

Program EVALPLOT
(Version 2021-1)

by

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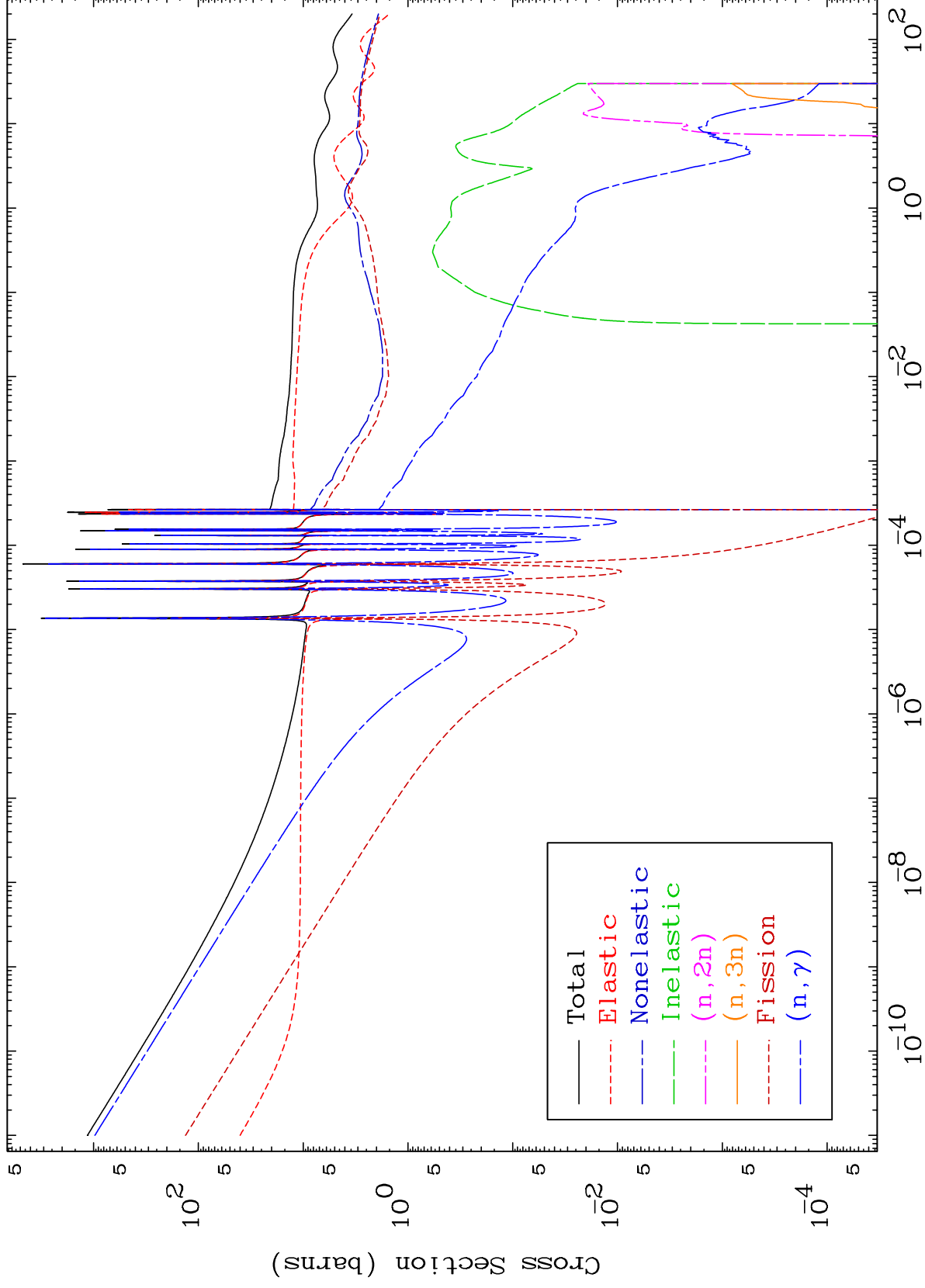
Press Mouse Button to Start

