

Program EVALPLOT
(Version 2021-1)

by

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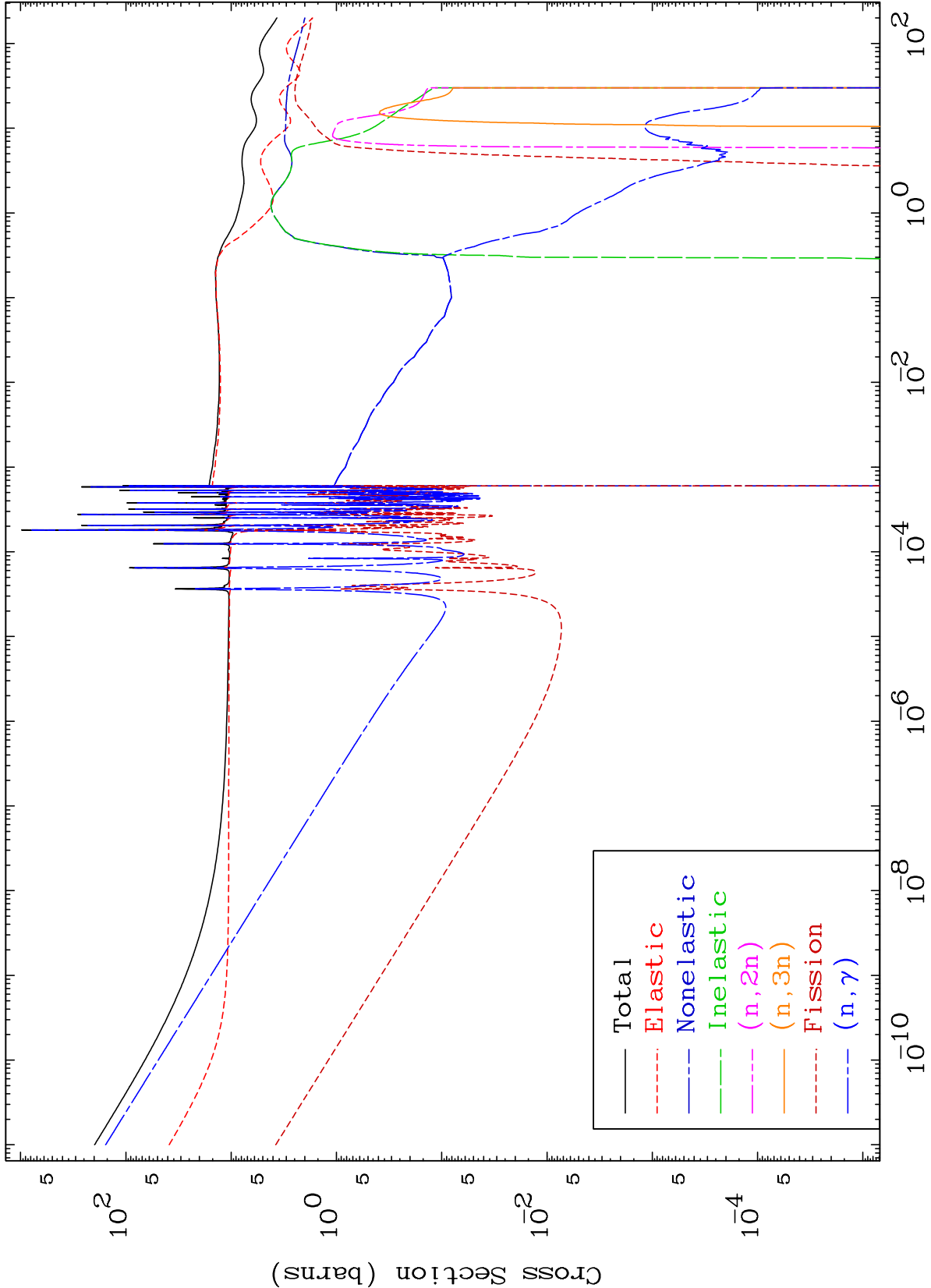
Press Mouse Button to Start

