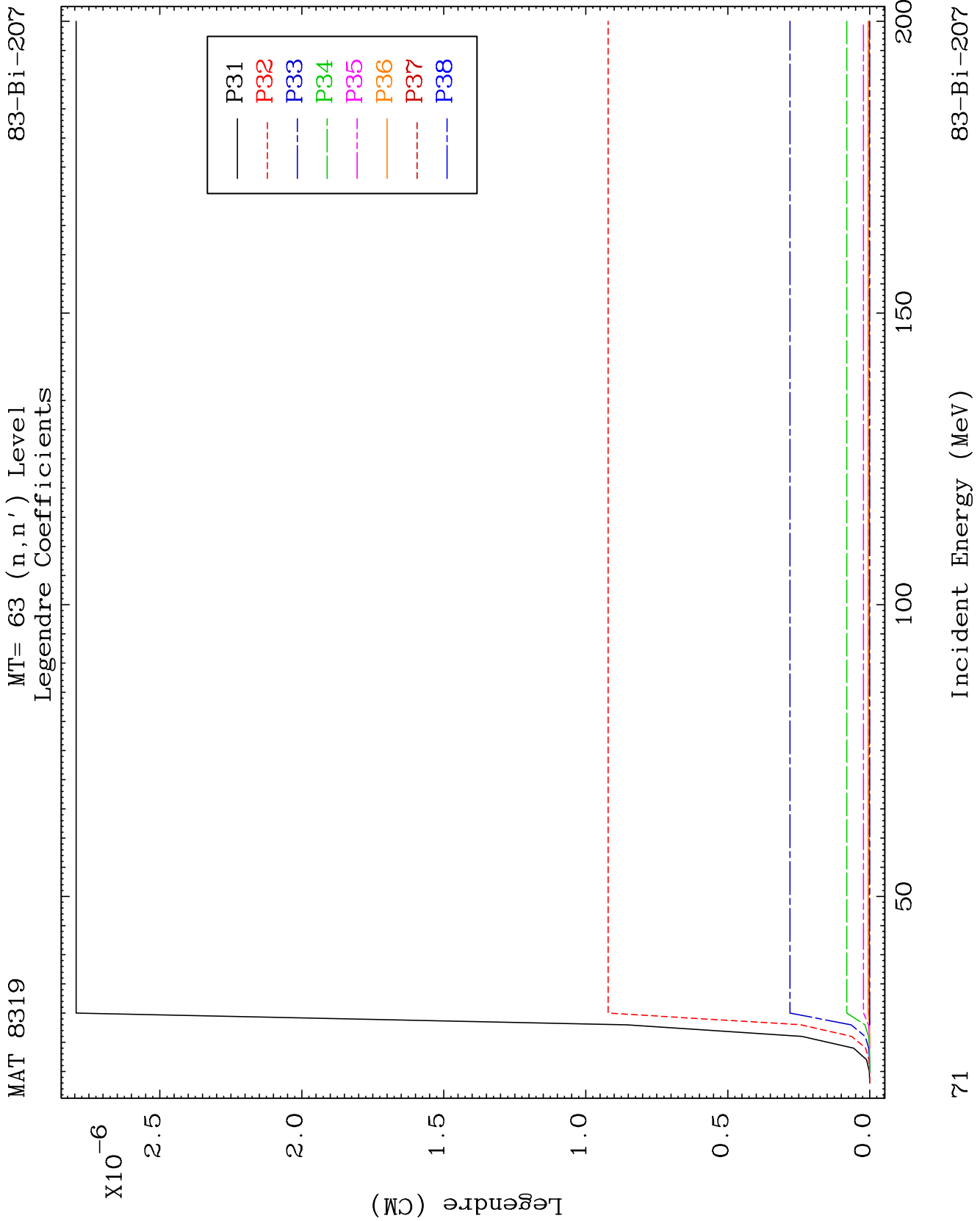
MT= 63 (n, n') Level
Legendre Coefficients

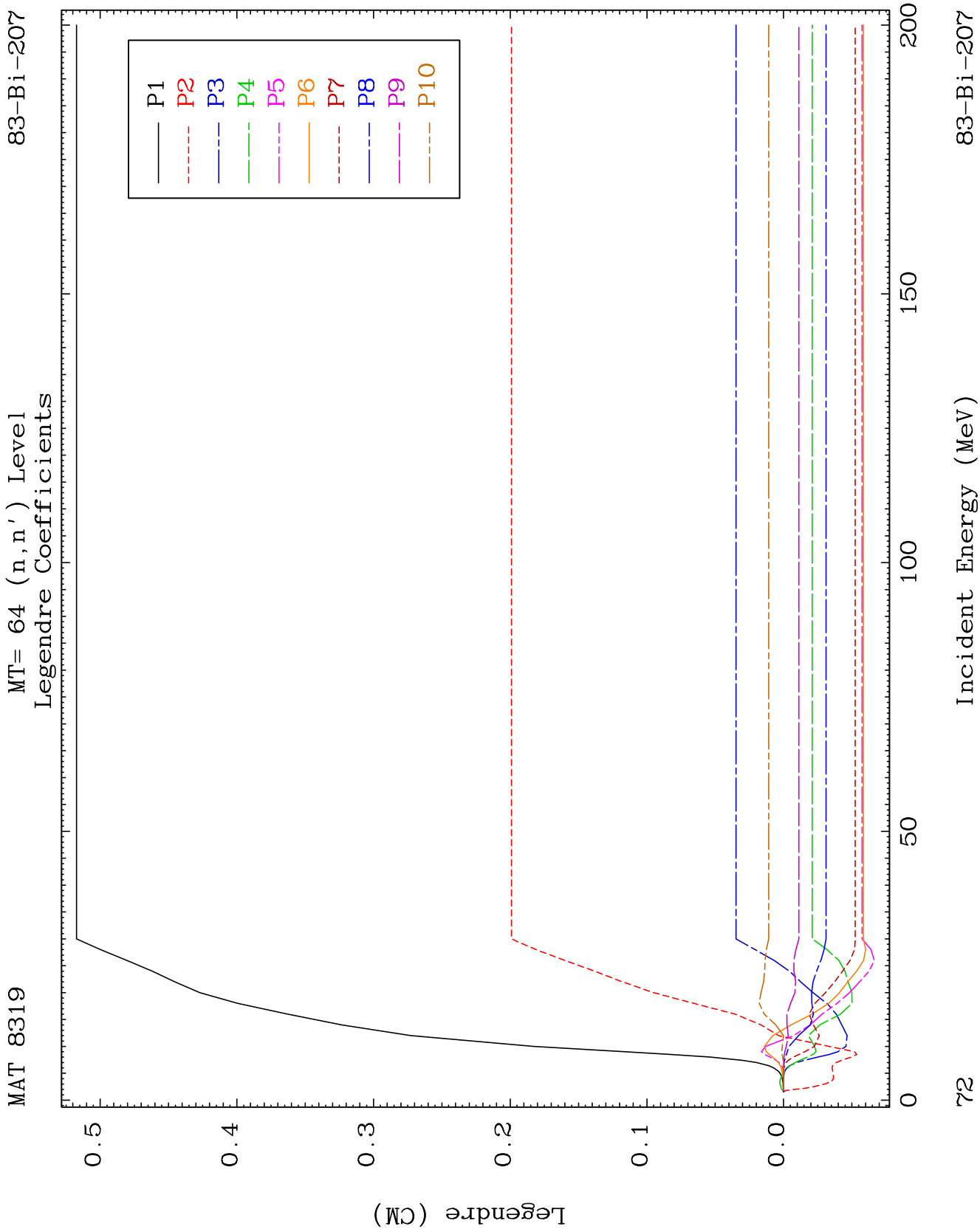
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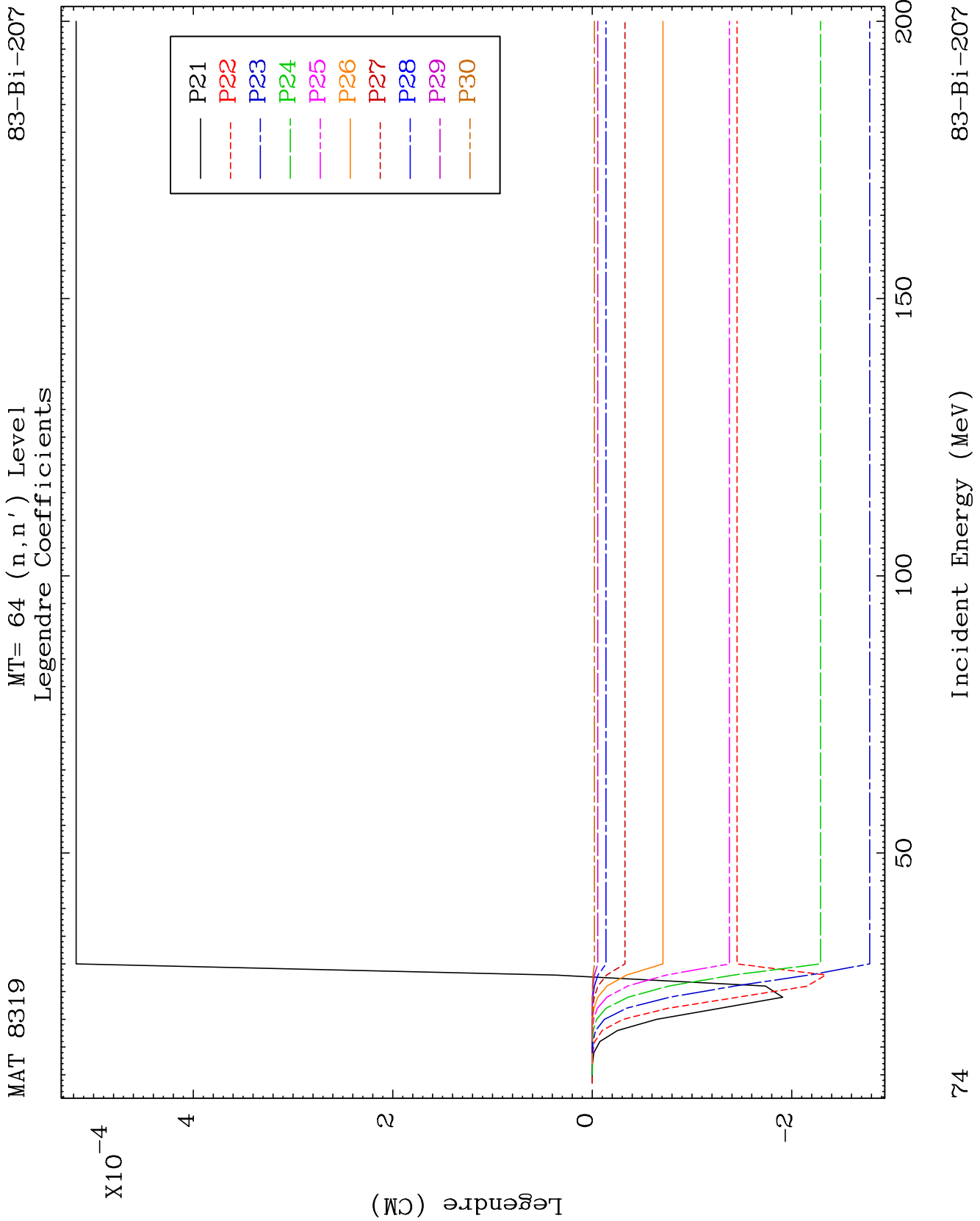


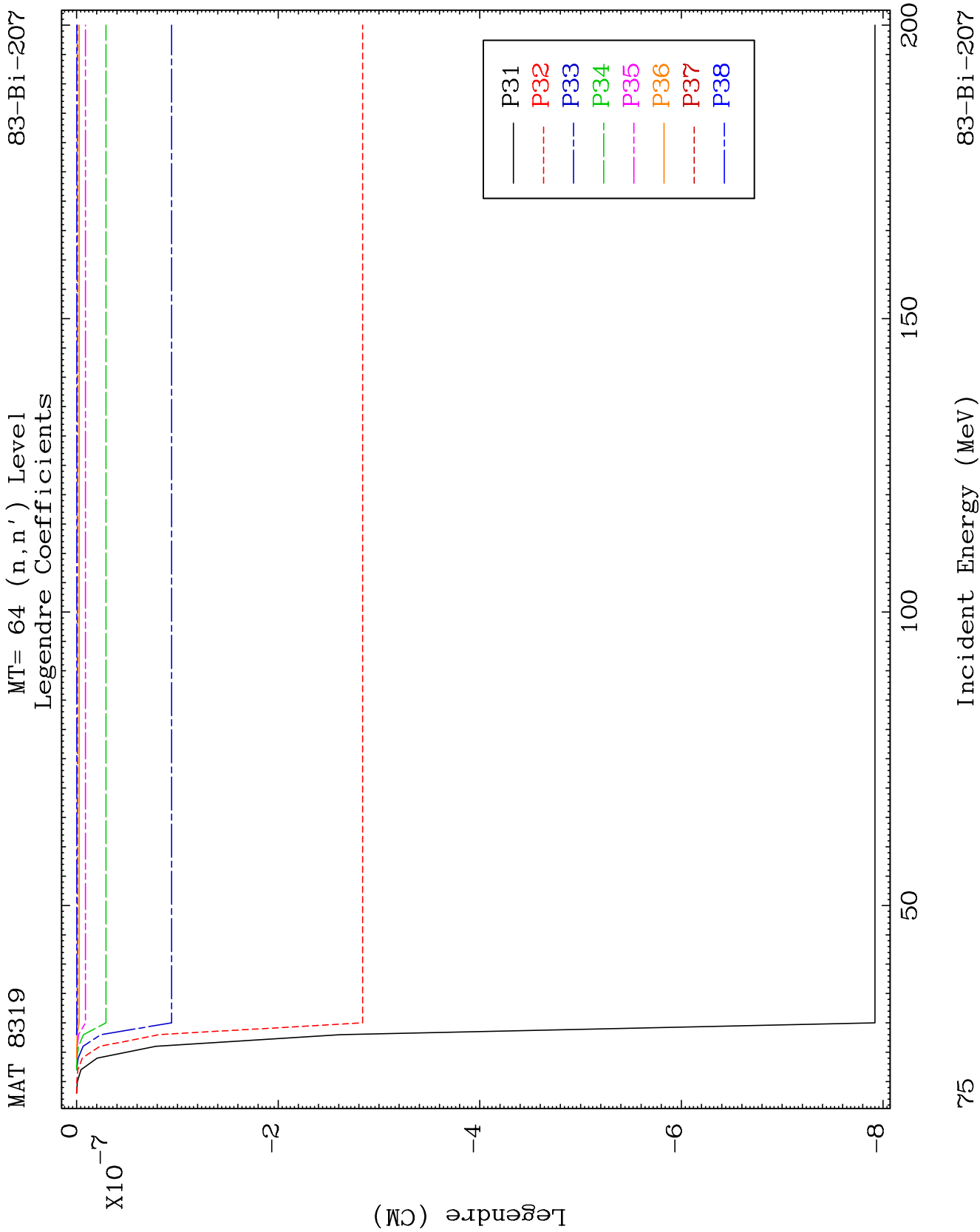








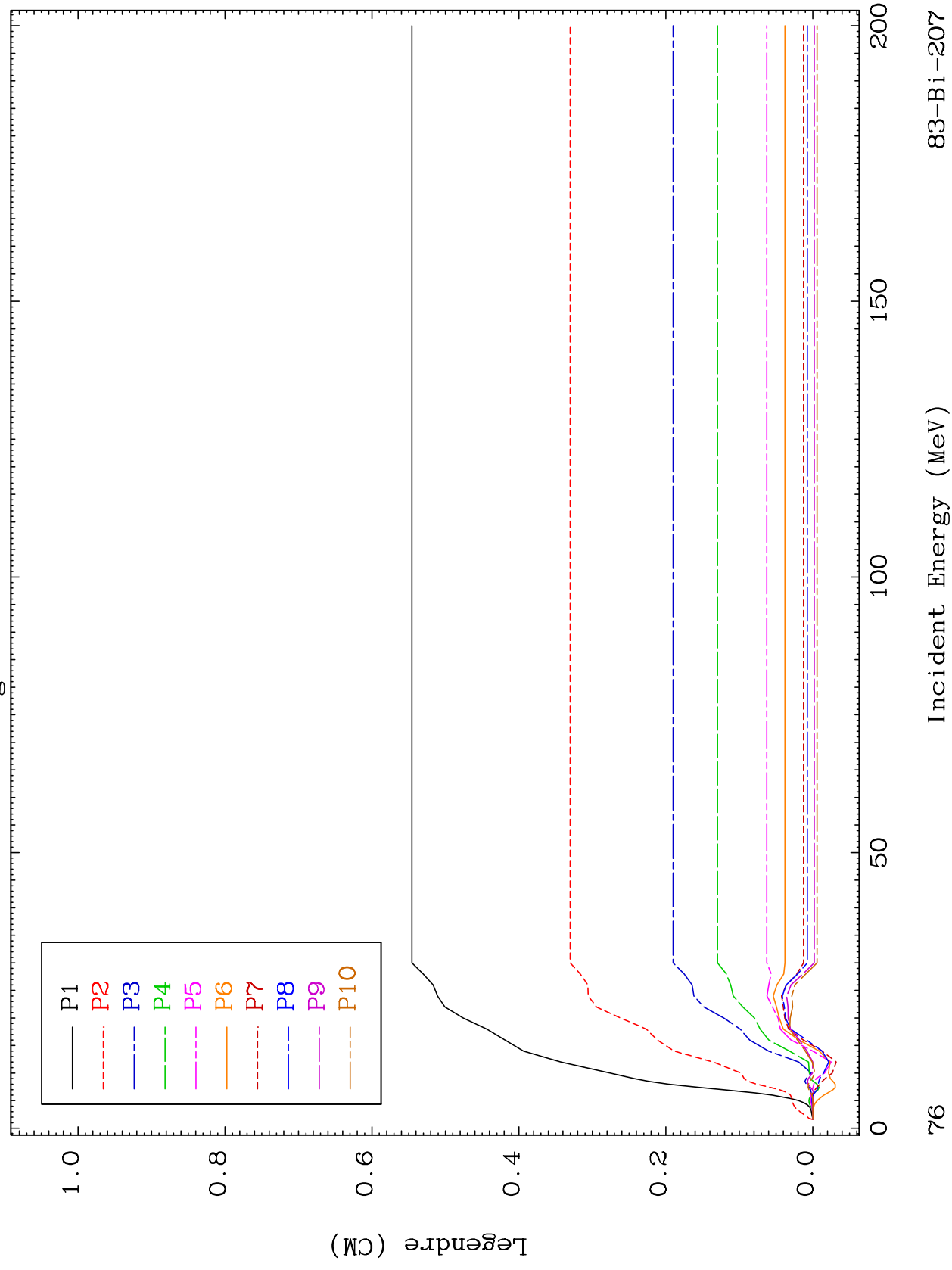


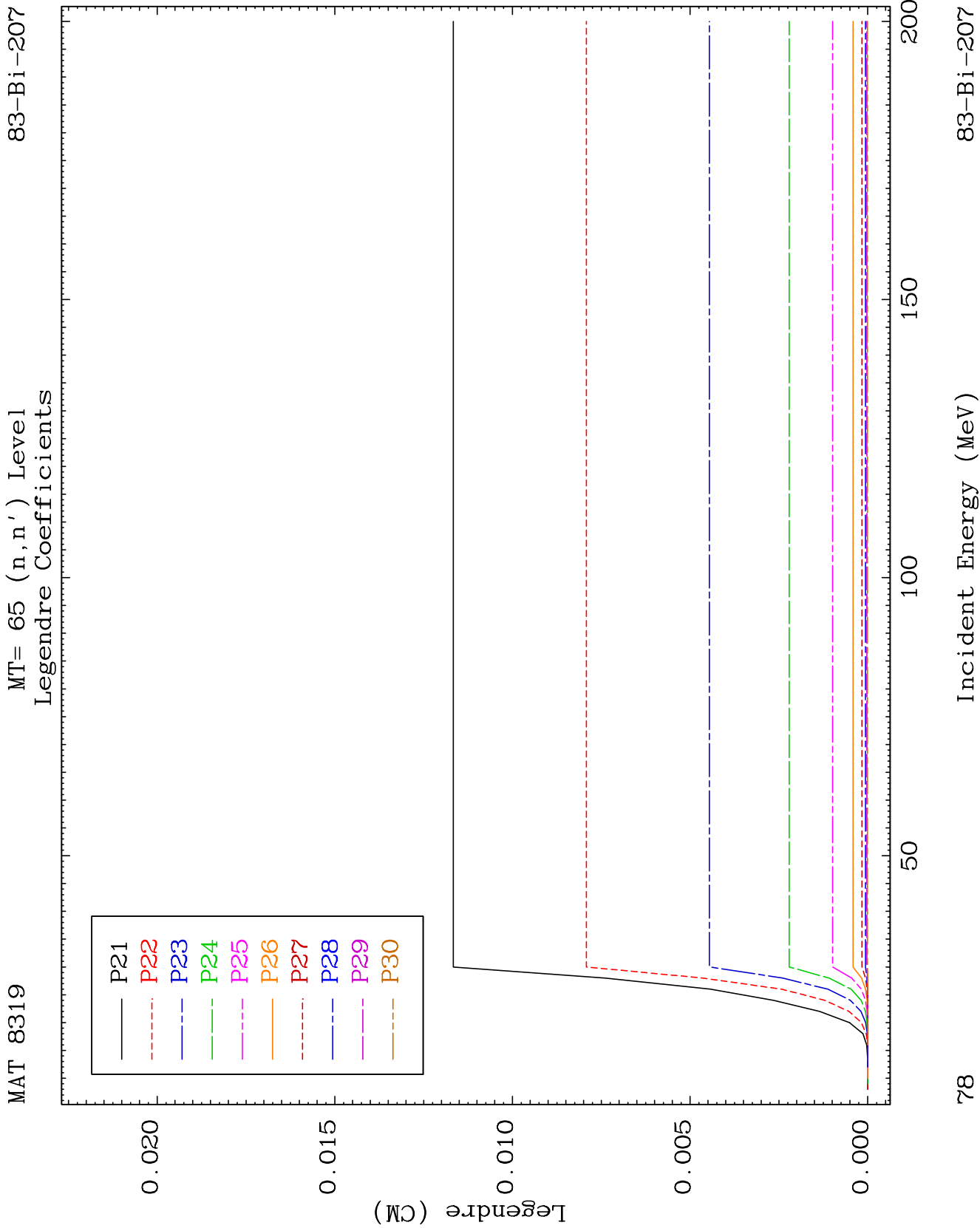


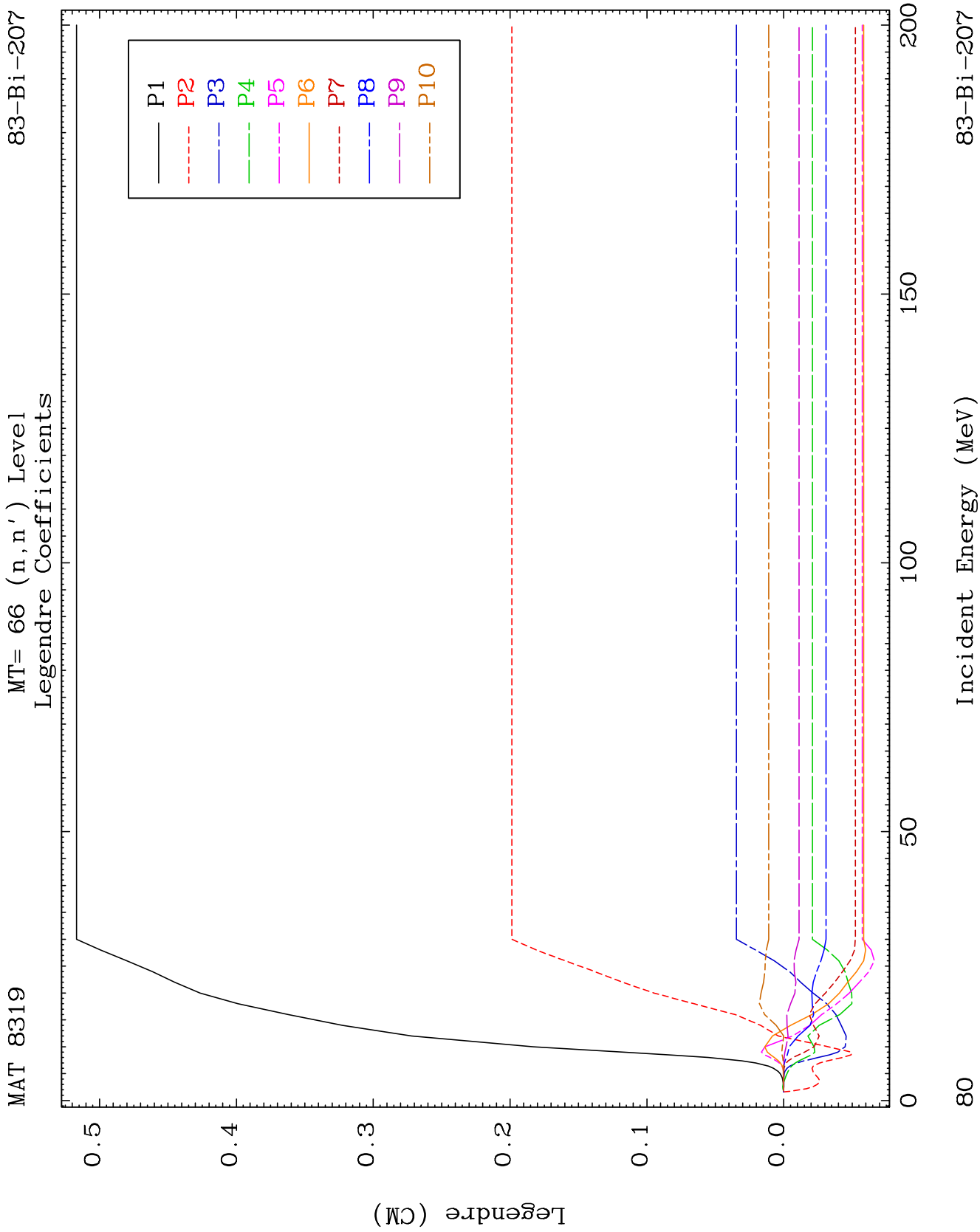
MAT 8319

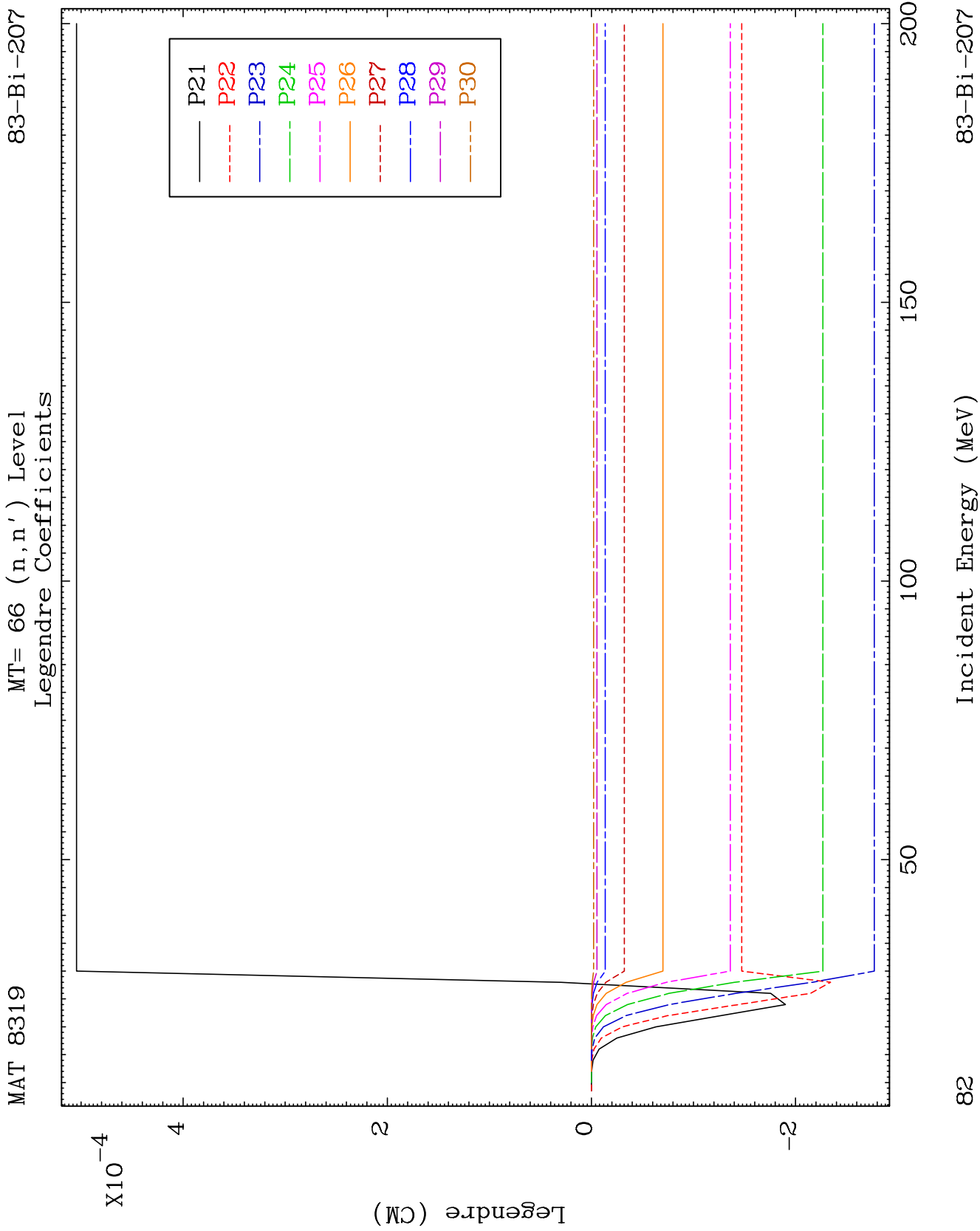
MT= 65 (n, n') Level
Legendre Coefficients