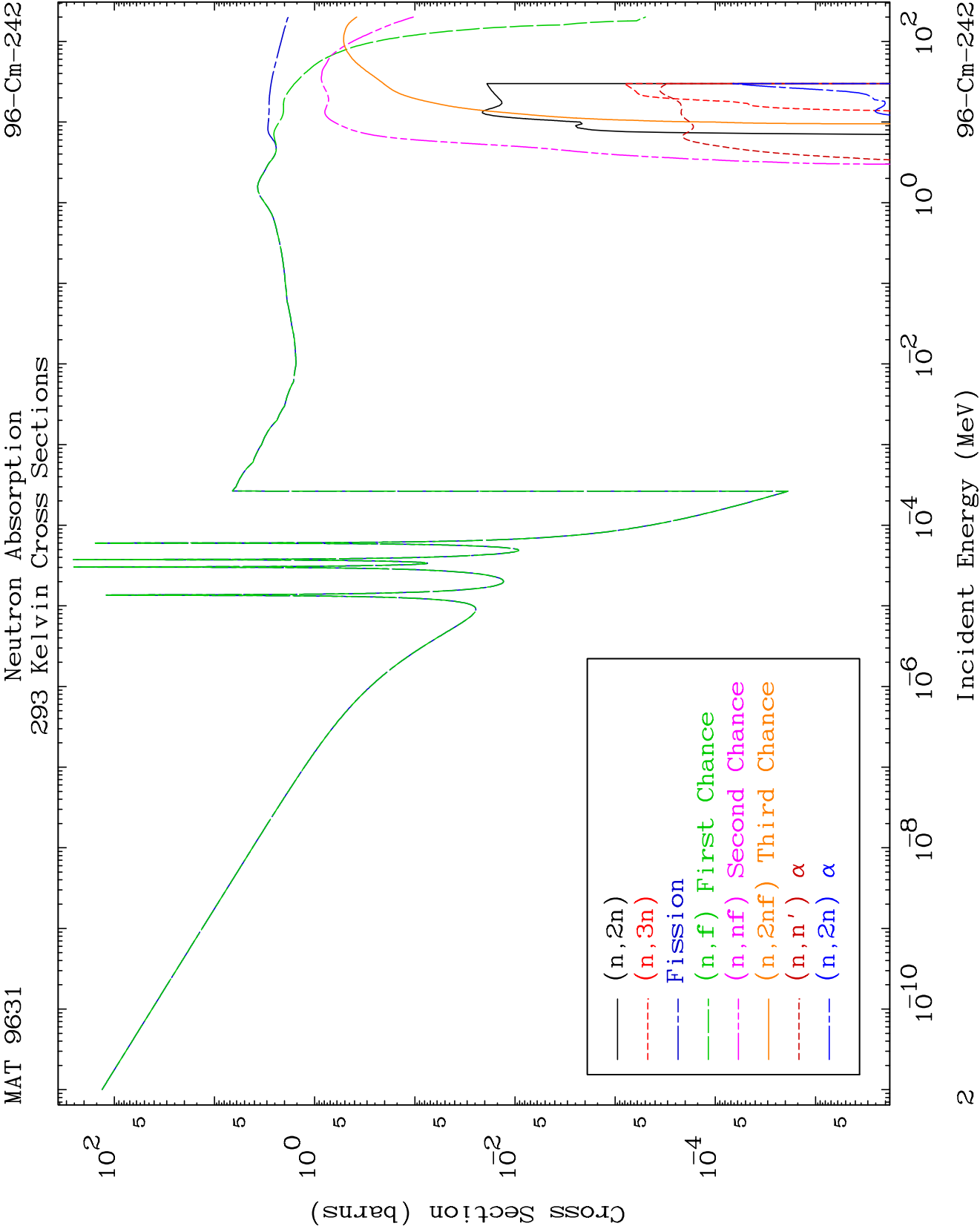
MAT 9631

Neutron Major

293 Kelvin Cross Sections

96-Cm-242

