

Program EVALPLOT
(Version 2021-1)

by

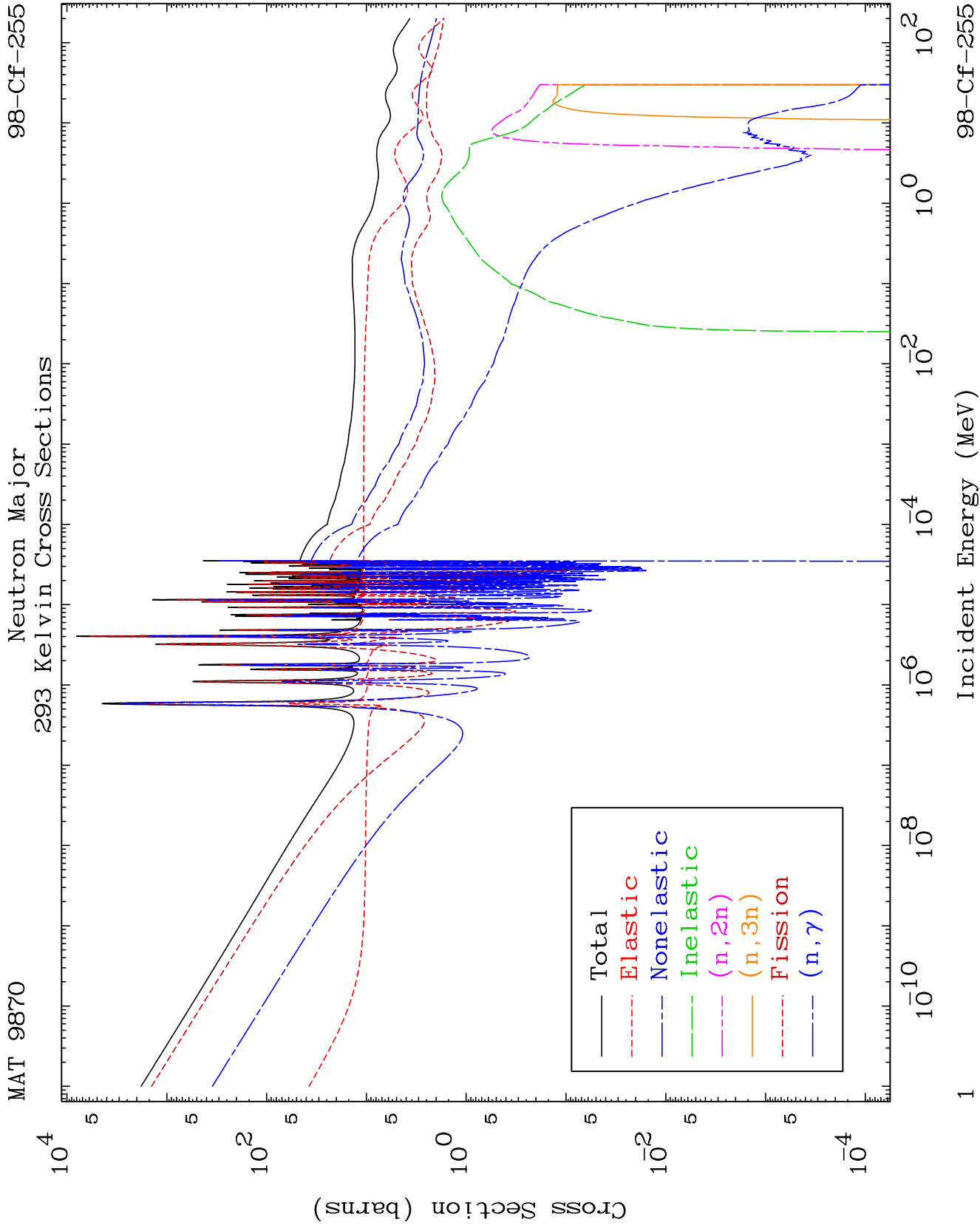
Dermott E. Cullen
(Present Contact Information)

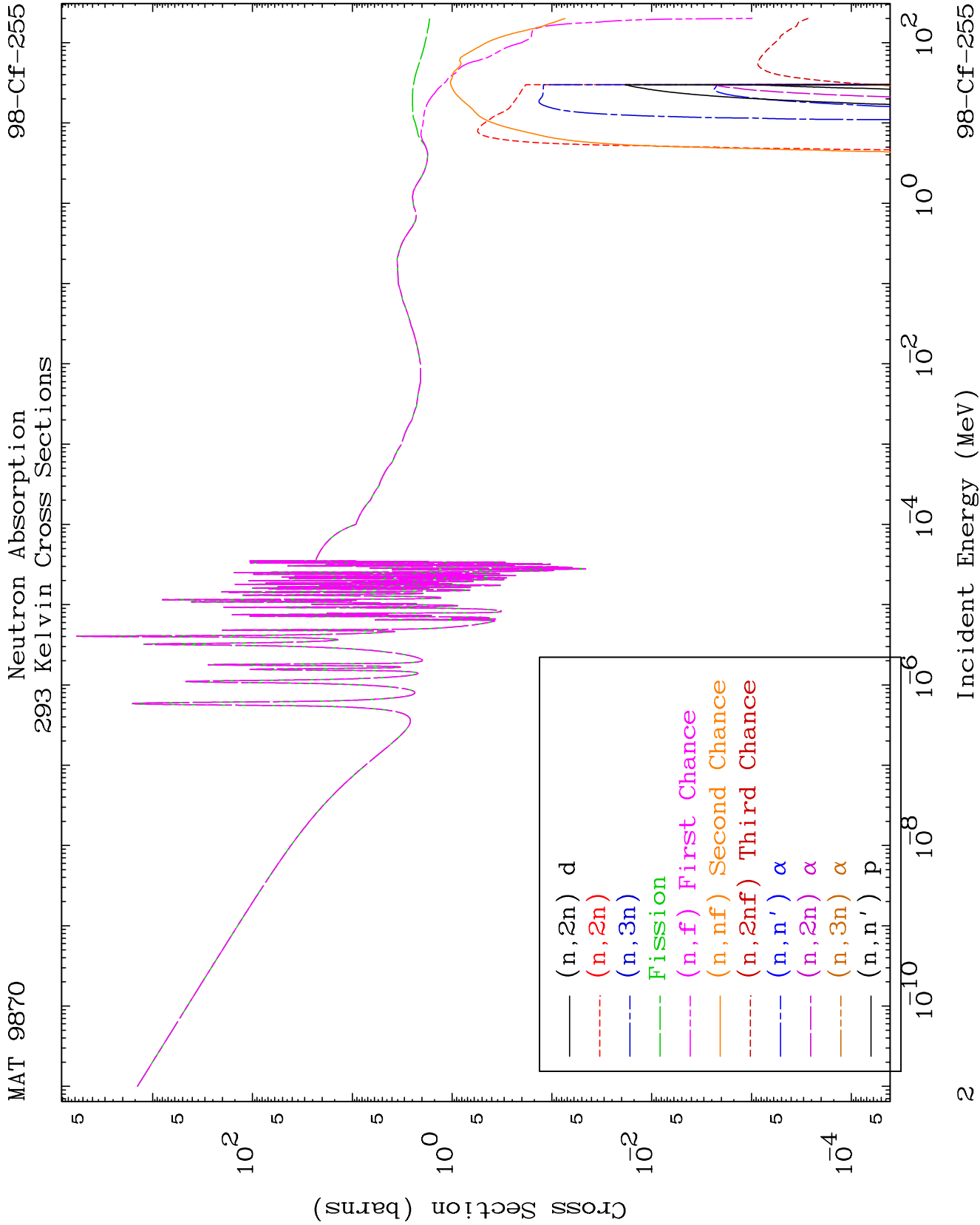
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

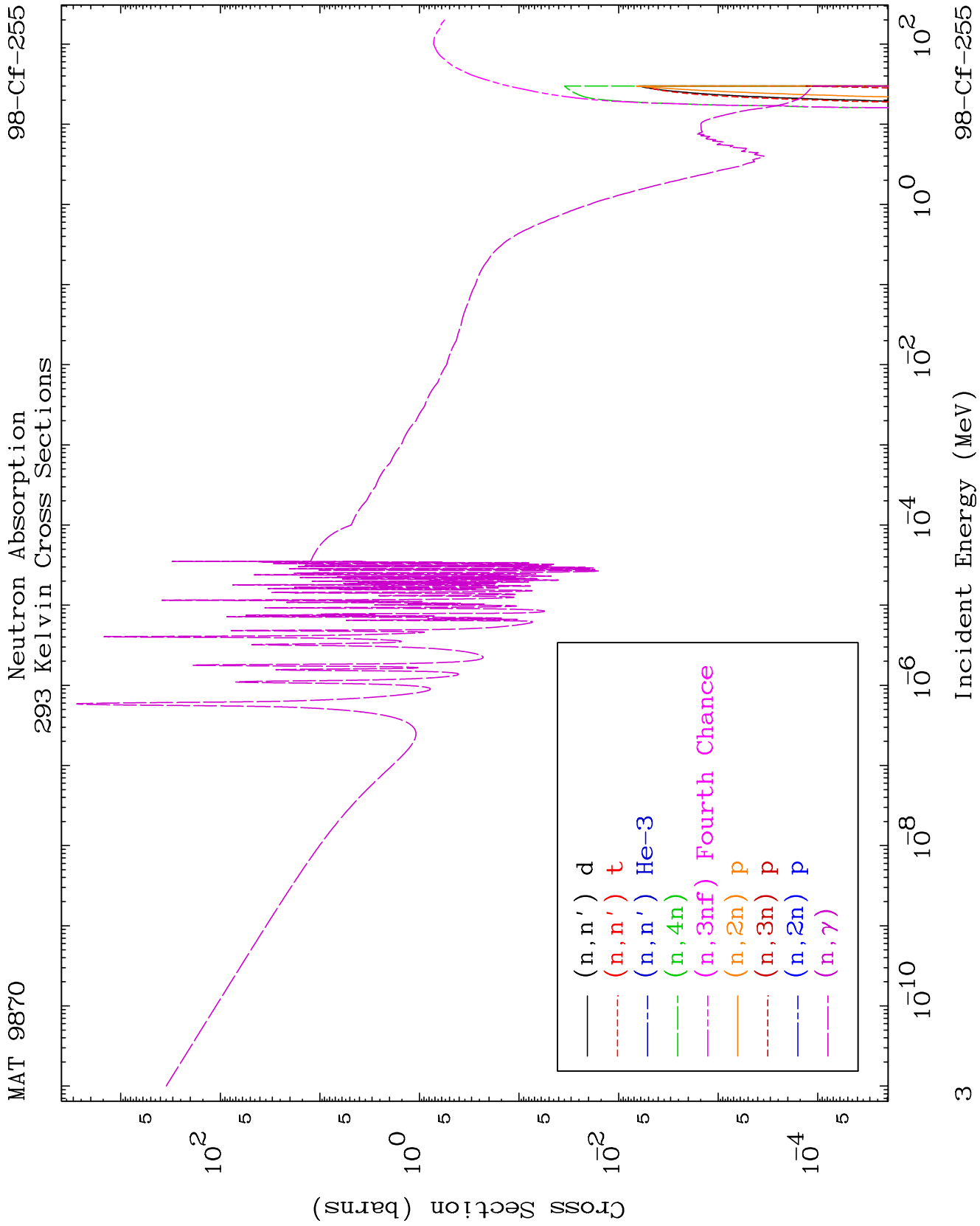
Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
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Press Mouse Button to Start