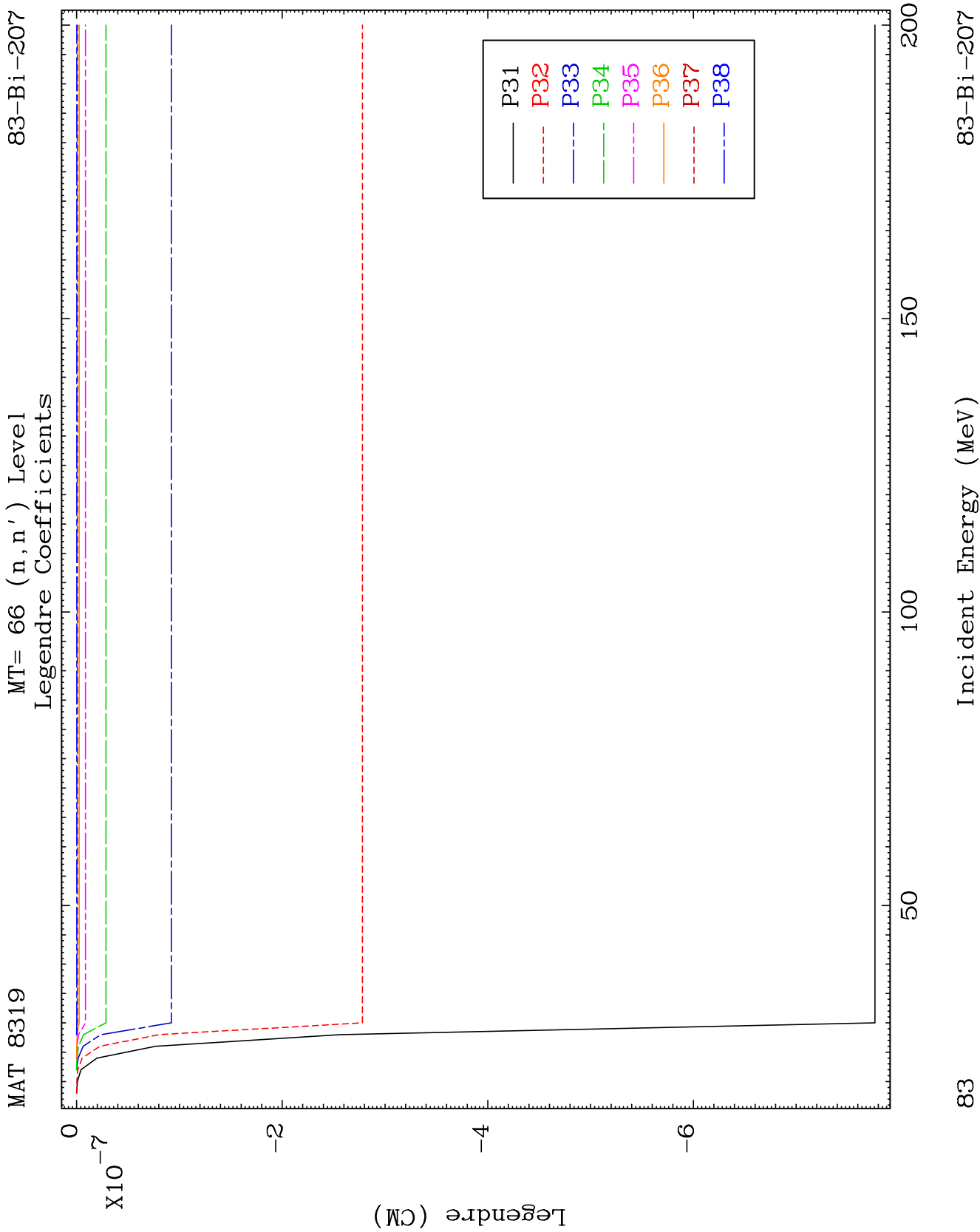
83-Bi-207

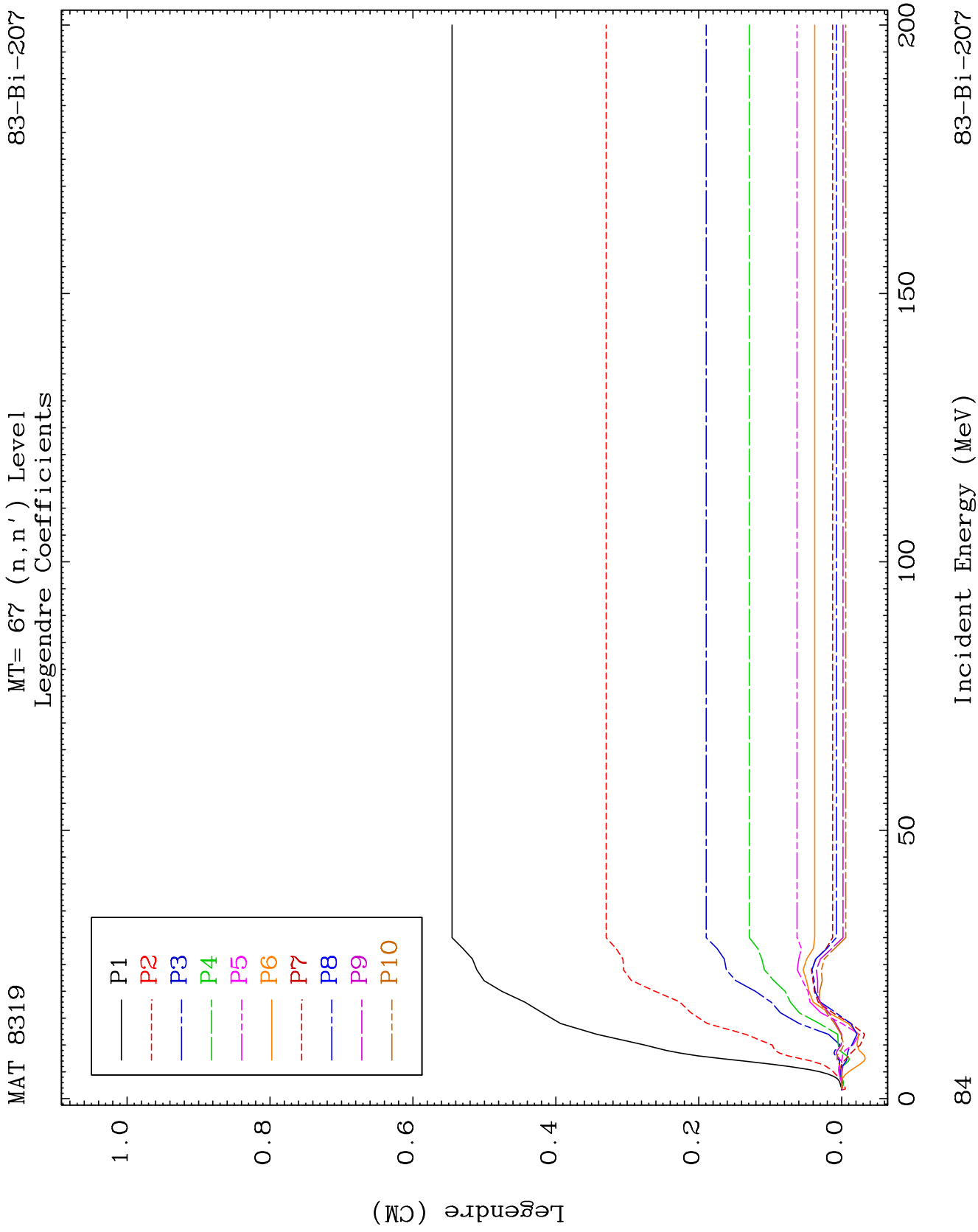


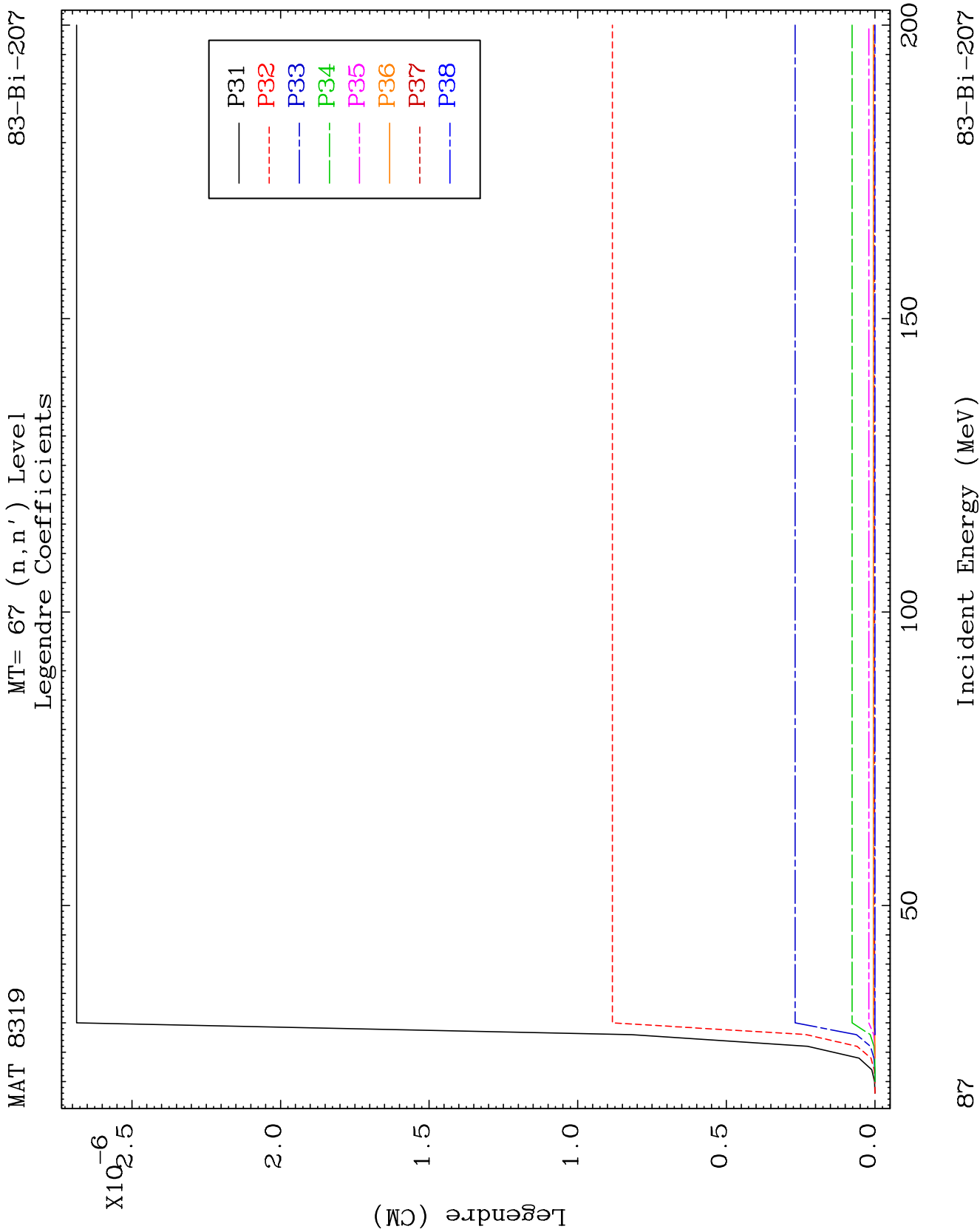


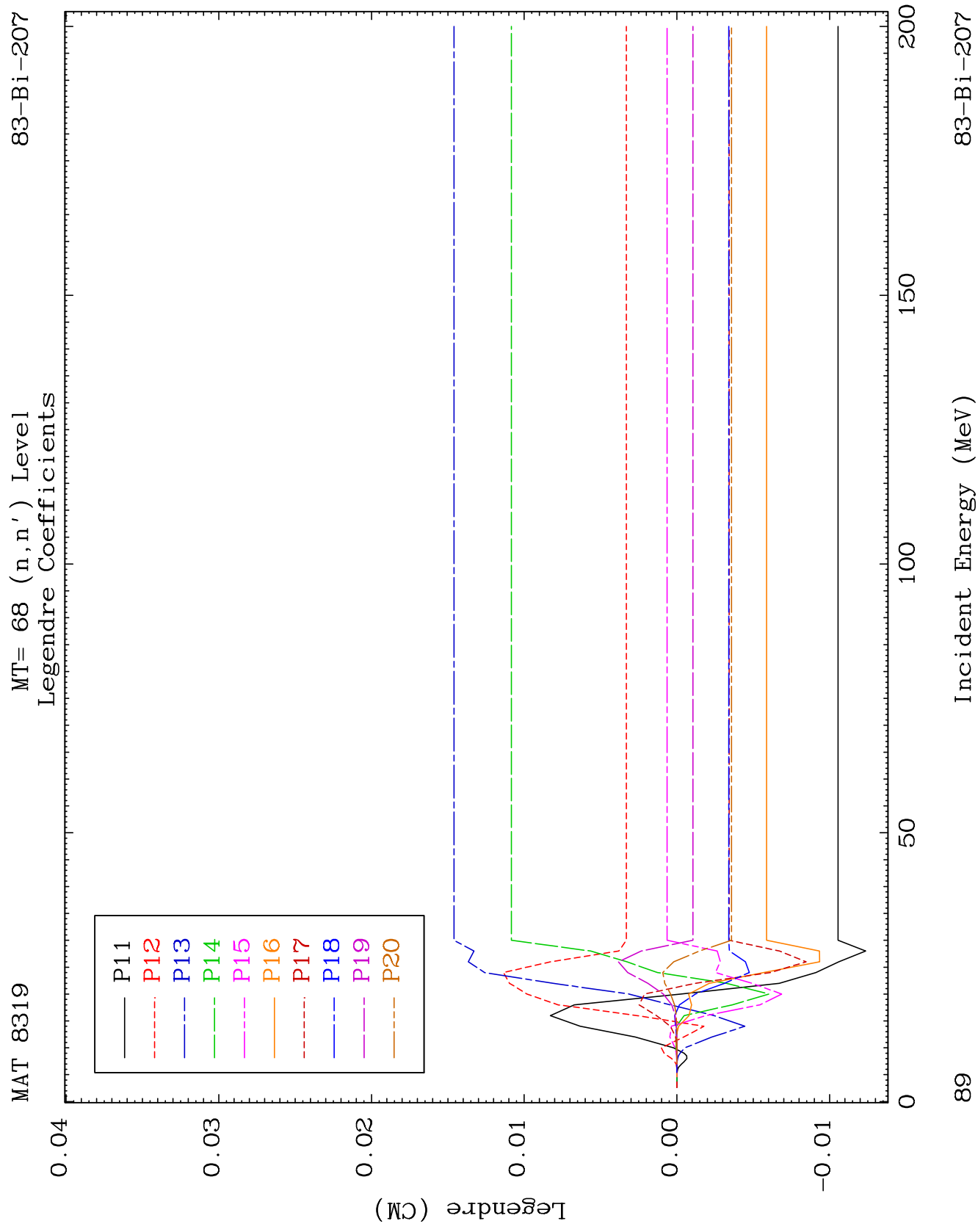


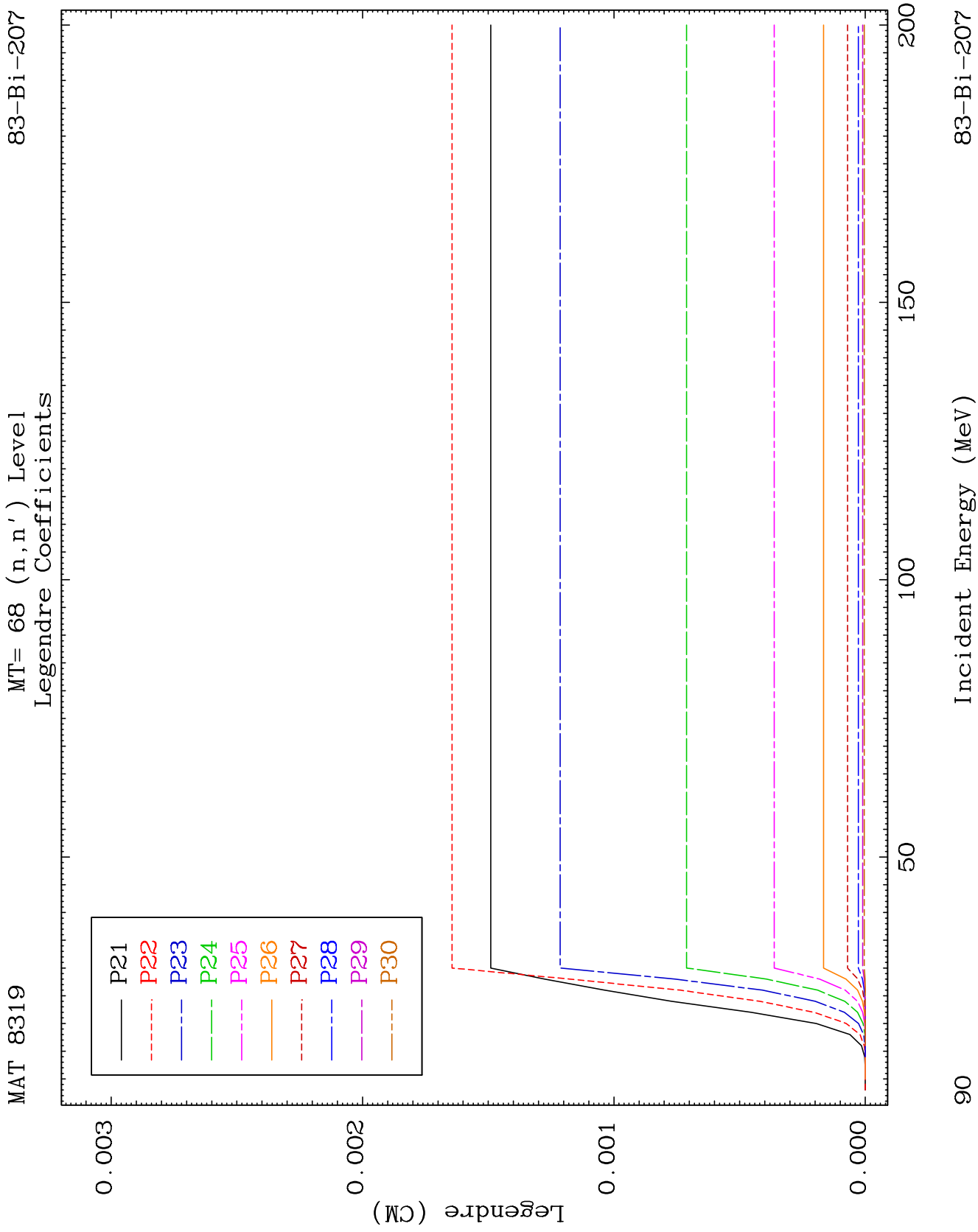


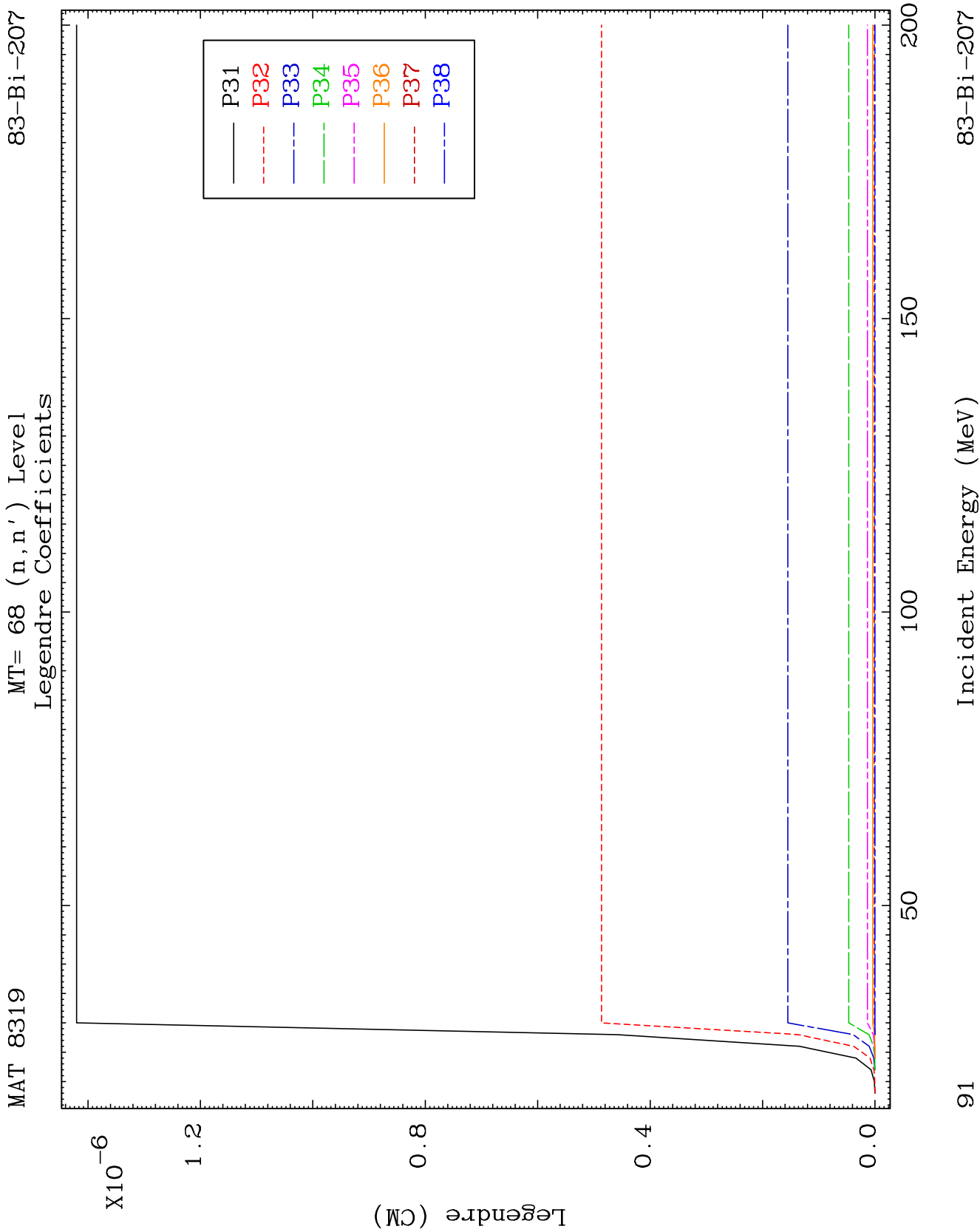


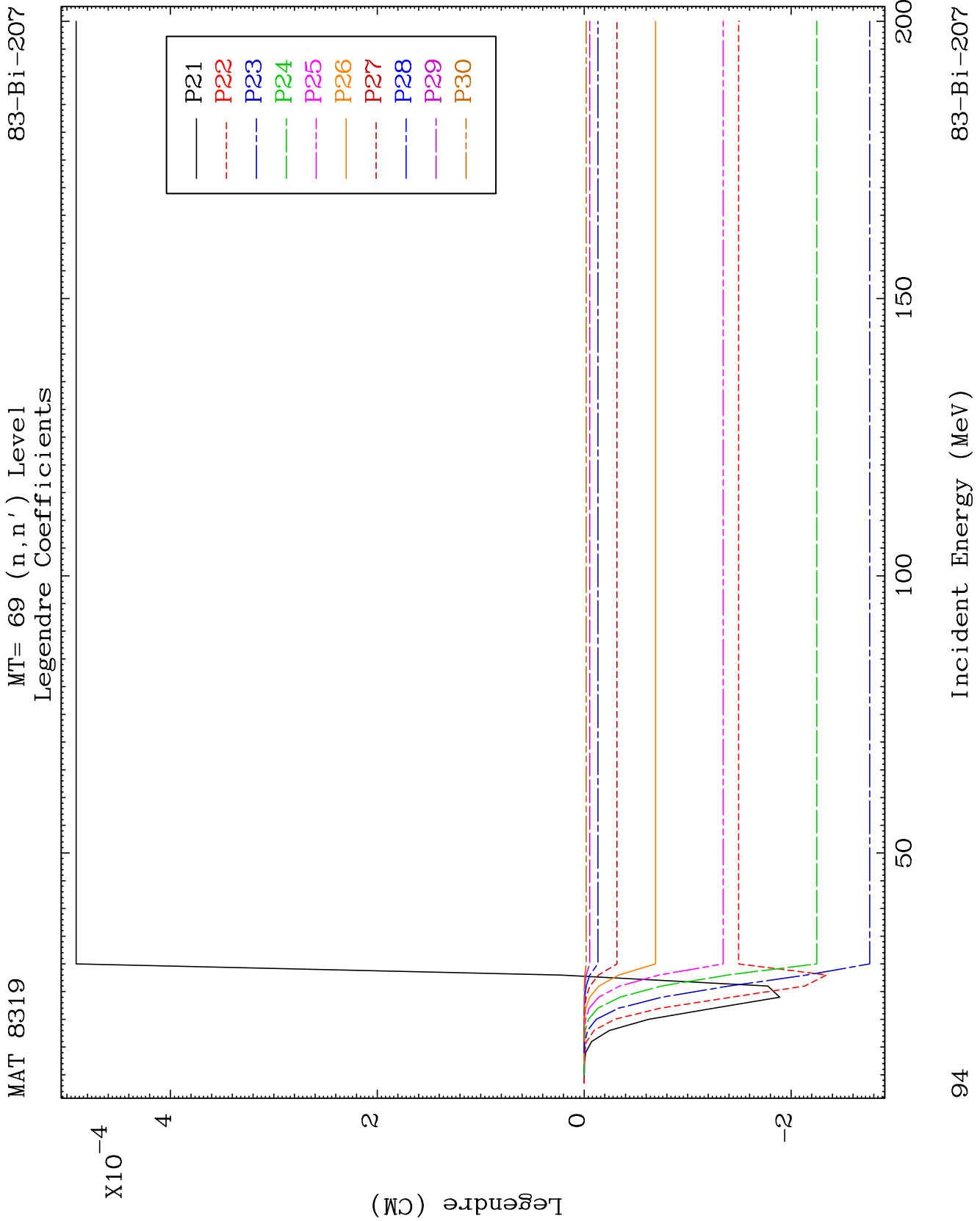


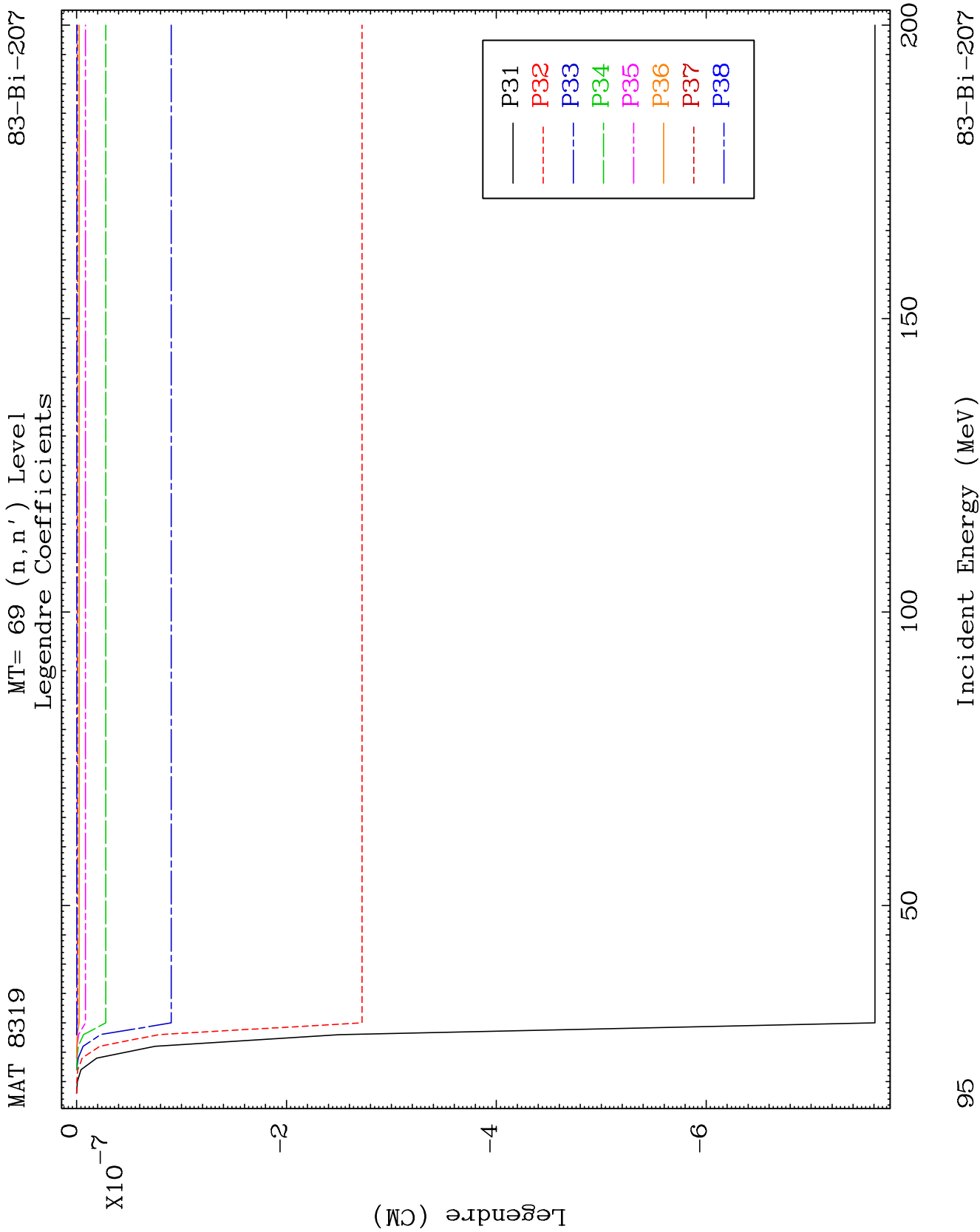