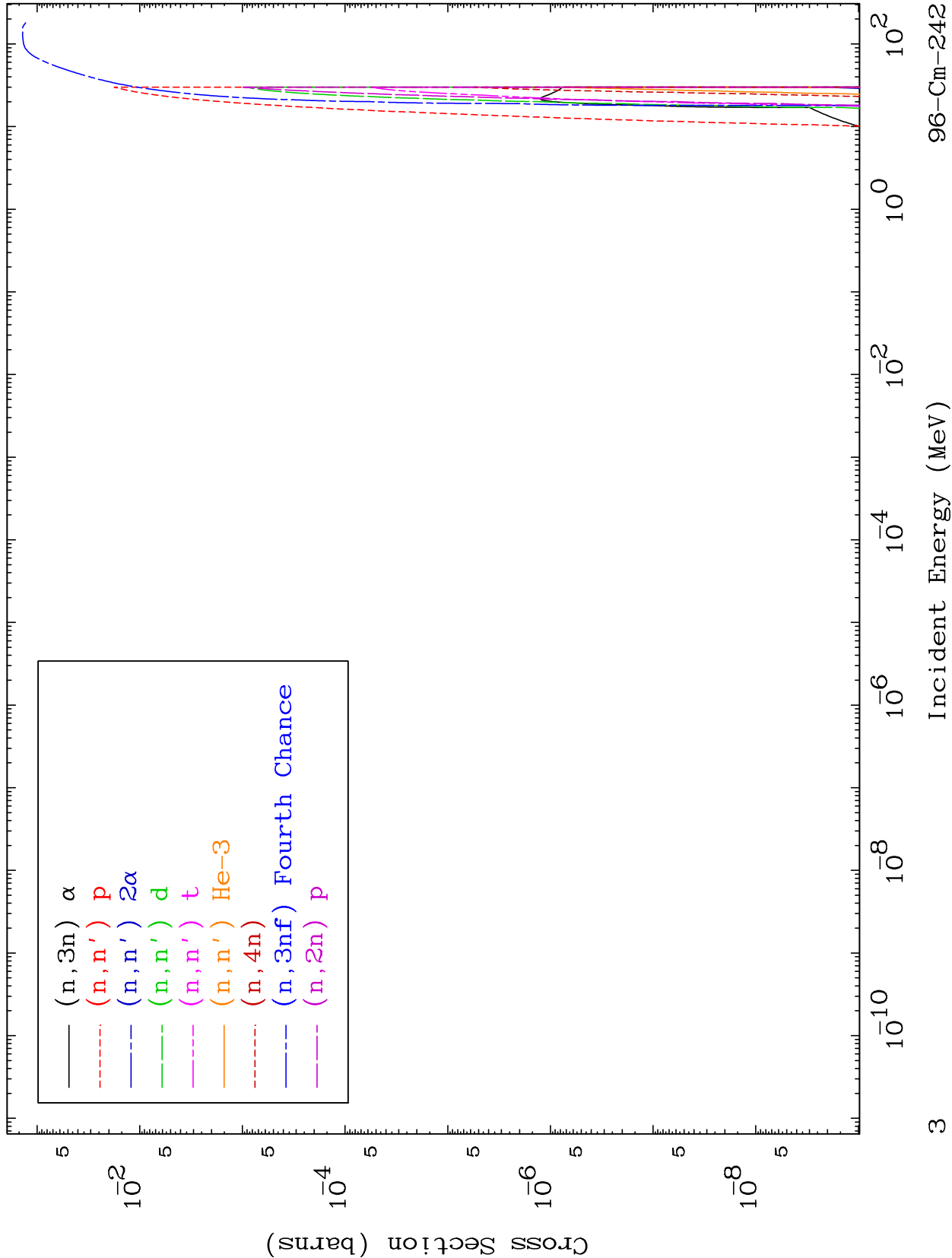


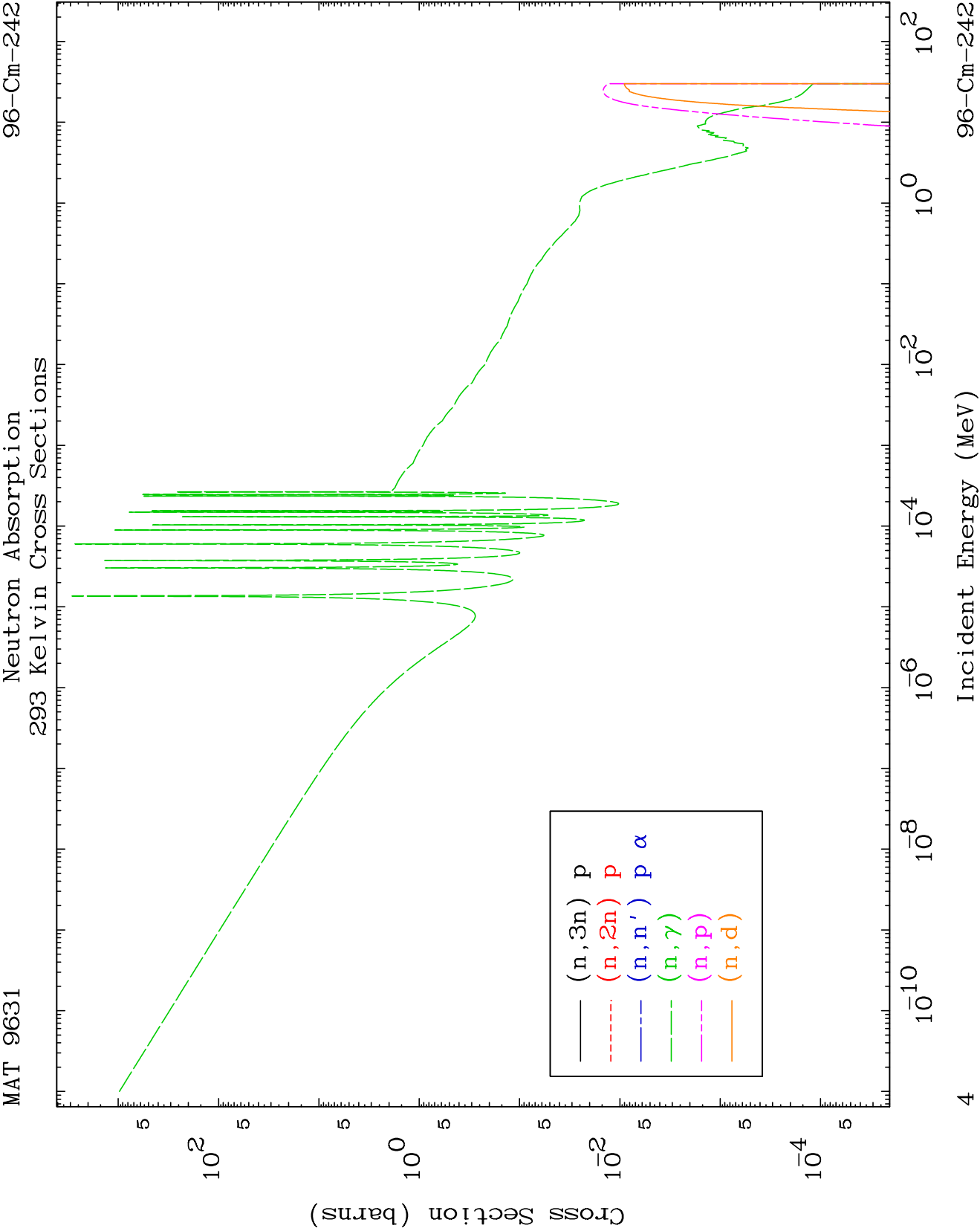


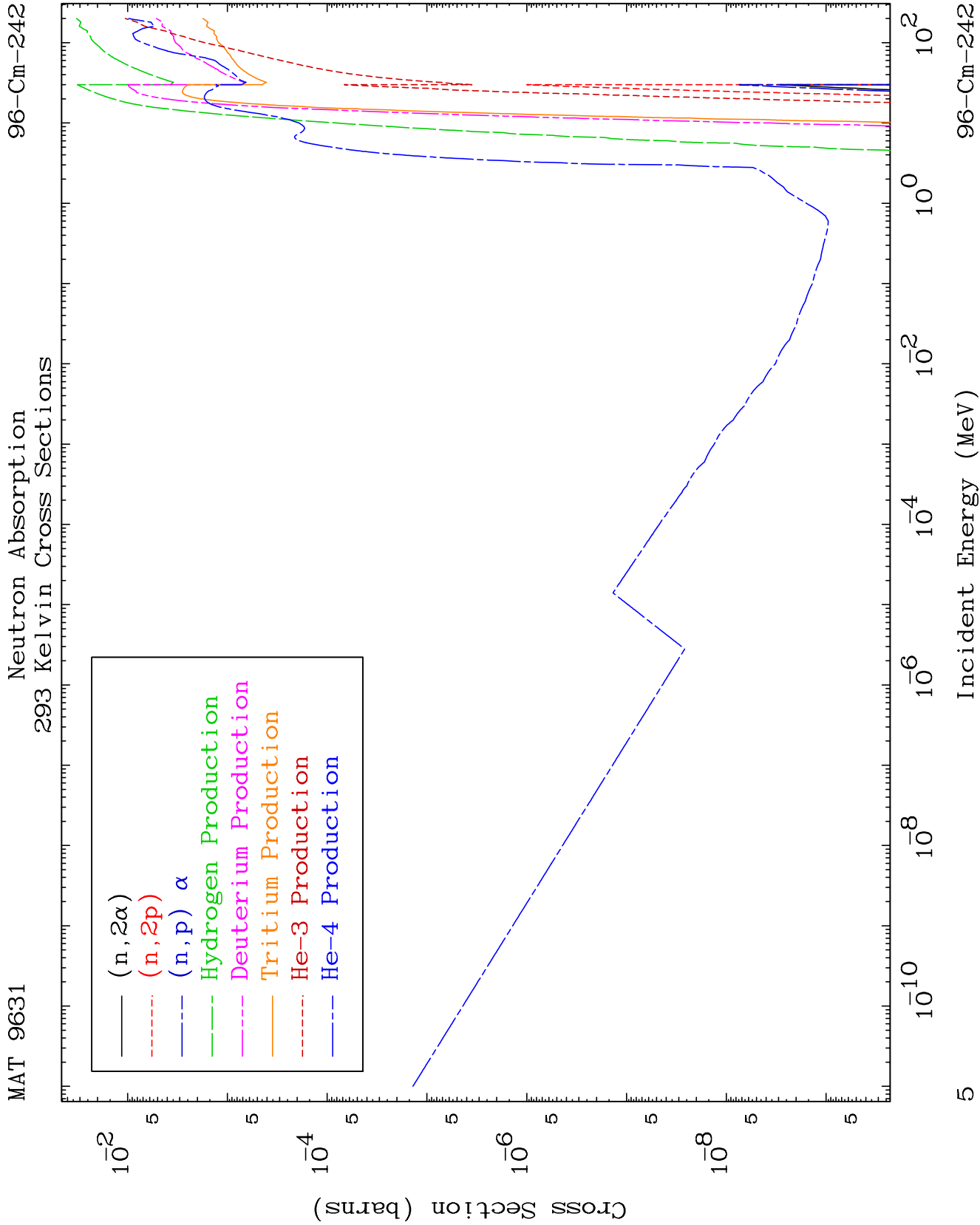