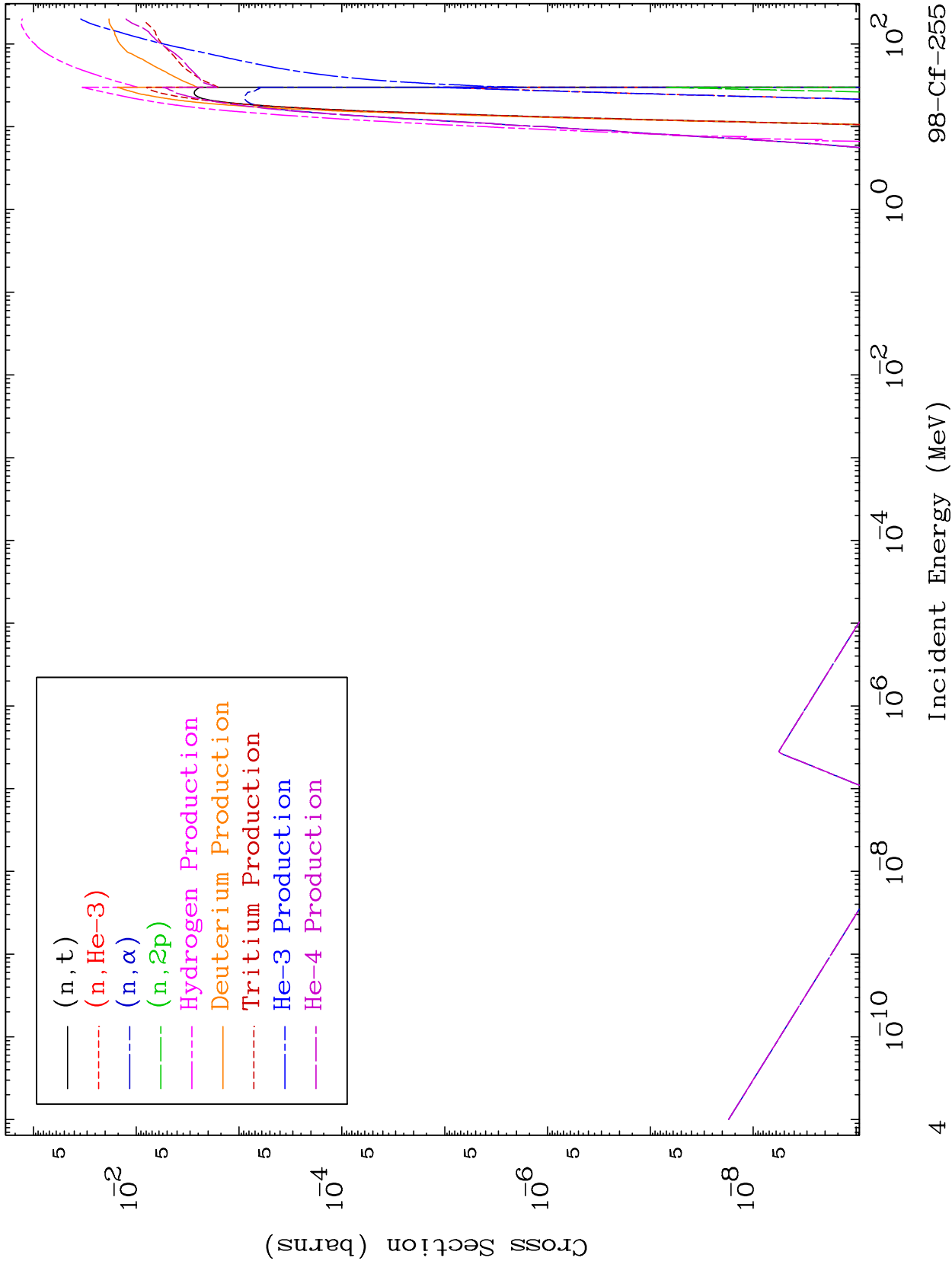


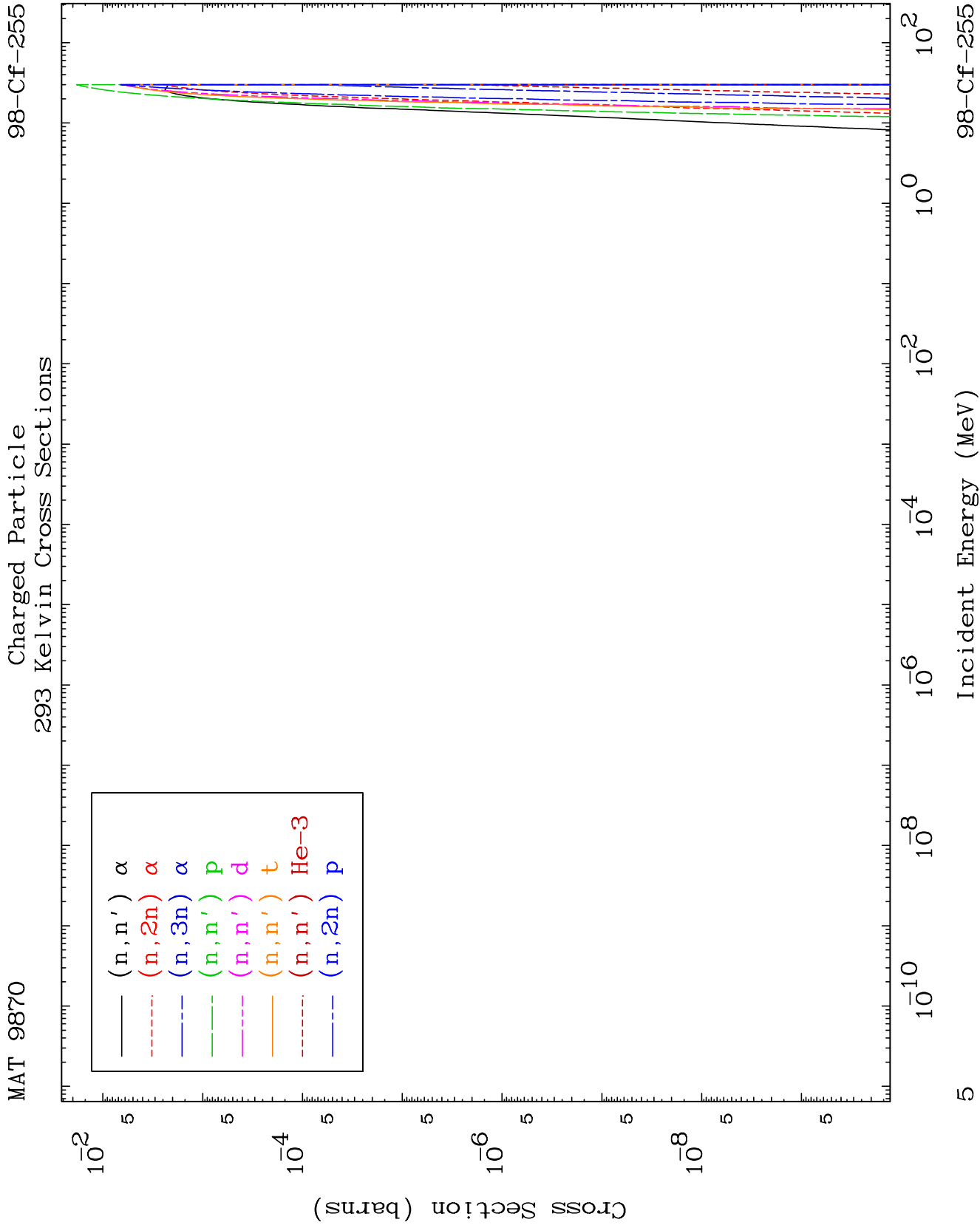


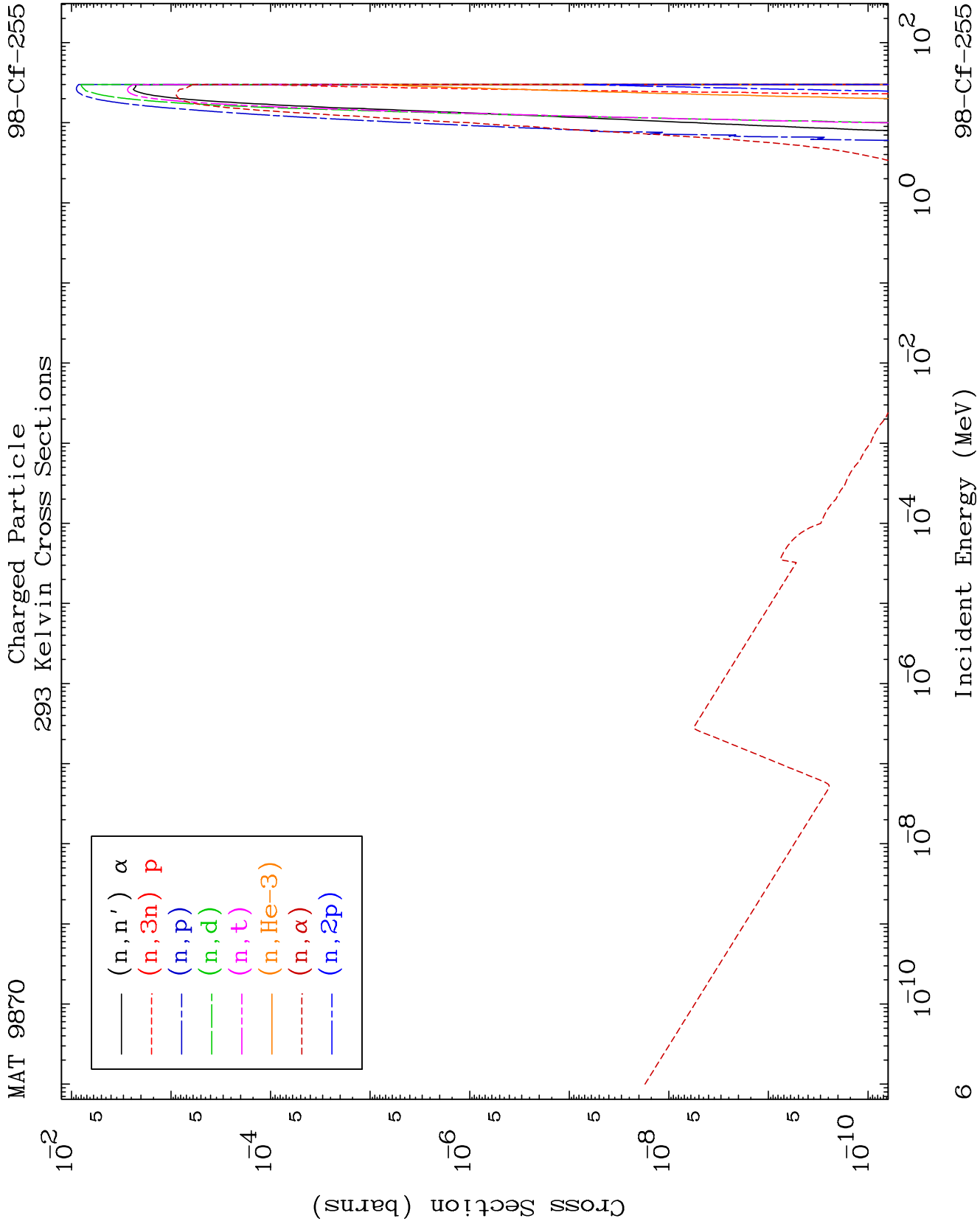
MAT 9870

Neutron Absorption
293 Kelvin Cross Sections

98-Cf-255



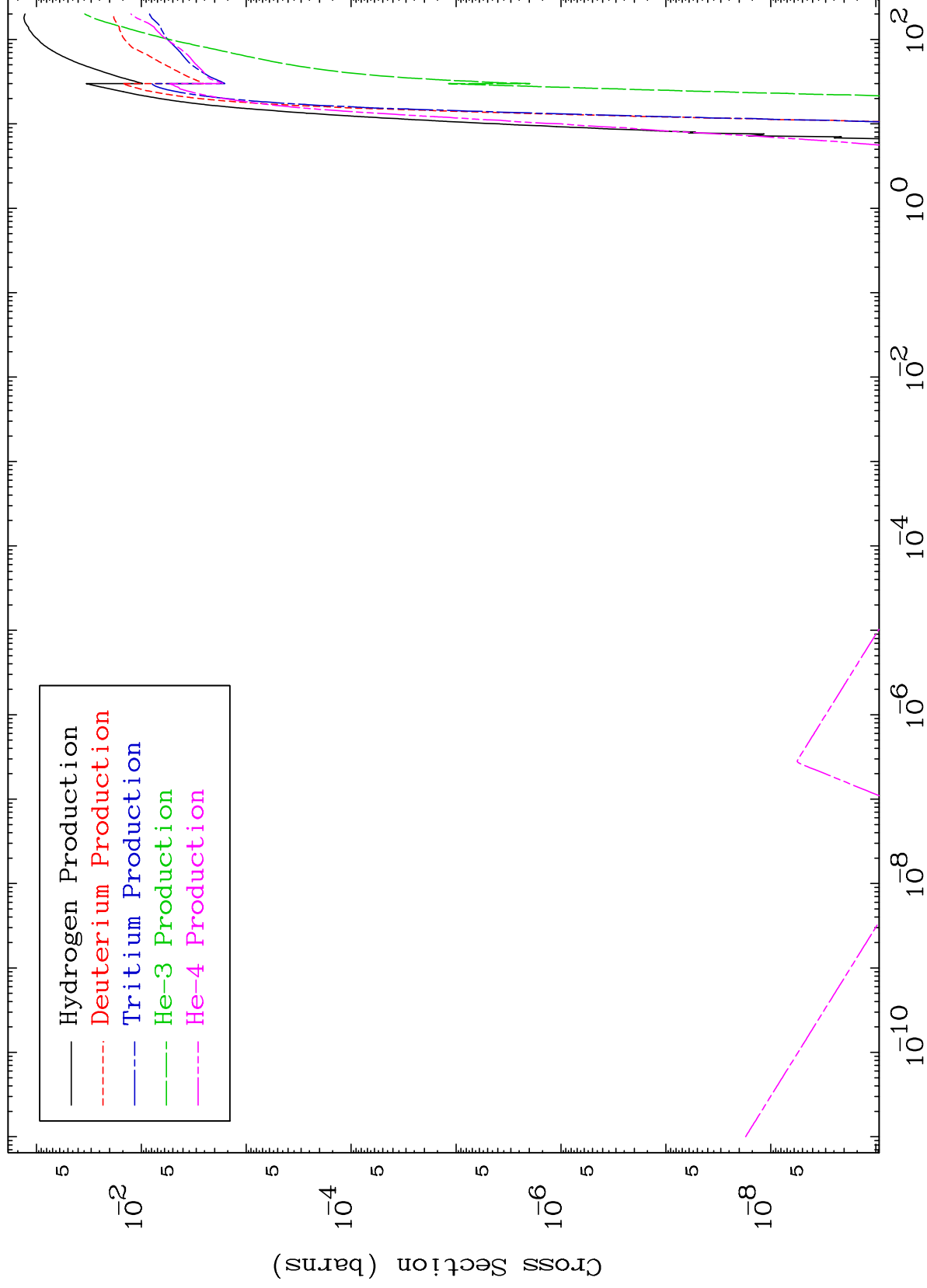