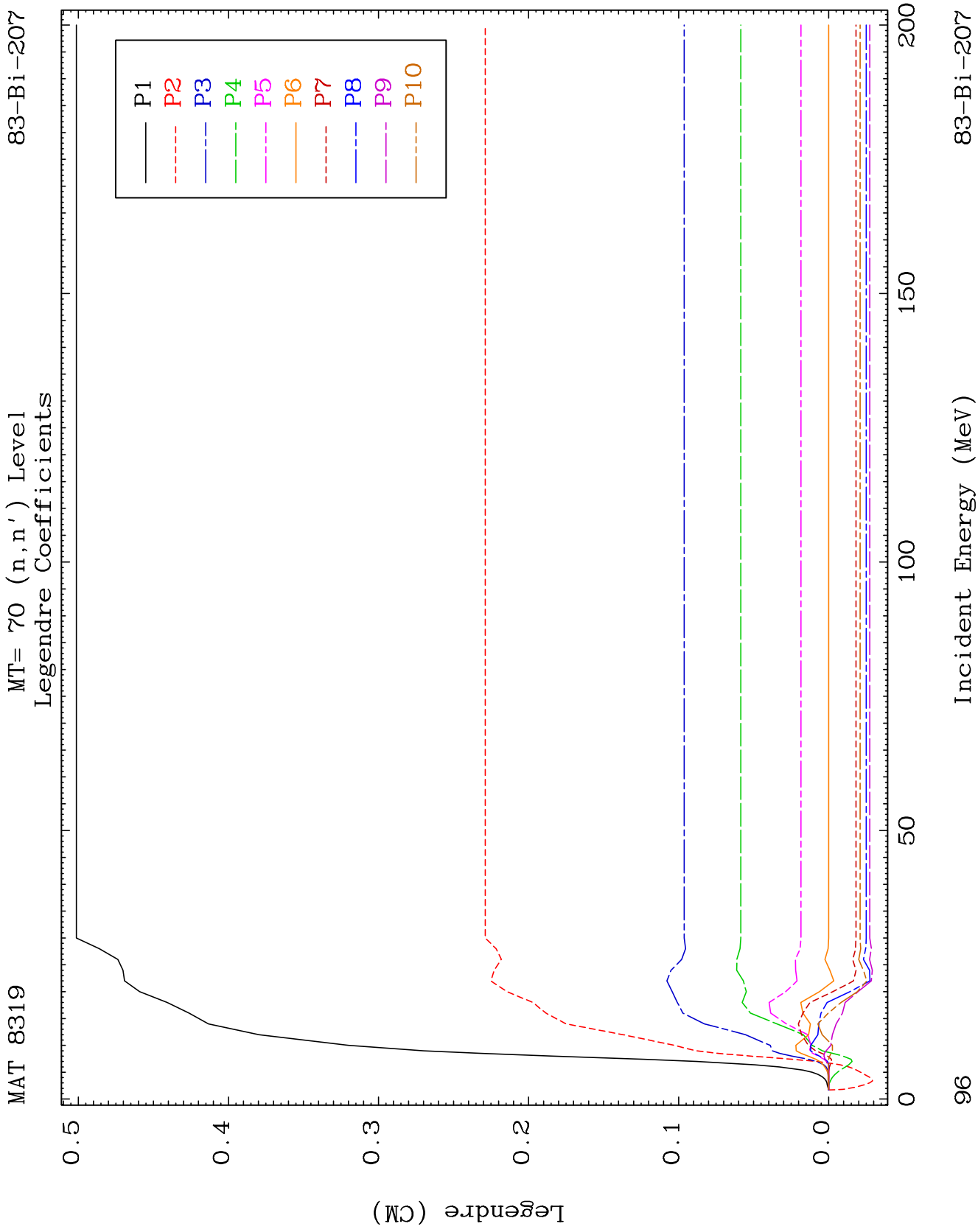


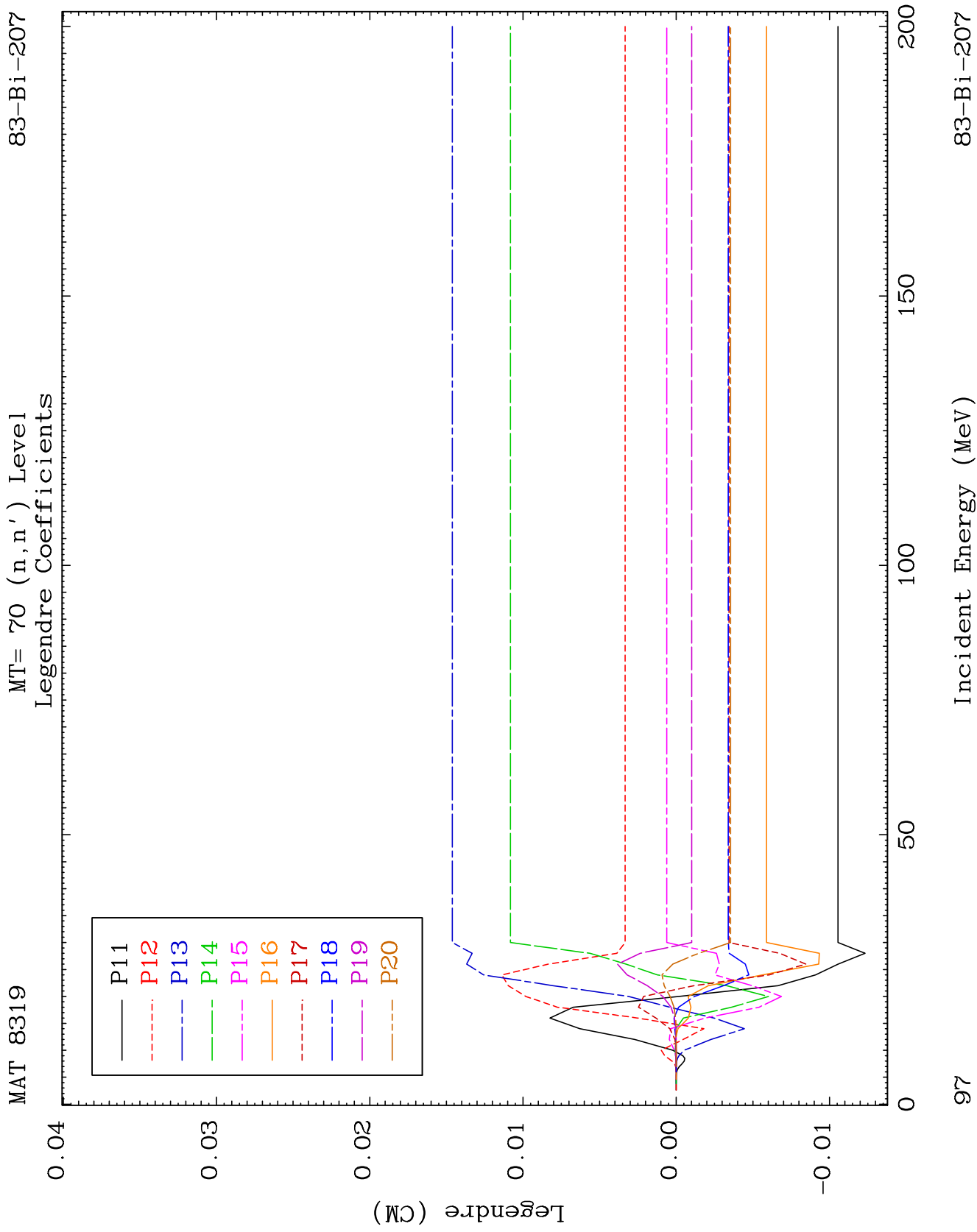


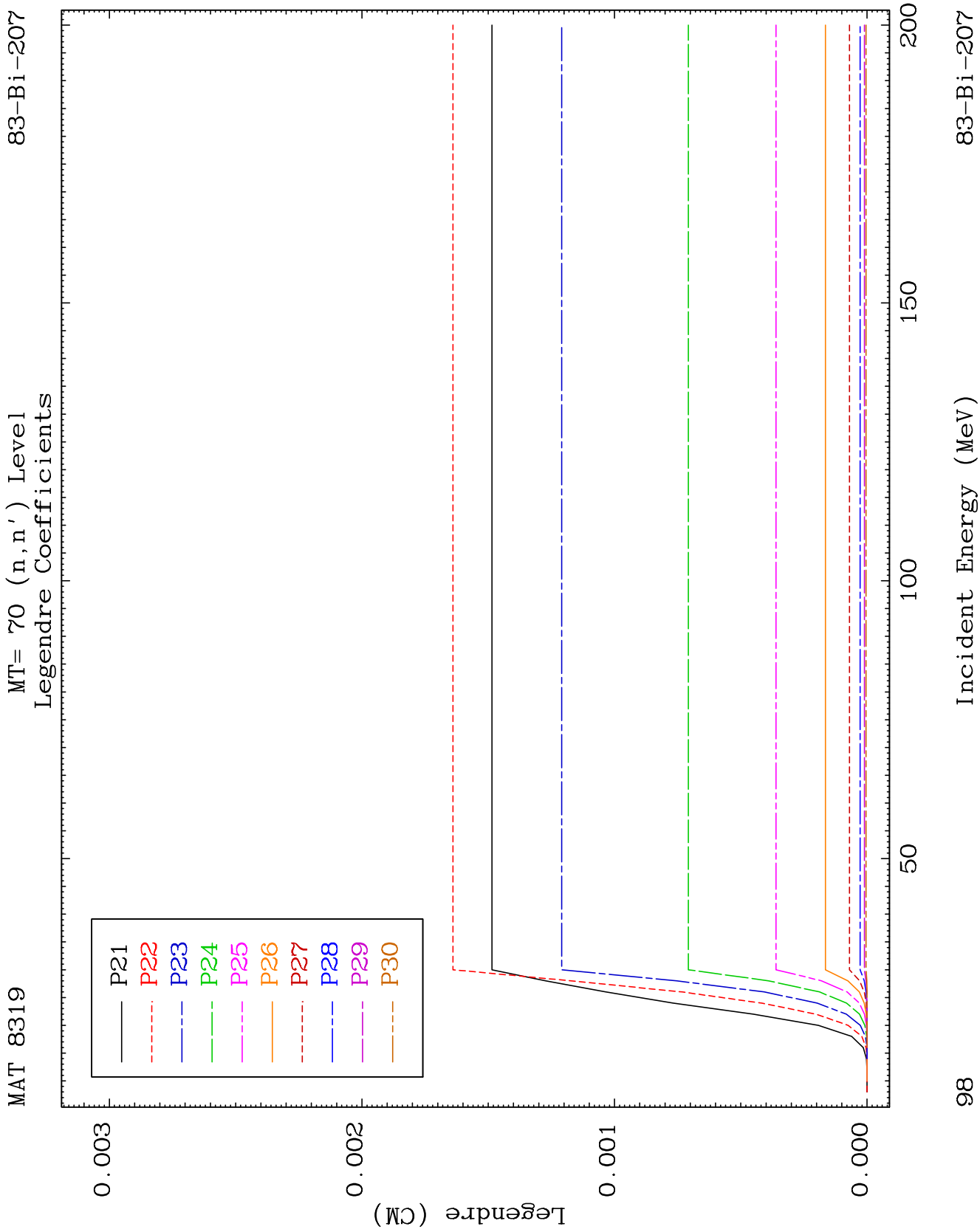








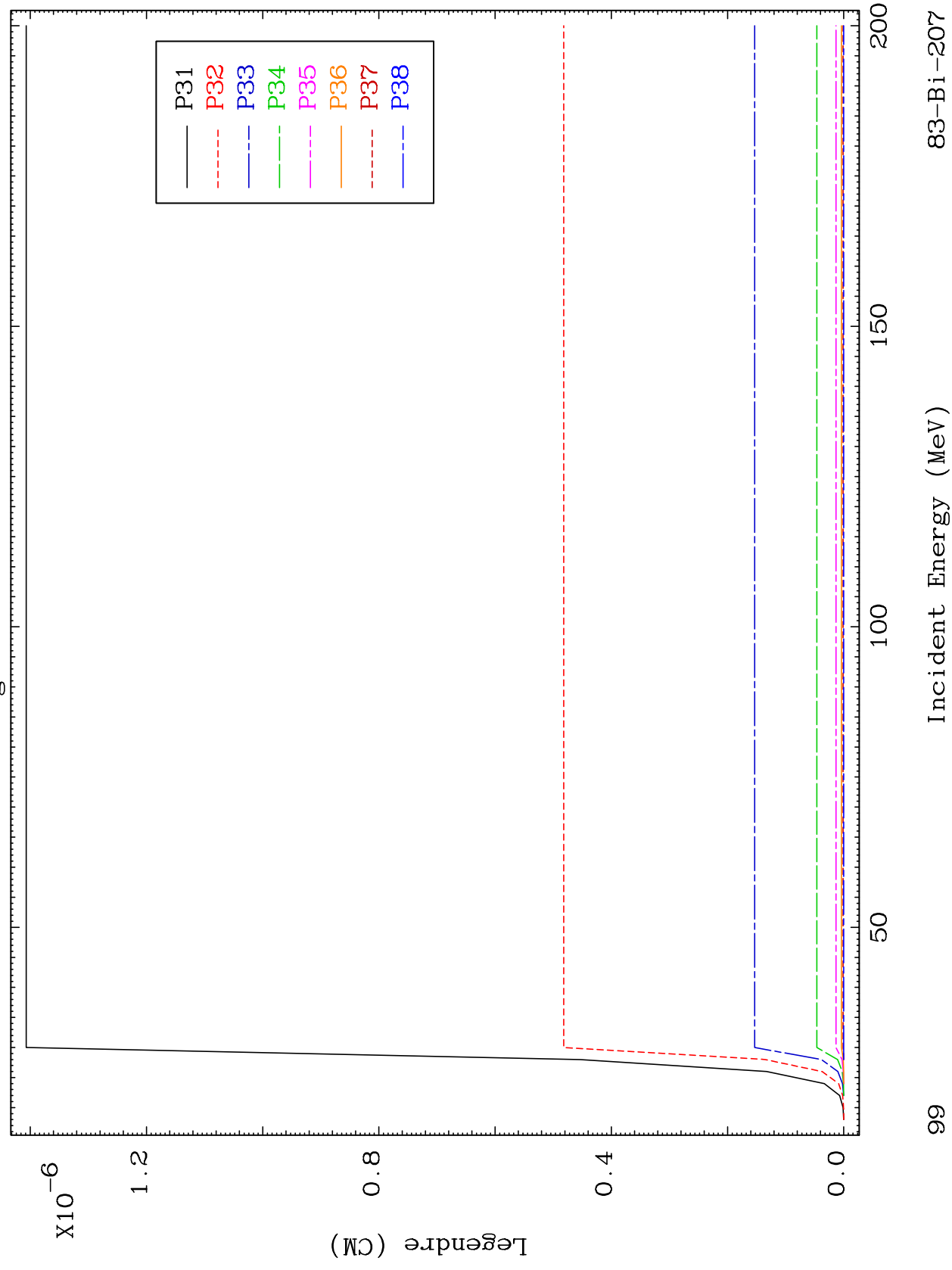


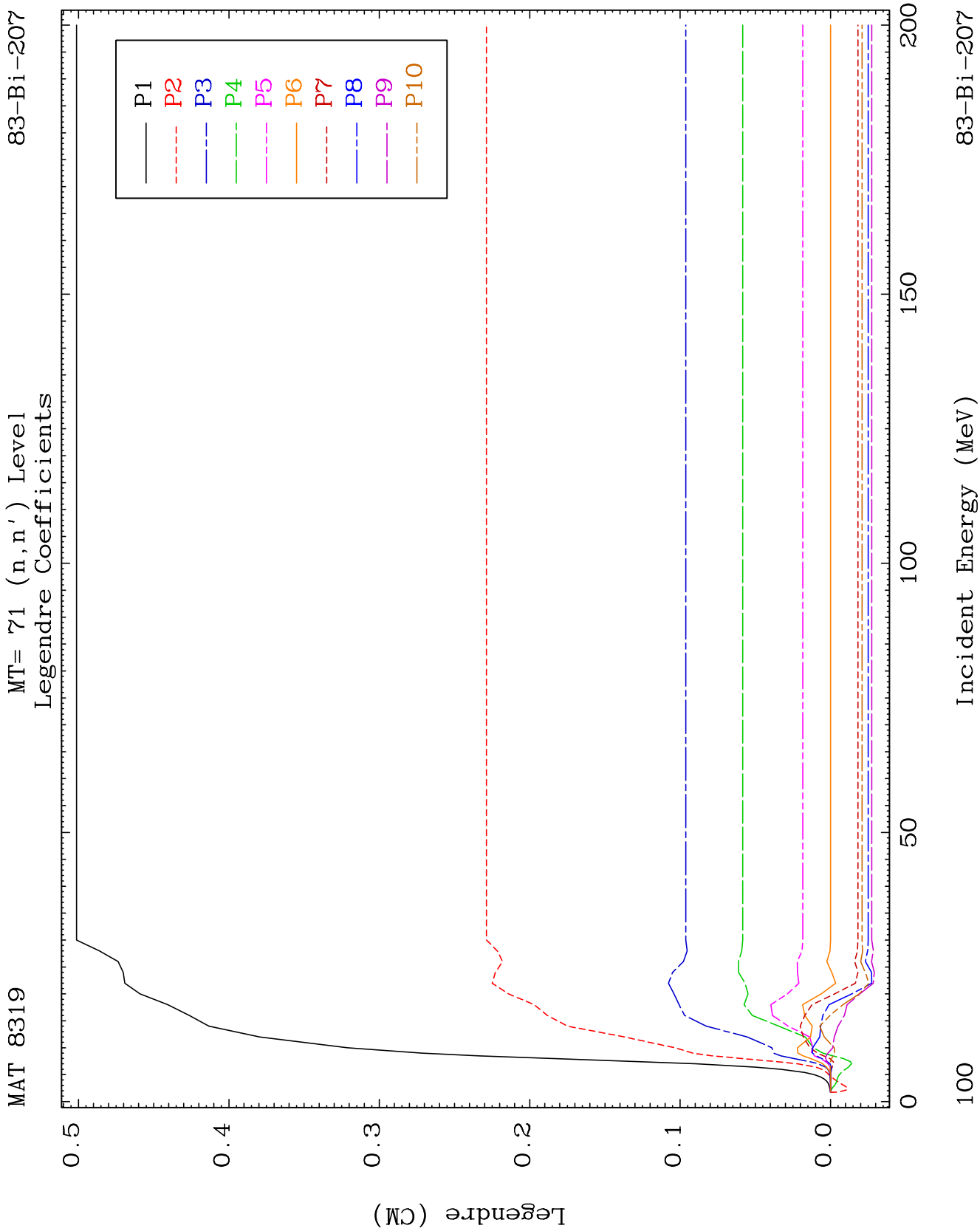


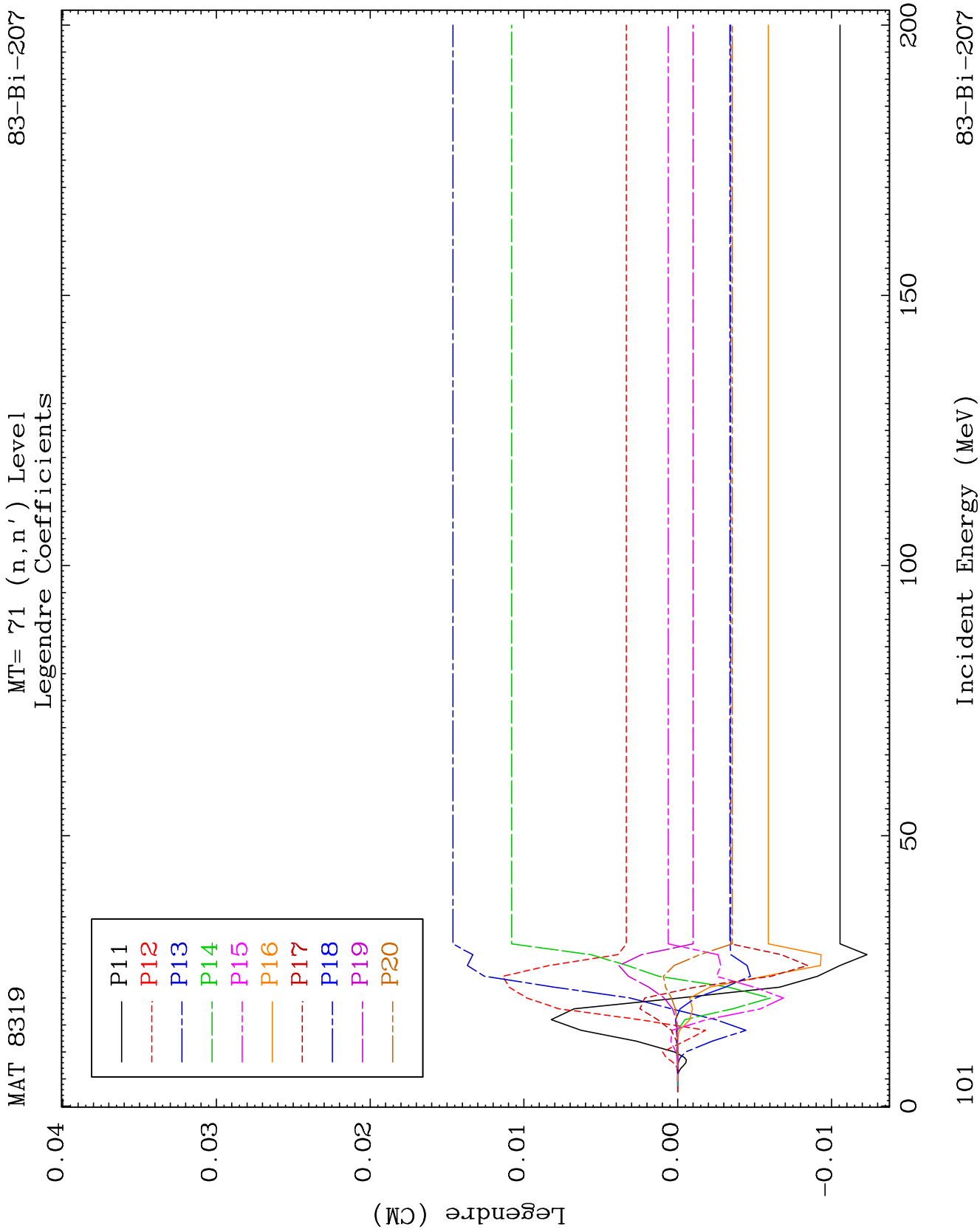
MAT 8319

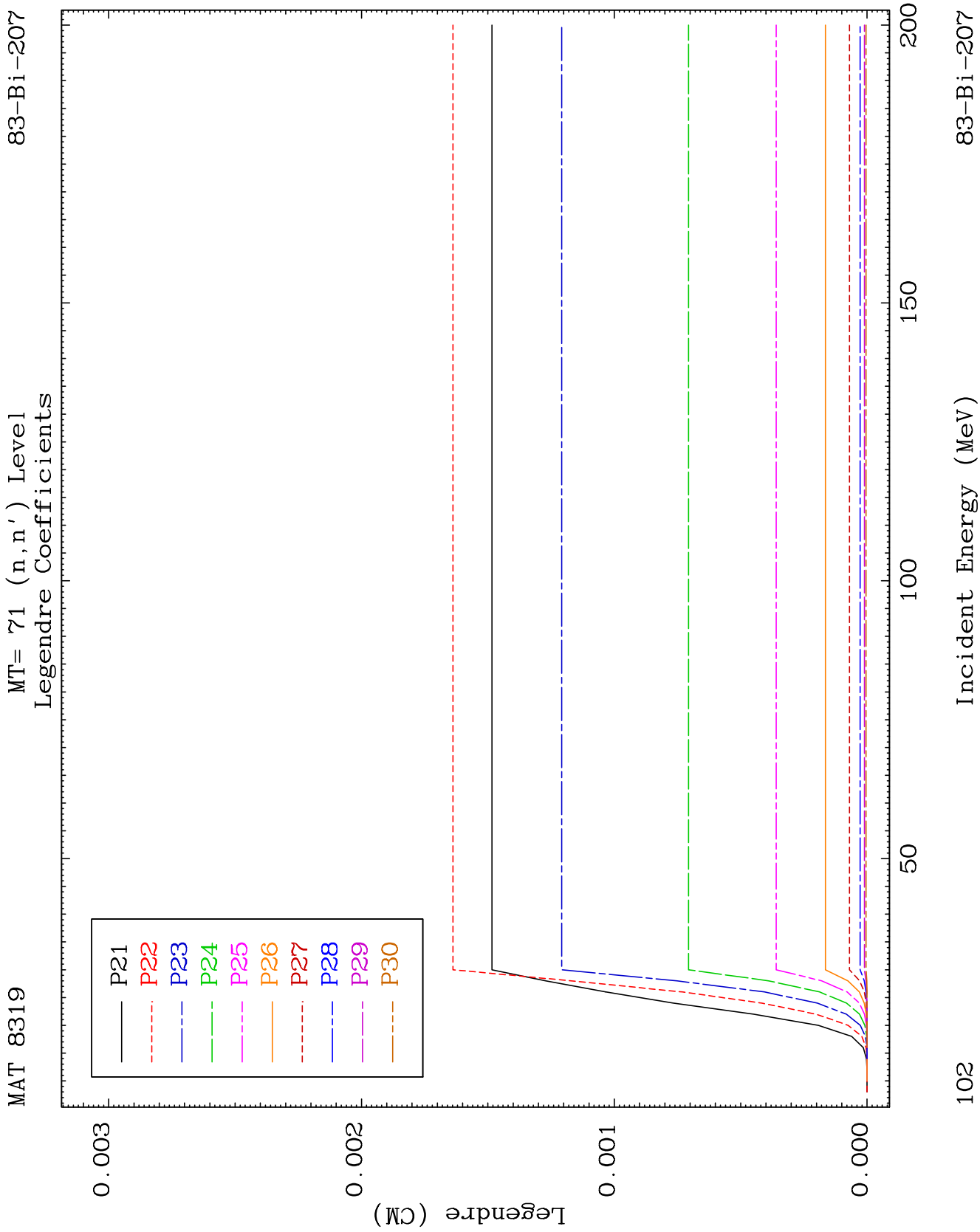
MT= 70 (n, n') Level
Legendre Coefficients

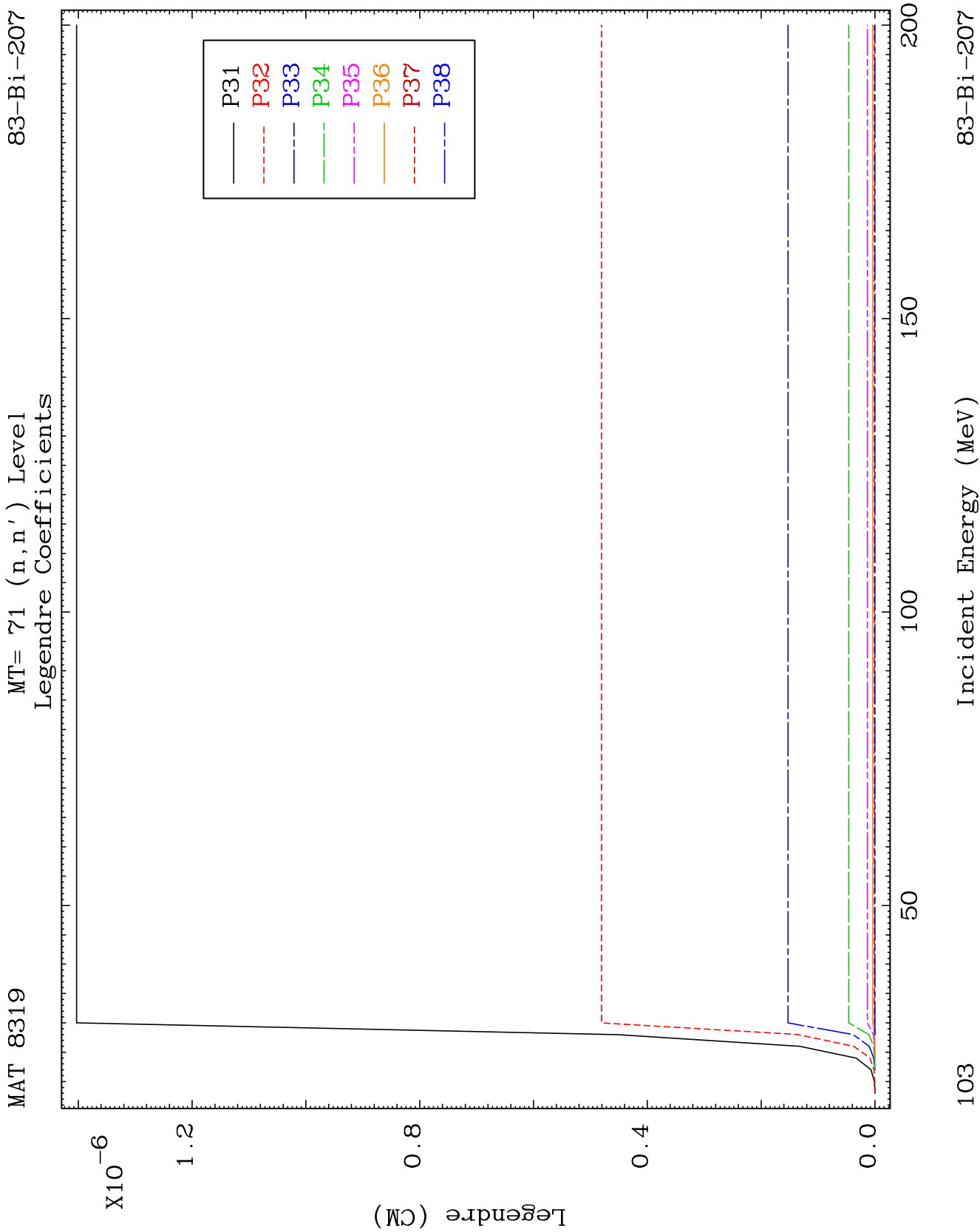