MAT 9631

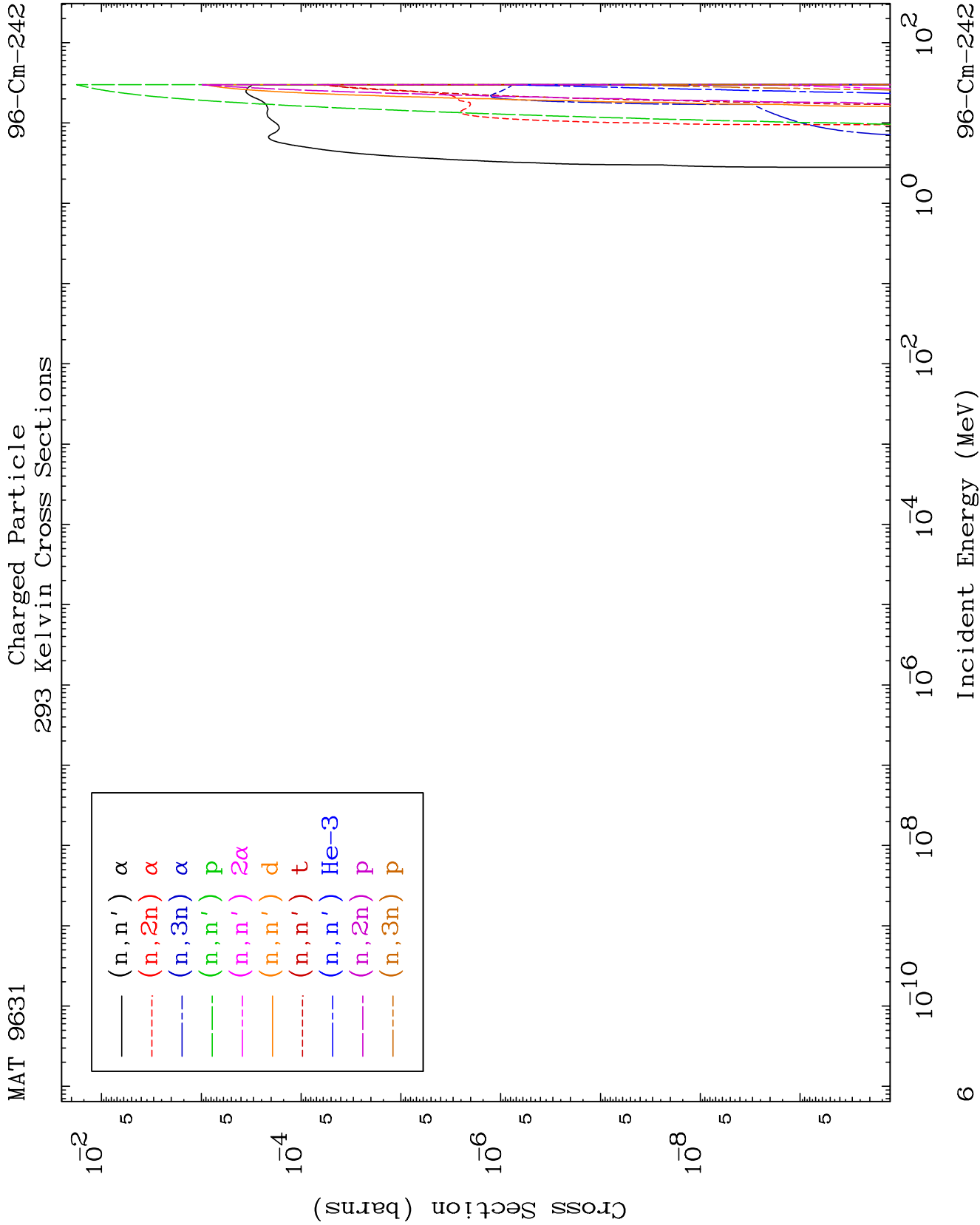
Neutron Absorption
293 Kelvin Cross Sections