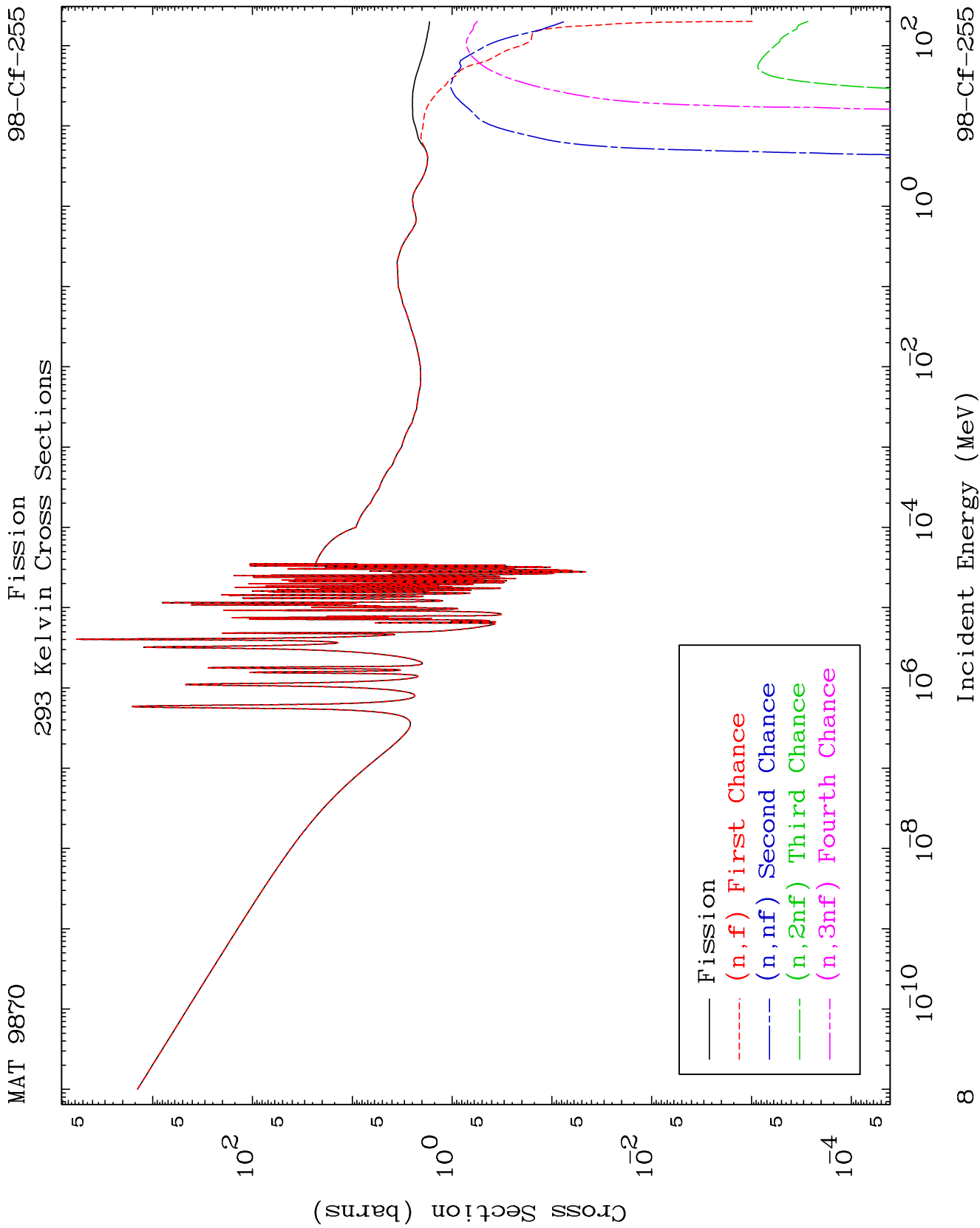


MAT 9870

Particle Production
293 Kelvin Cross Sections

98-Cf-255



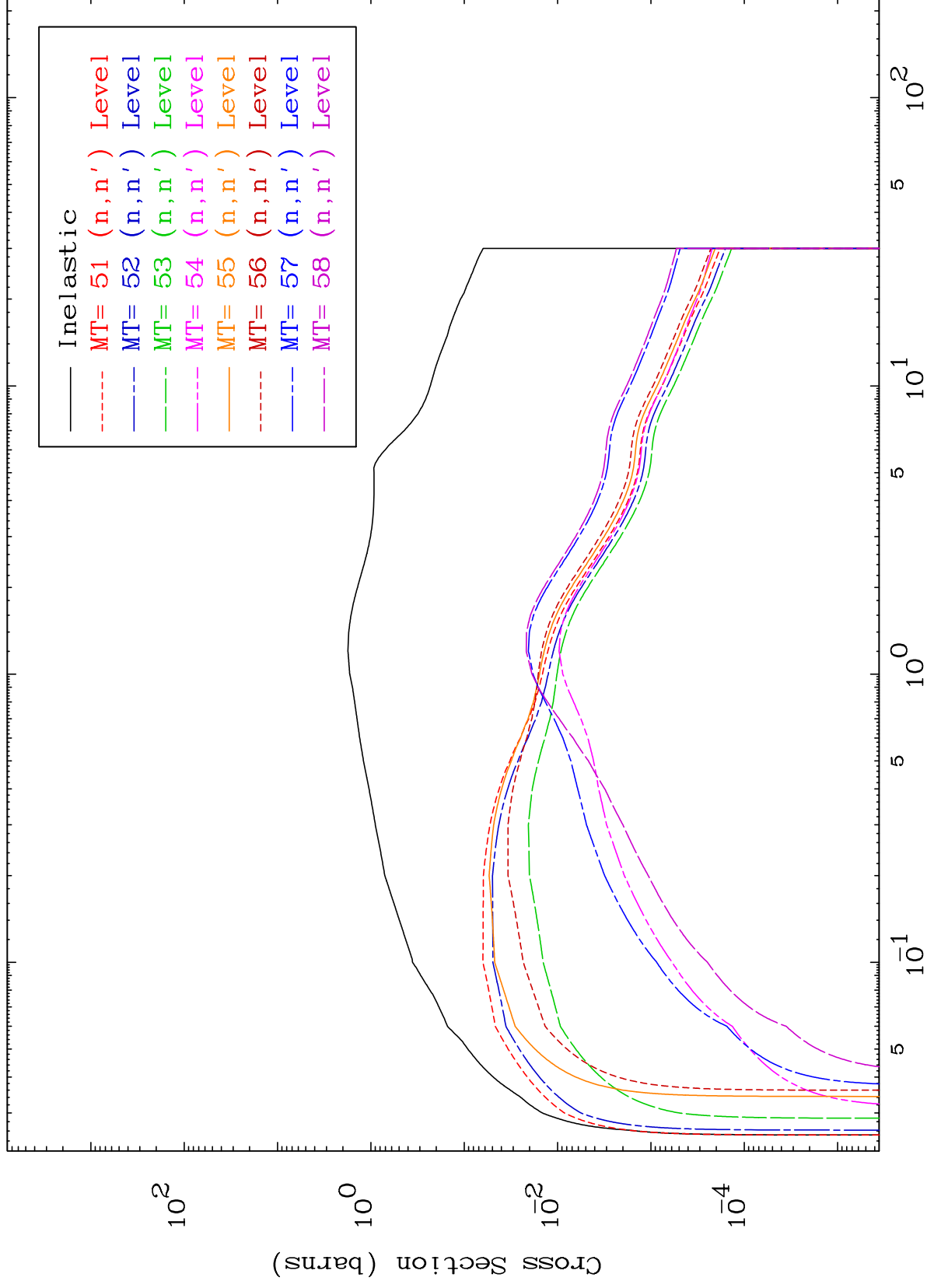


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(n,n') Levels

98-Cf-255

293 Kelvin Cross Sections

