

Program EVALPLOT
(Version 2021-1)

by

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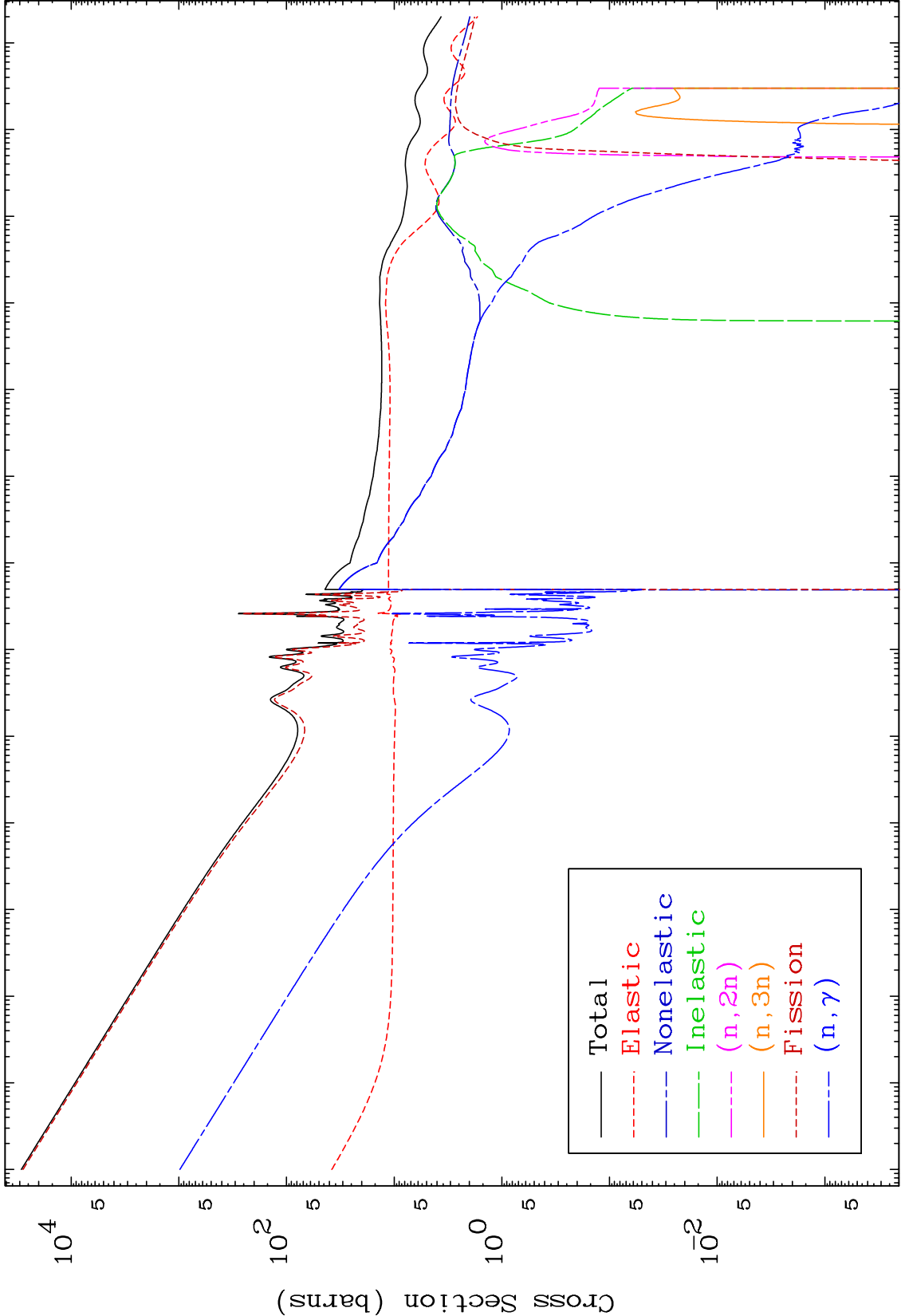
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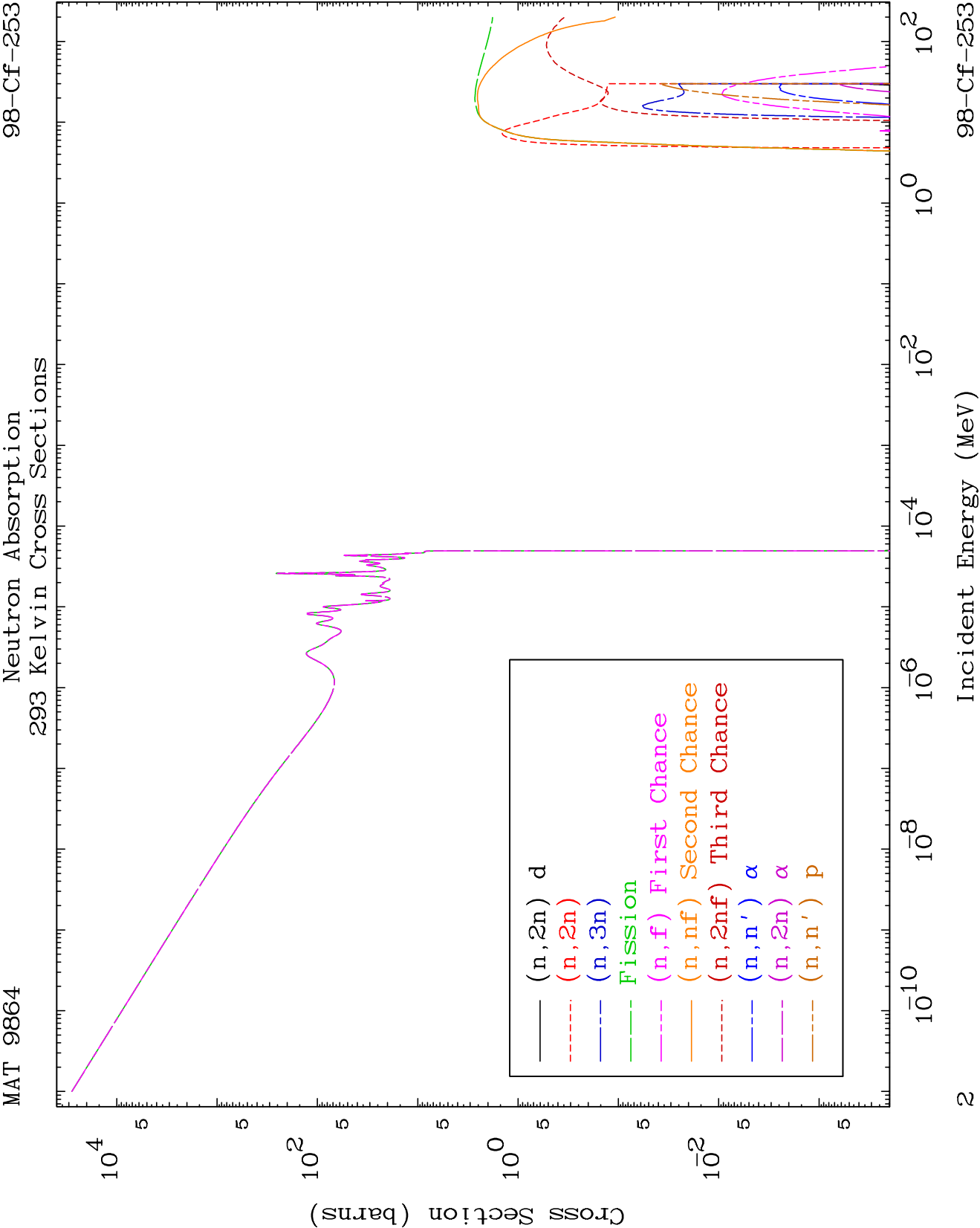
Press Mouse Button to Start