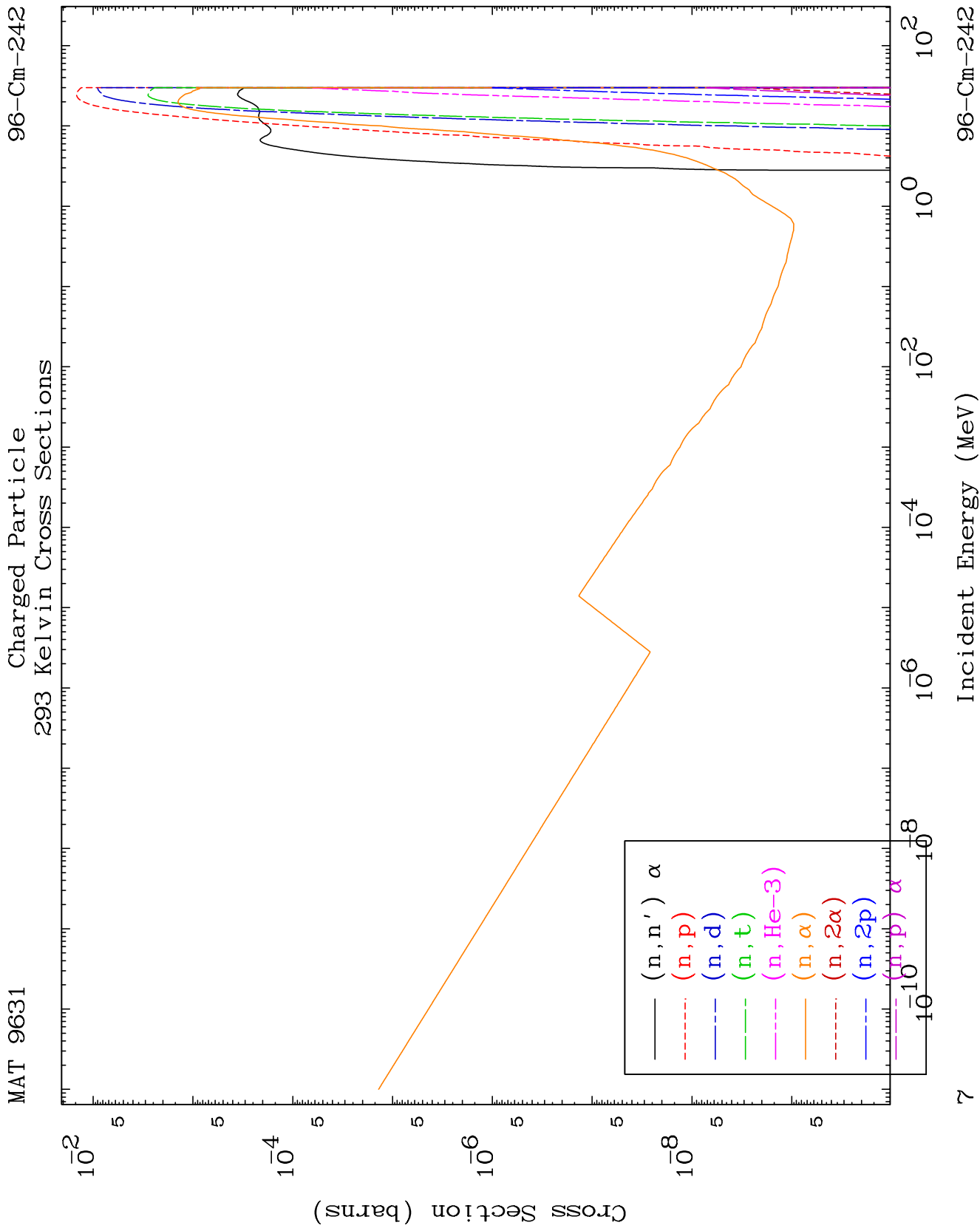
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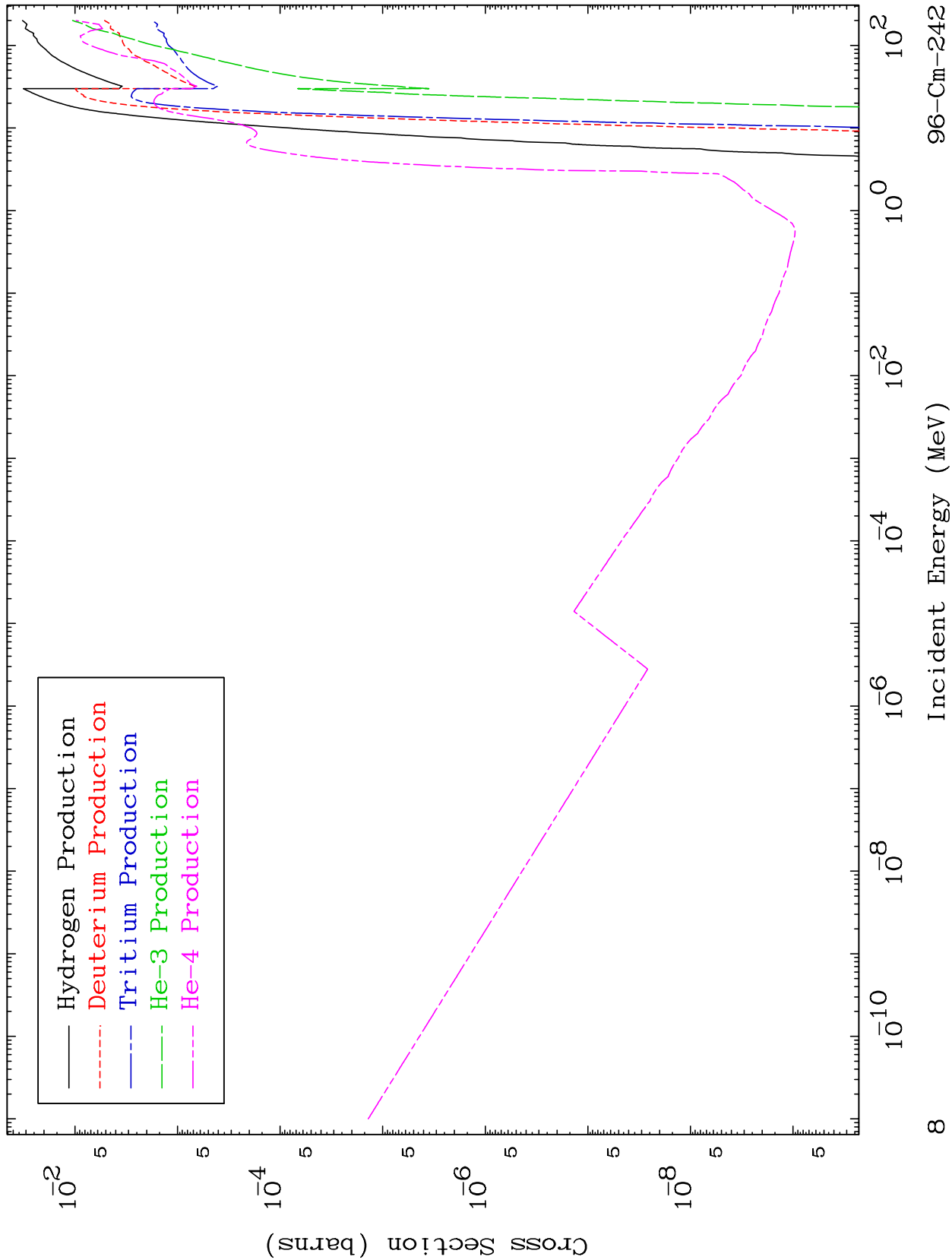