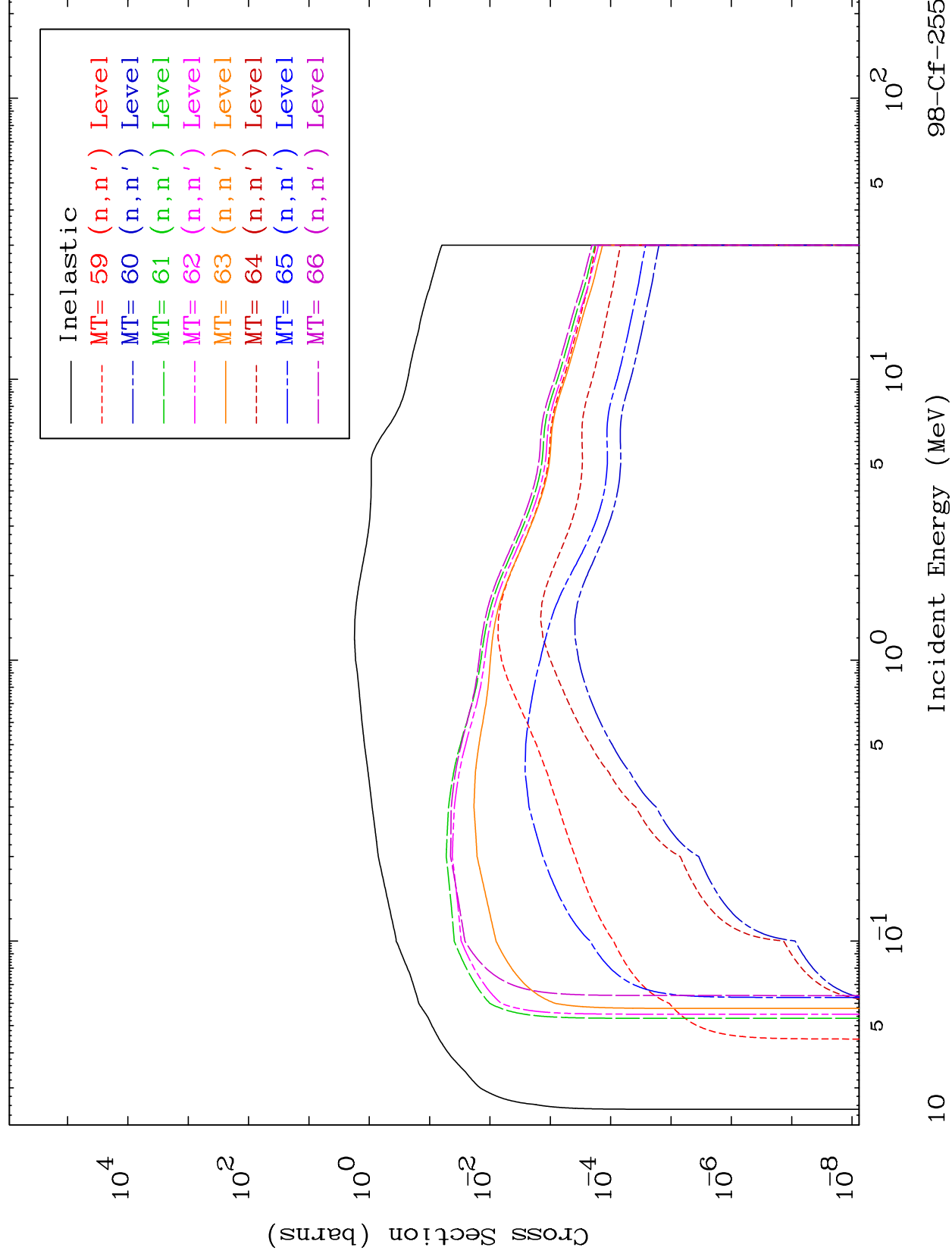


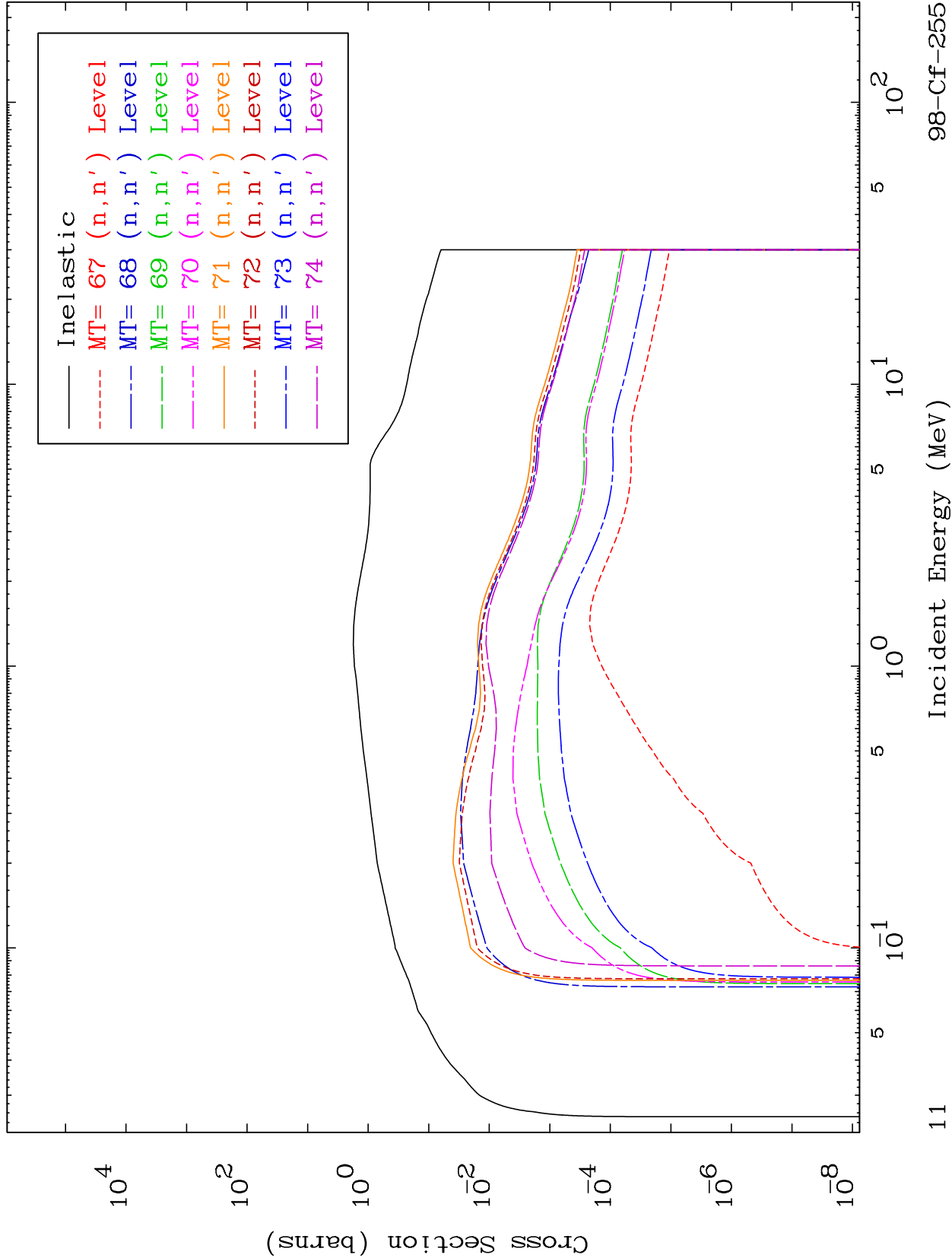
MAT 9870

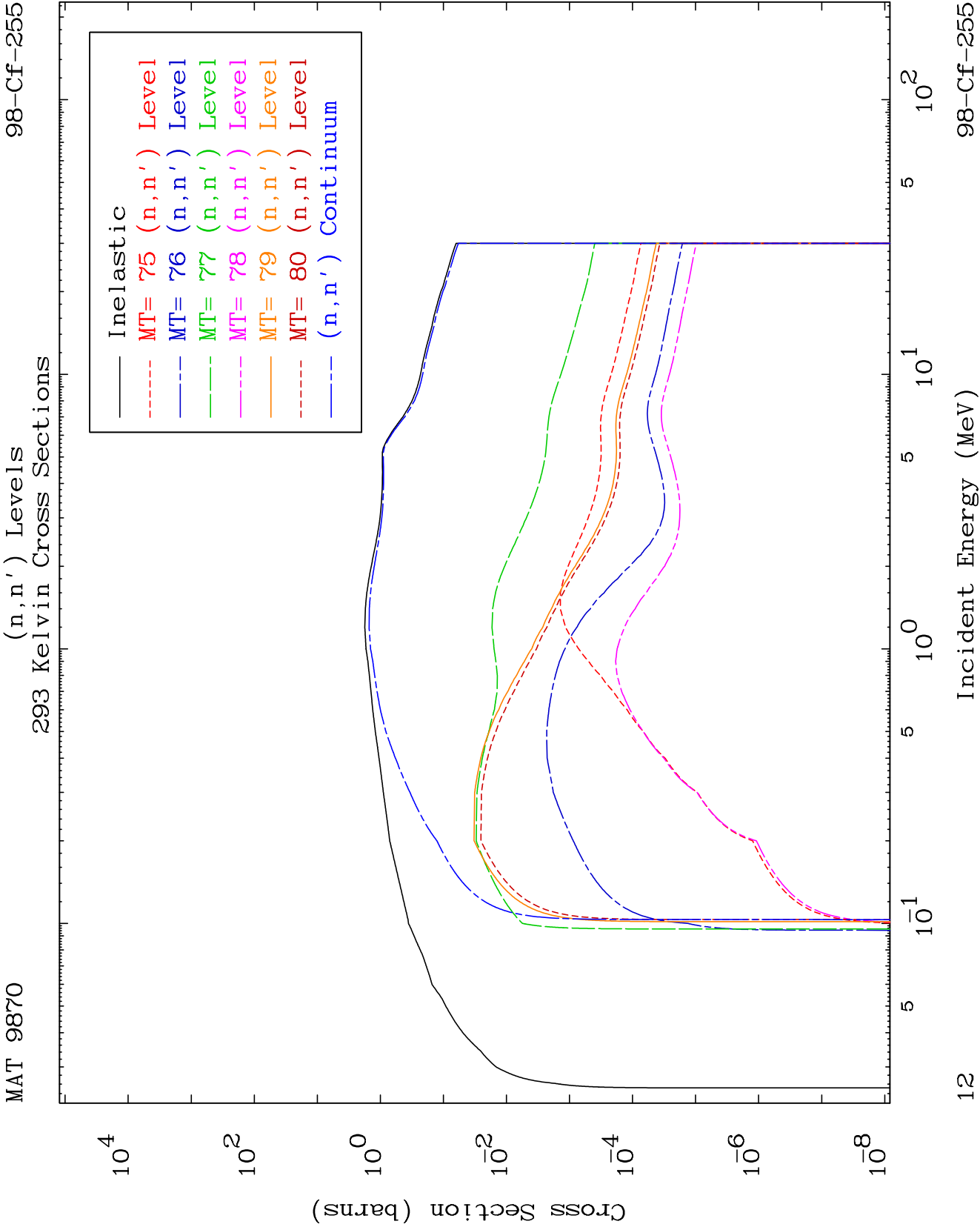
(n, n') Levels

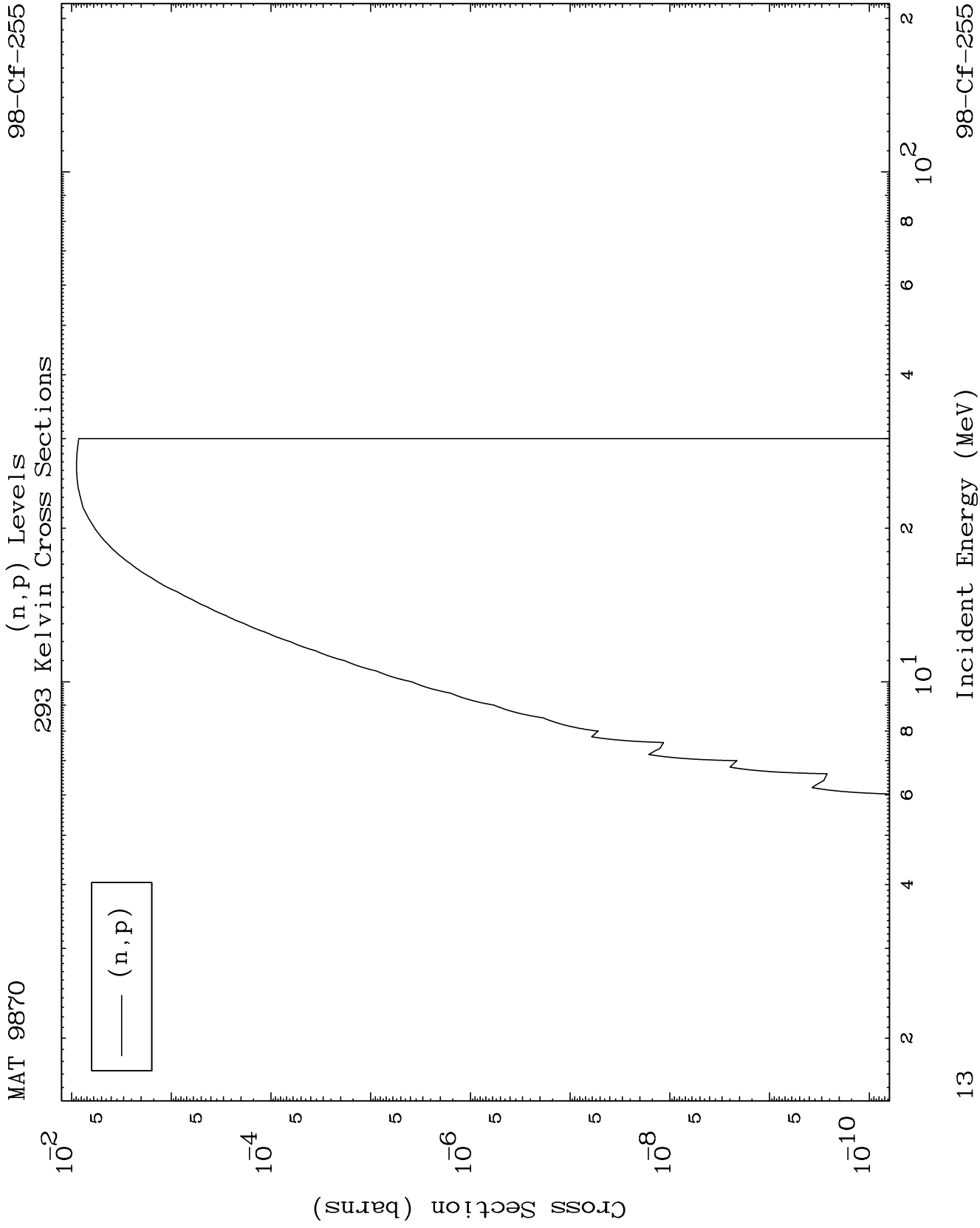
293 Kelvin Cross Sections

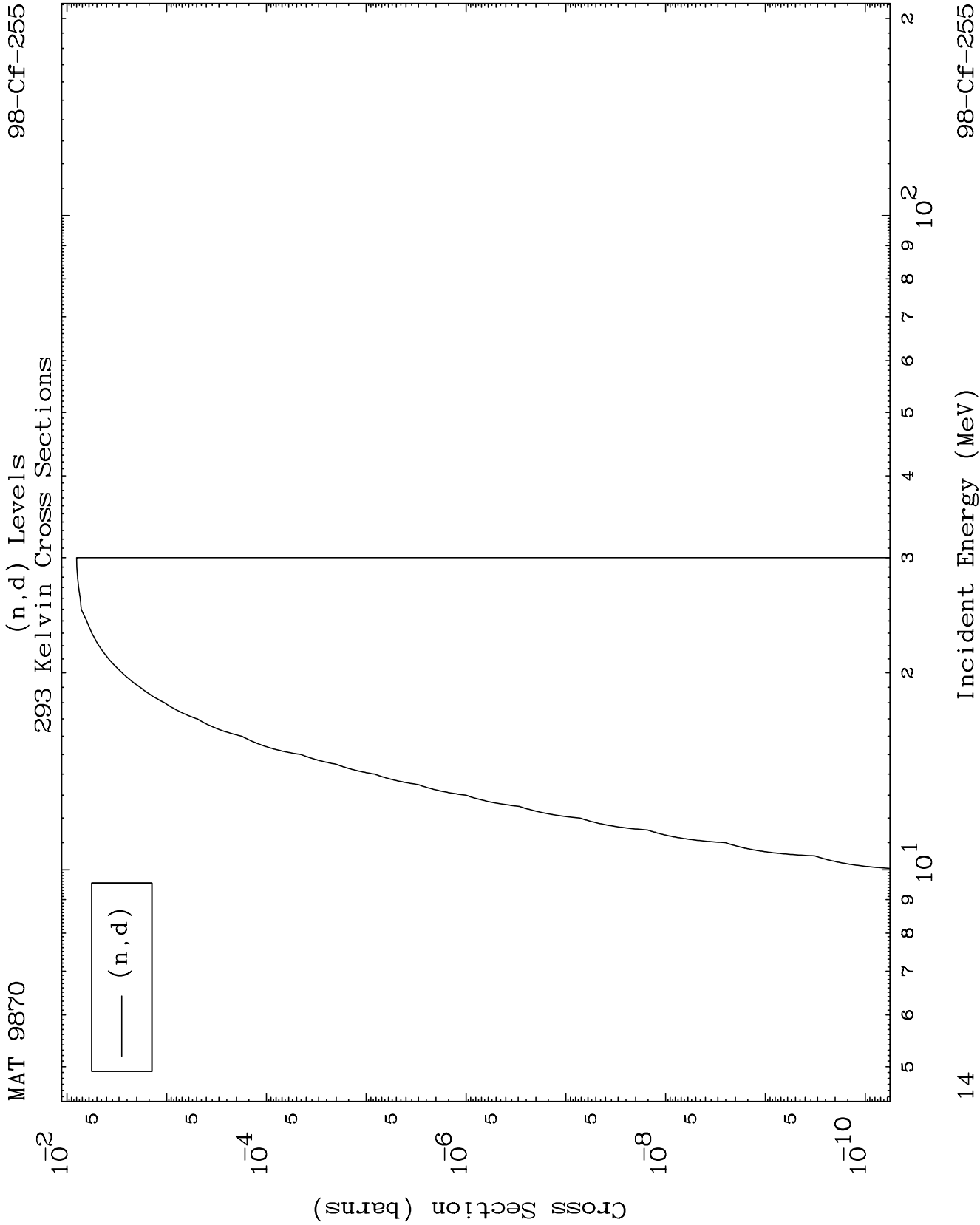
