

Program EVALPLOT
(Version 2021-1)

by

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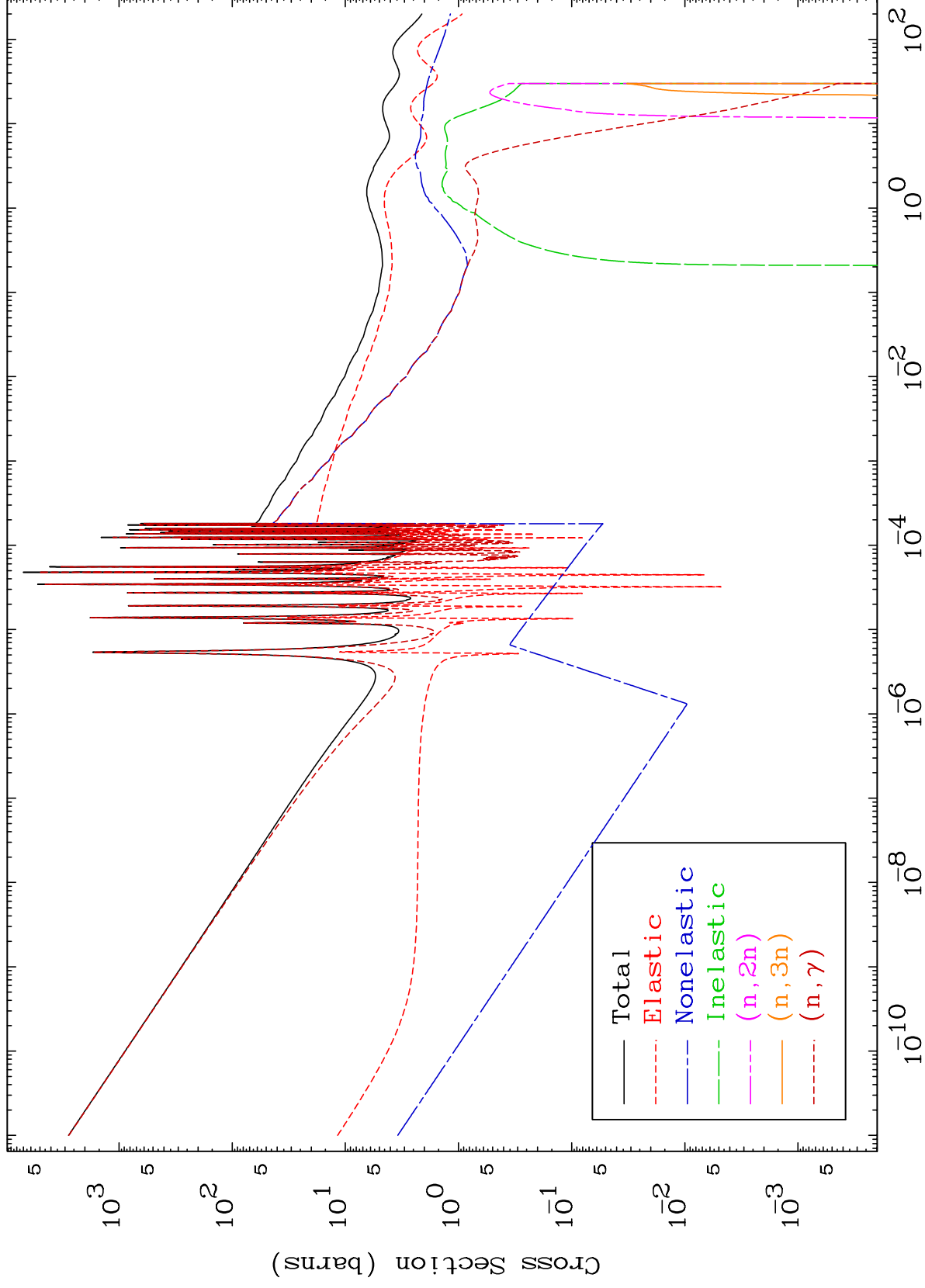
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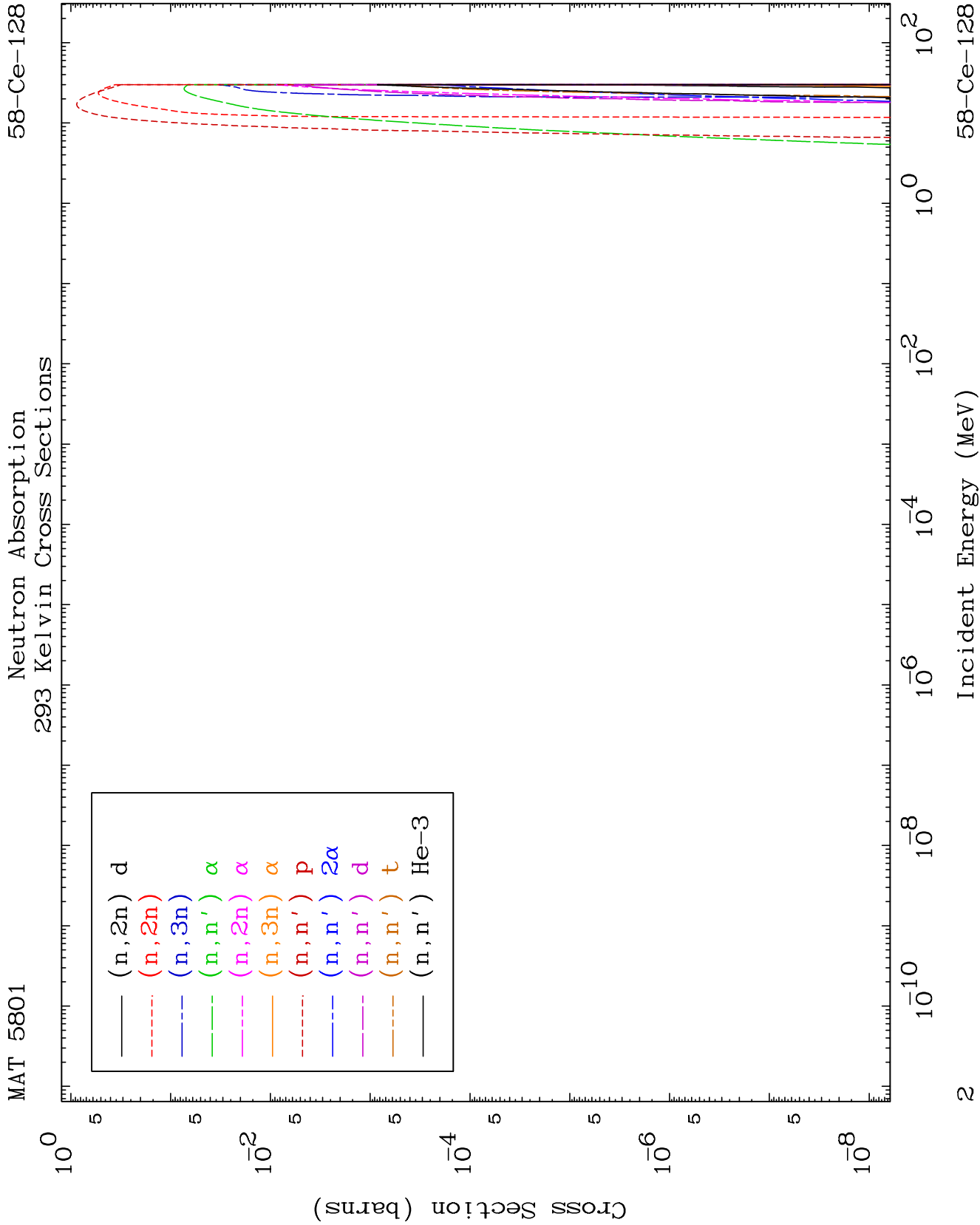
Press Mouse Button to Start

MAT 5801

Neutron Major
293 Kelvin Cross Sections

58-Ce-128

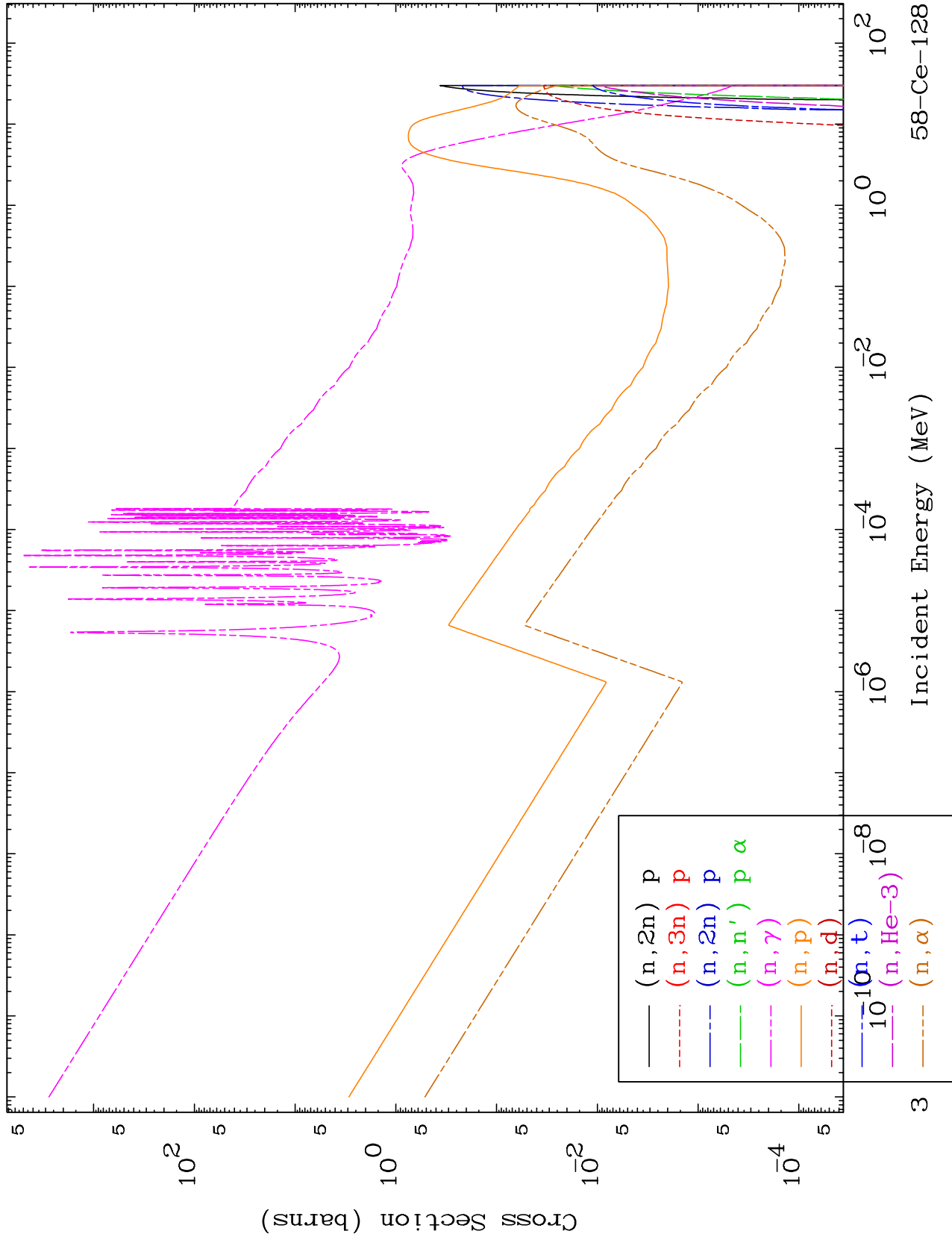


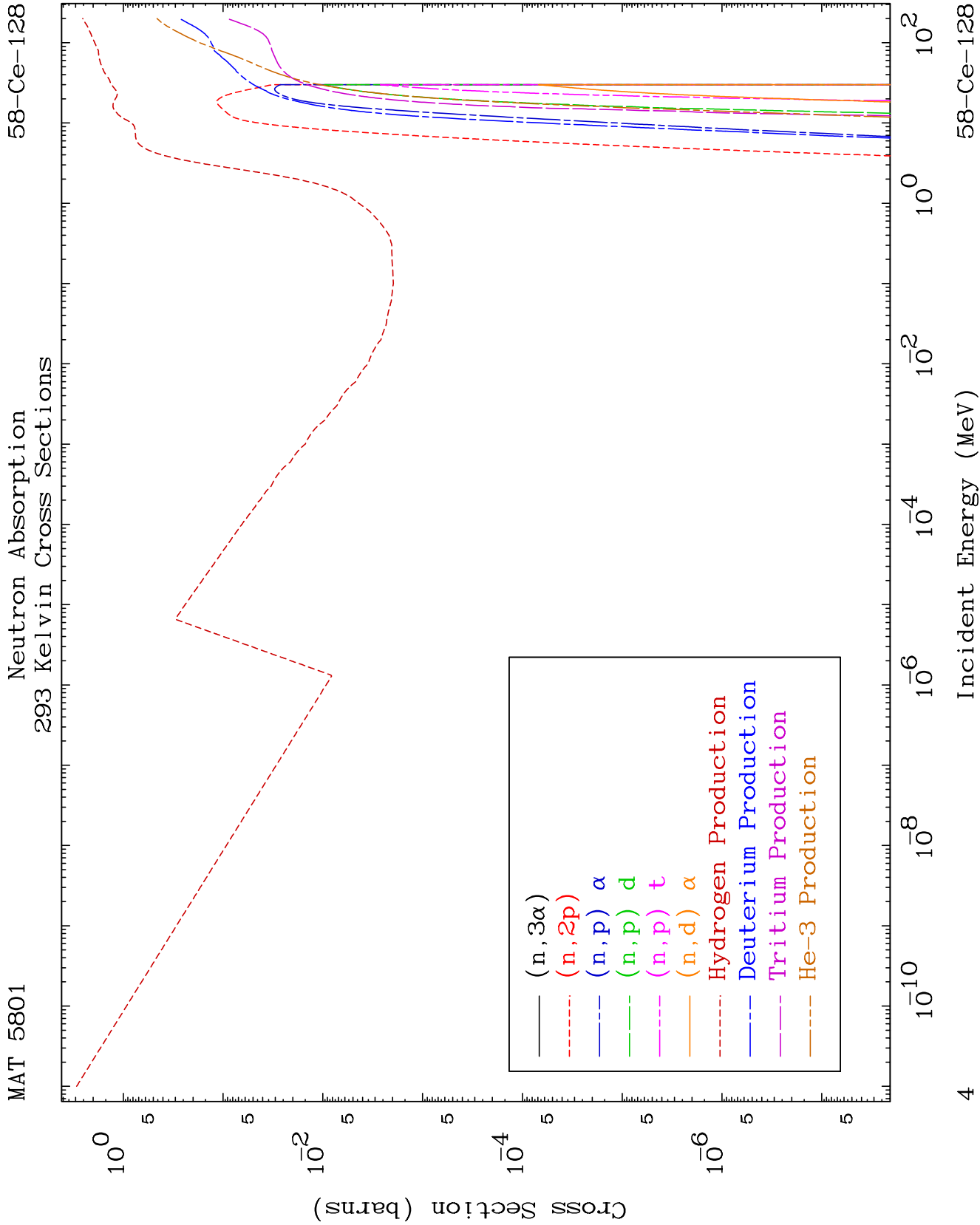


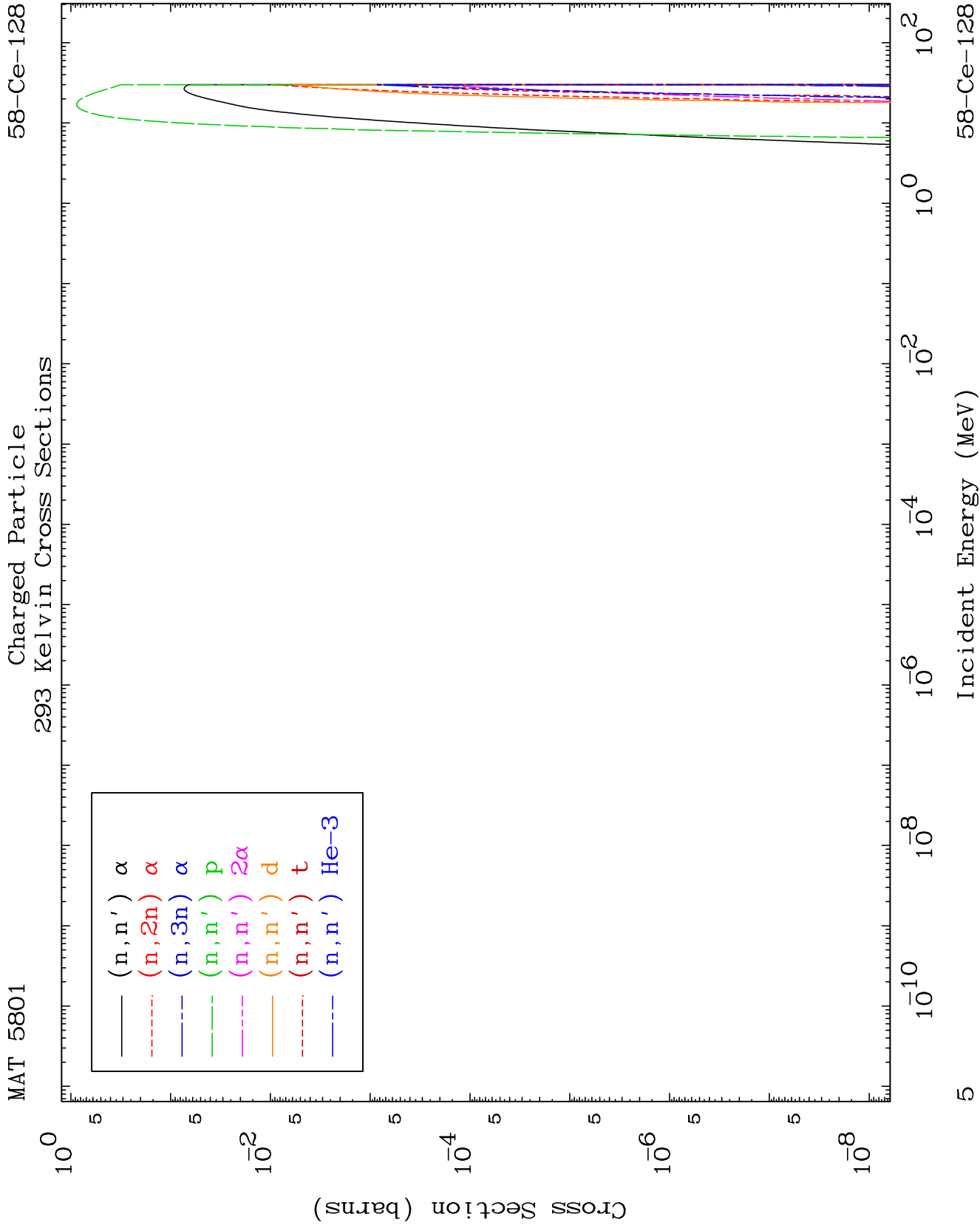
MAT 5801

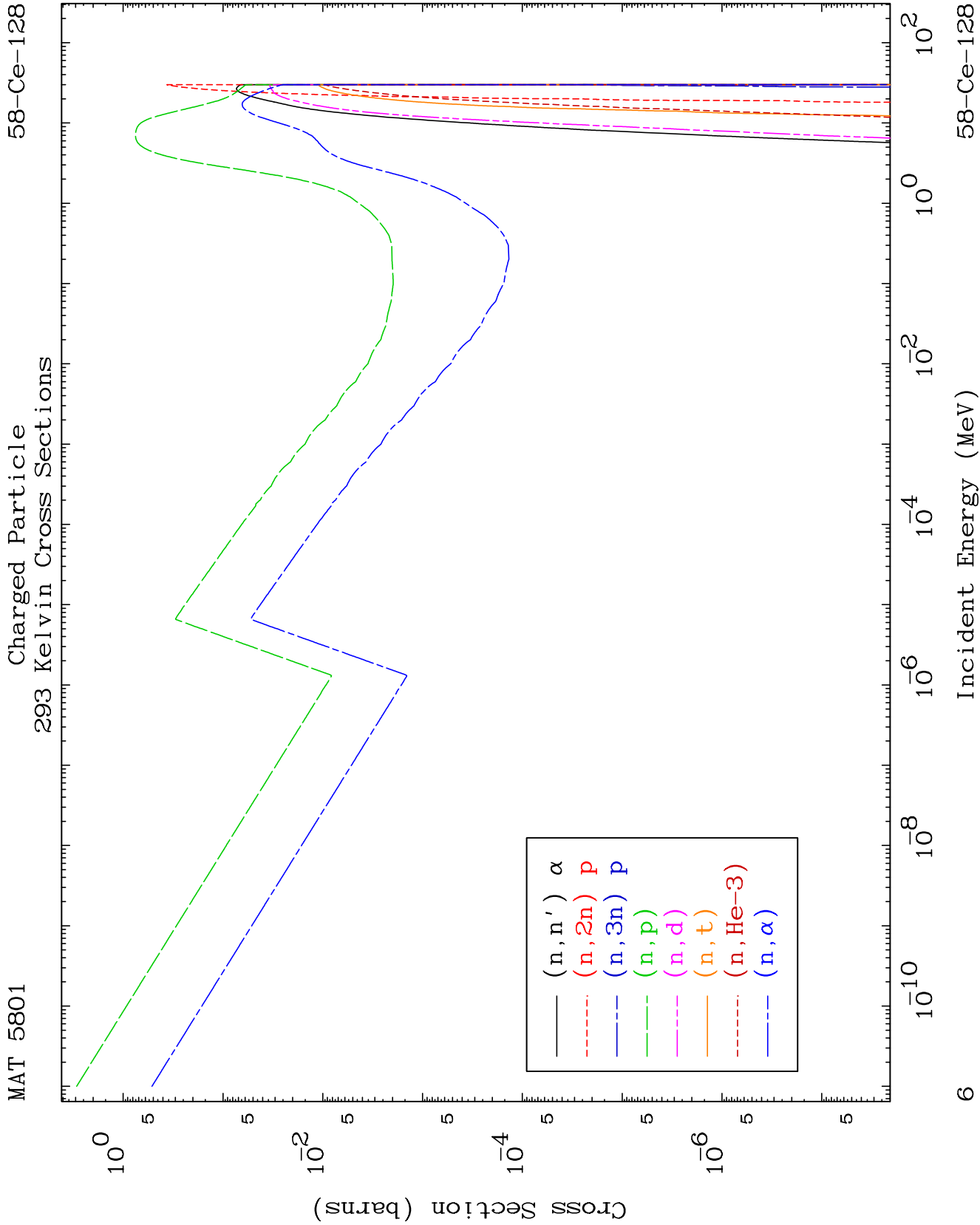
Neutron Absorption
293 Kelvin Cross Sections

58-Ce-128





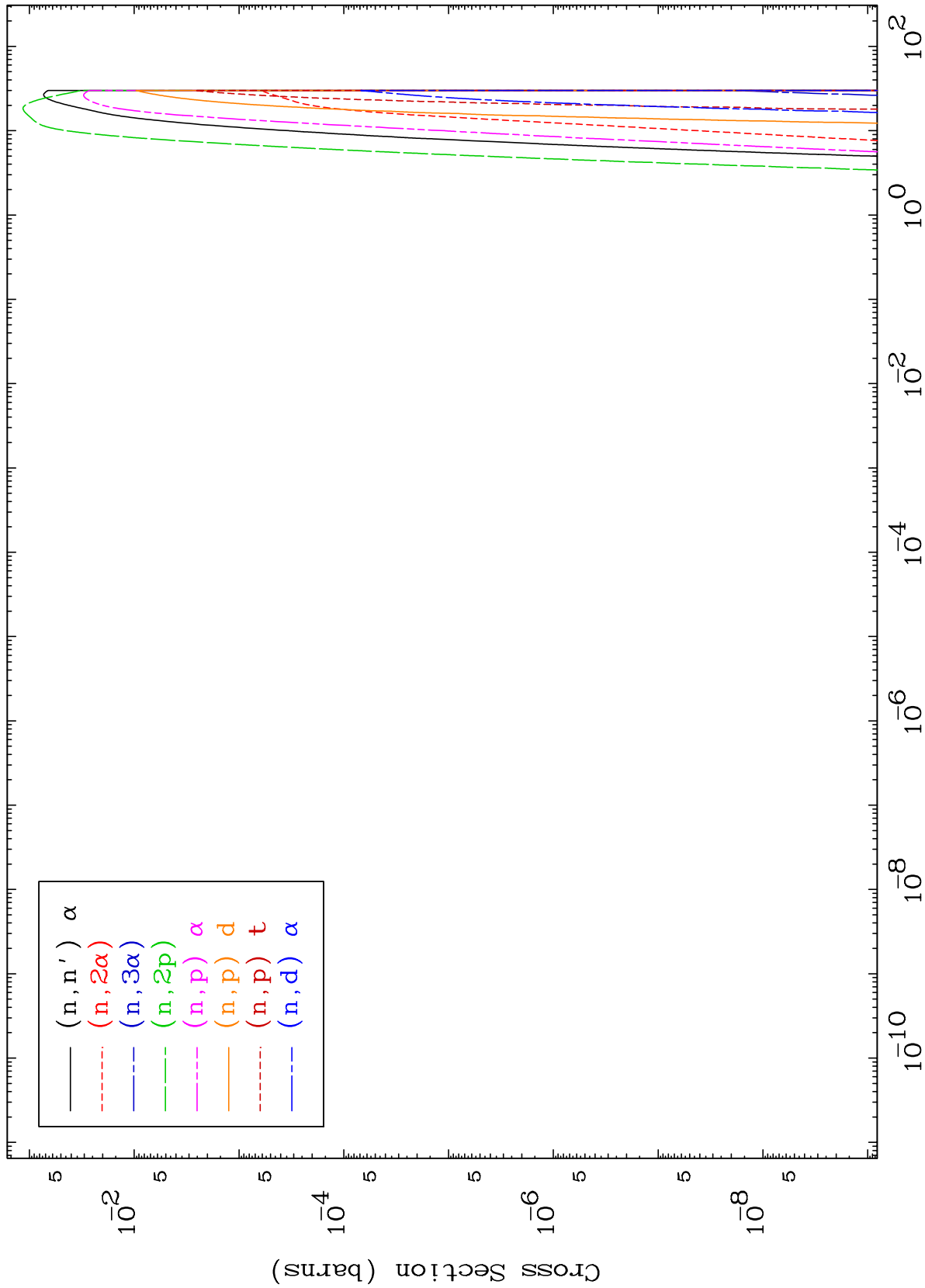


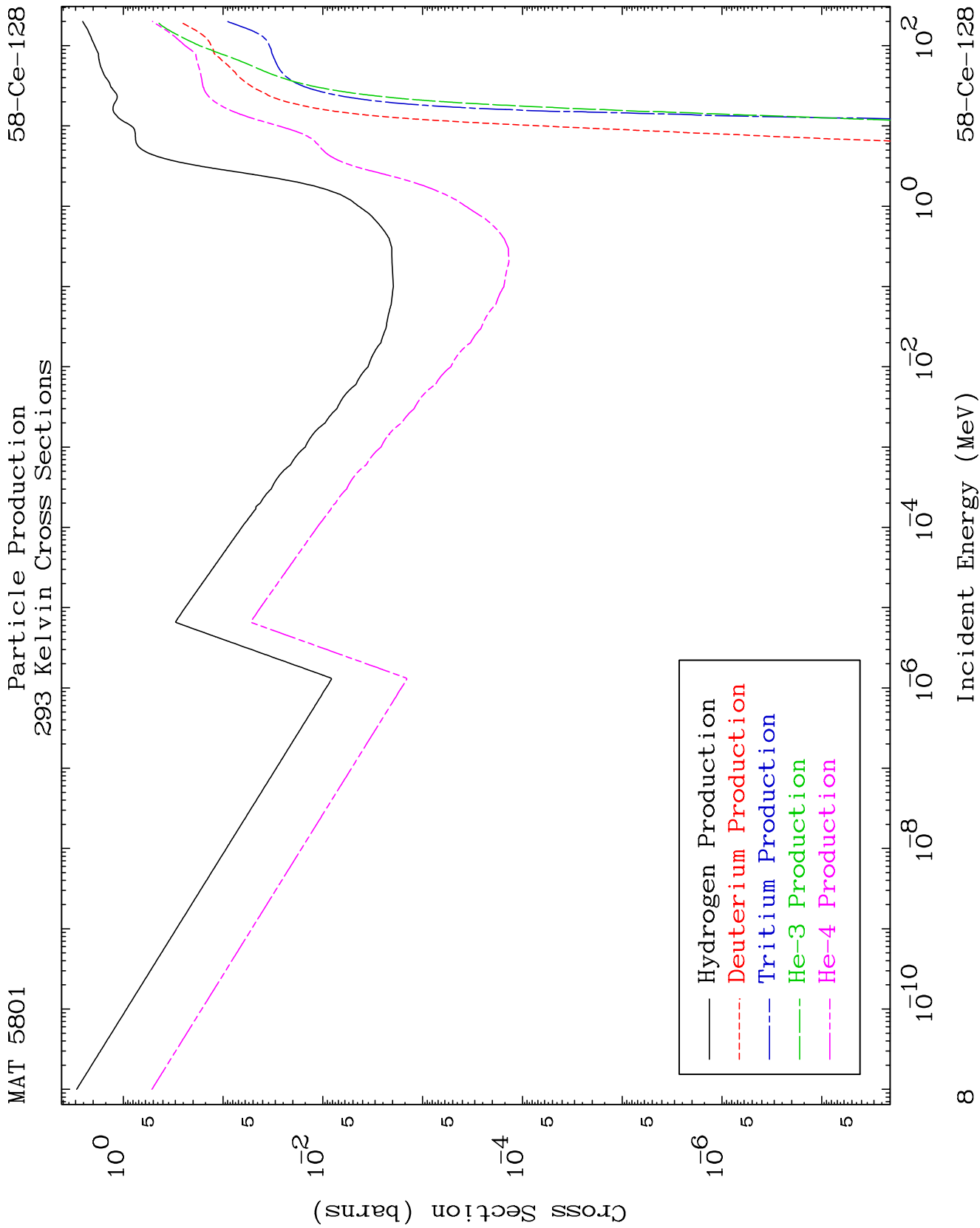


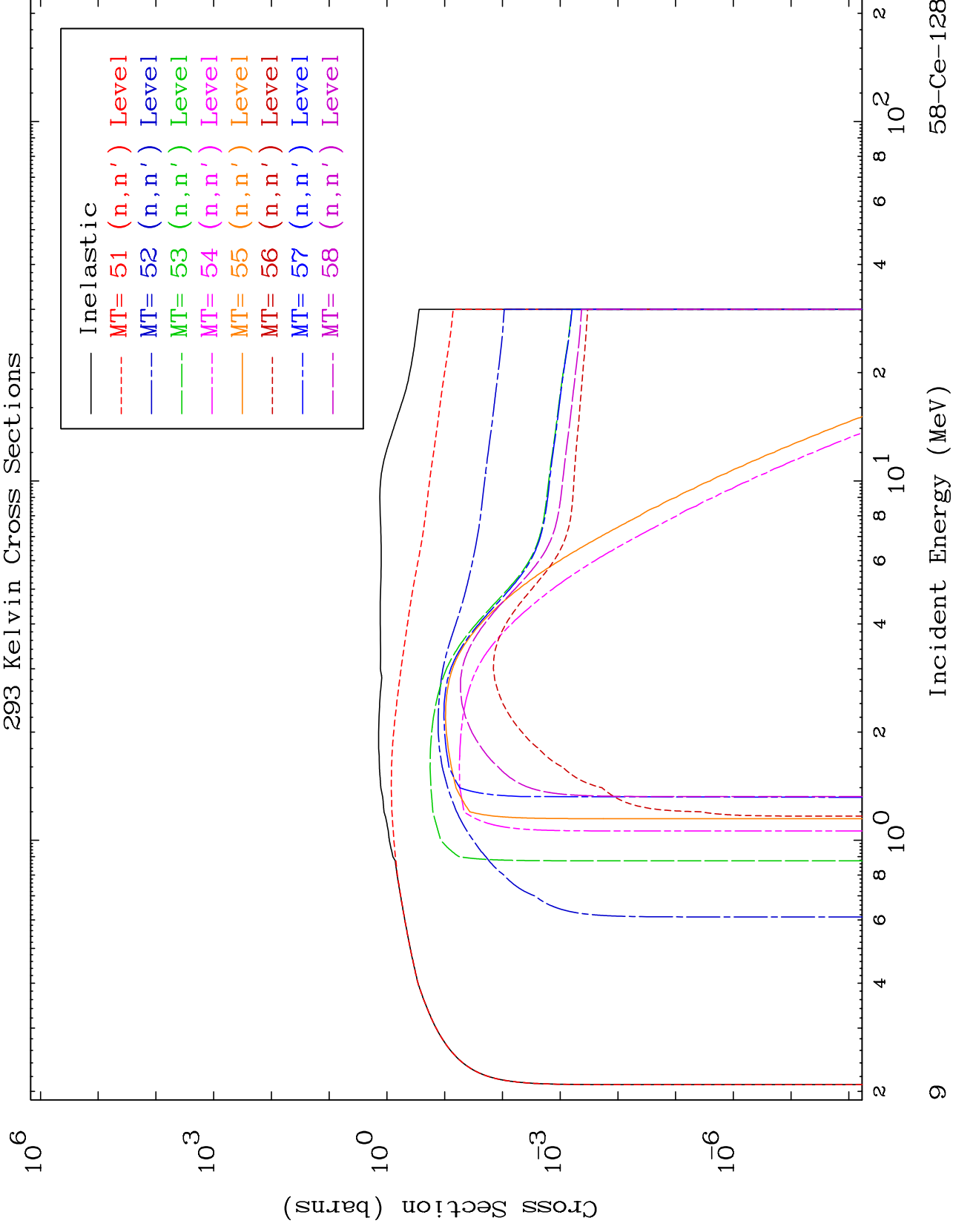
MAT 5801

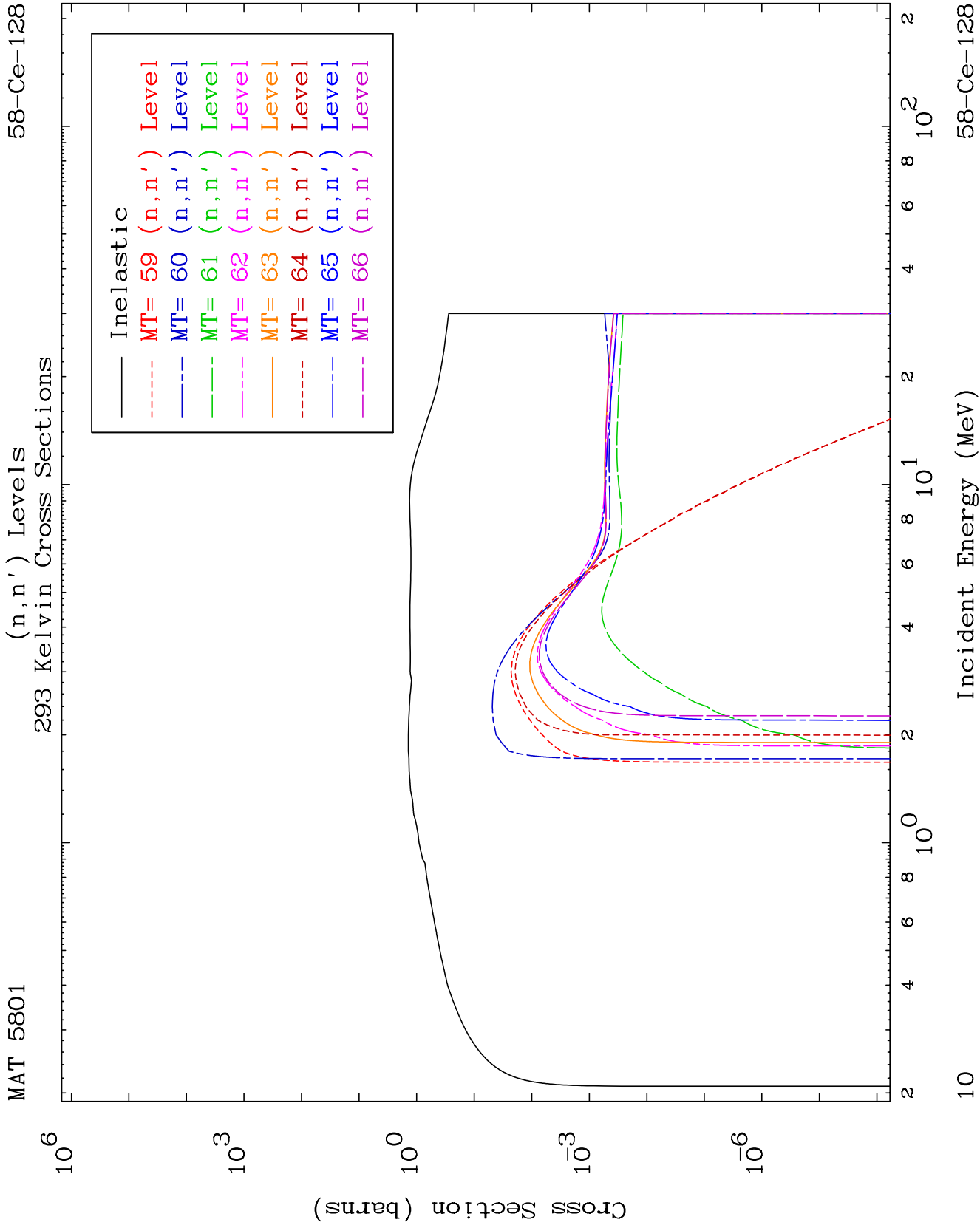
Charged Particle
293 Kelvin Cross Sections