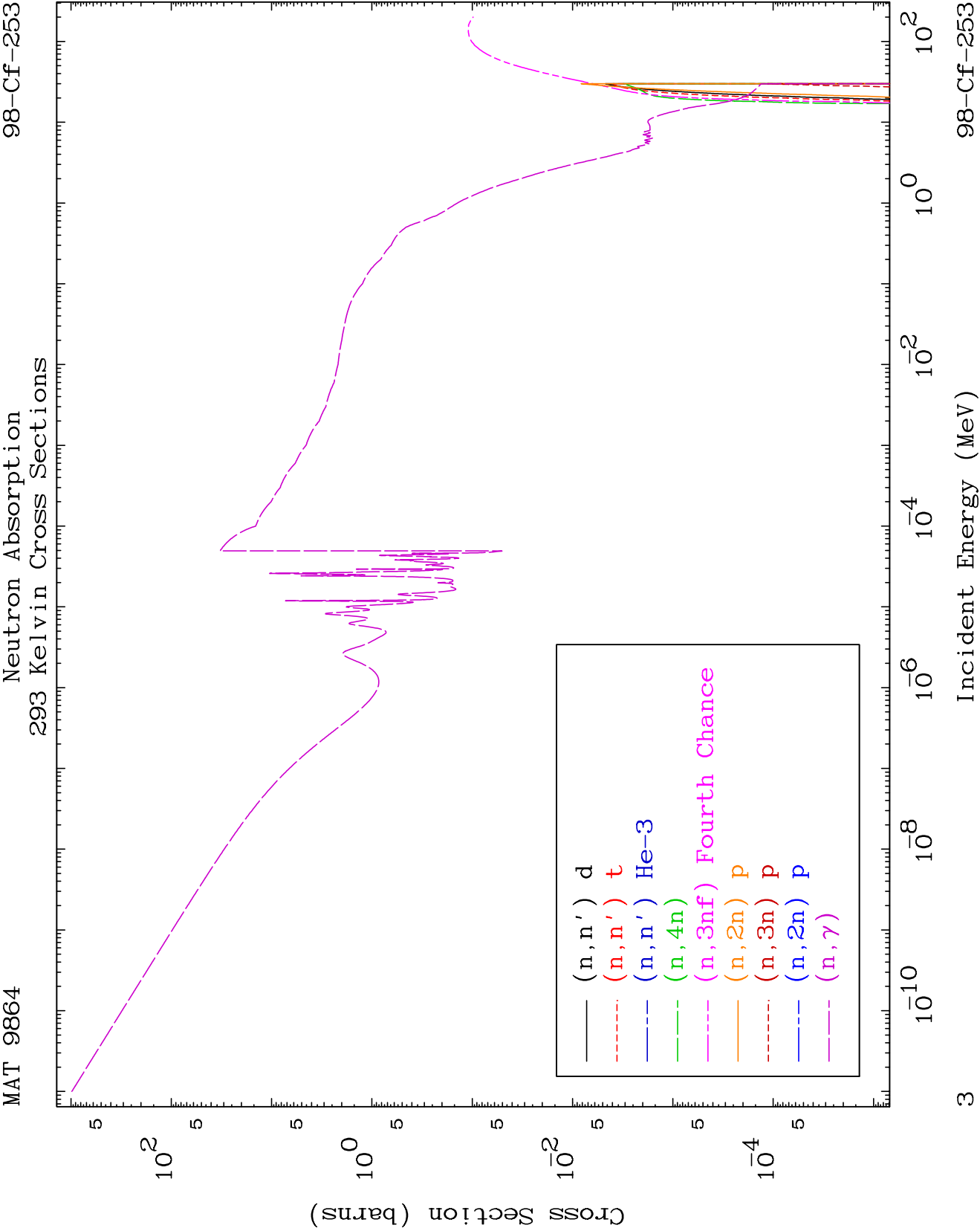
MAT 9864

Neutron Major
293 Kelvin Cross Sections

98-Cf-253



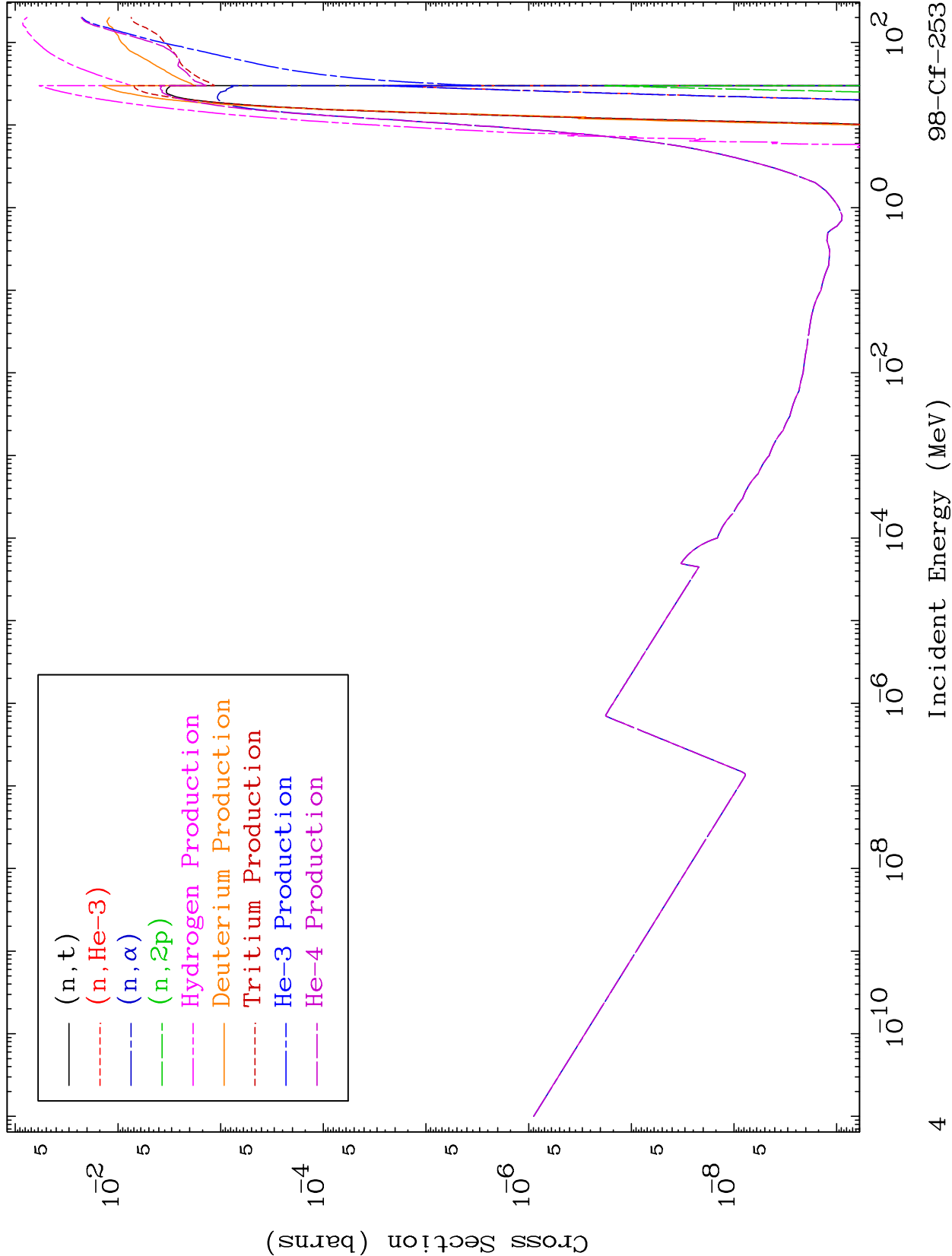


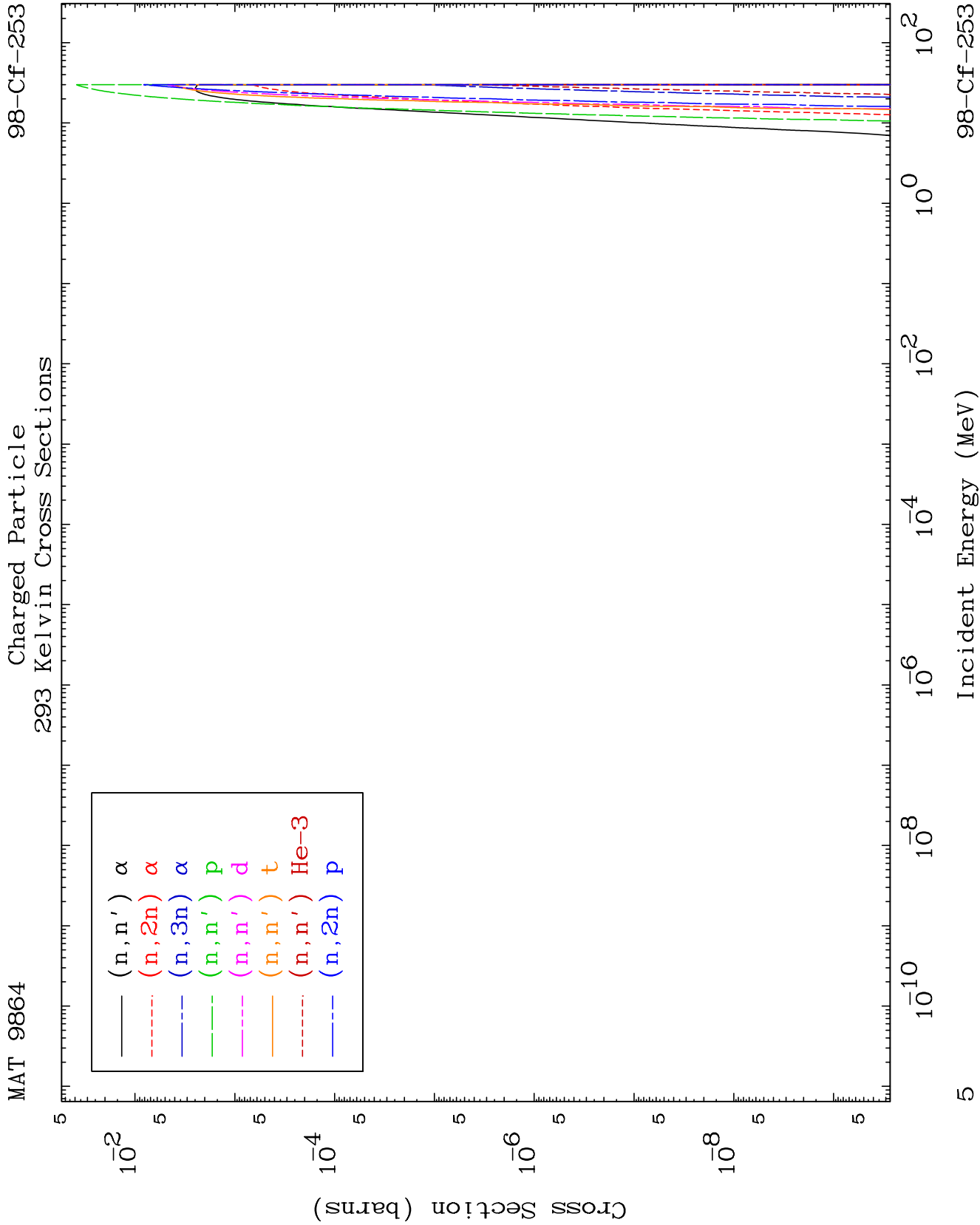


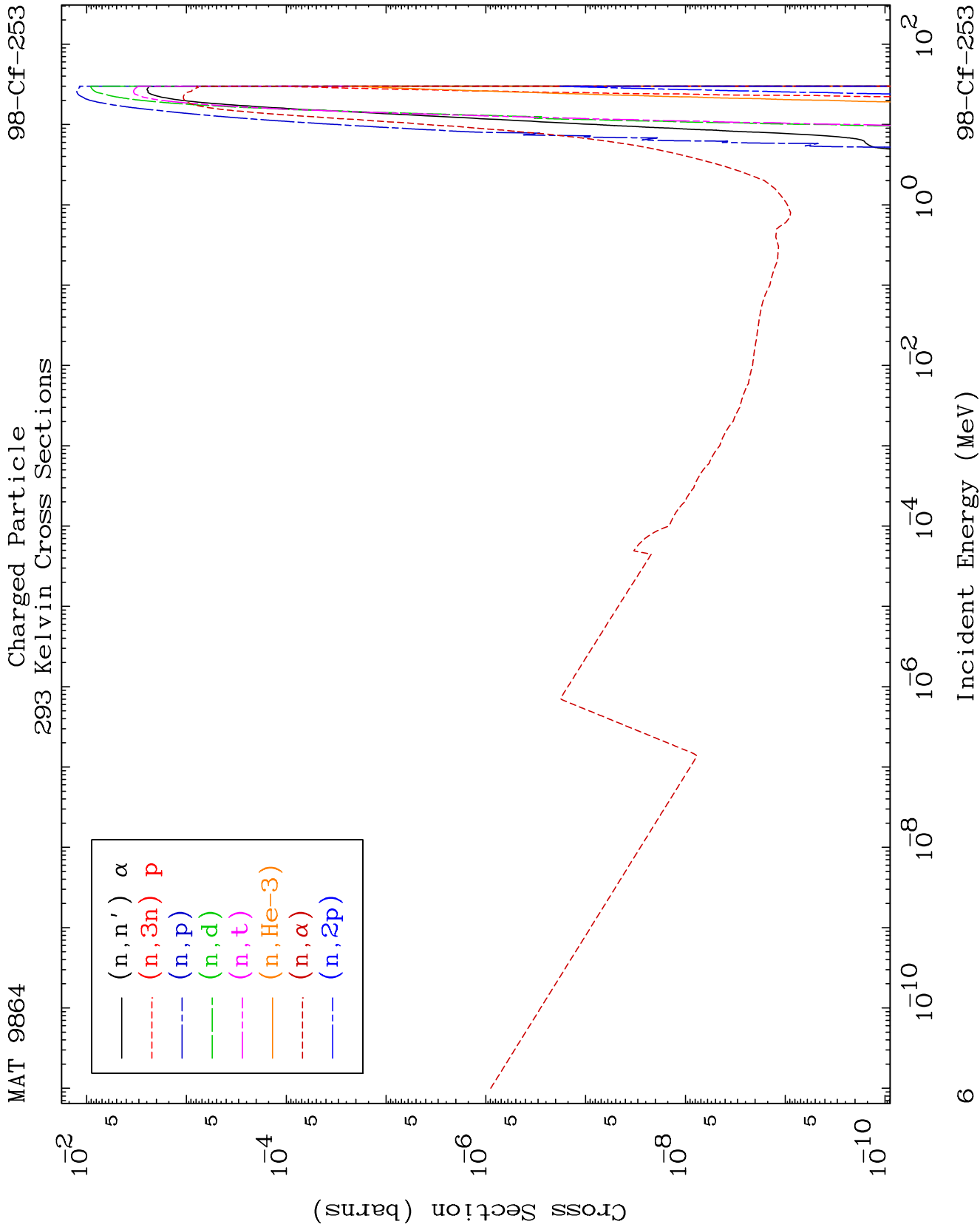
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Neutron Absorption
293 Kelvin Cross Sections