98-Cf-255







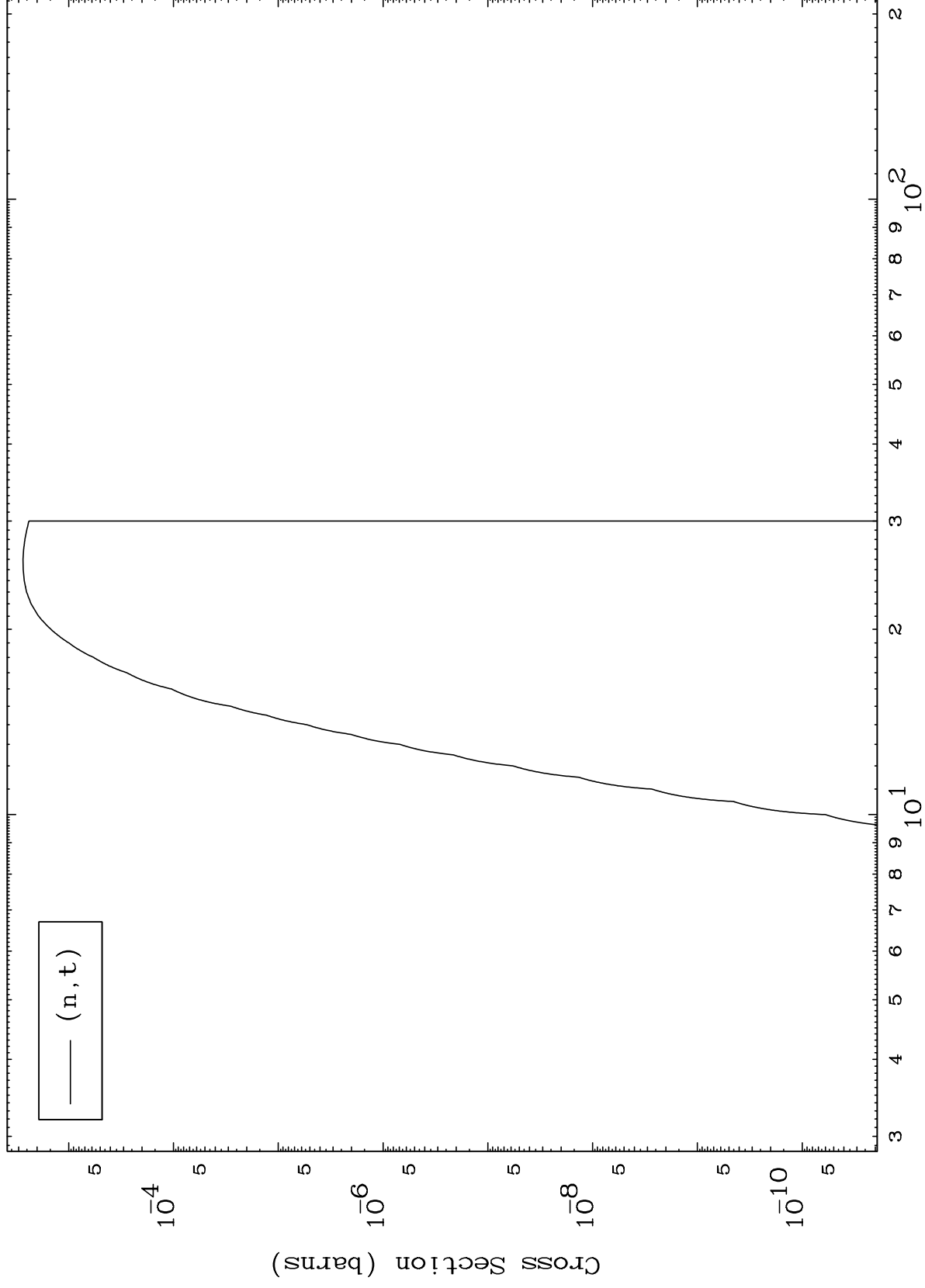




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98-Cf-255

(n,t) Levels
293 Kelvin Cross Sections



15

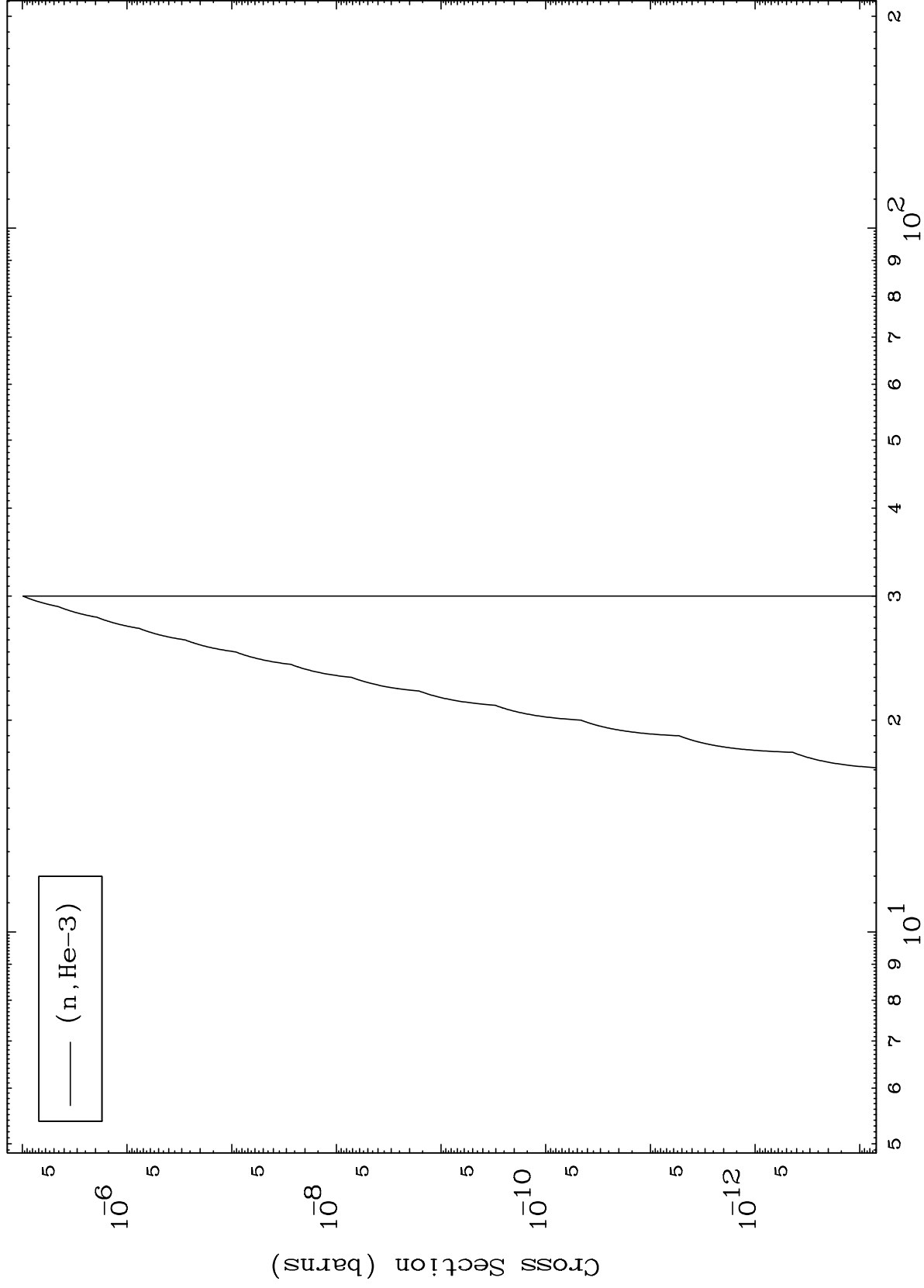
Incident Energy (MeV)