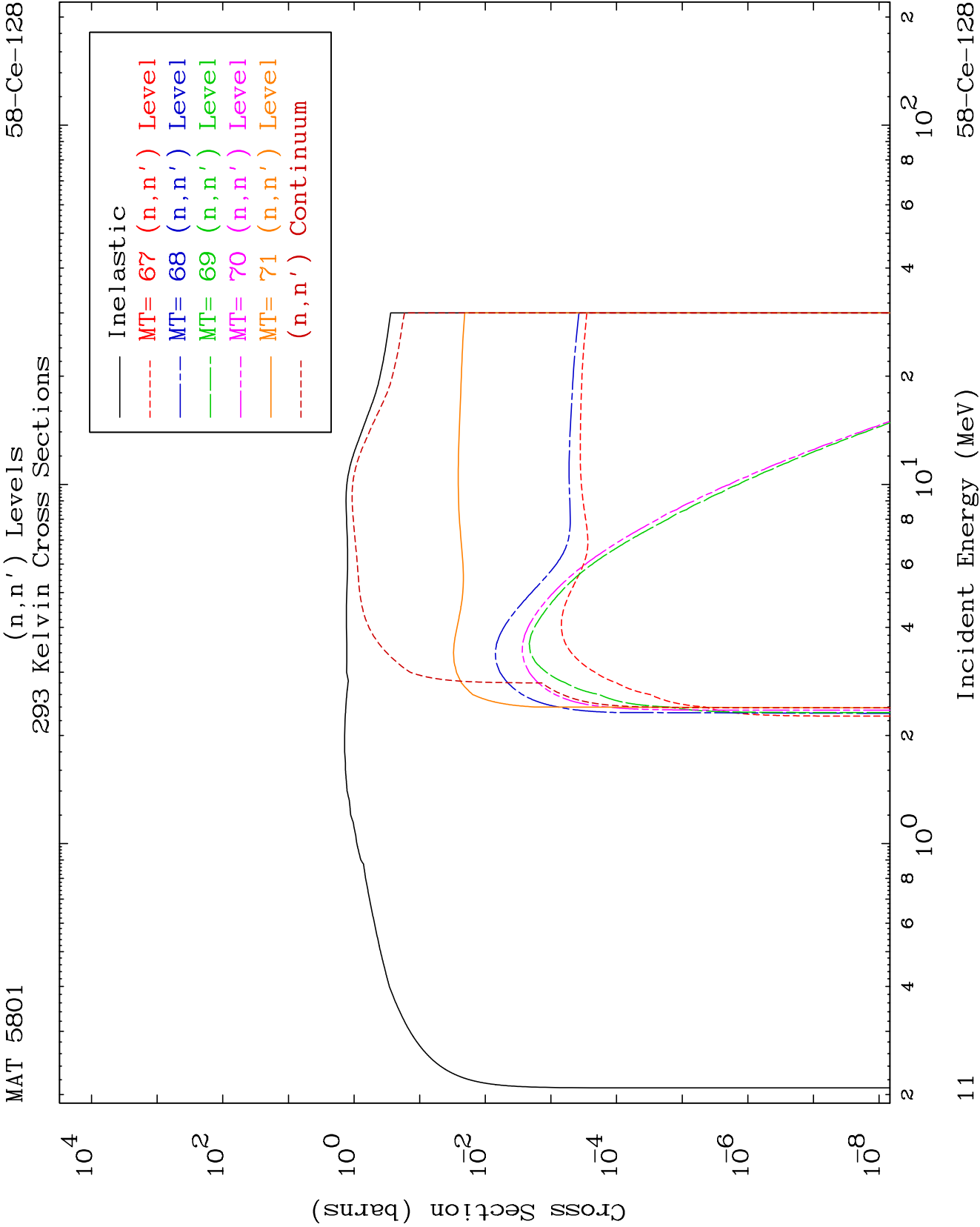
58-Ce-128

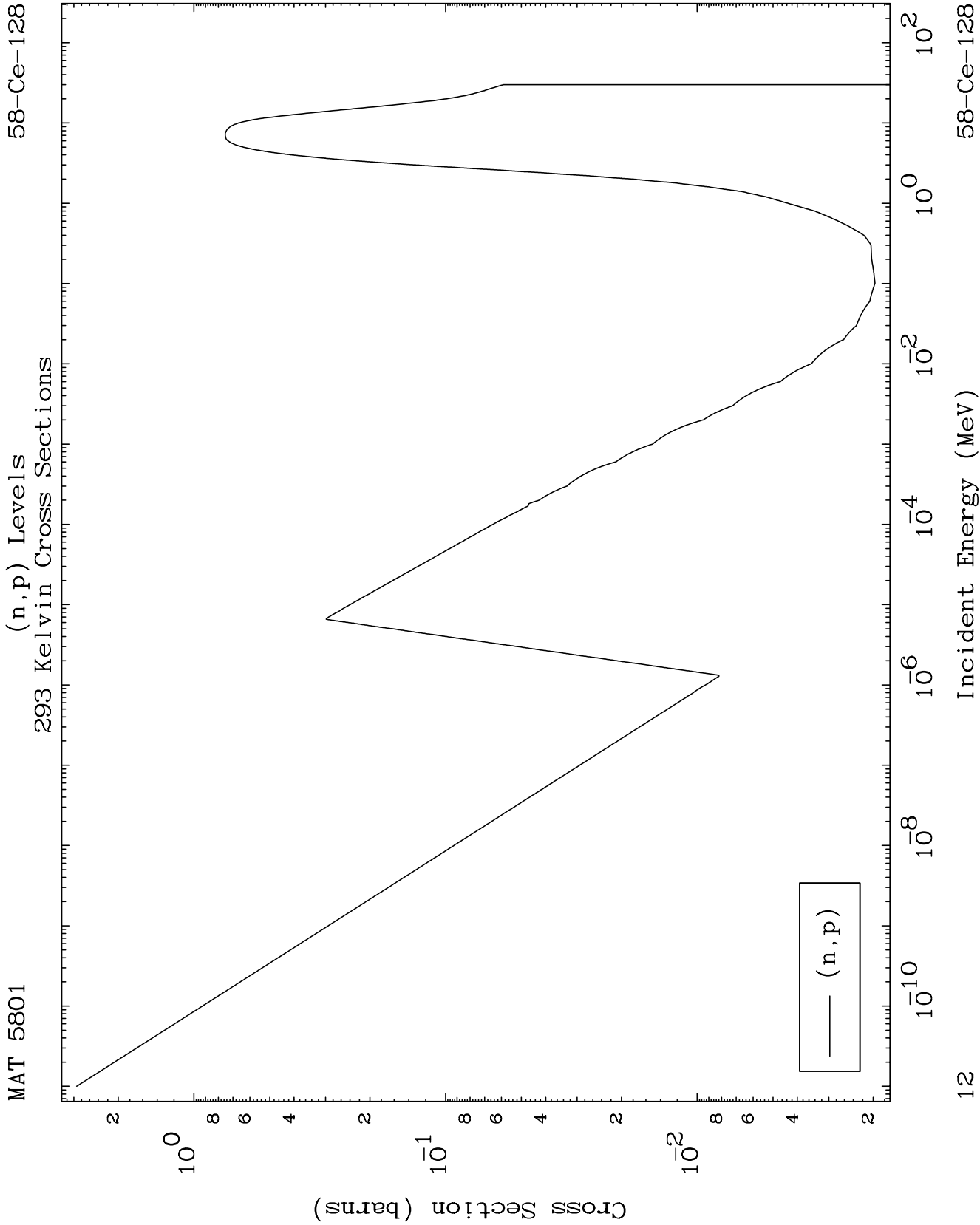








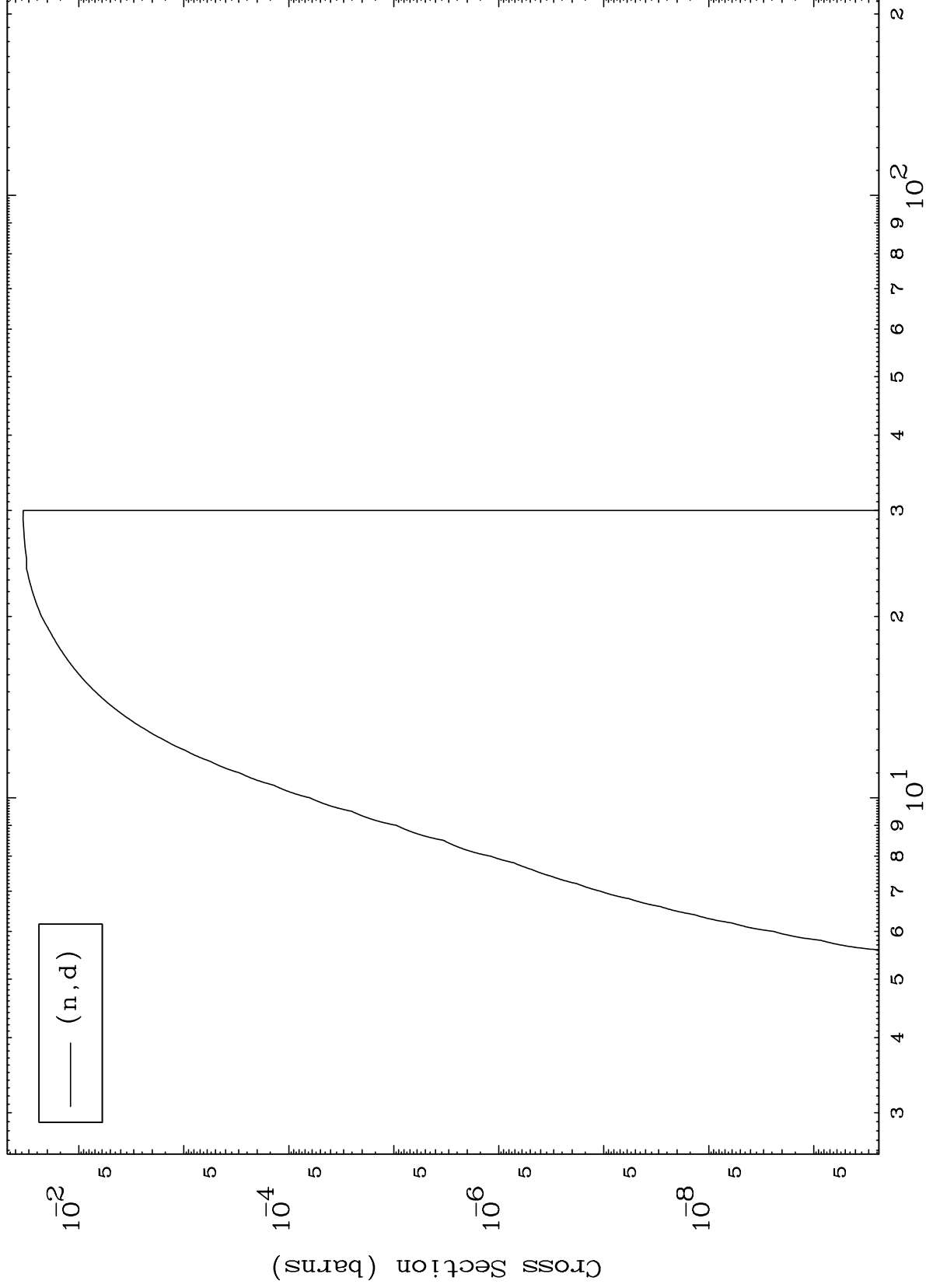




MAT 5801

58-Ce-128

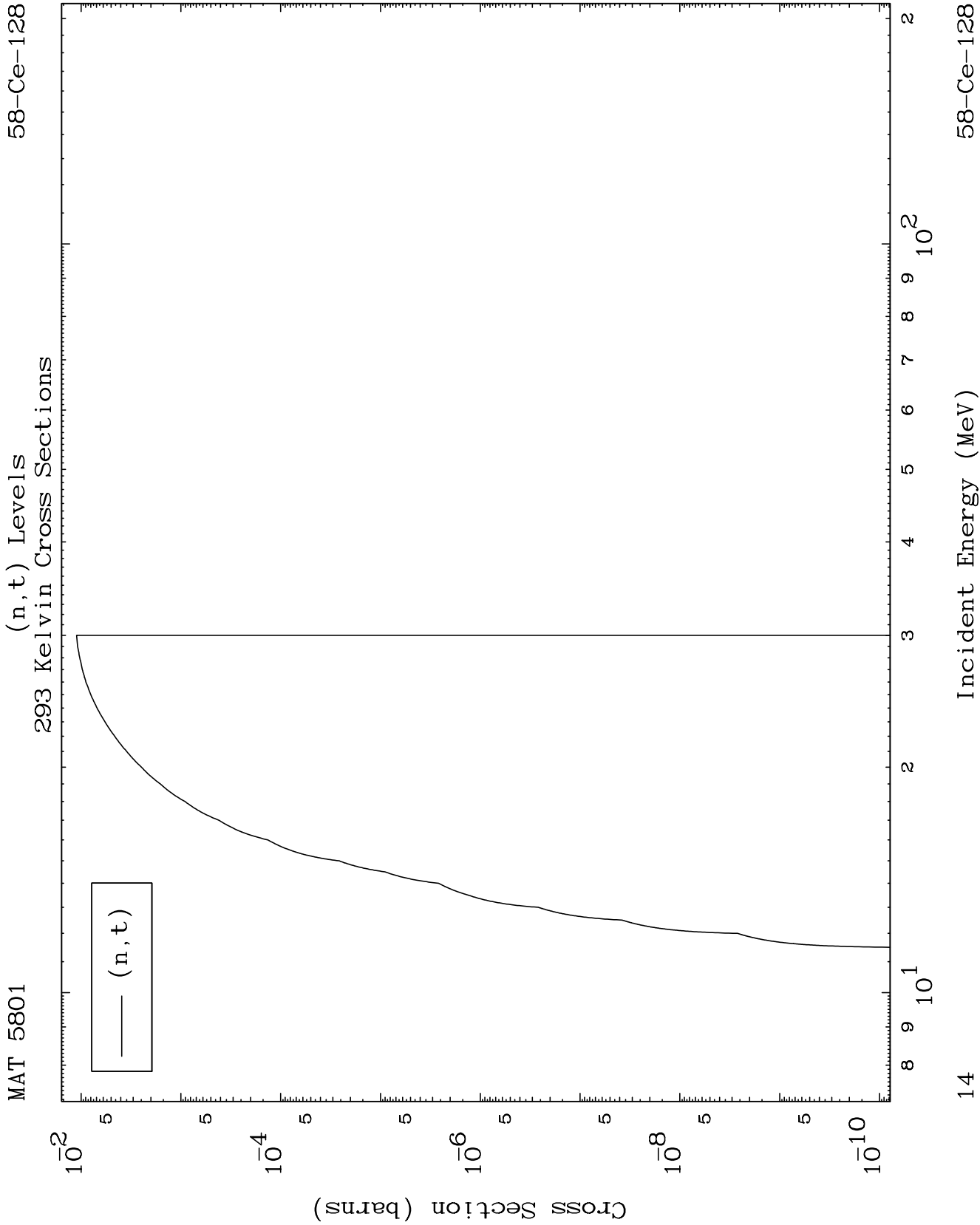
(n,d) Levels
293 Kelvin Cross Sections

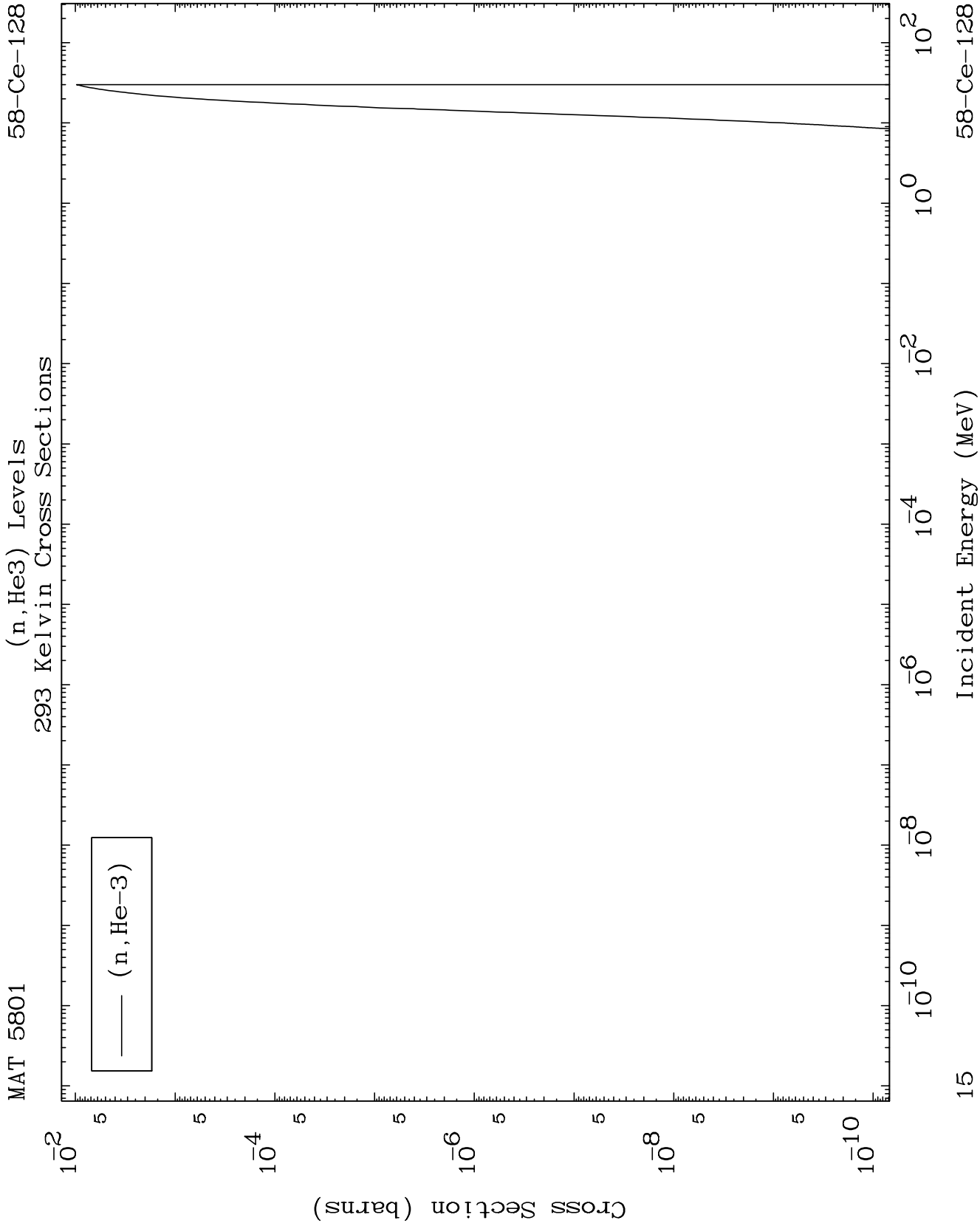


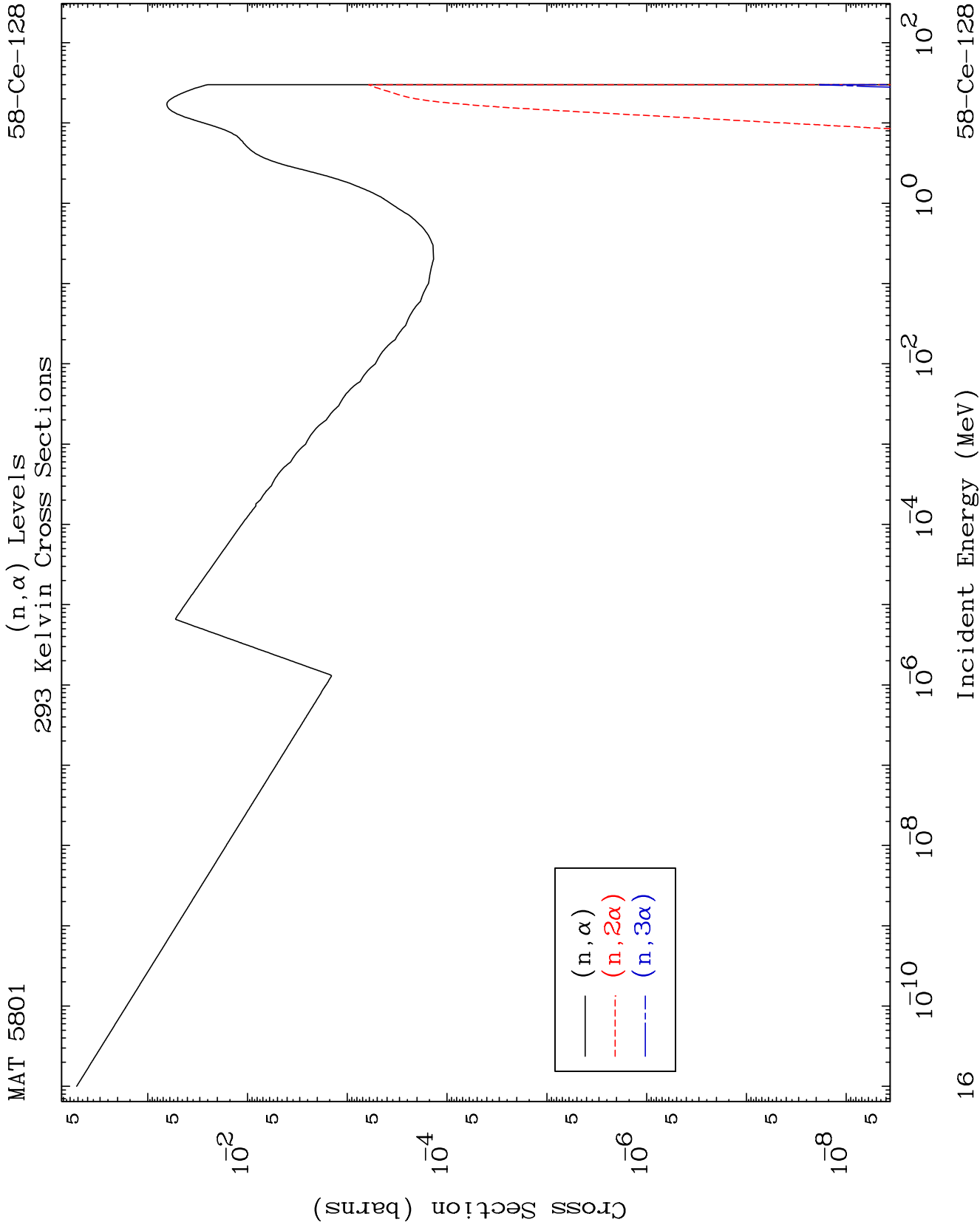
58-Ce-128

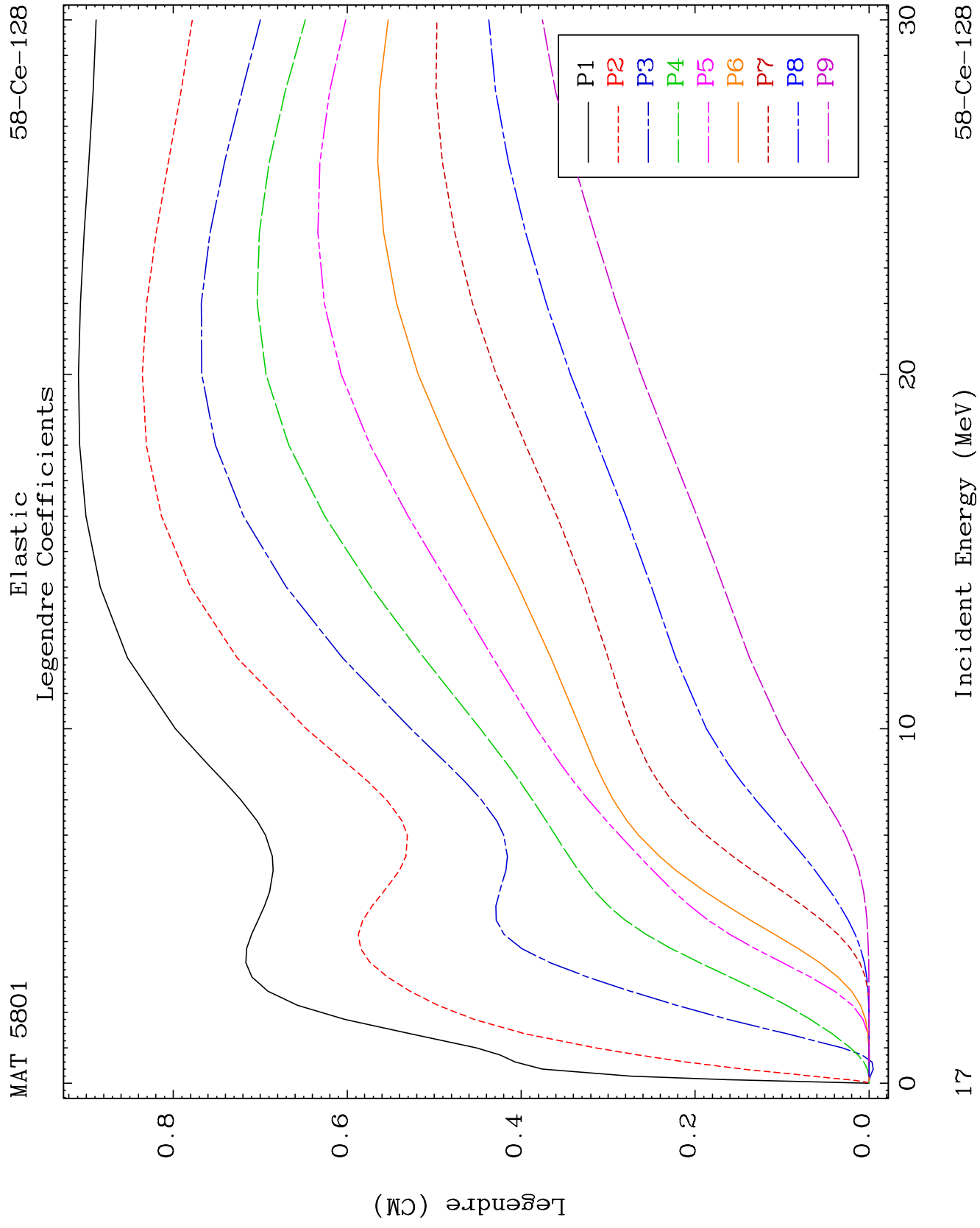
Incident Energy (MeV)

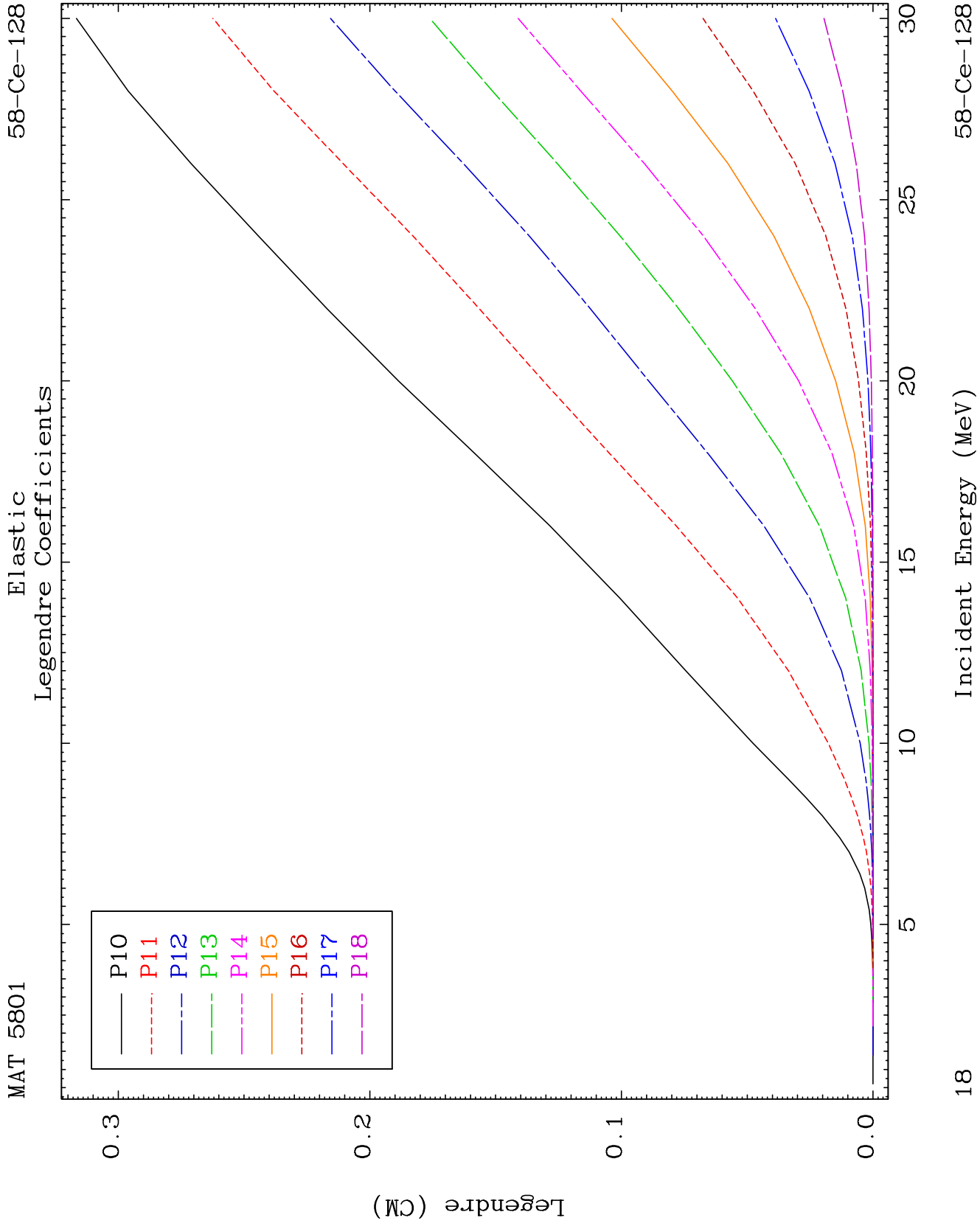
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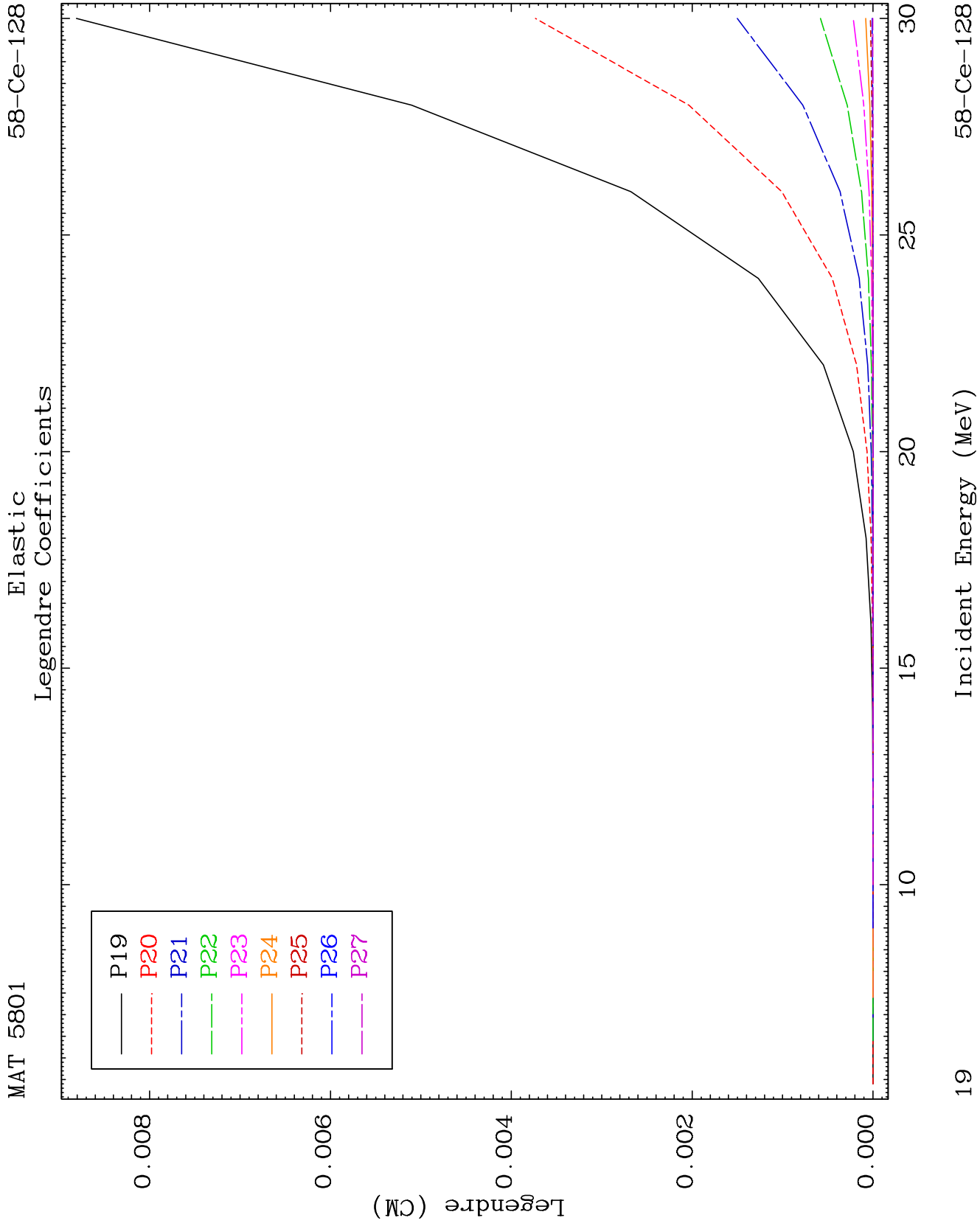


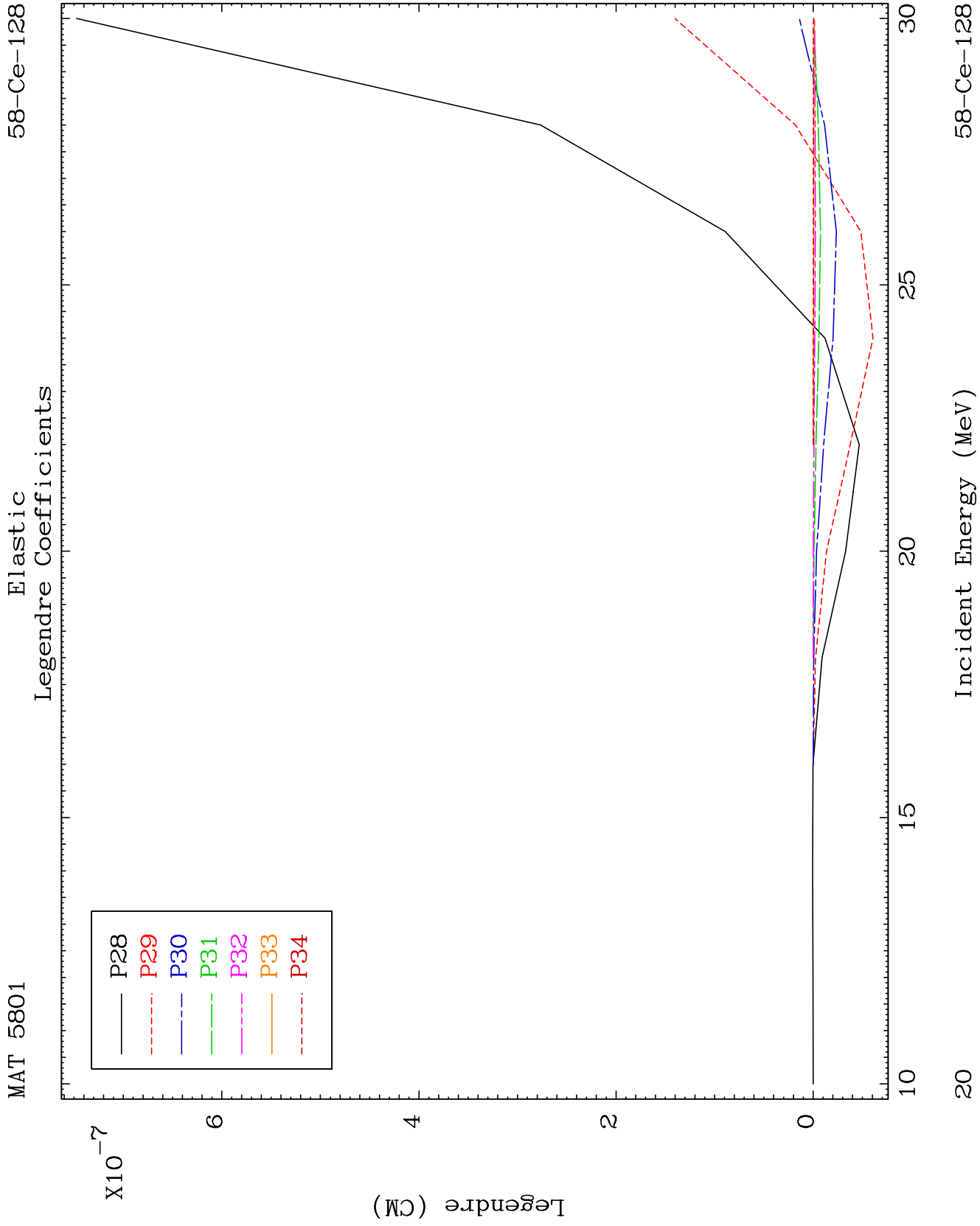








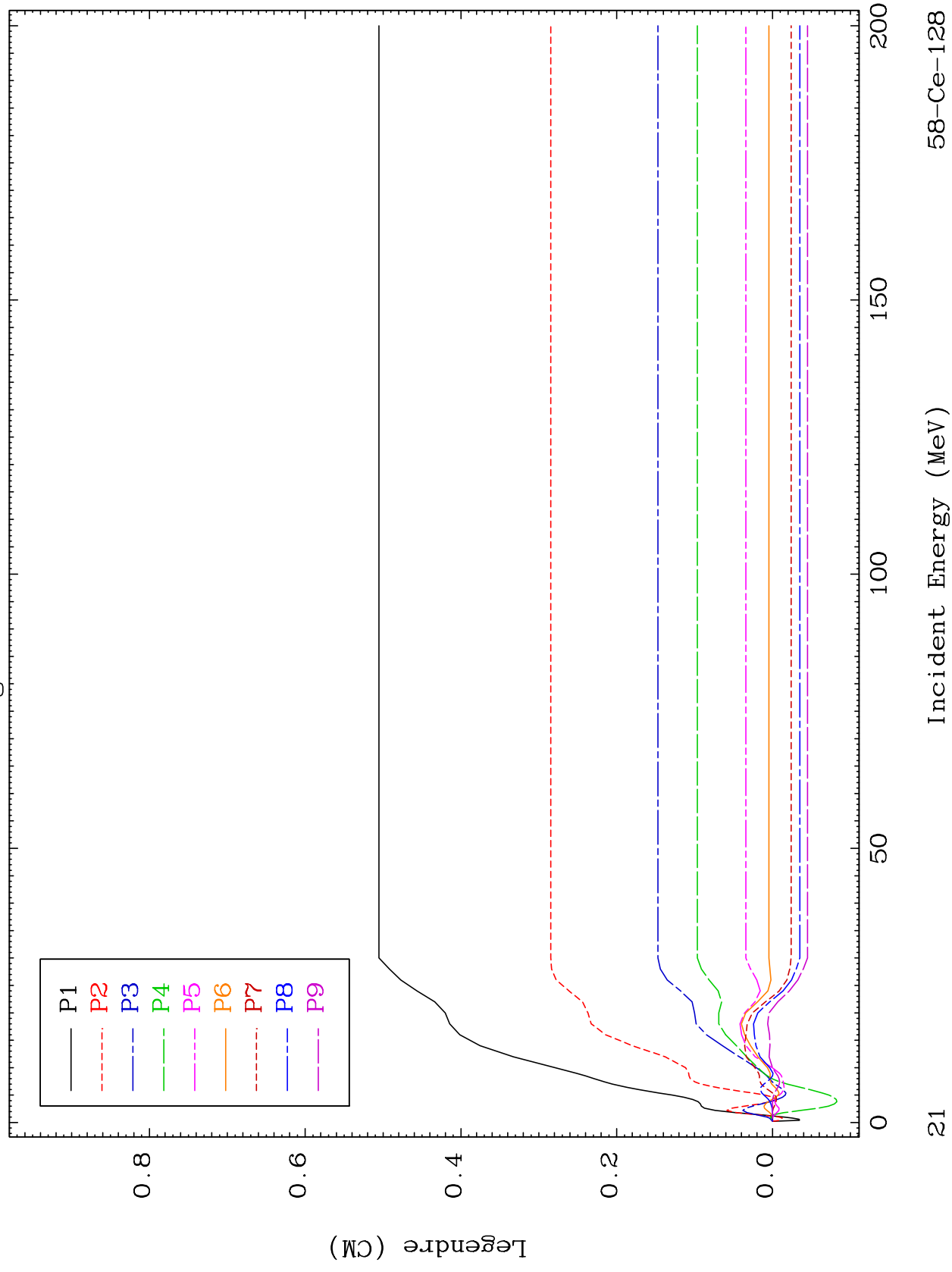


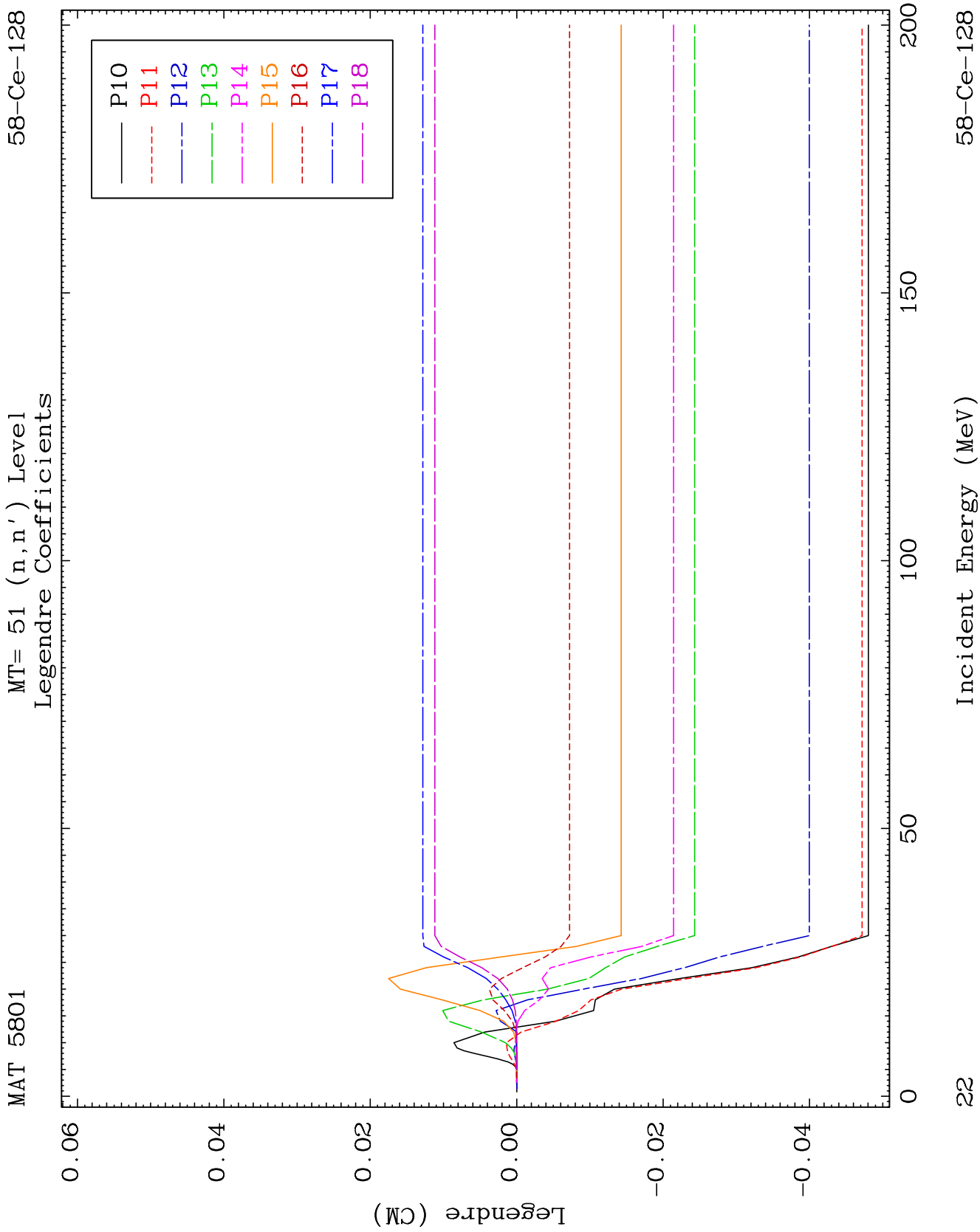


MAT 5801

MT= 51 (n, n') Level
Legendre Coefficients

58-Ce-128

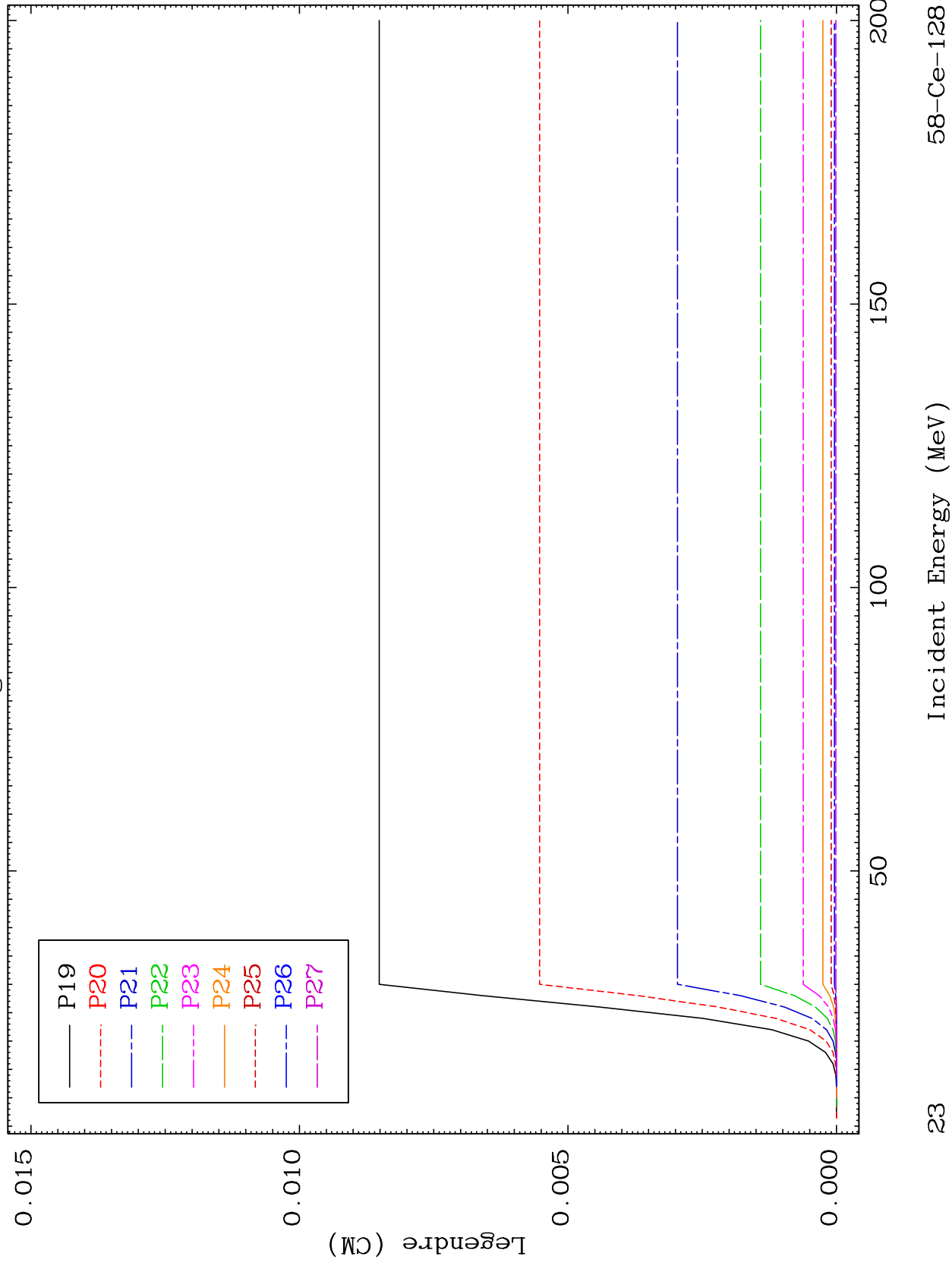


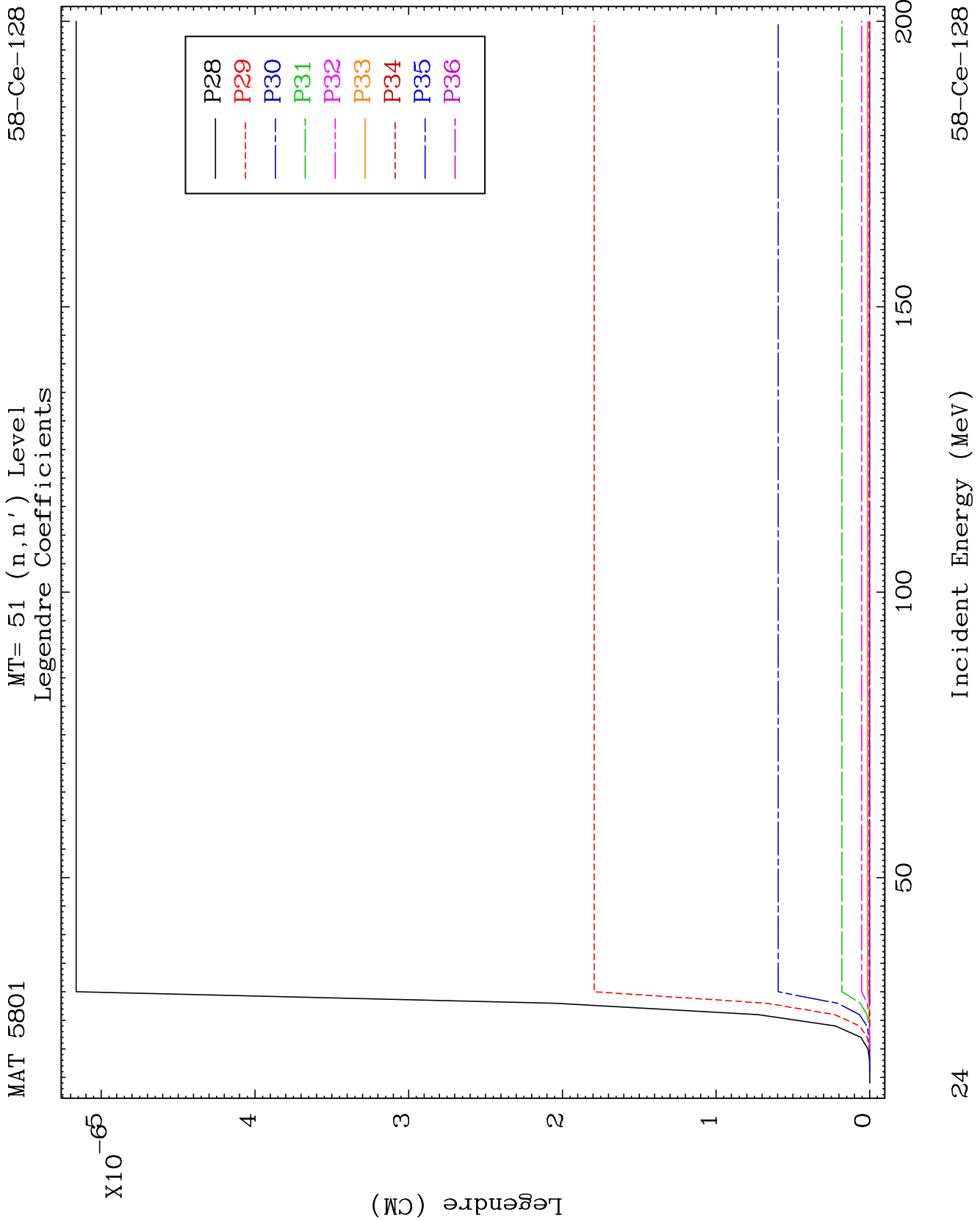


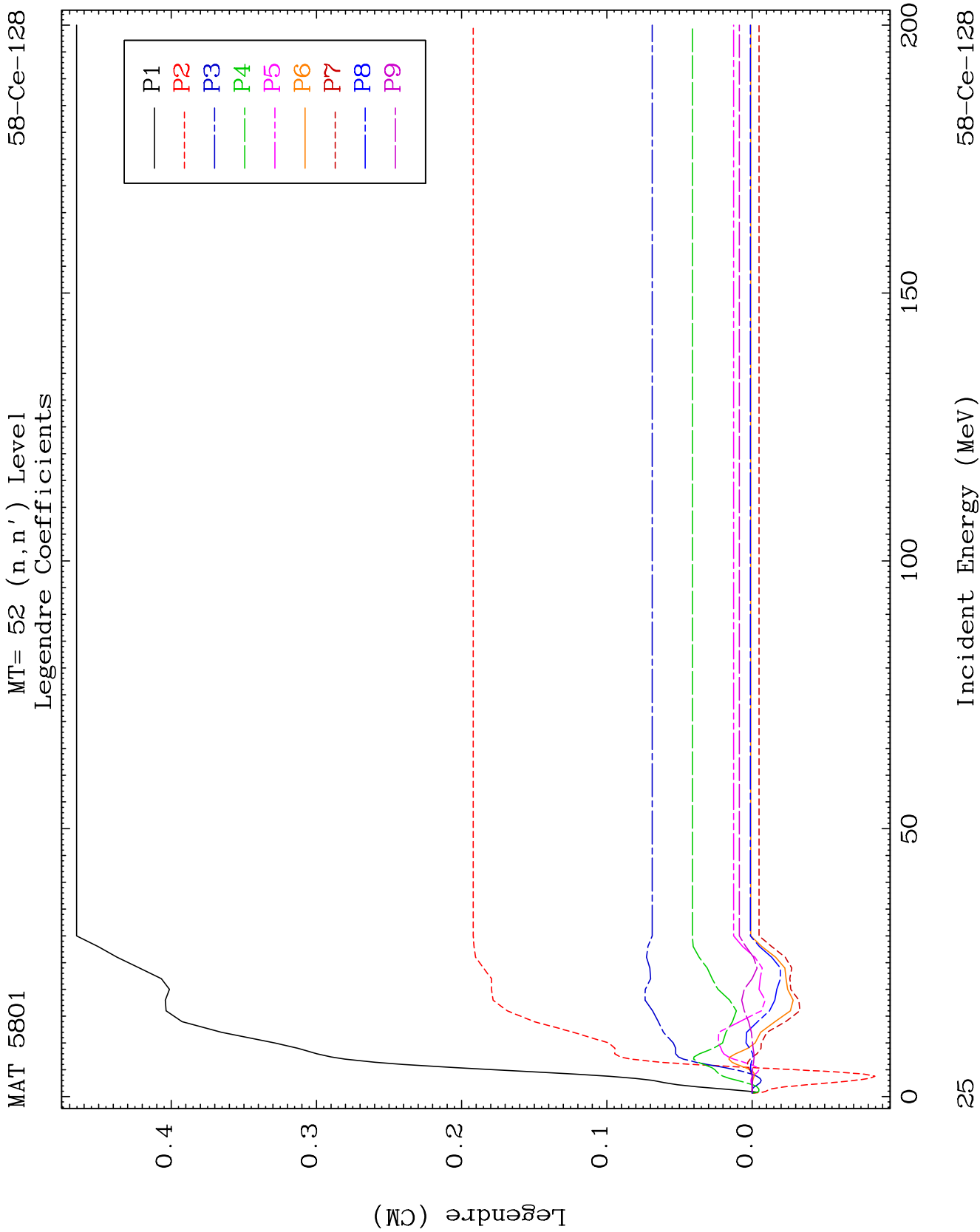
MAT 5801

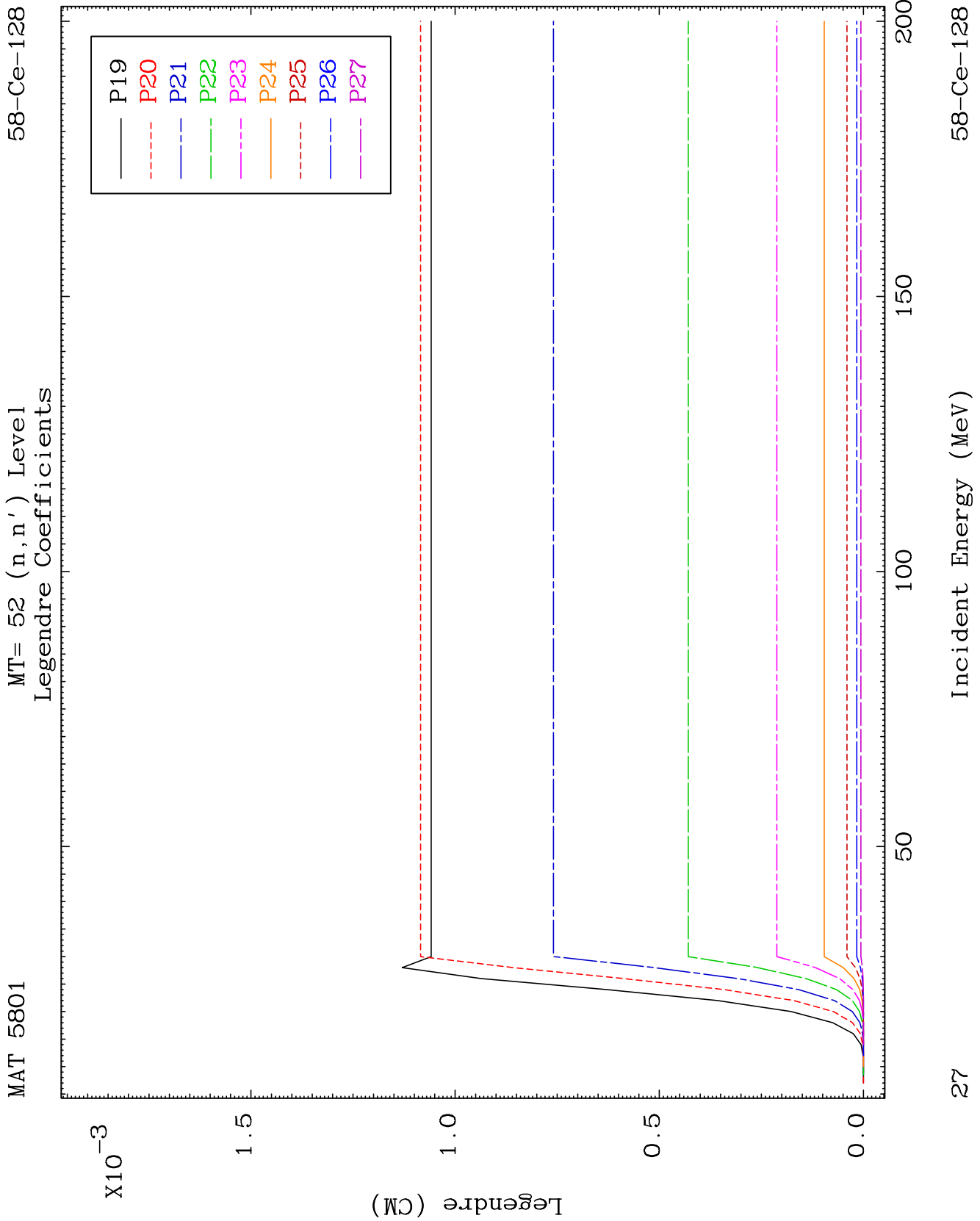
MT= 51 (n, n') Level
Legendre Coefficients