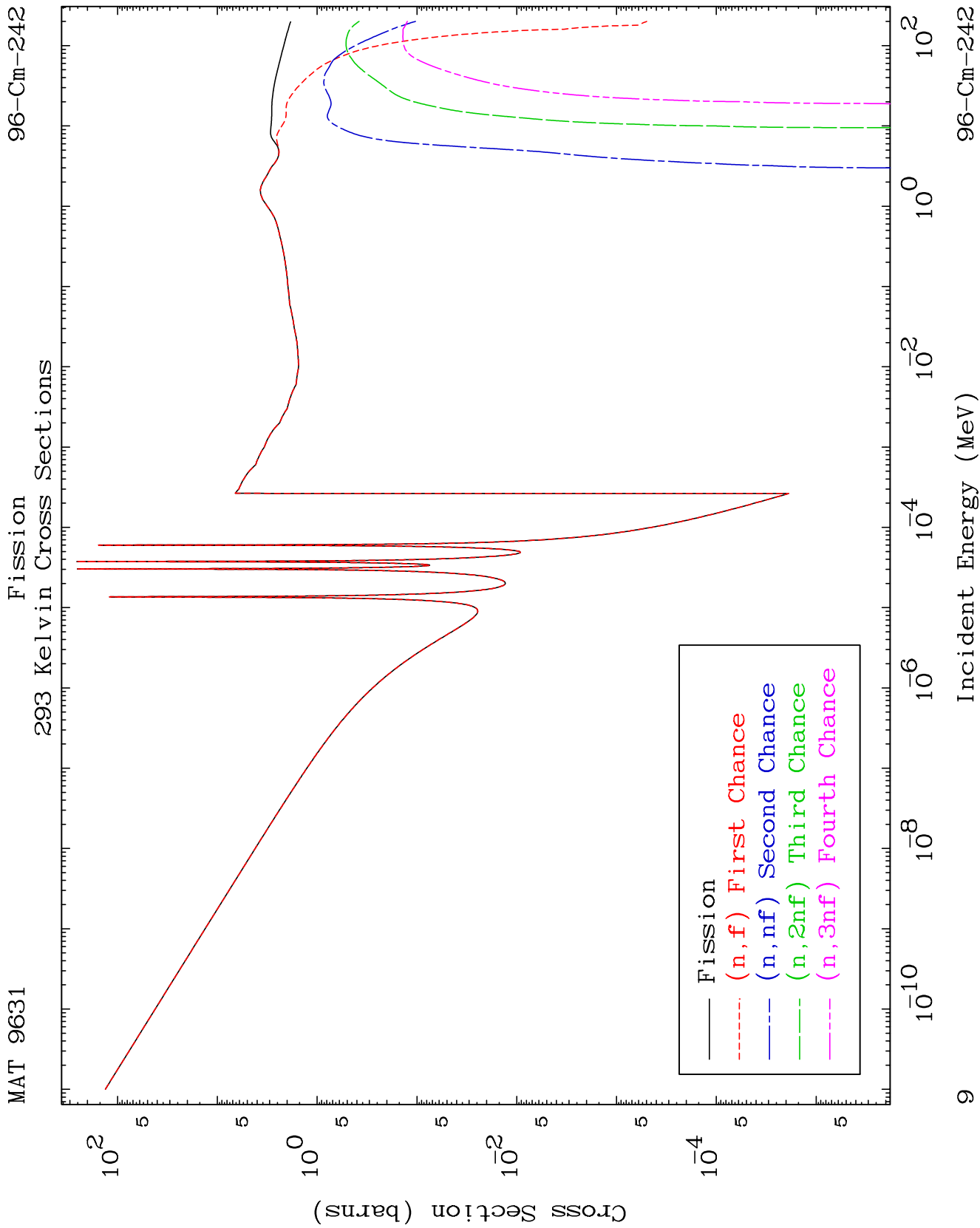


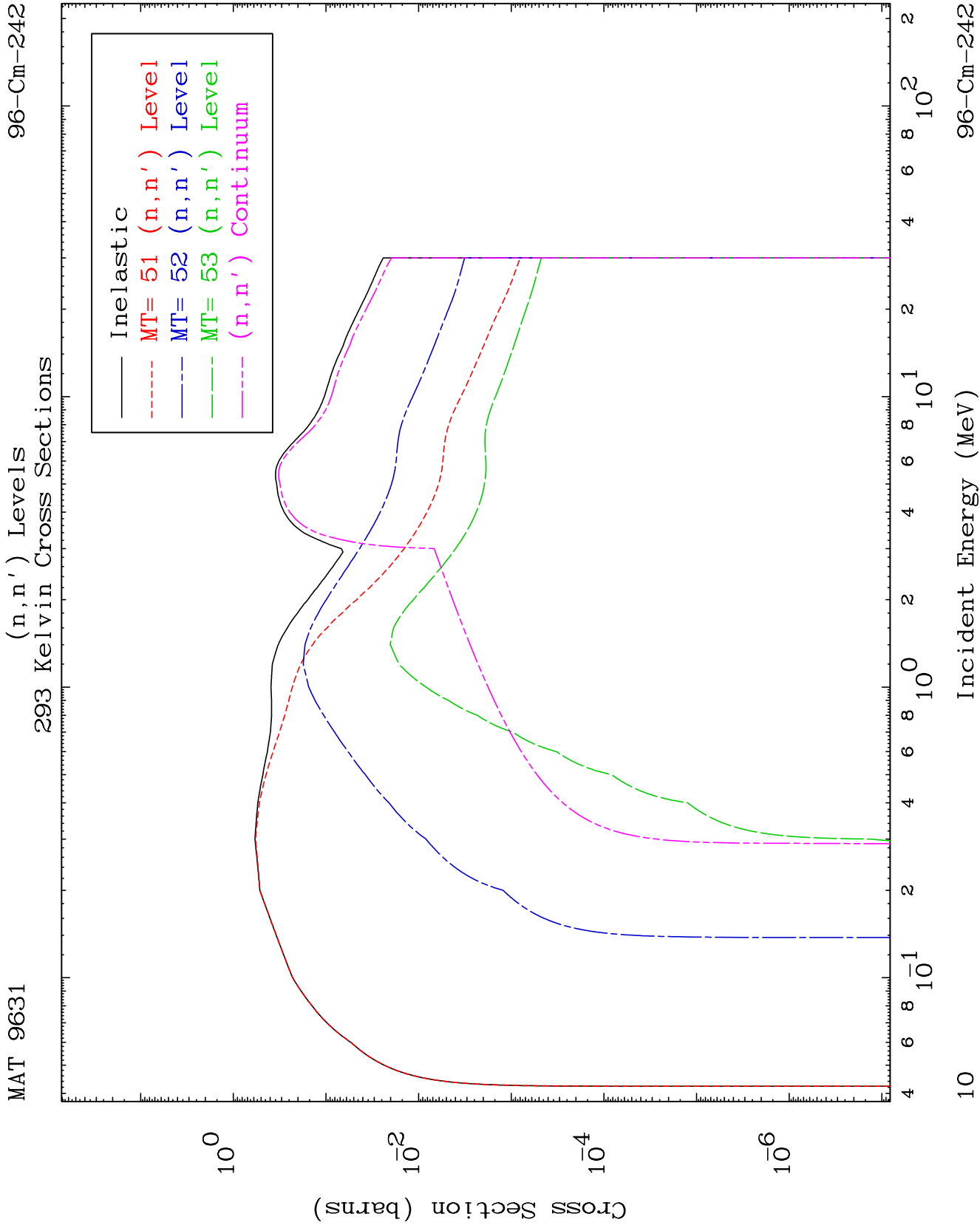
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Particle Production
293 Kelvin Cross Sections