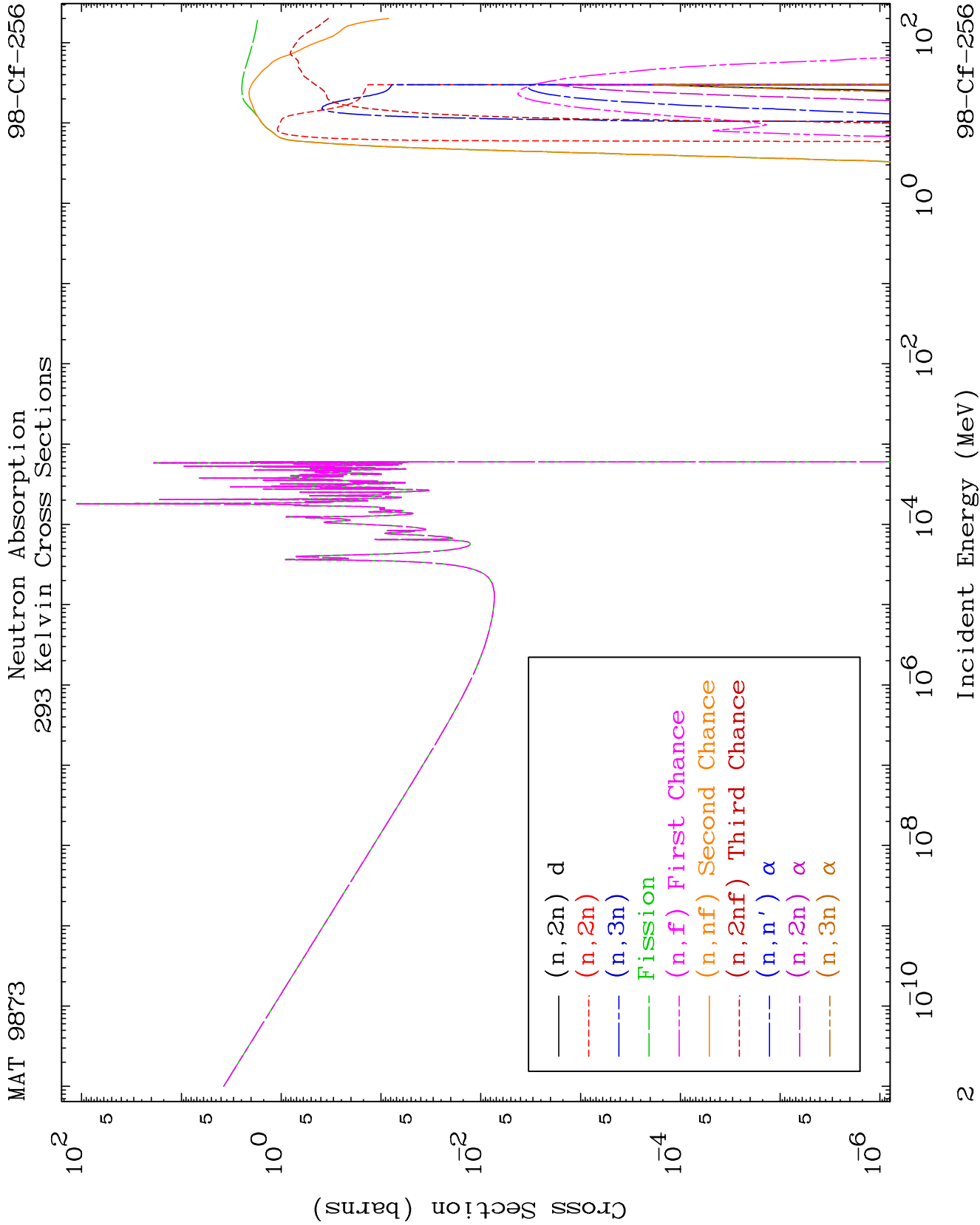
MAT 9873

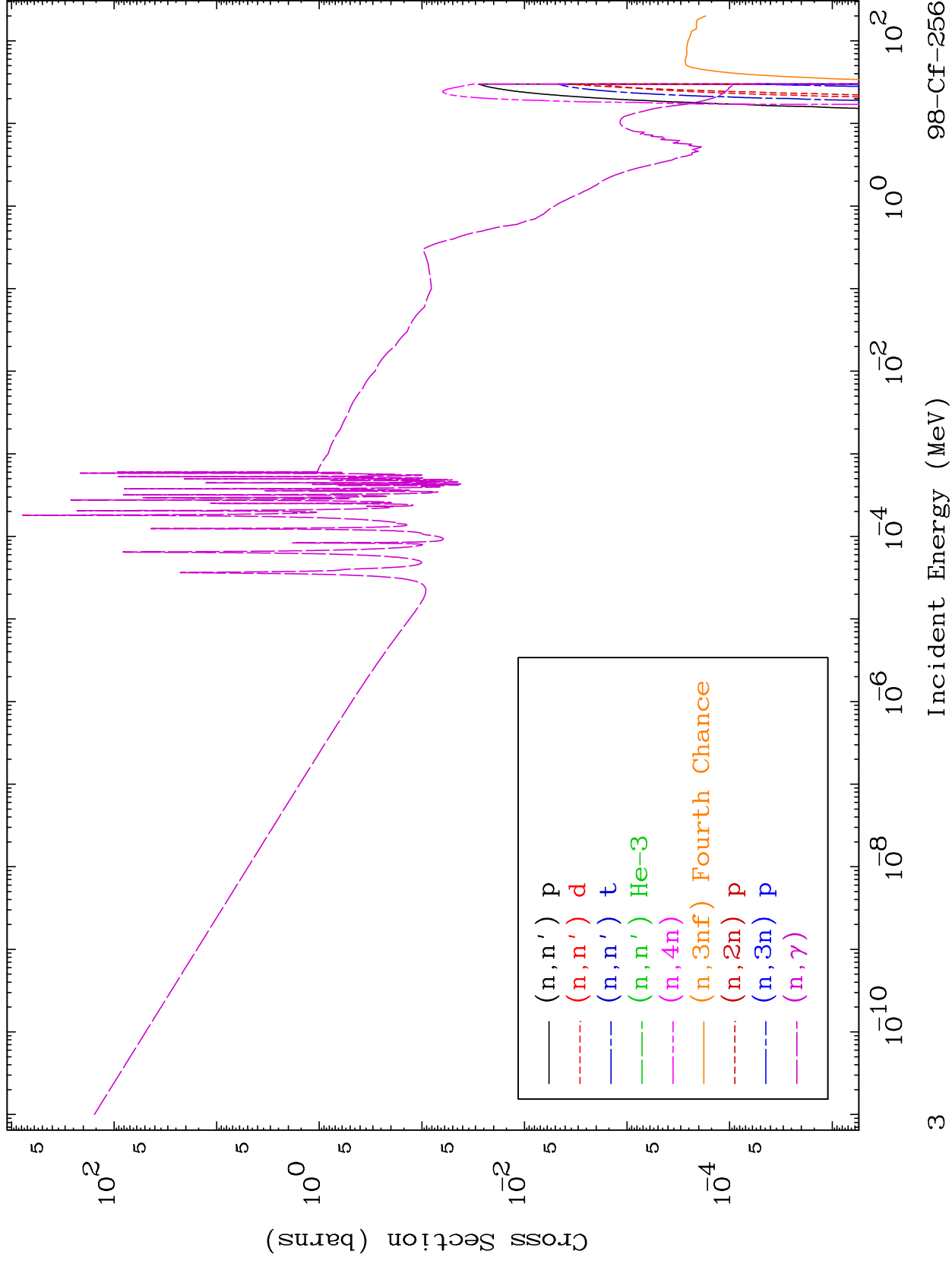
Neutron Major

293 Kelvin Cross Sections

98-Cf-256



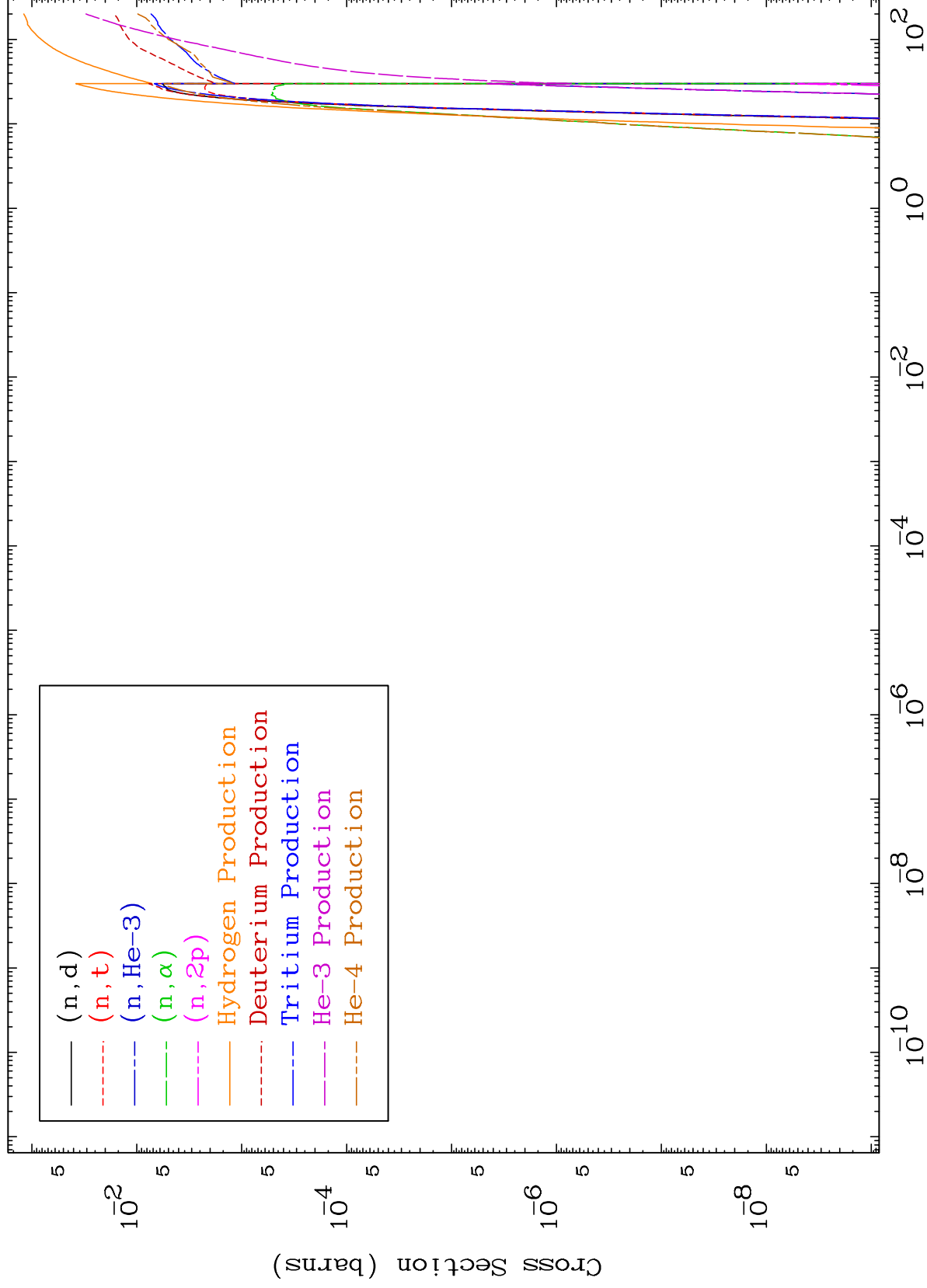




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Neutron Absorption
293 Kelvin Cross Sections

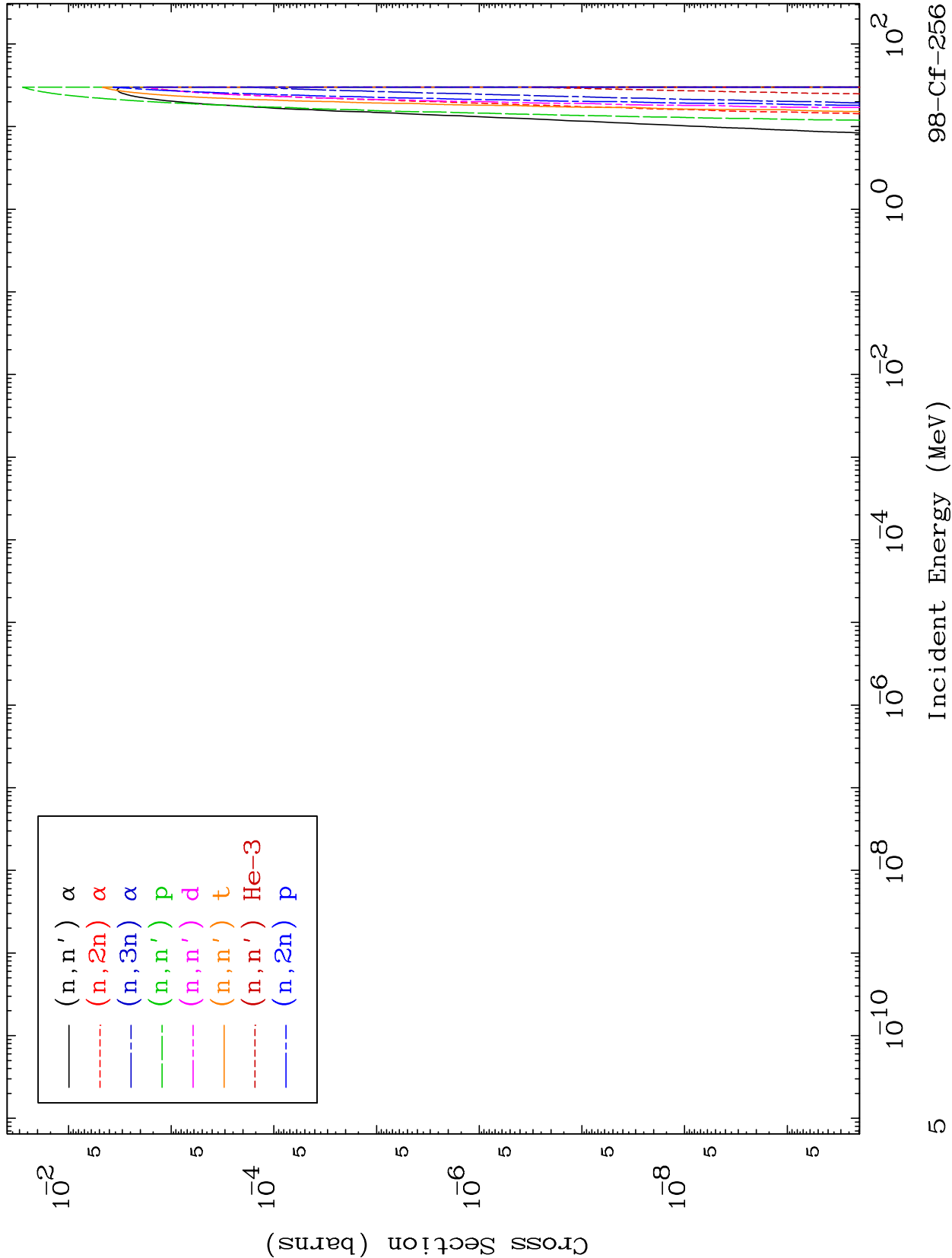
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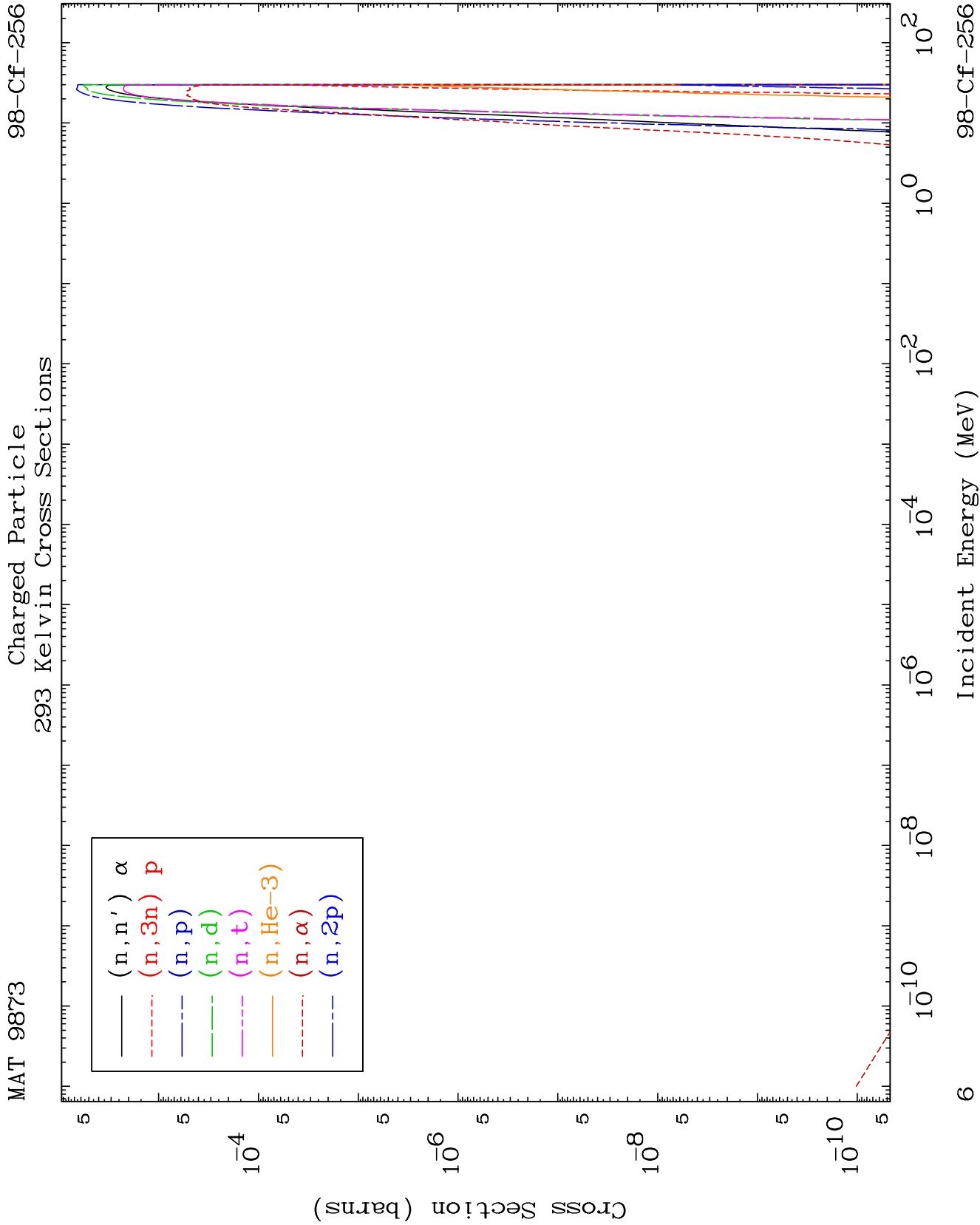