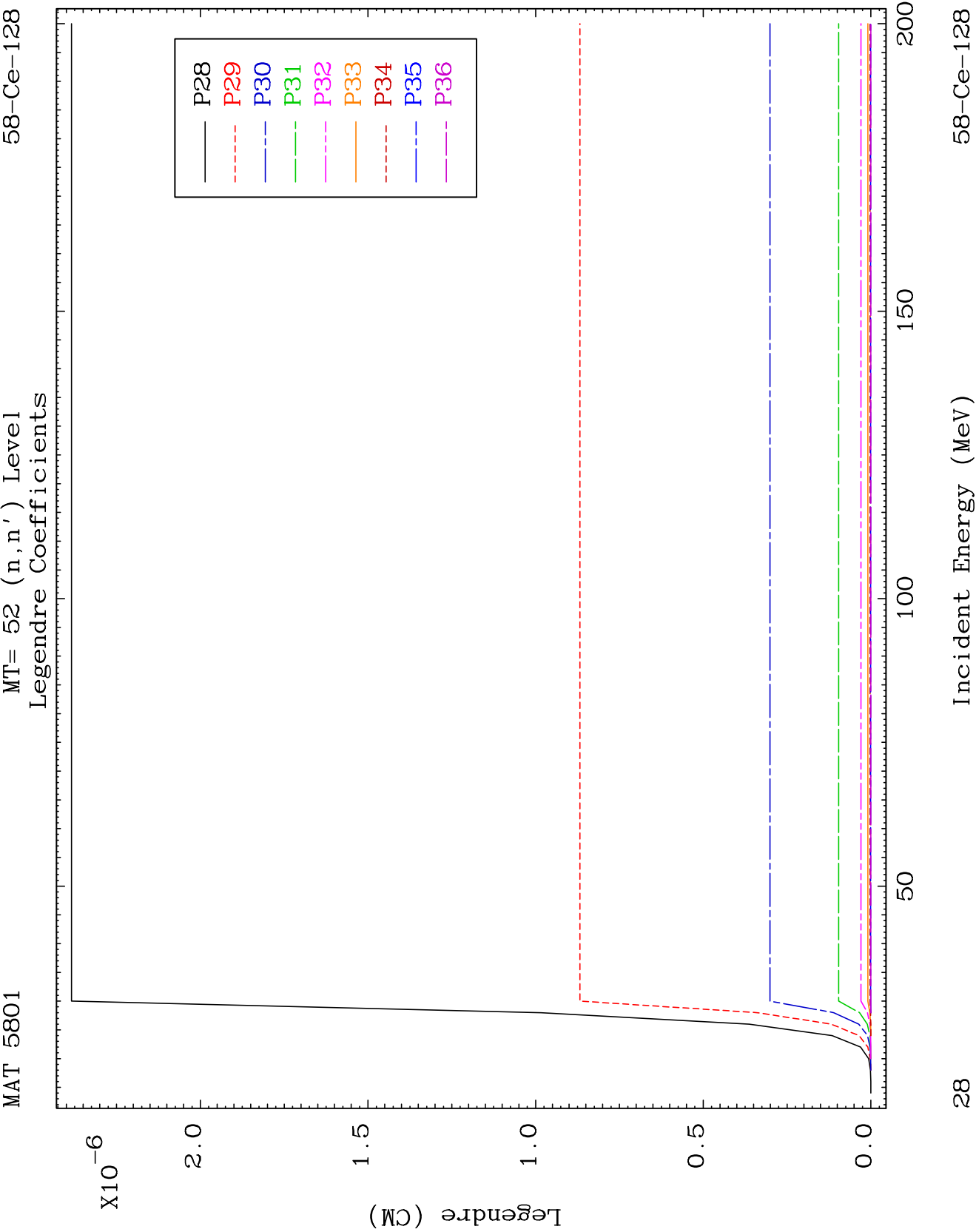
58-Ce-128







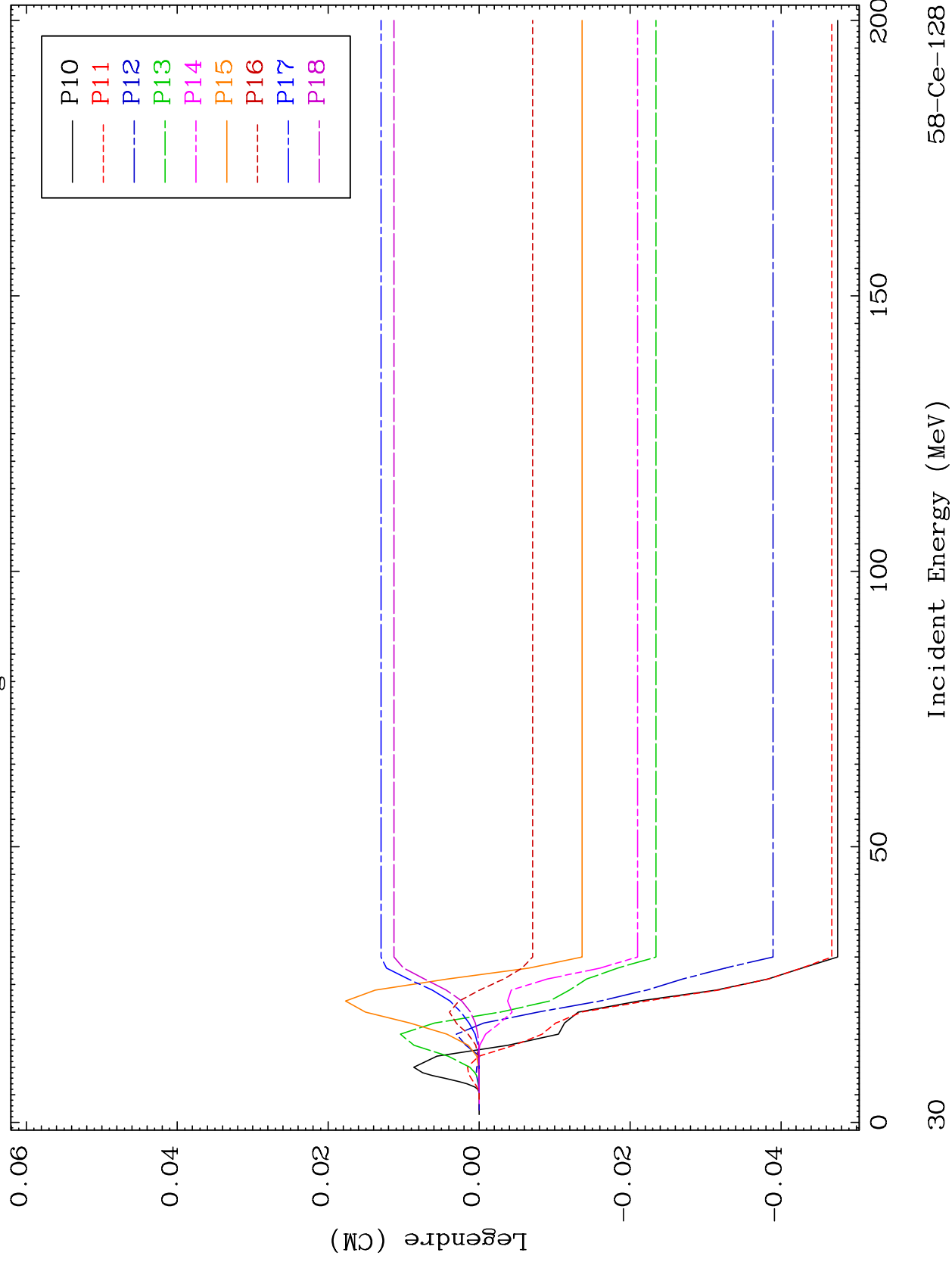


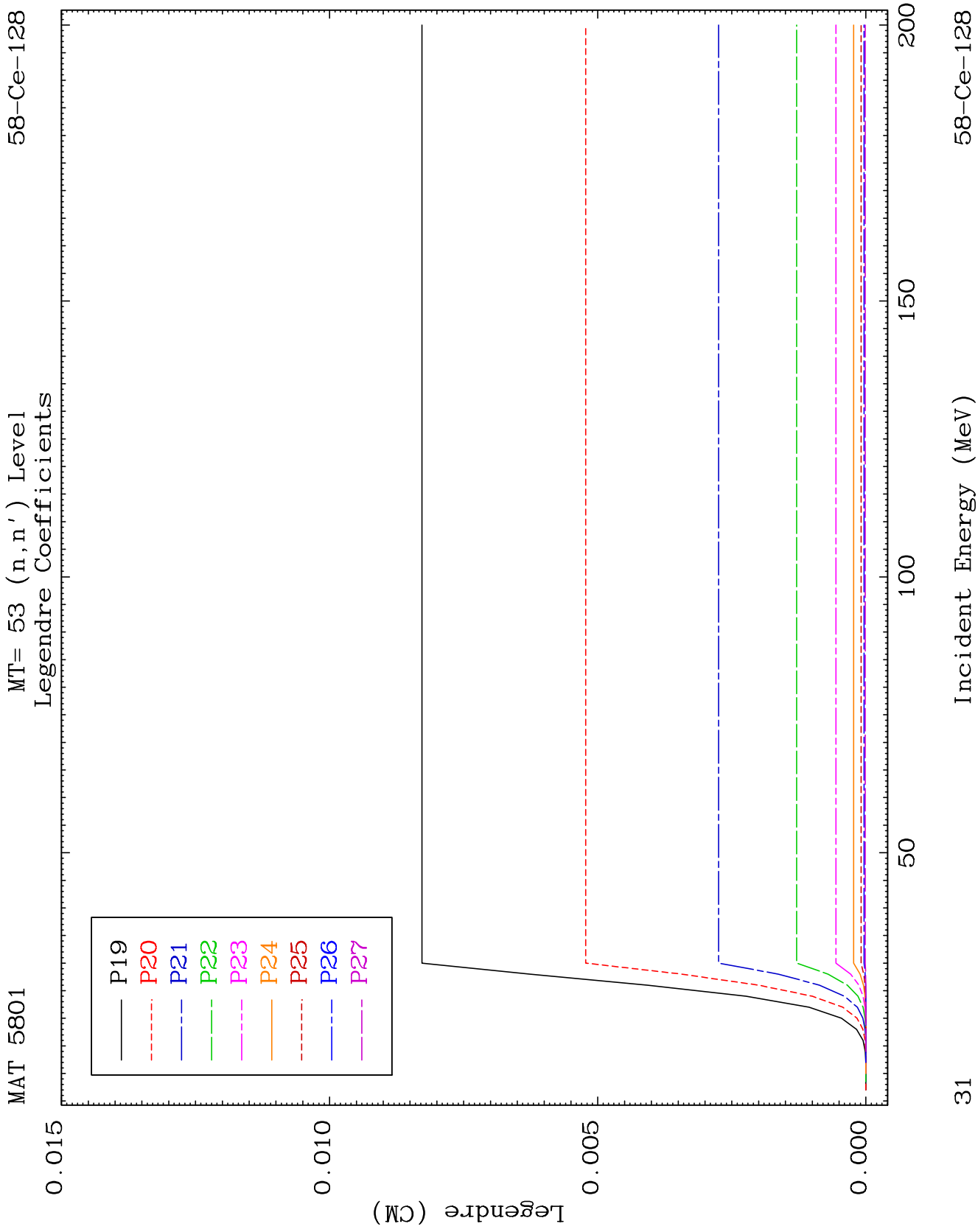


MAT 5801

MT= 53 (n, n') Level
Legendre Coefficients

58-Ce-128

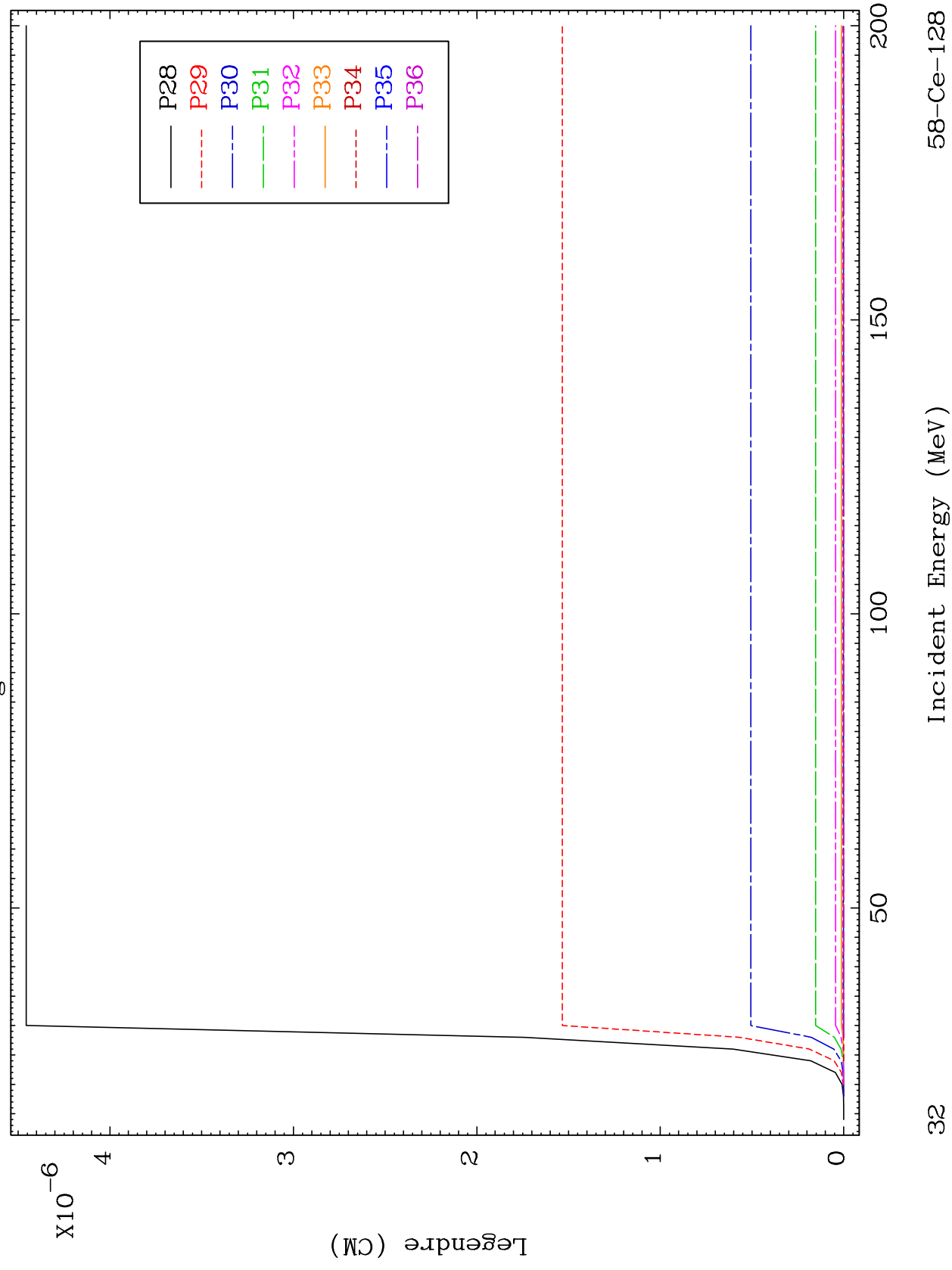




MAT 5801

MT= 53 (n, n') Level
Legendre Coefficients

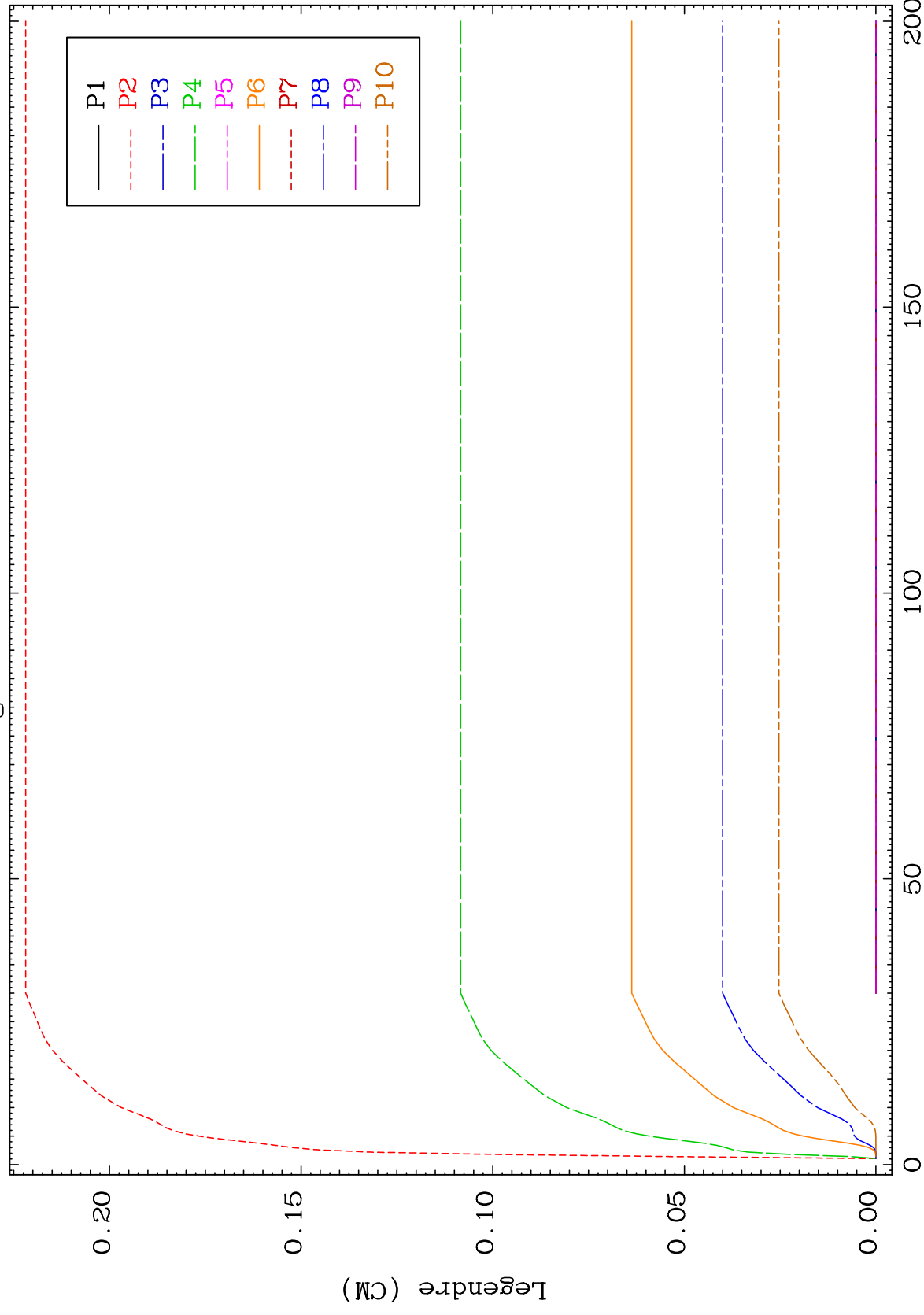
58-Ce-128



MAT 5801

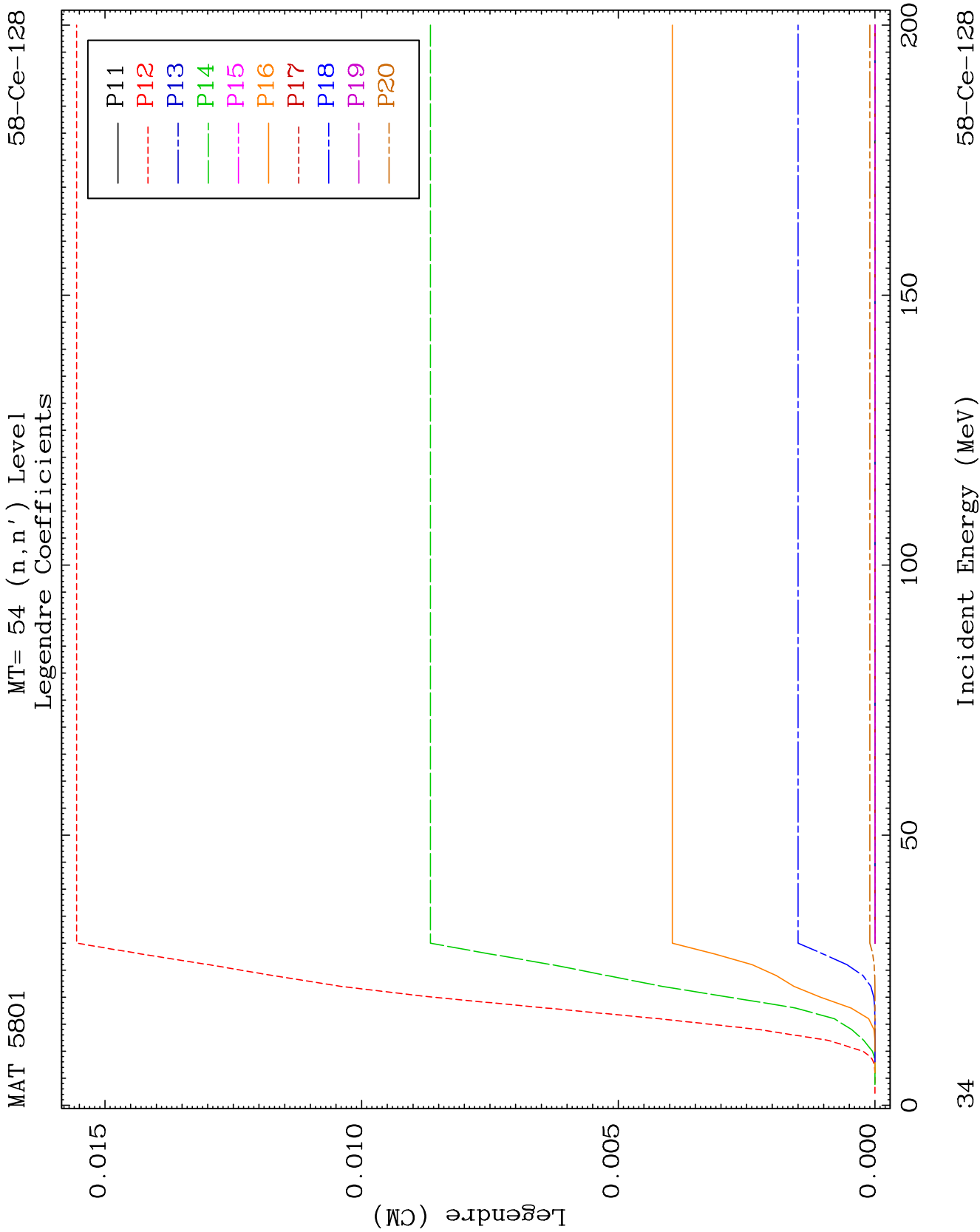
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Legendre Coefficients

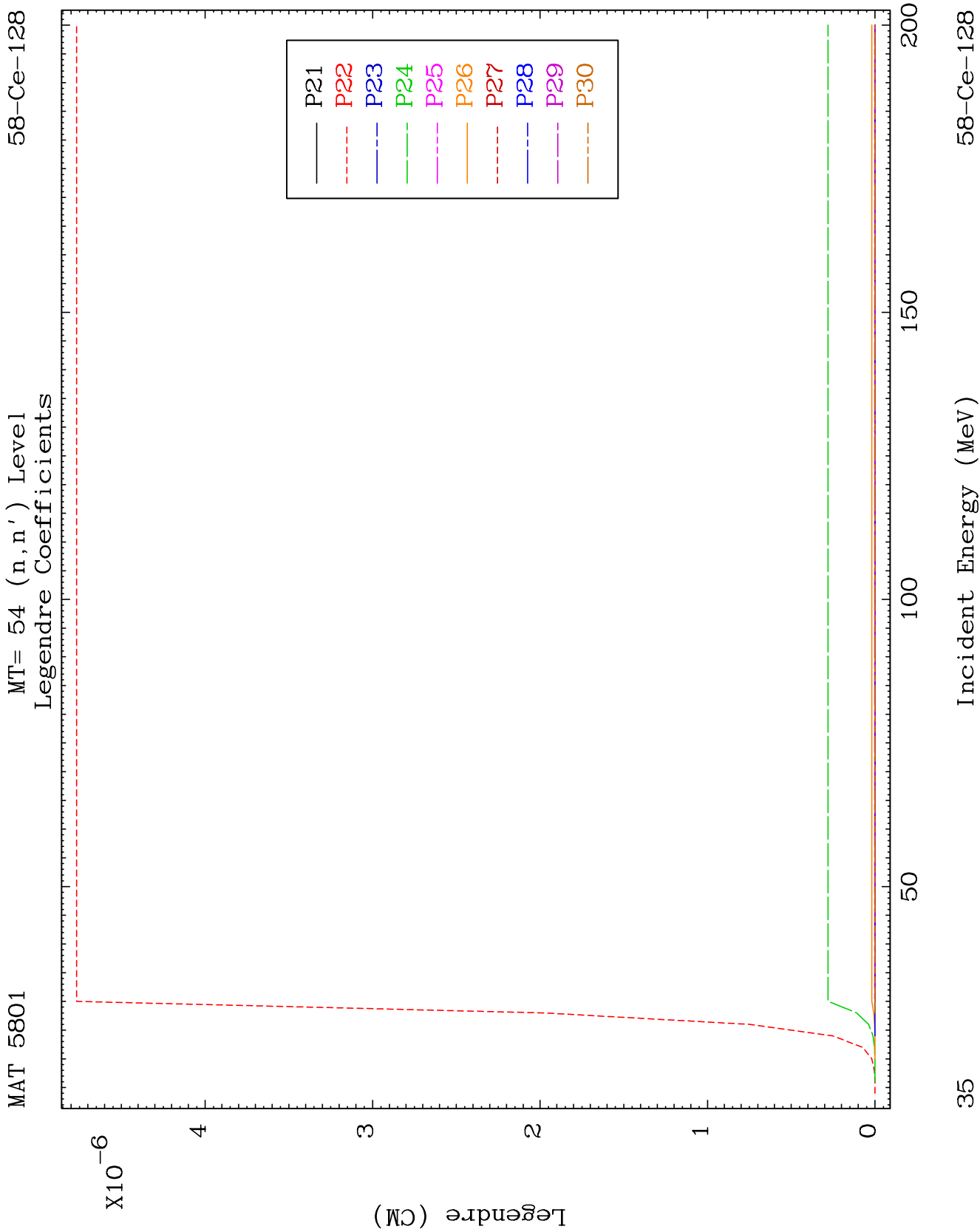
58-Ce-128

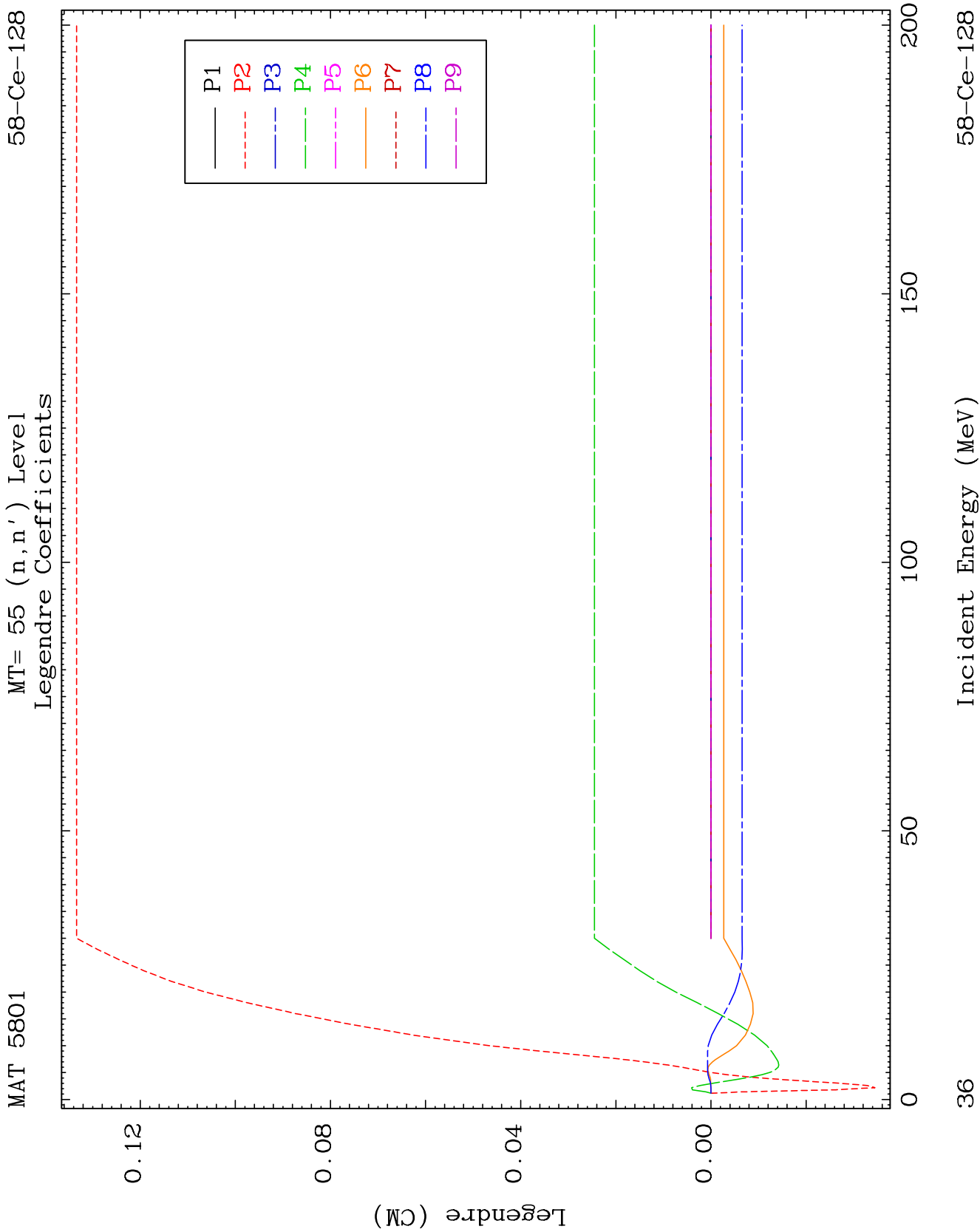


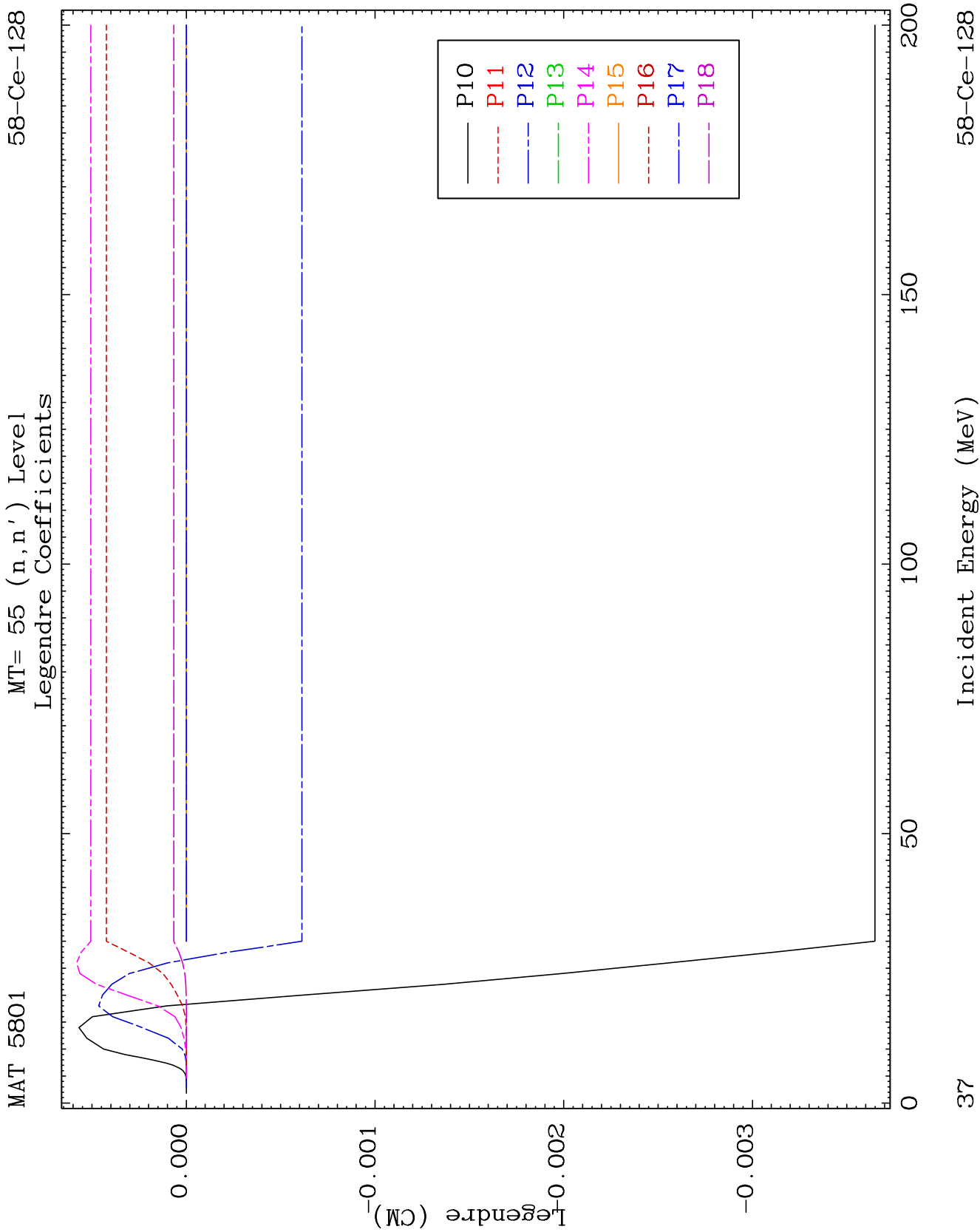
58-Ce-128

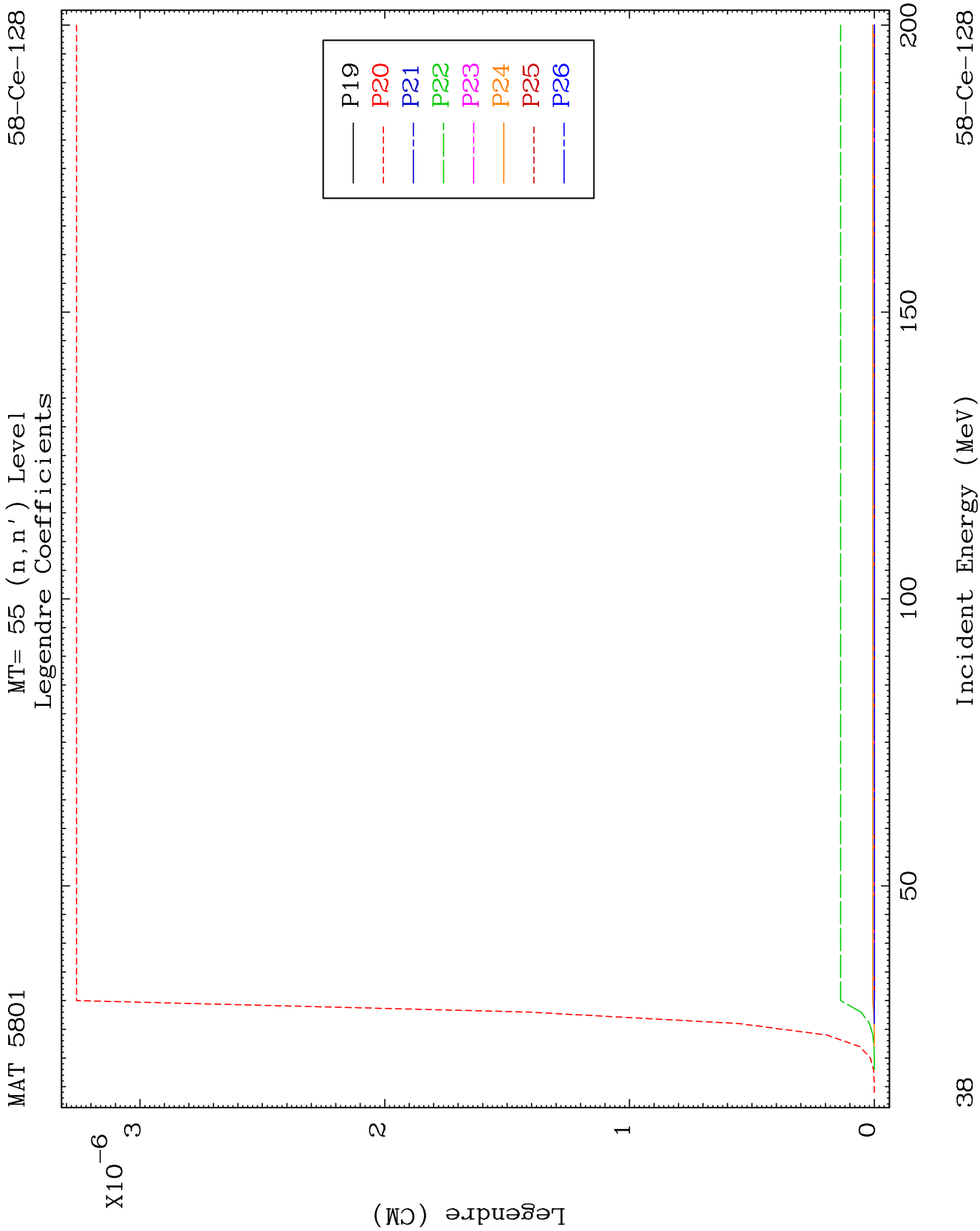
Incident Energy (MeV)