83-Bi-207







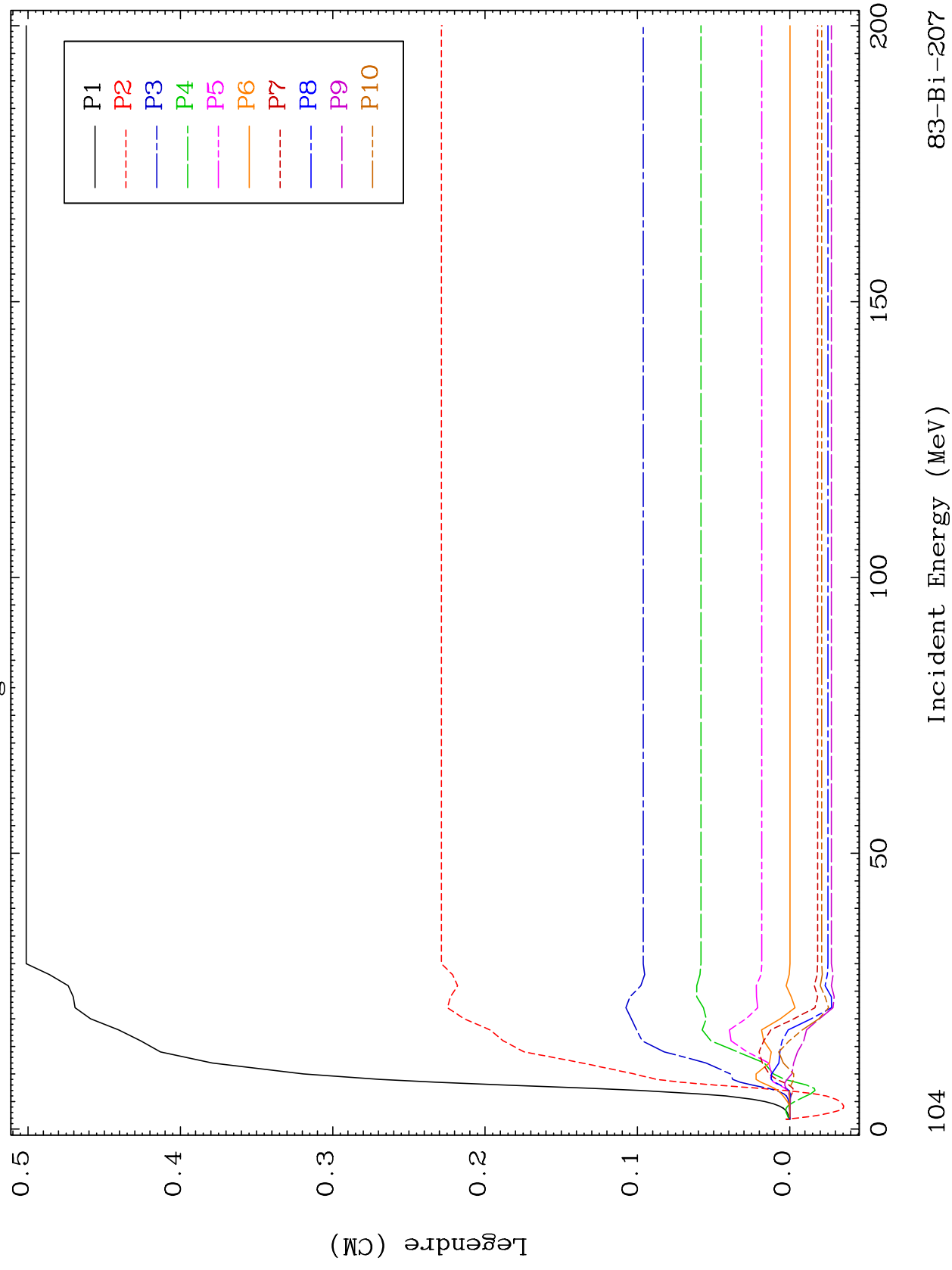




MAT 8319

MT= 72 (n, n') Level
Legendre Coefficients

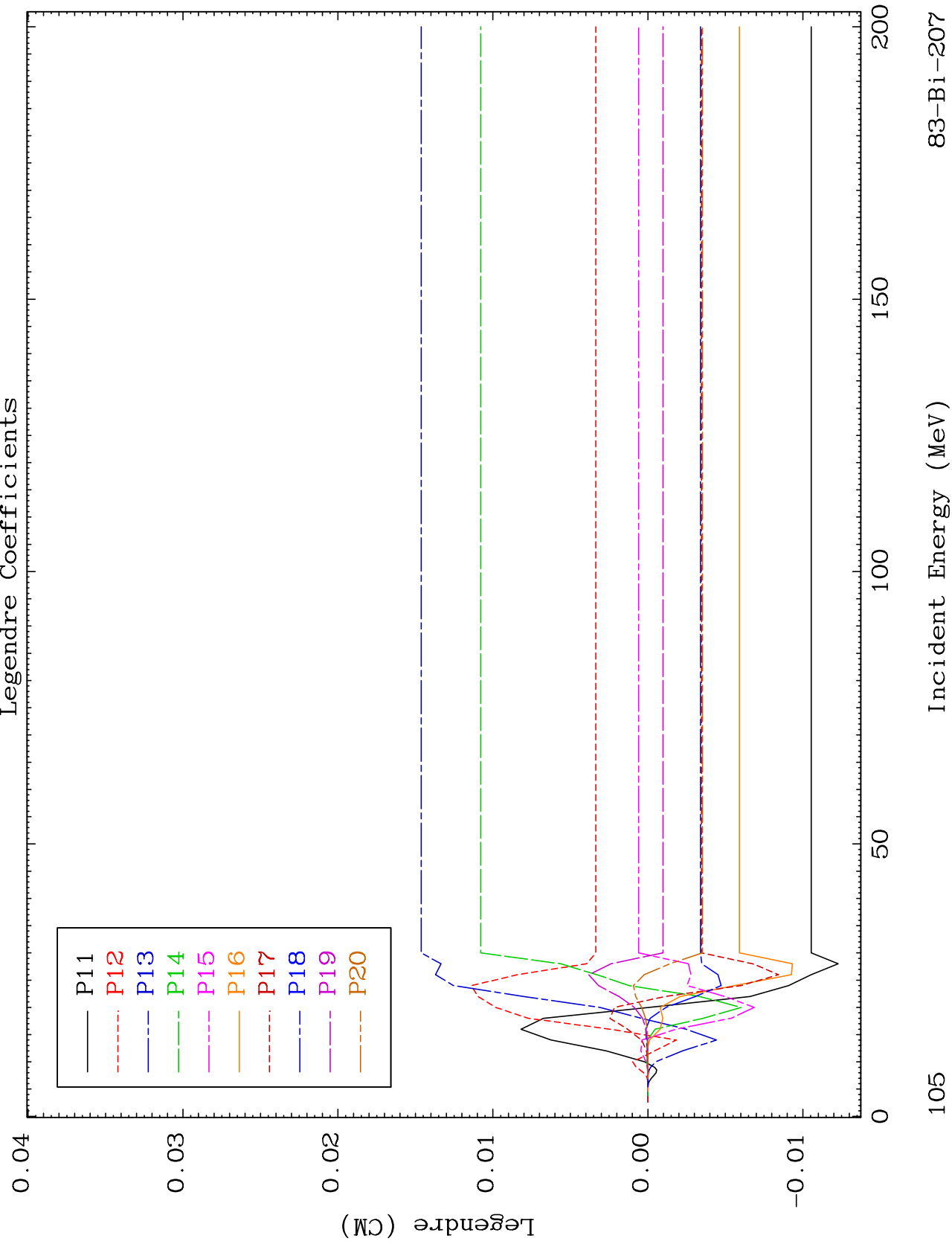
83-Bi-207

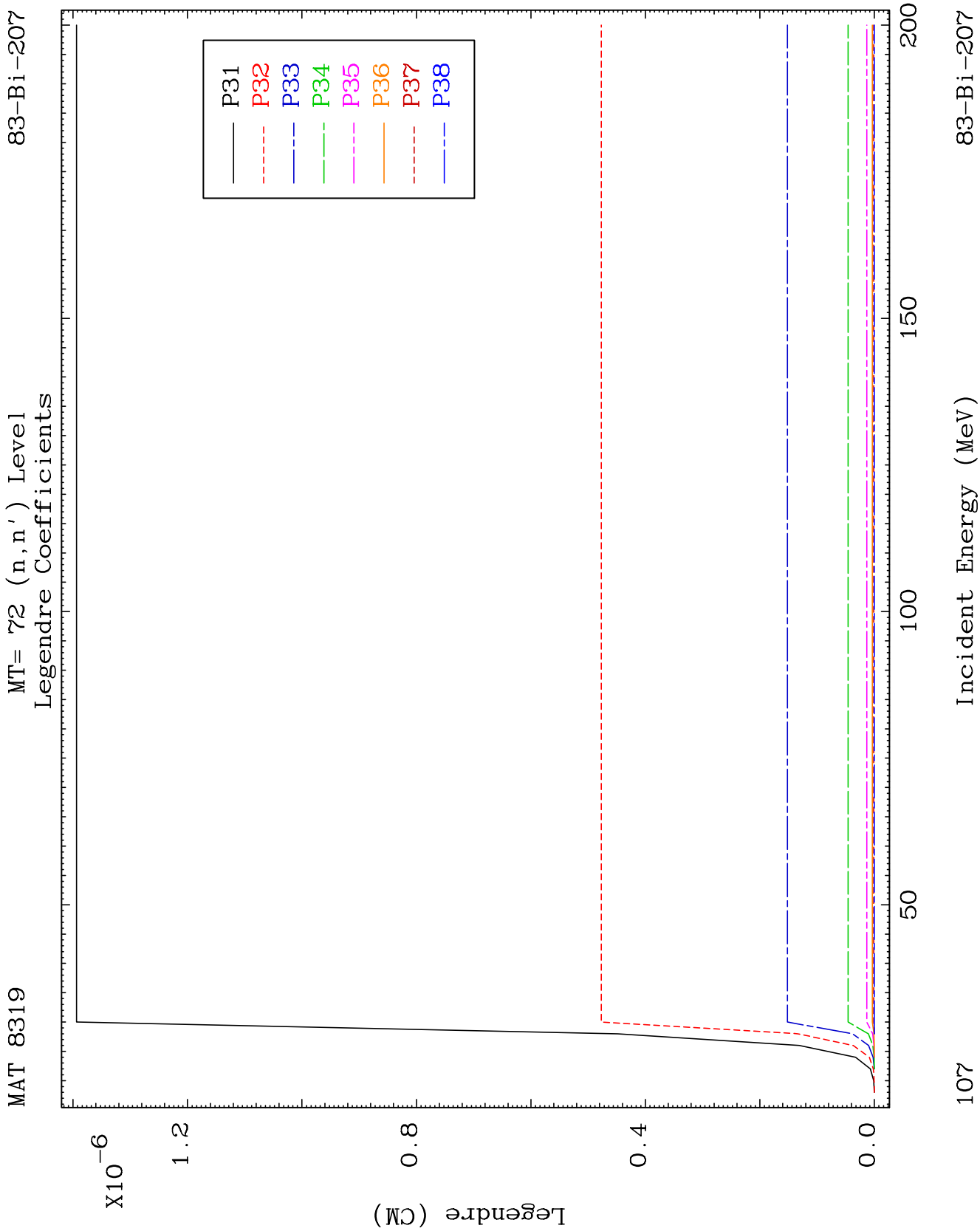


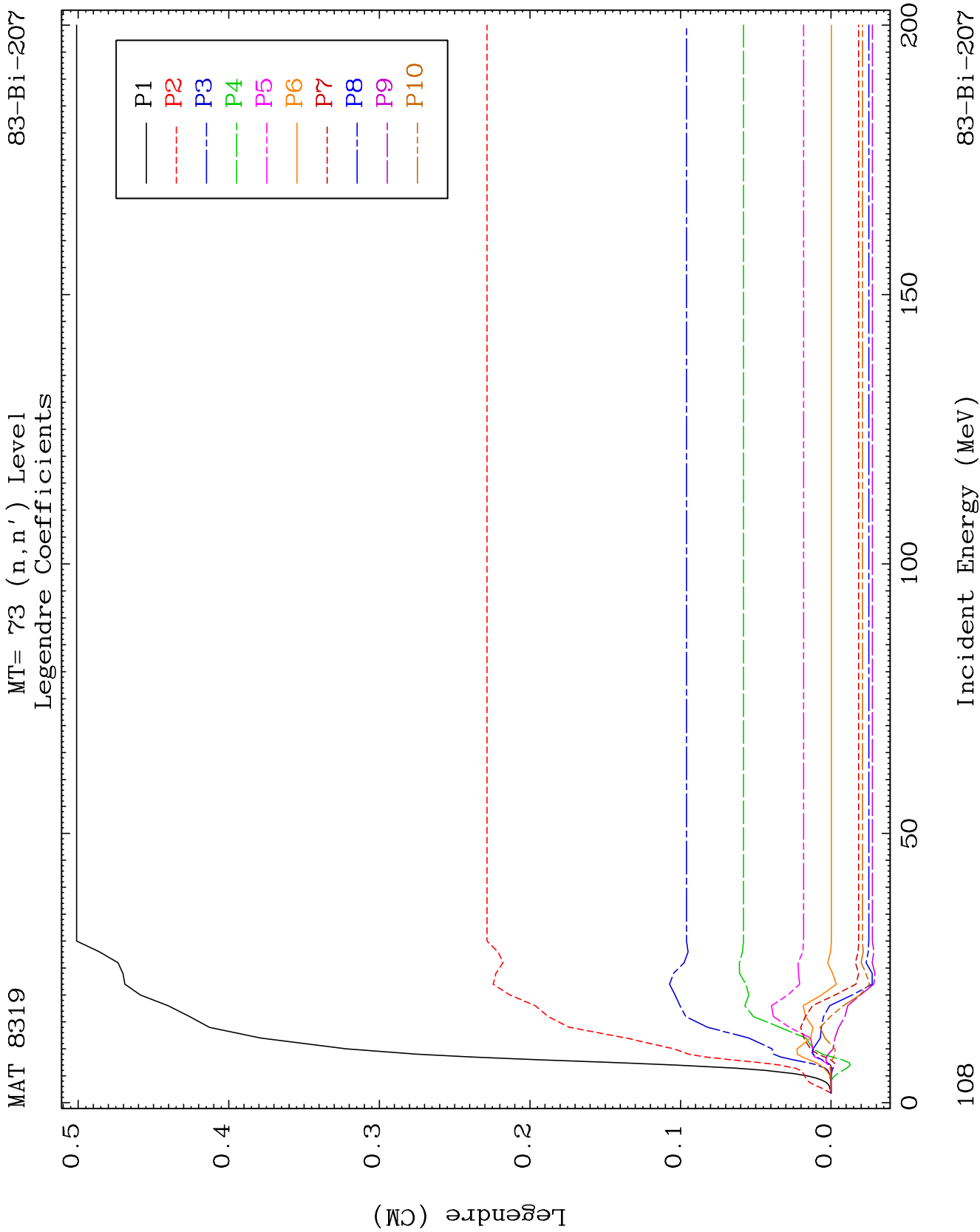
MAT 8319

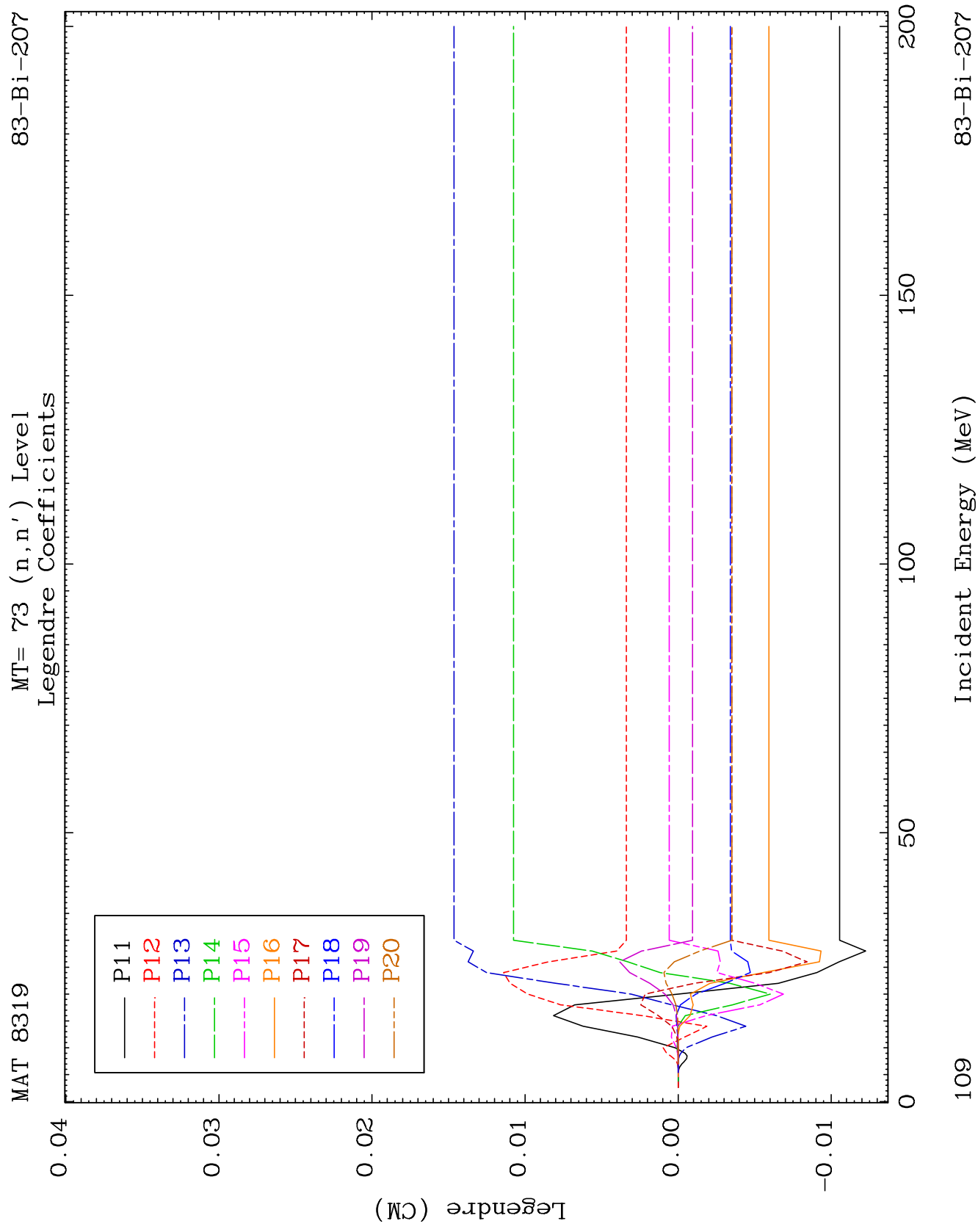
MT= 72 (n, n') Level
Legendre Coefficients