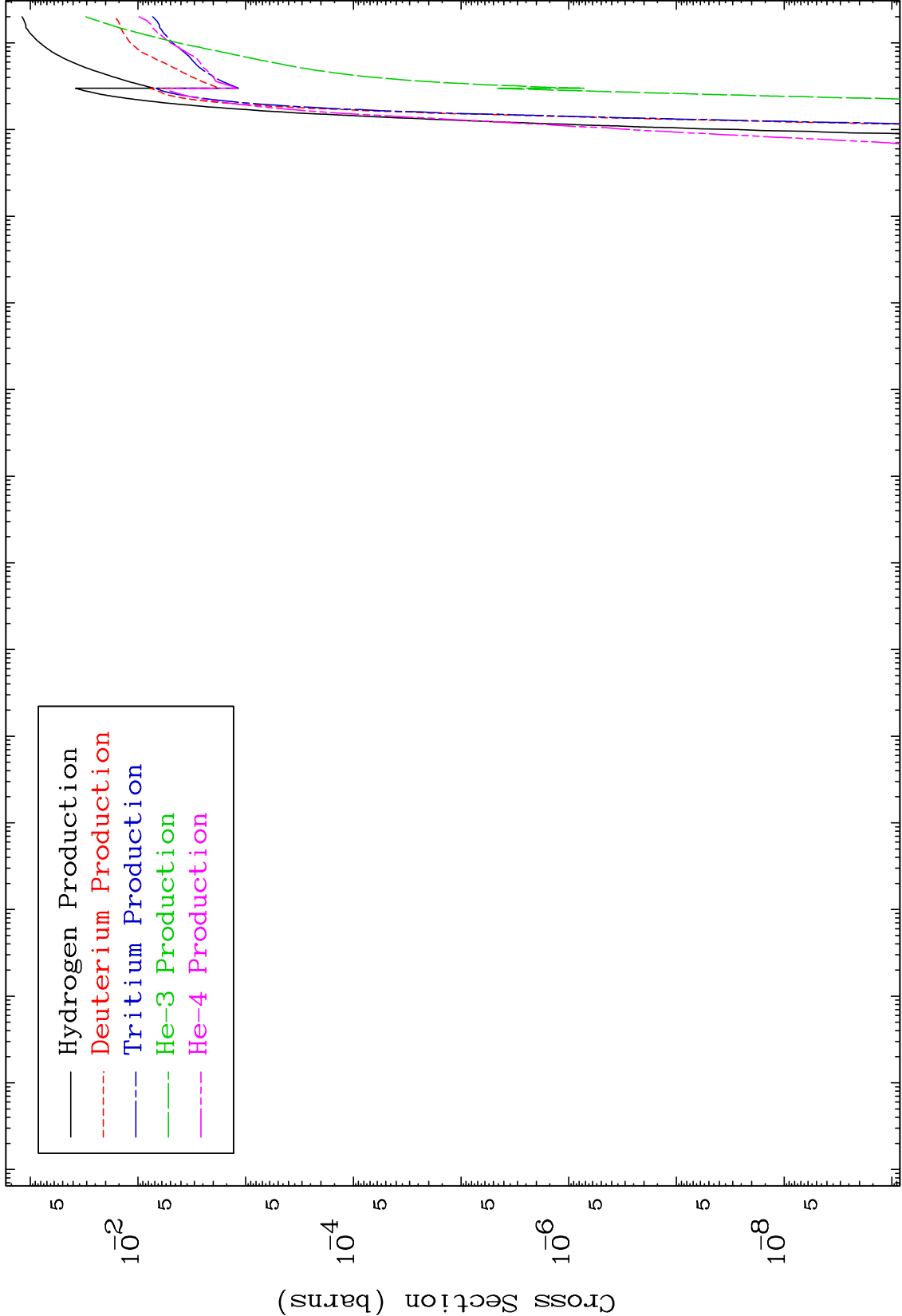
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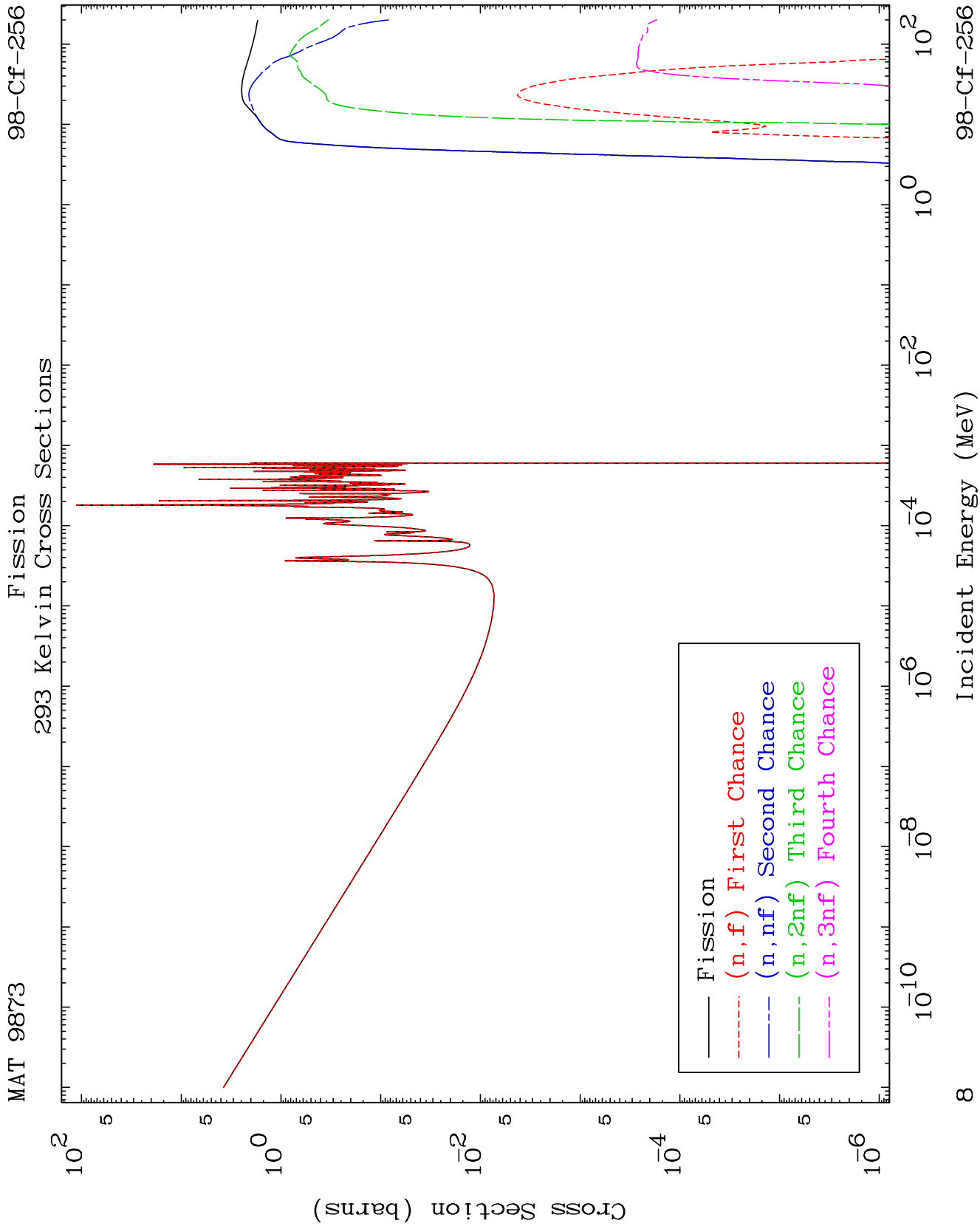
Charged Particle
293 Kelvin Cross Sections