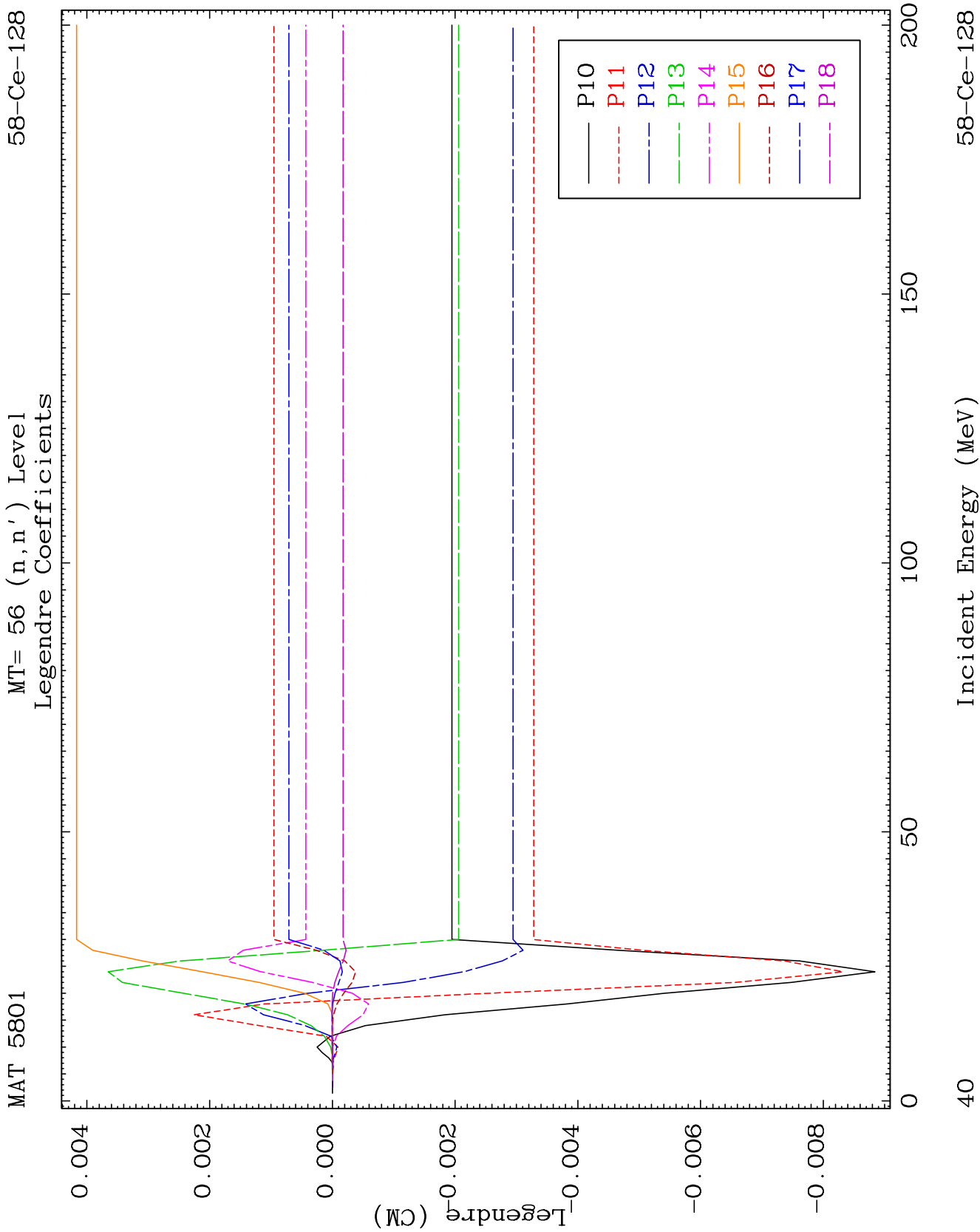


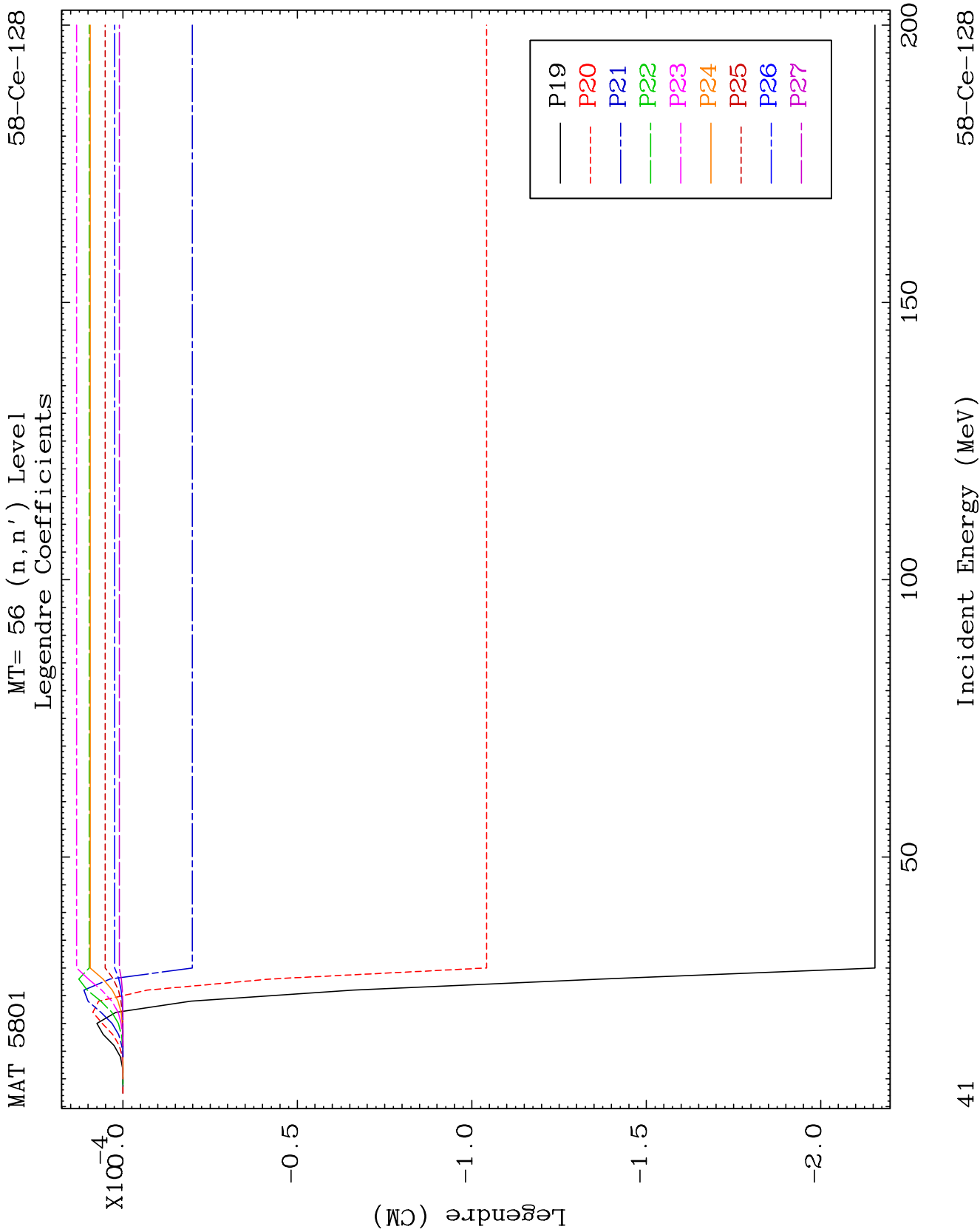


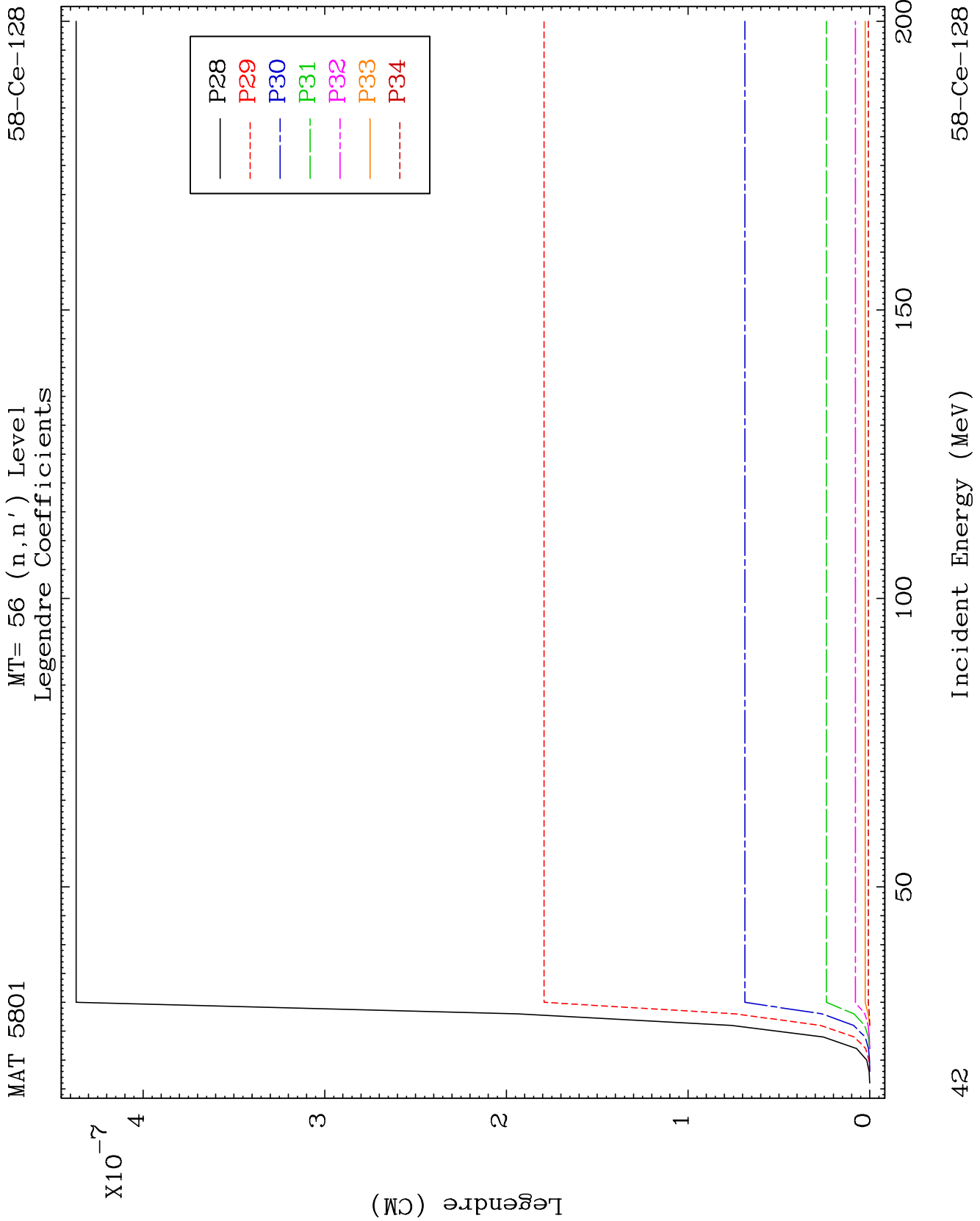


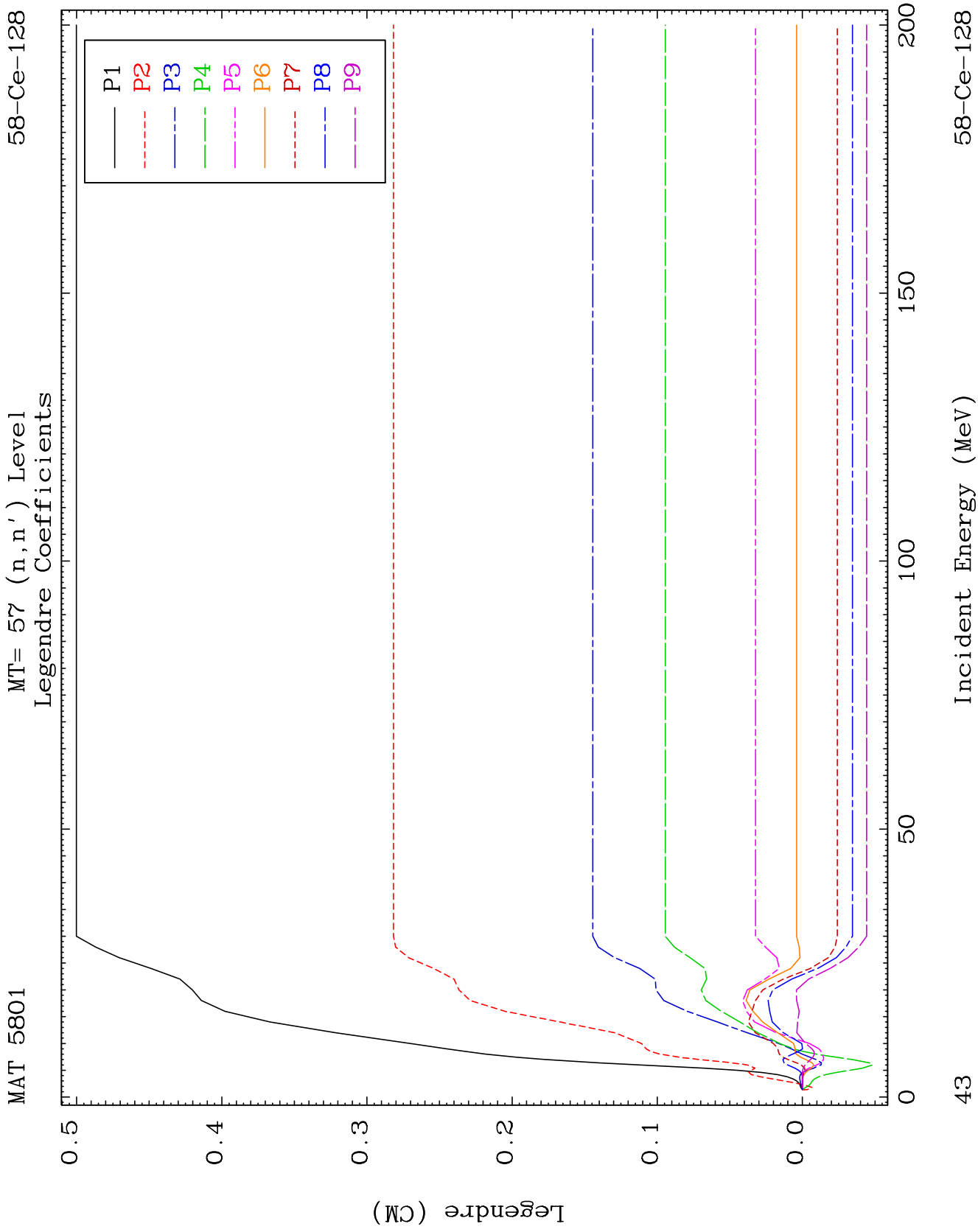


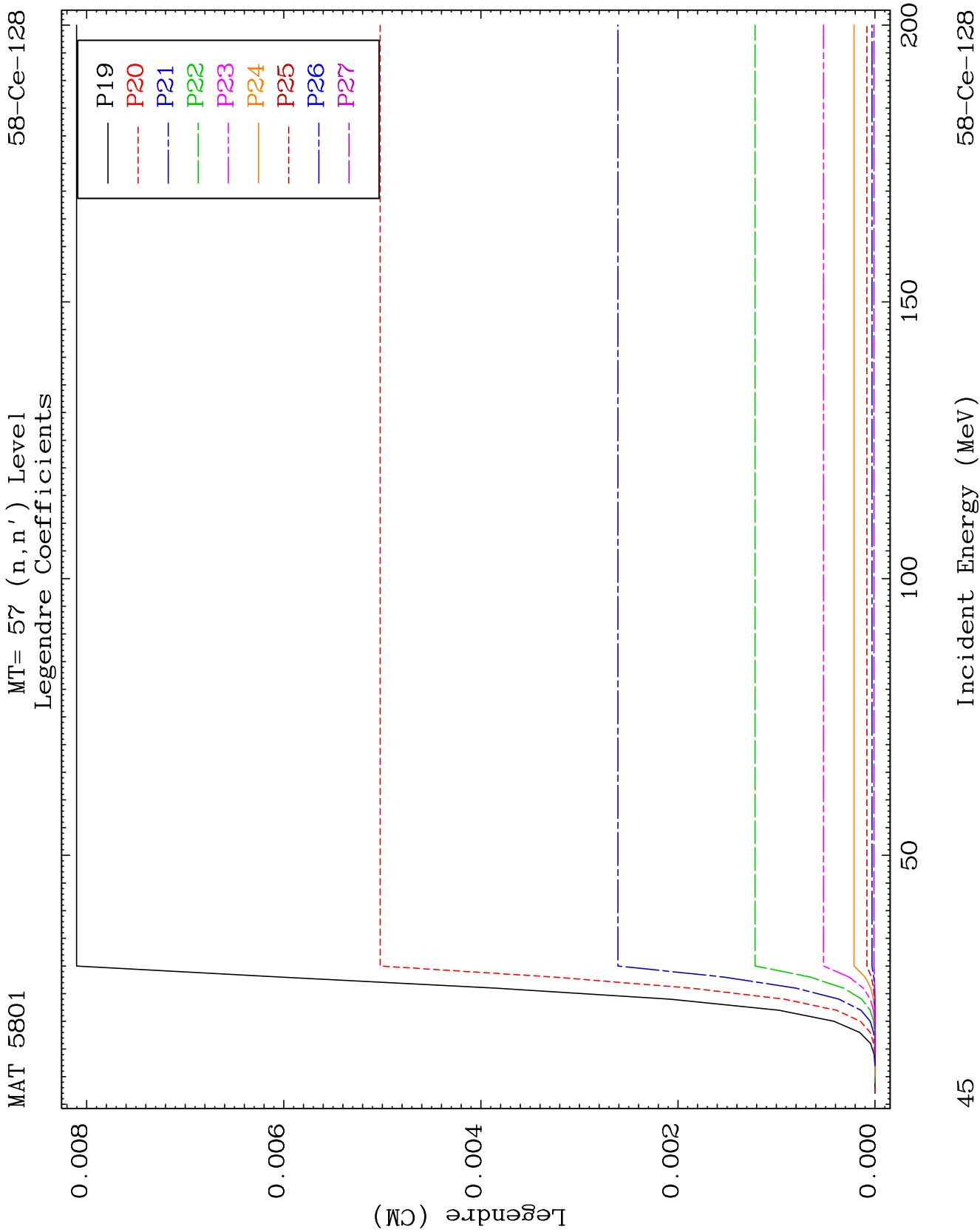


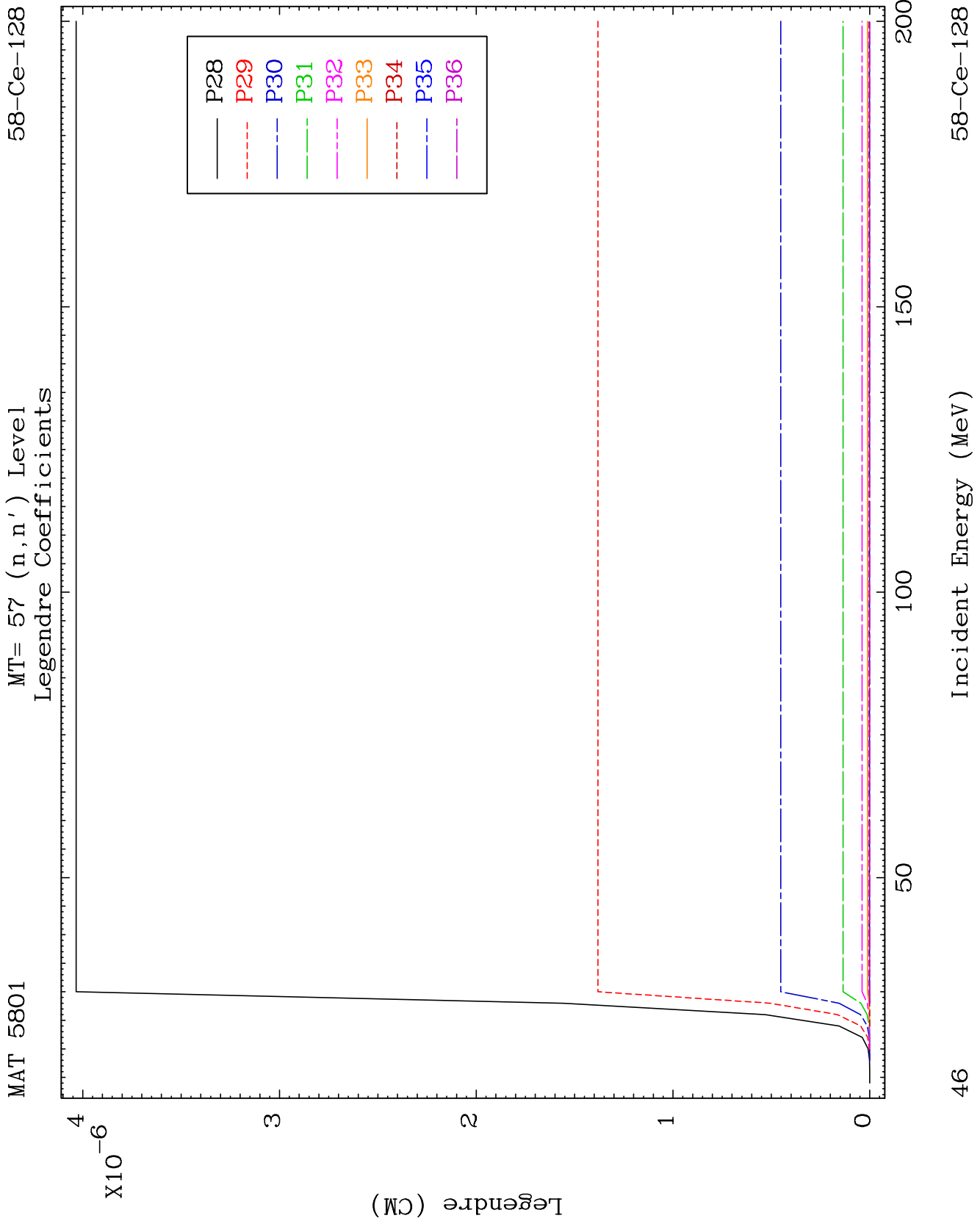


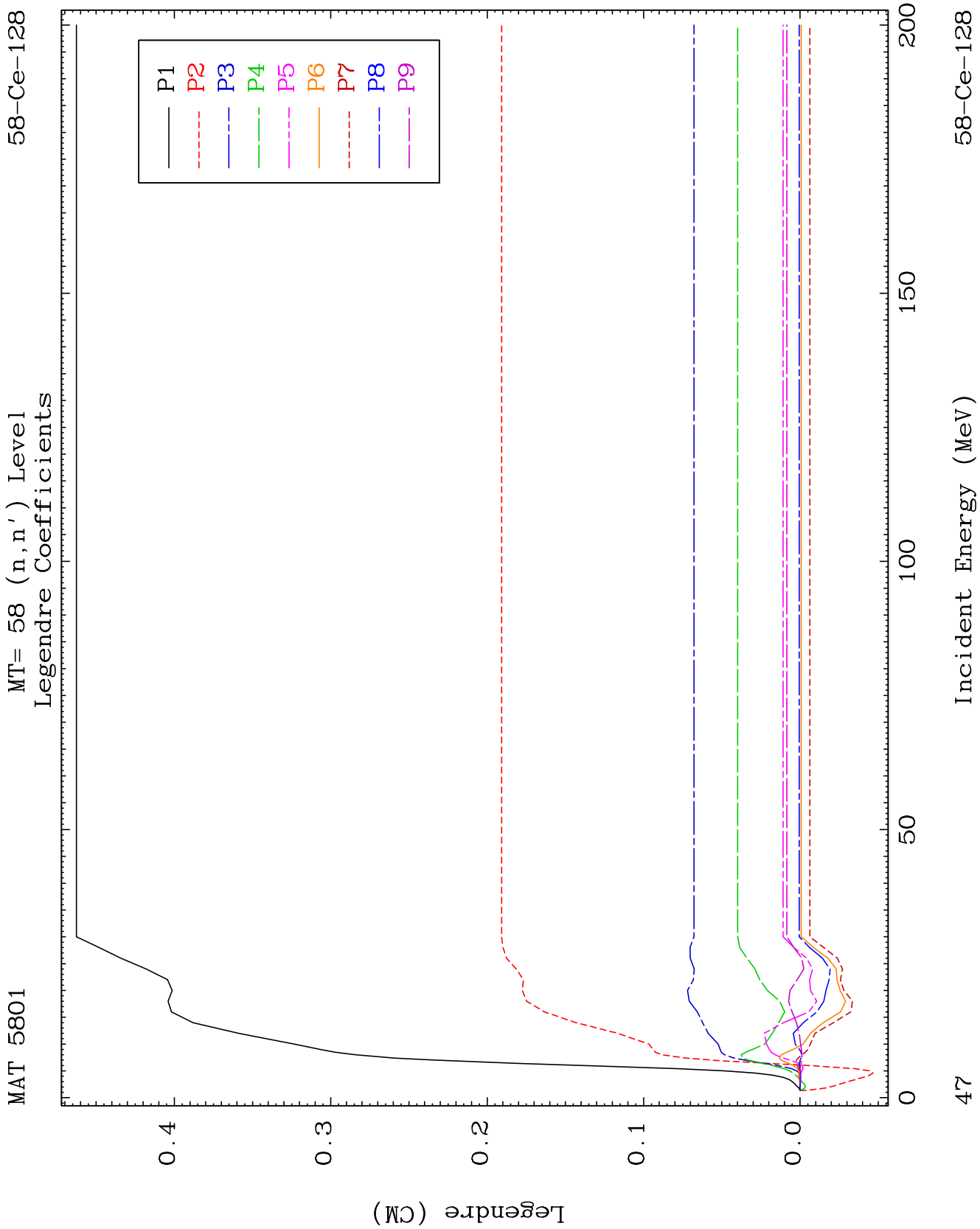








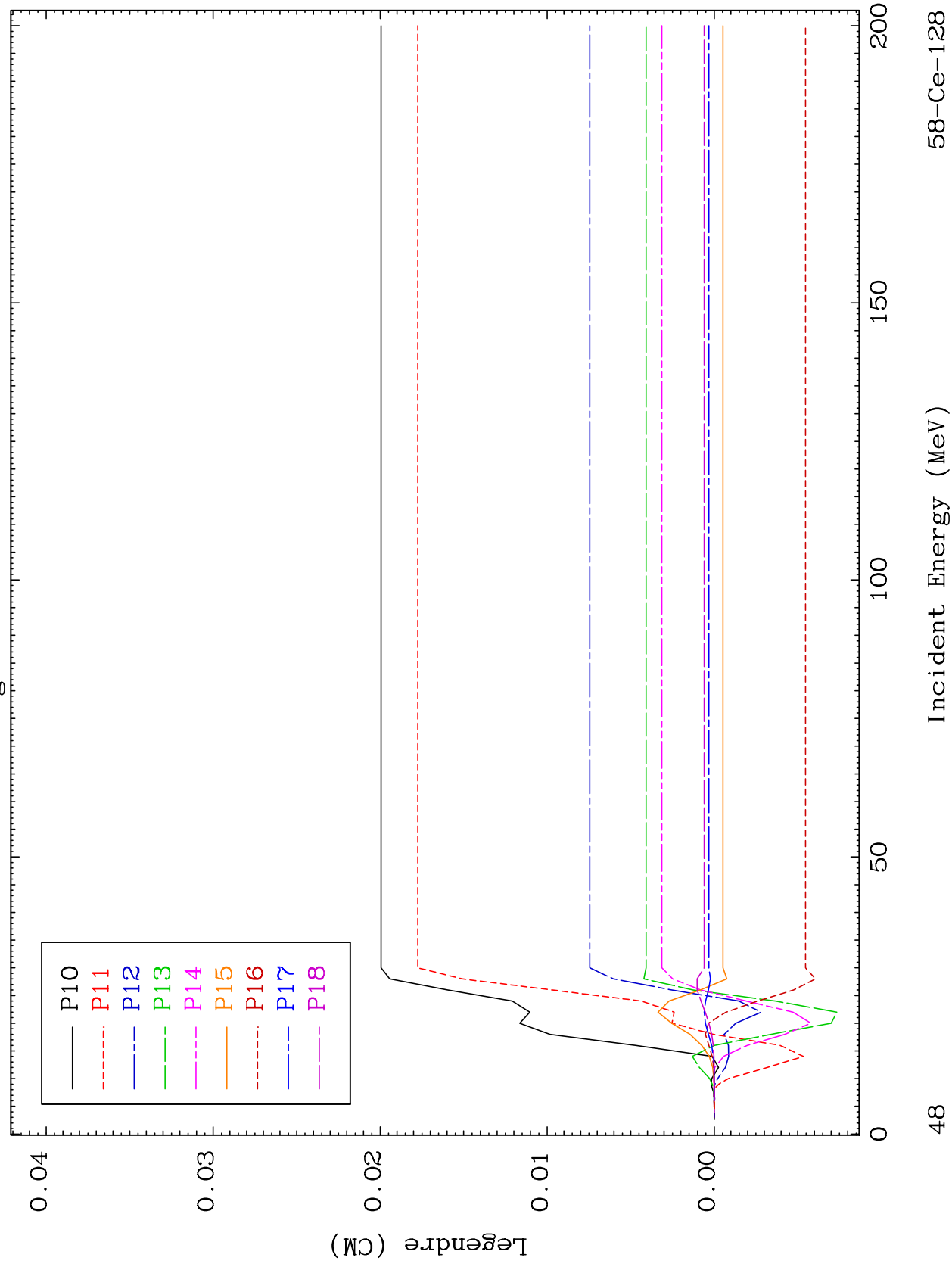


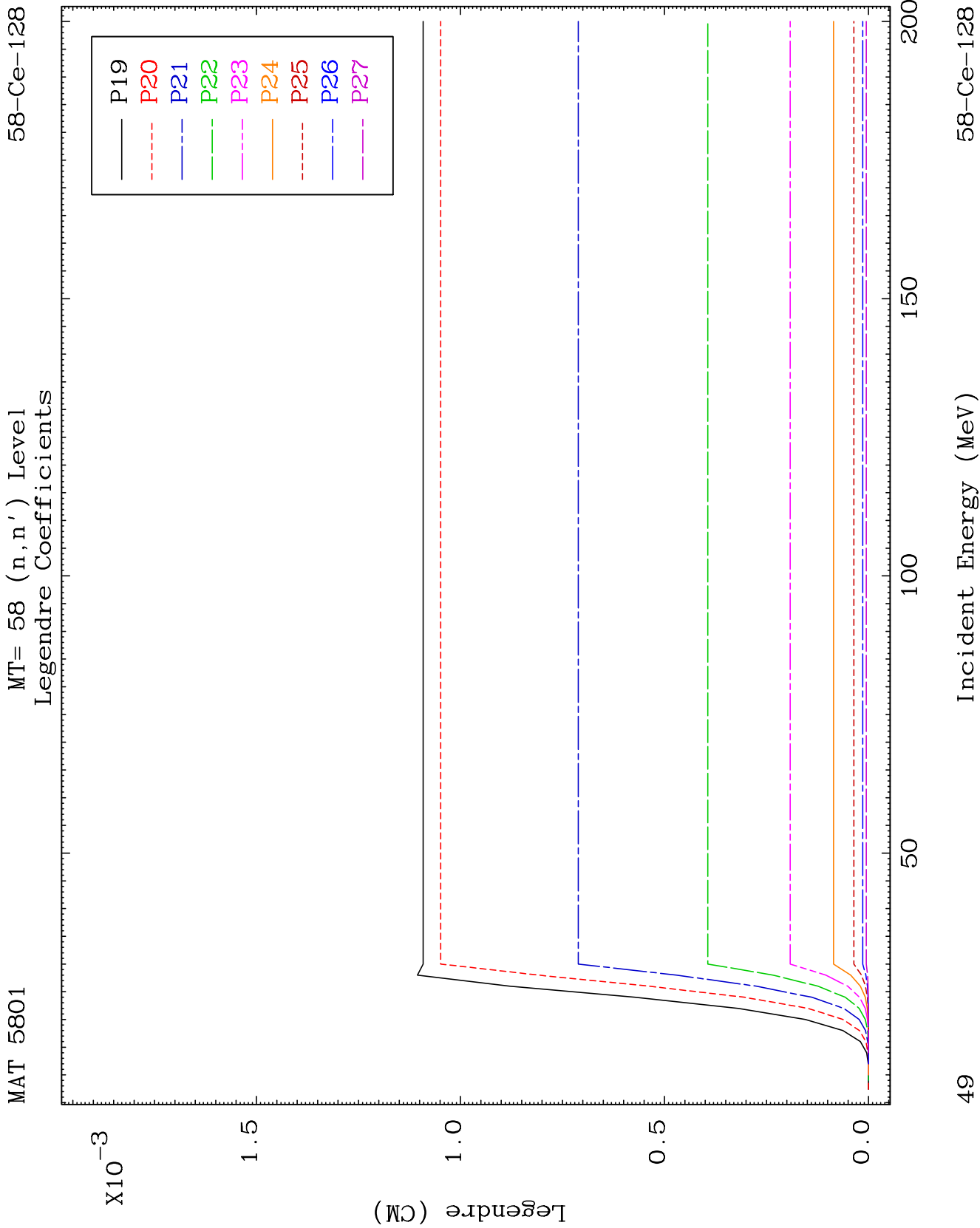


MAT 5801

MT= 58 (n, n') Level
Legendre Coefficients

58-Ce-128

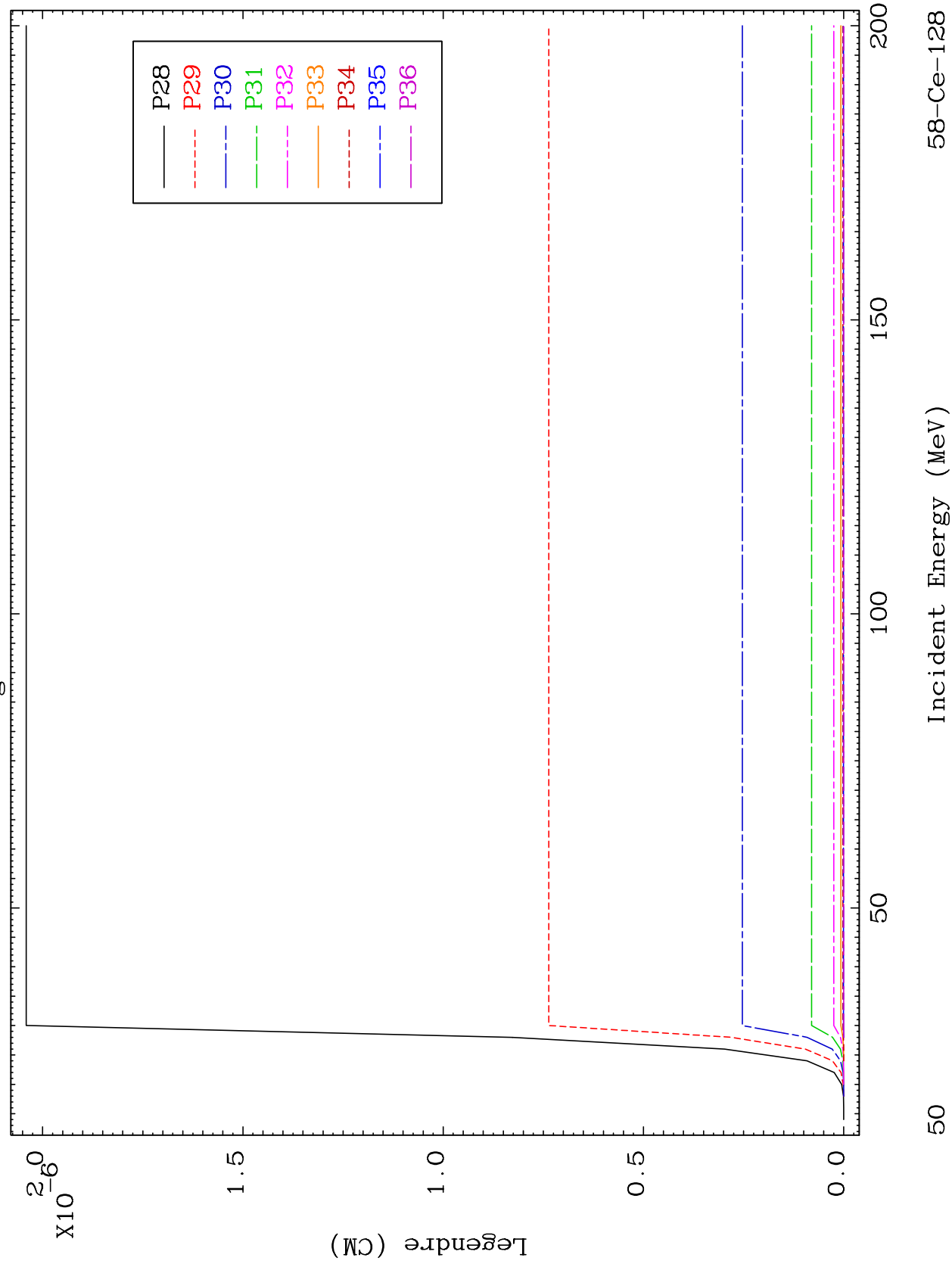




MAT 5801

MT= 58 (n, n') Level
Legendre Coefficients

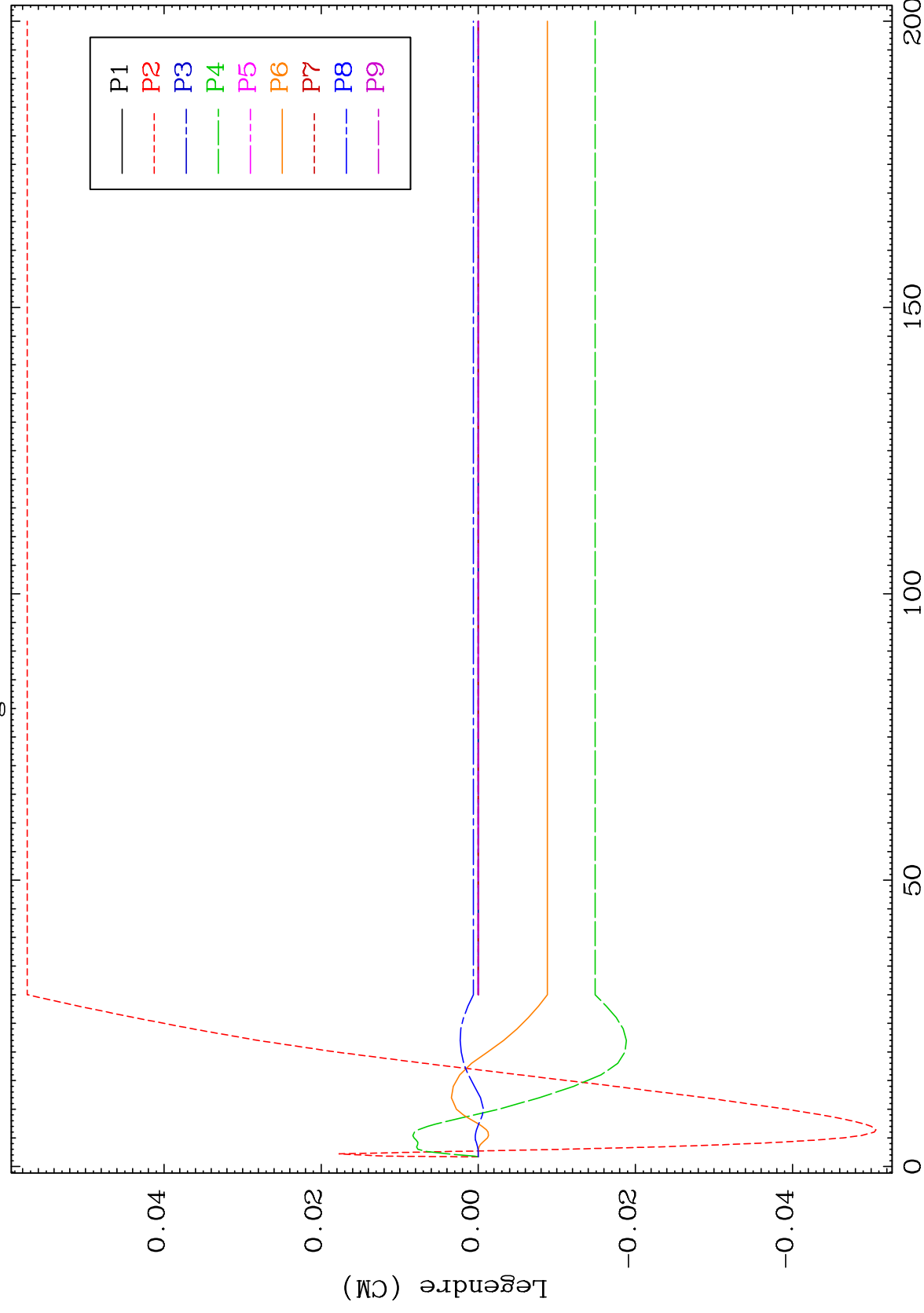
58-Ce-128



MAT 5801

MT= 59 (n, n') Level
Legendre Coefficients

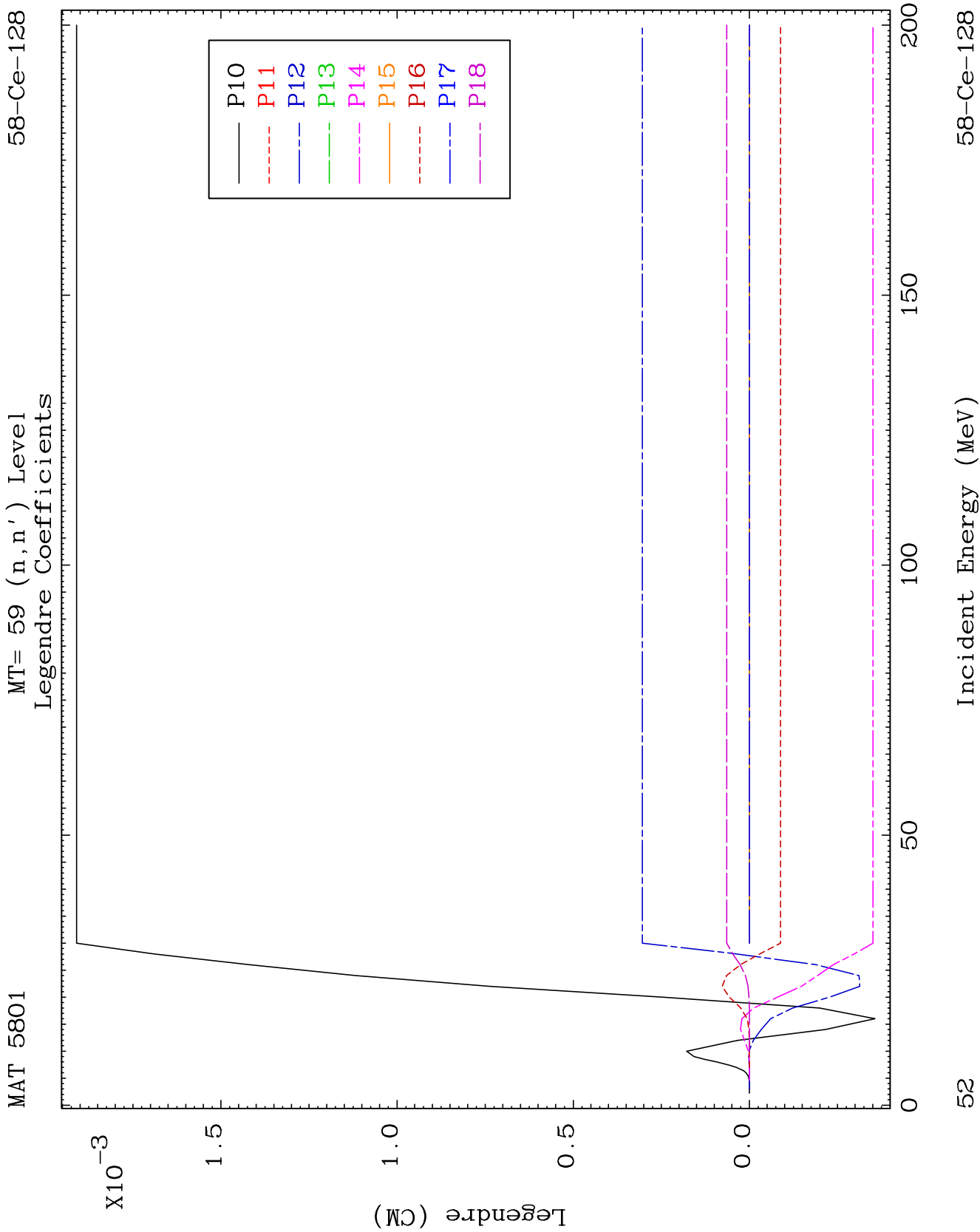
58-Ce-128

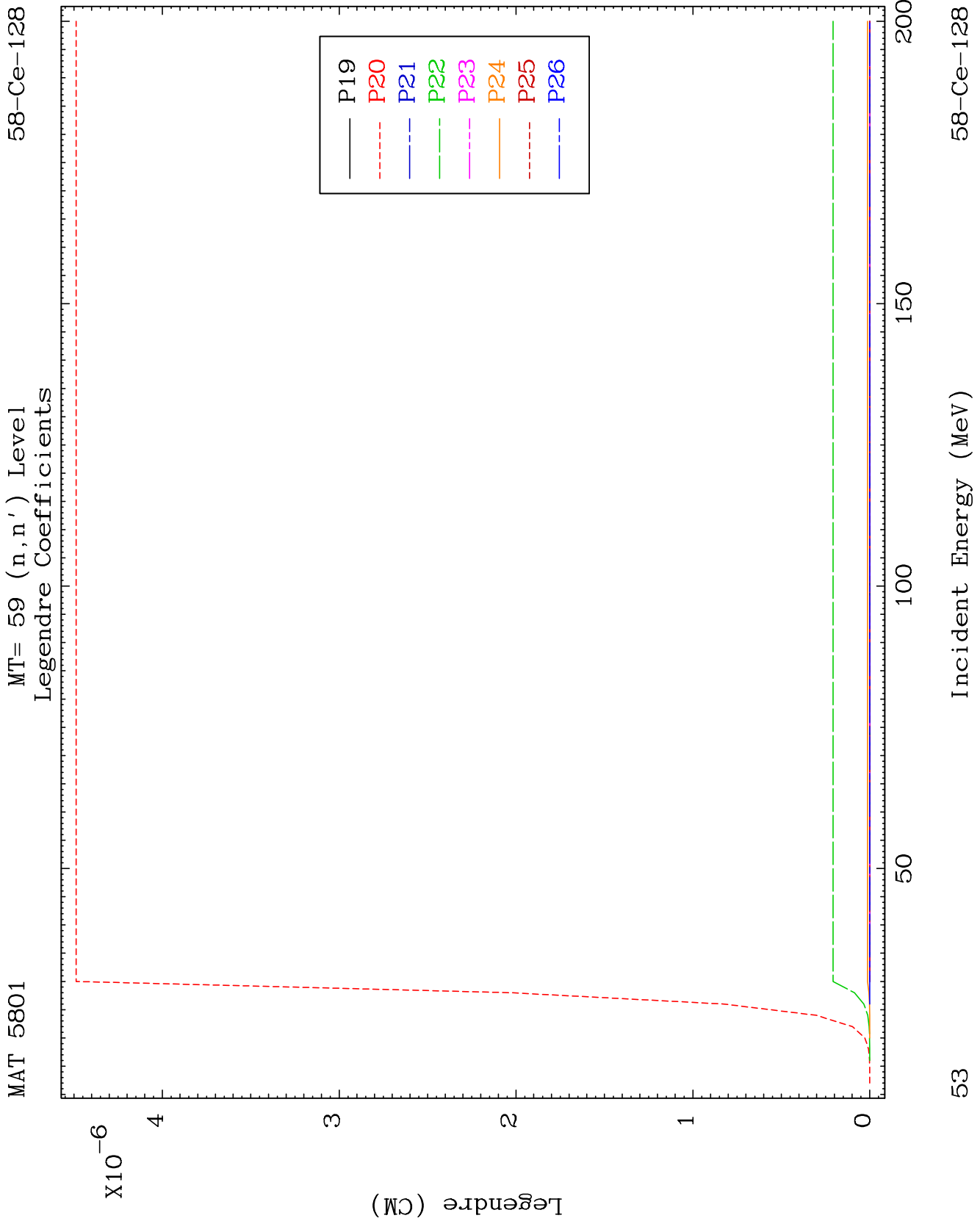


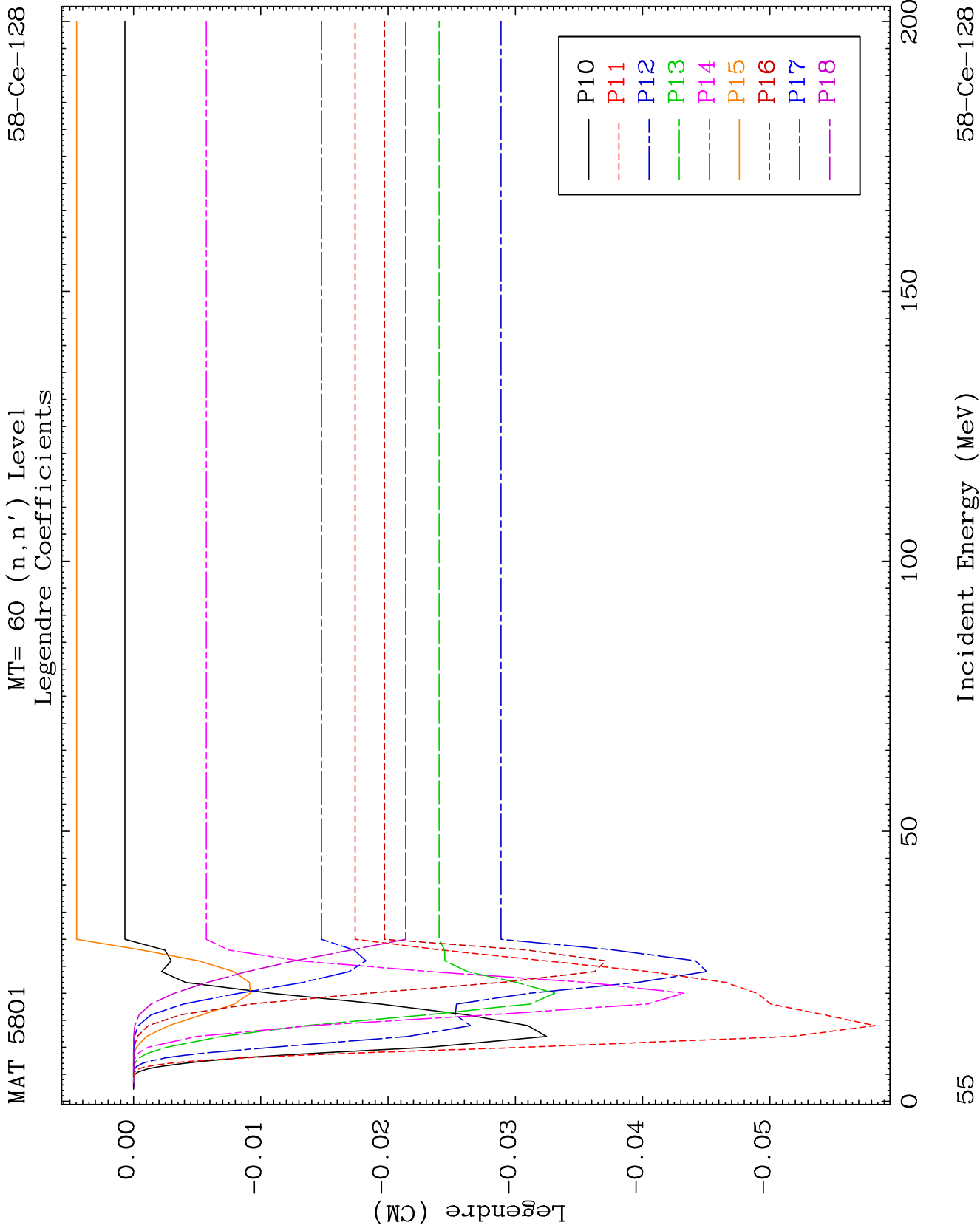
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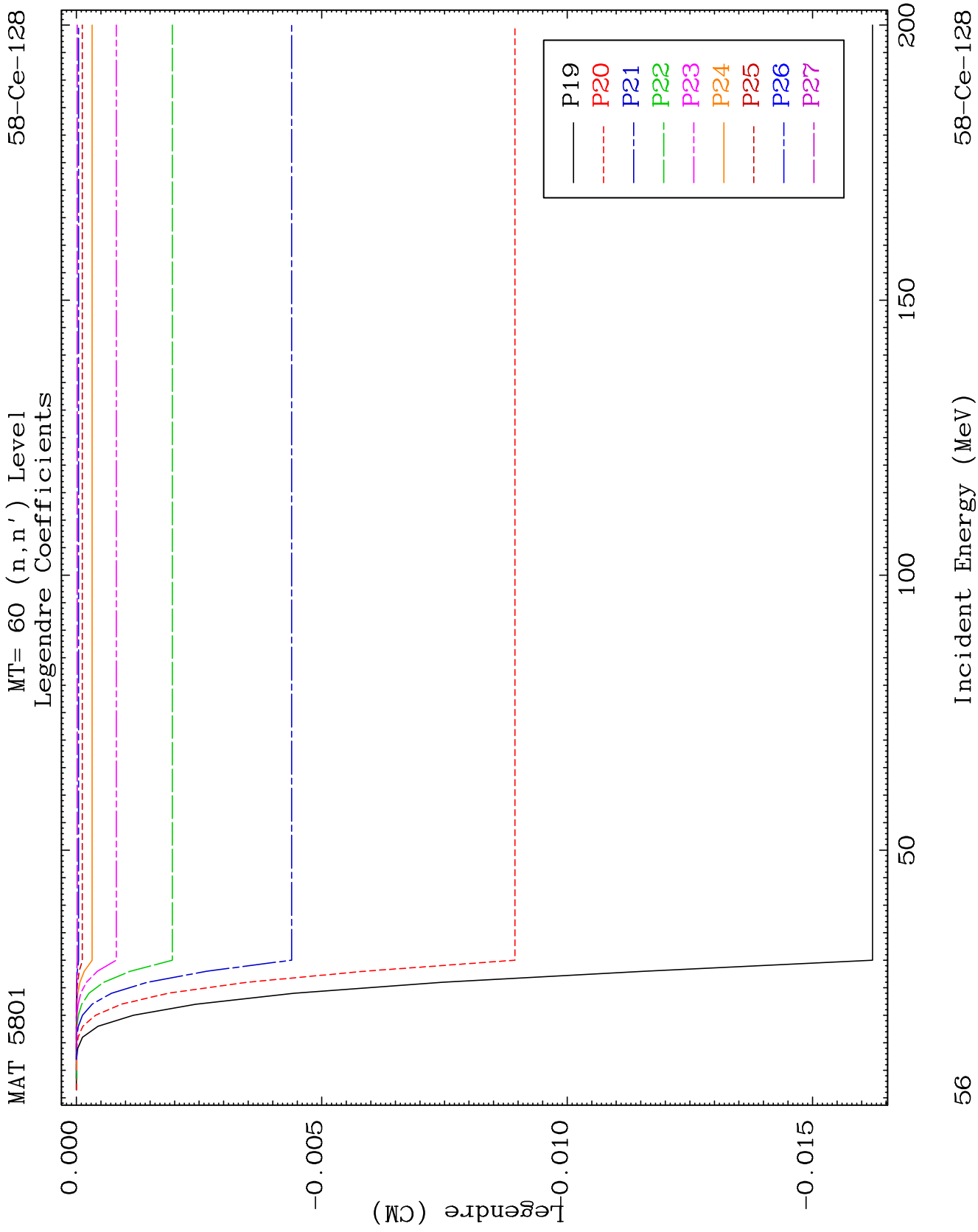
Incident Energy (MeV)