98-Cf-253



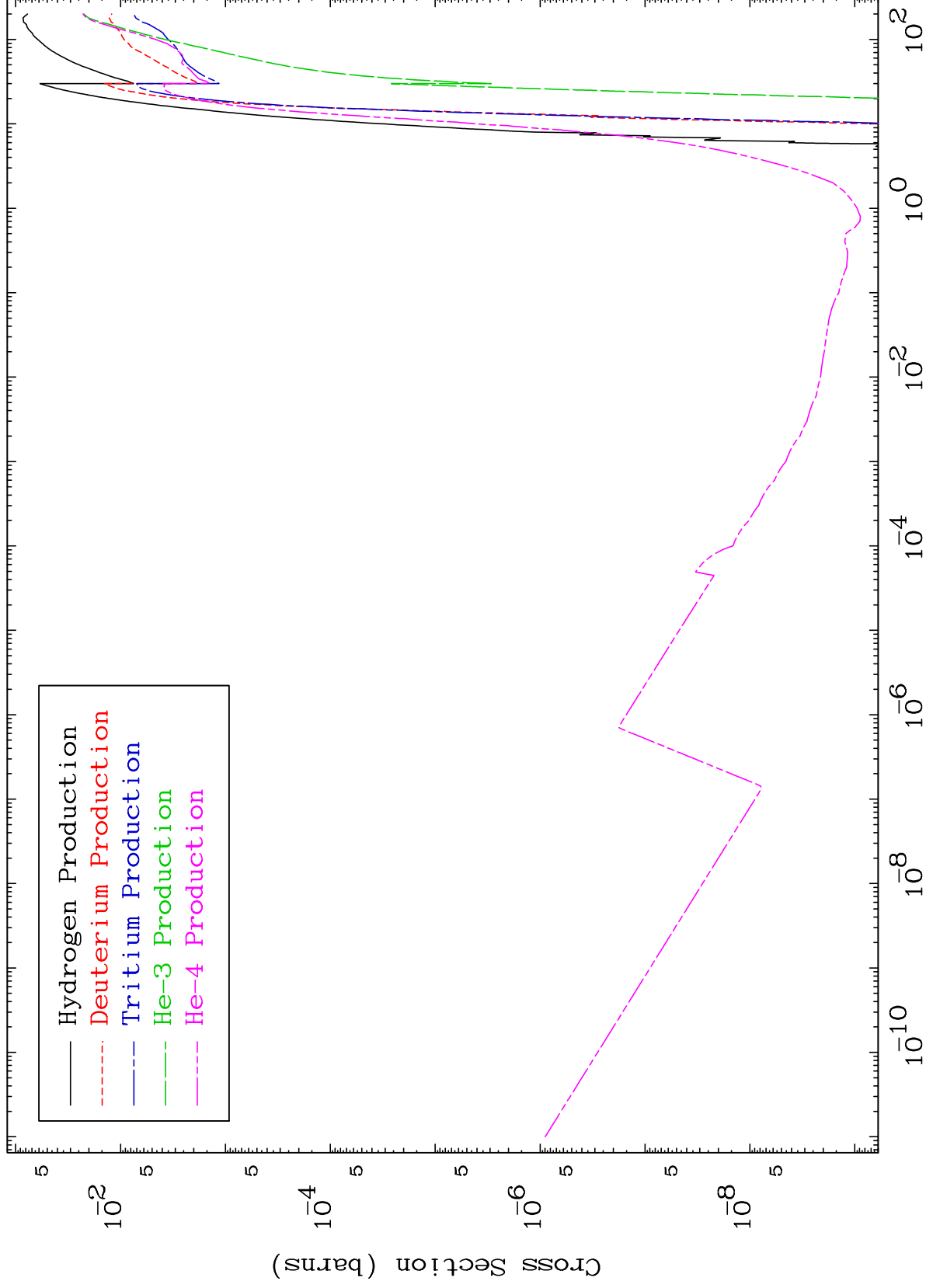




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Particle Production
293 Kelvin Cross Sections

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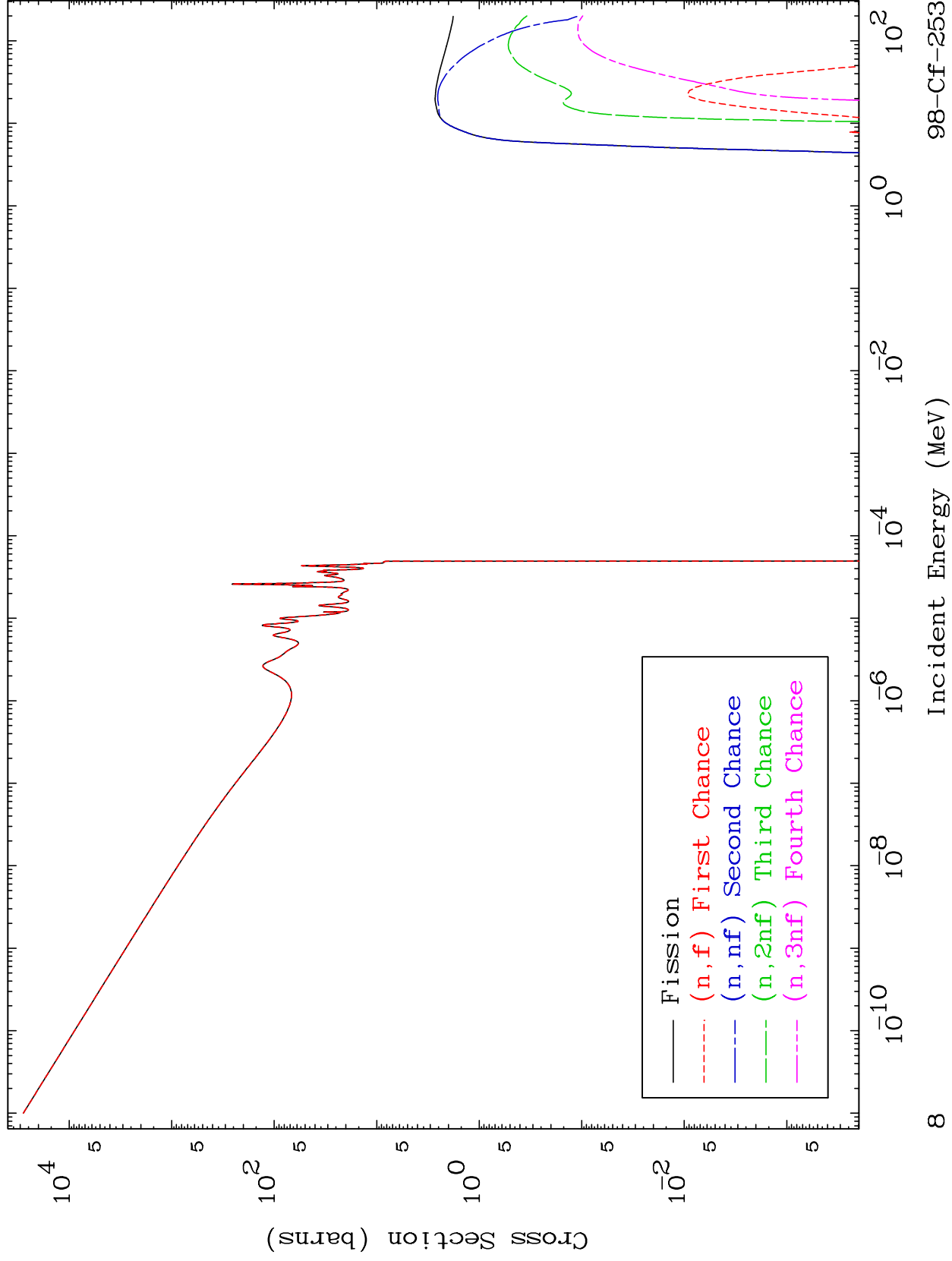


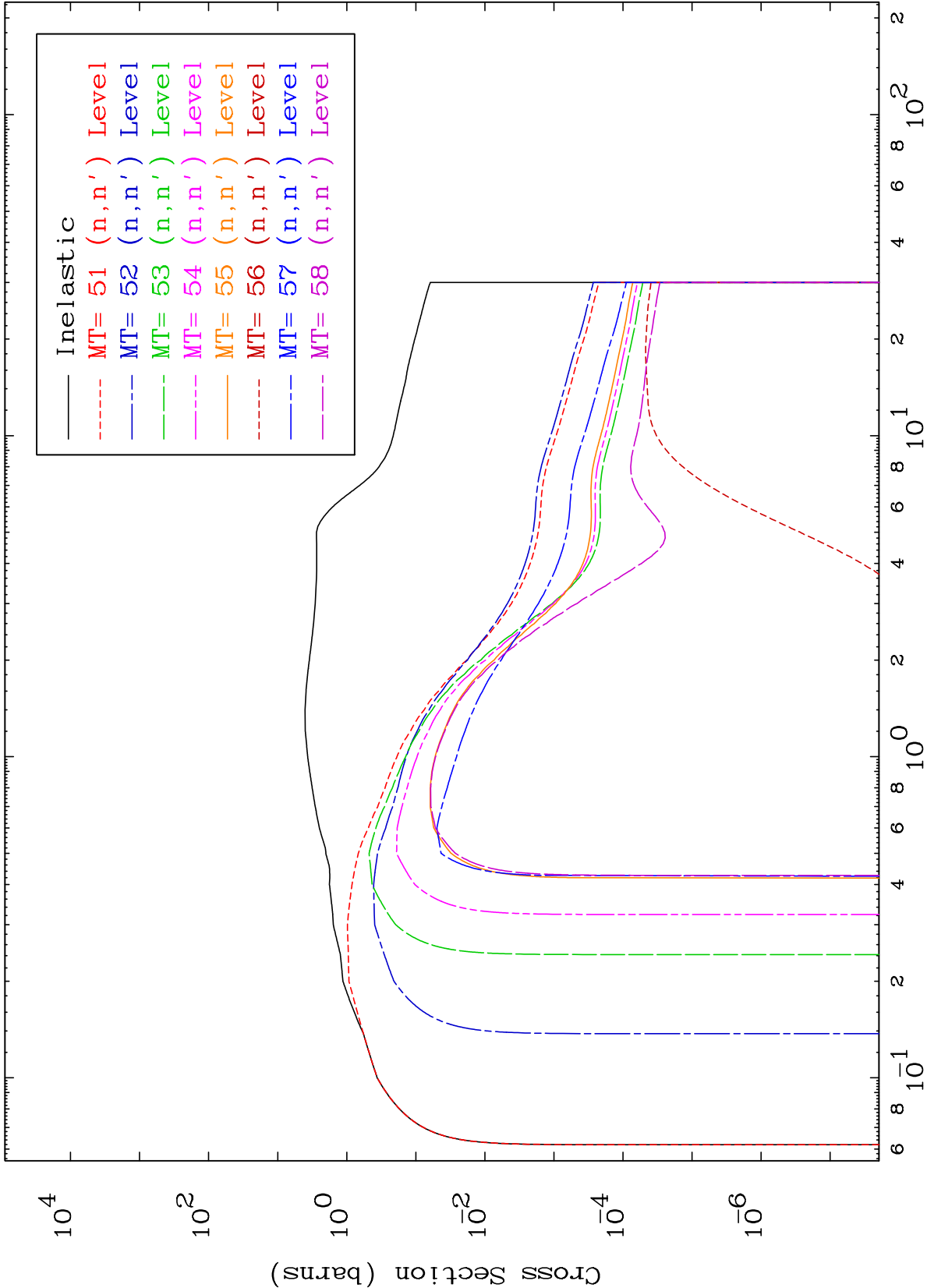
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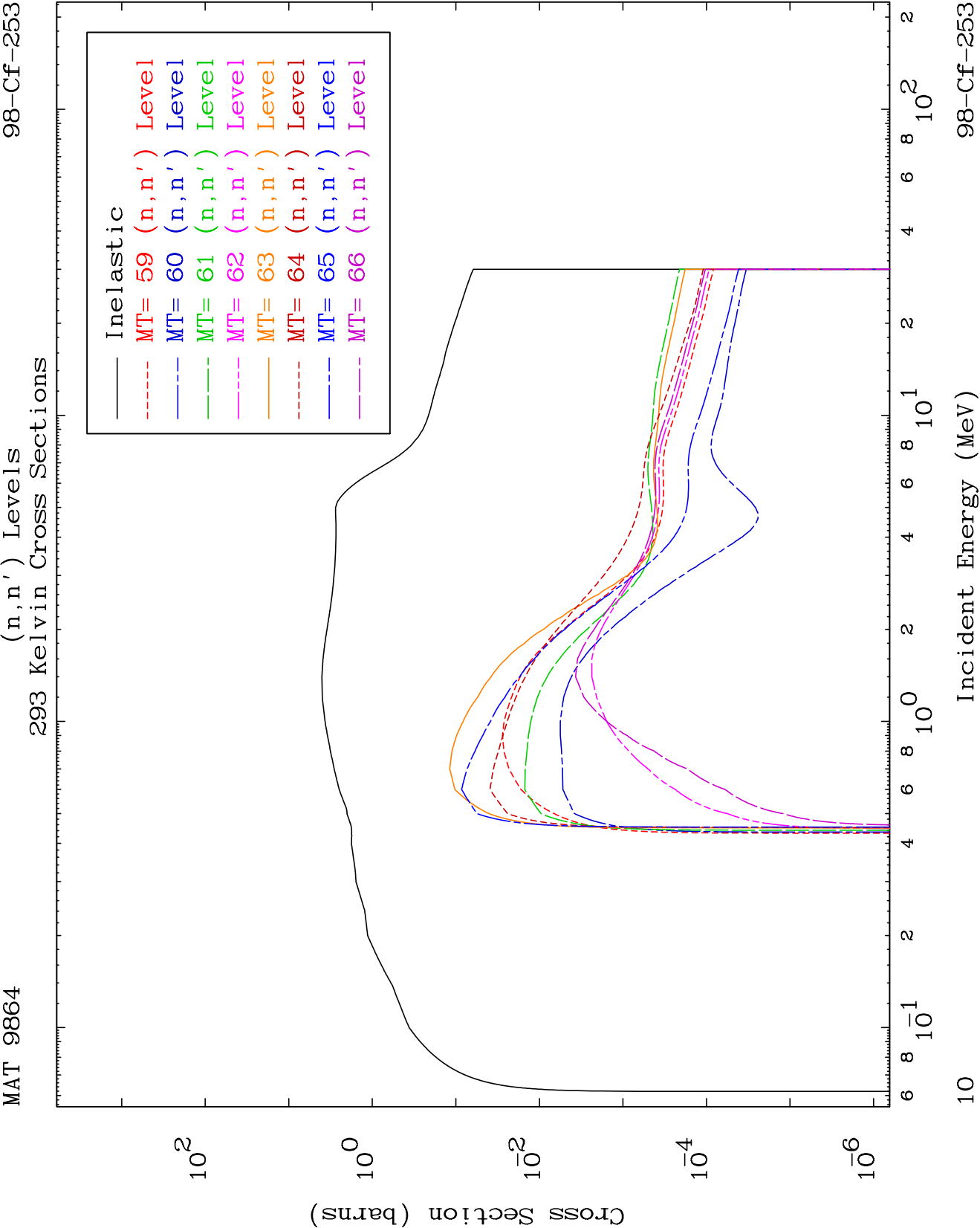
Fission