98-Cf-256

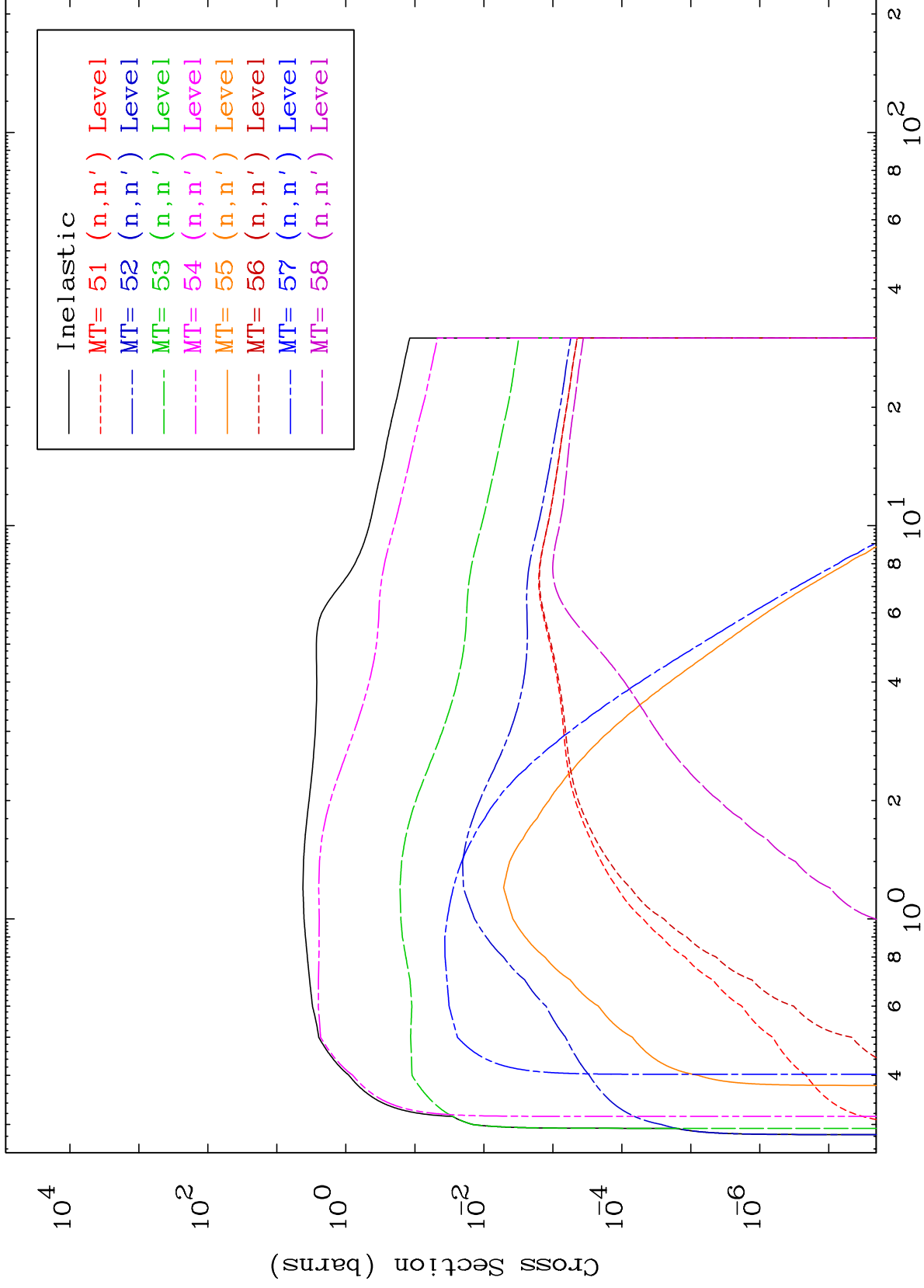


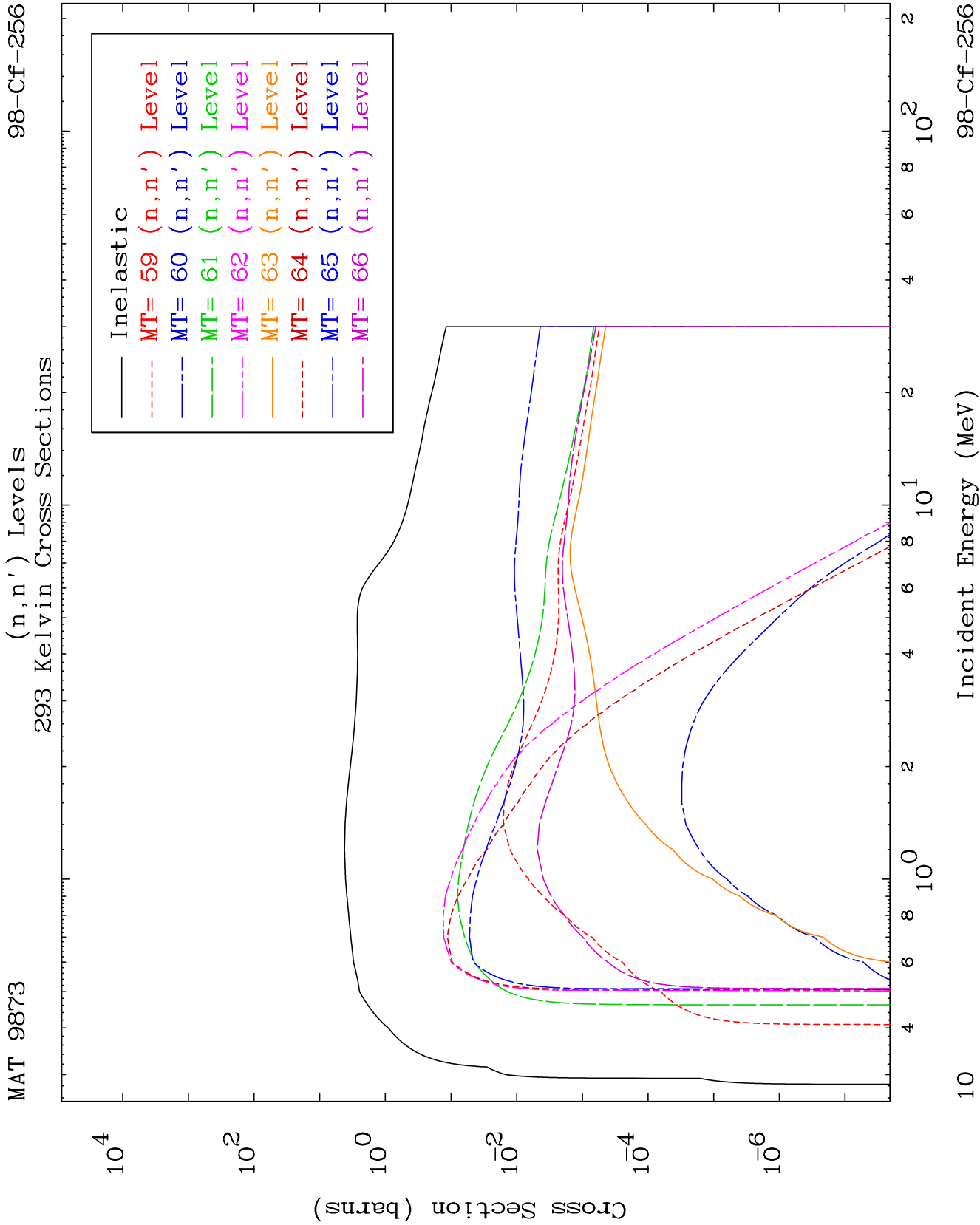


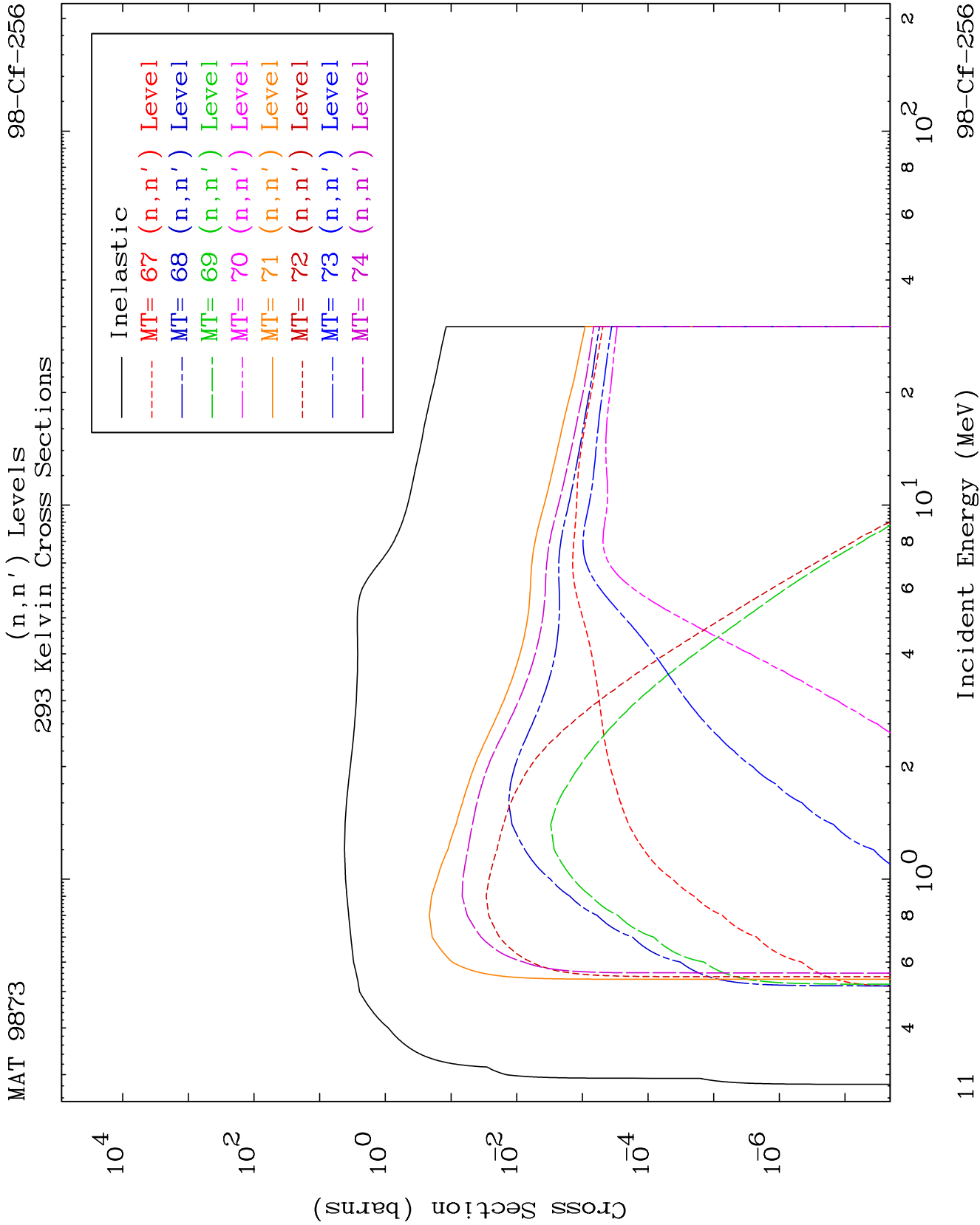


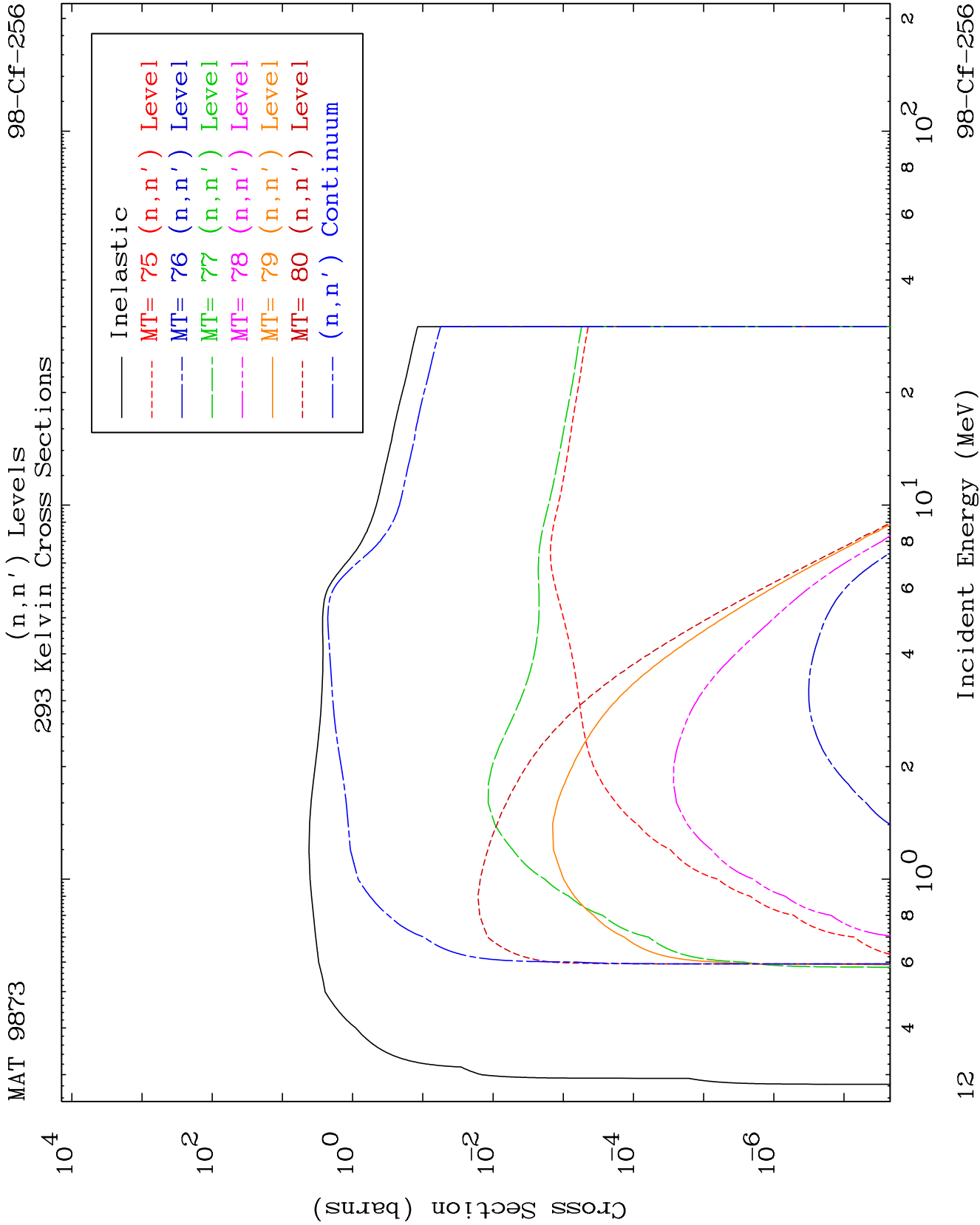


293 Kelvin Cross Sections





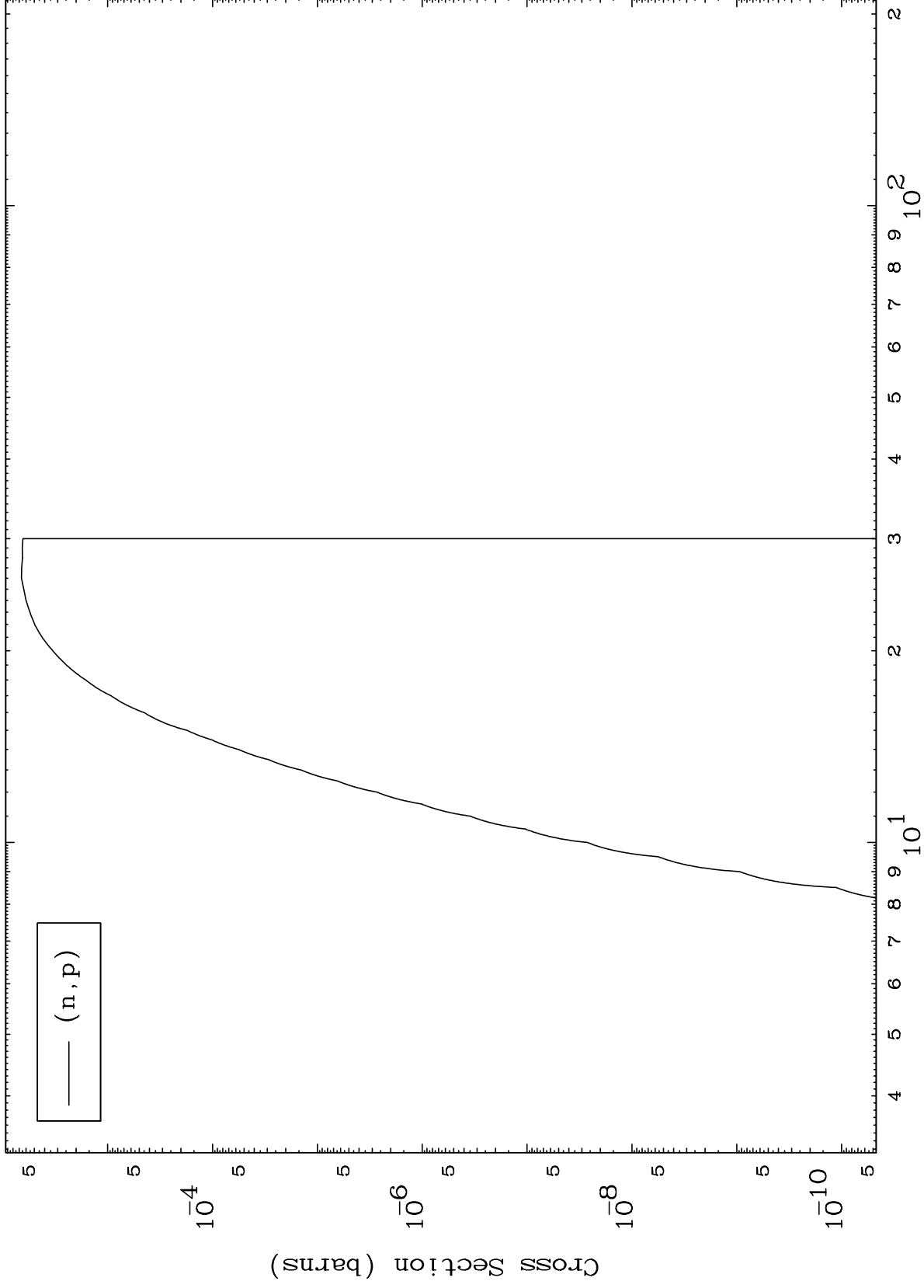




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(n,p) Levels
293 Kelvin Cross Sections