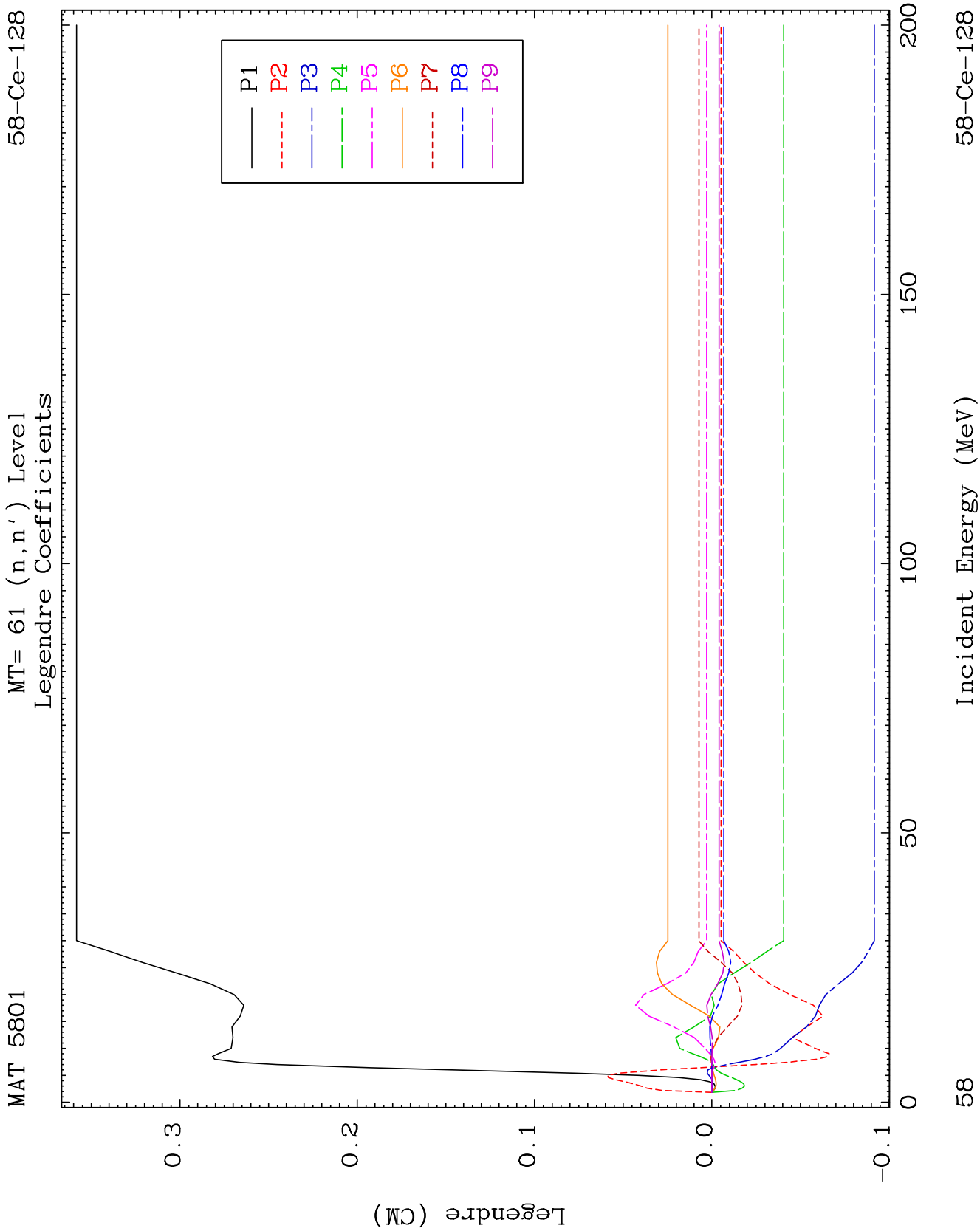
58-Ce-128

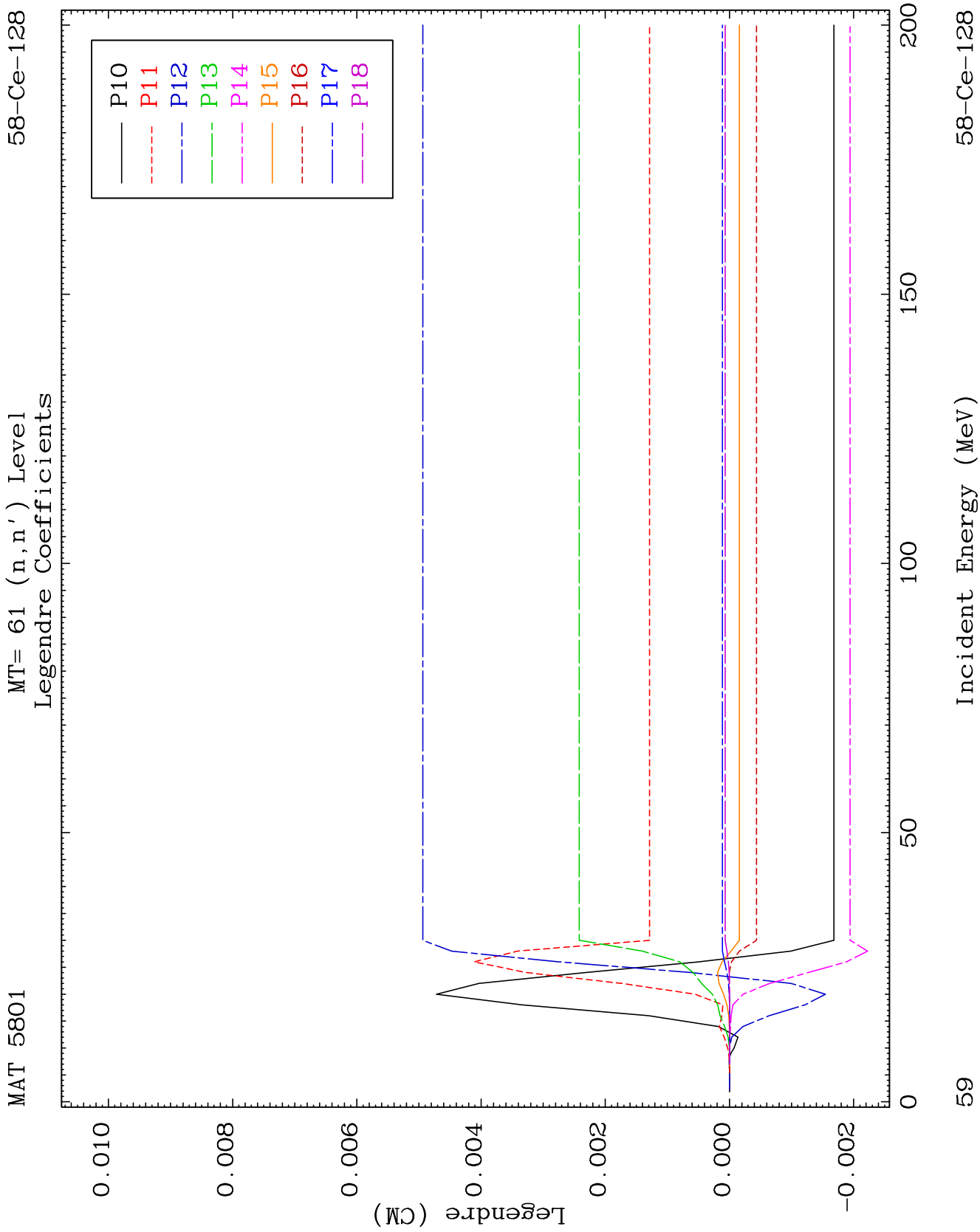








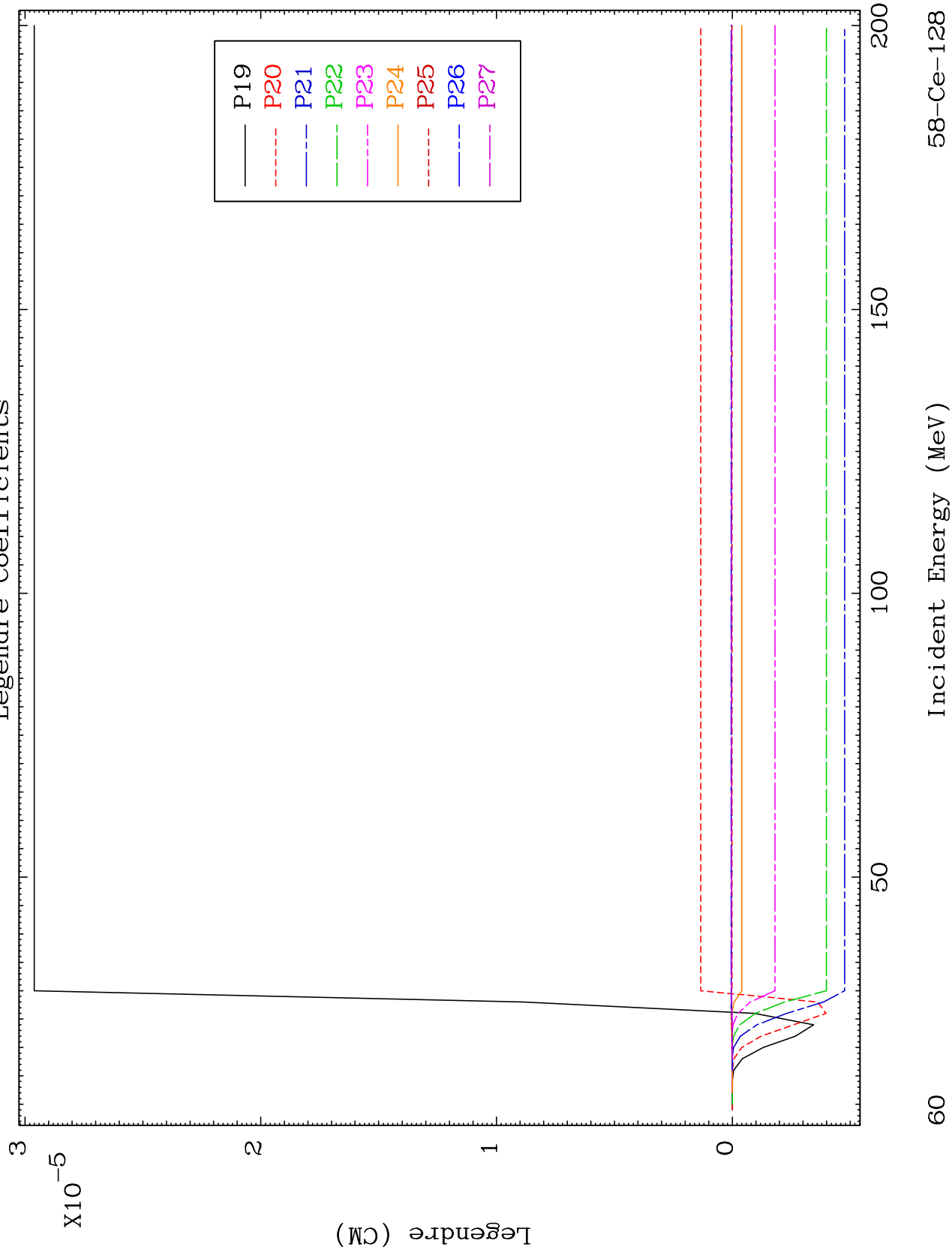


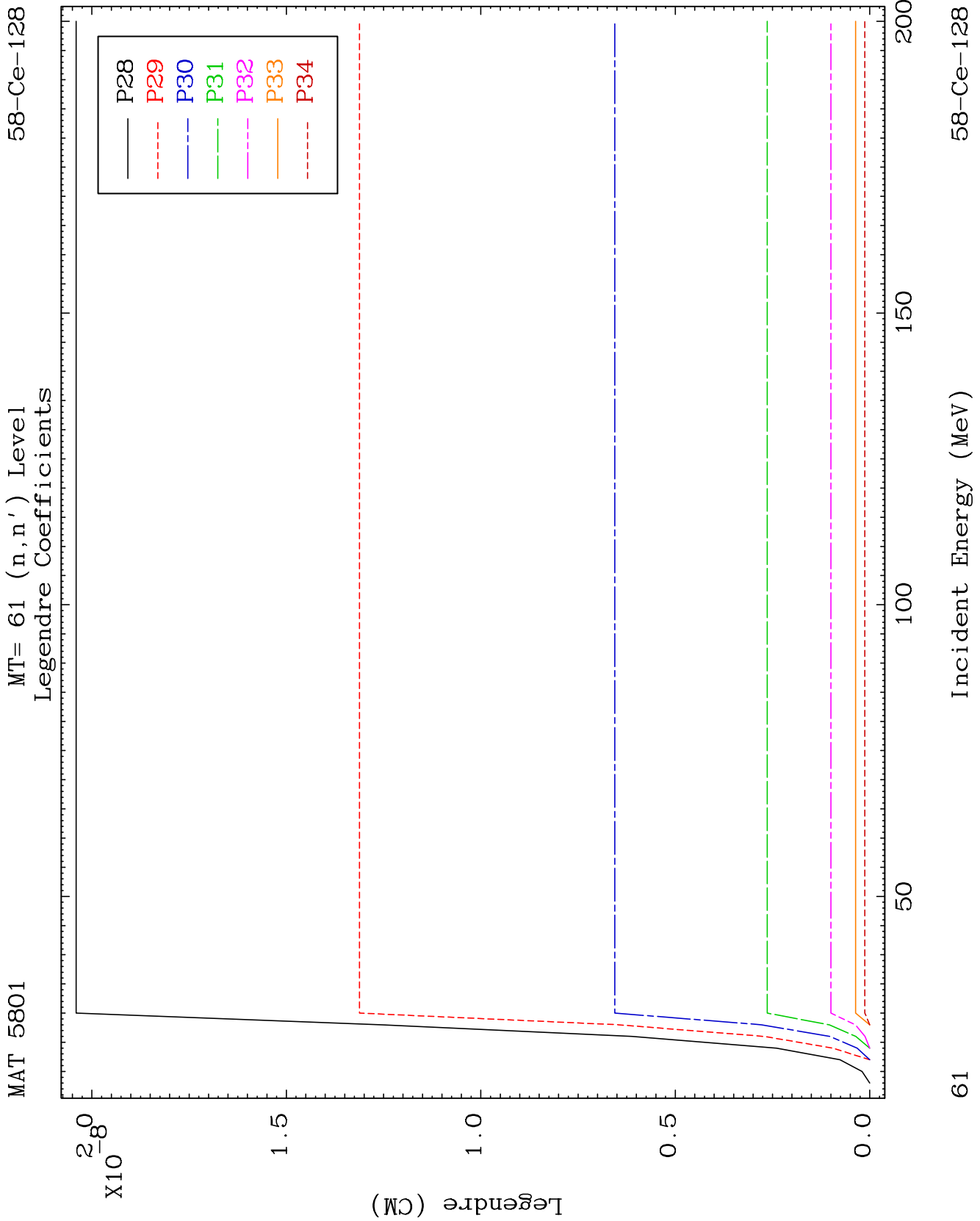


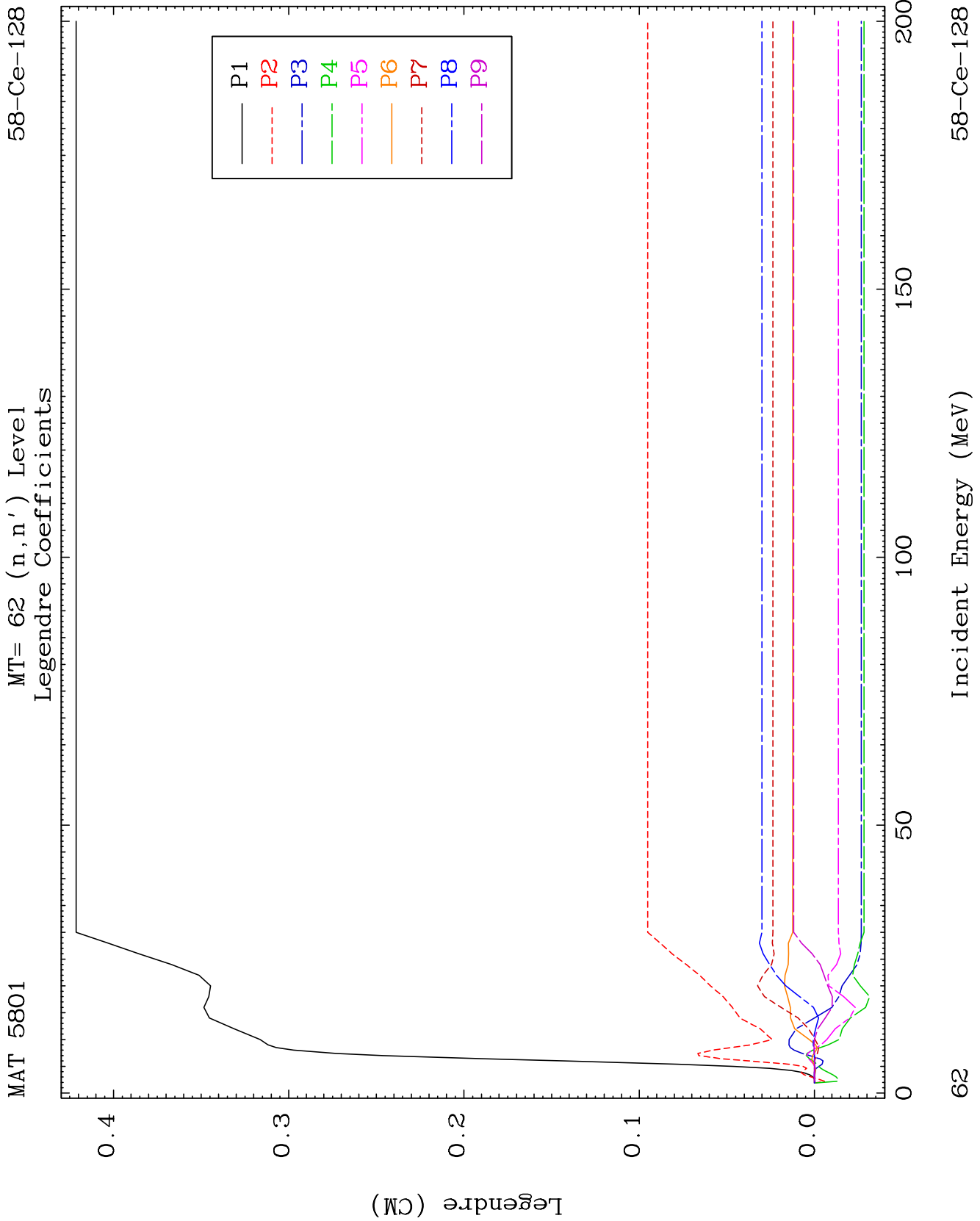
MAT 5801

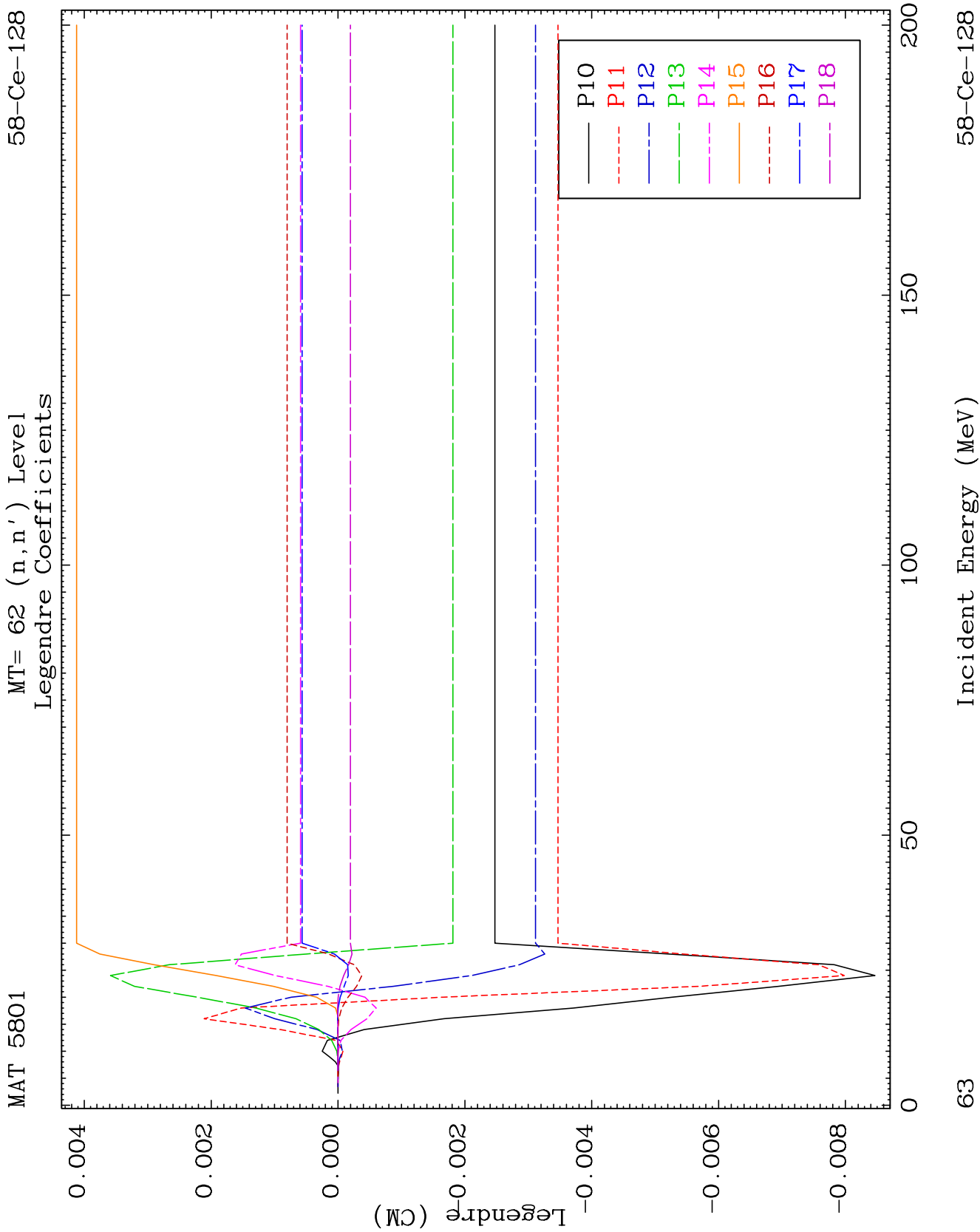
MT= 61 (n, n') Level
Legendre Coefficients

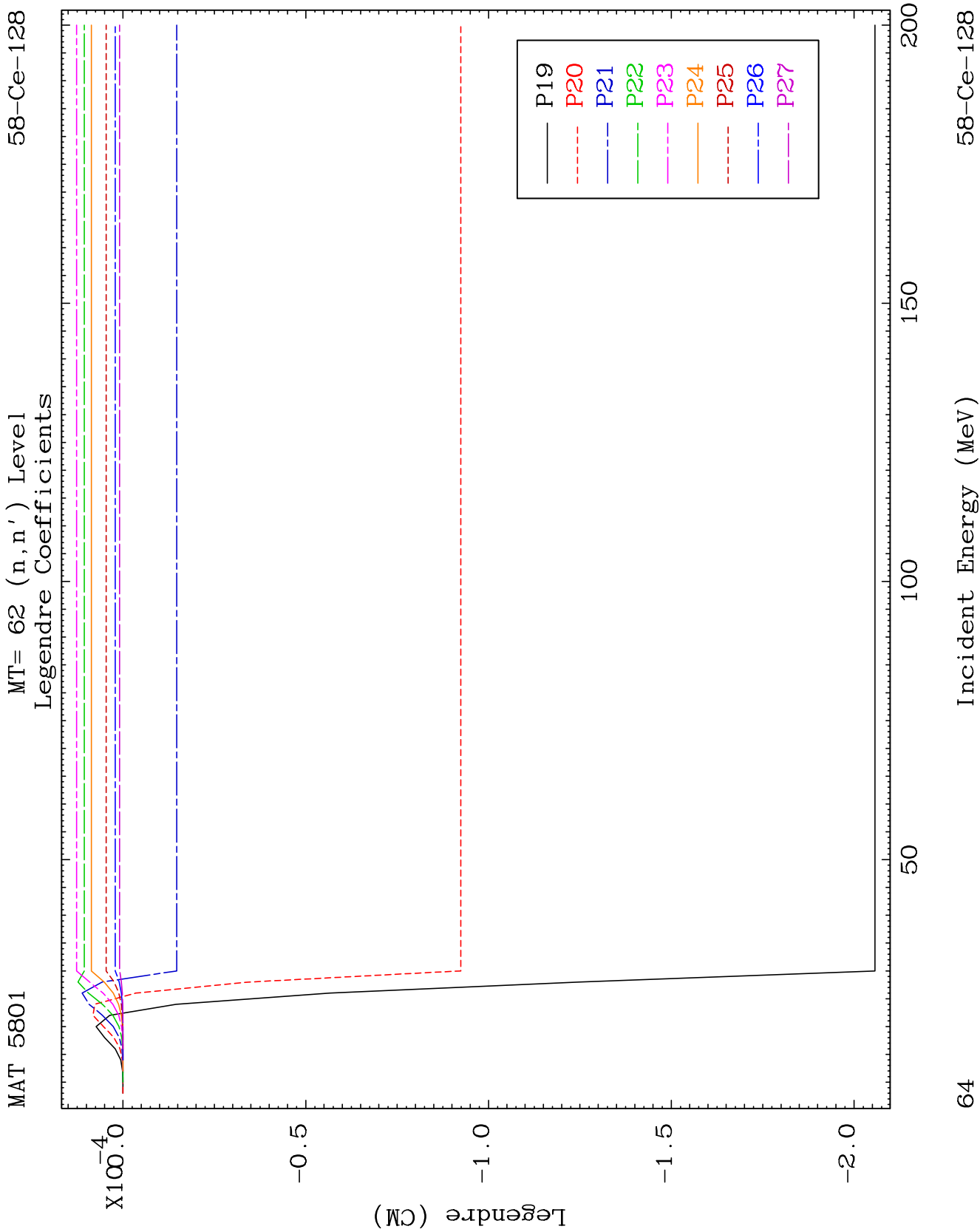
58-Ce-128

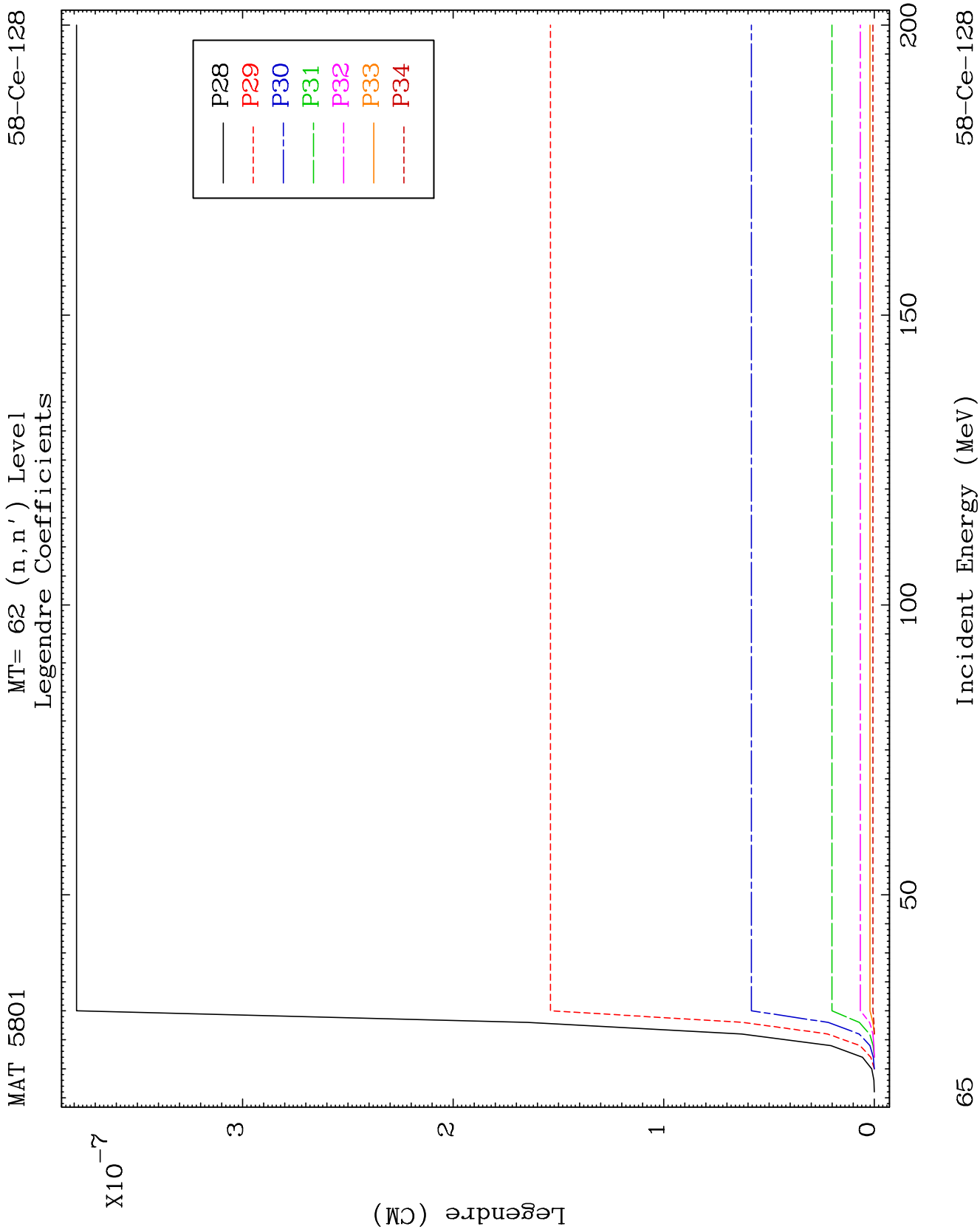


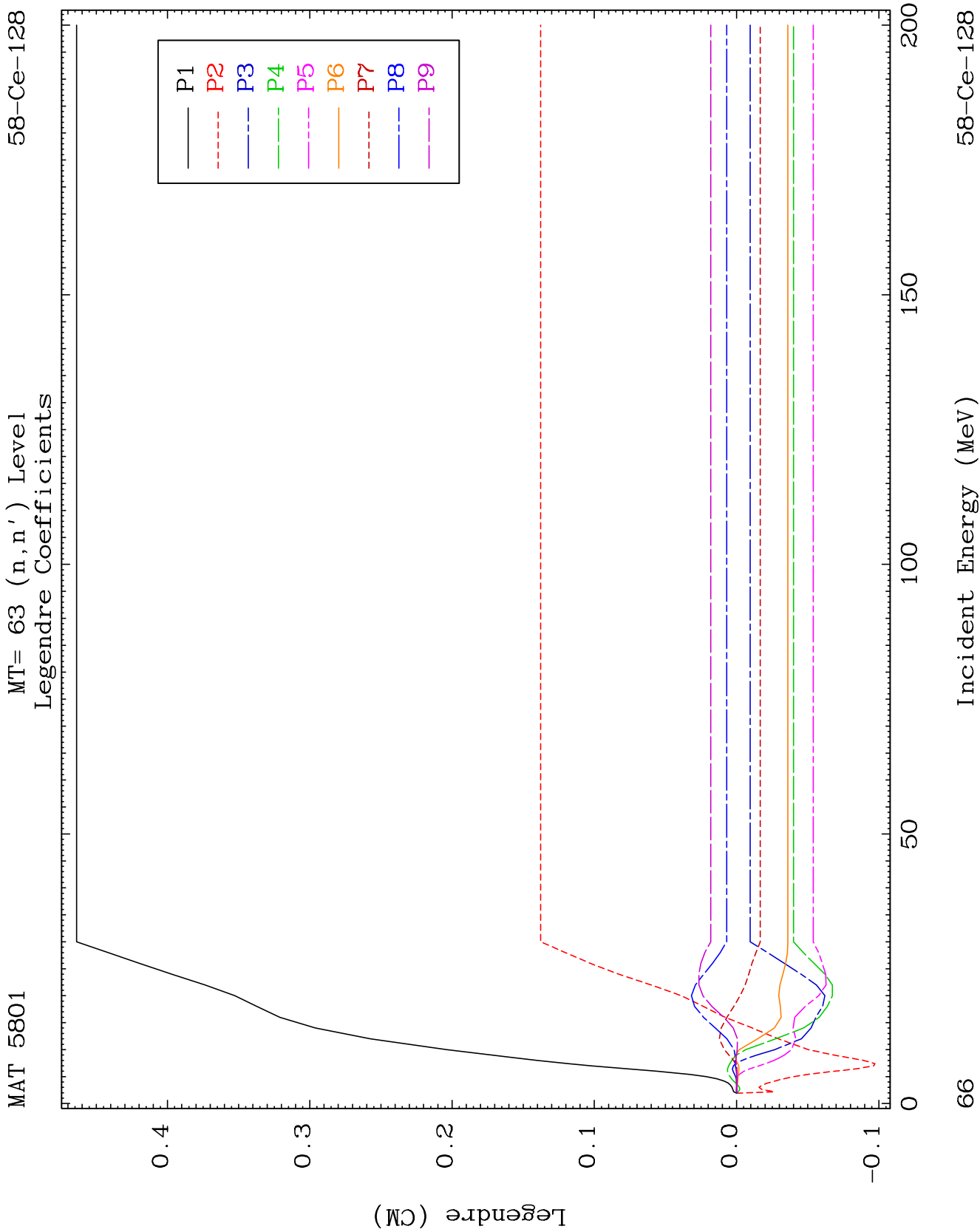








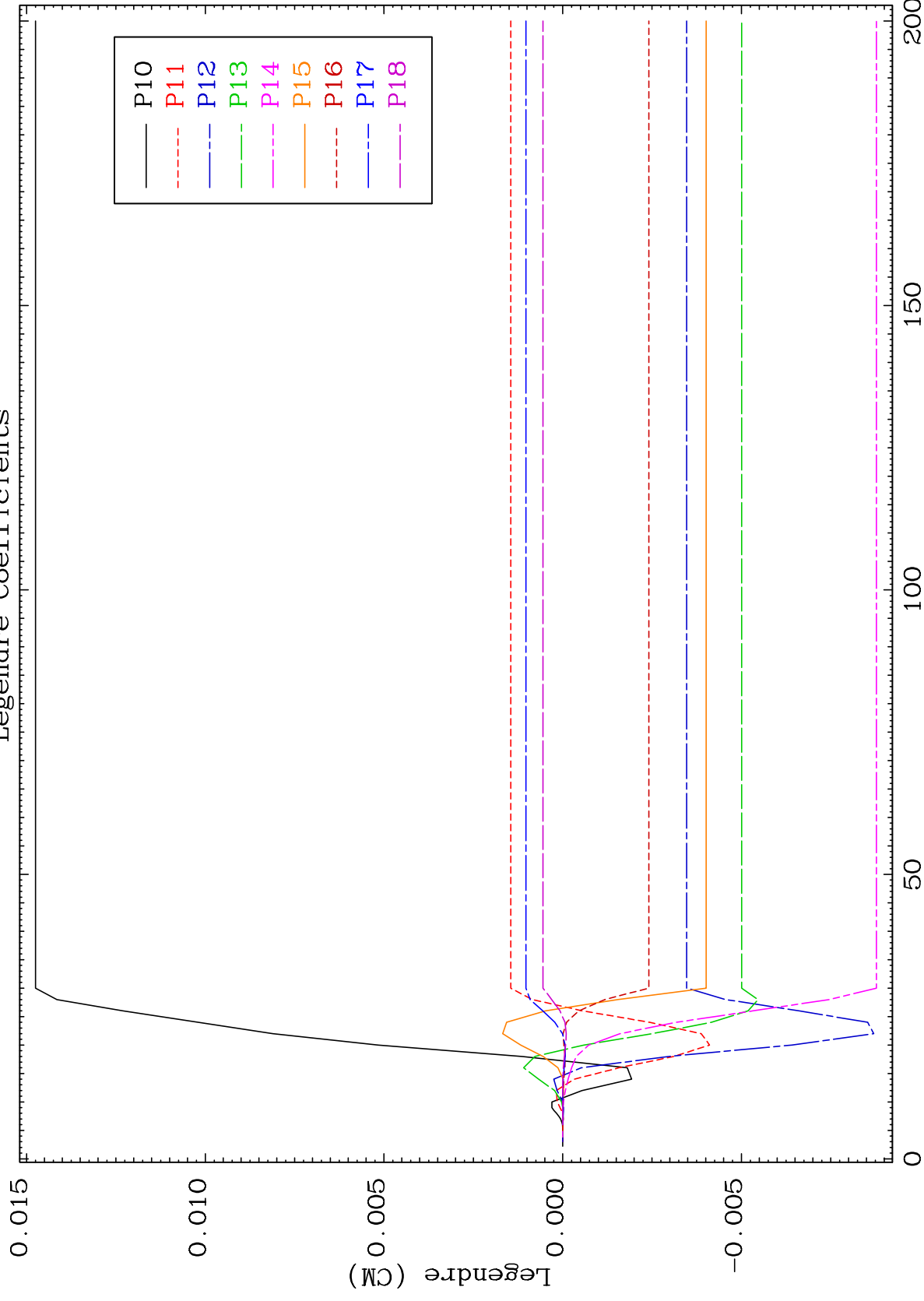




MAT 5801

MT= 63 (n, n') Level
Legendre Coefficients

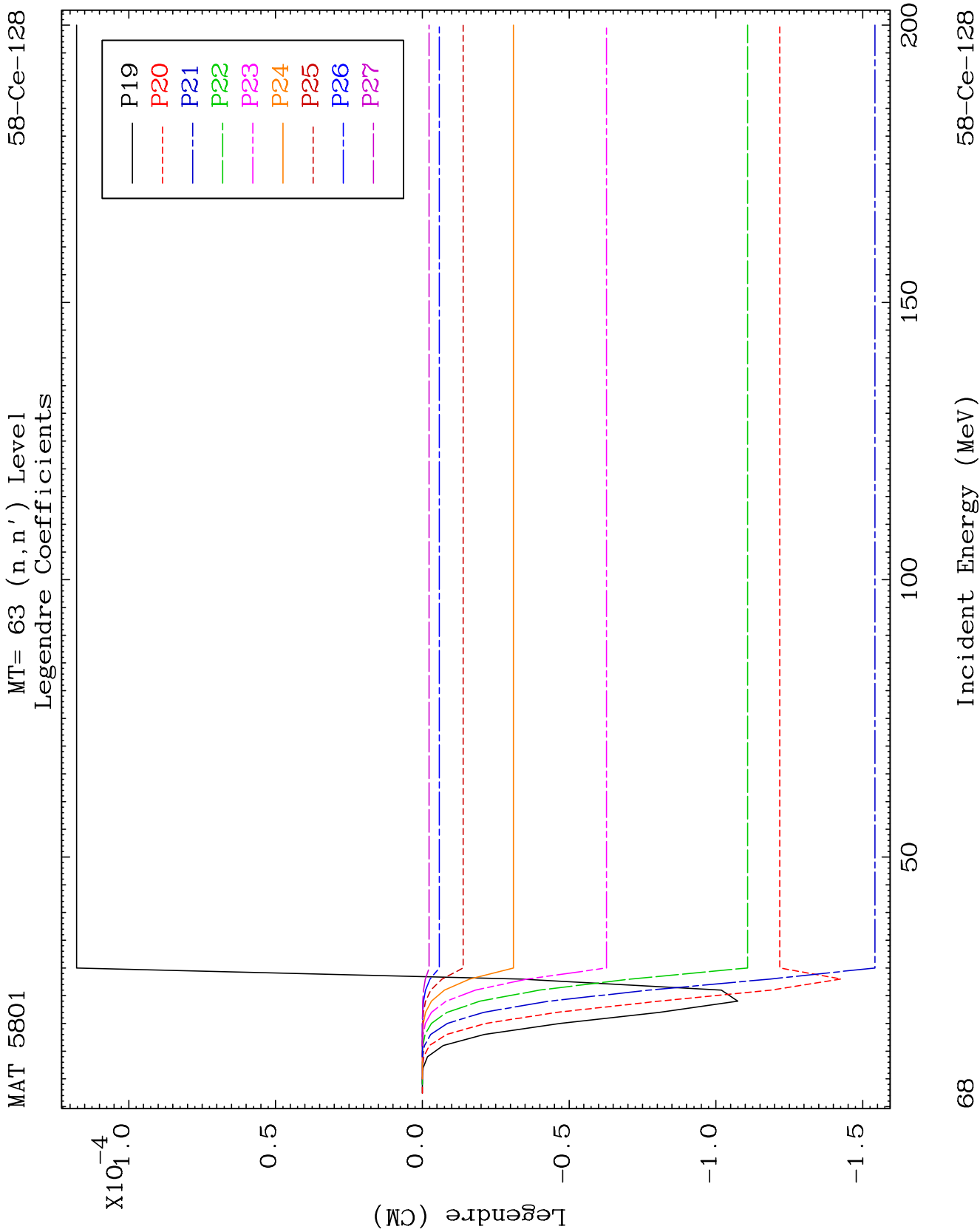
58-Ce-128

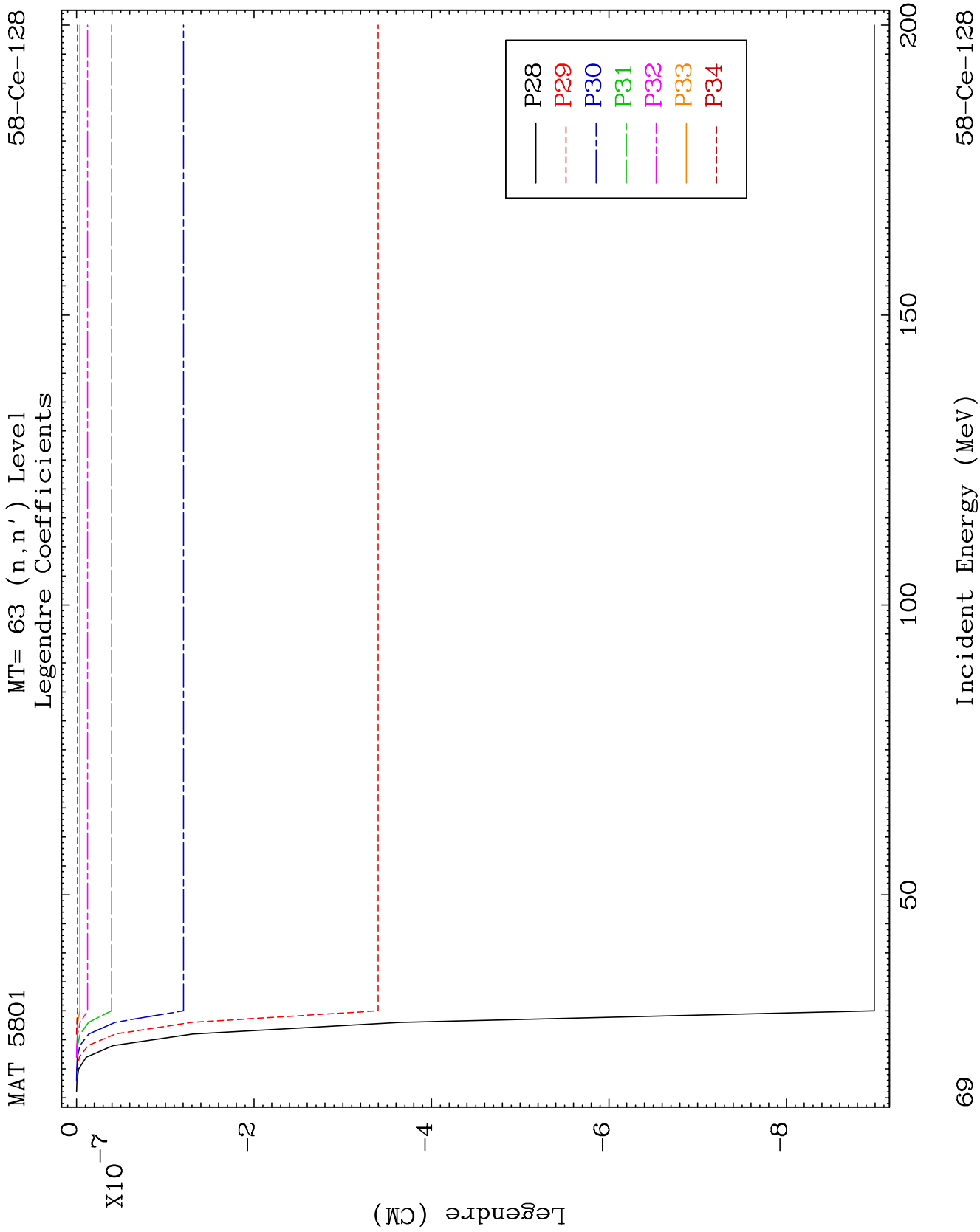


29

Incident Energy (MeV)