98-Cf-255

MAT 9870

98-Cf-255

(n,He3) Levels
293 Kelvin Cross Sections



98-Cf-255

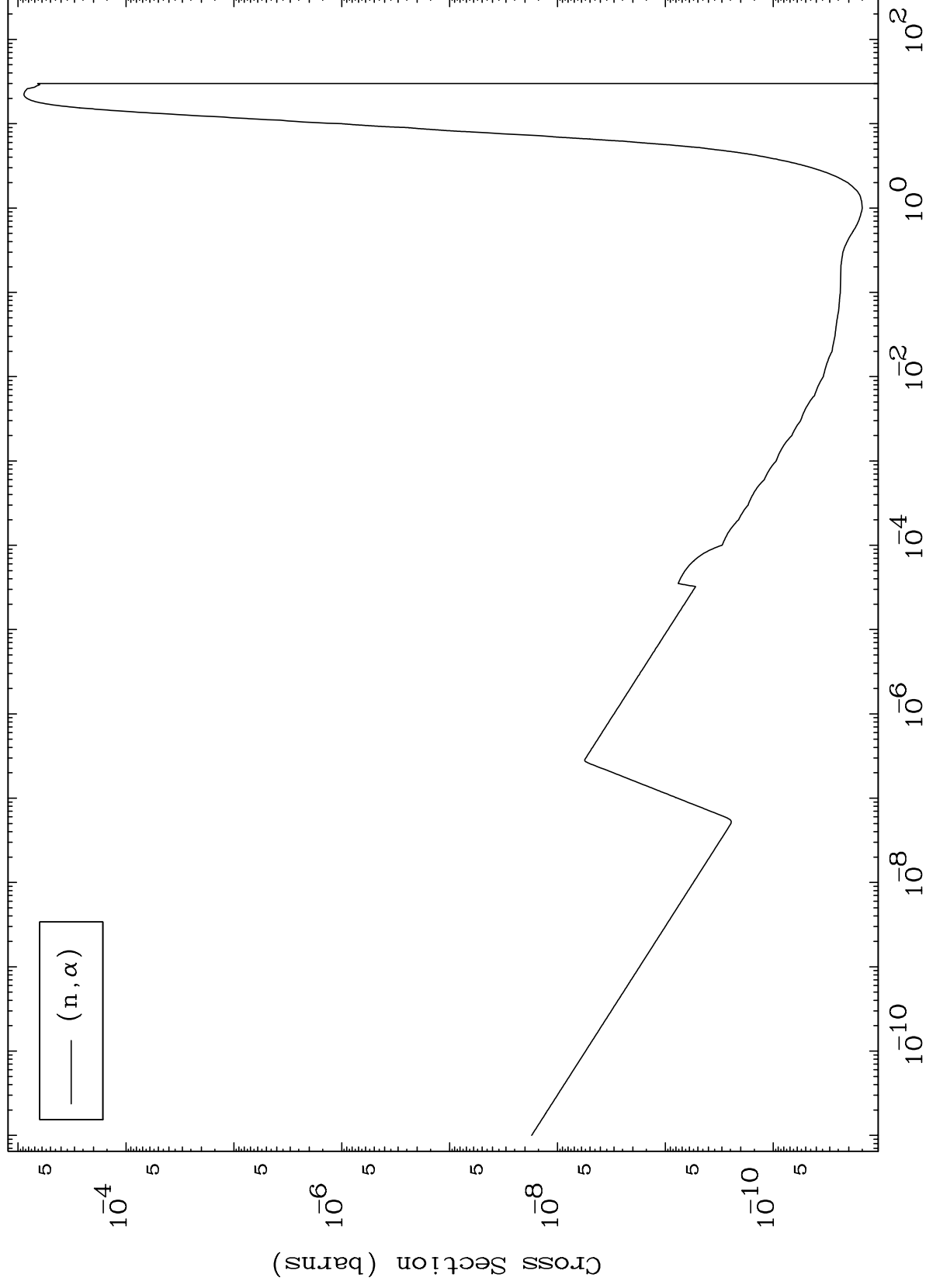
Incident Energy (MeV)

16

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(n, α) Levels
293 Kelvin Cross Sections

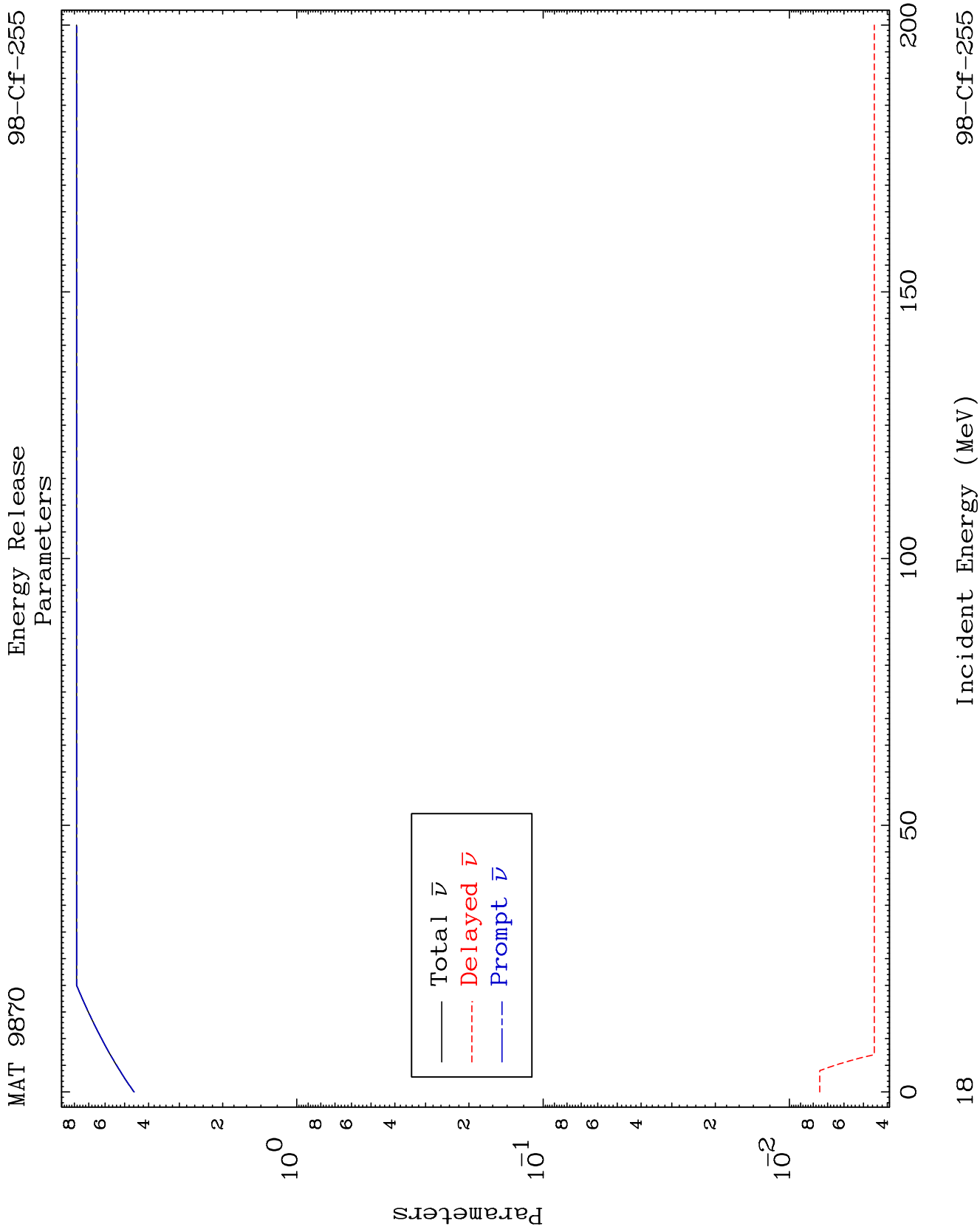
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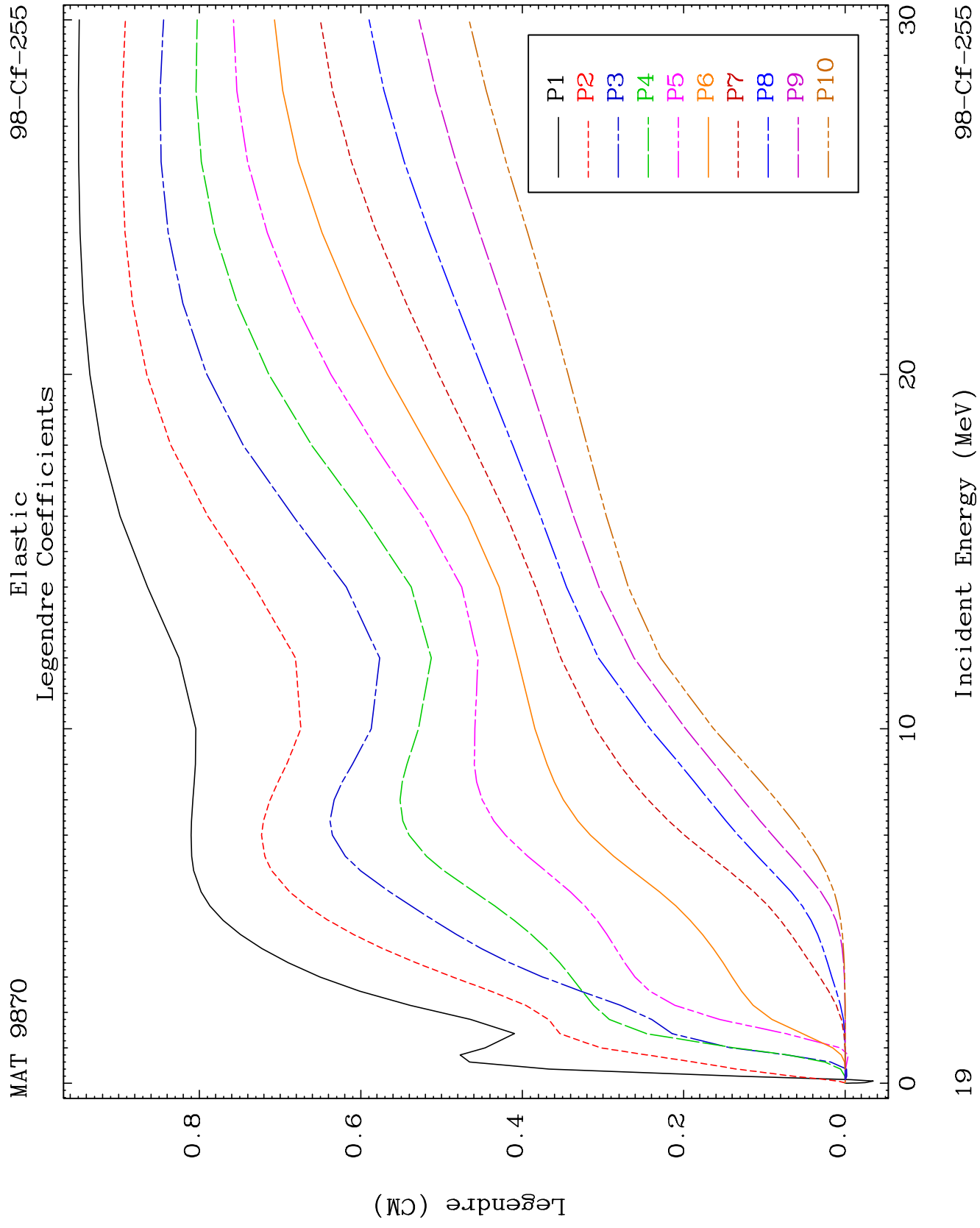