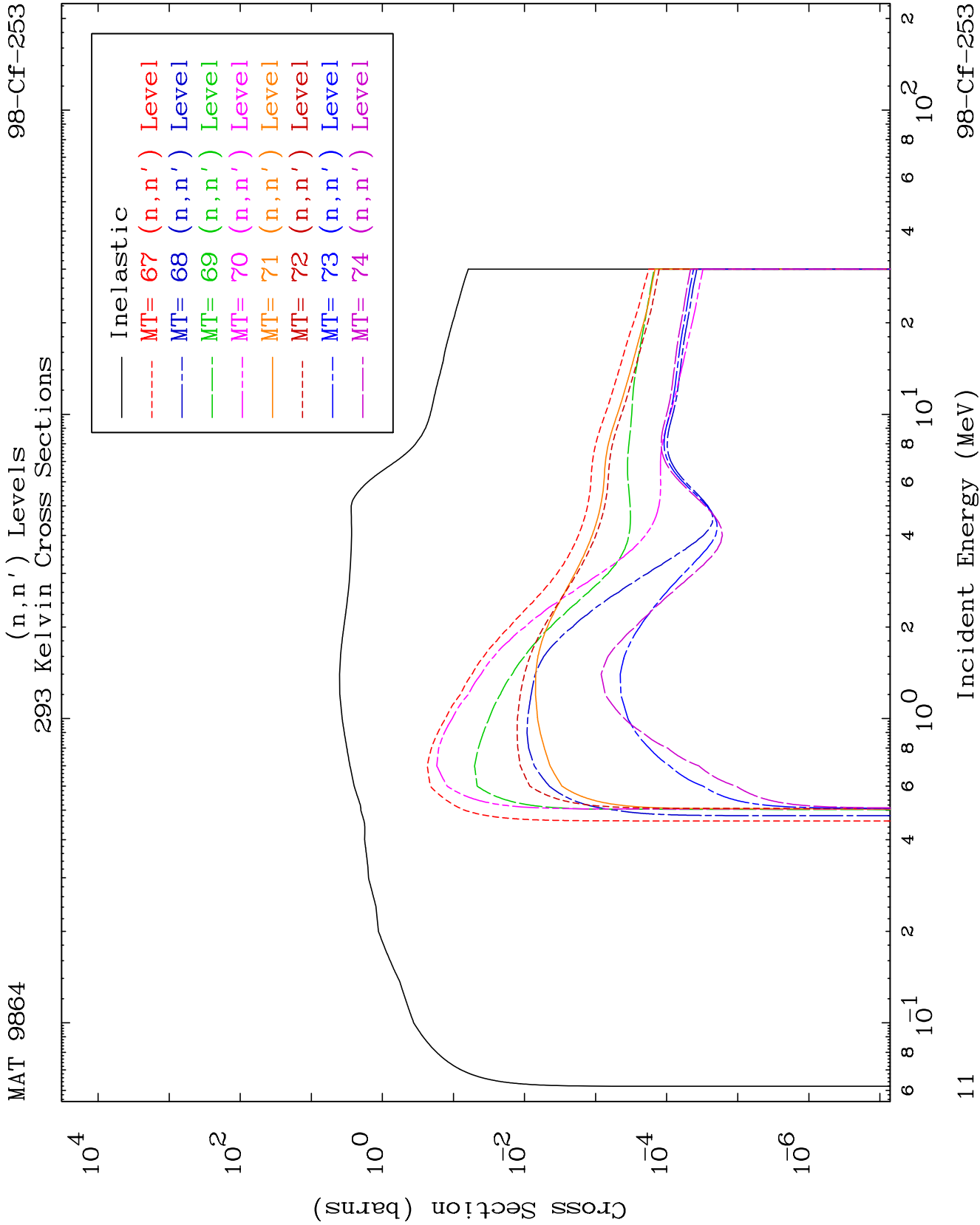
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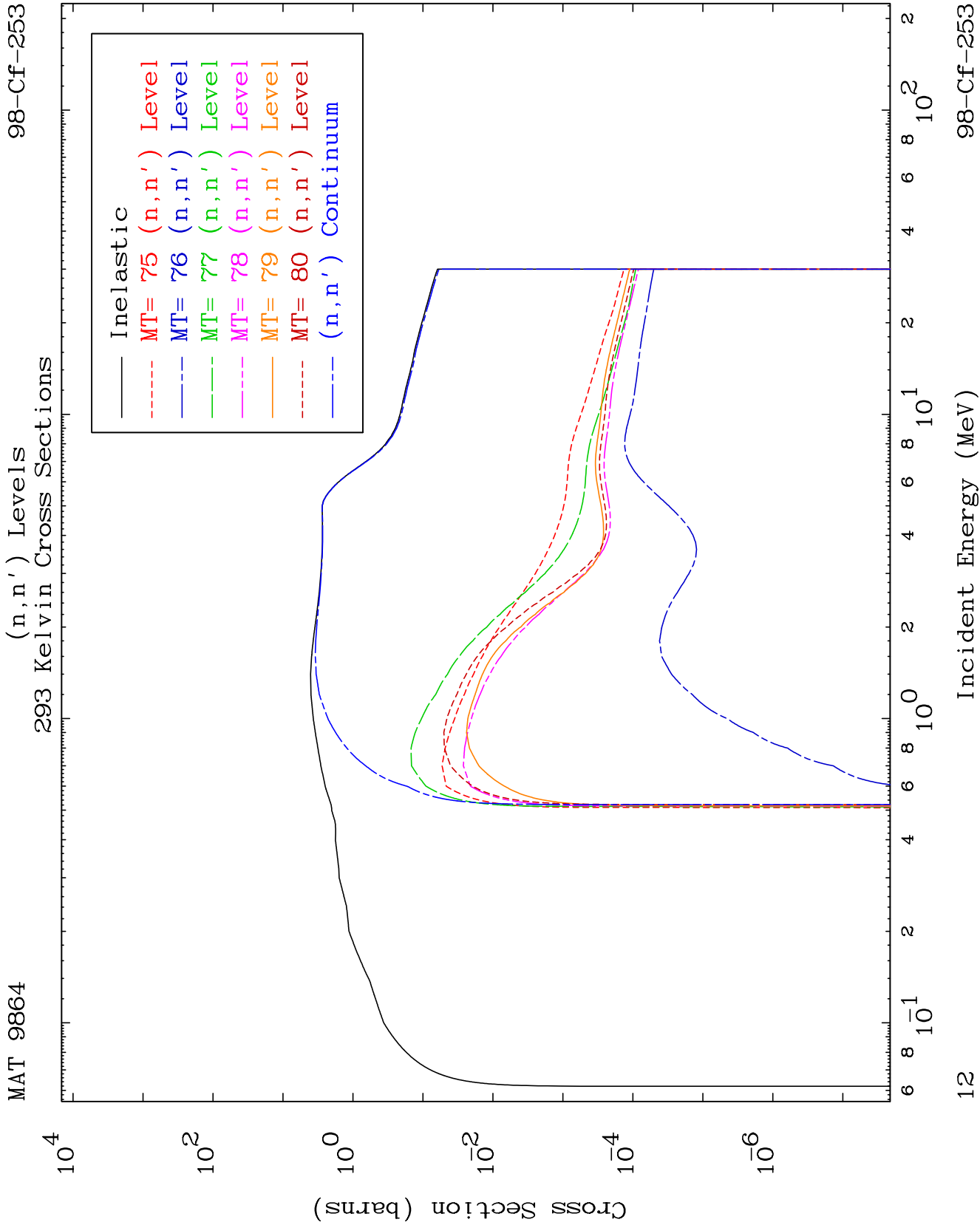
293 Kelvin Cross Sections