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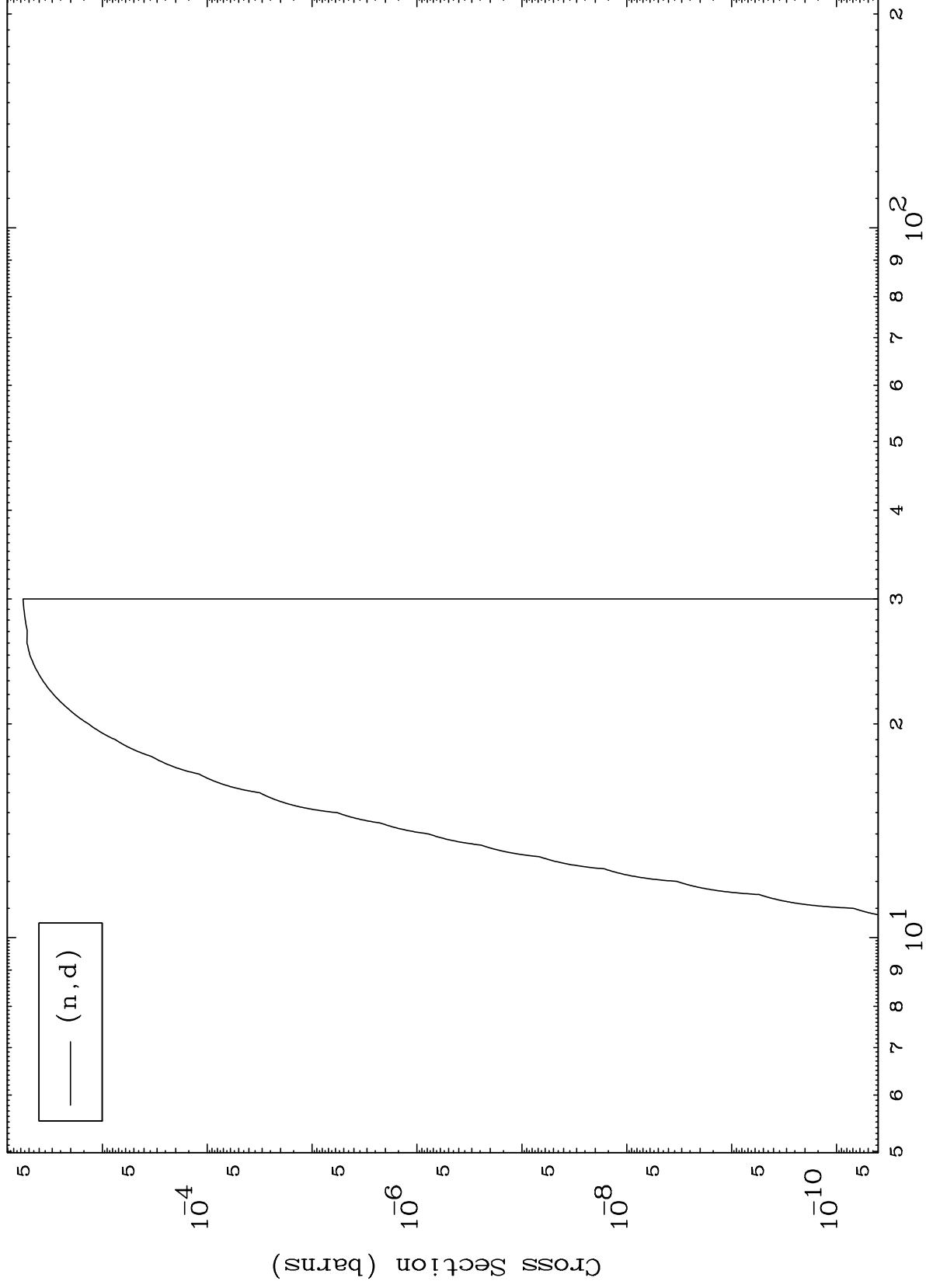
Incident Energy (MeV)

98-Cf-256

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98-Cf-256

(n,d) Levels
293 Kelvin Cross Sections



14

Incident Energy (MeV)

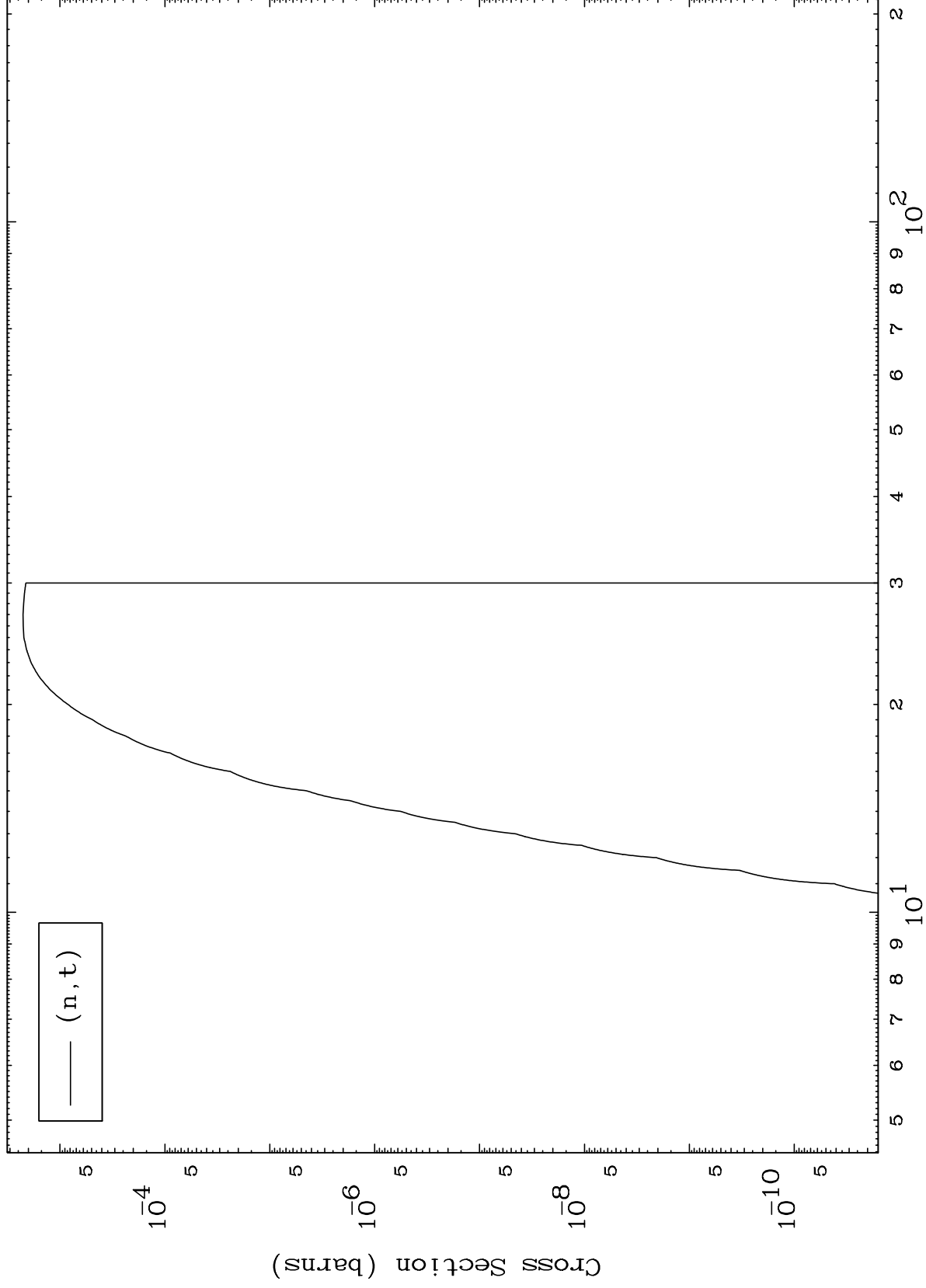
98-Cf-256

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(n,t) Levels

98-Cf-256

293 Kelvin Cross Sections



15

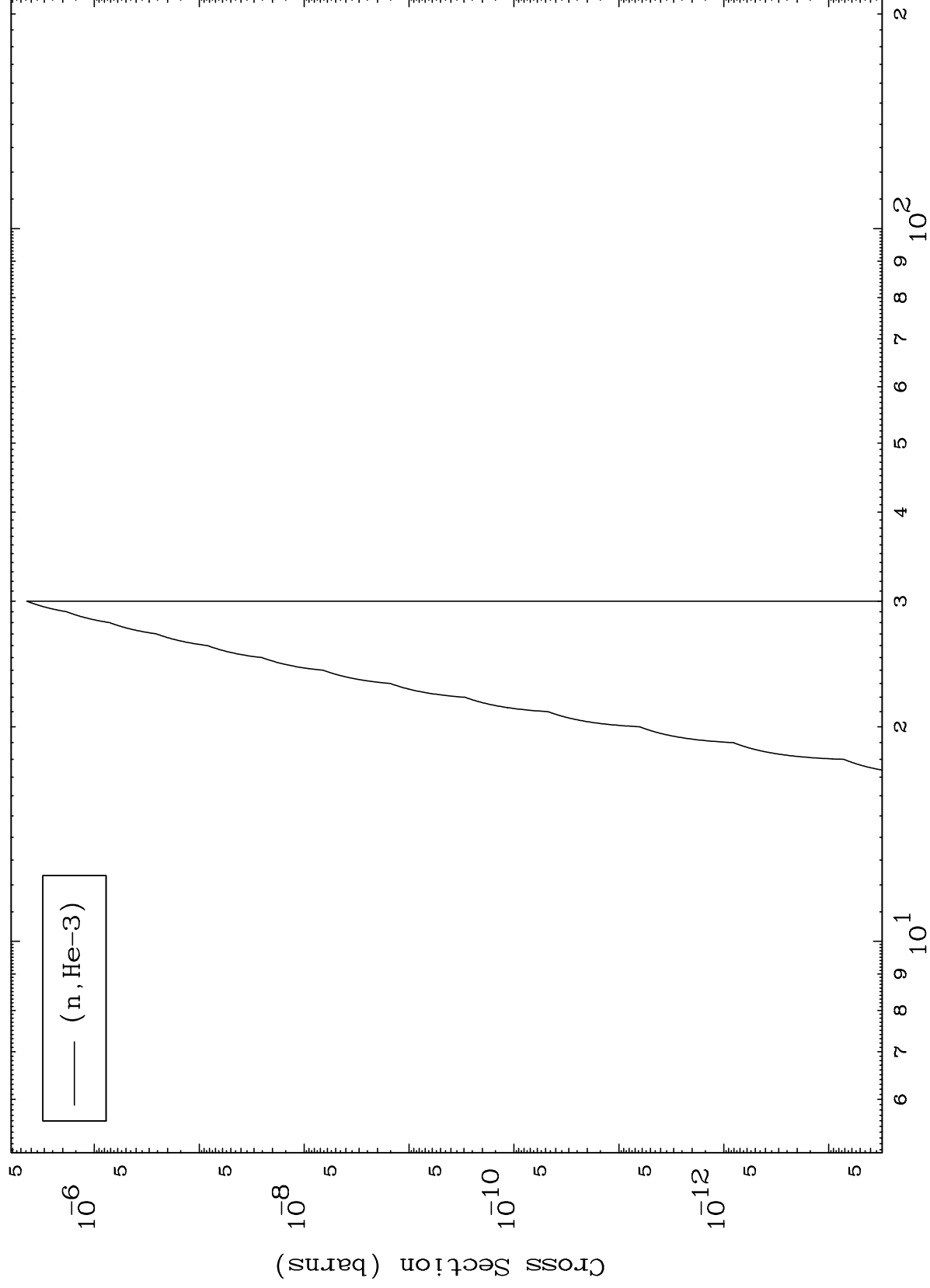
Incident Energy (MeV)