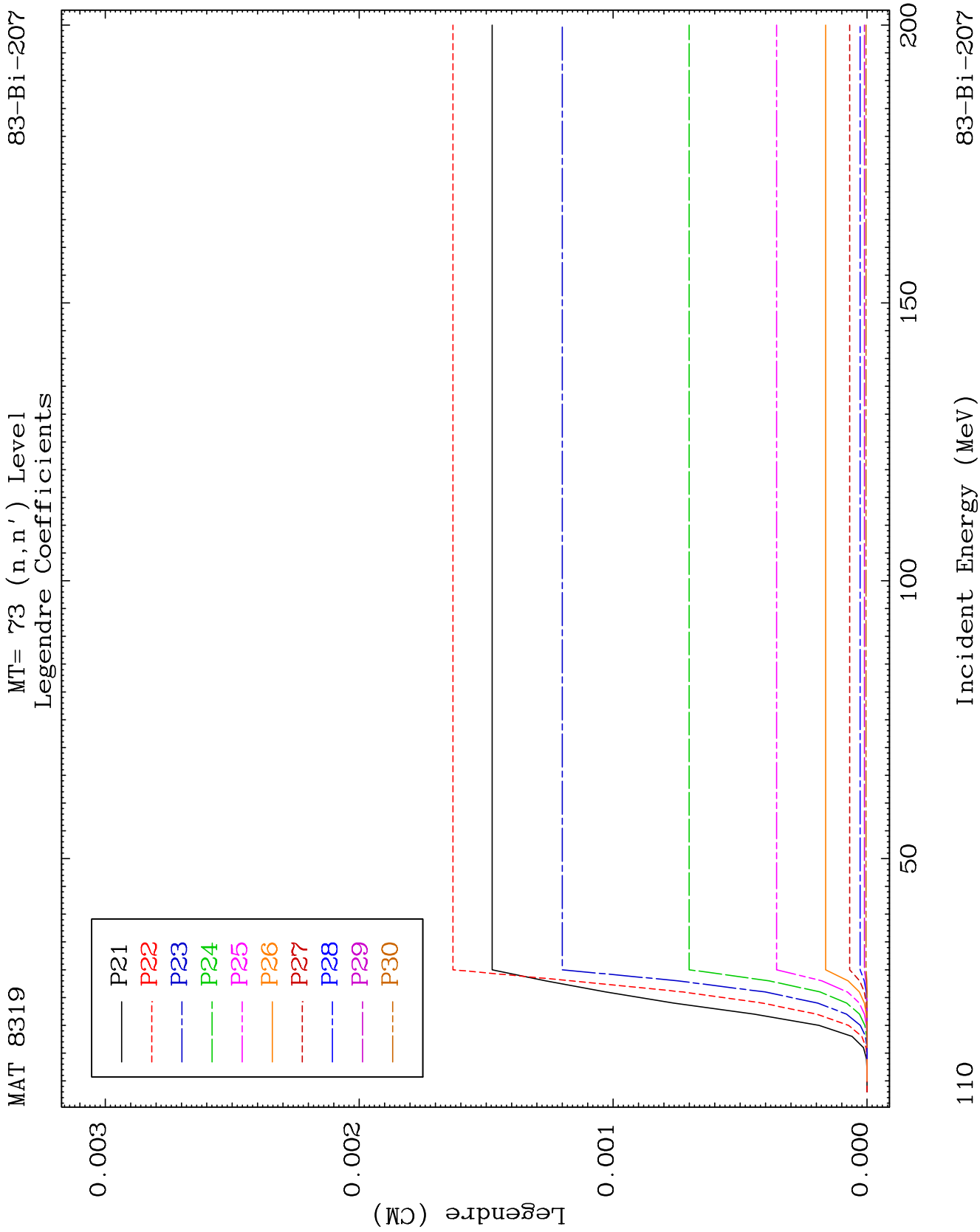
83-Bi-207

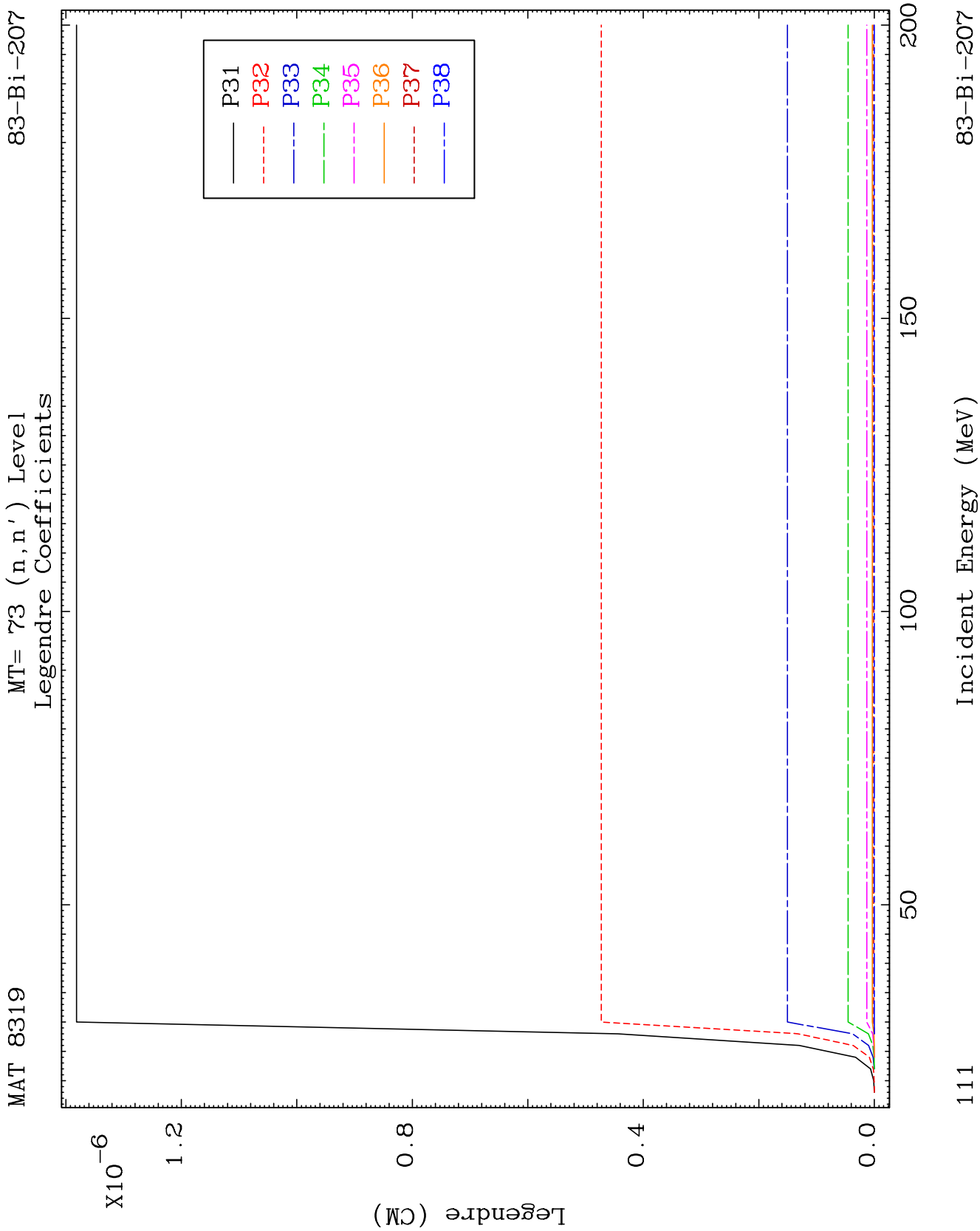


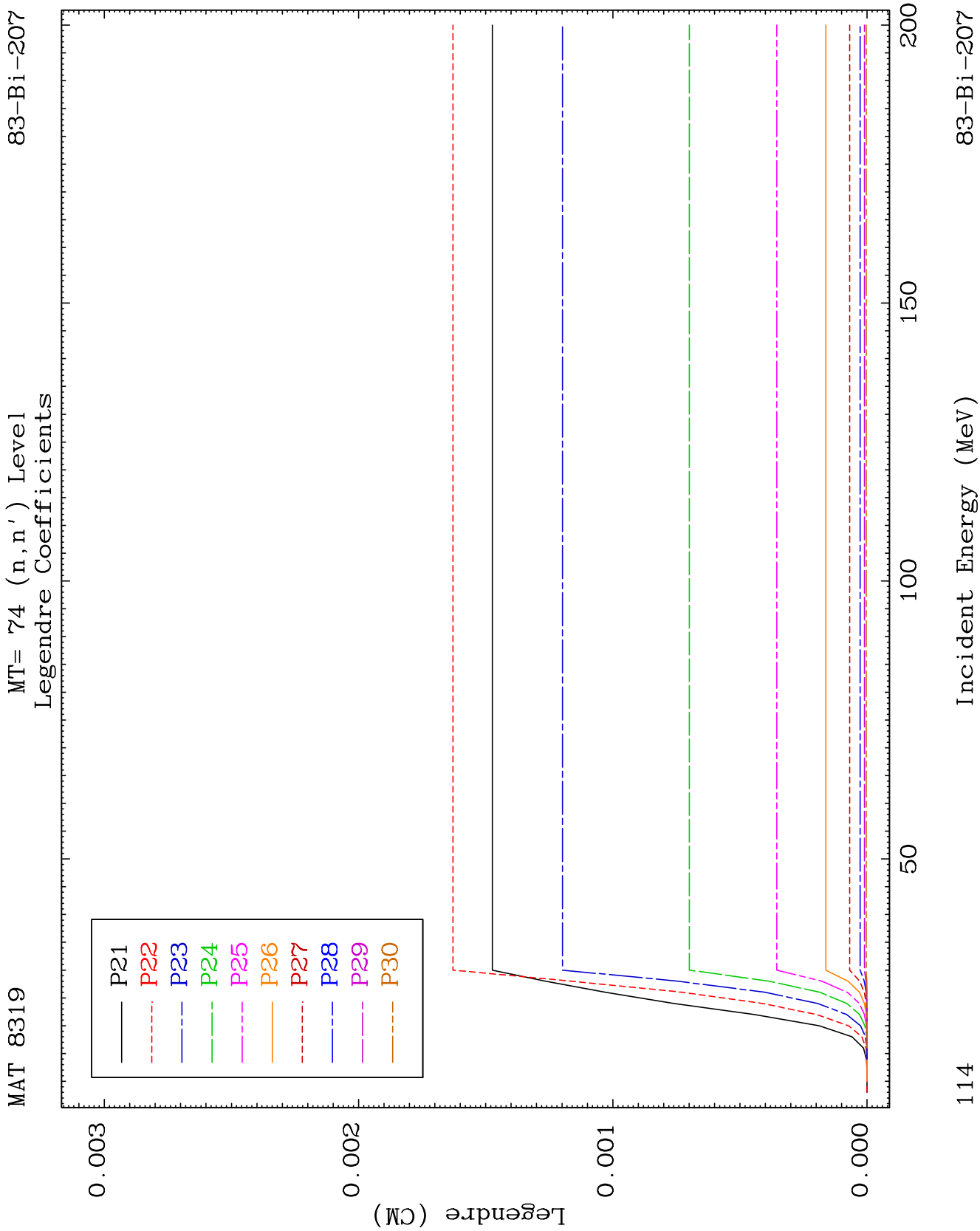


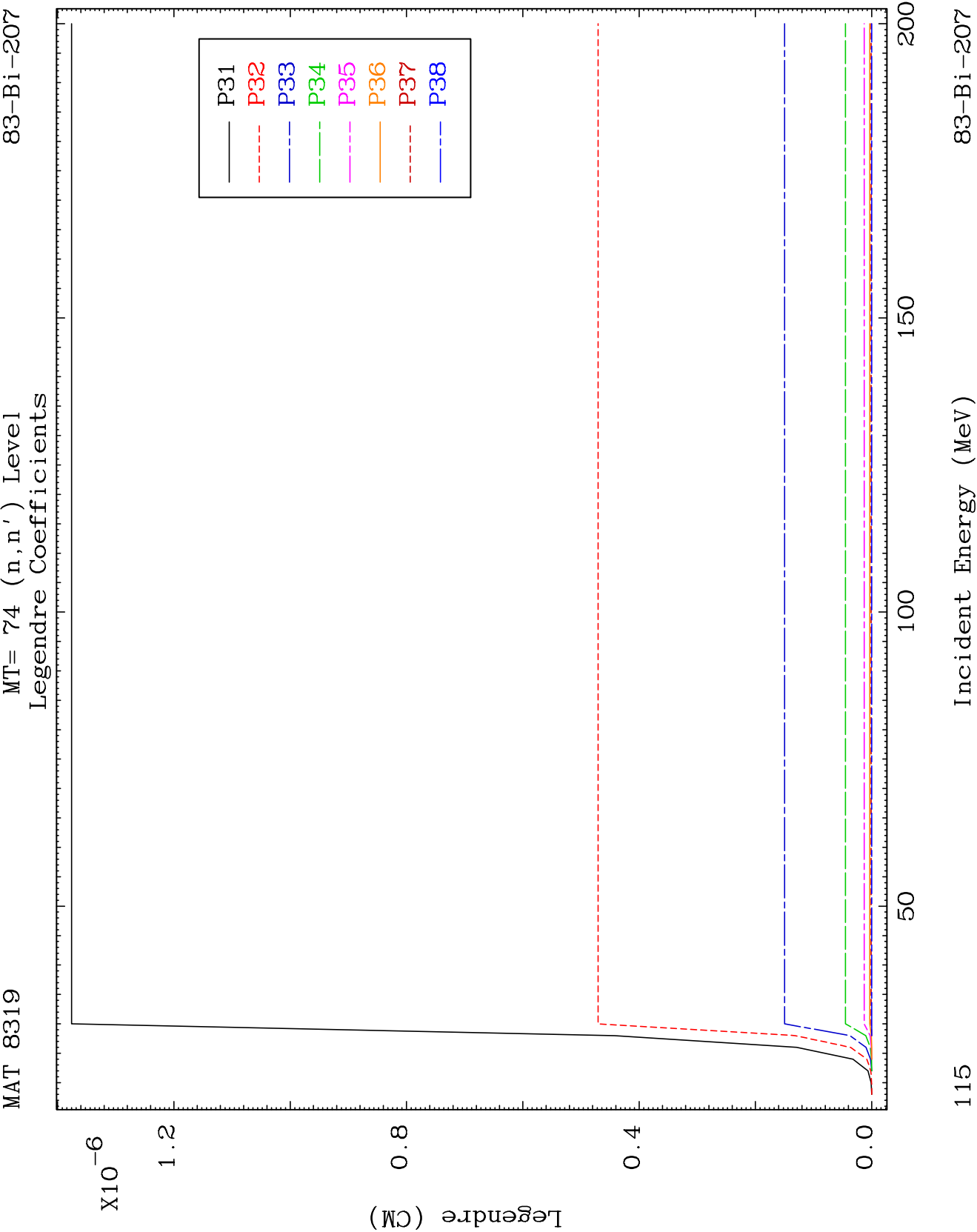


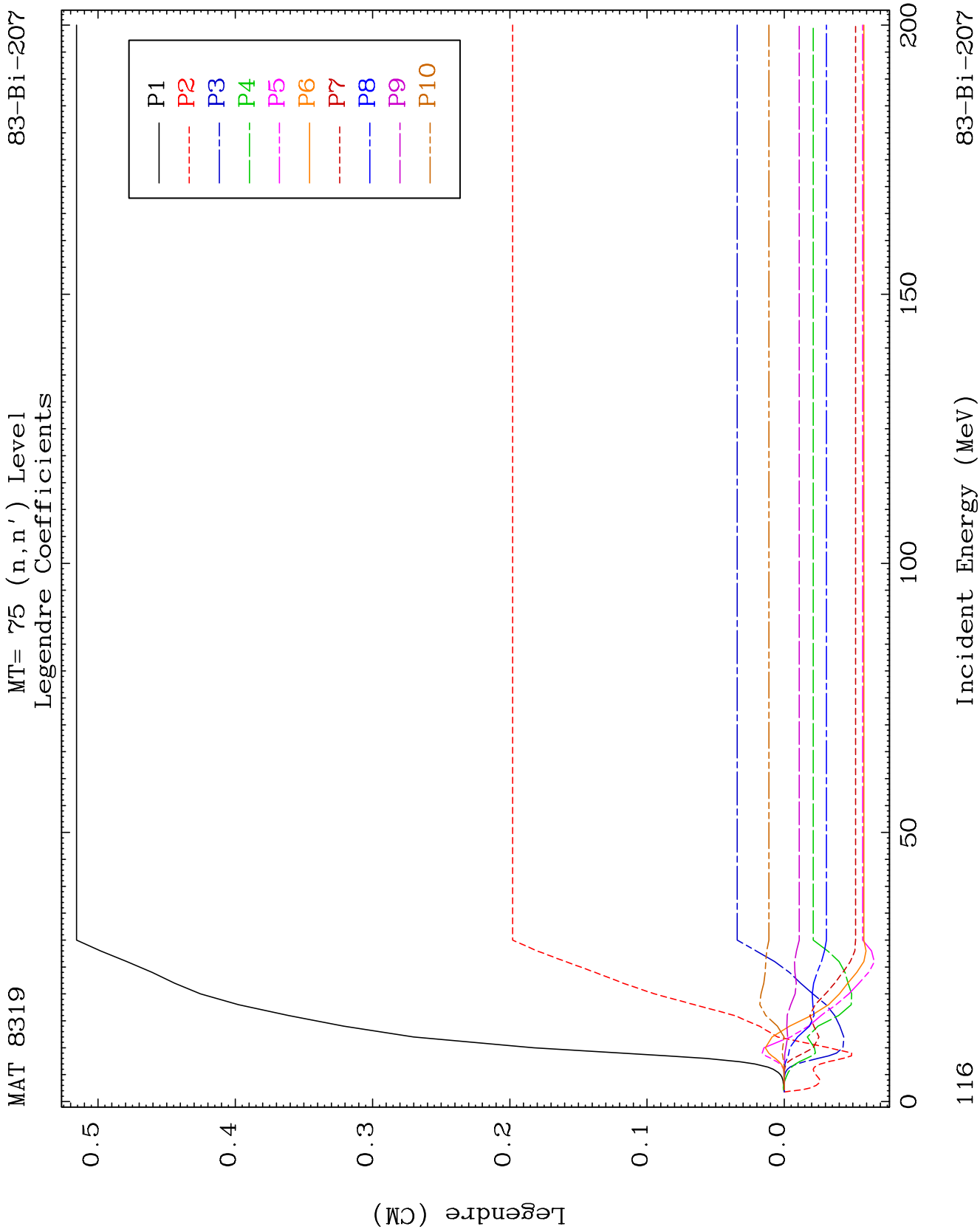


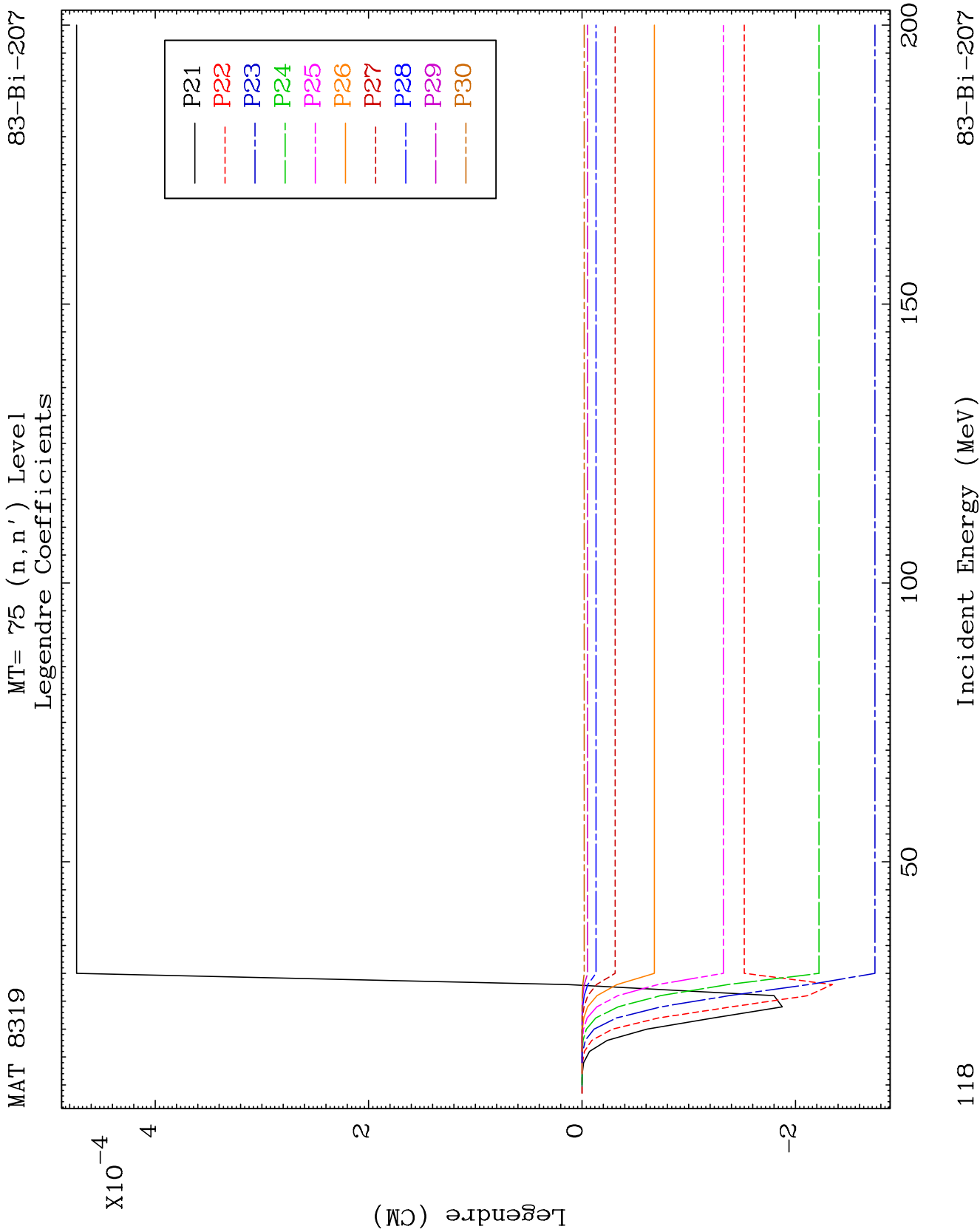


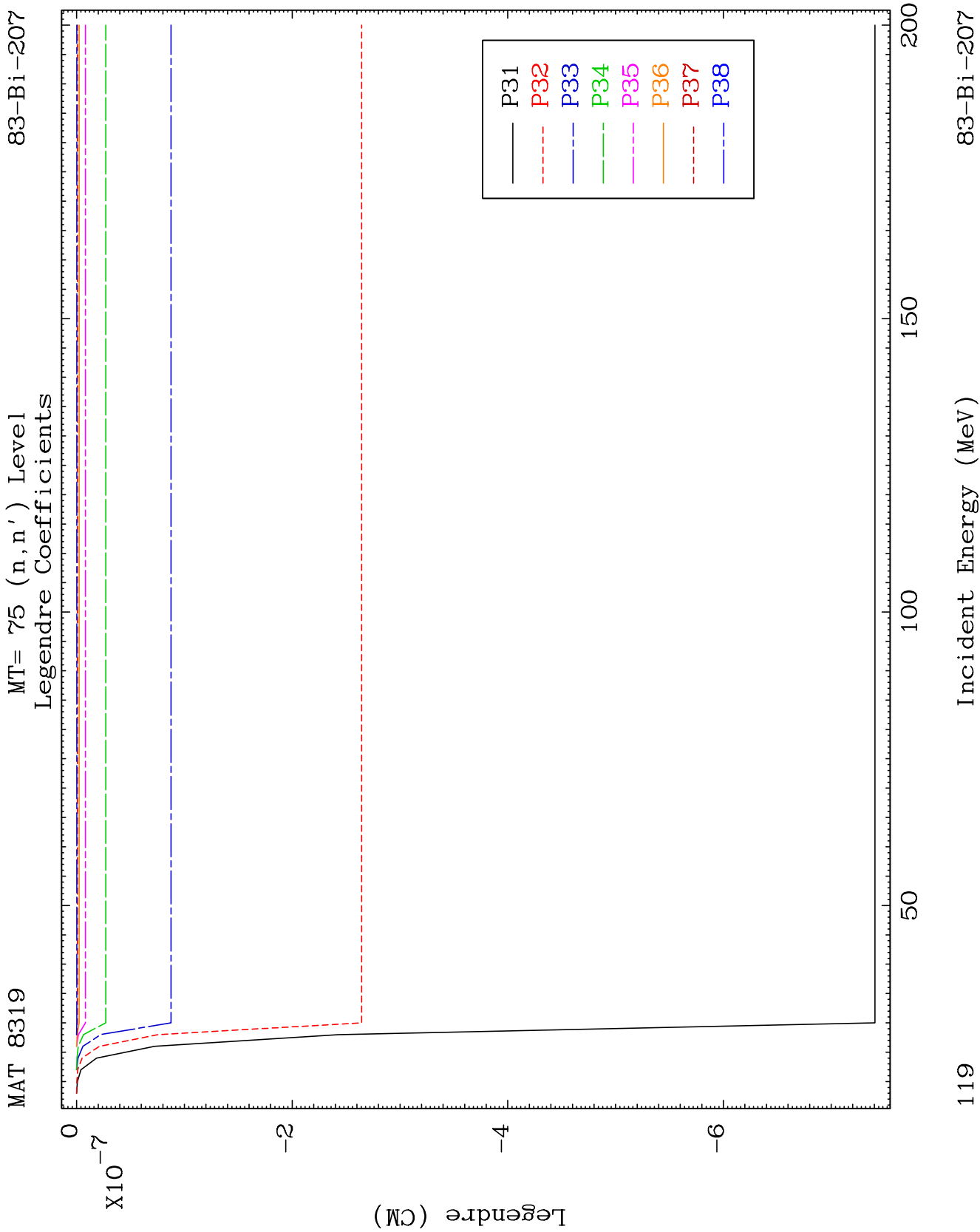


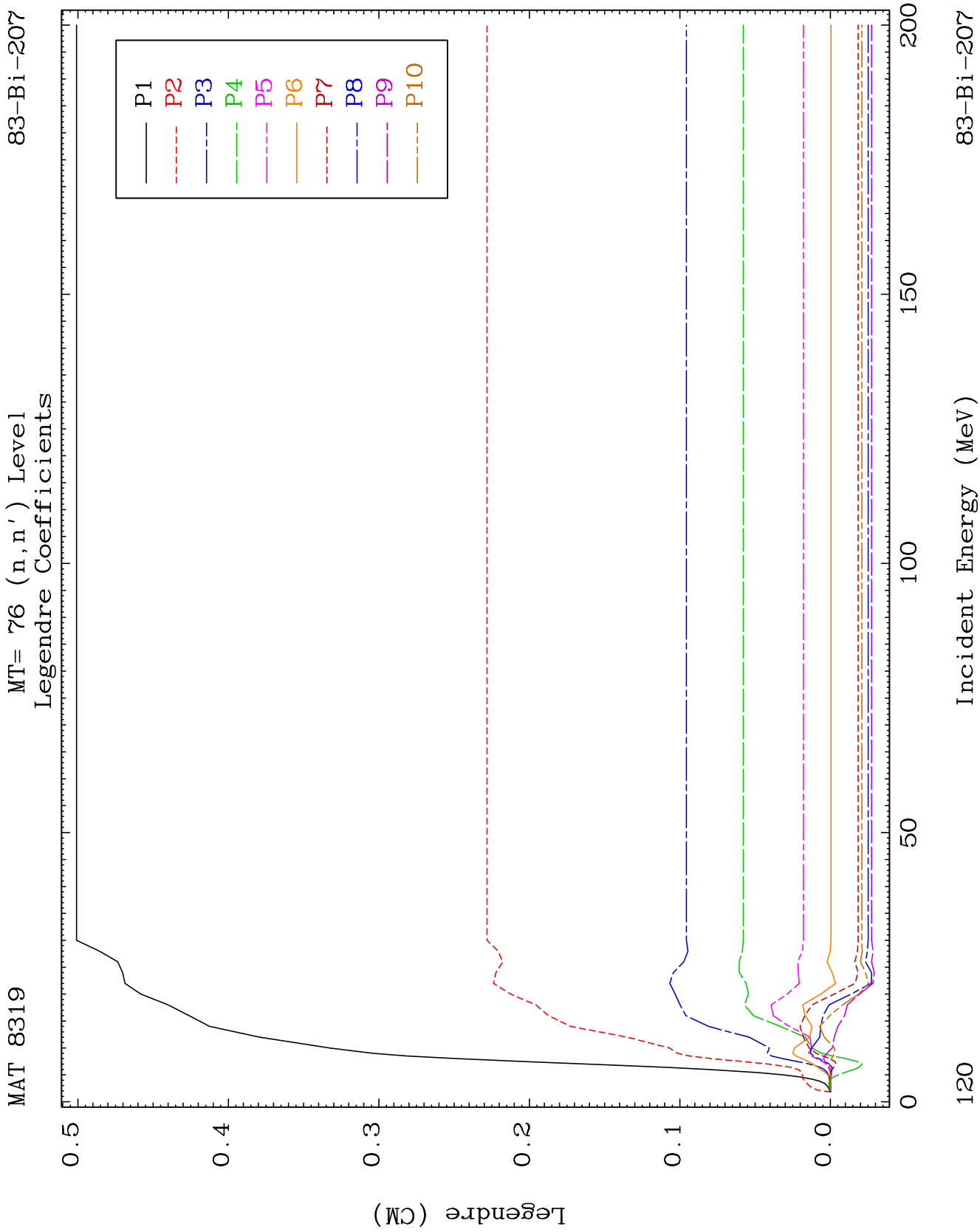