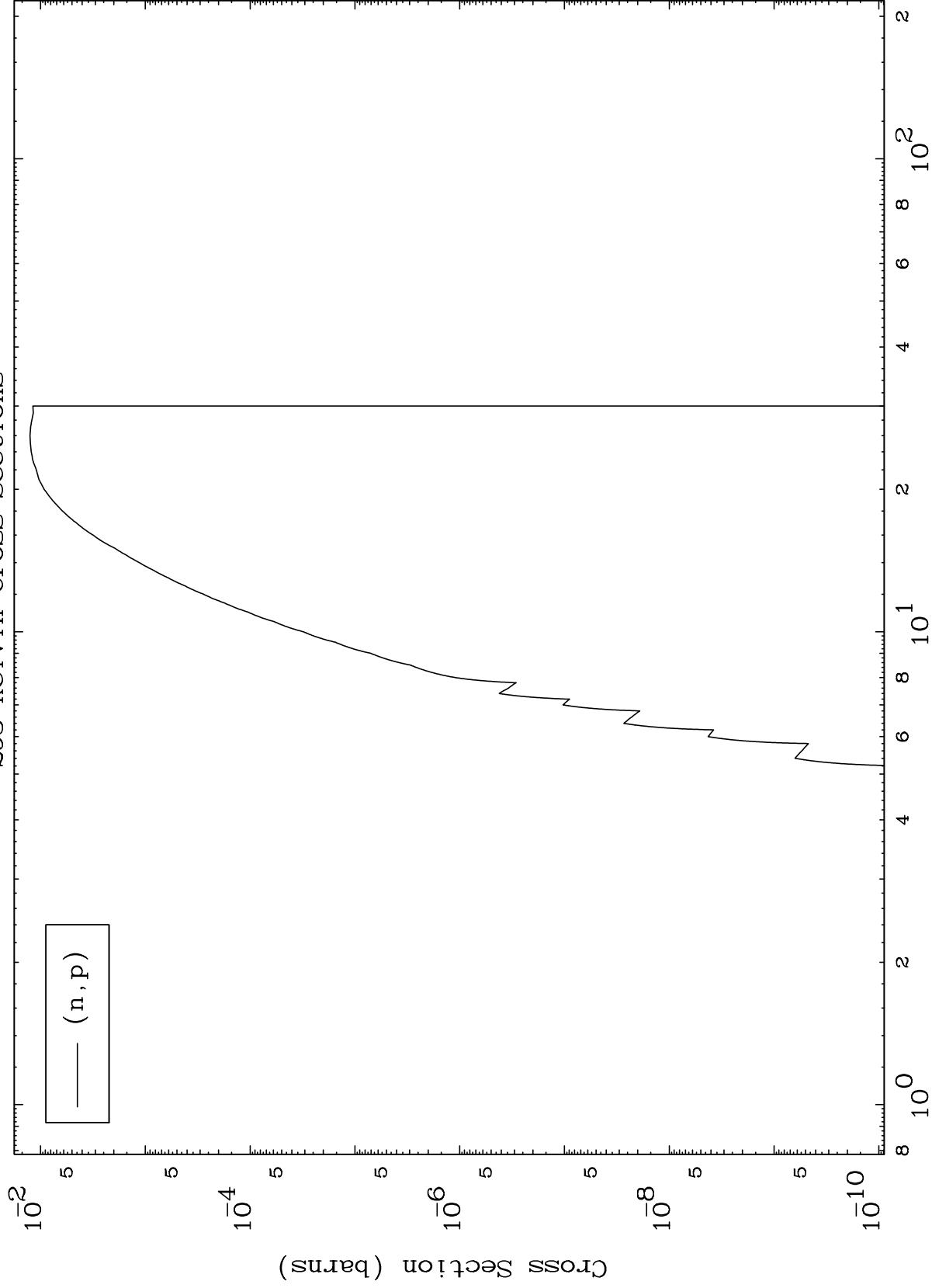




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98-Cf-253

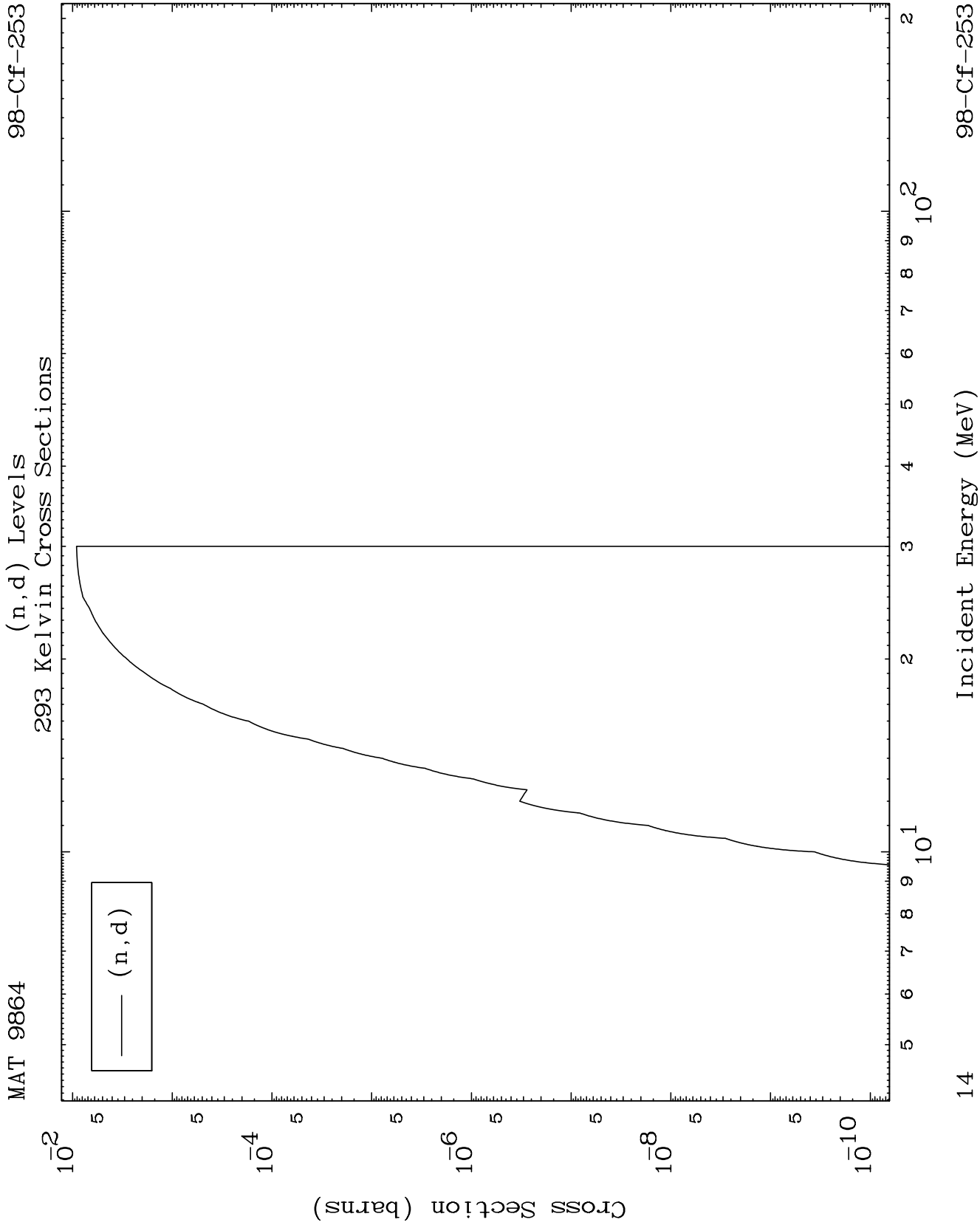
(n,p) Levels
293 Kelvin Cross Sections



98-Cf-253

Incident Energy (MeV)

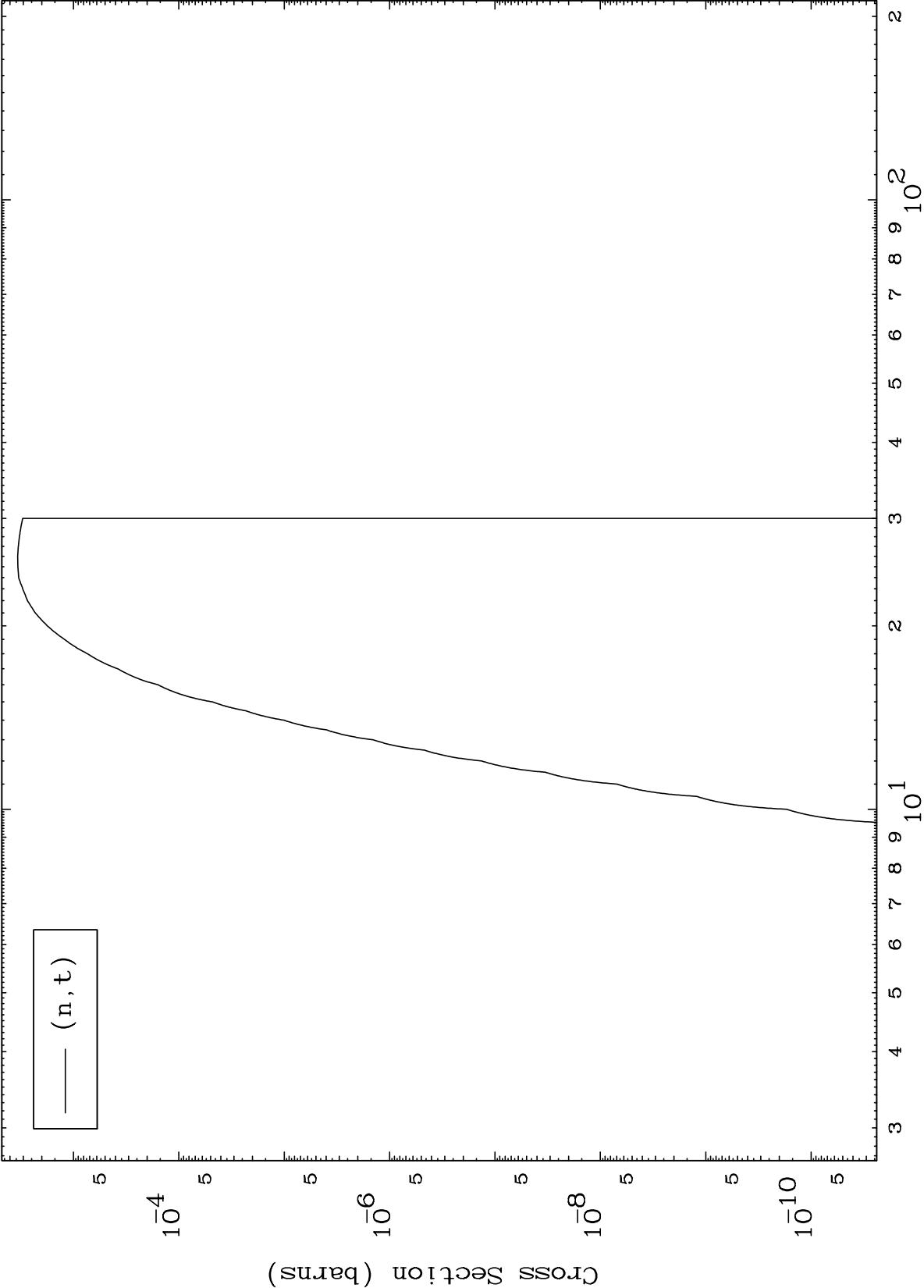
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(n,t) Levels
293 Kelvin Cross Sections



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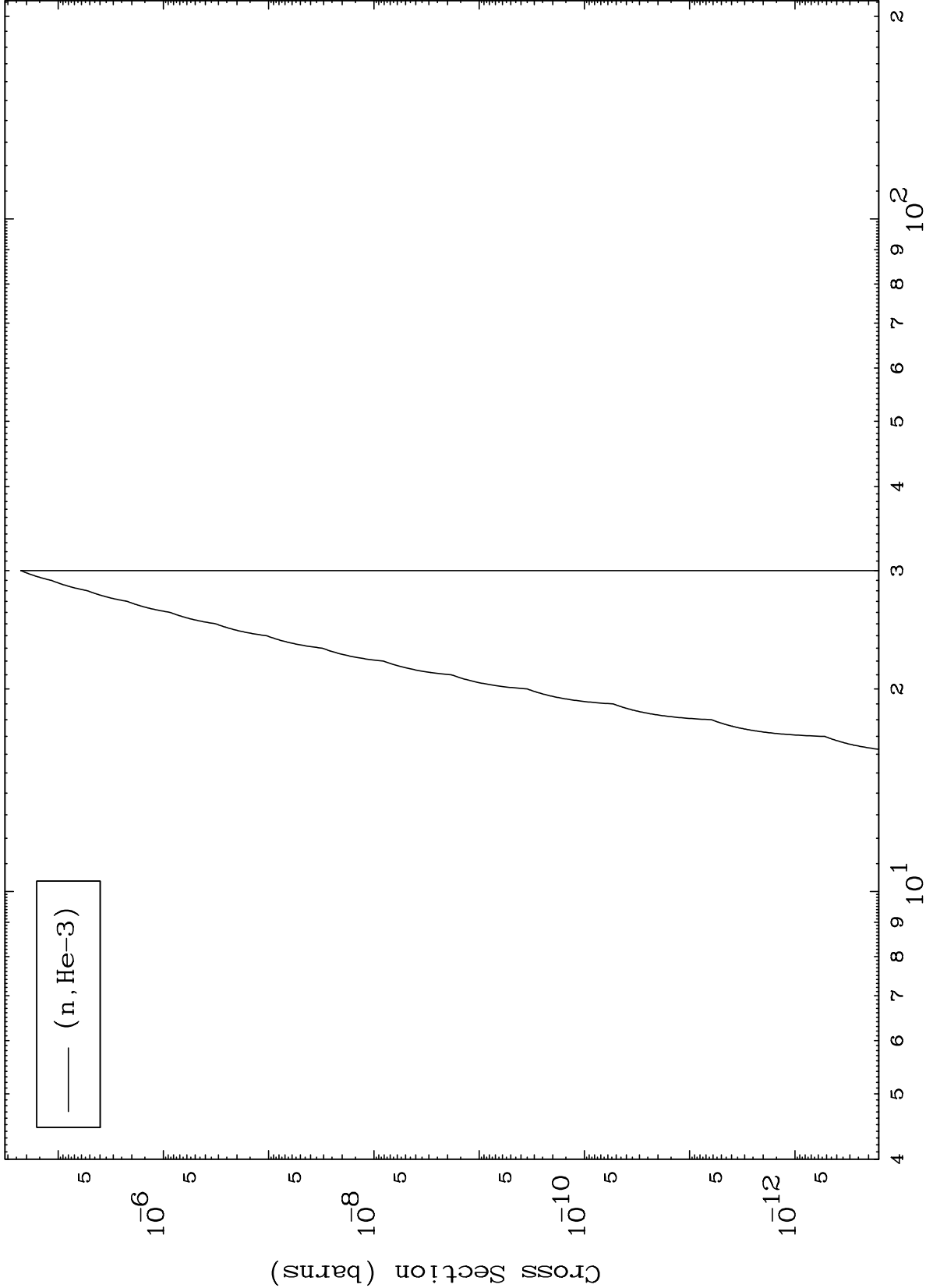
Incident Energy (MeV)

98-Cf-253

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(n,He3) Levels
293 Kelvin Cross Sections