98-Cf-256

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98-Cf-256

(n,He3) Levels
293 Kelvin Cross Sections



98-Cf-256

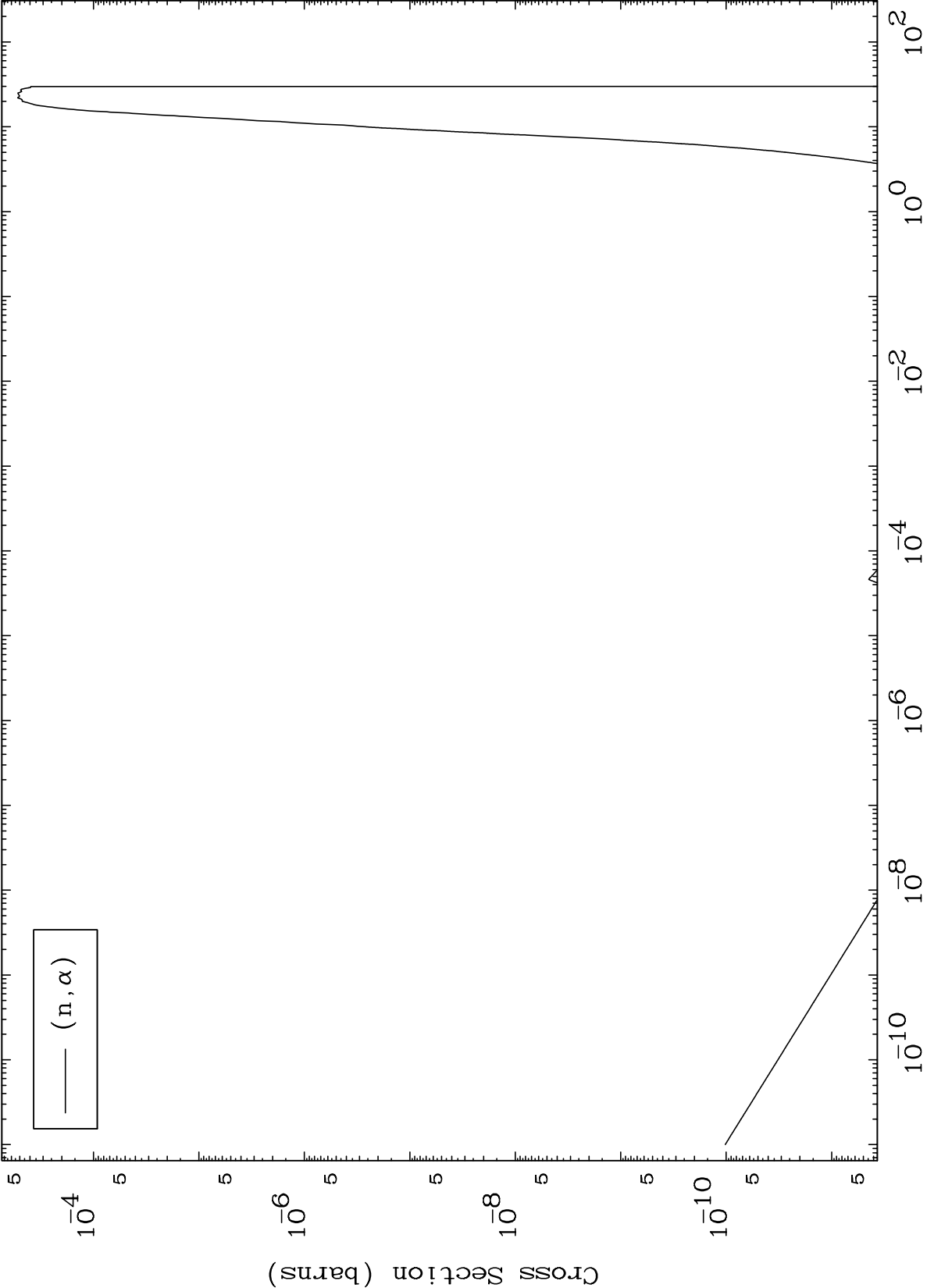
Incident Energy (MeV)

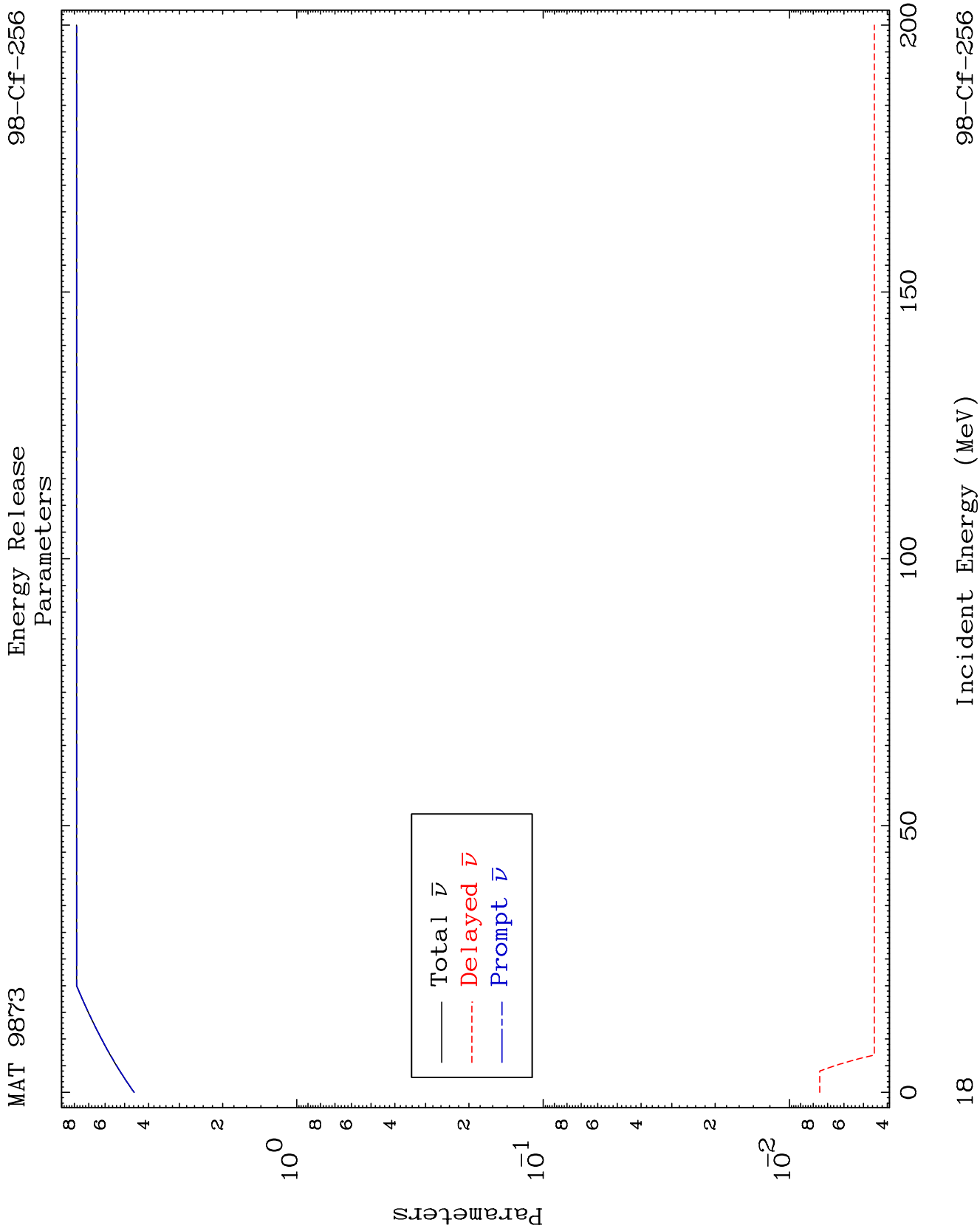
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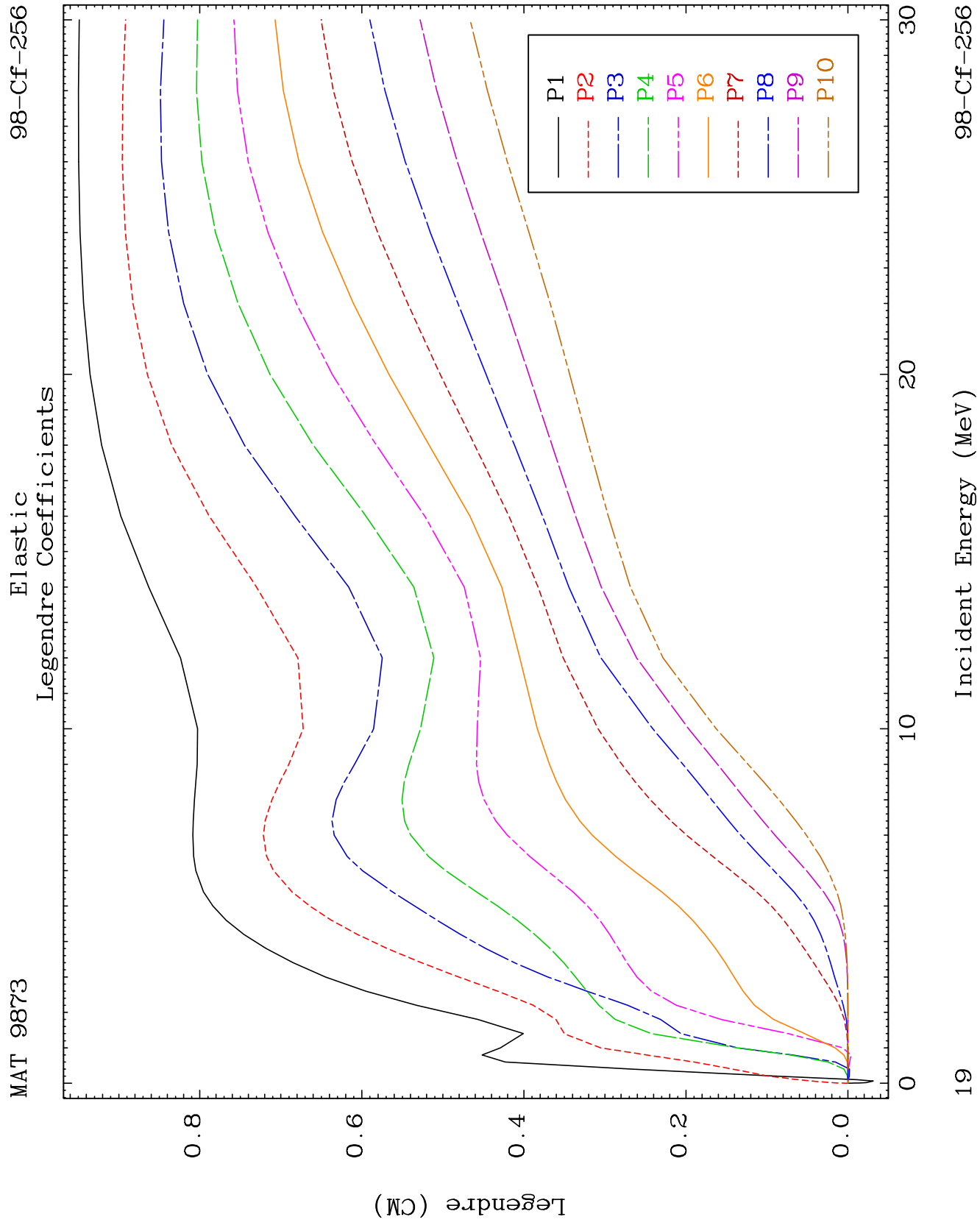
MAT 9873