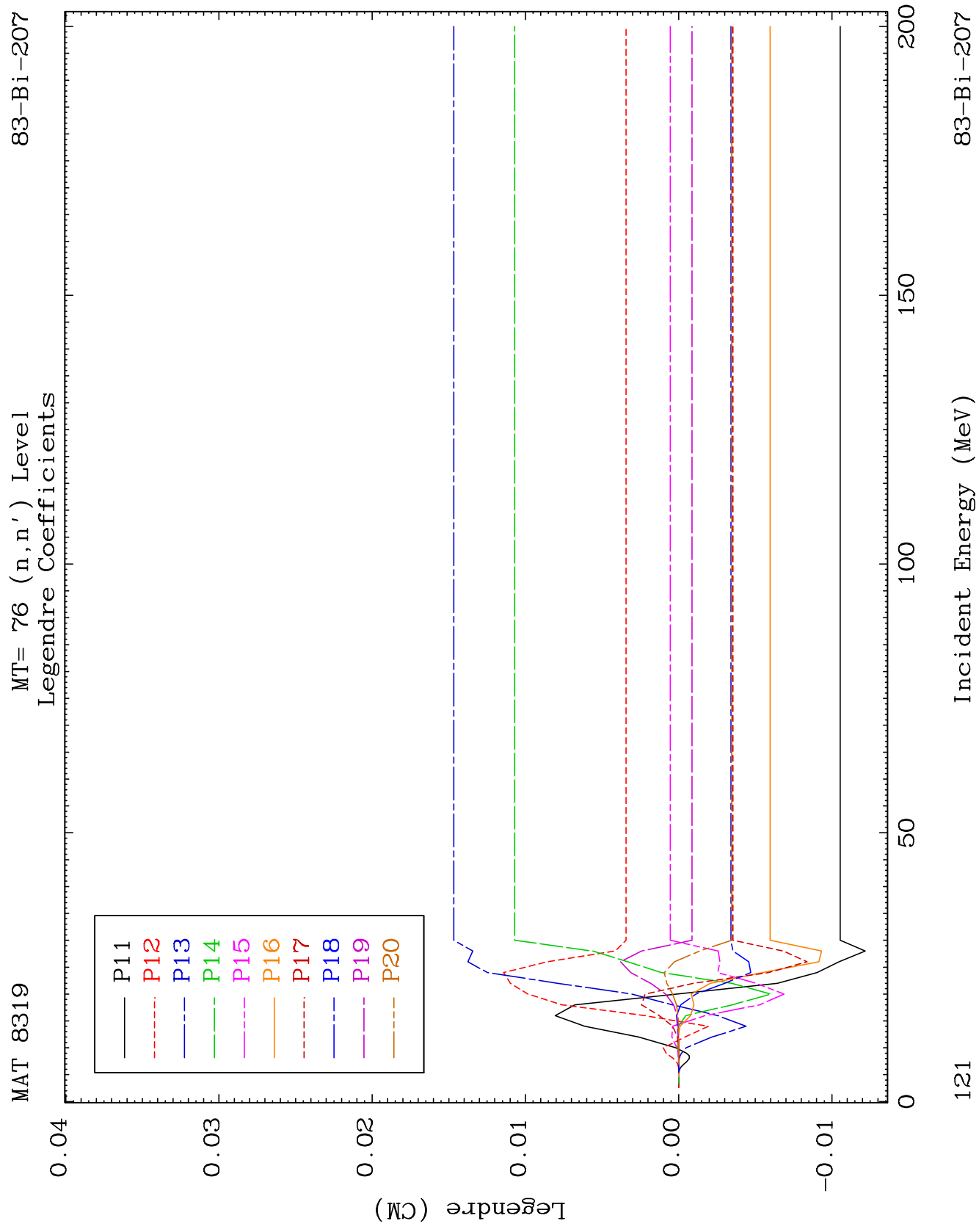


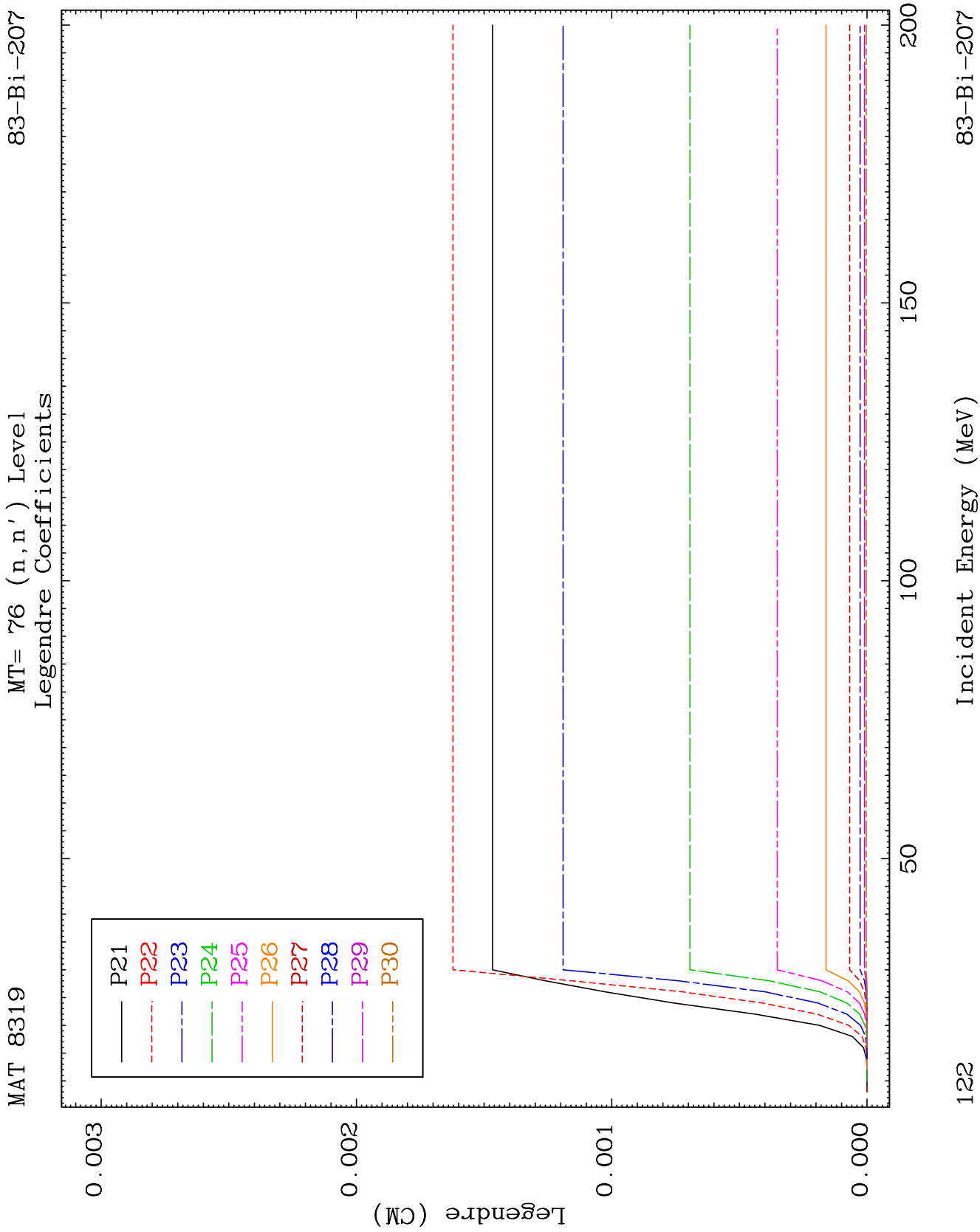


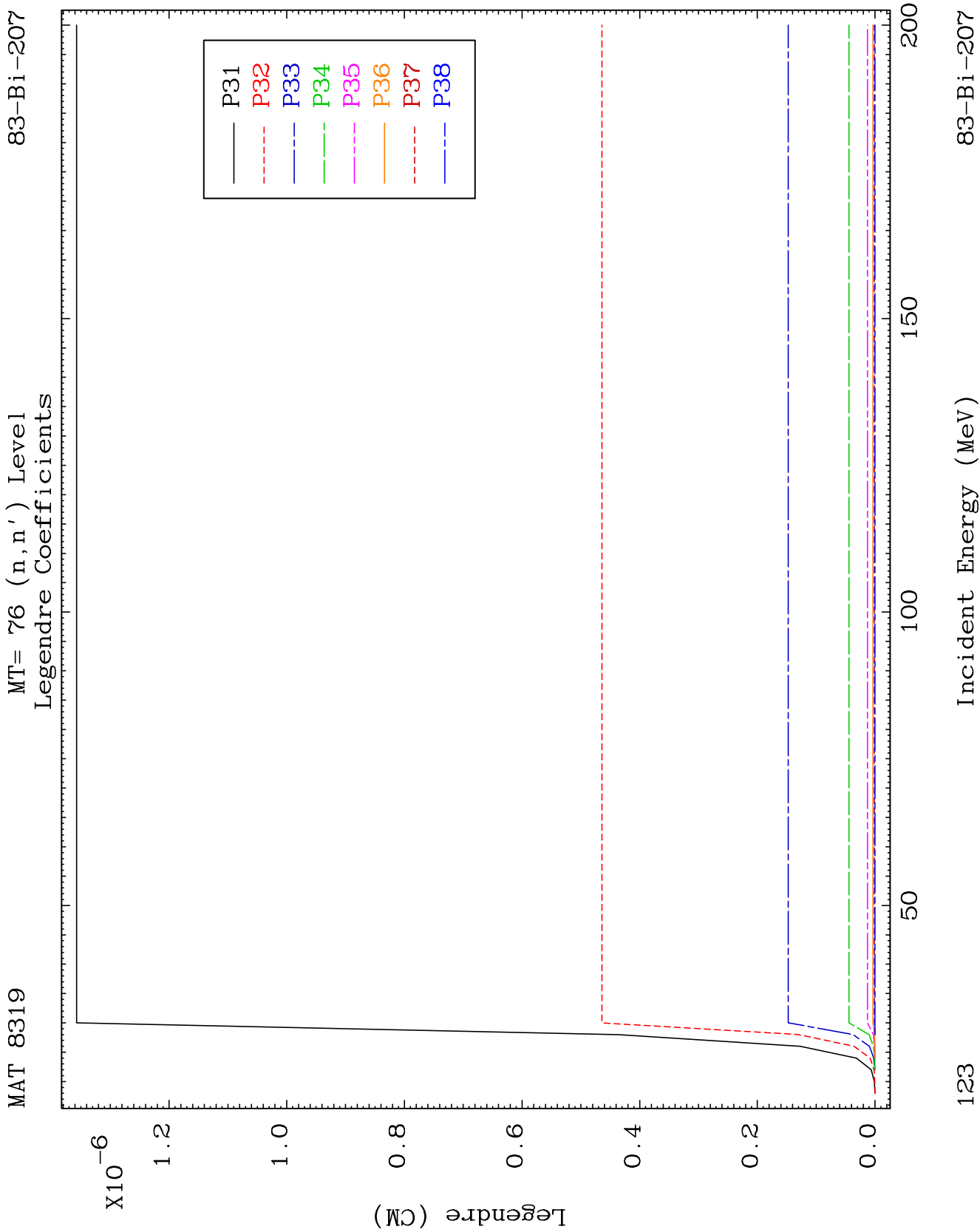








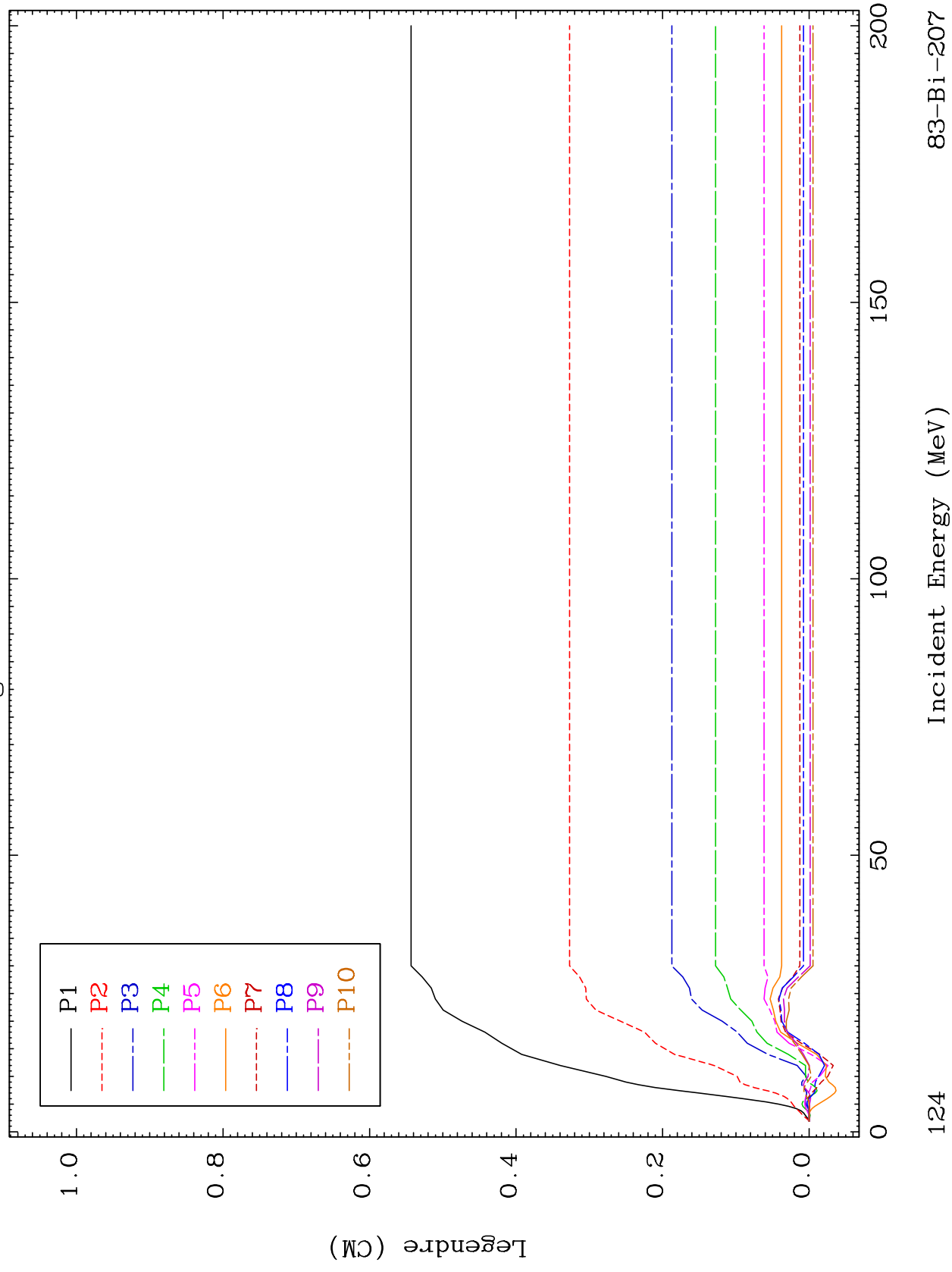


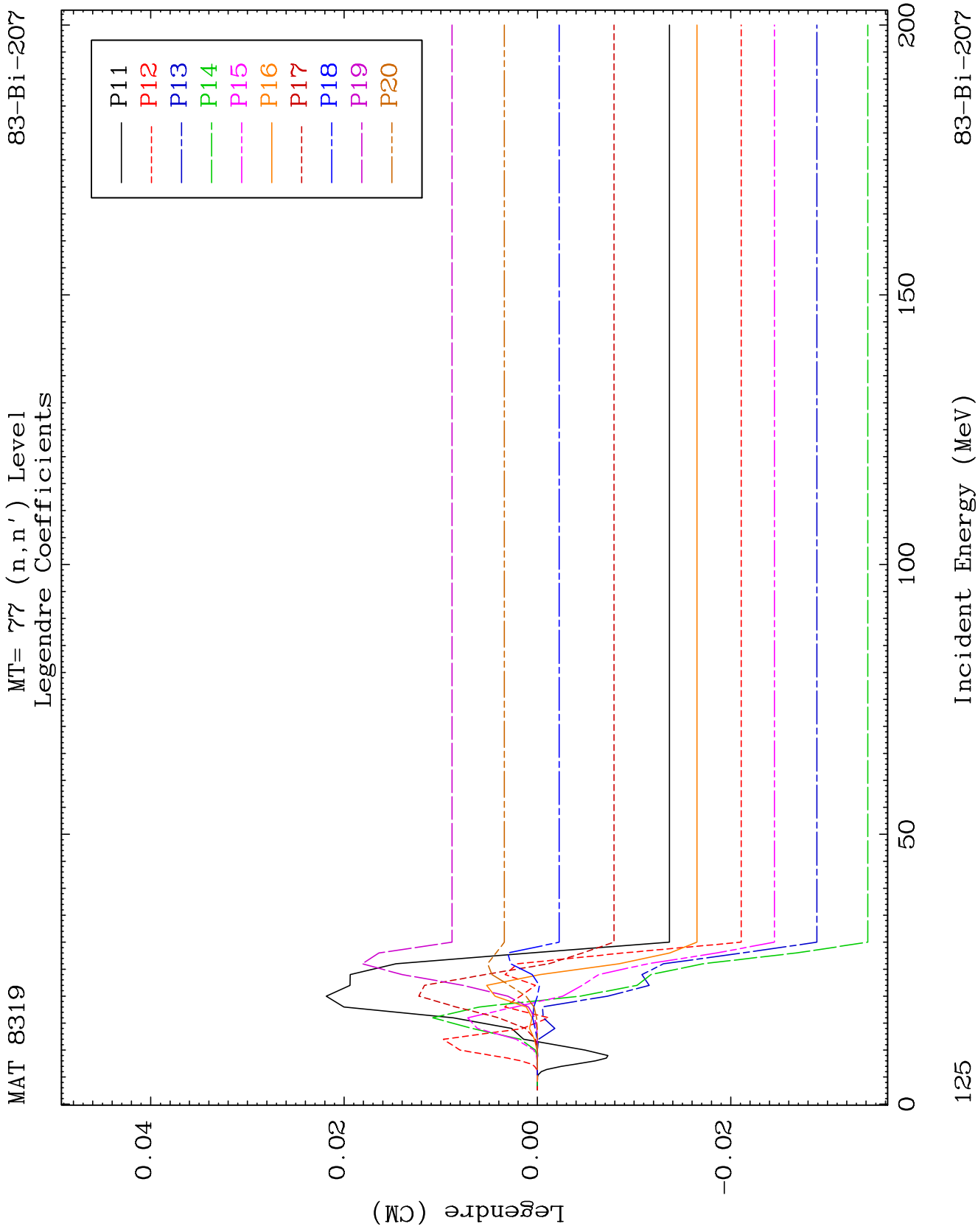


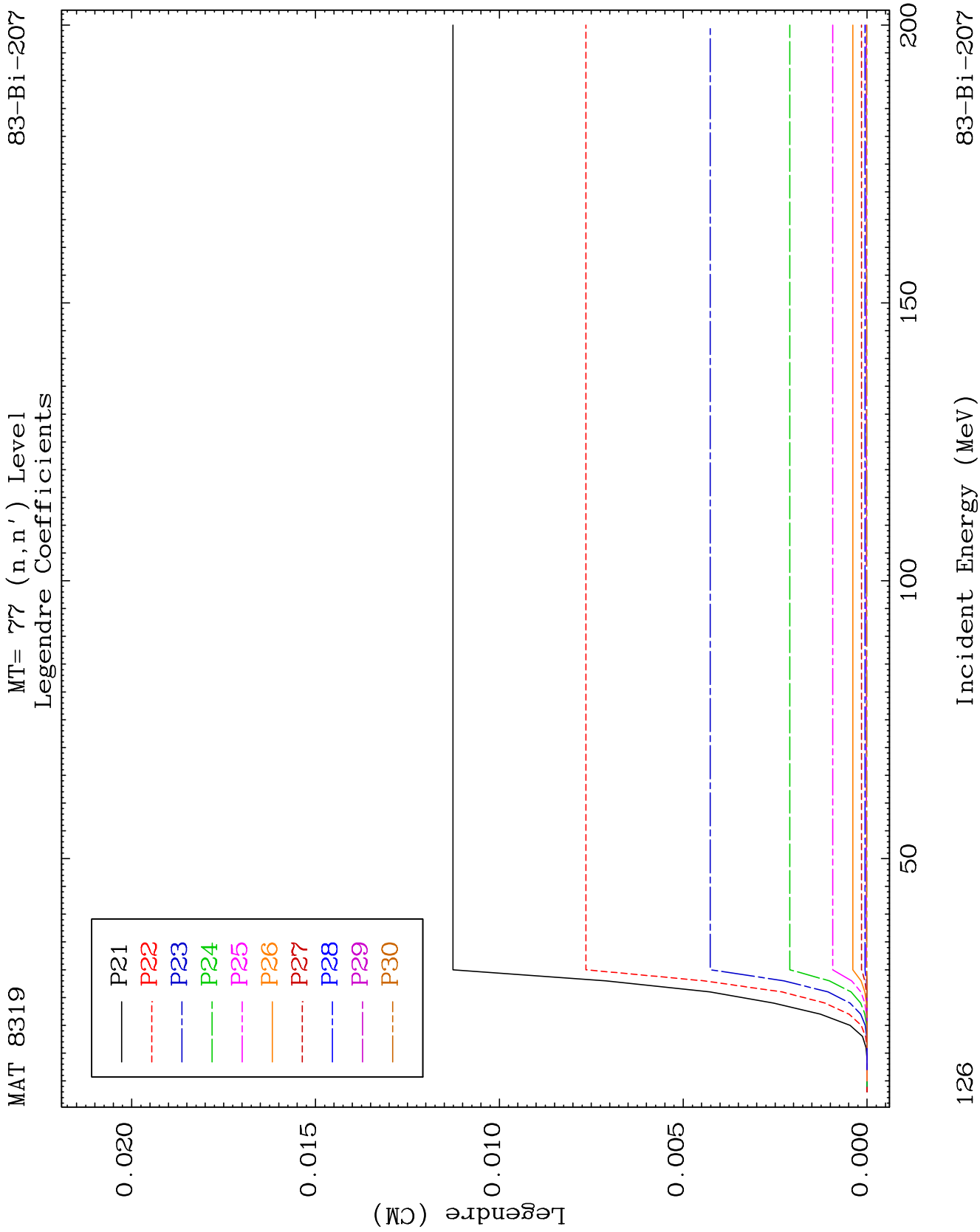
MAT 8319

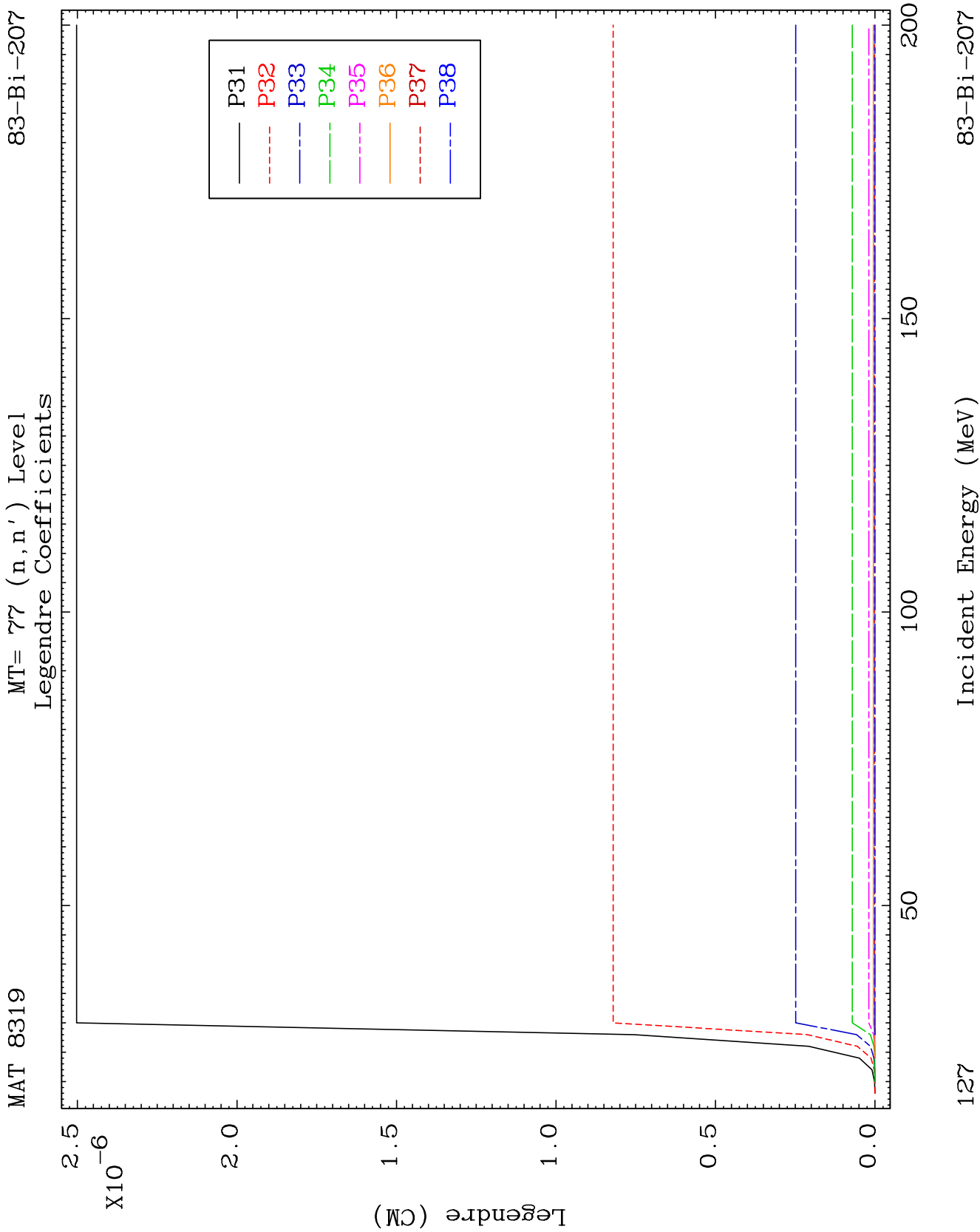
MT= 77 (n, n') Level
Legendre Coefficients