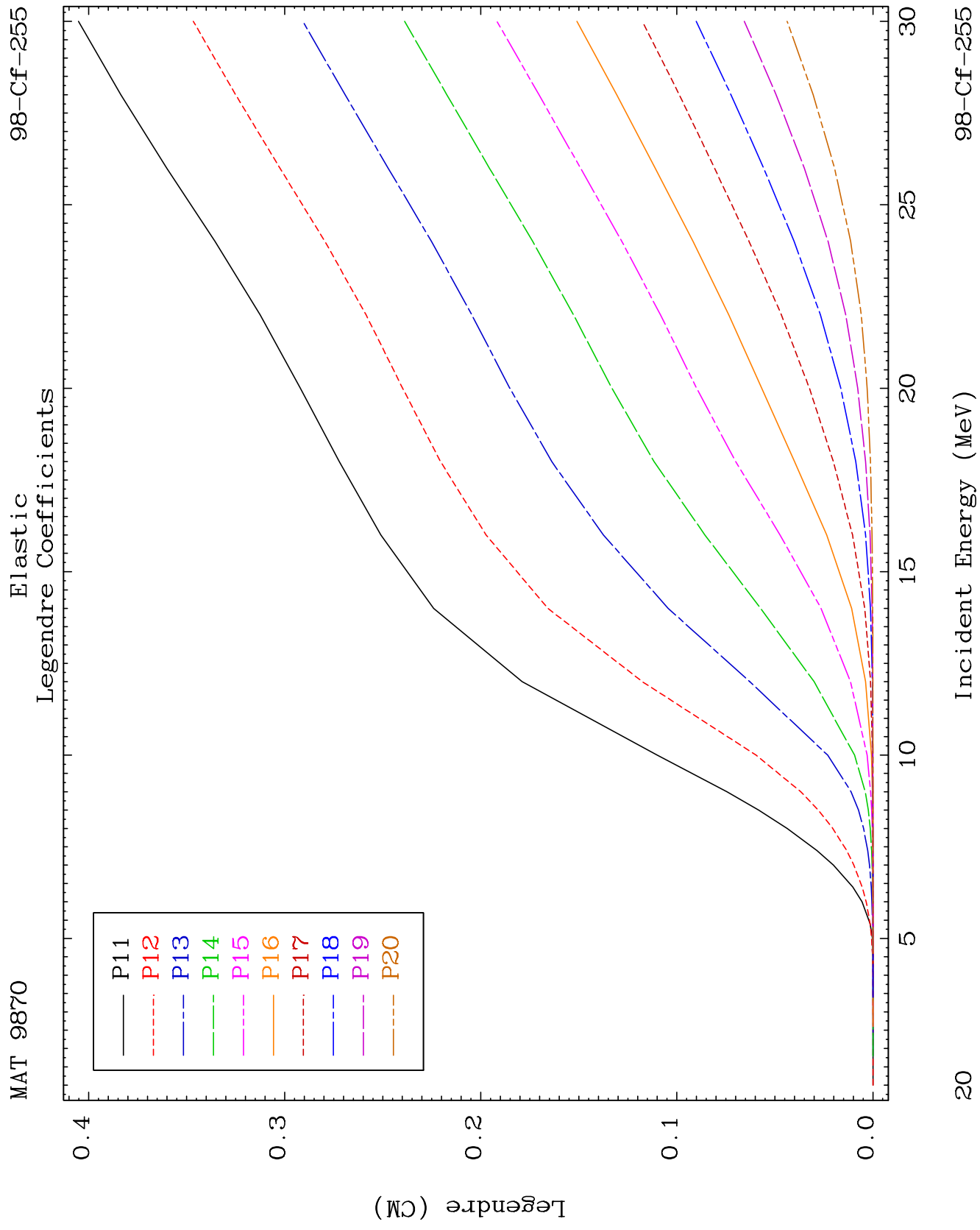
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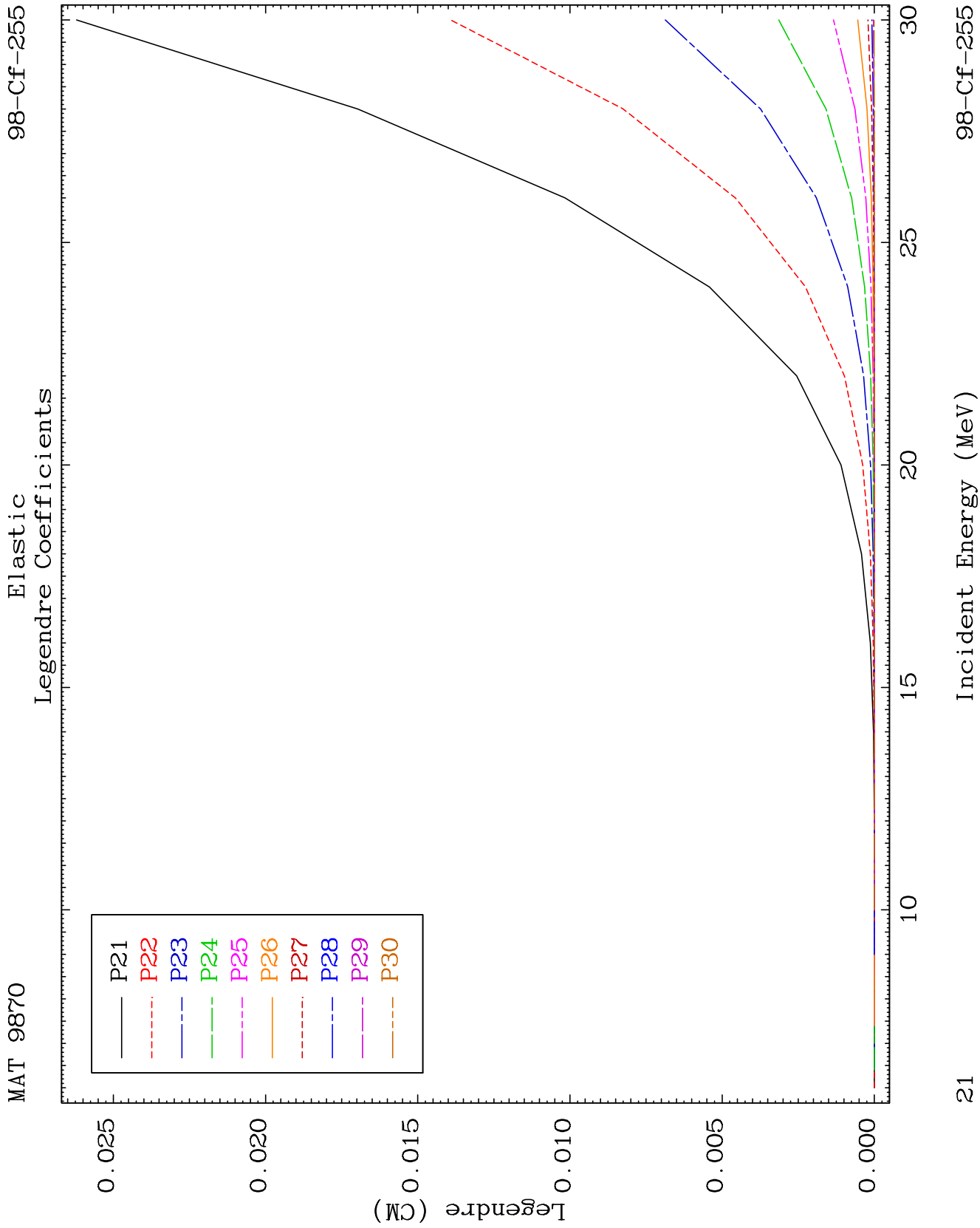
Incident Energy (MeV)

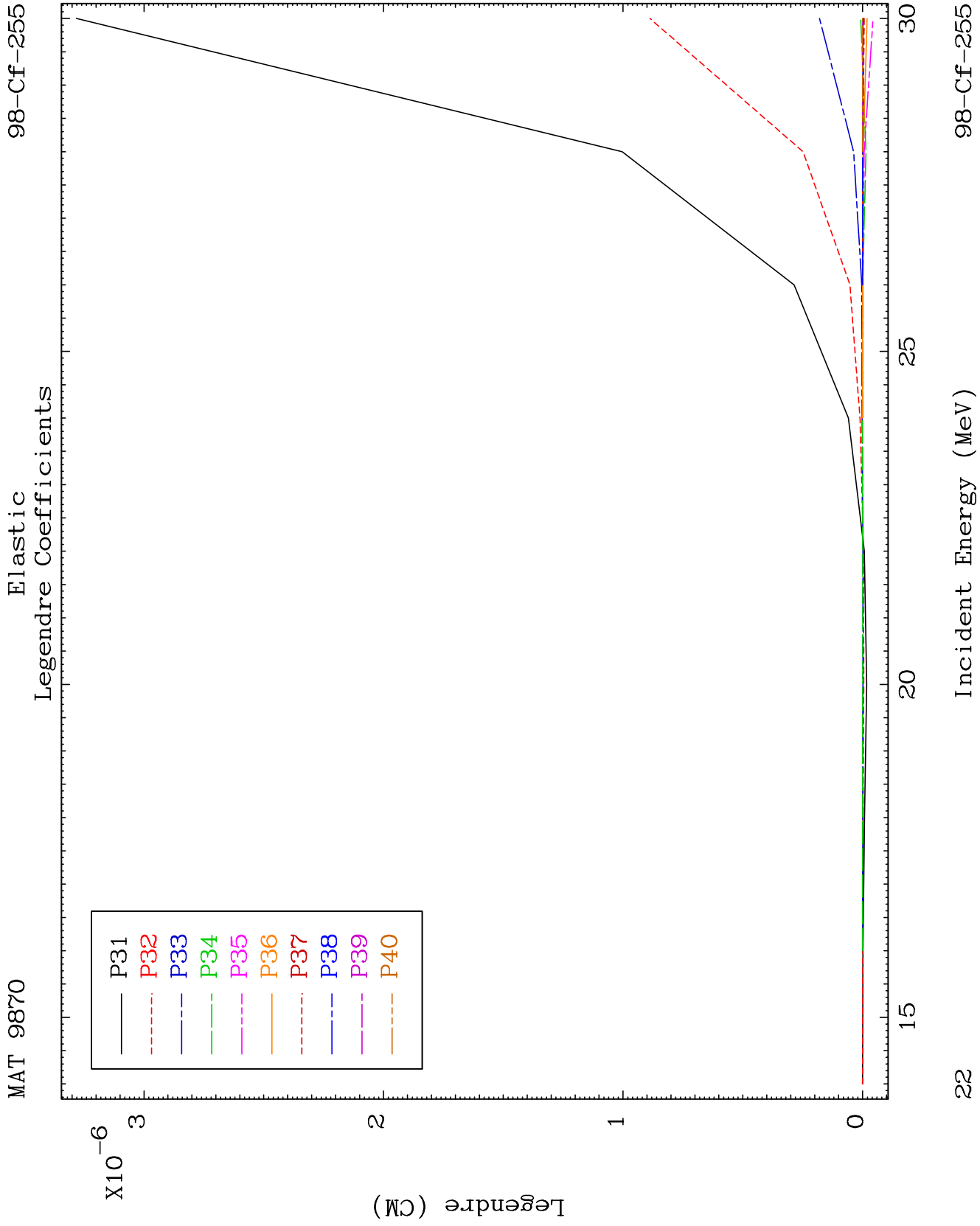
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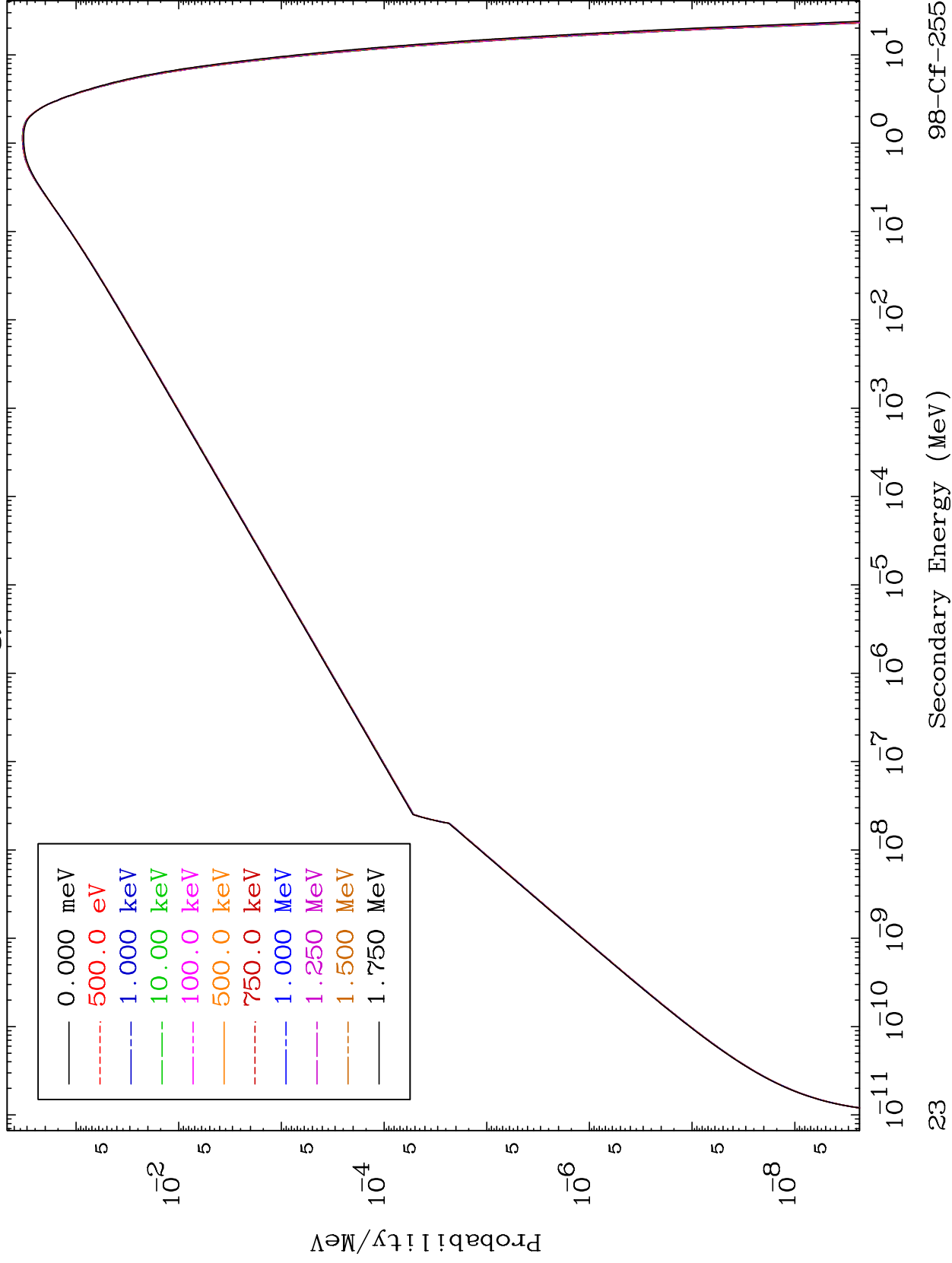