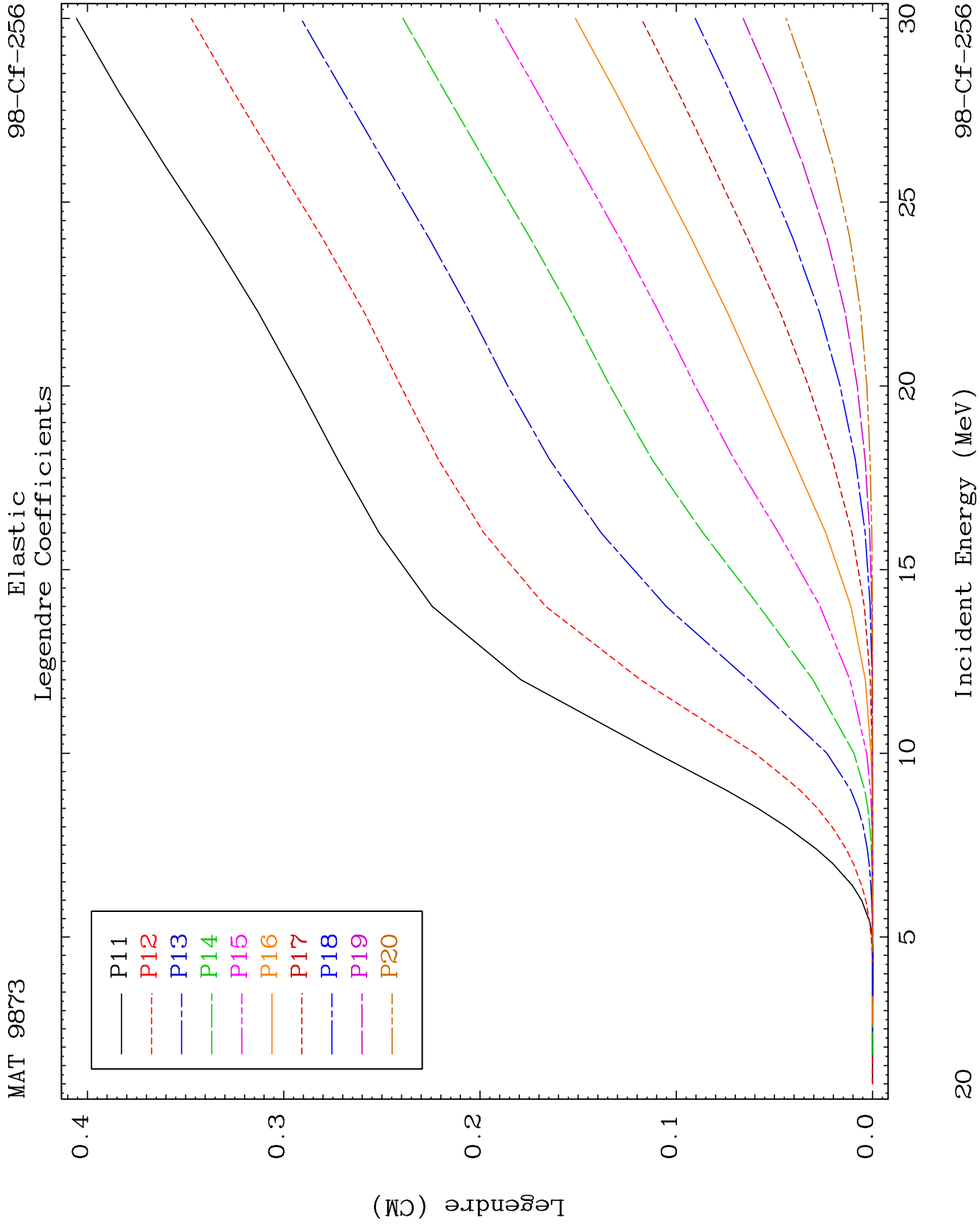
(n, α) Levels
293 Kelvin Cross Sections

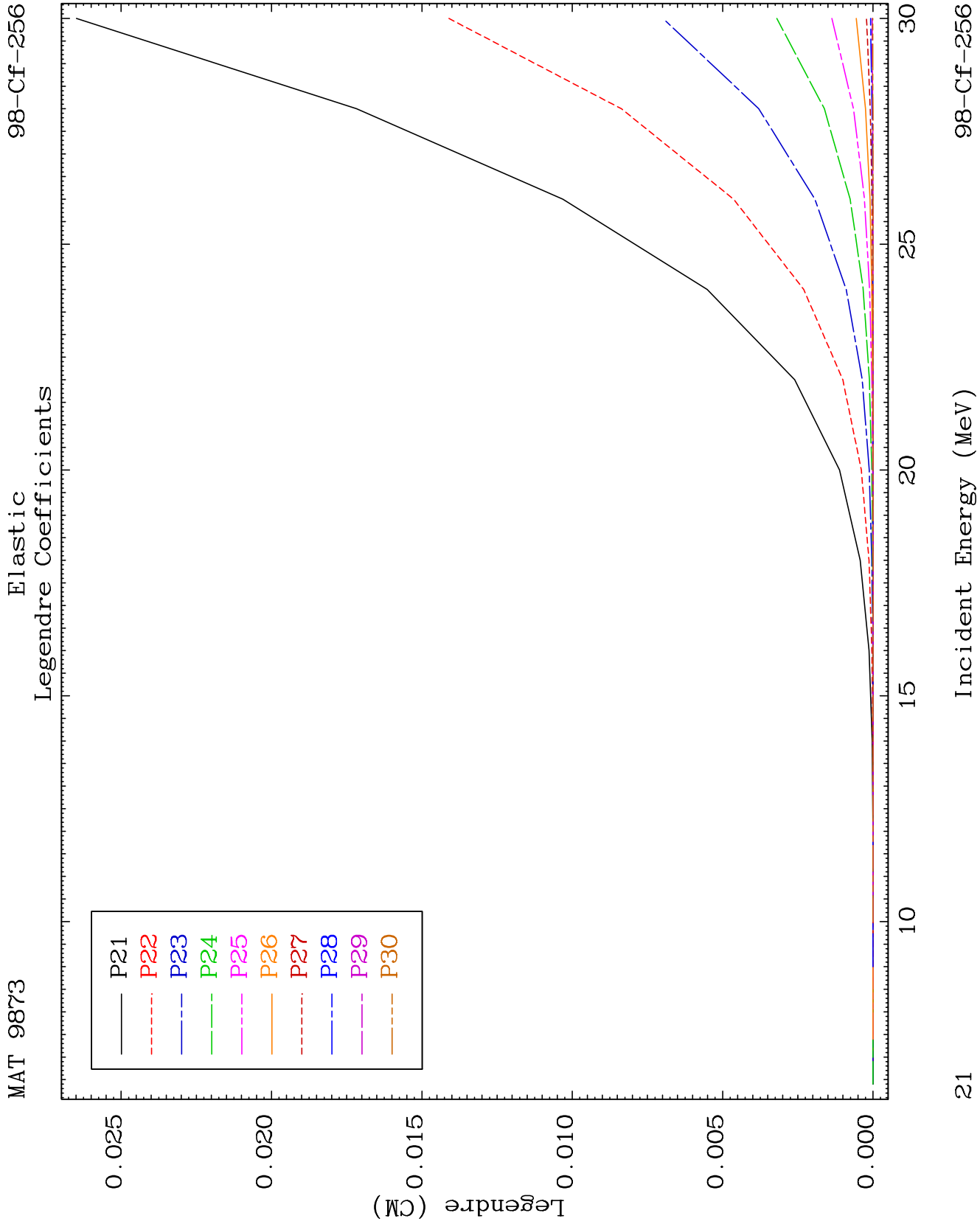
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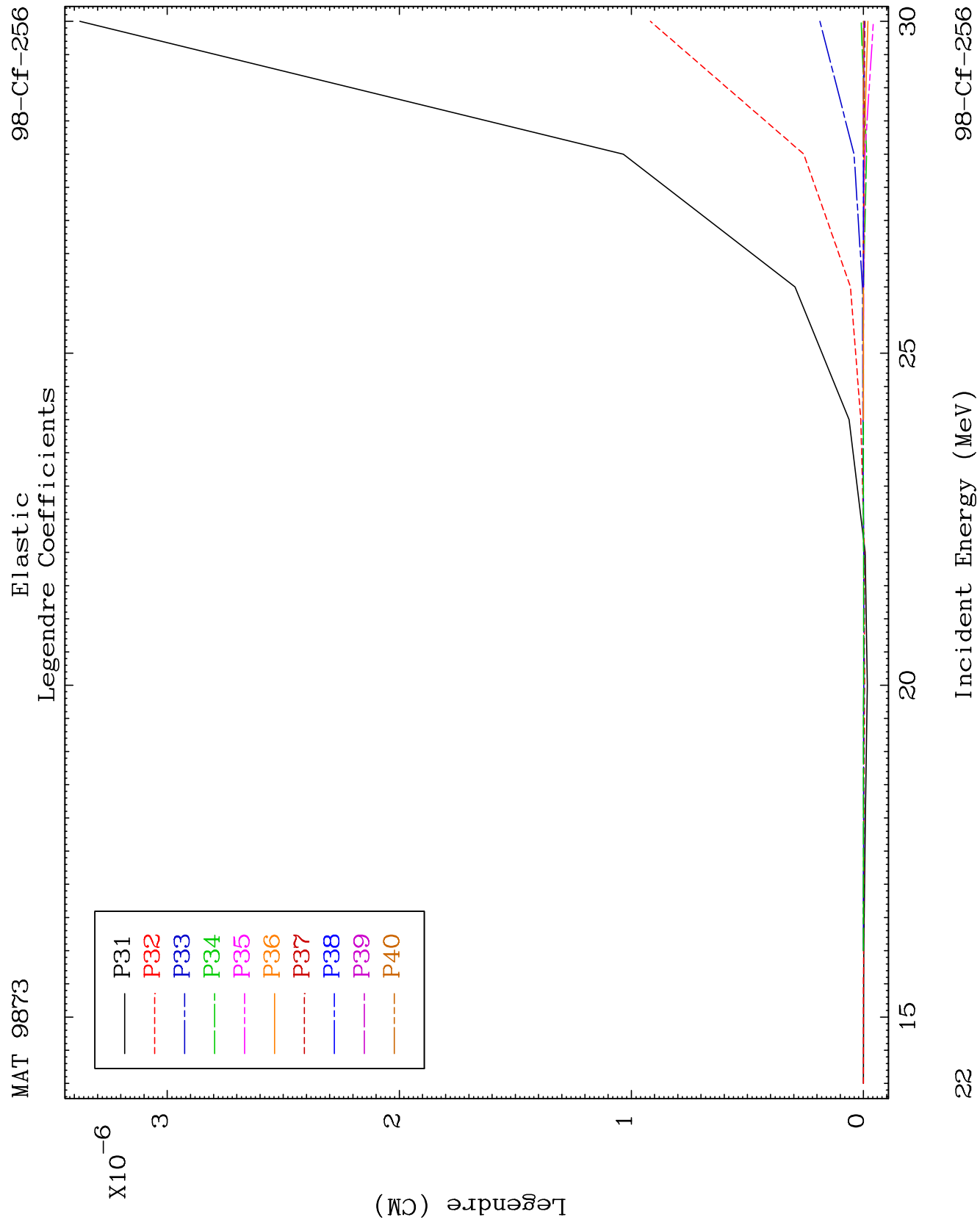








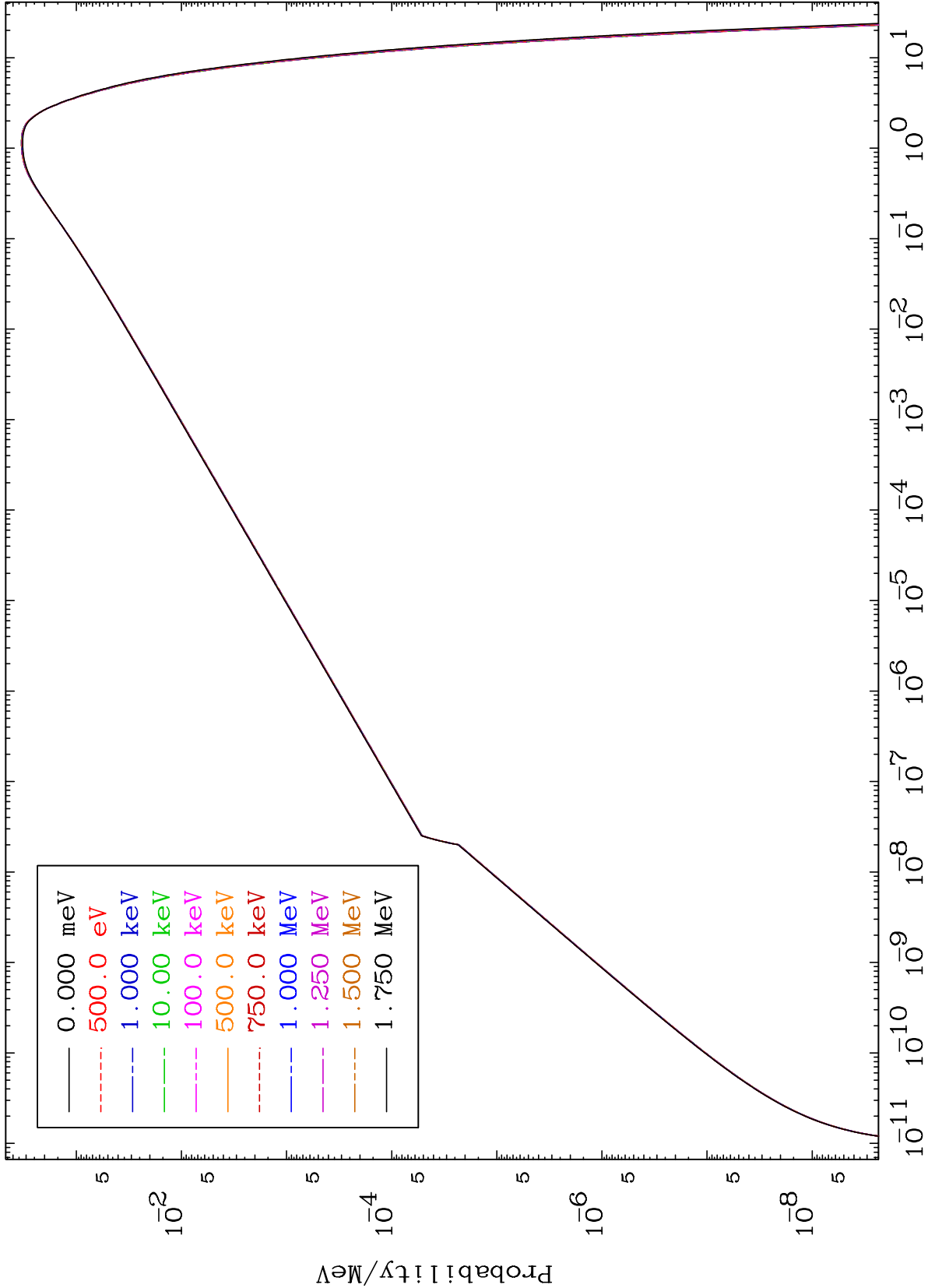




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Fission
Energy Distributions