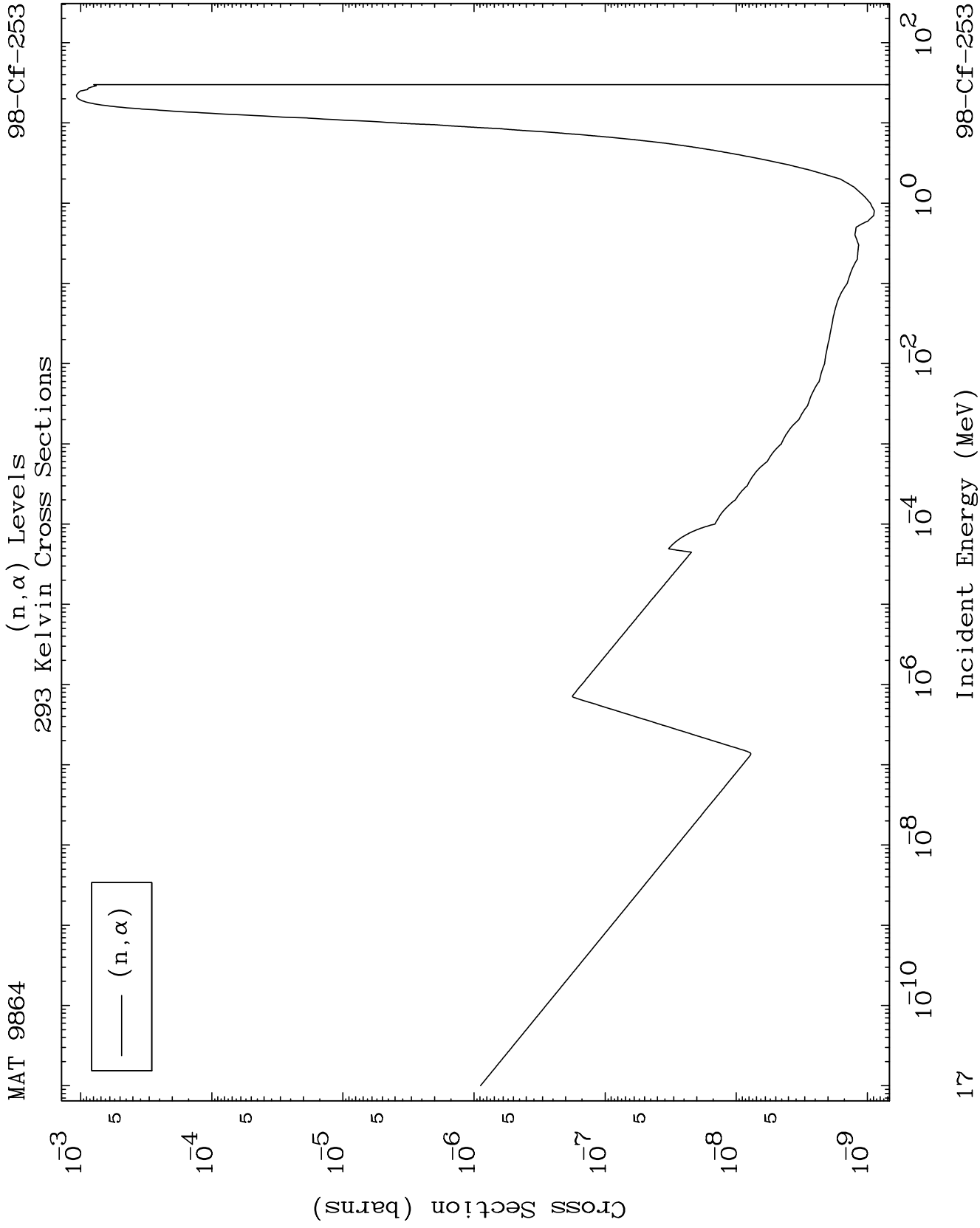
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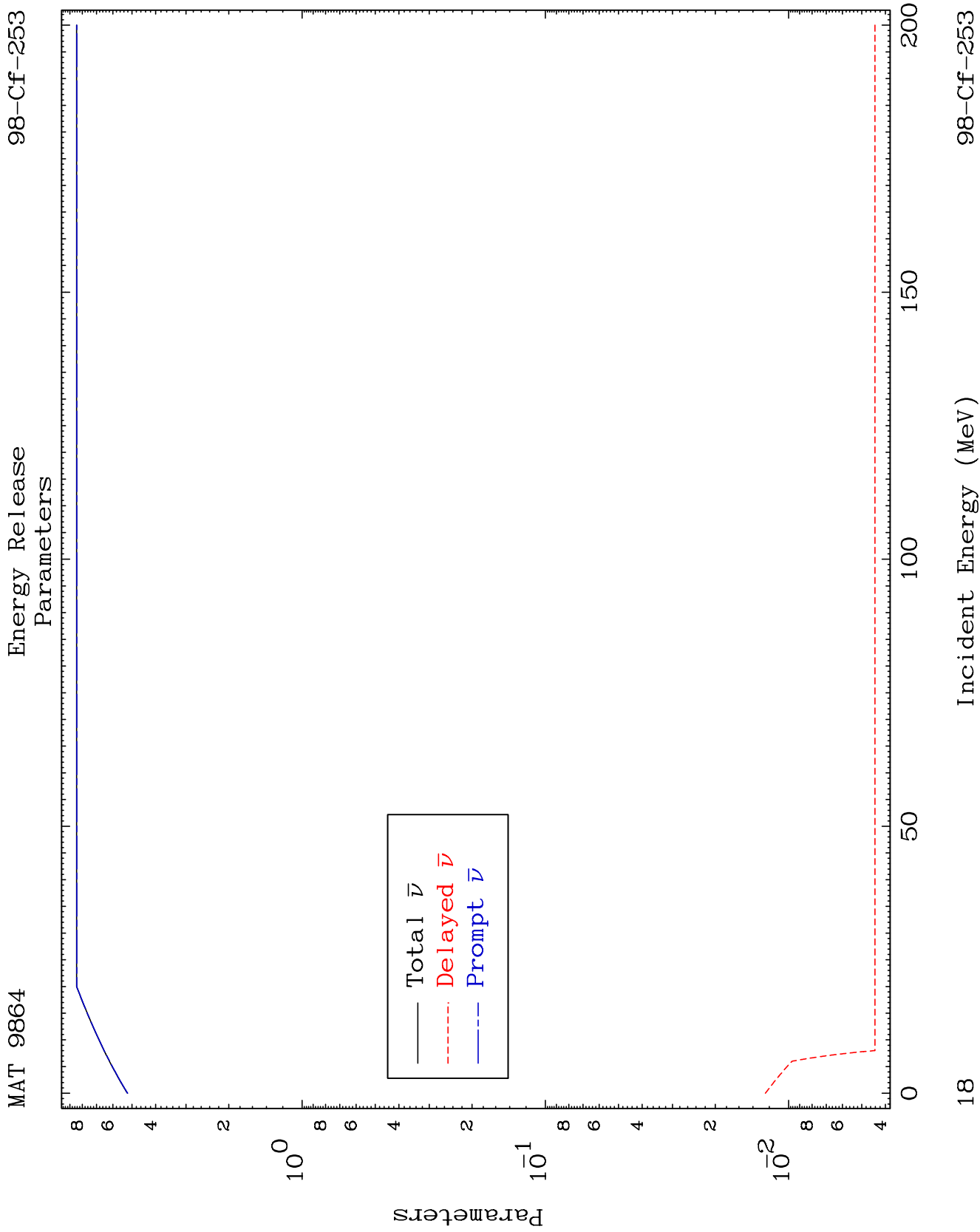


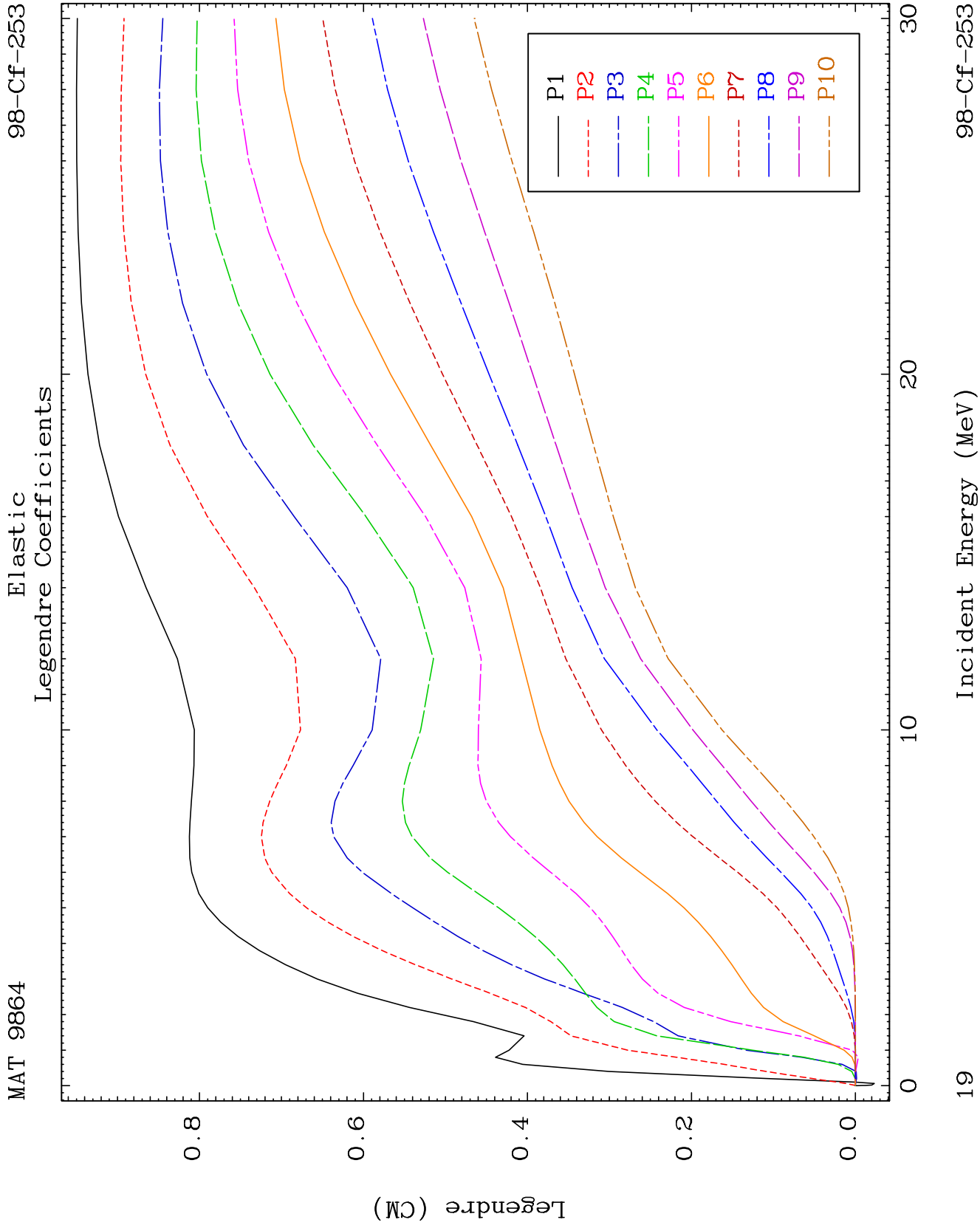
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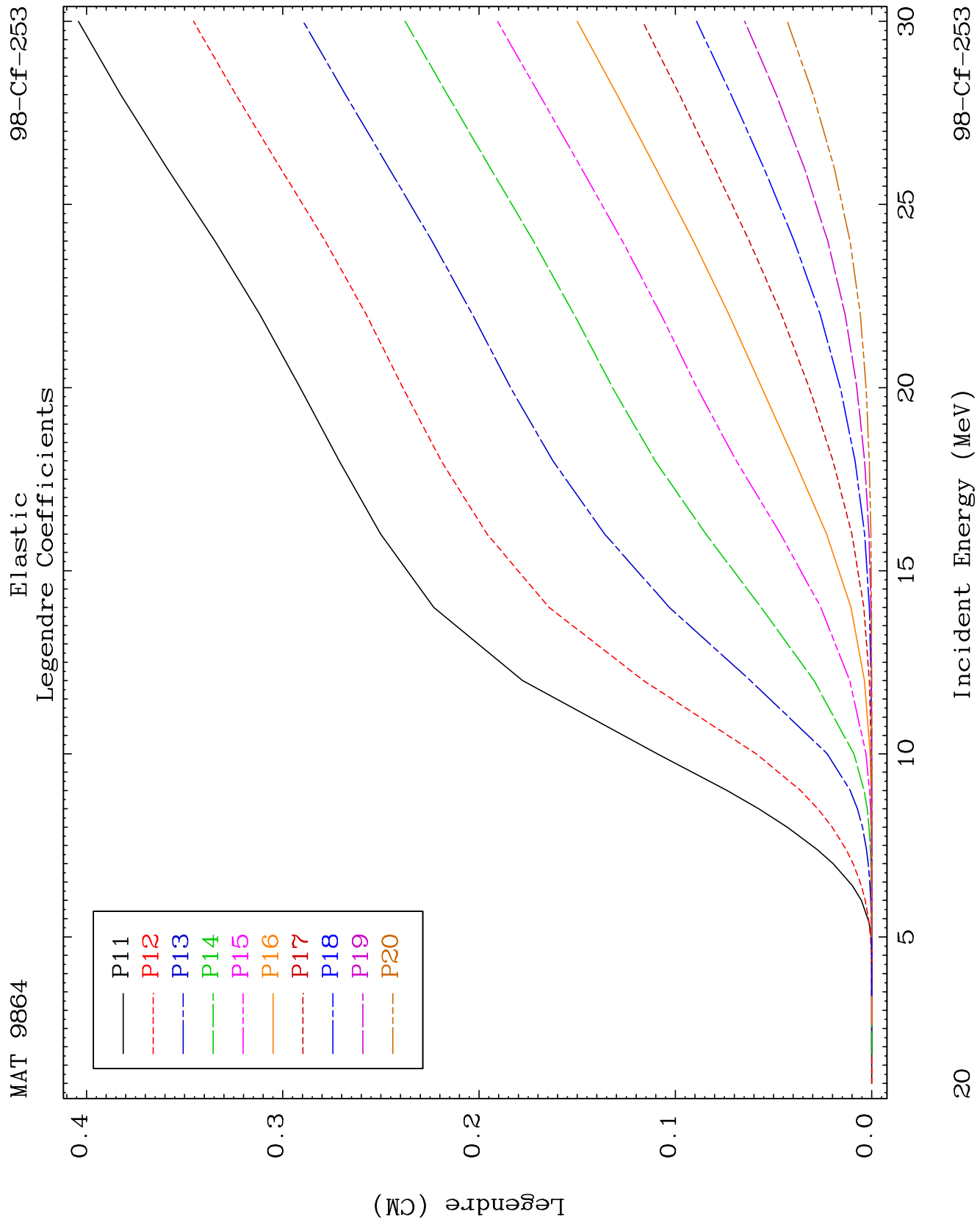
Incident Energy (MeV)