83-Bi-207





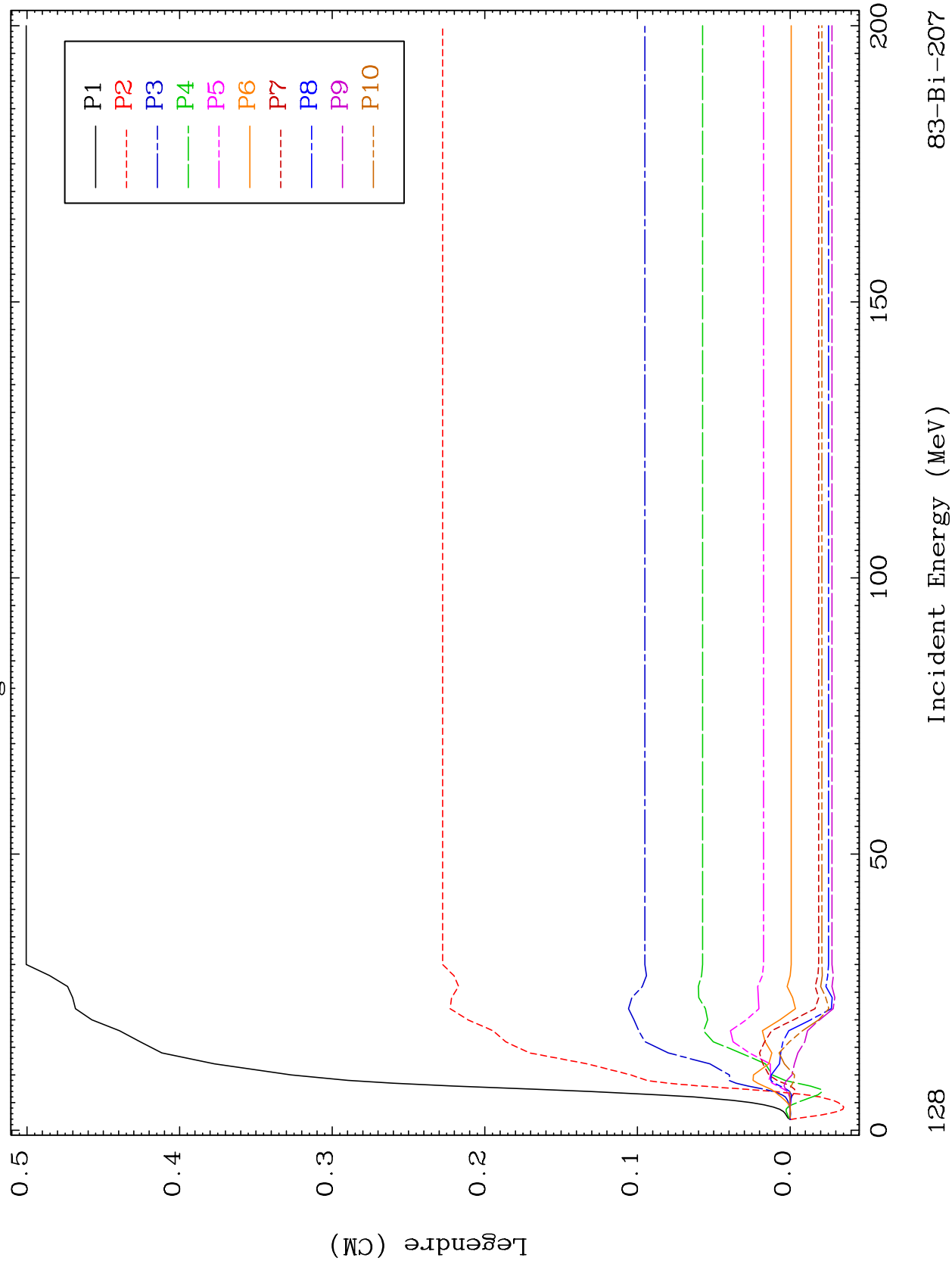


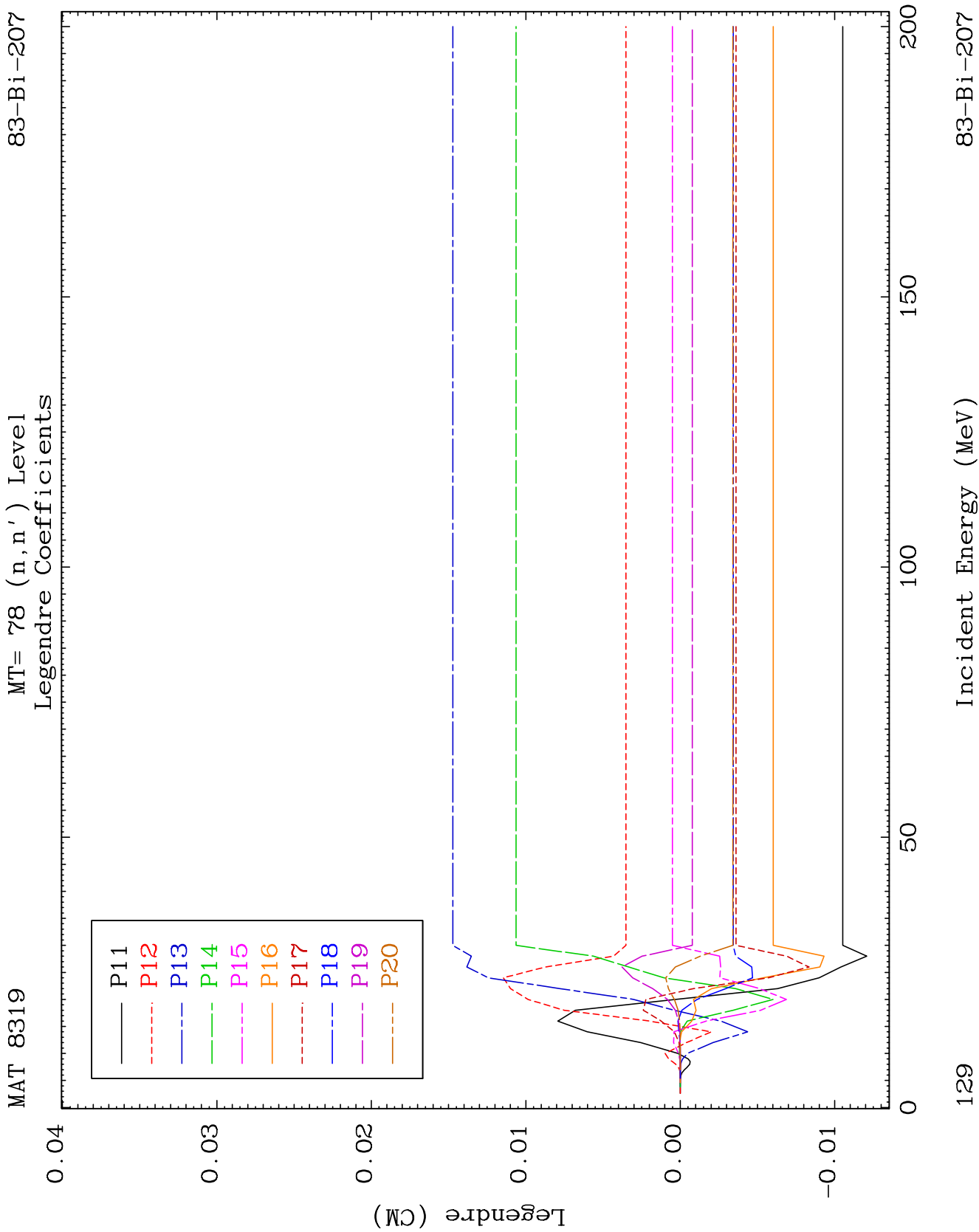


MAT 8319

MT= 78 (n, n') Level
Legendre Coefficients

83-Bi-207

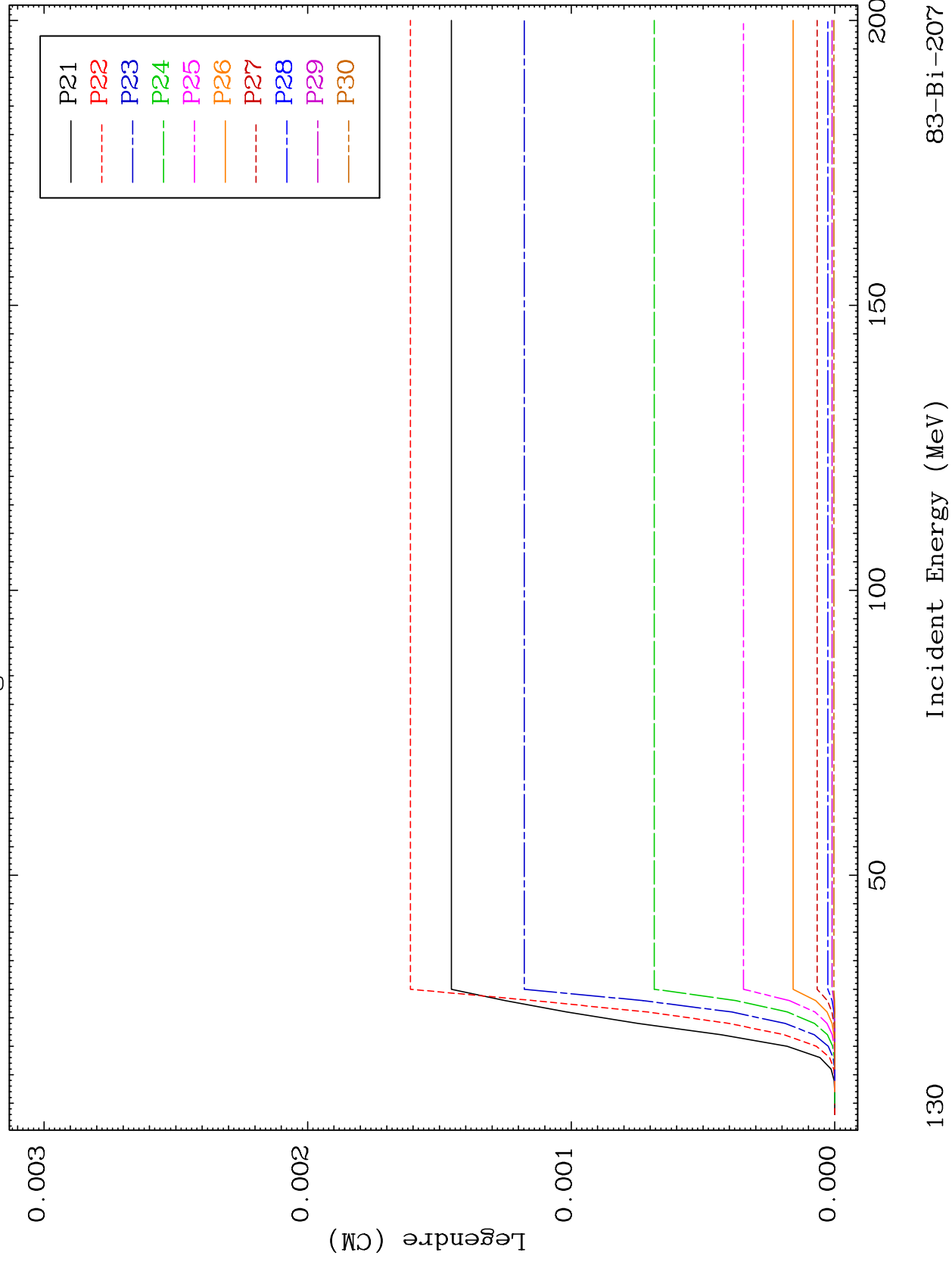


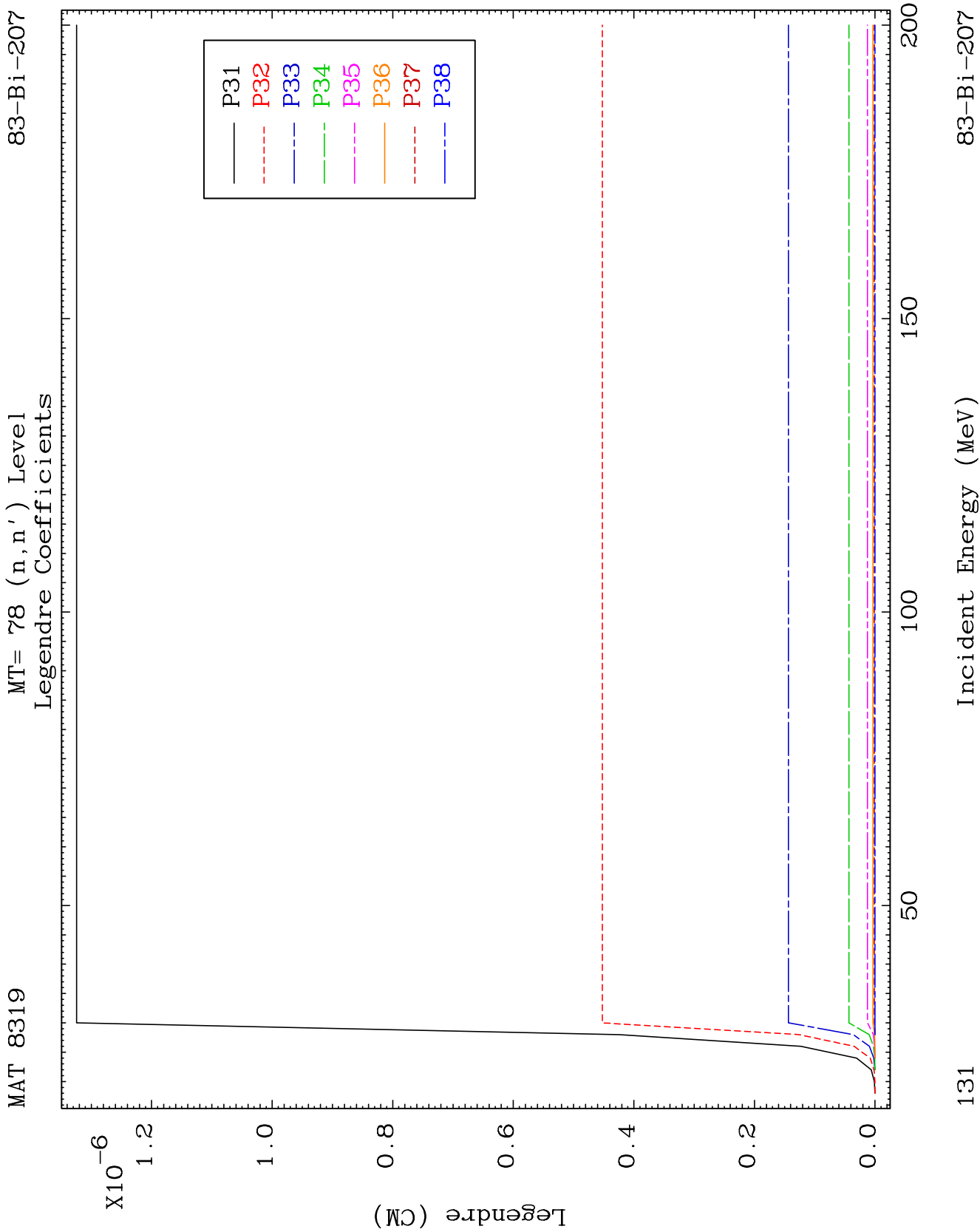


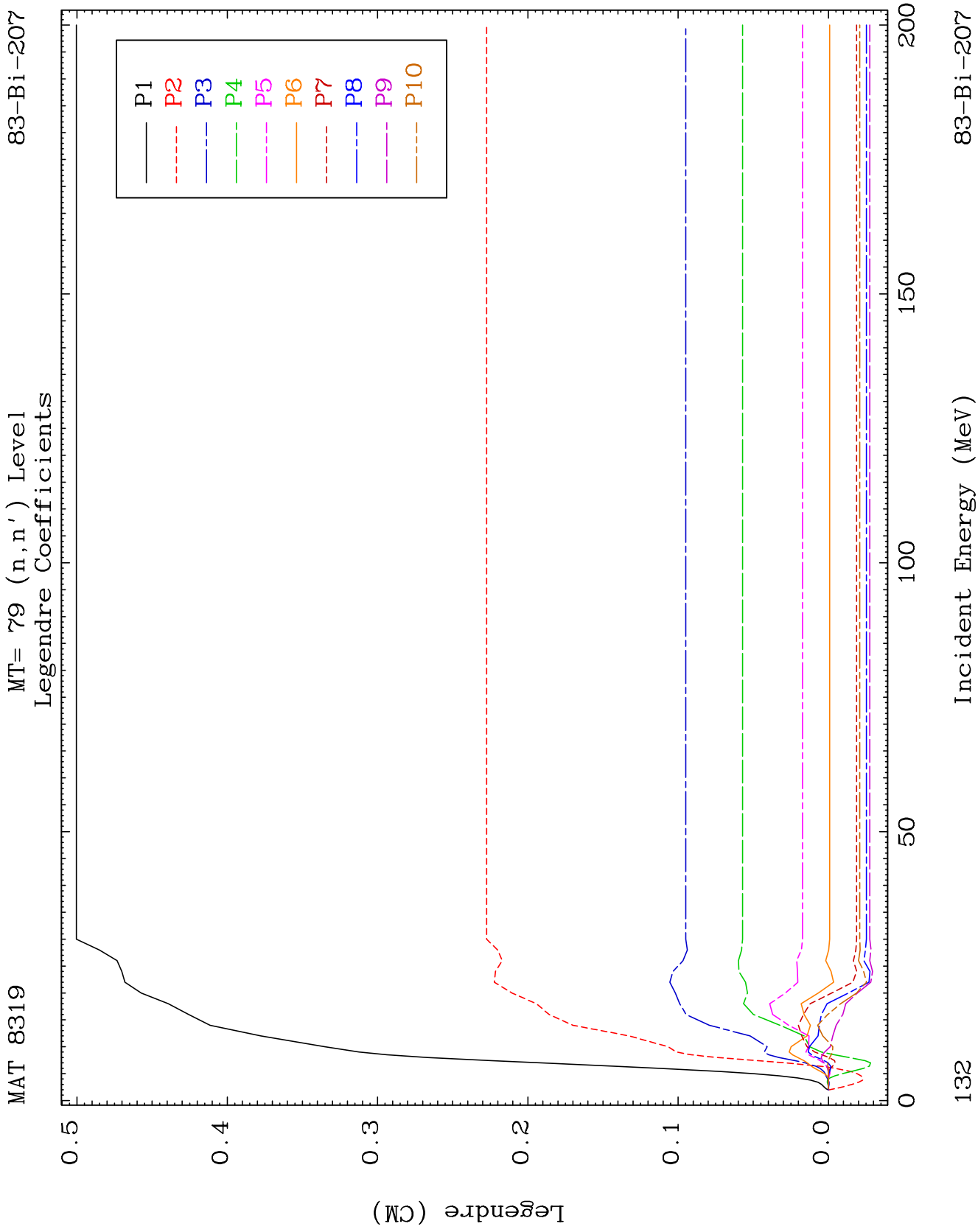
MAT 8319

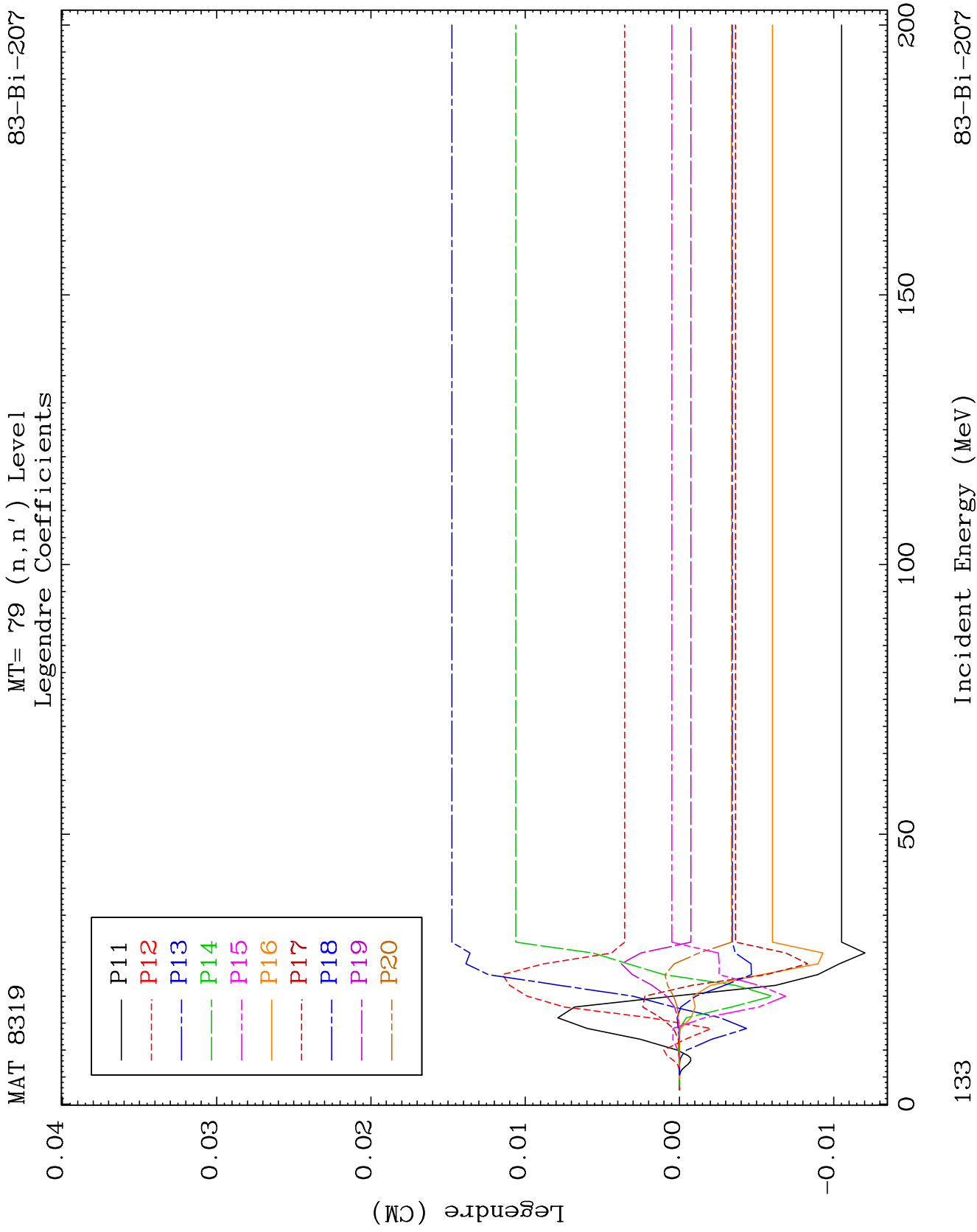
MT= 78 (n, n') Level
Legendre Coefficients