58-Ce-128

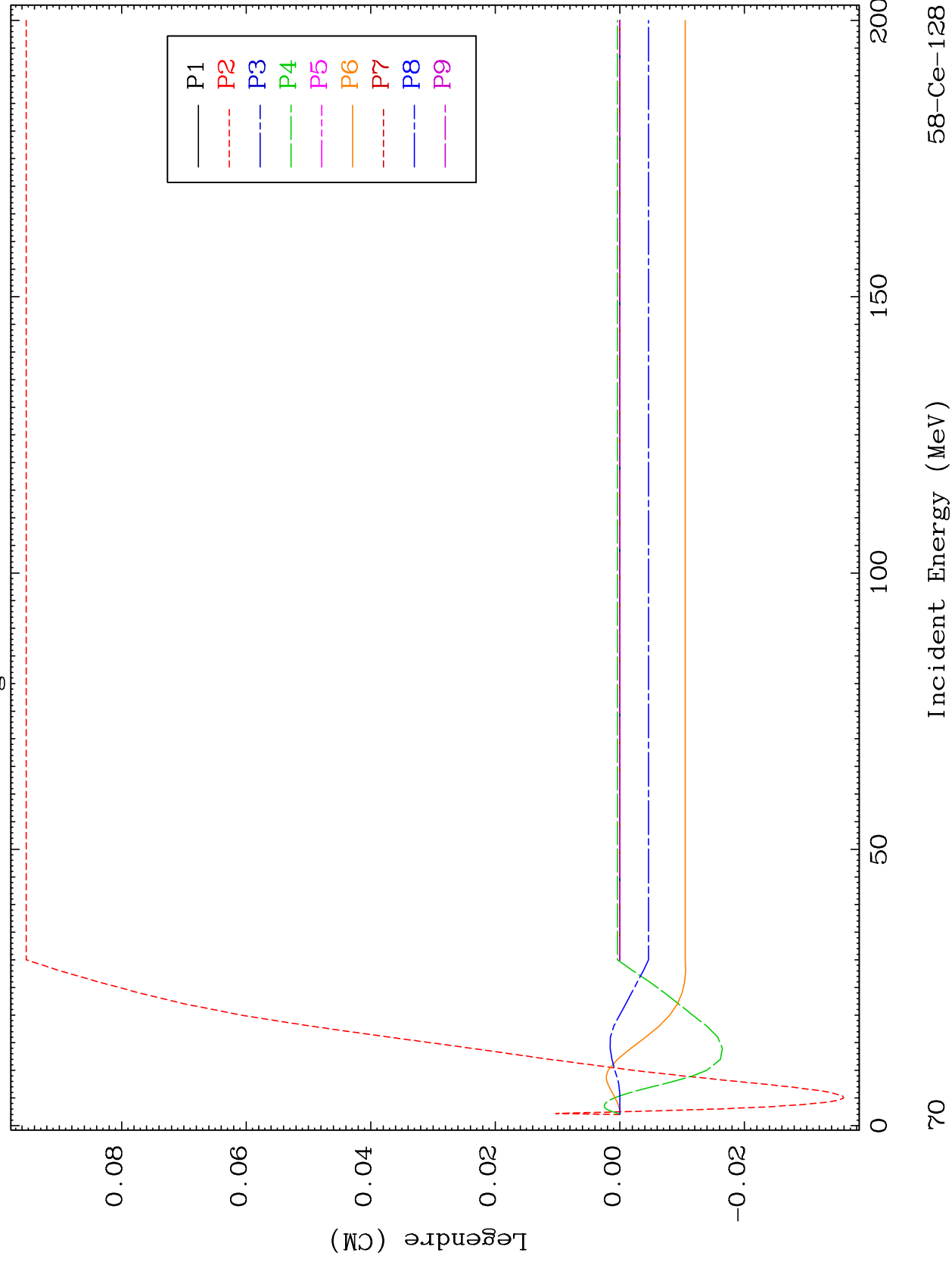


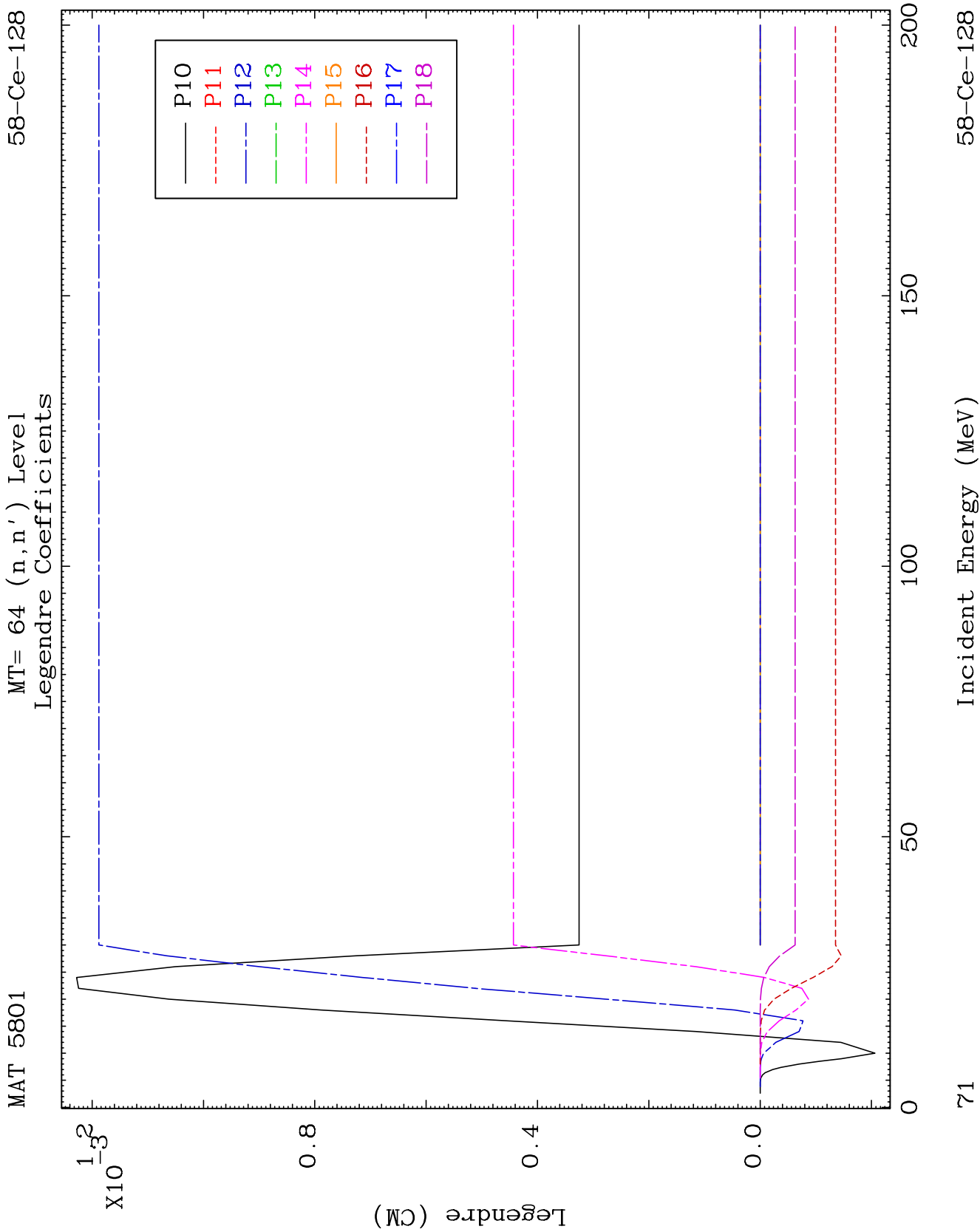


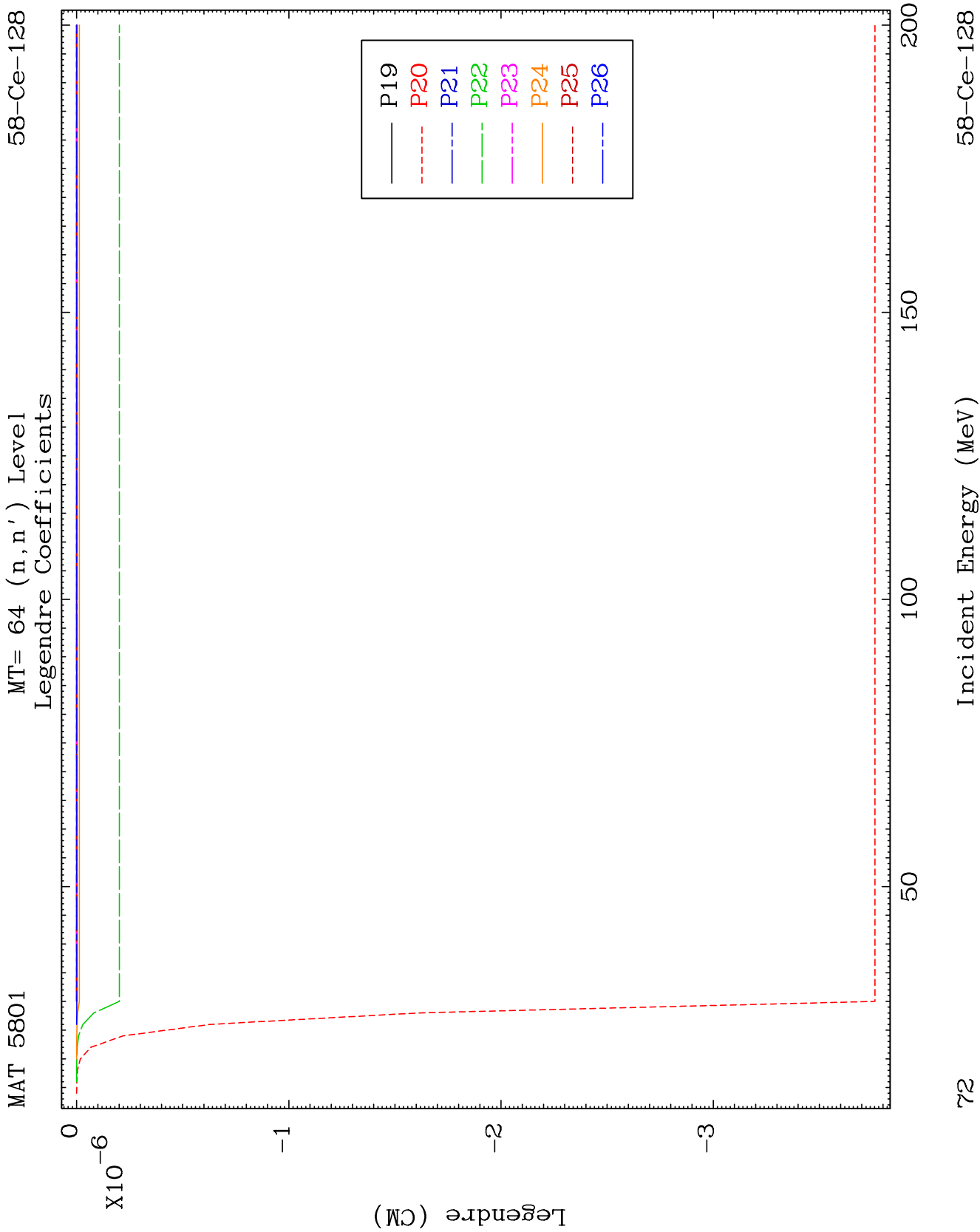
MAT 5801

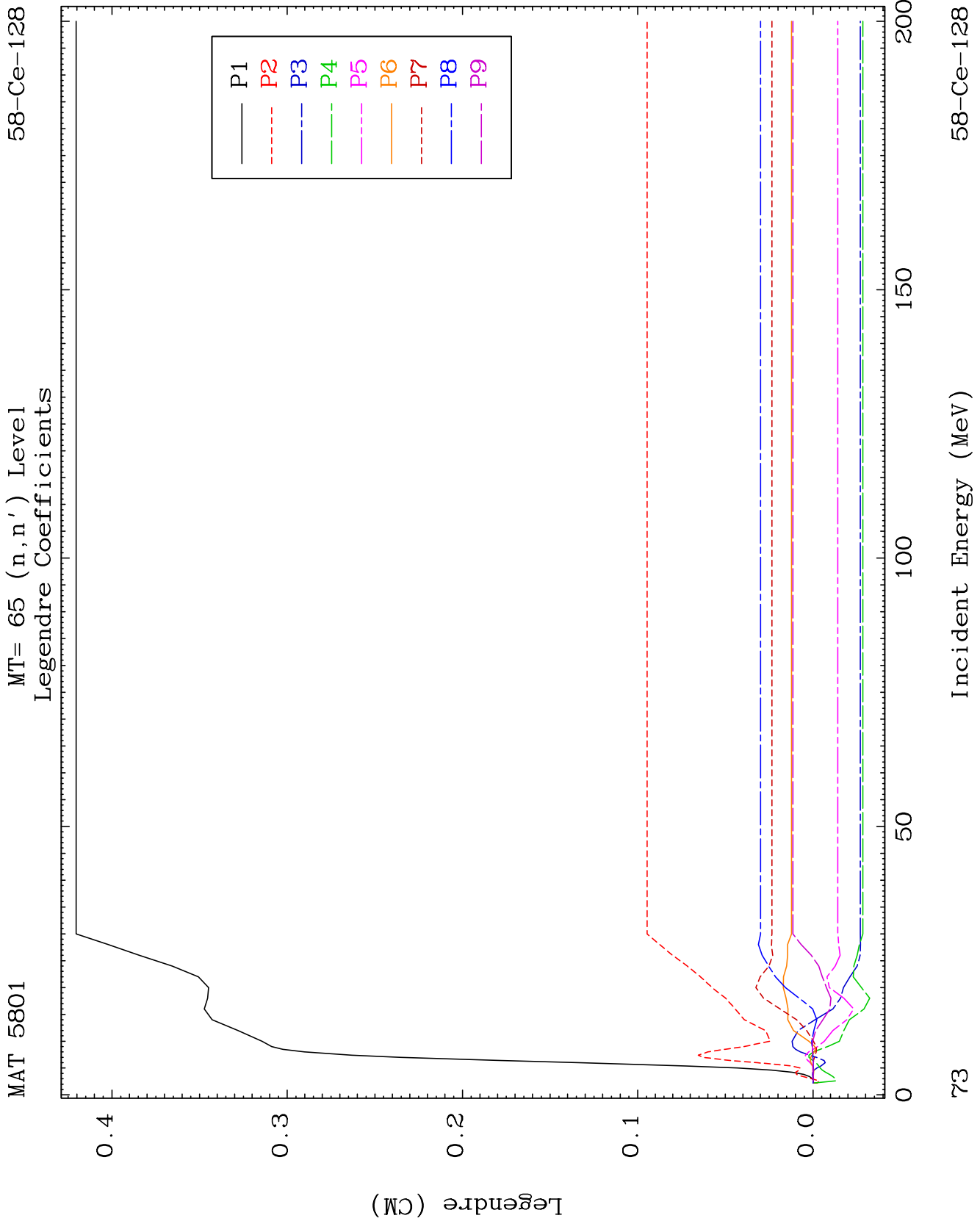
MT= 64 (n, n') Level
Legendre Coefficients

58-Ce-128





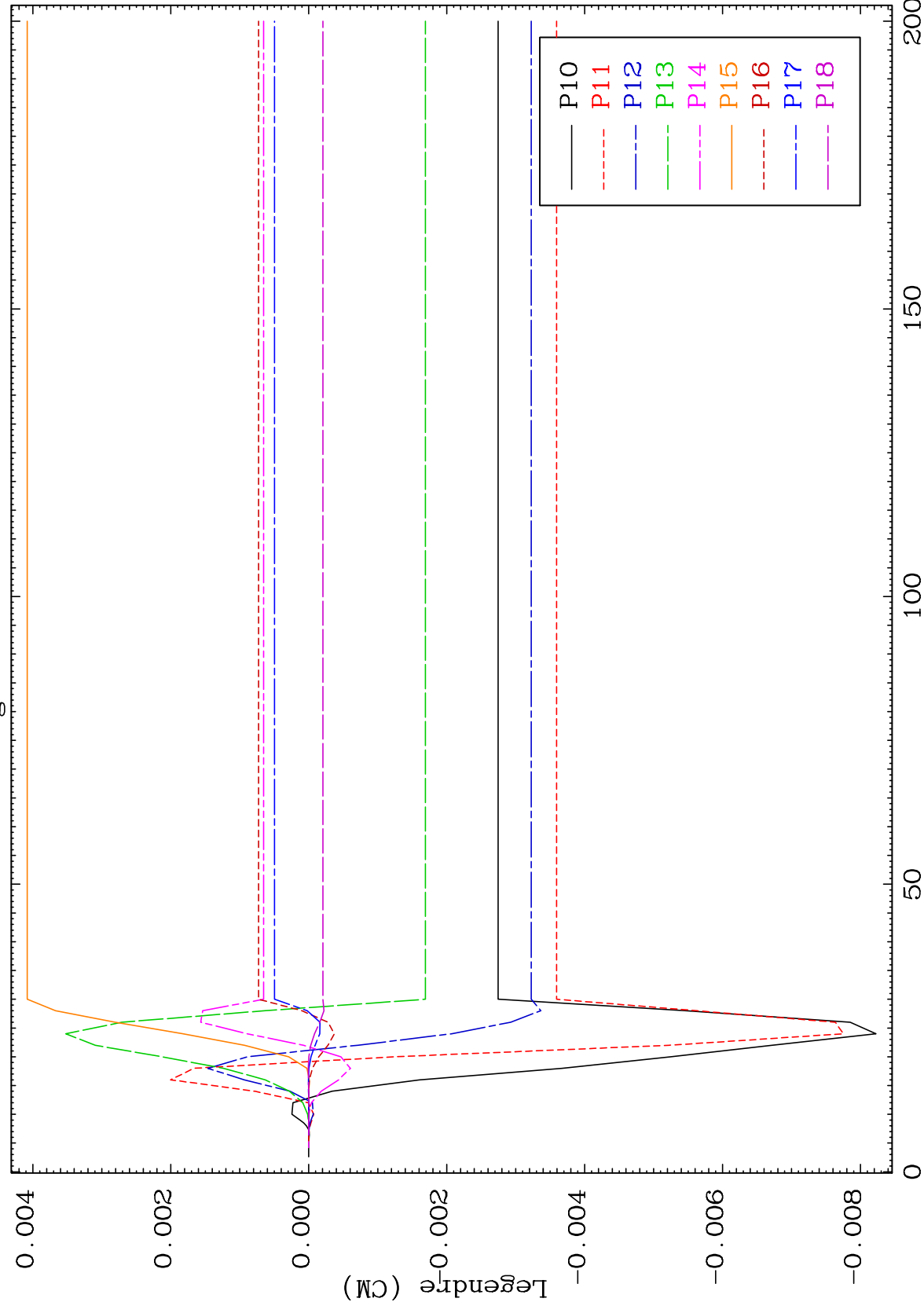




MAT 5801

MT= 65 (n, n') Level
Legendre Coefficients

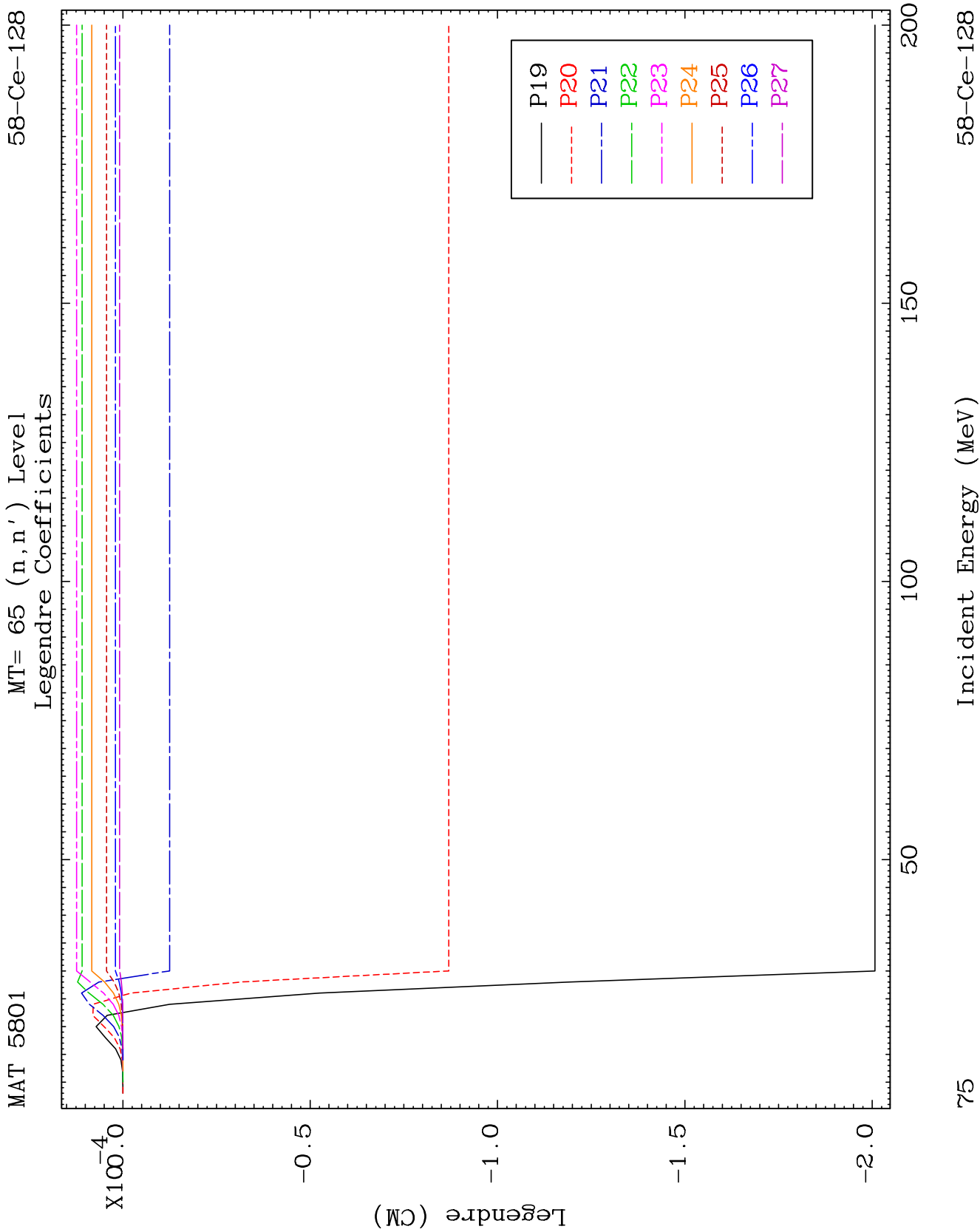
58-Ce-128

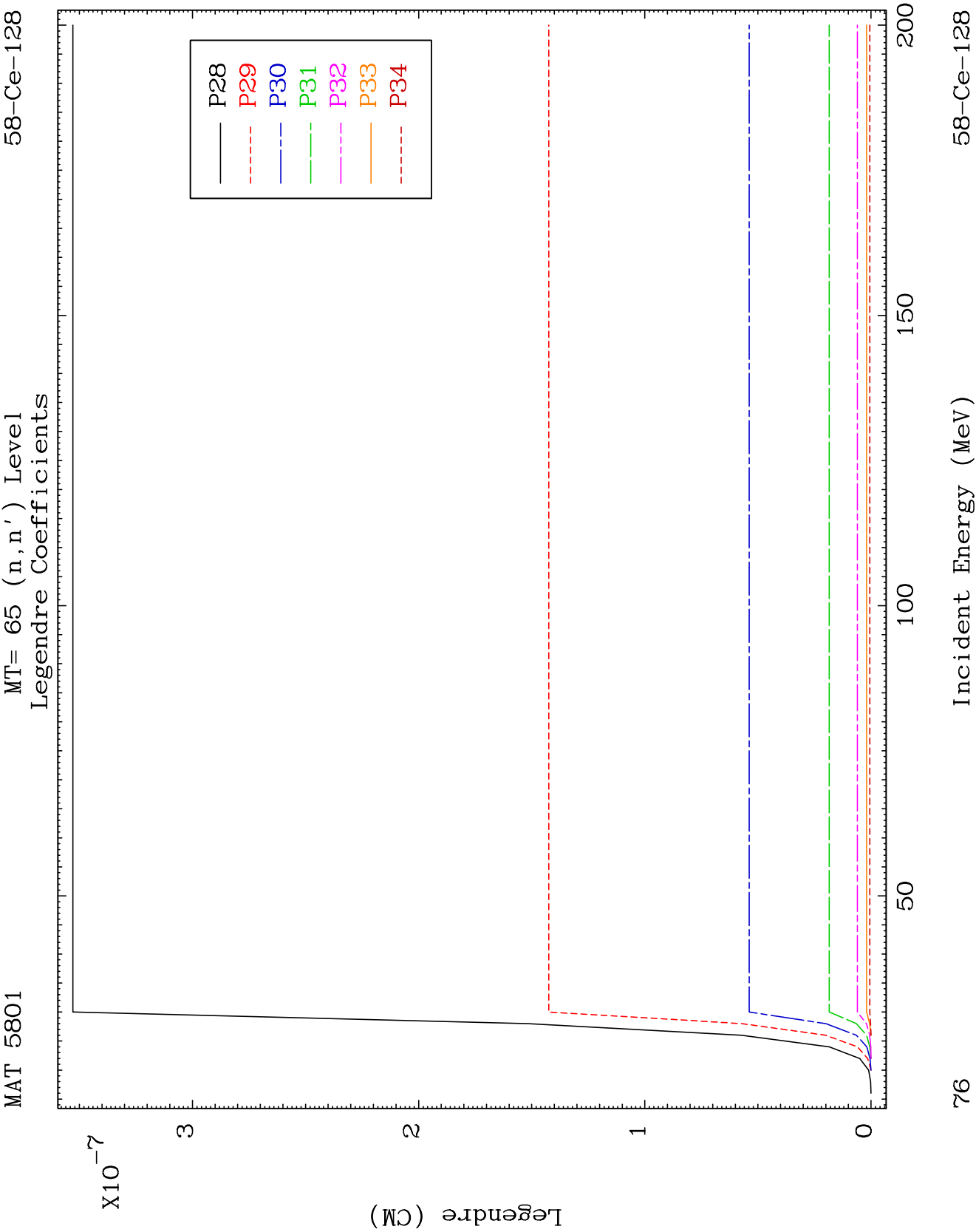


42

Incident Energy (MeV)

58-Ce-128

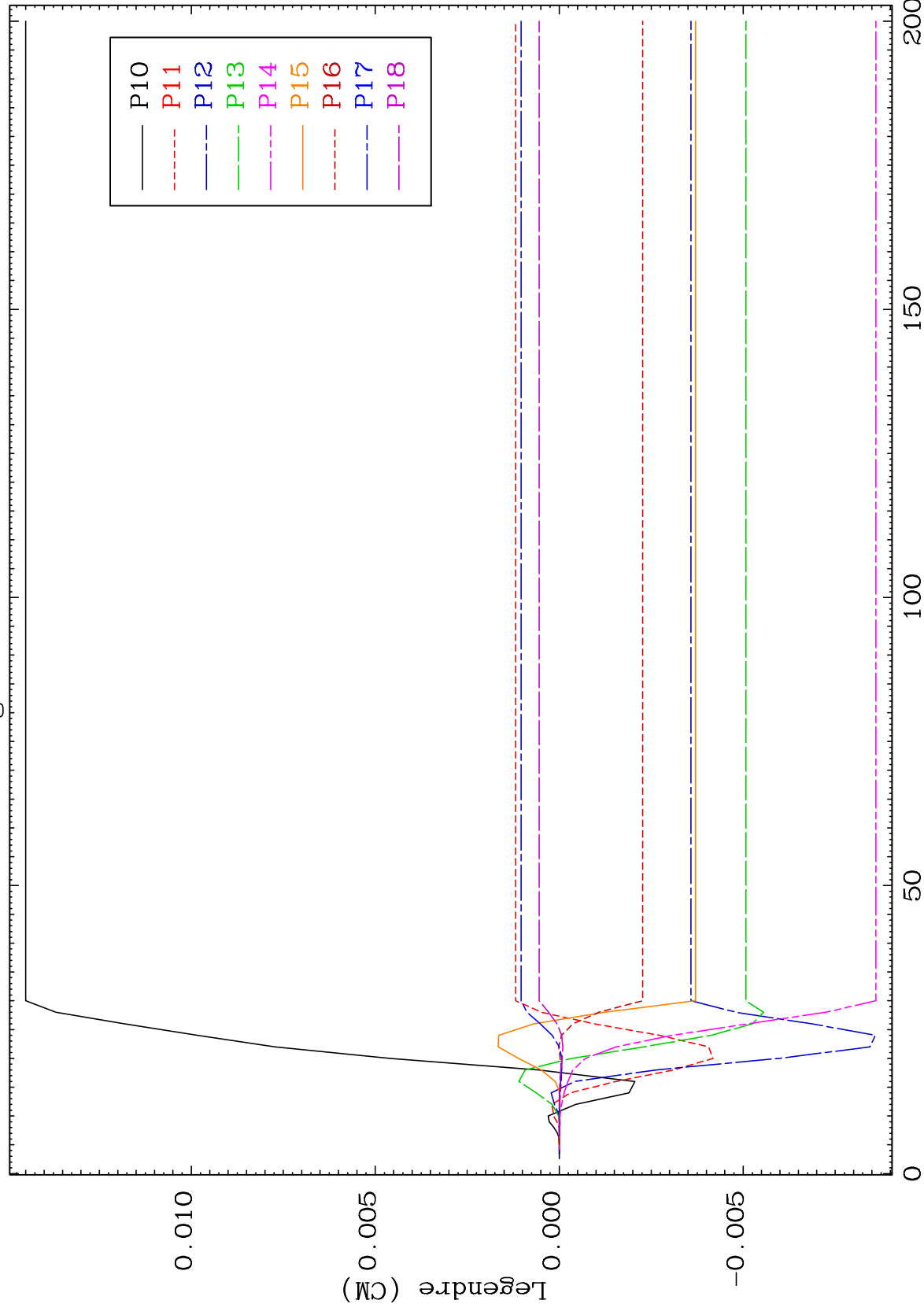




MAT 5801

MT= 66 (n, n') Level
Legendre Coefficients

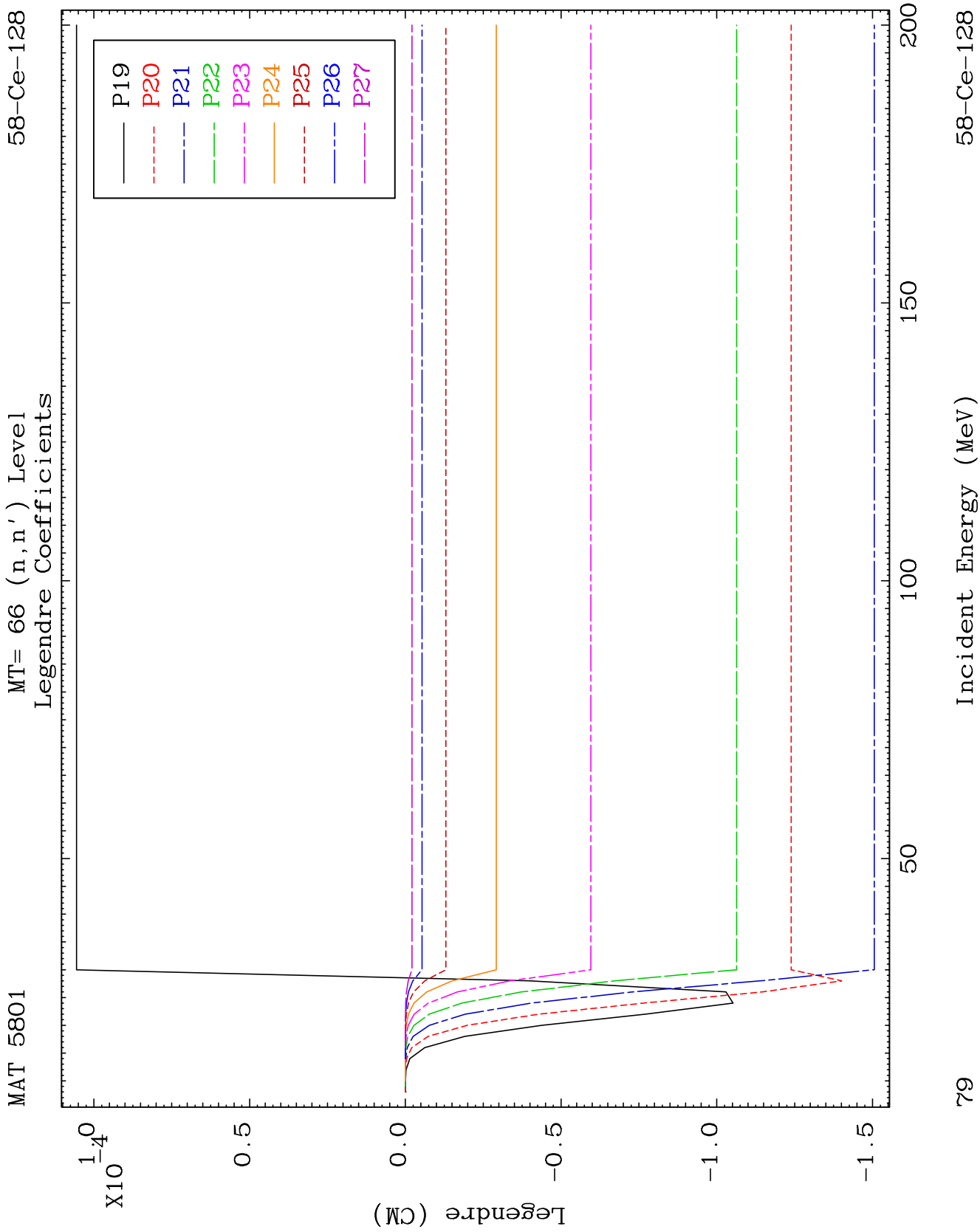
58-Ce-128

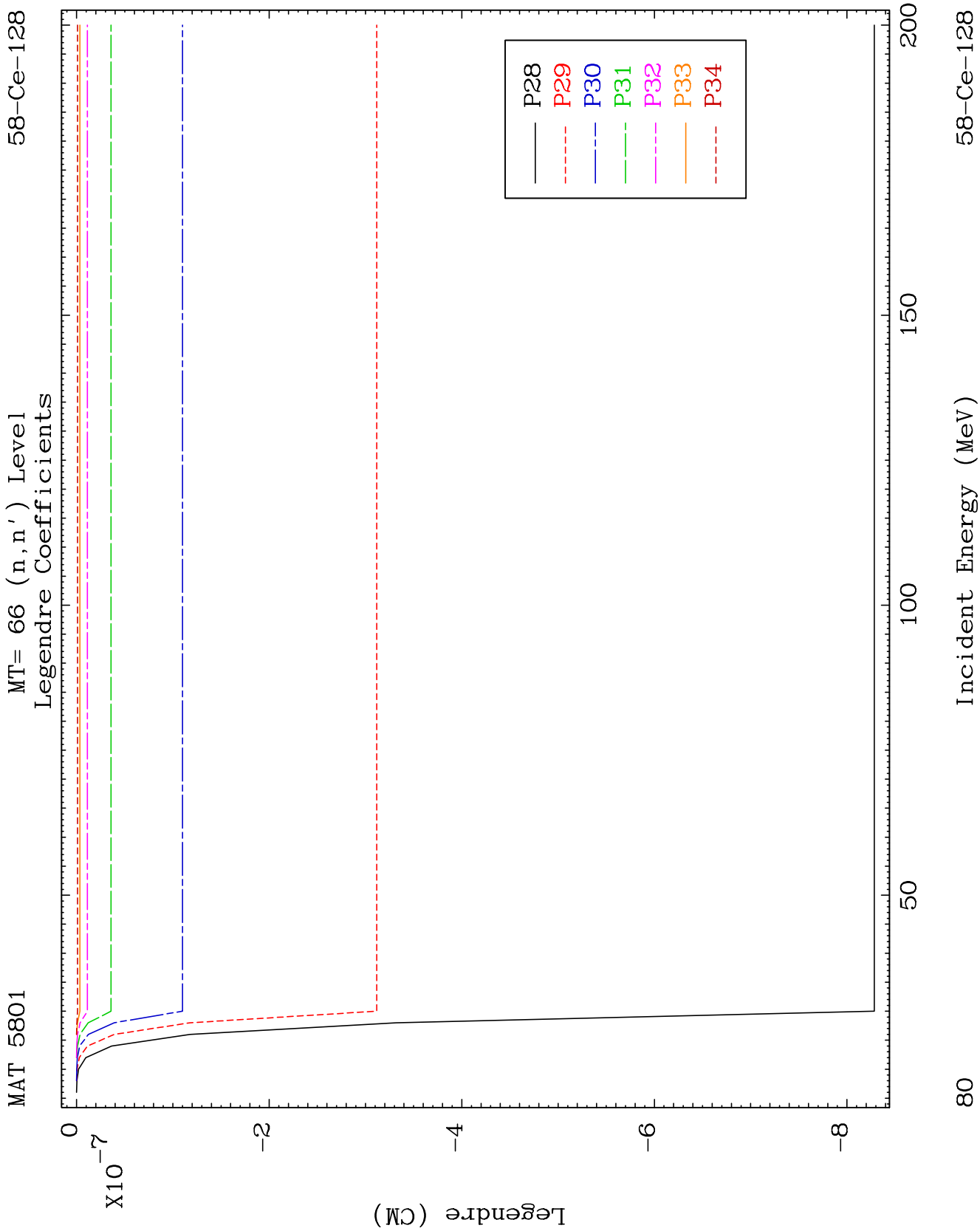


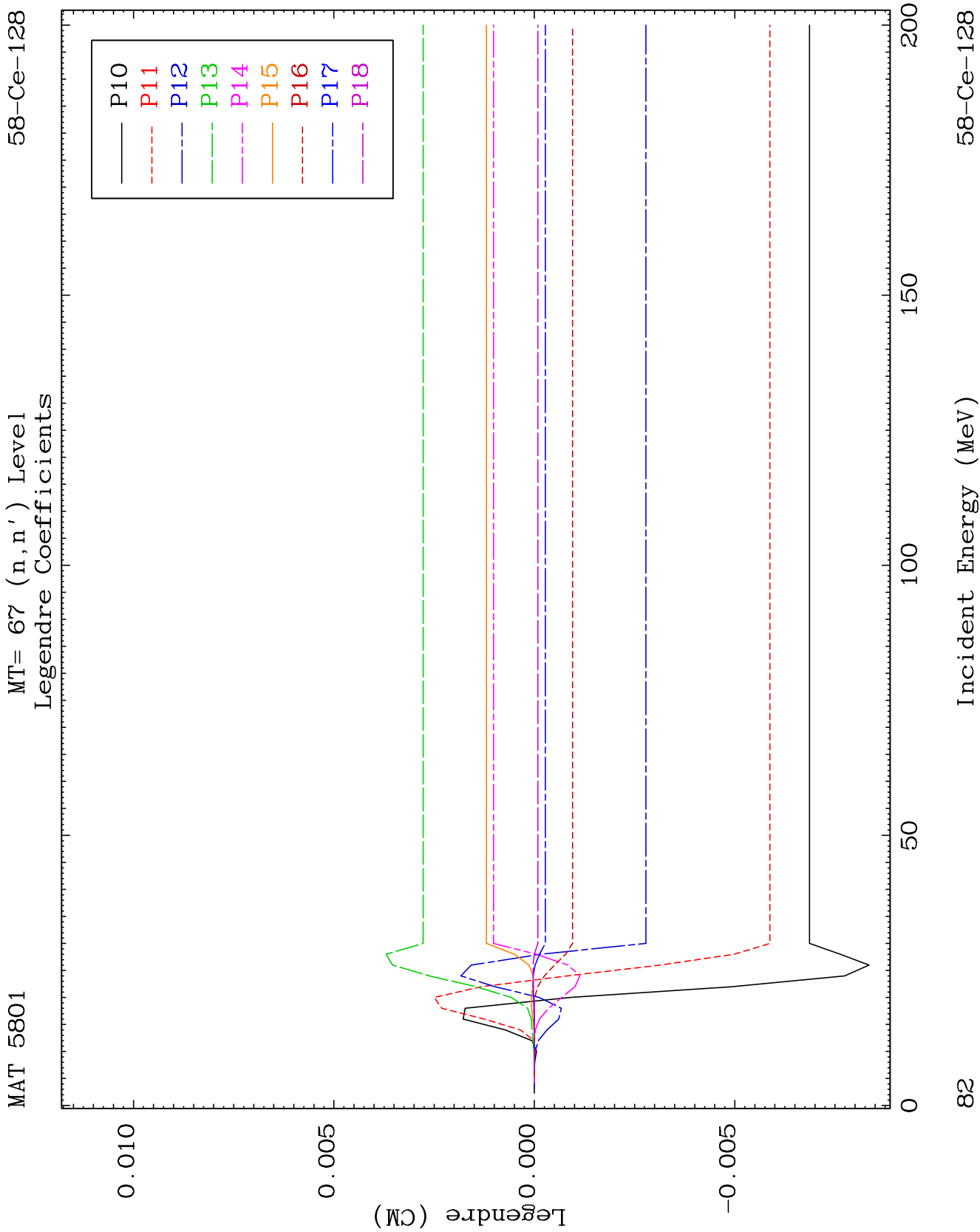
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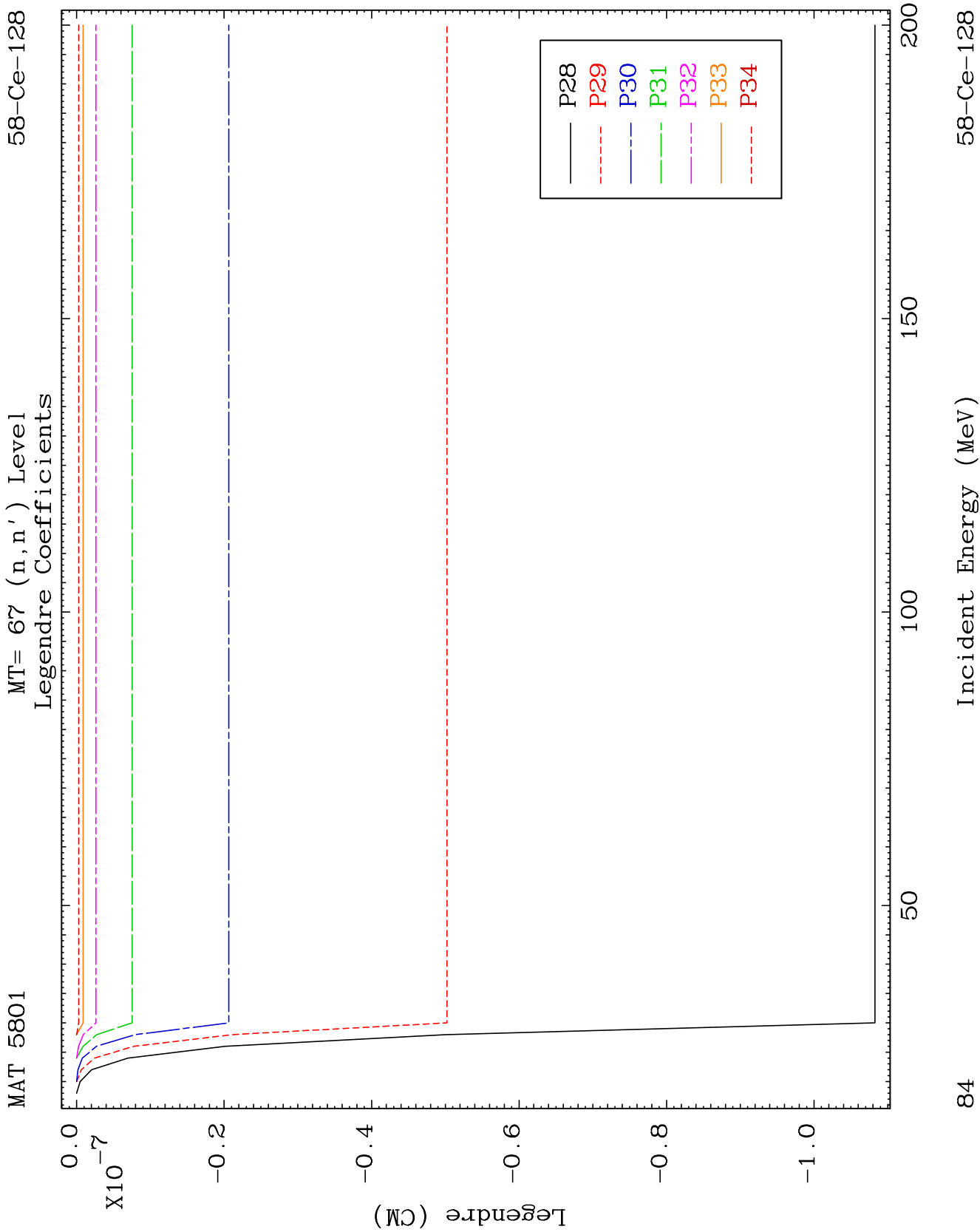
Incident Energy (MeV)