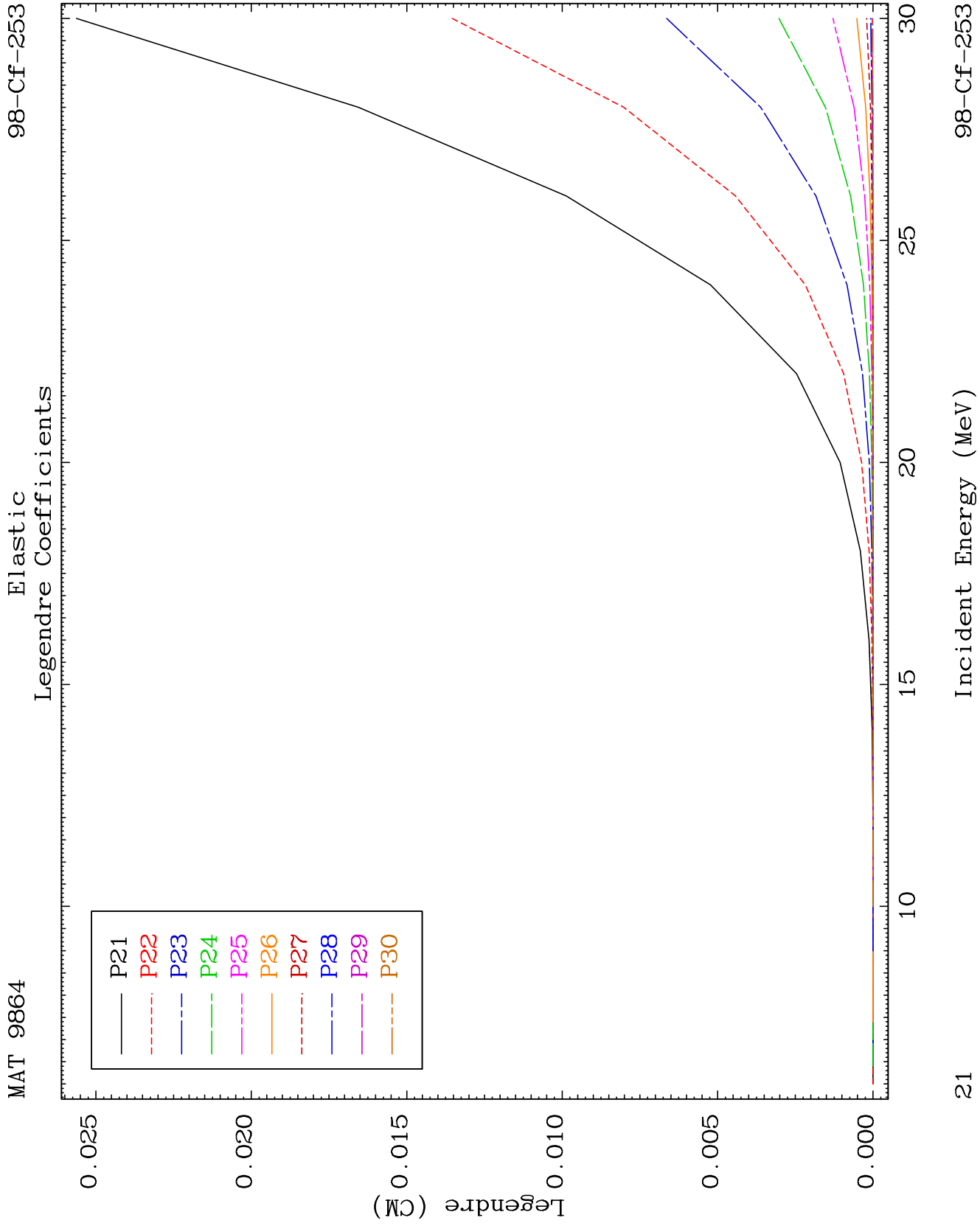
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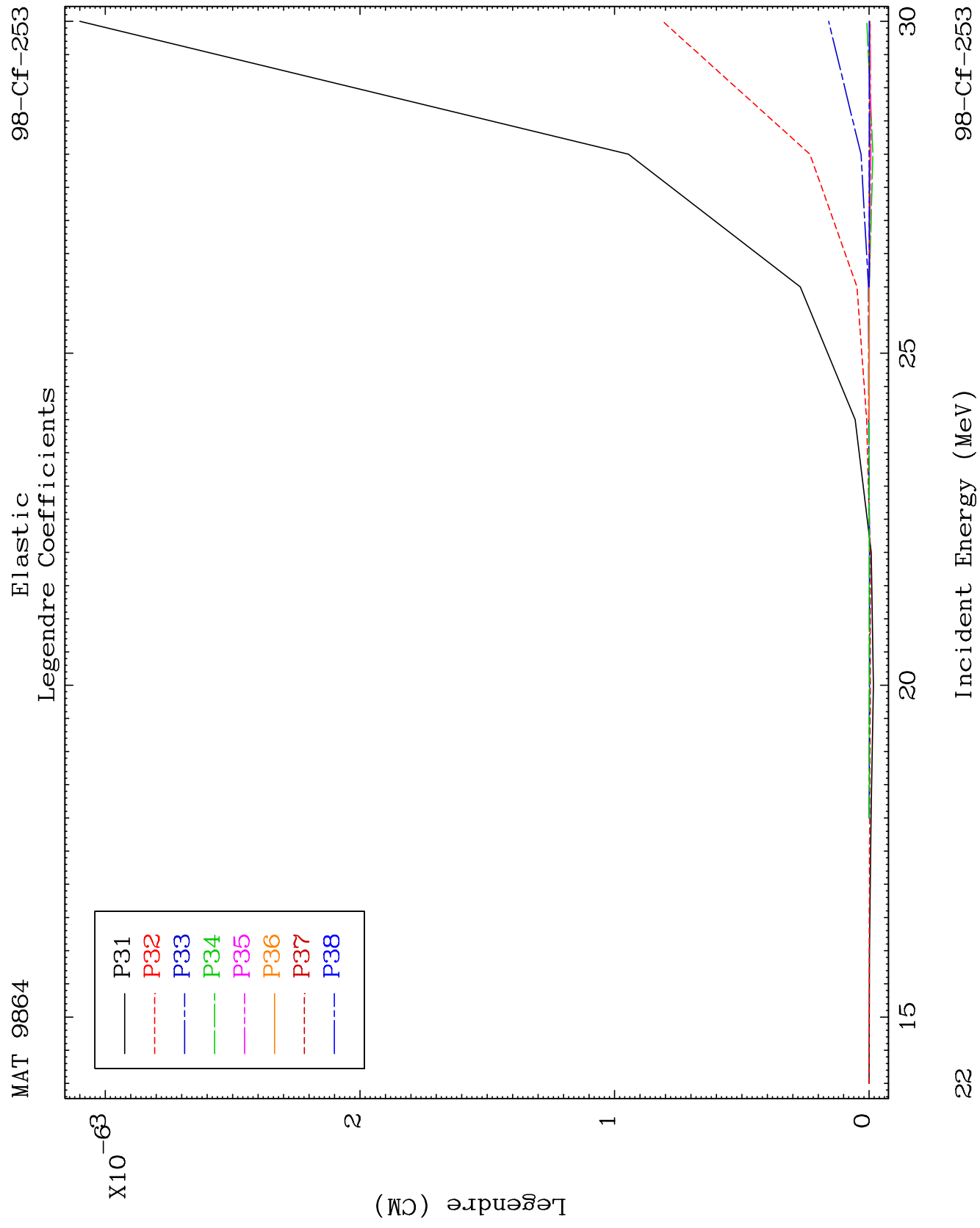