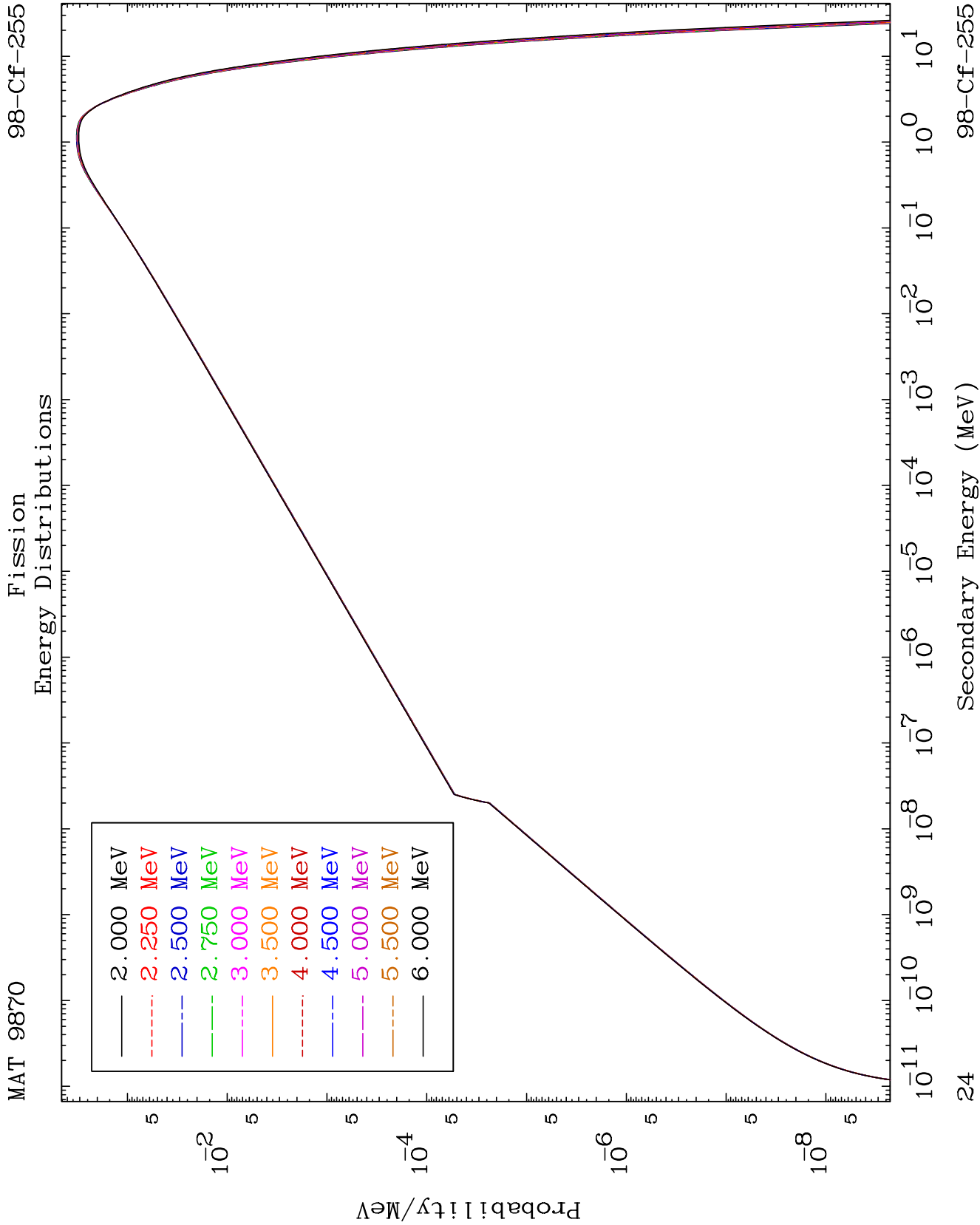


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Fission
Energy Distributions

98-Cf-255

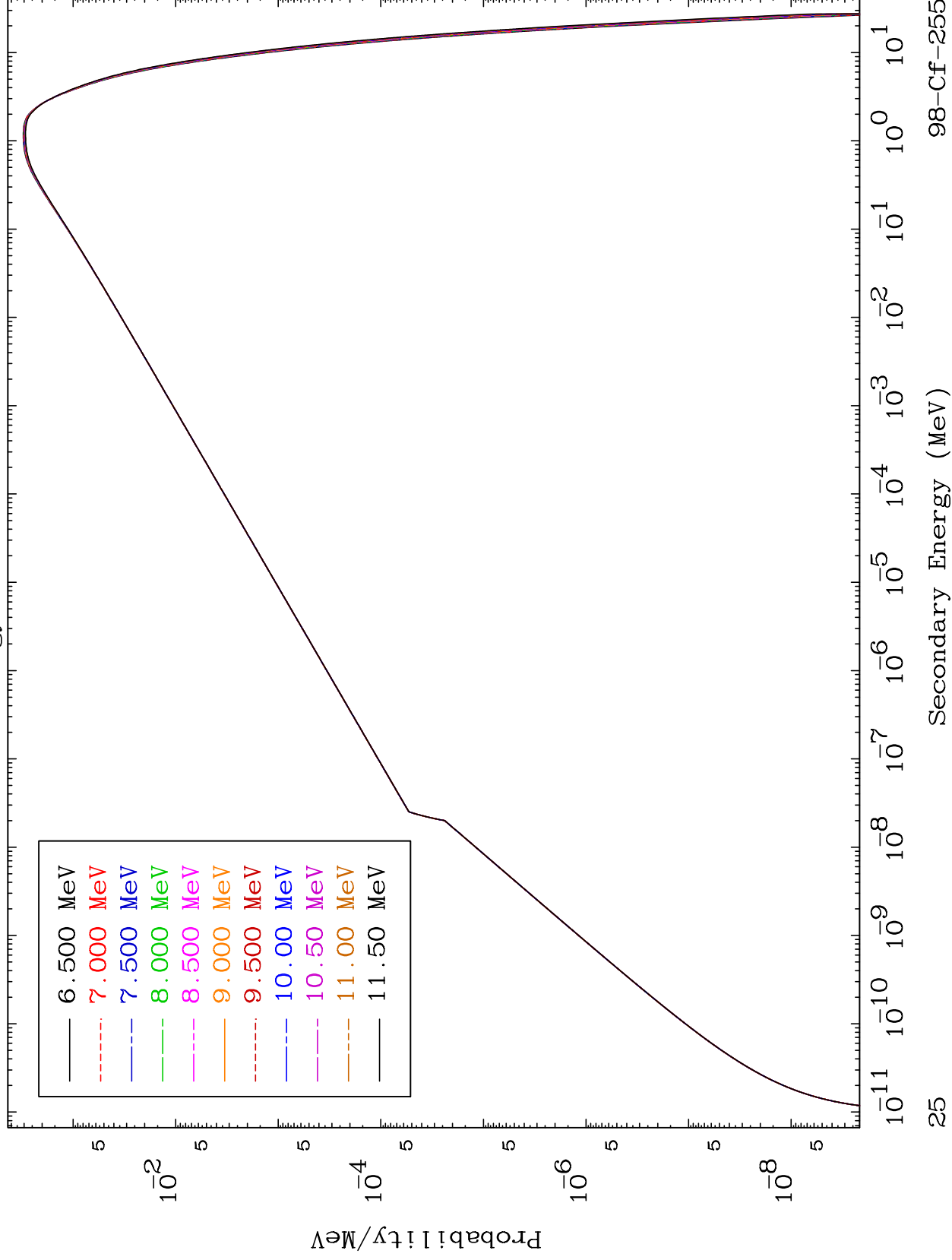




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Fission
Energy Distributions

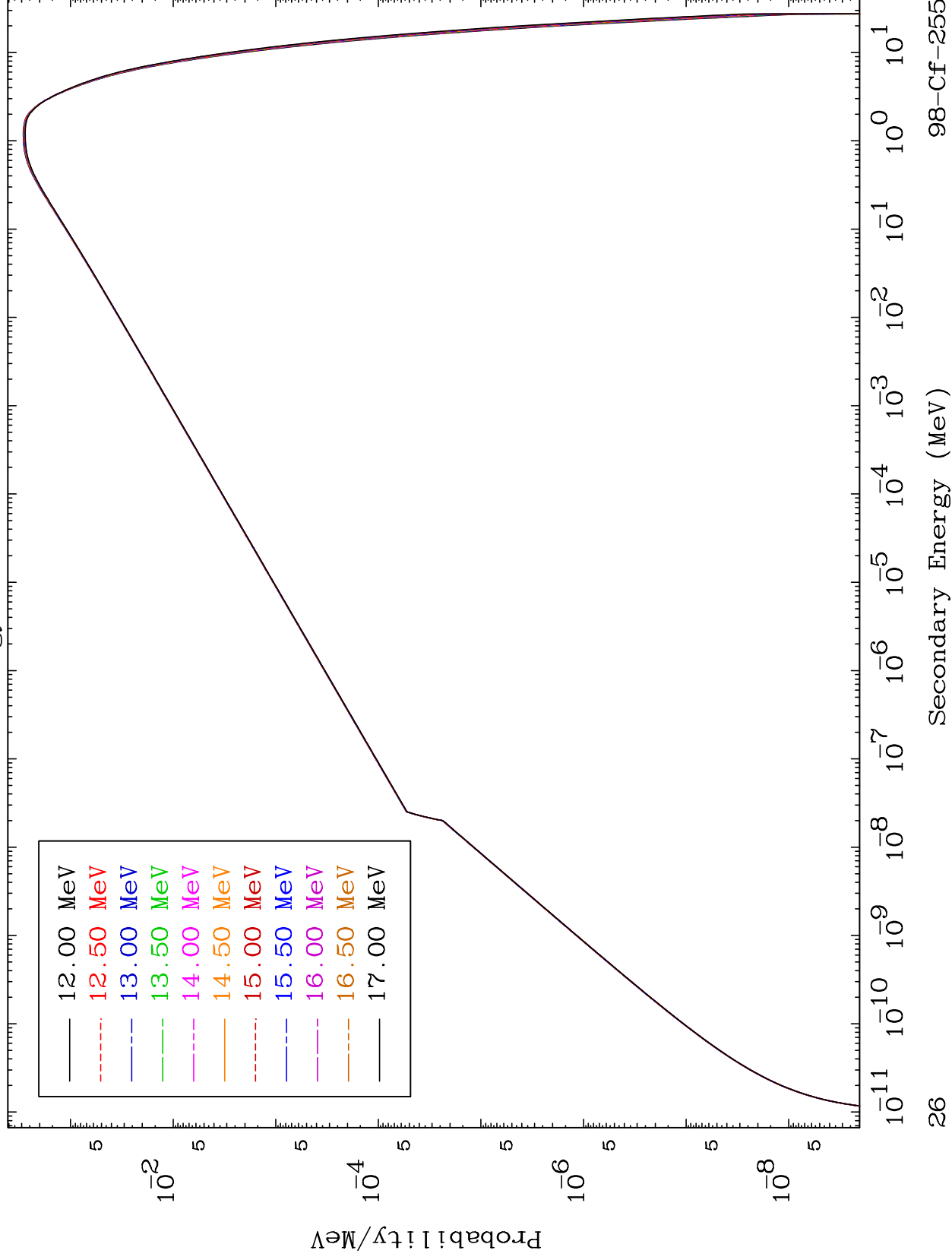
98-Cf-255



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Fission
Energy Distributions

98-Cf-255



MAT 9870

Fission
Energy Distributions

98-Cf-255