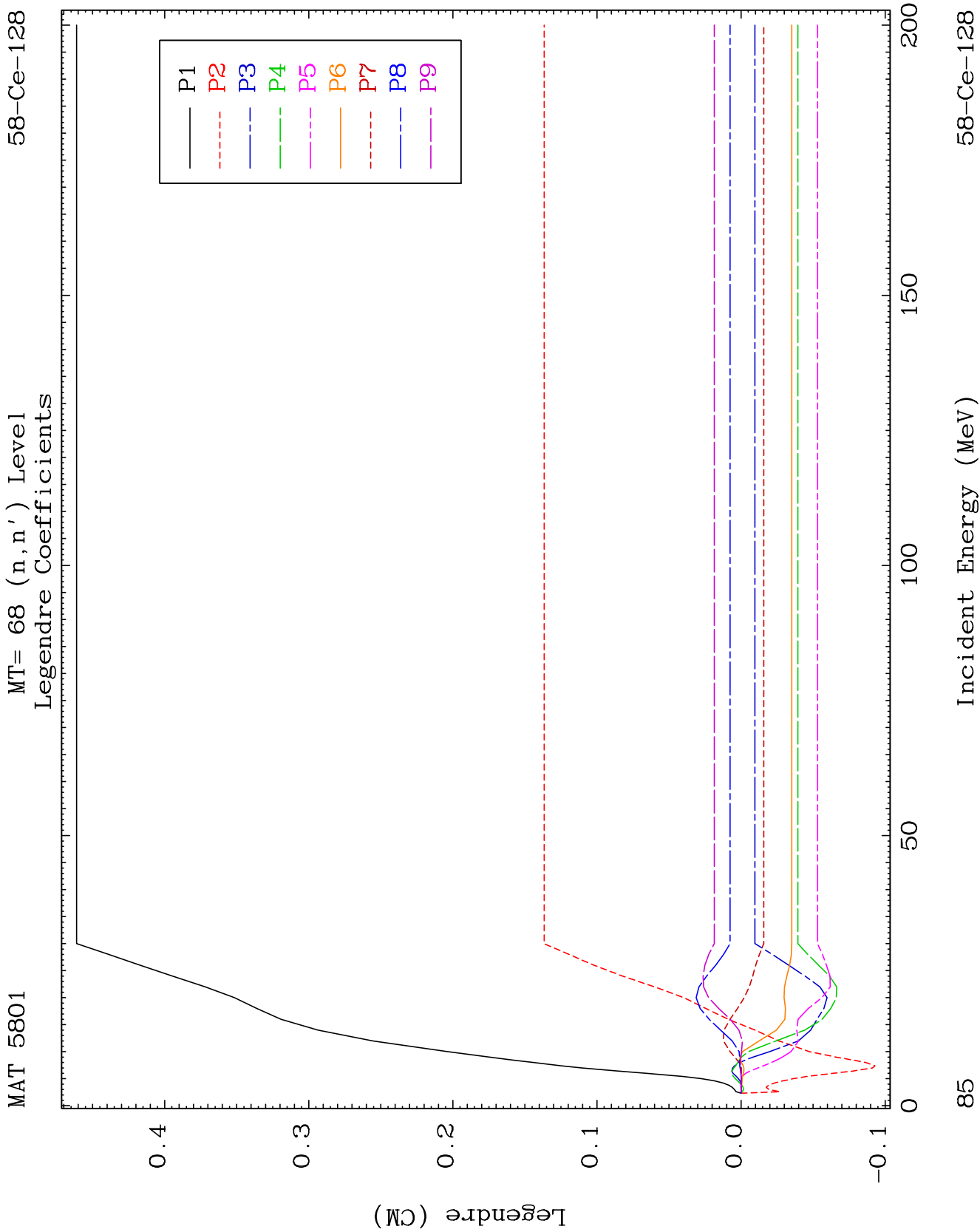
58-Ce-128







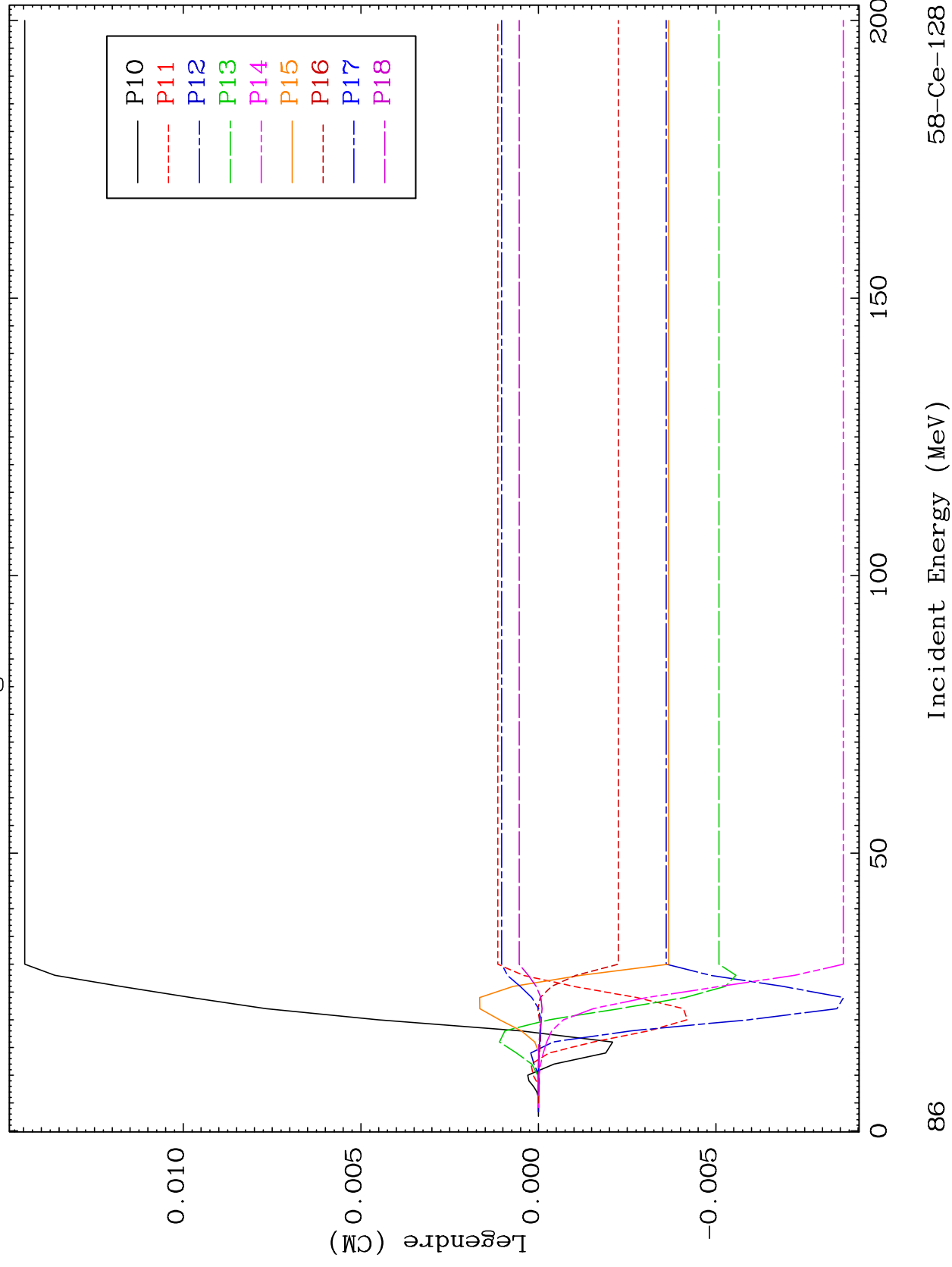


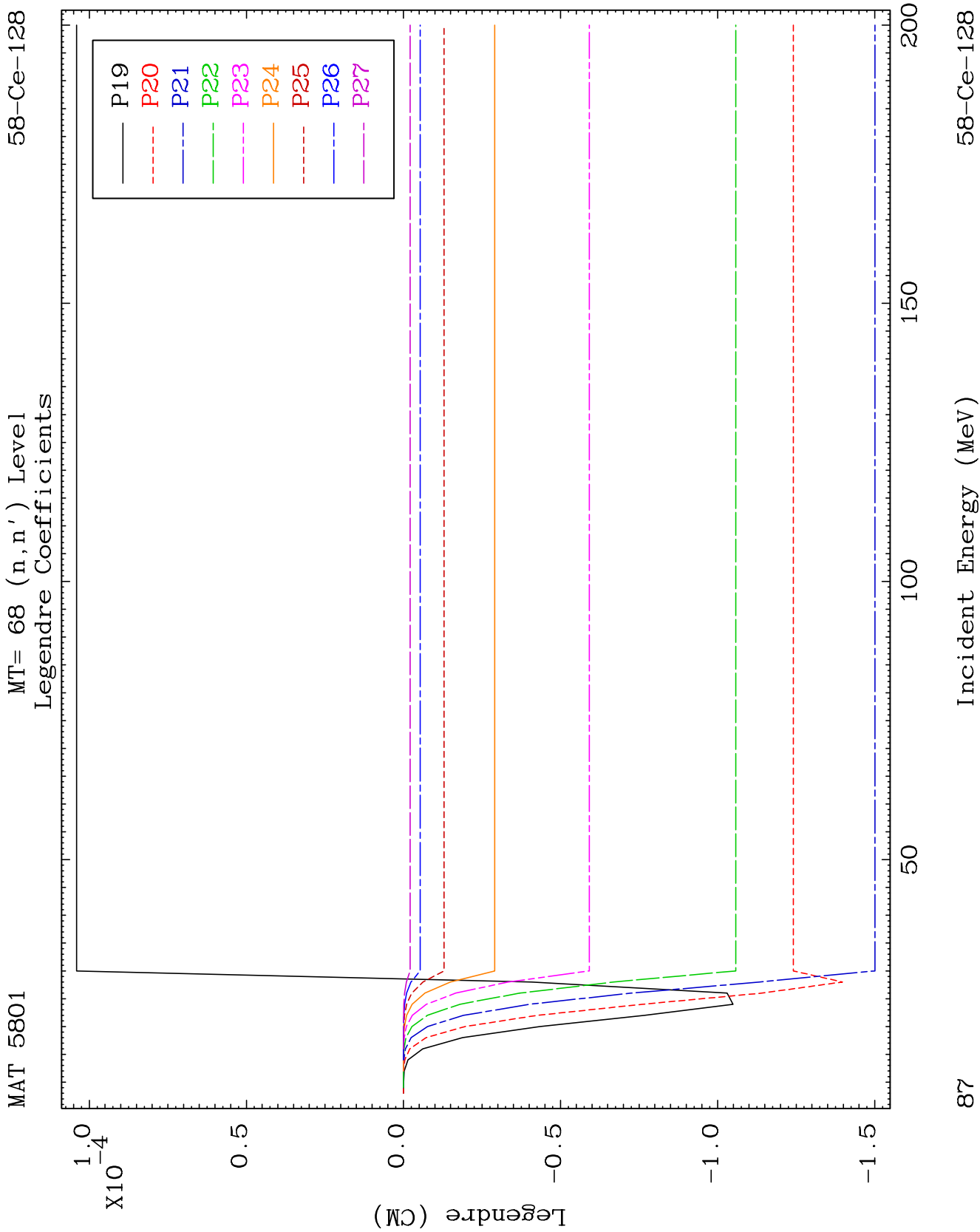


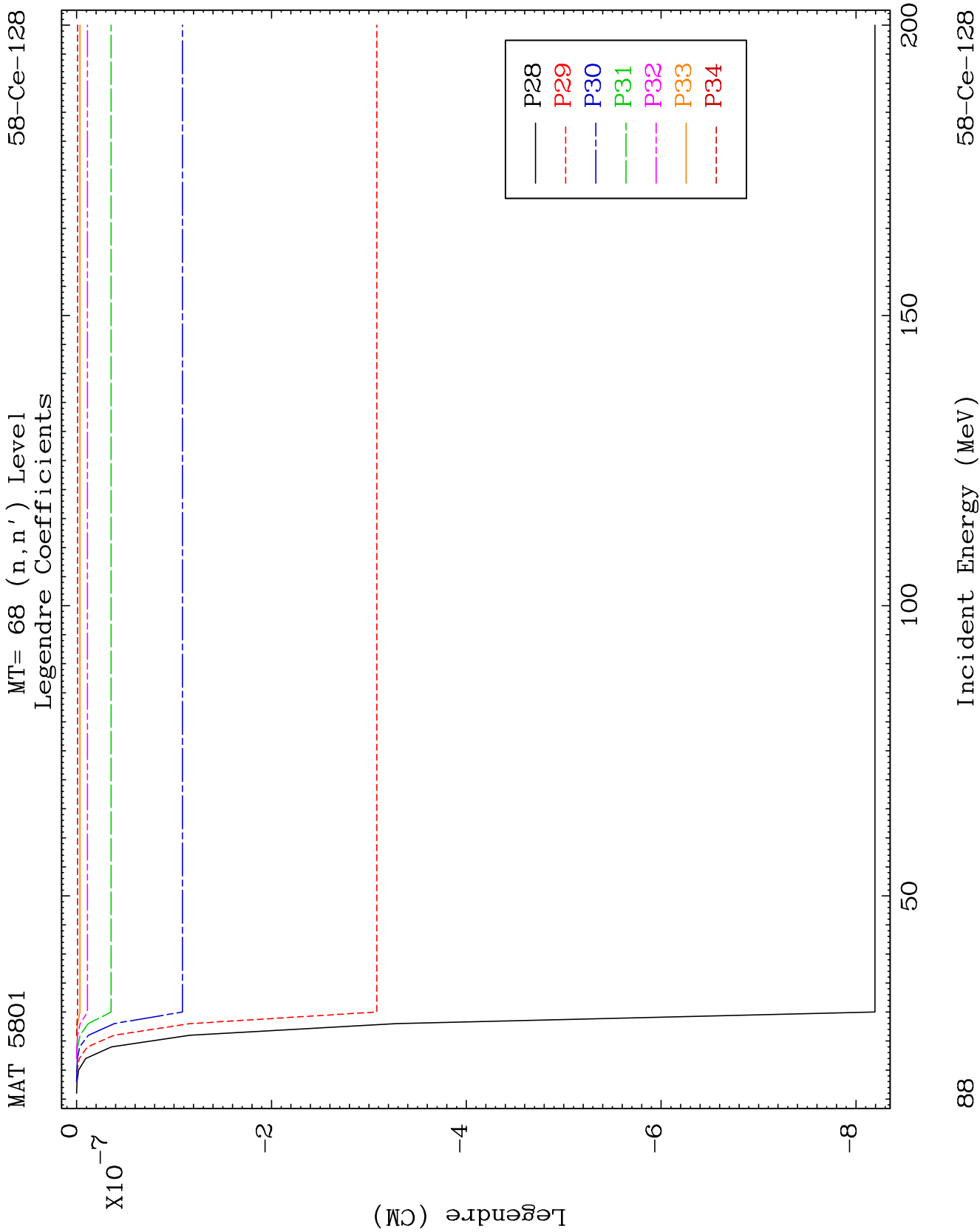
MAT 5801

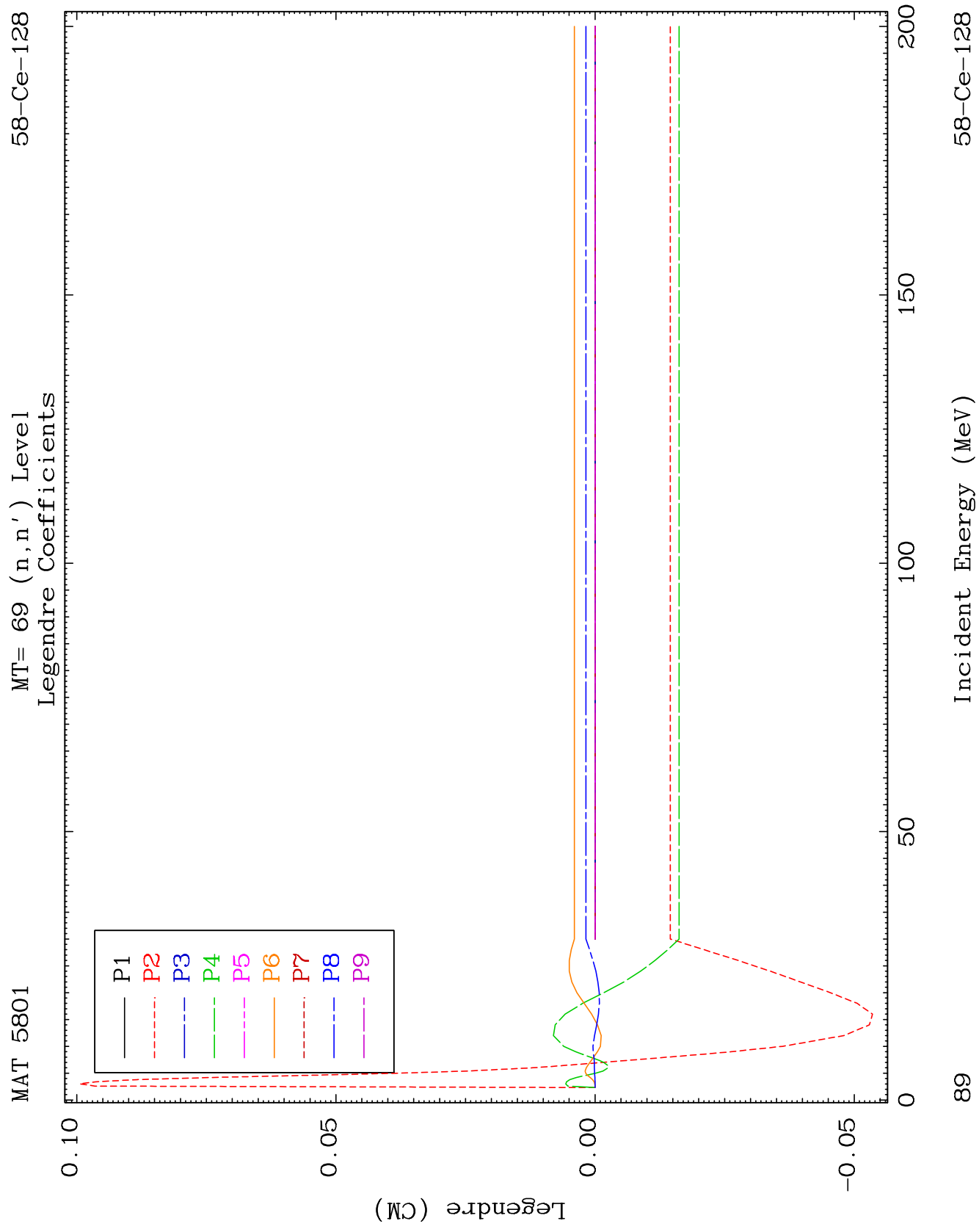
MT= 68 (n, n') Level
Legendre Coefficients

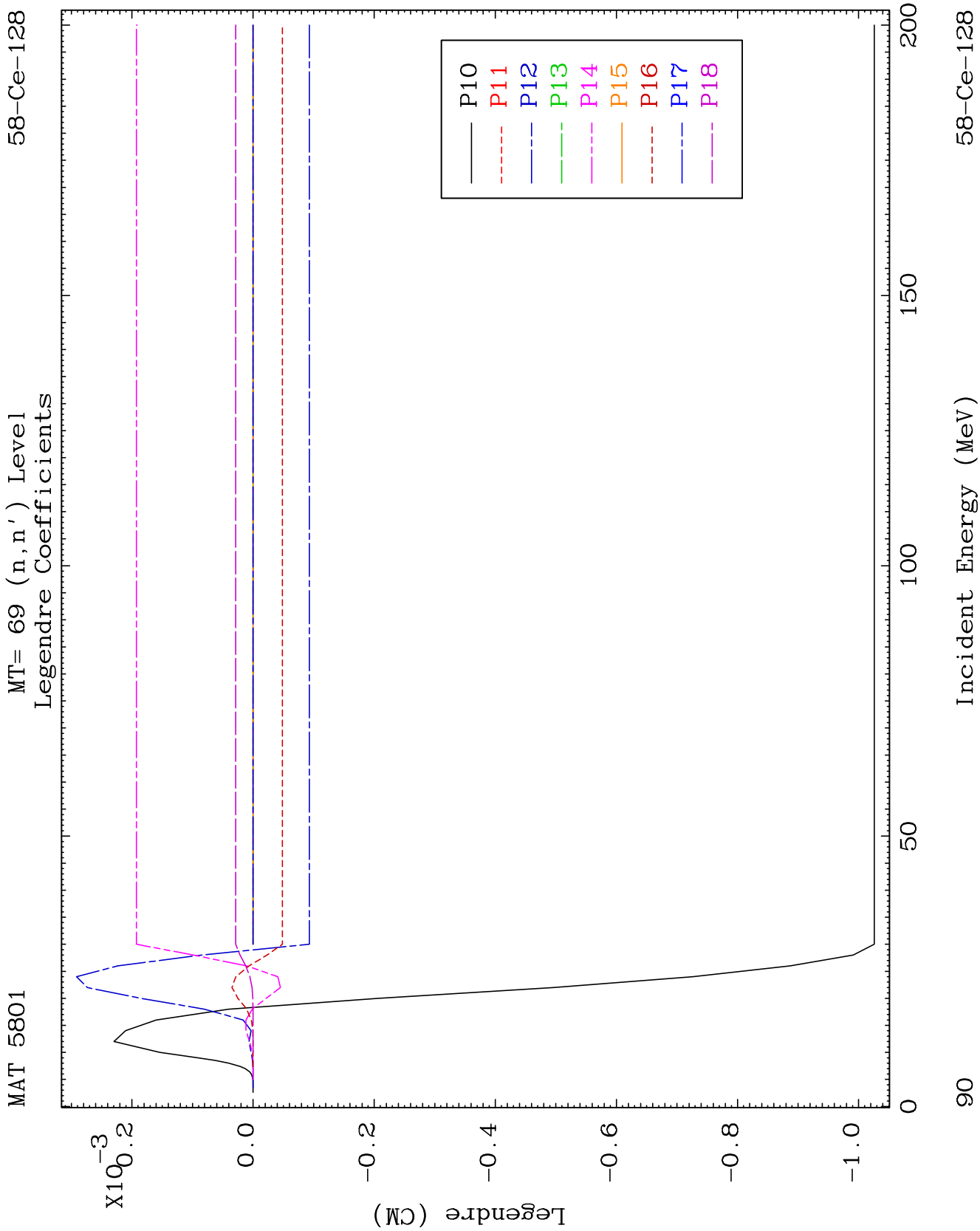
58-Ce-128

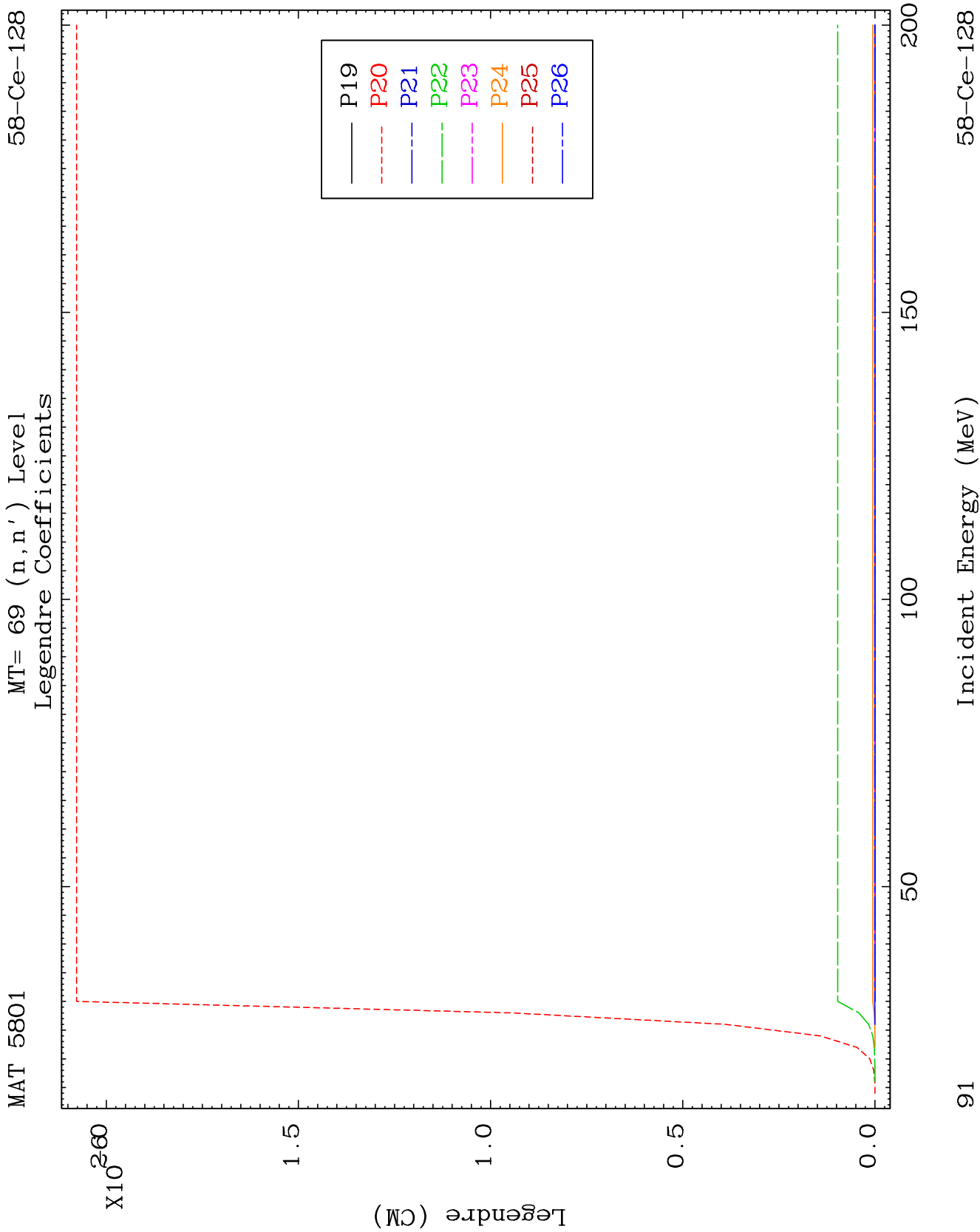


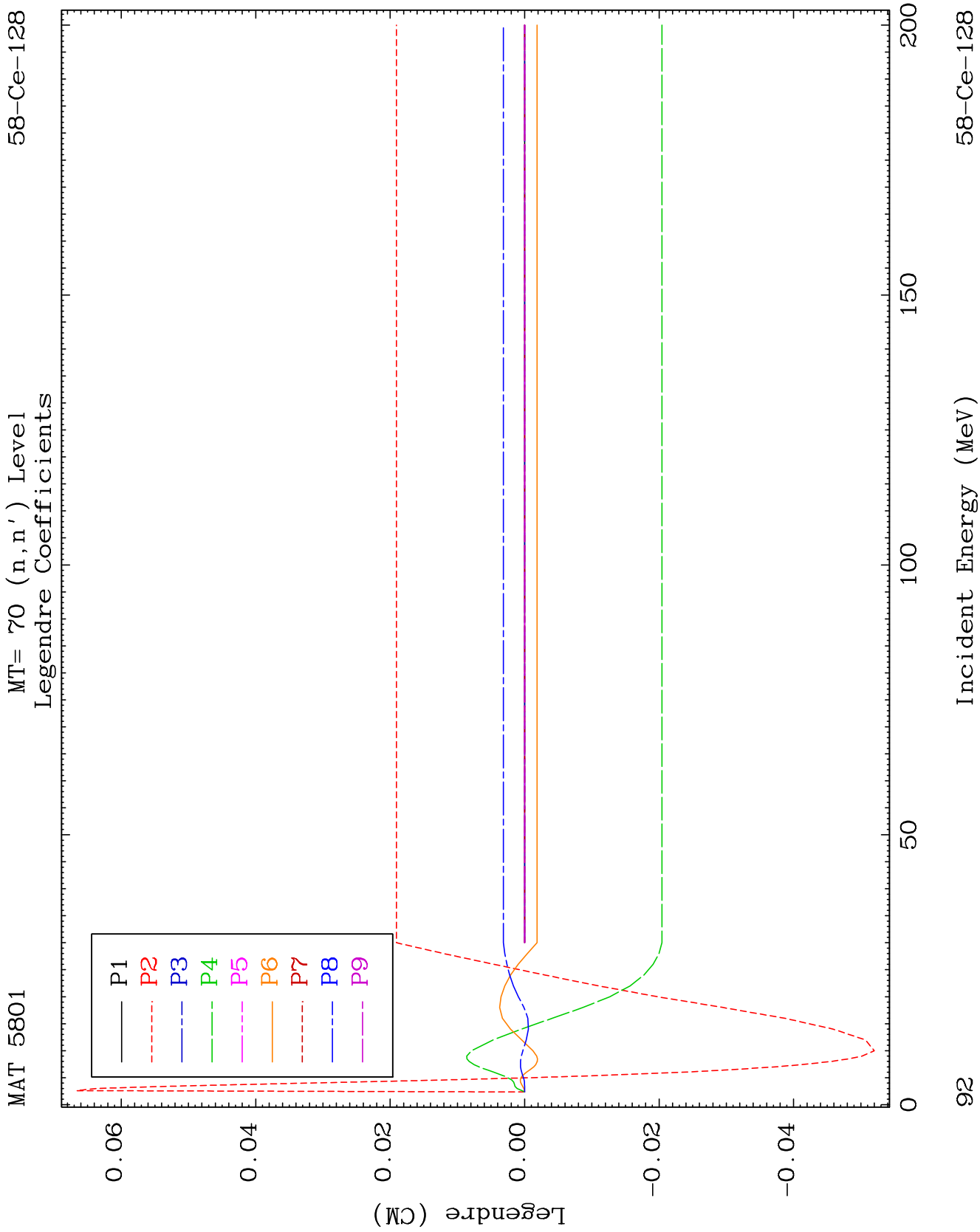








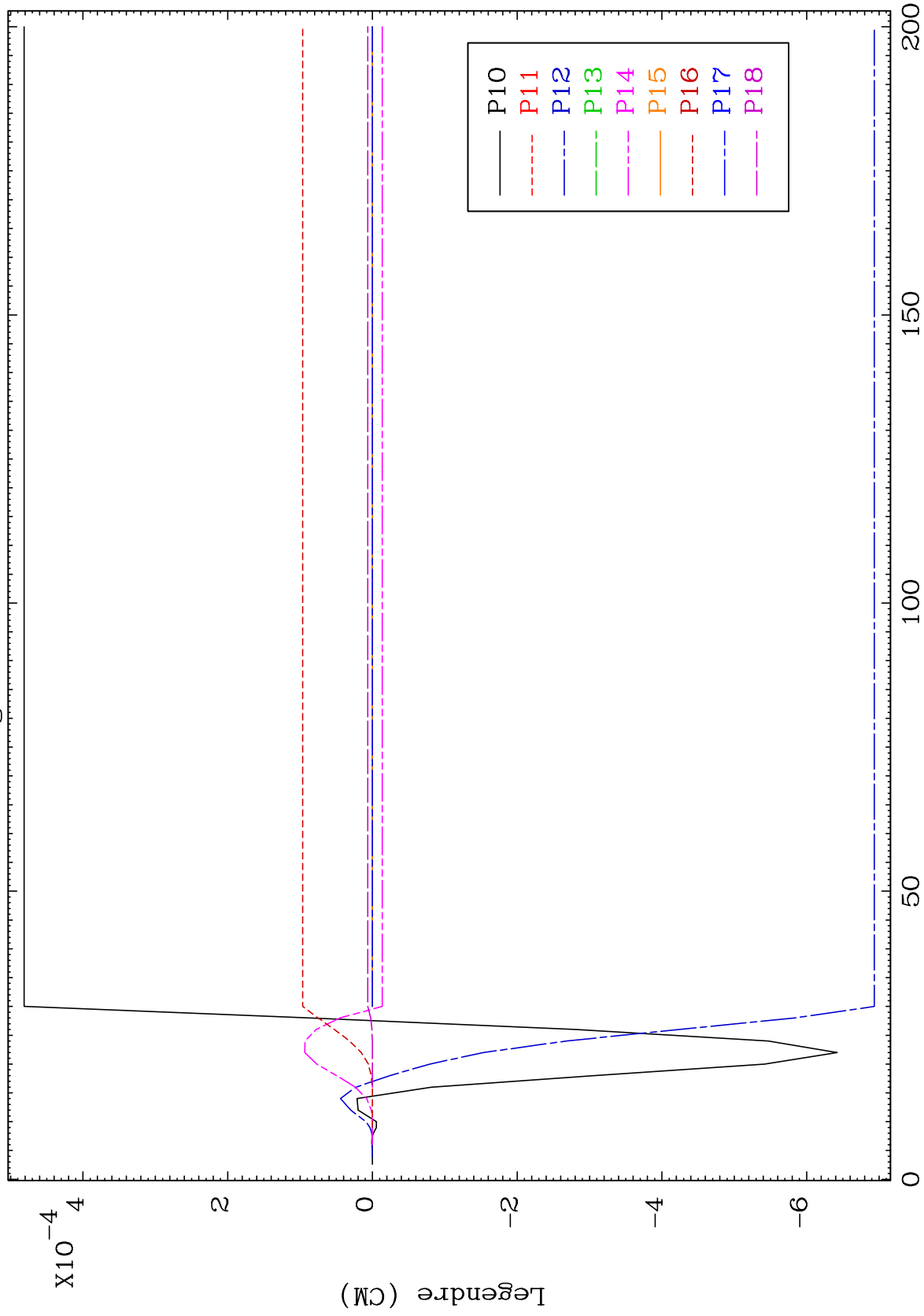




MAT 5801

MT= 70 (n,n') Level
Legendre Coefficients

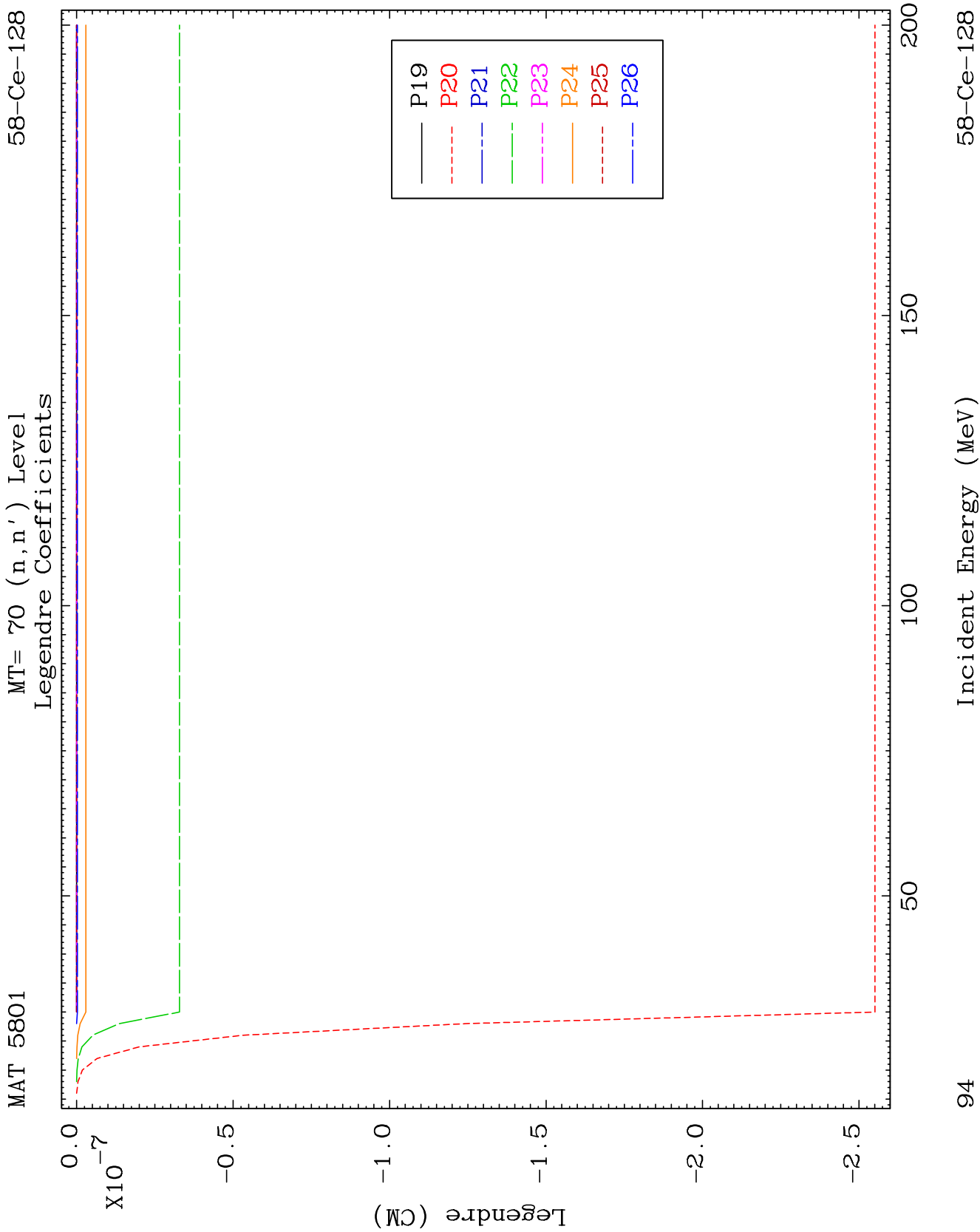
58-Ce-128

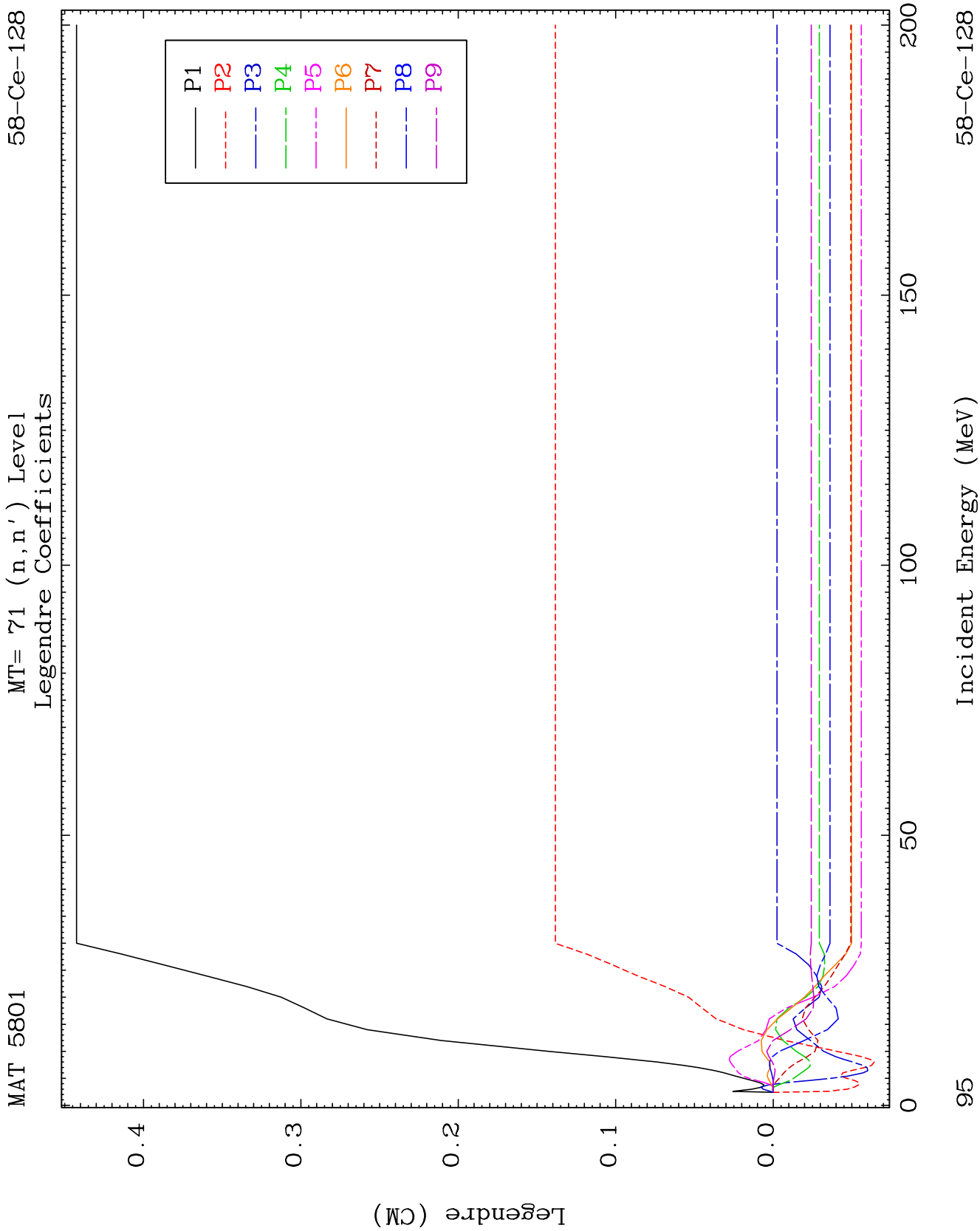


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Incident Energy (MeV)