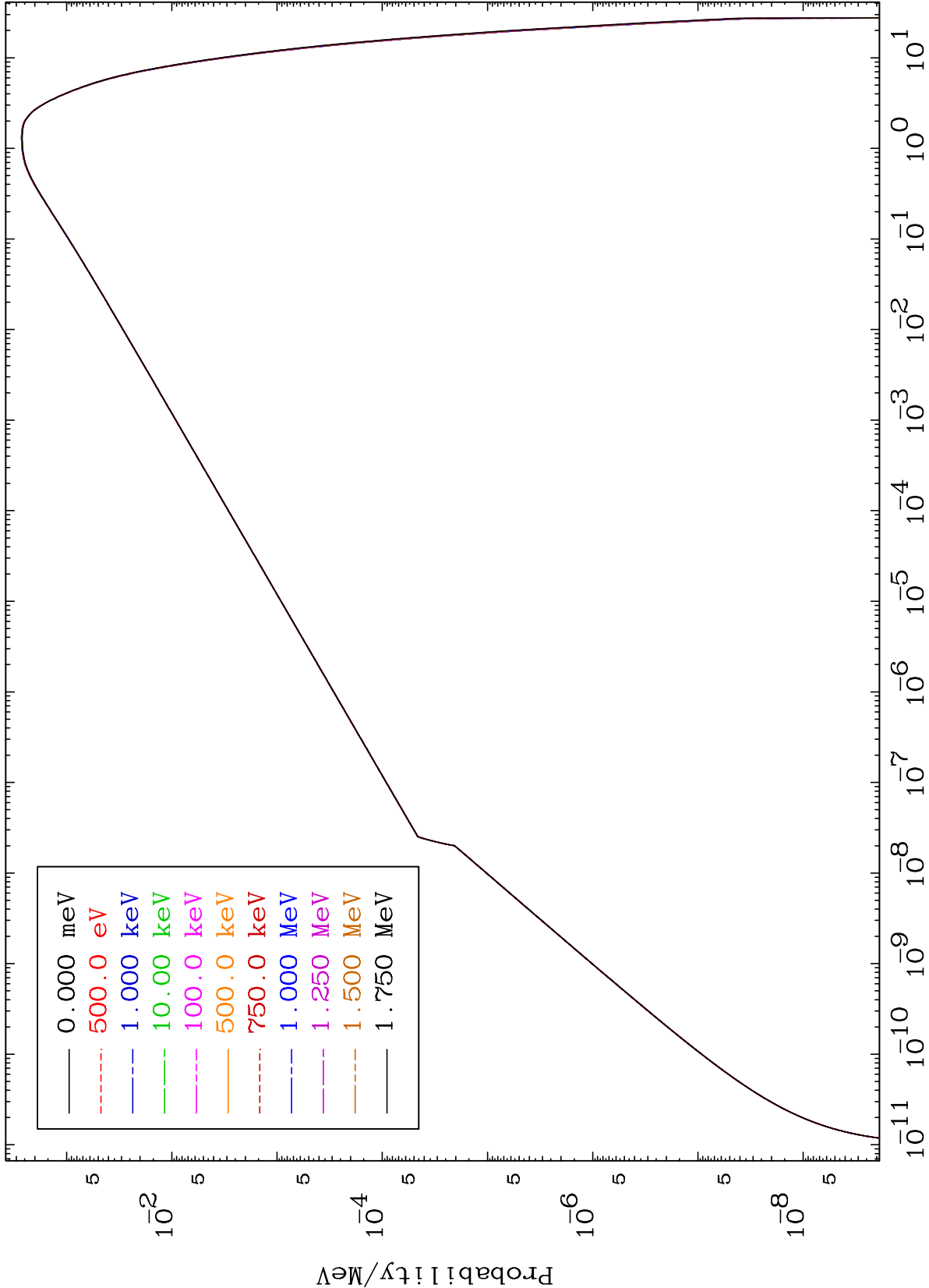


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Fission

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Energy Distributions



98-Cf-253

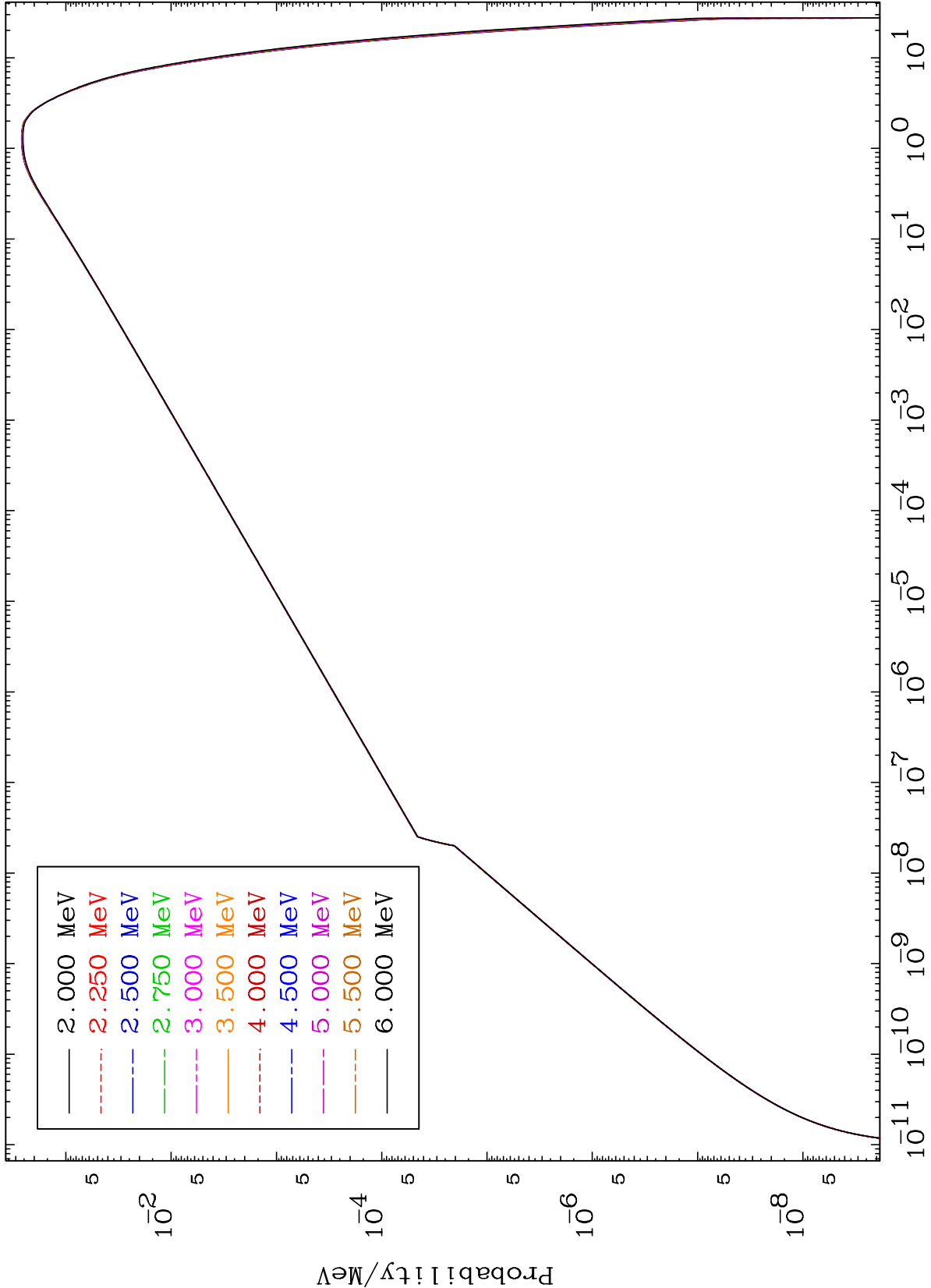
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Fission

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Energy Distributions



Secondary Energy (MeV)

98-Cf-253

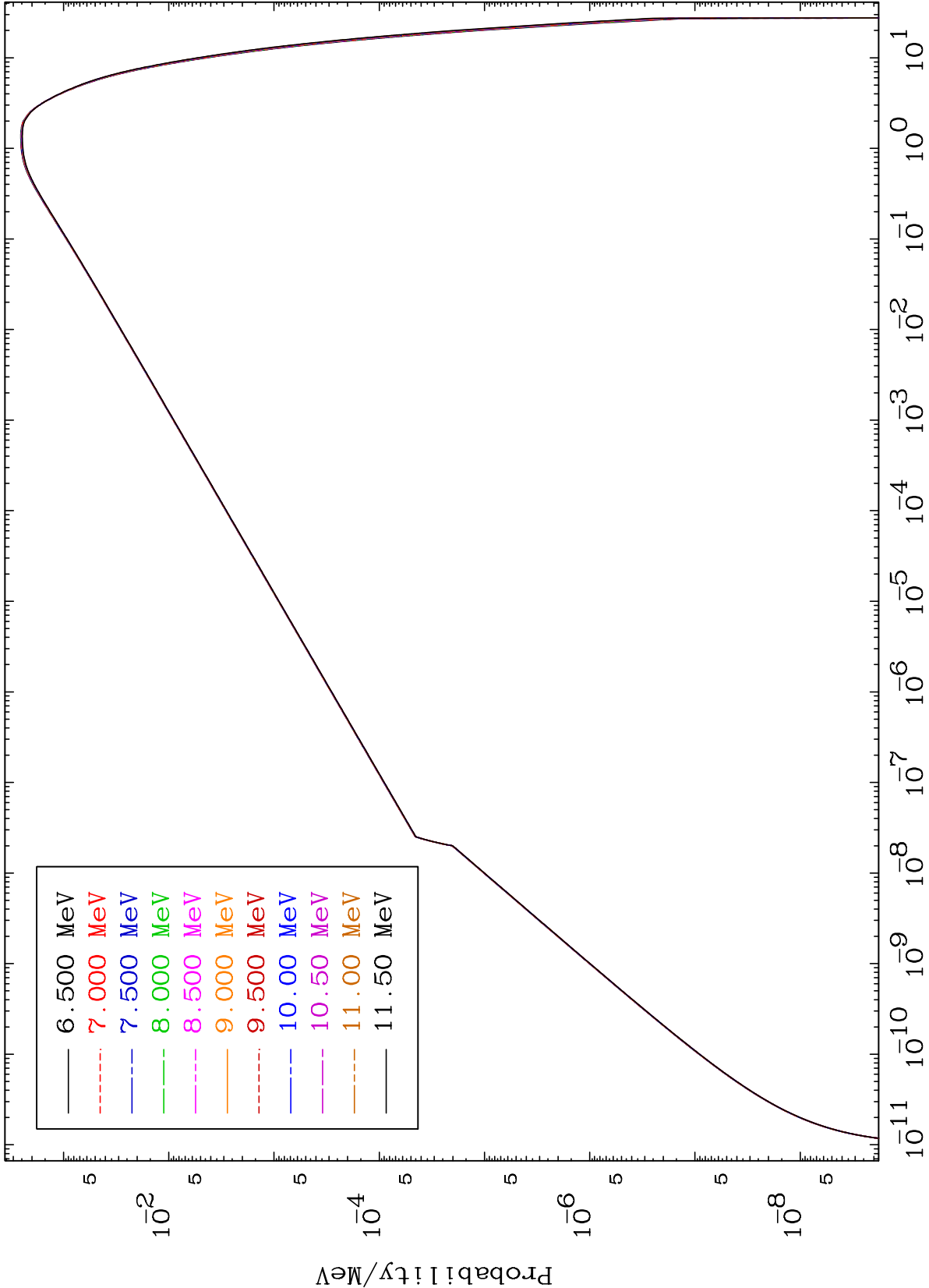
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Fission

98-Cf-253

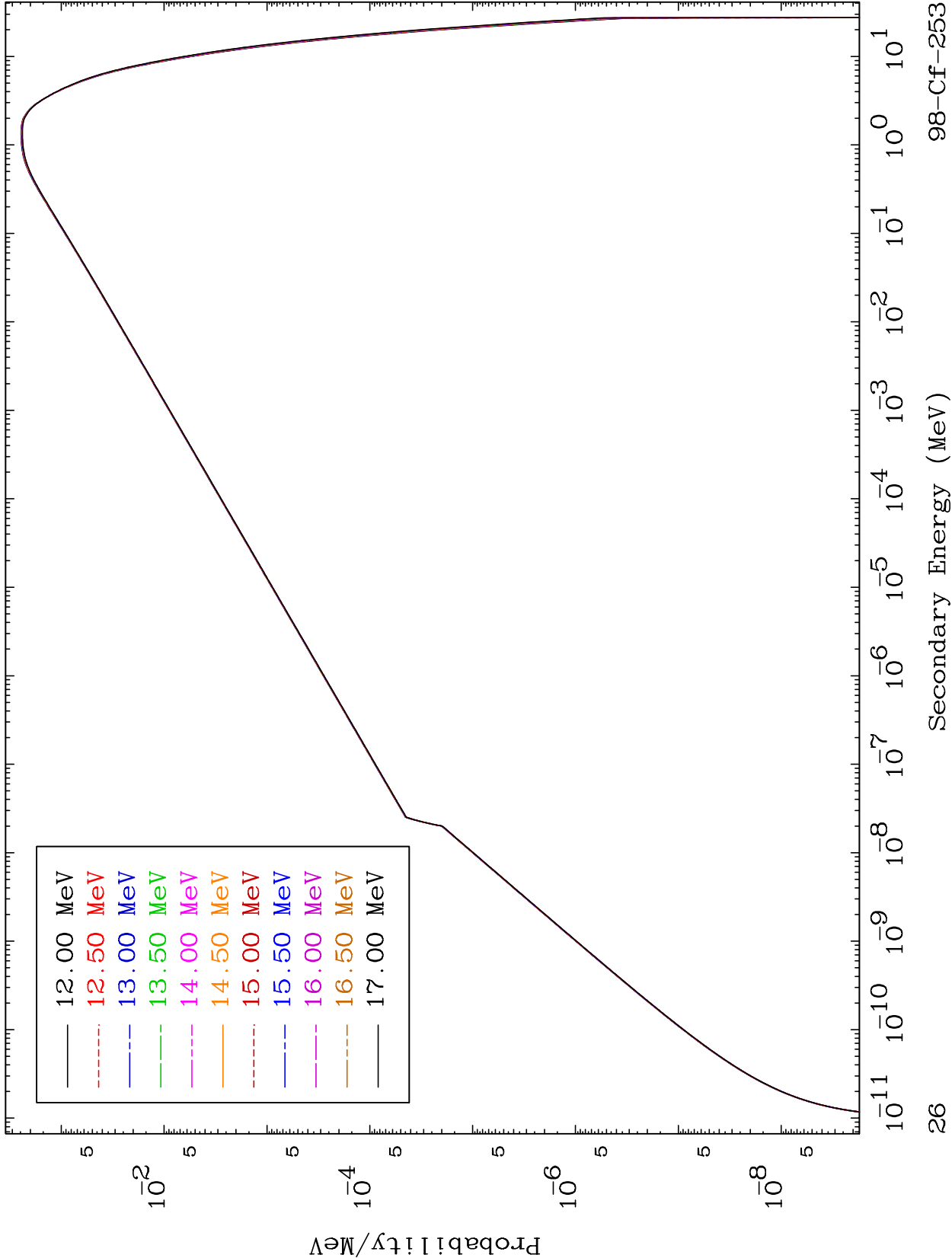
Energy Distributions



Secondary Energy (MeV)

98-Cf-253

25



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Fission
Energy Distributions

98-Cf-253