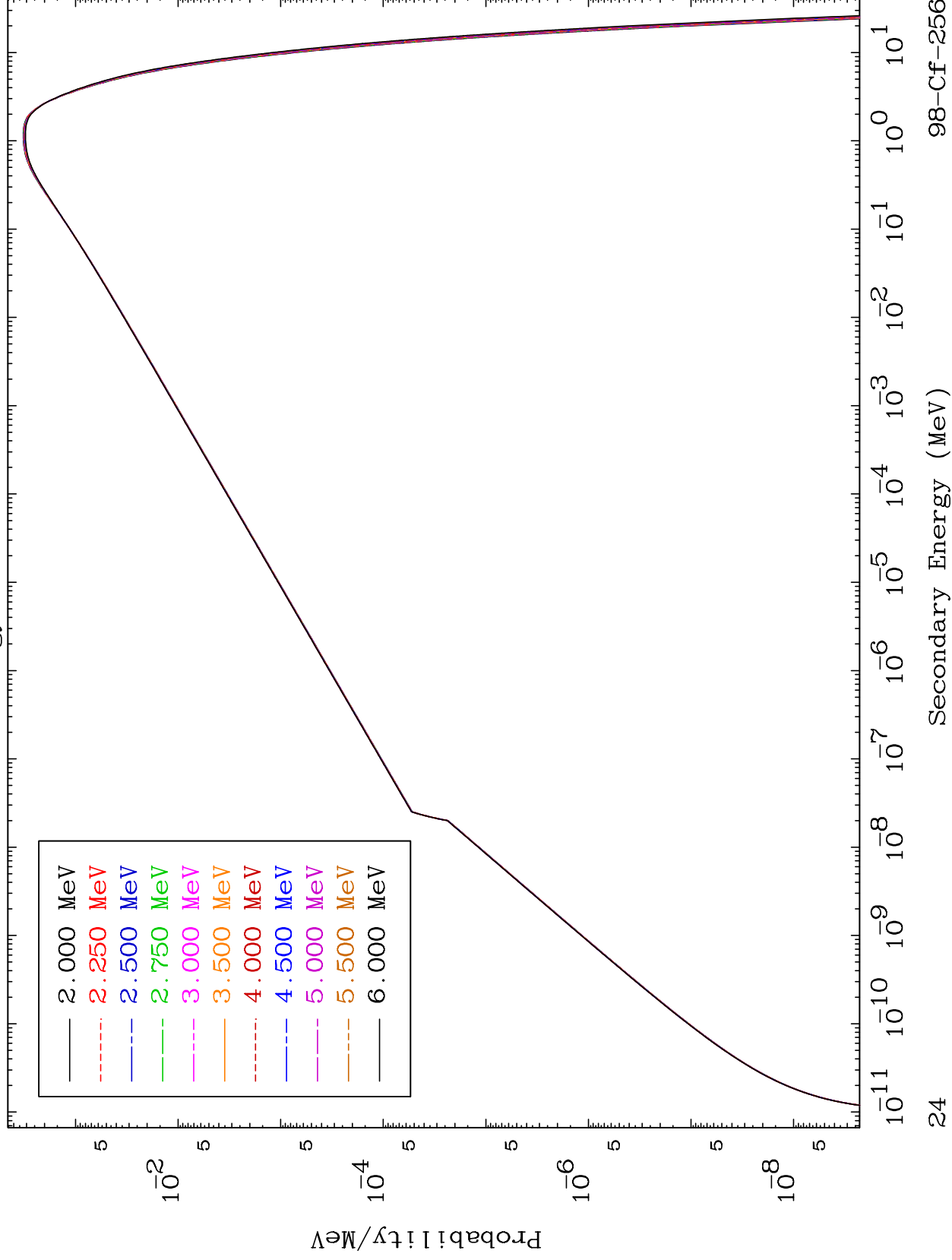
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MAT 9873

Fission
Energy Distributions

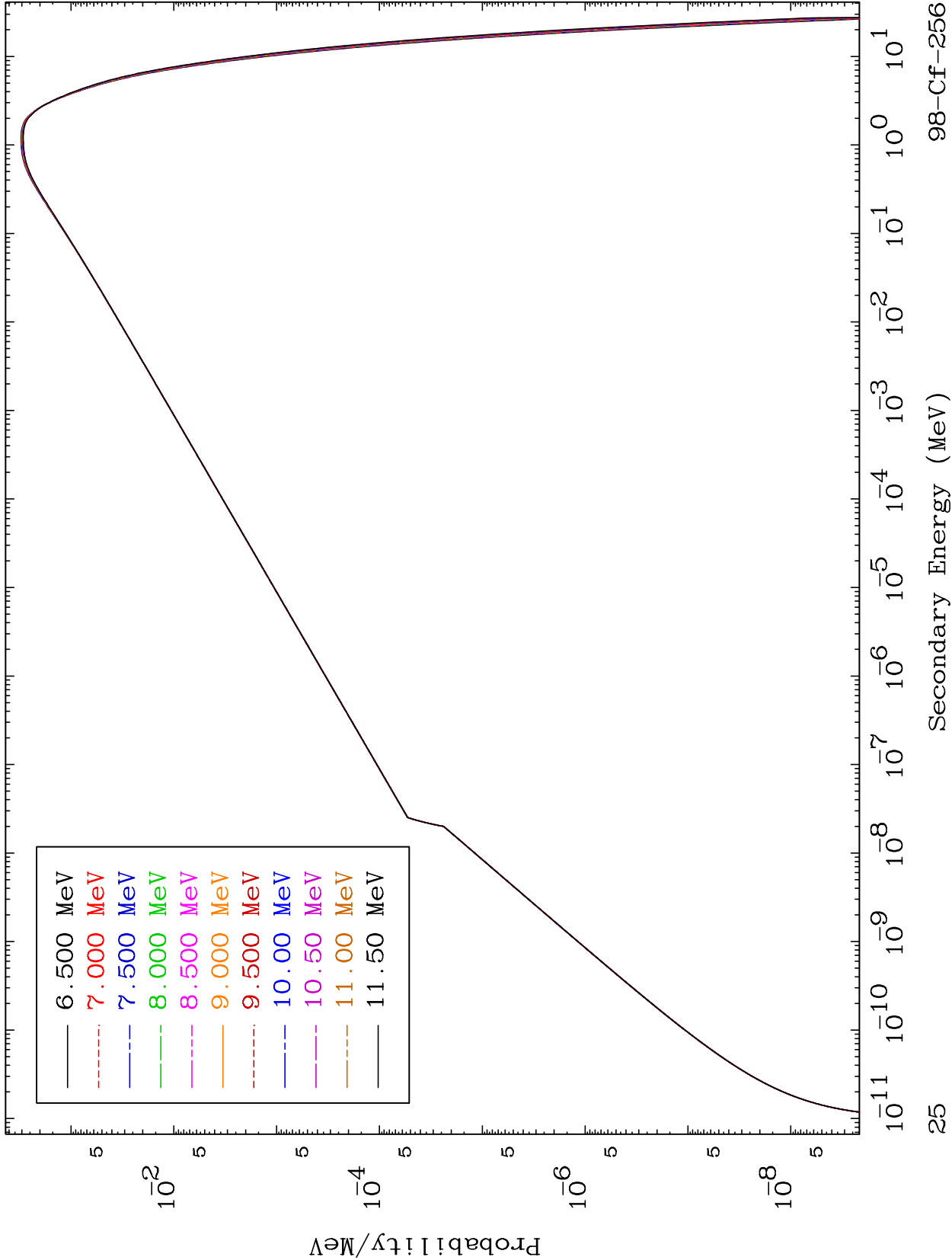
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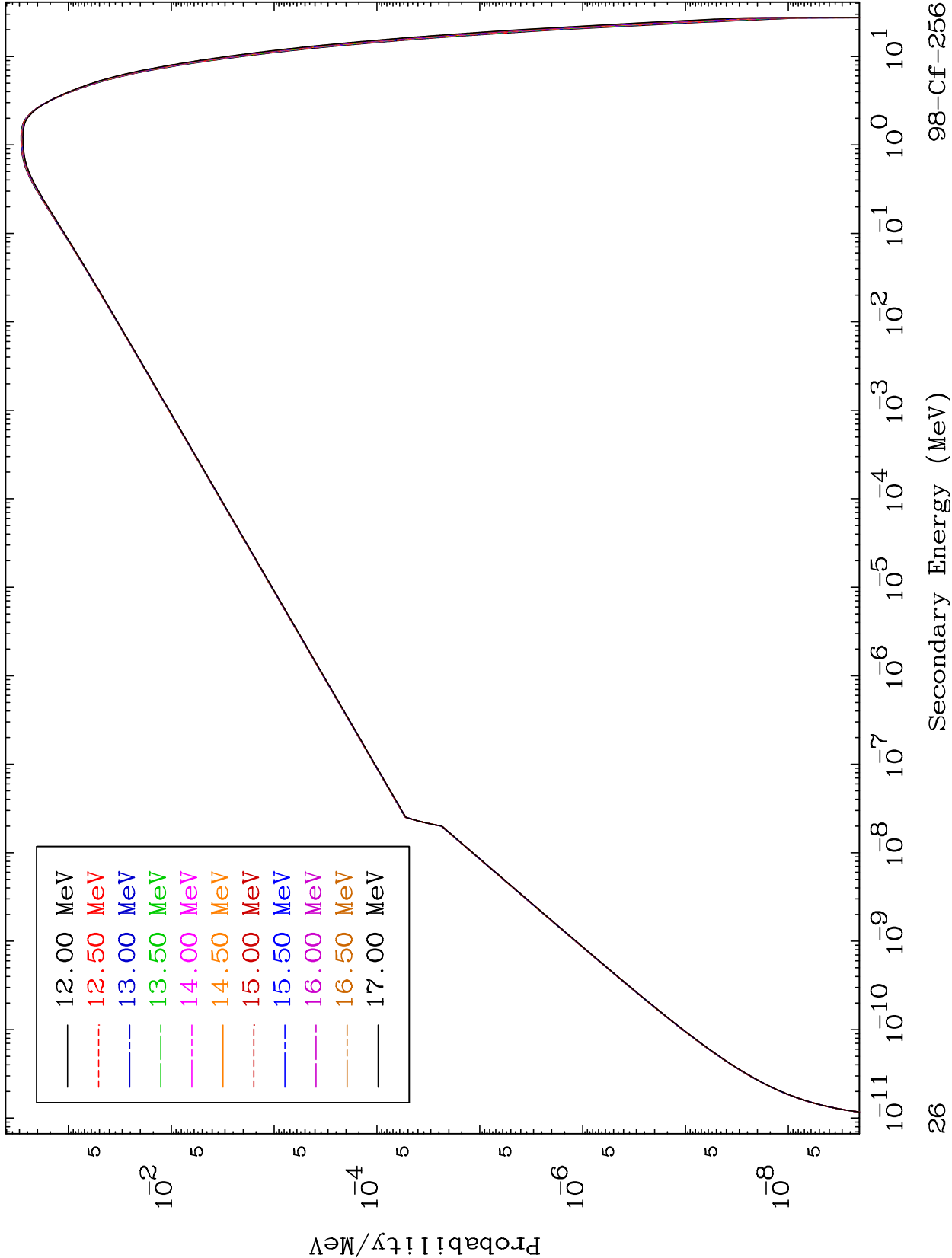


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Fission
Energy Distributions

98-Cf-256





MAT 9873

Fission
Energy Distributions

98-Cf-256