58-Ce-128

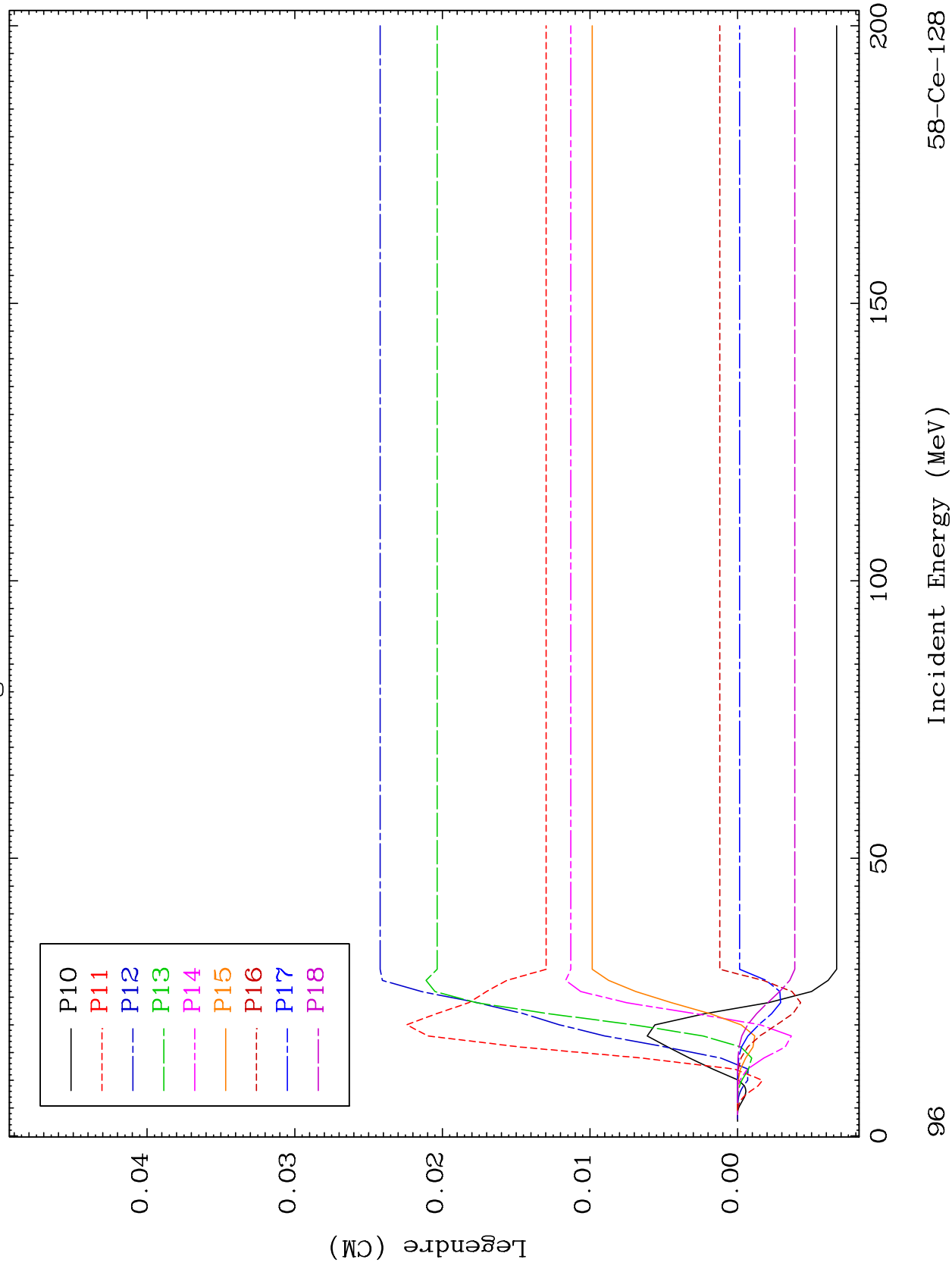


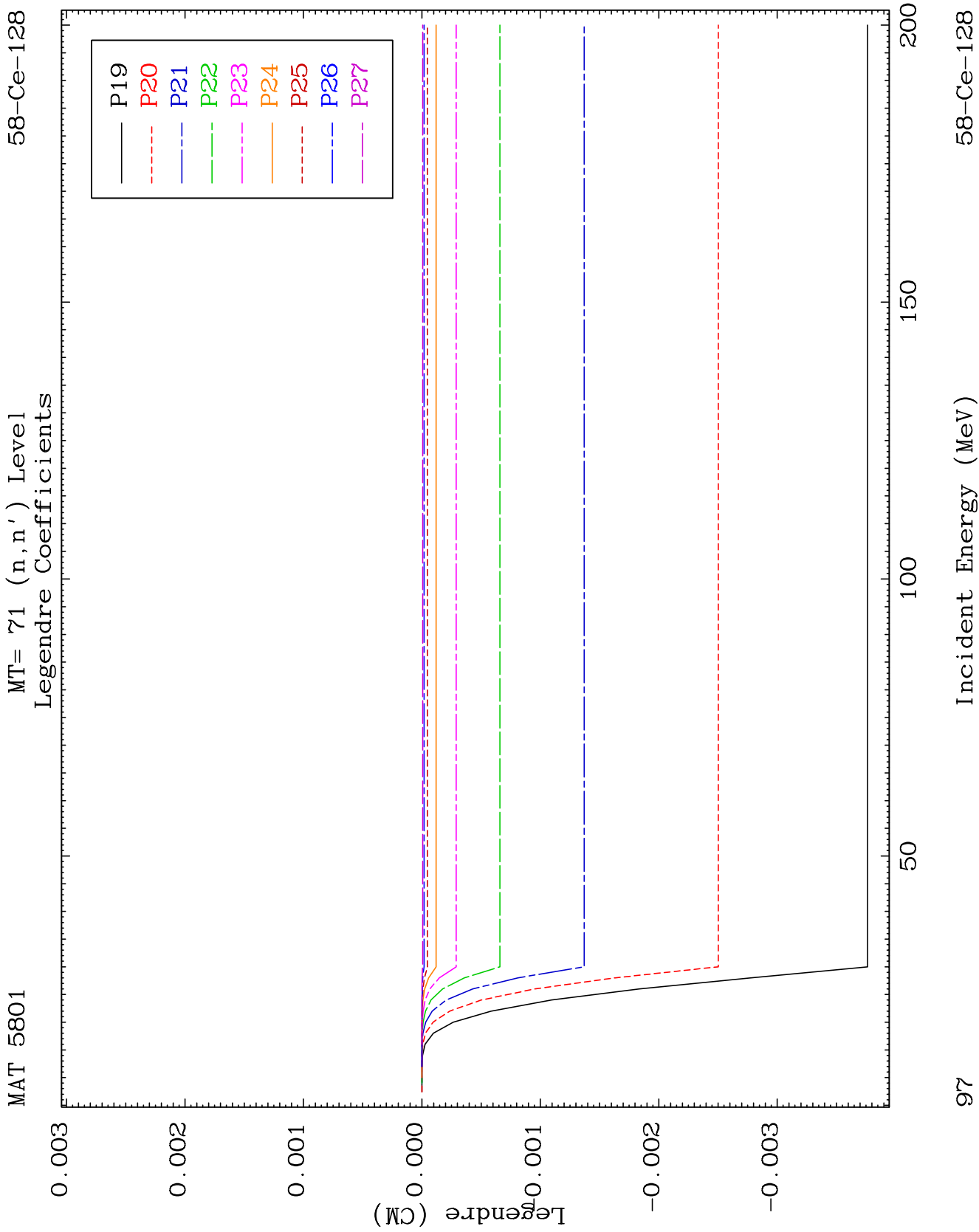


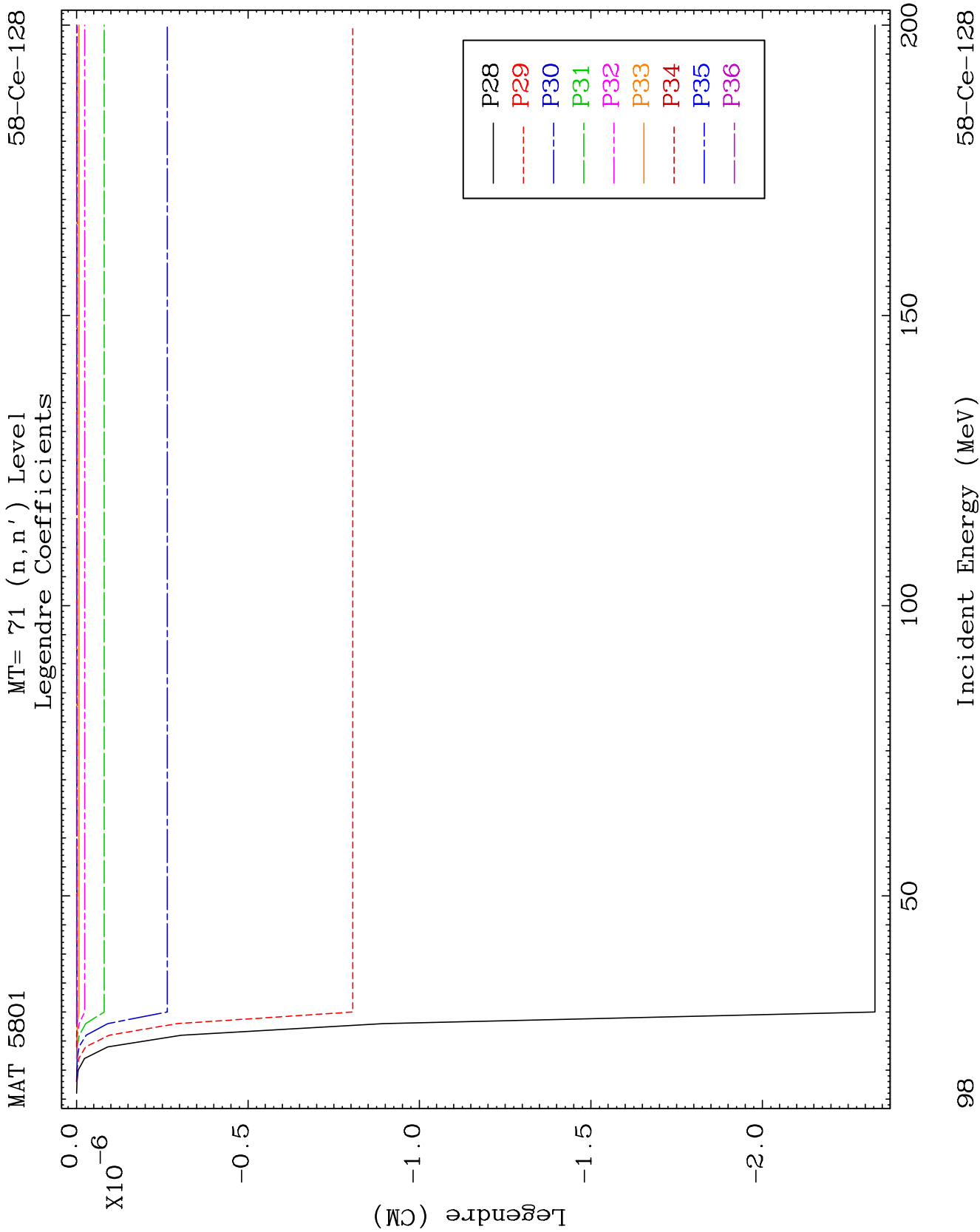
MAT 5801

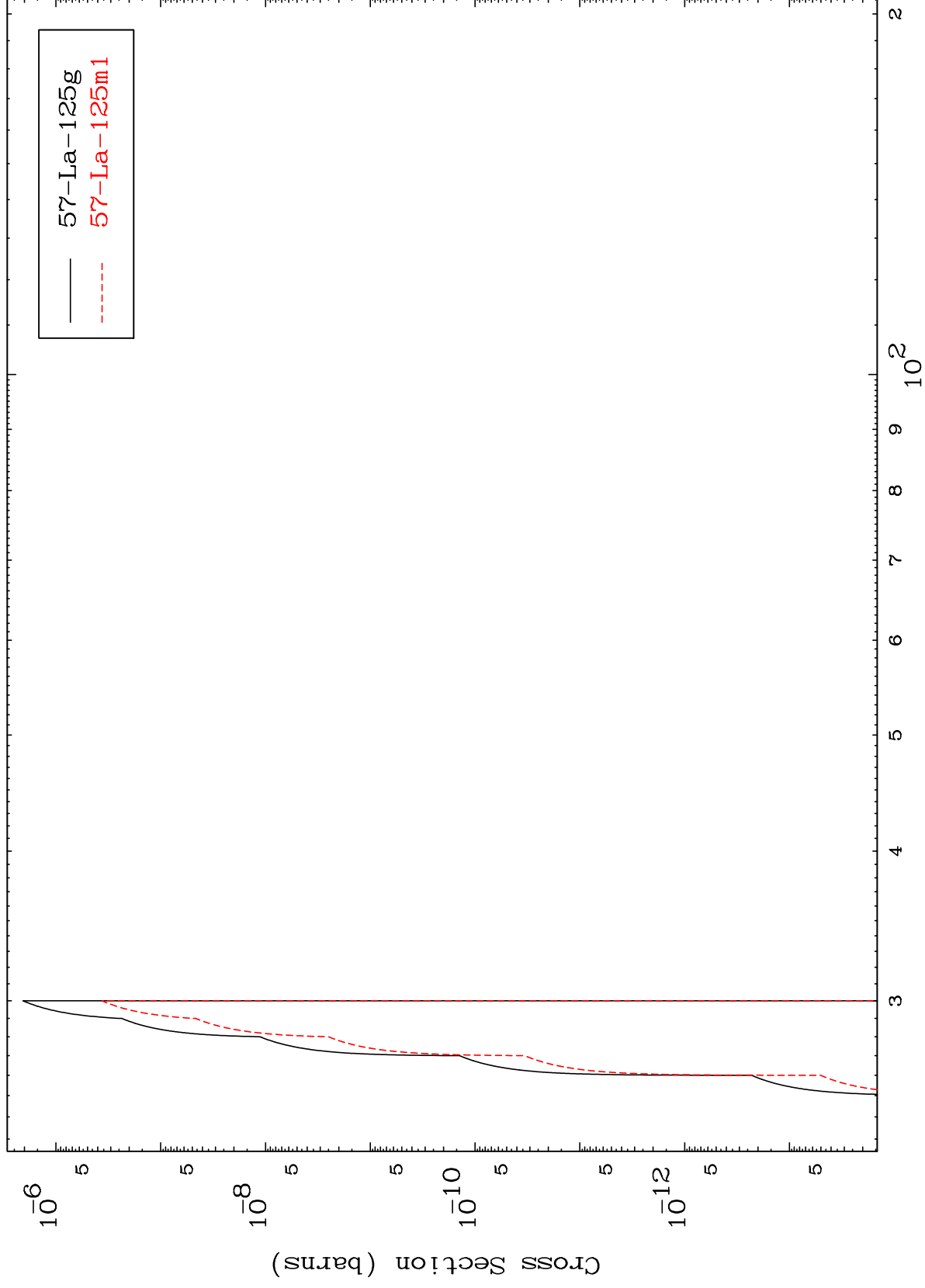
MT= 71 (n, n') Level
Legendre Coefficients

58-Ce-128





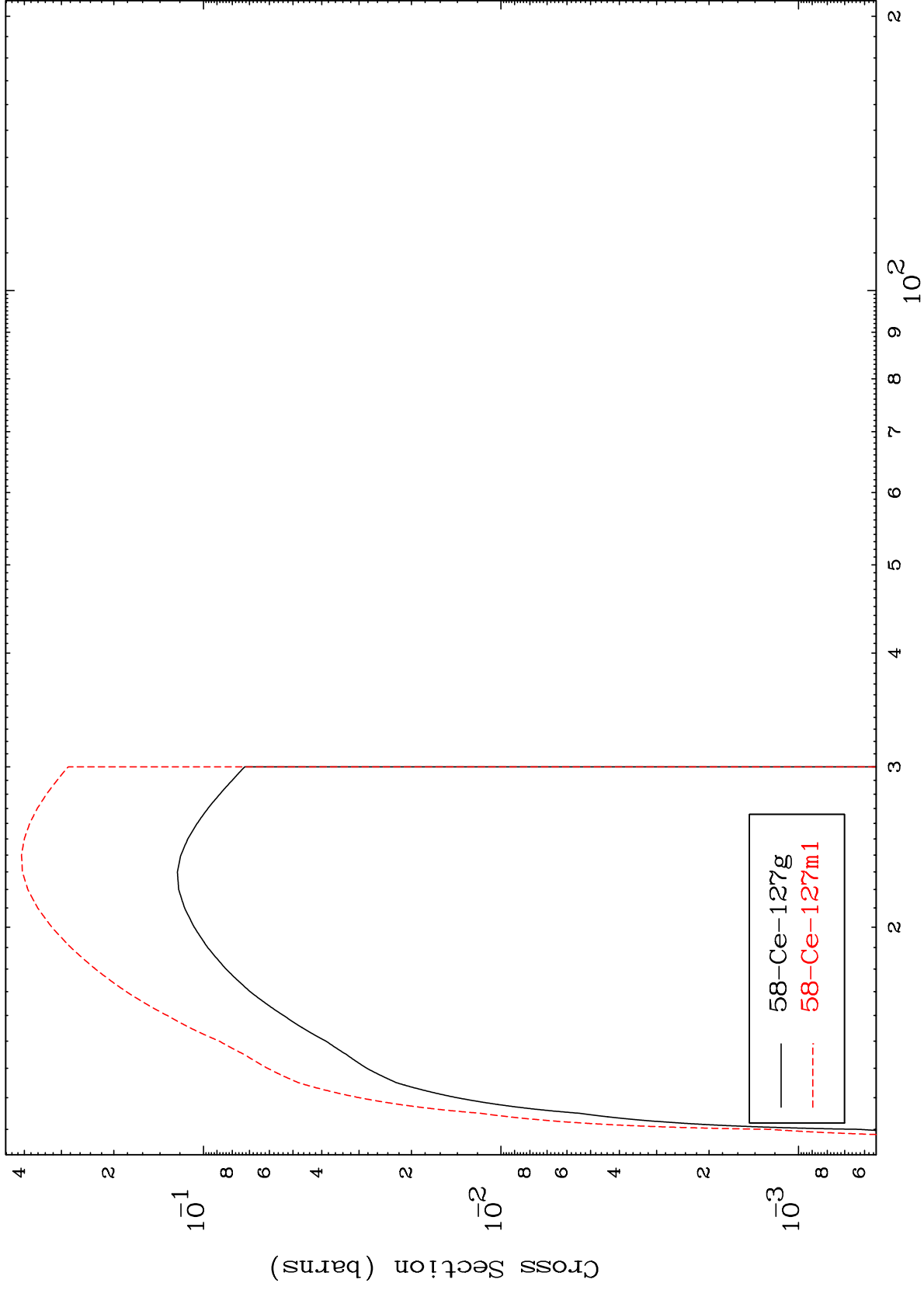


$(n,2n)$ d
Radionuclide Production Cross Section

MAT 5801

58-Ce-128

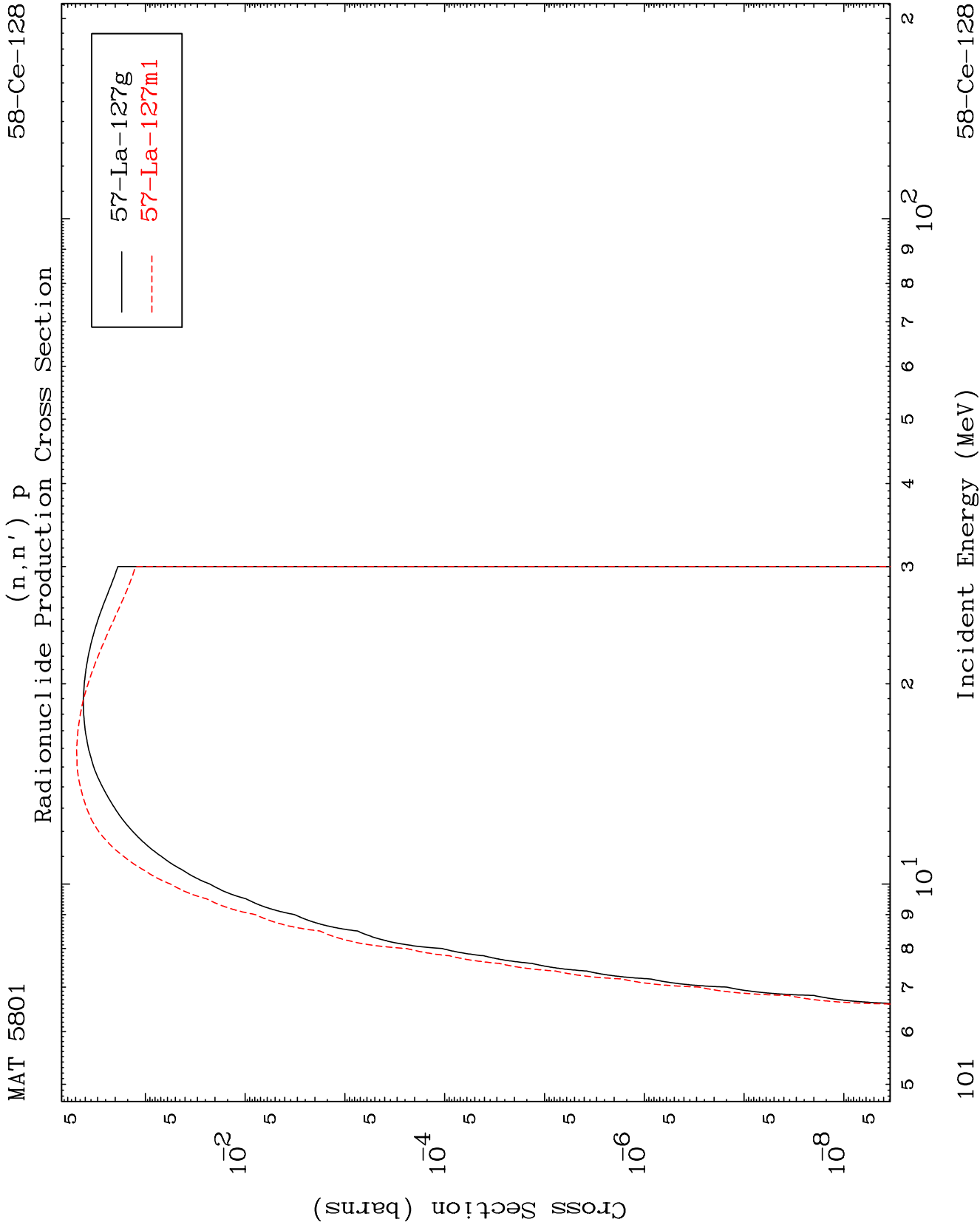
(n,2n)
Radionuclide Production Cross Section

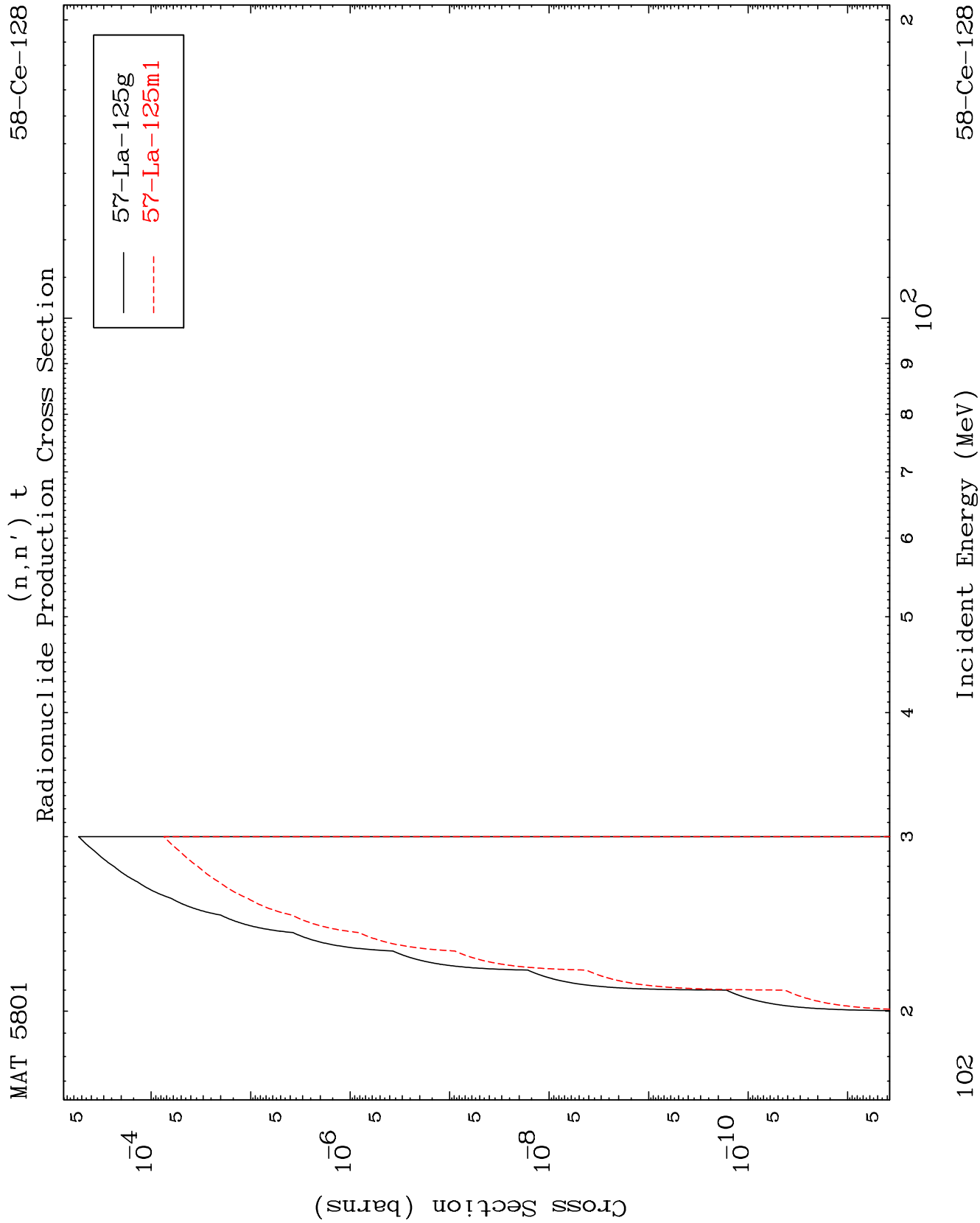


58-Ce-128

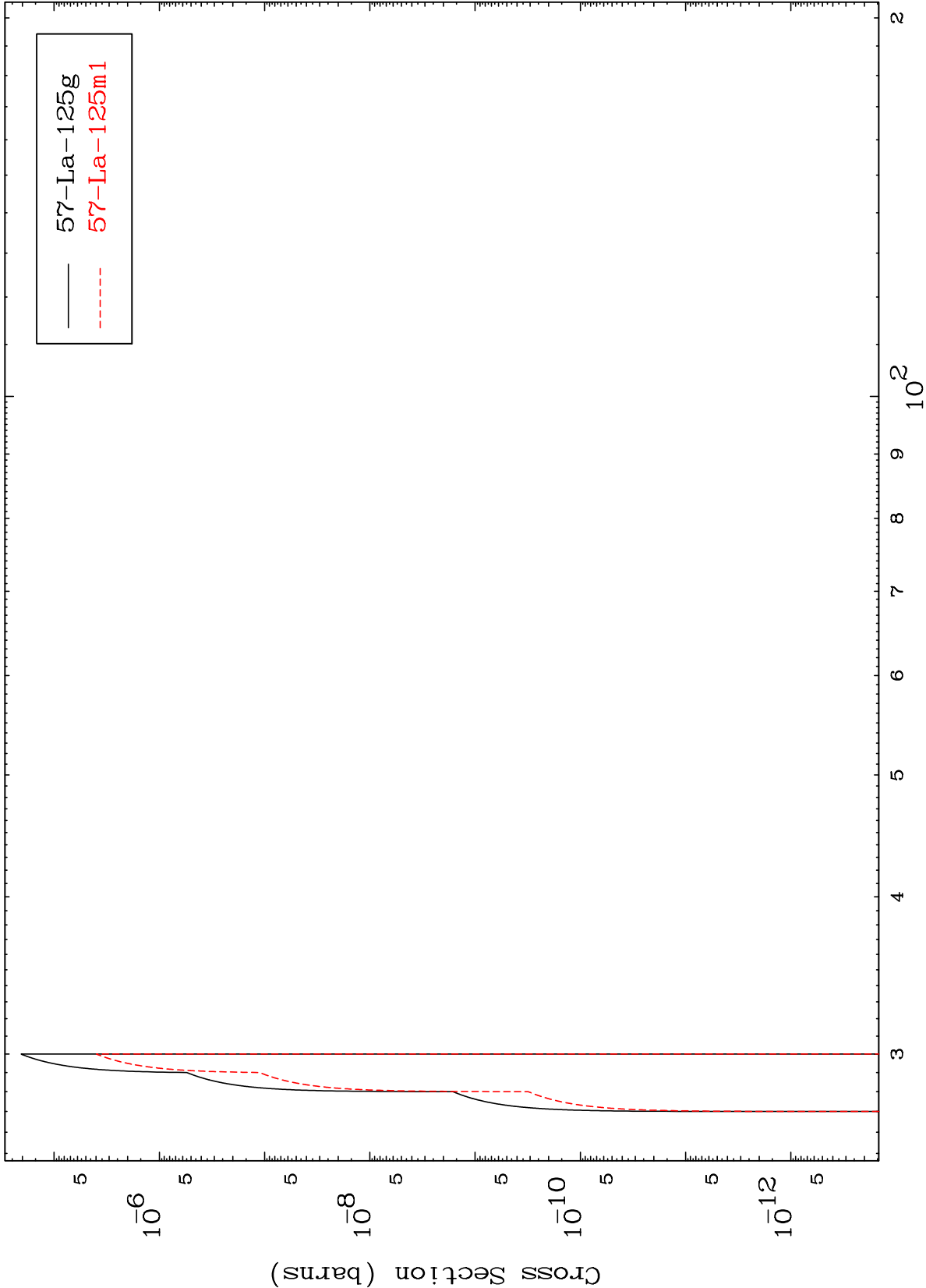
Incident Energy (MeV)

100





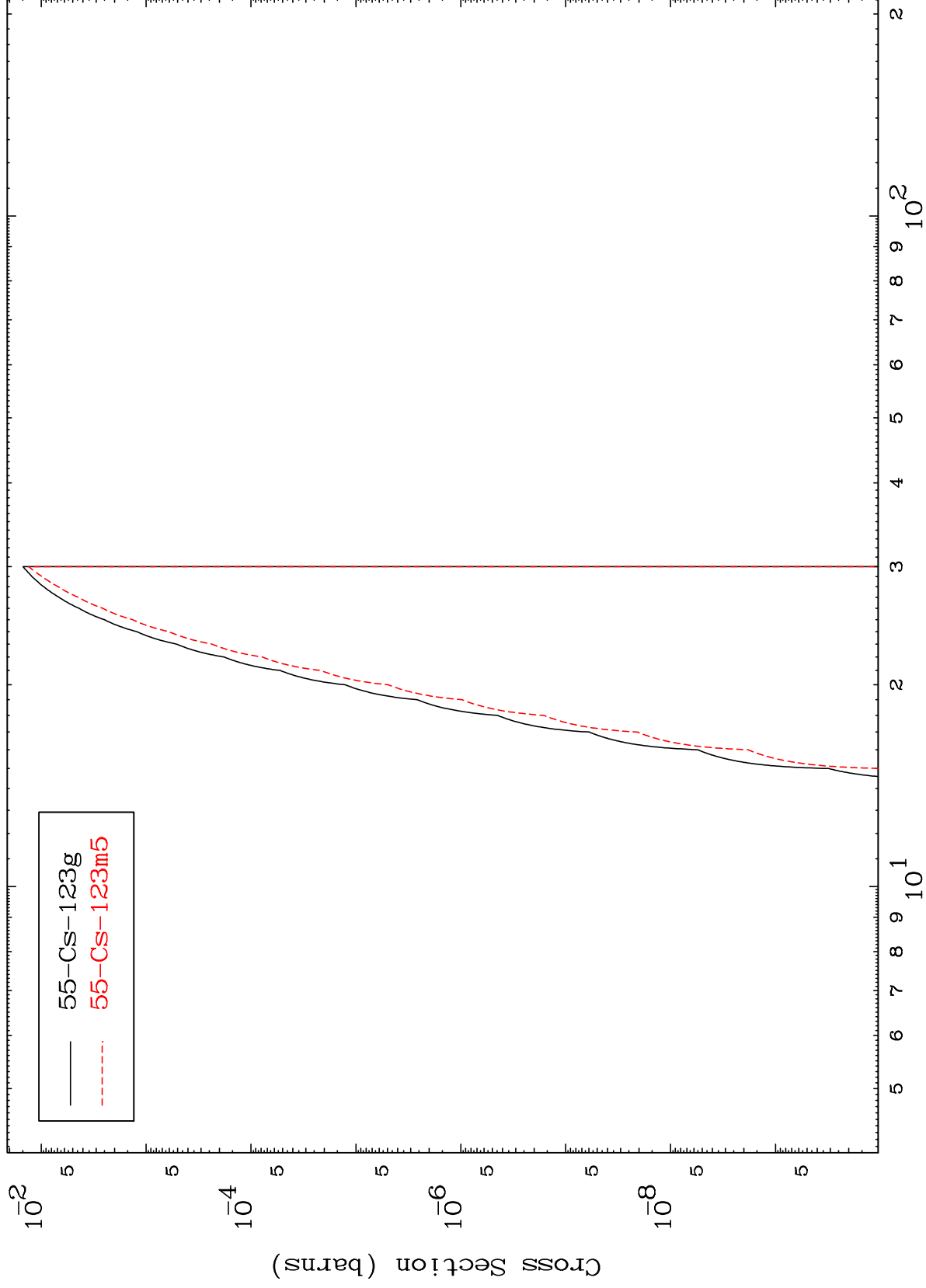
(n,3n) p
Radionuclide Production Cross Section



MAT 5801

58-Ce-128

(n,n') p α
Radionuclide Production Cross Section



104

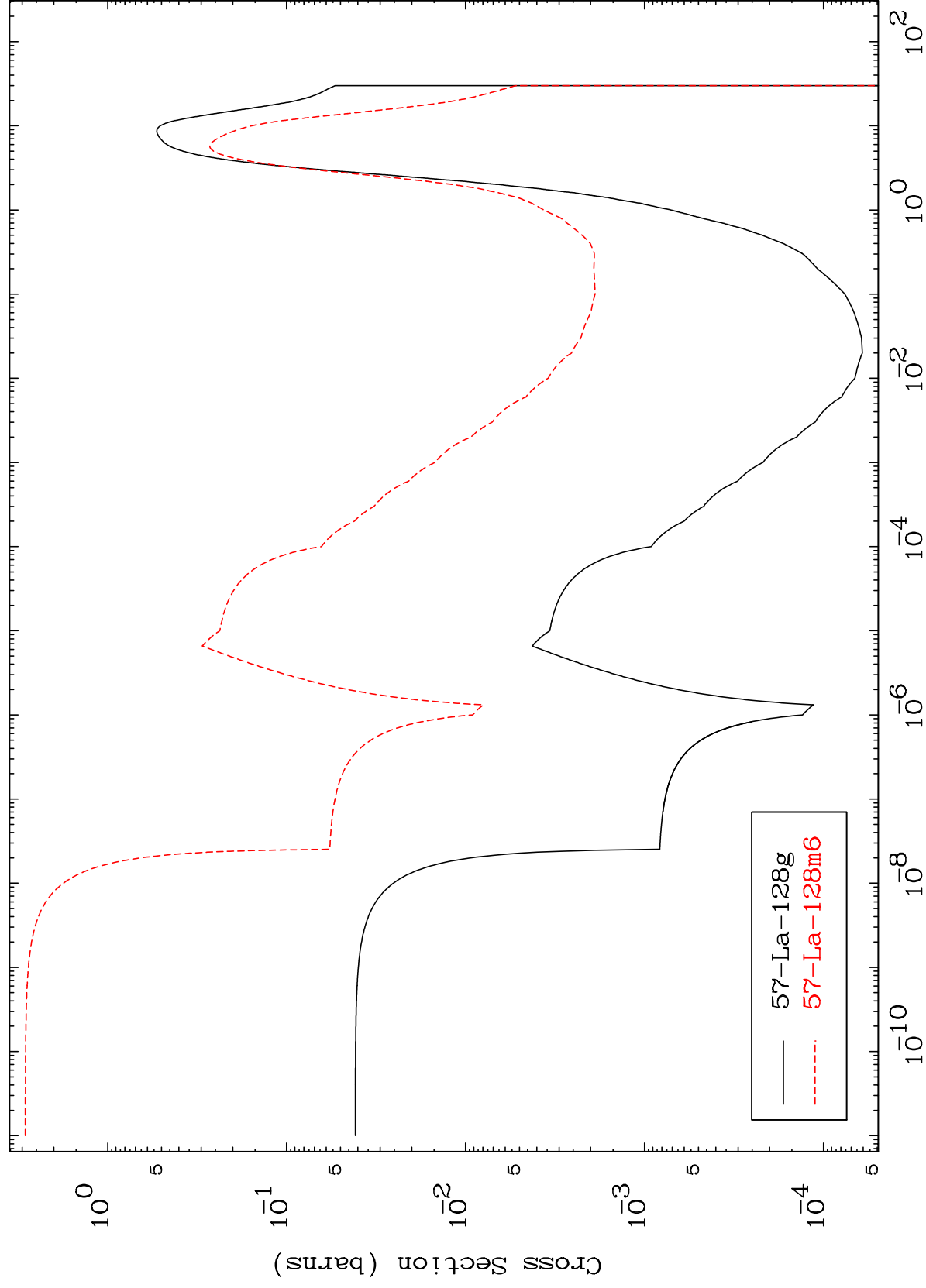
Incident Energy (MeV)

58-Ce-128

MAT 5801

58-Ce-128

(n,p)
Radionuclide Production Cross Section



58-Ce-128

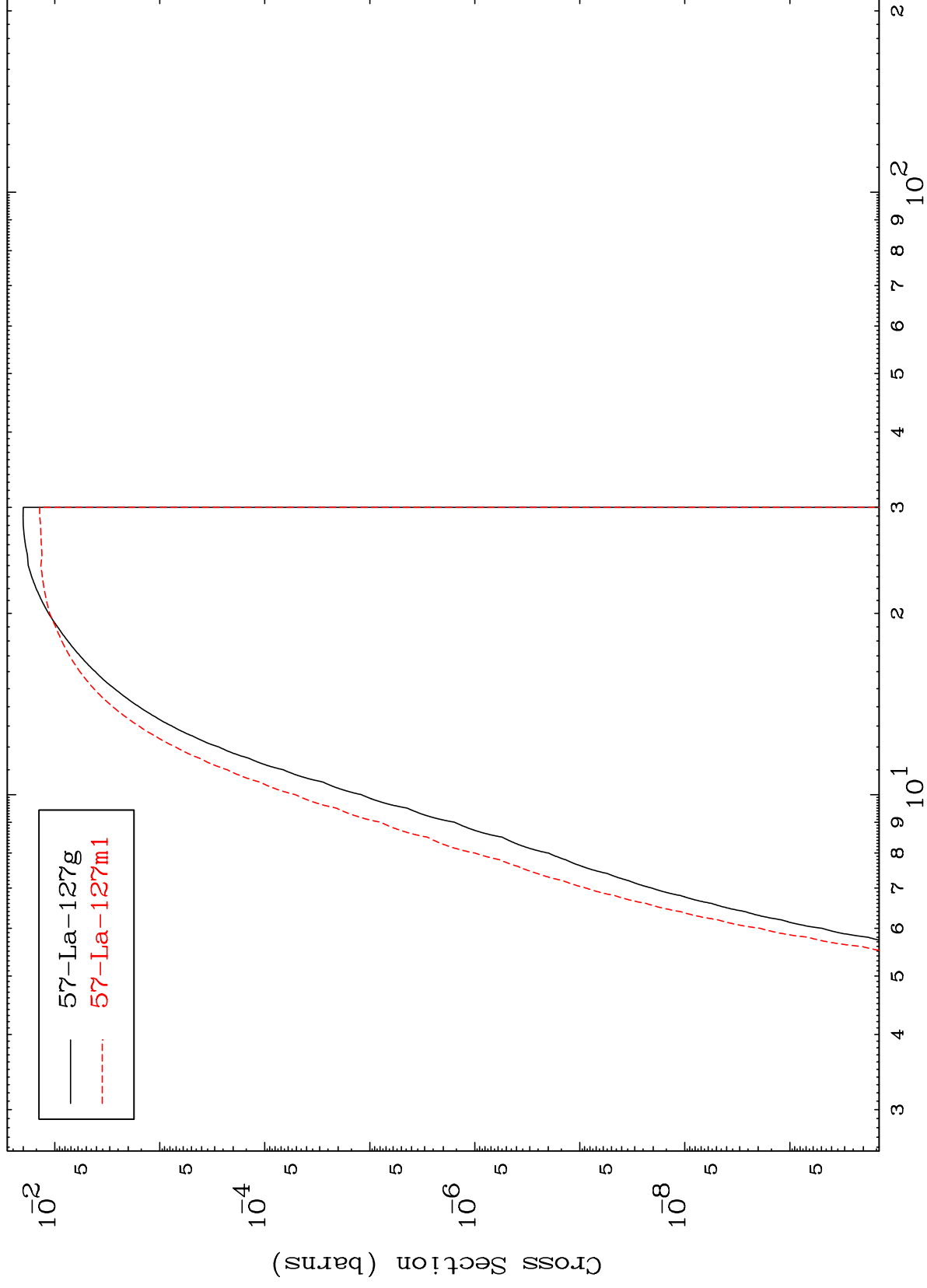
Incident Energy (MeV)

105

MAT 5801

58-Ce-128

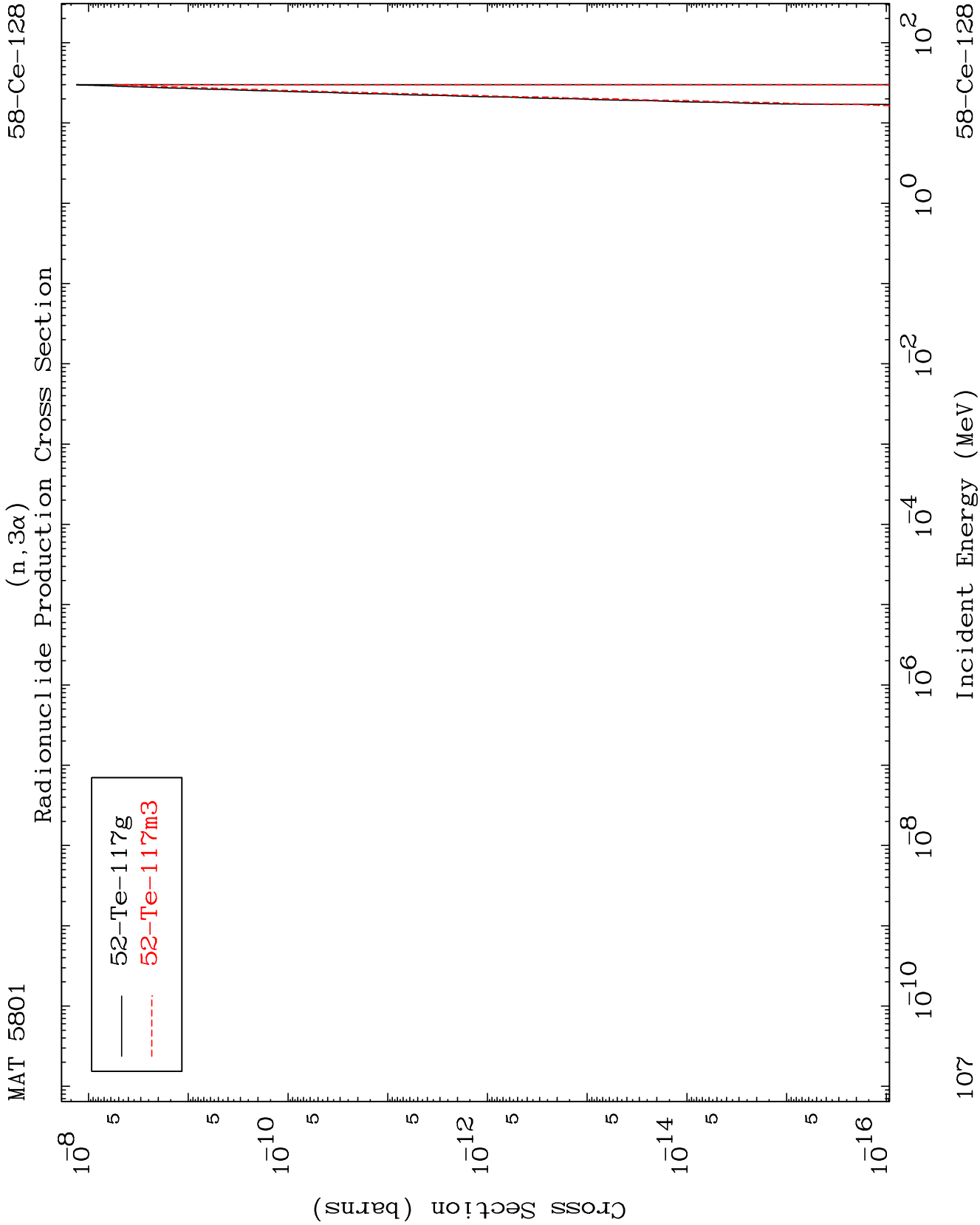
(n,d)
Radionuclide Production Cross Section



106

Incident Energy (MeV)

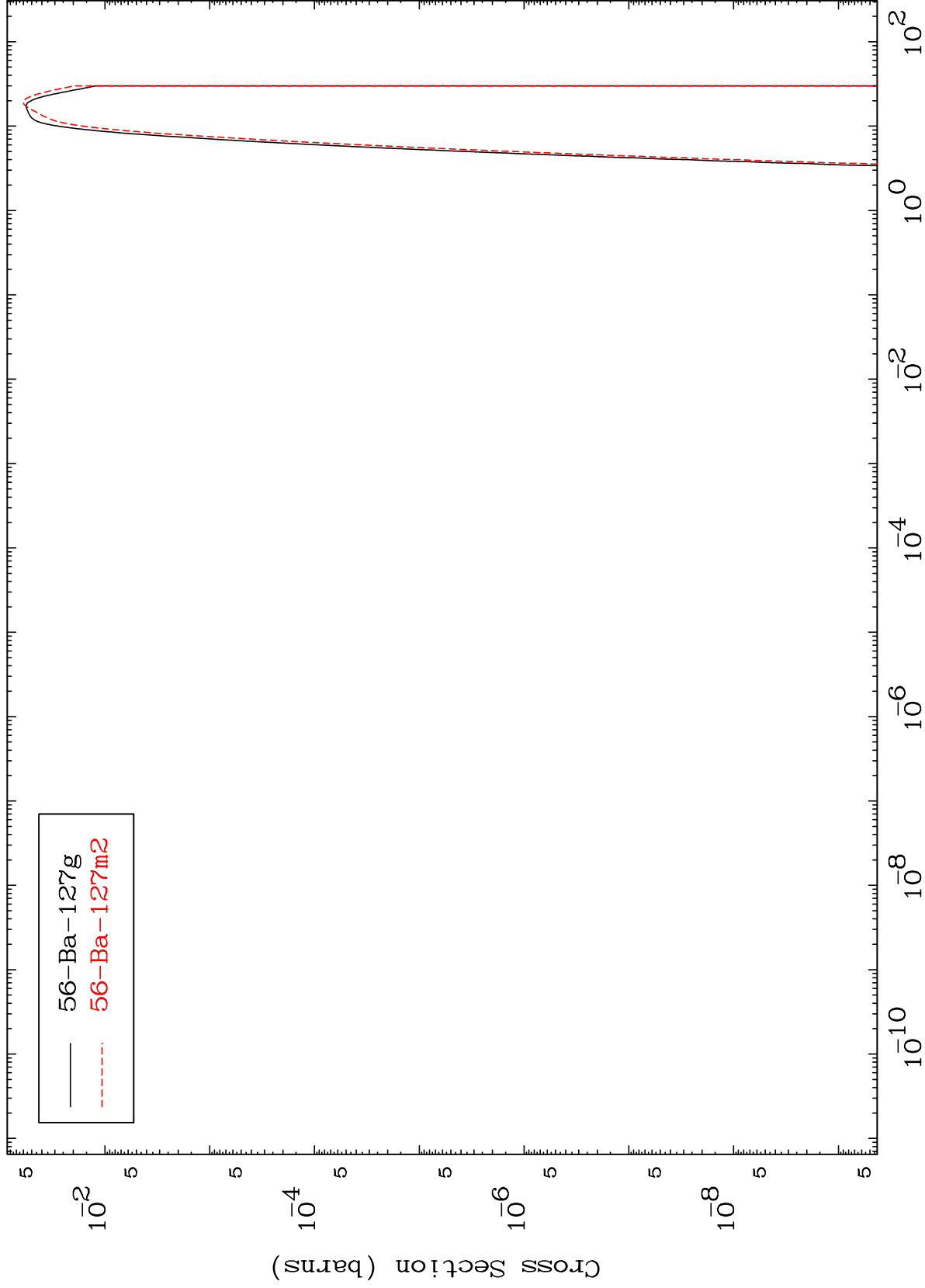
58-Ce-128



MAT 5801

58-Ce-128

(n,2p)
Radionuclide Production Cross Section



58-Ce-128

Incident Energy (MeV)

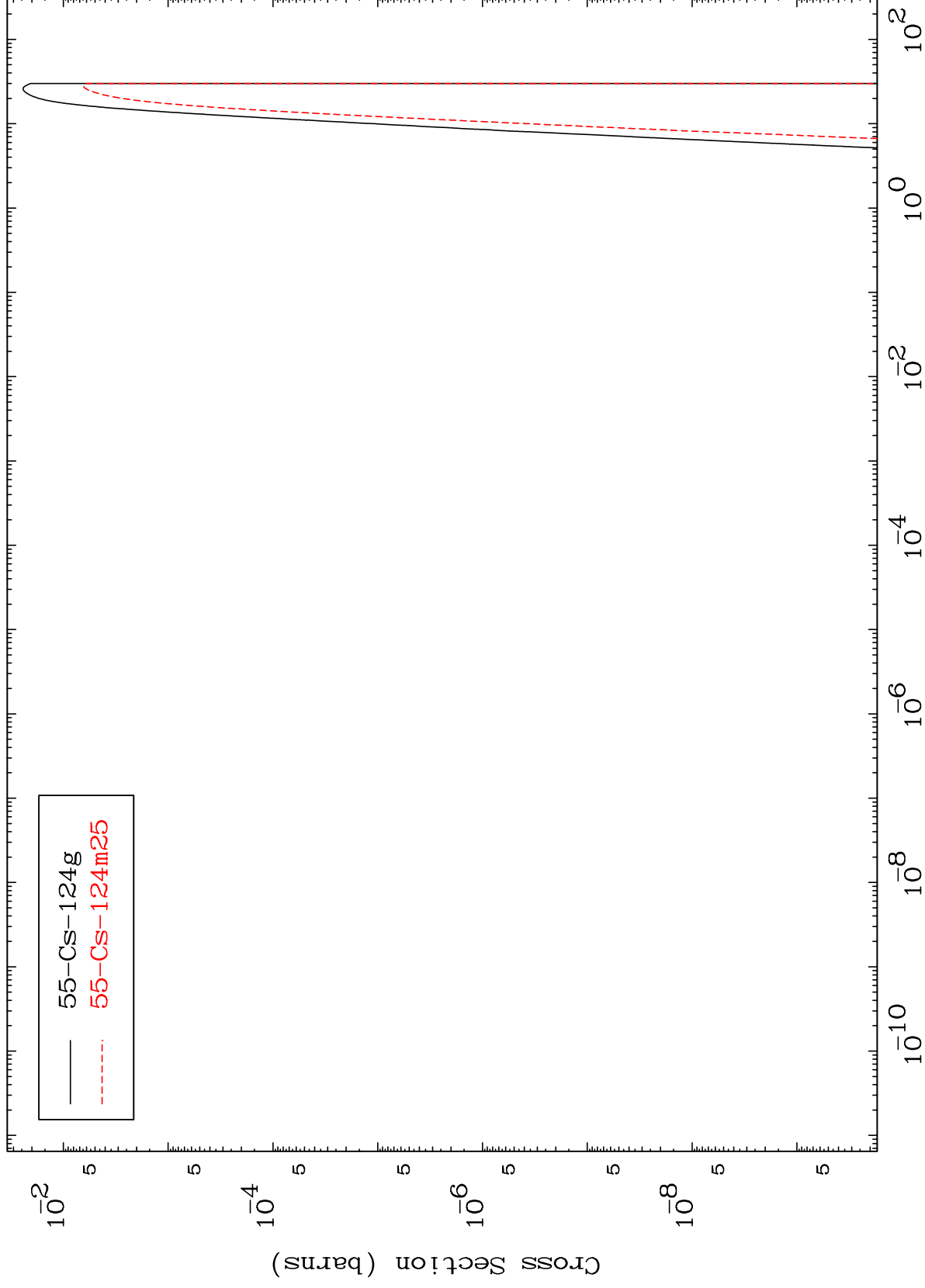
108

MAT 5801

(n,p) α

58-Ce-128

Radionuclide Production Cross Section



109

Incident Energy (MeV)

58-Ce-128