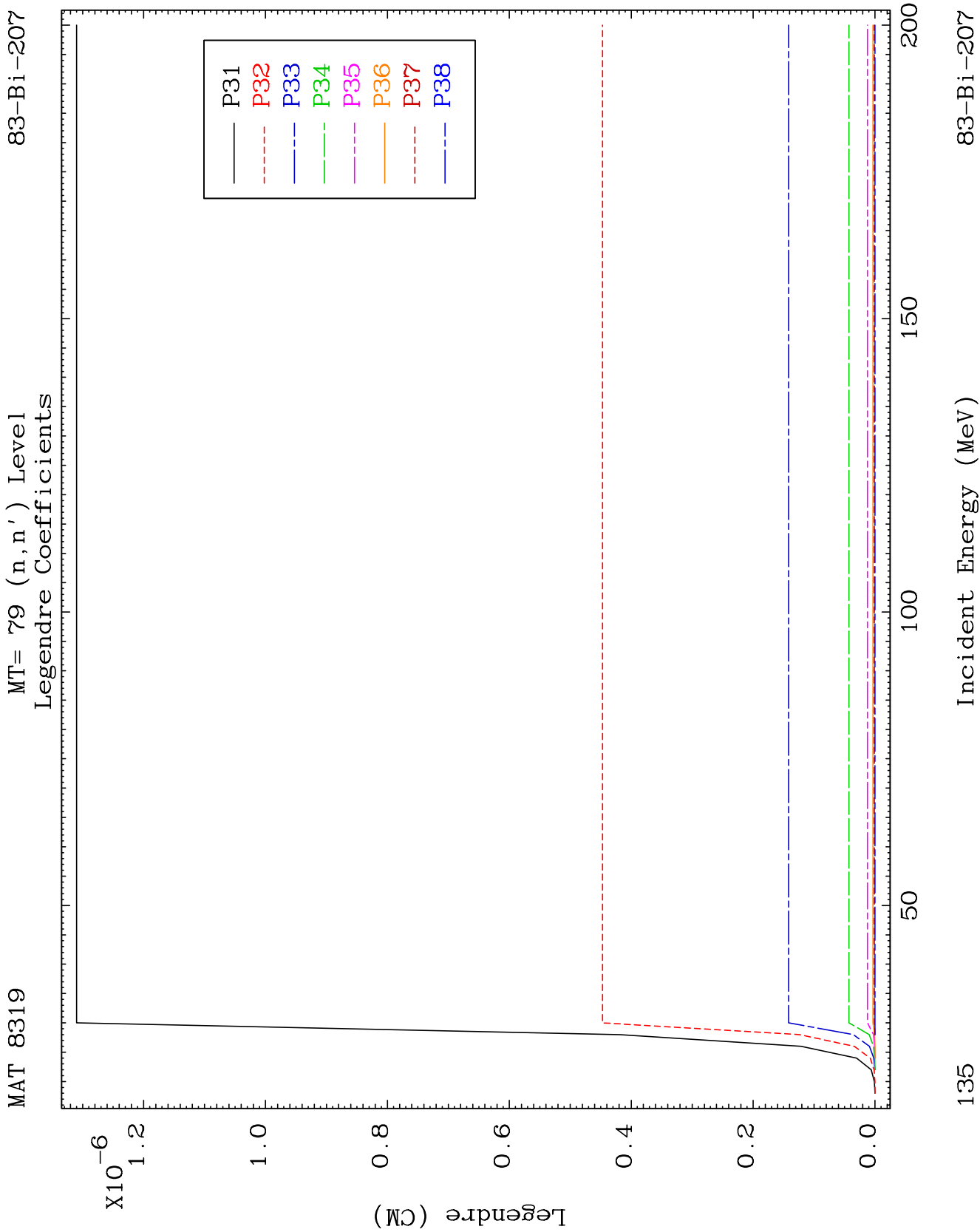
83-Bi-207



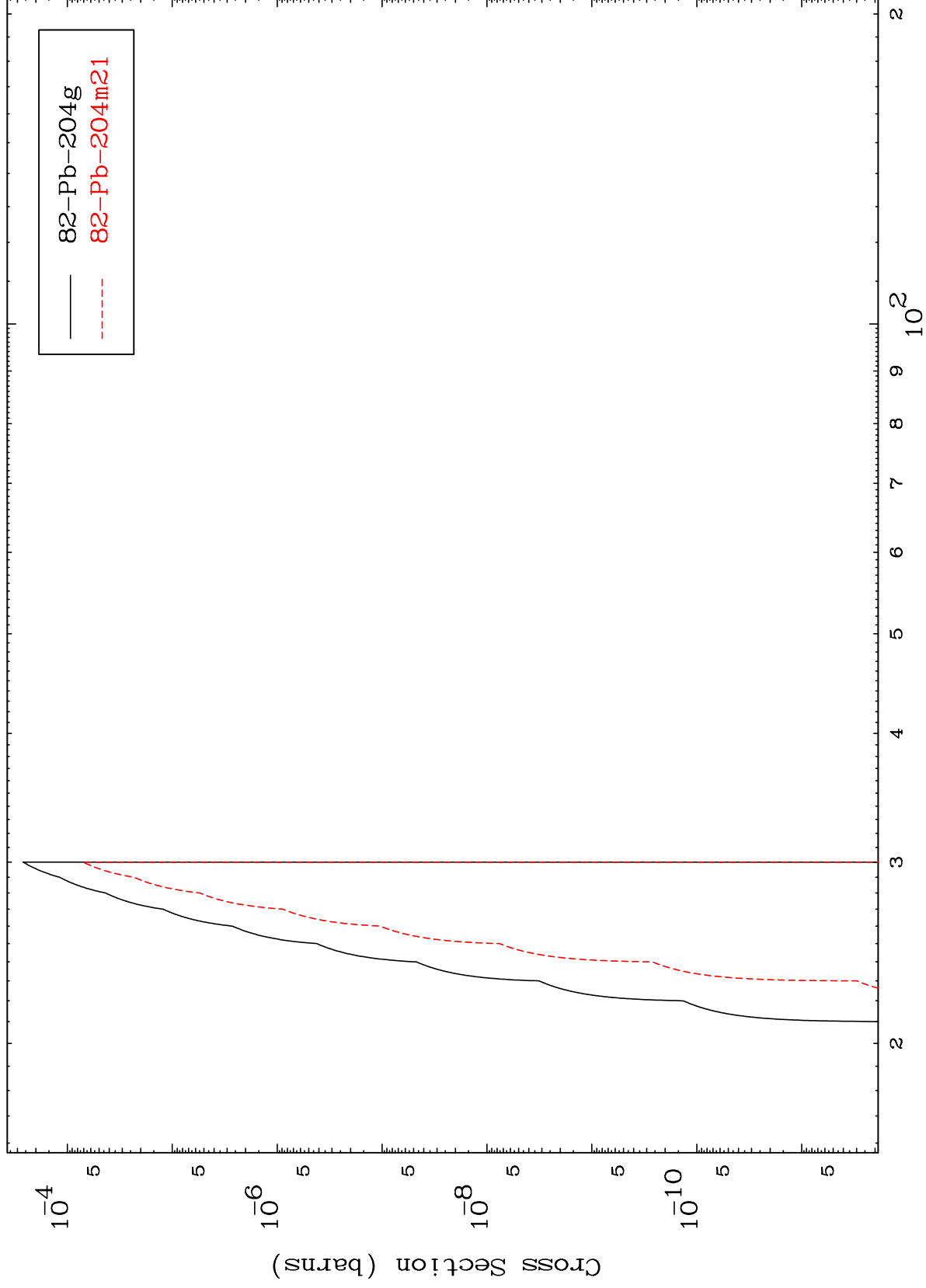








(n,2n) d
Radionuclide Production Cross Section

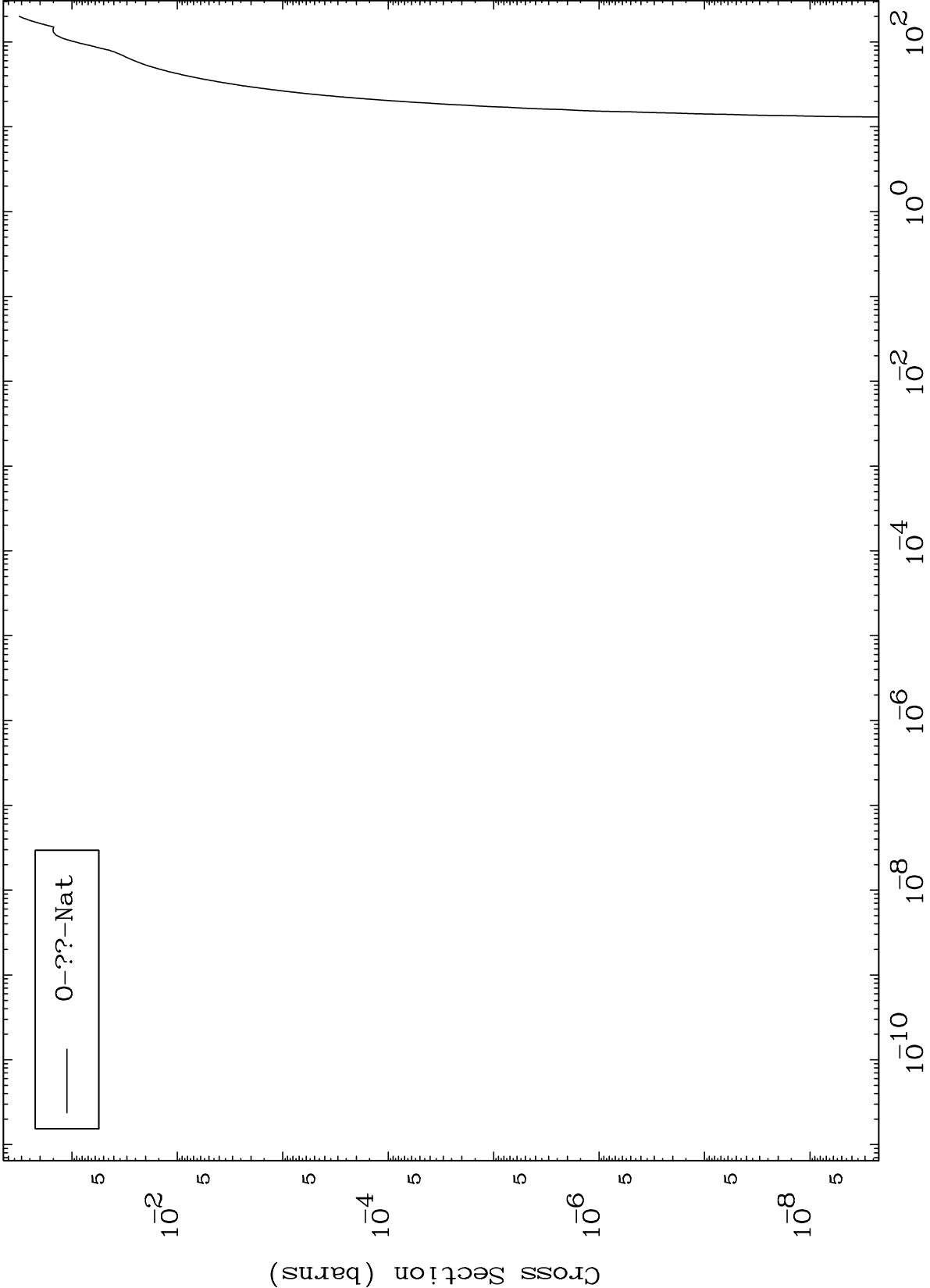


MAT 8319

Fission

83-Bi-207

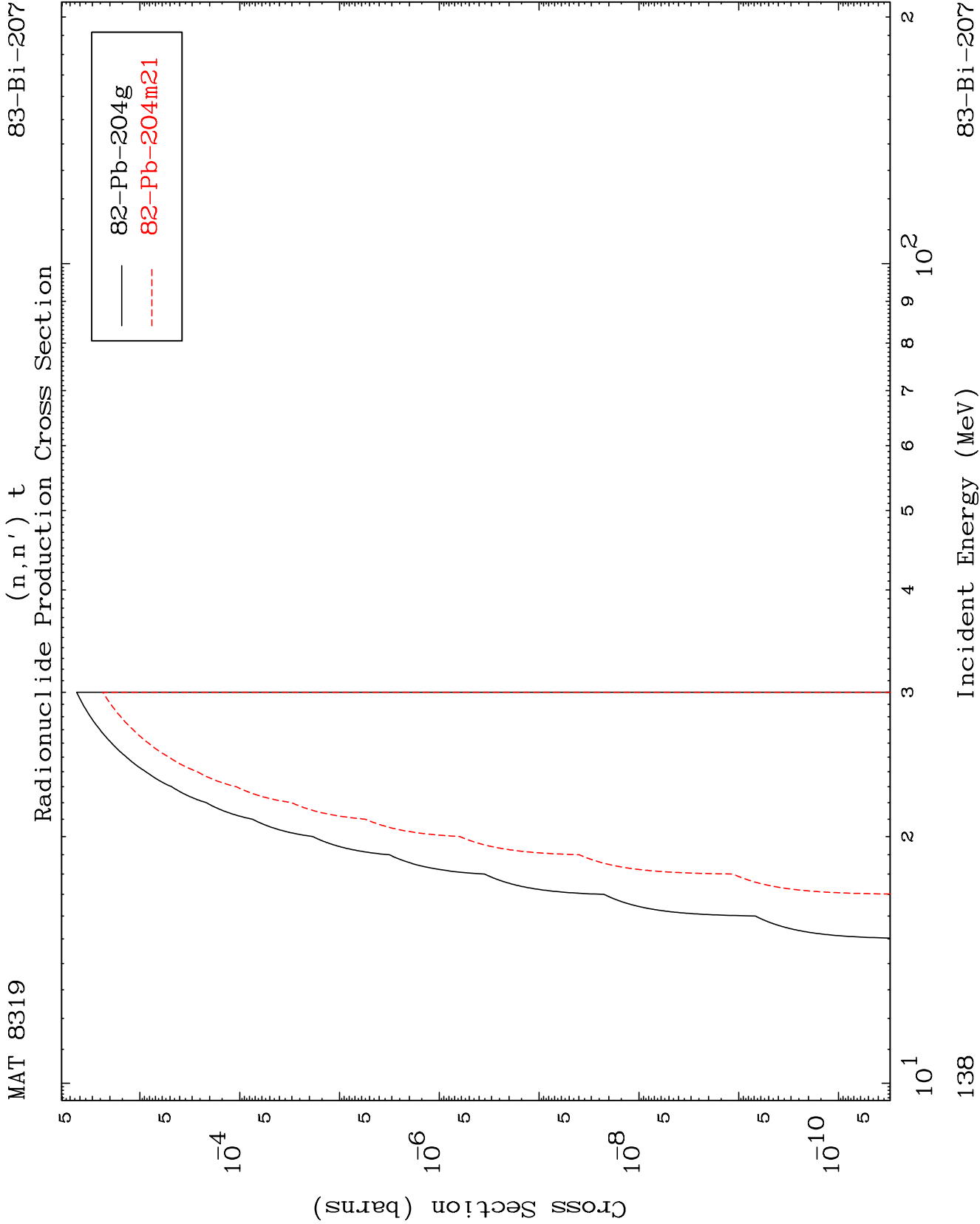
Radionuclide Production Cross Section



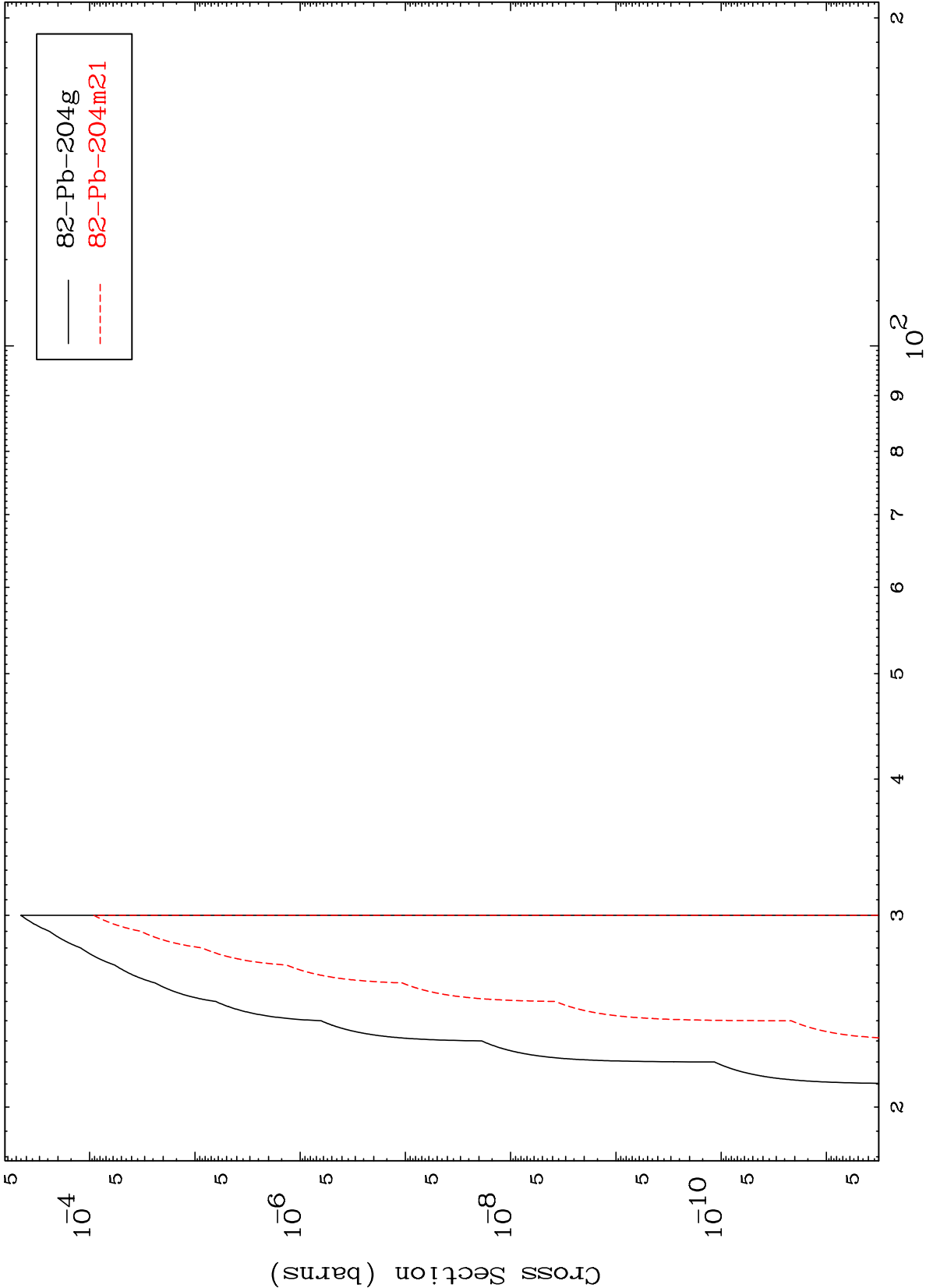
137

Incident Energy (MeV)

83-Bi-207



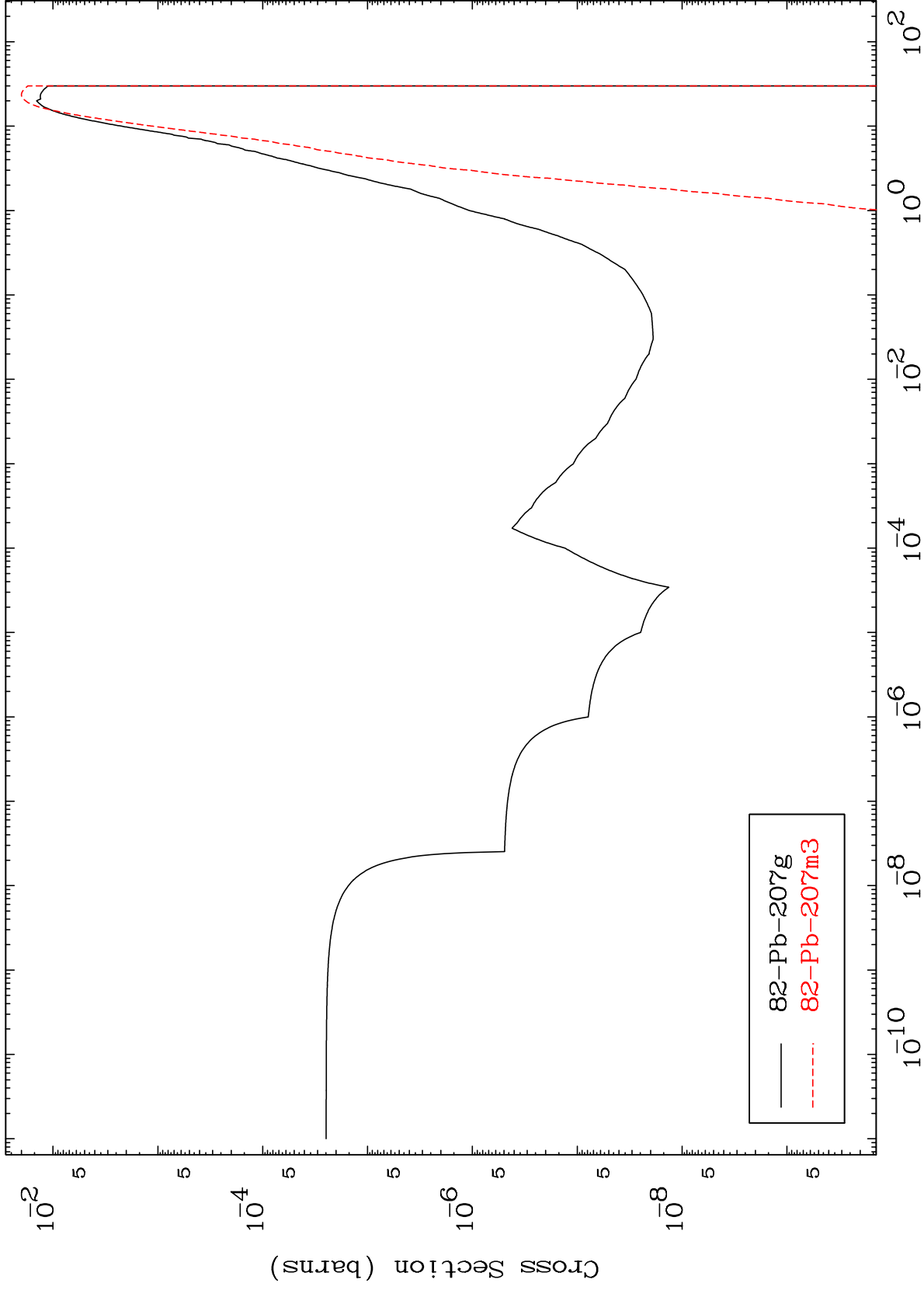
Radionuclide Production Cross Section



MAT 8319

83-Bi-207

(n,p)
Radionuclide Production Cross Section



83-Bi-207

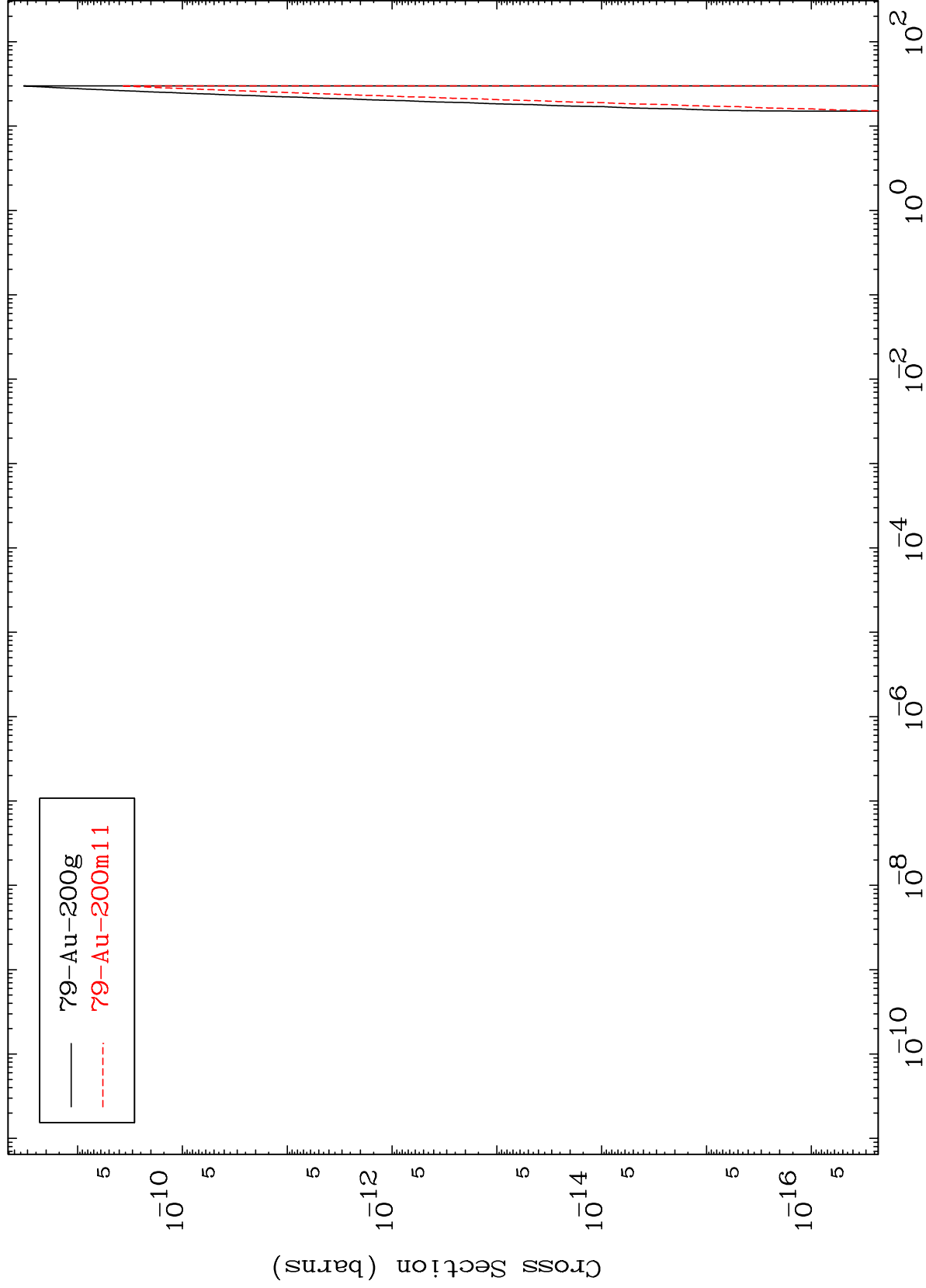
Incident Energy (MeV)

140

MAT 8319

83-Bi-207

(n,2α)
Radionuclide Production Cross Section



— 79-Au-200g
- - - 79-Au-200m11

83-Bi-207

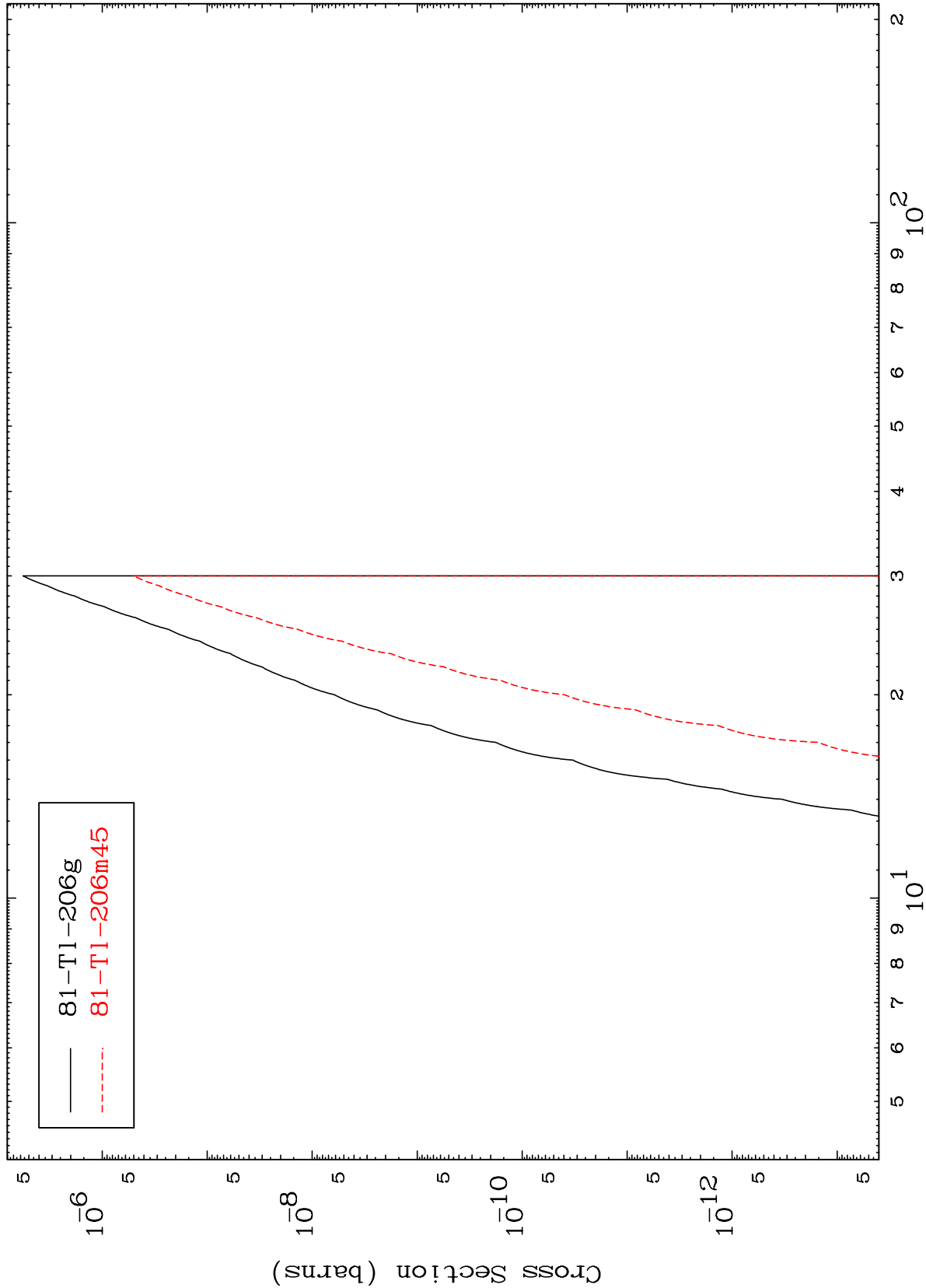
Incident Energy (MeV)

141

MAT 8319

83-Bi-207

(n,2p)
Radionuclide Production Cross Section



142

Incident Energy (MeV)

83-Bi-207