96-Cm-242



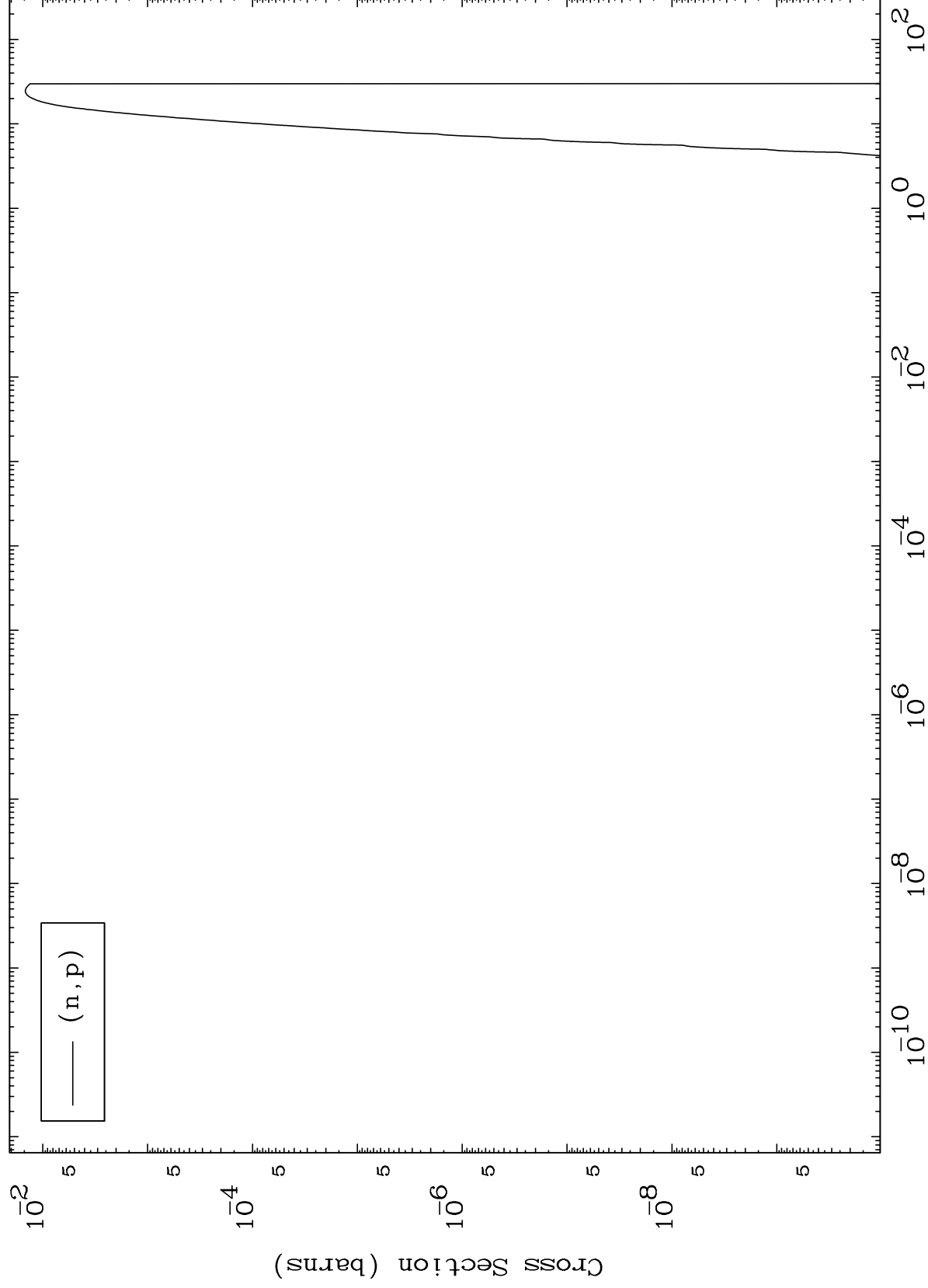


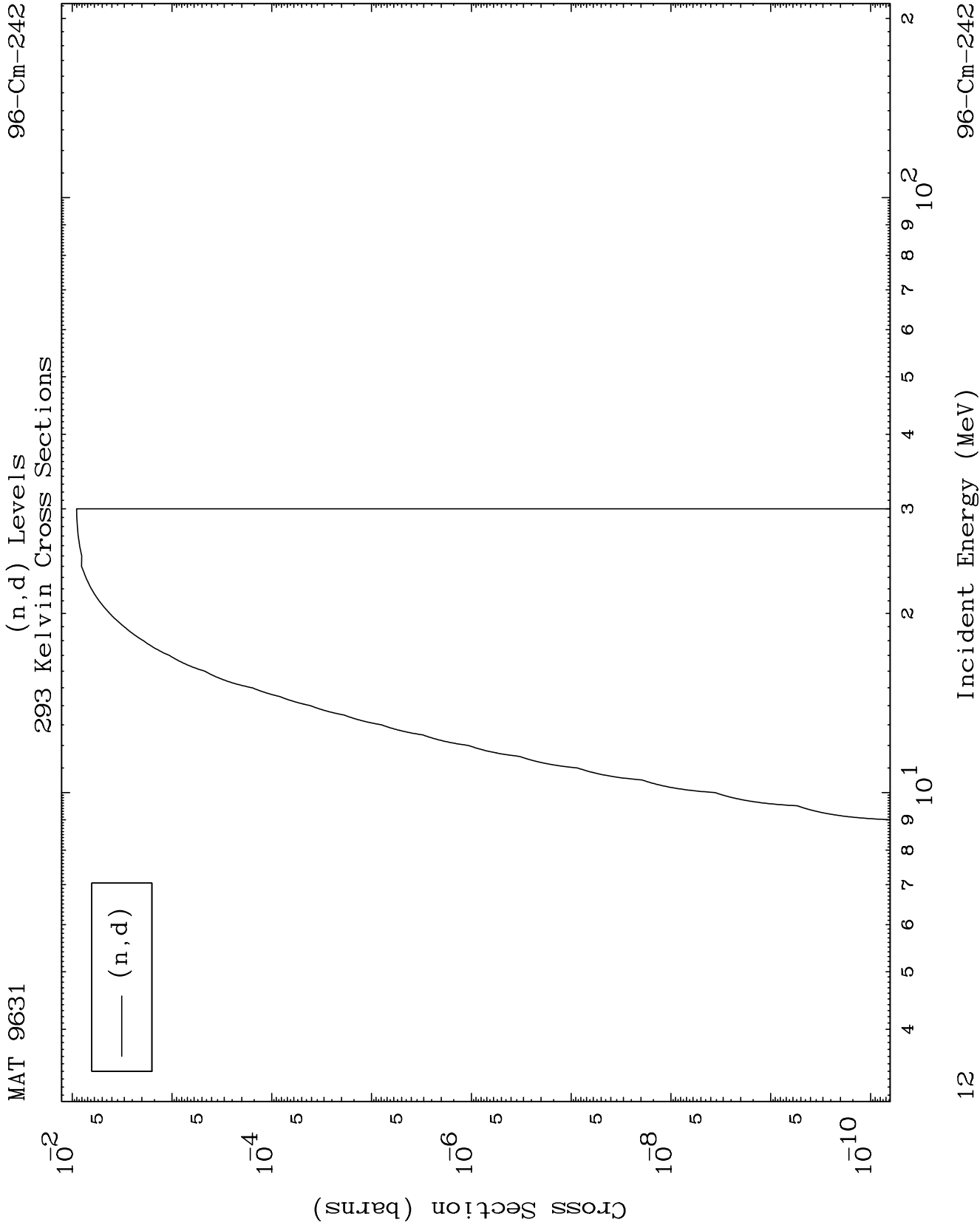


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(n,p) Levels
293 Kelvin Cross Sections

96-Cm-242

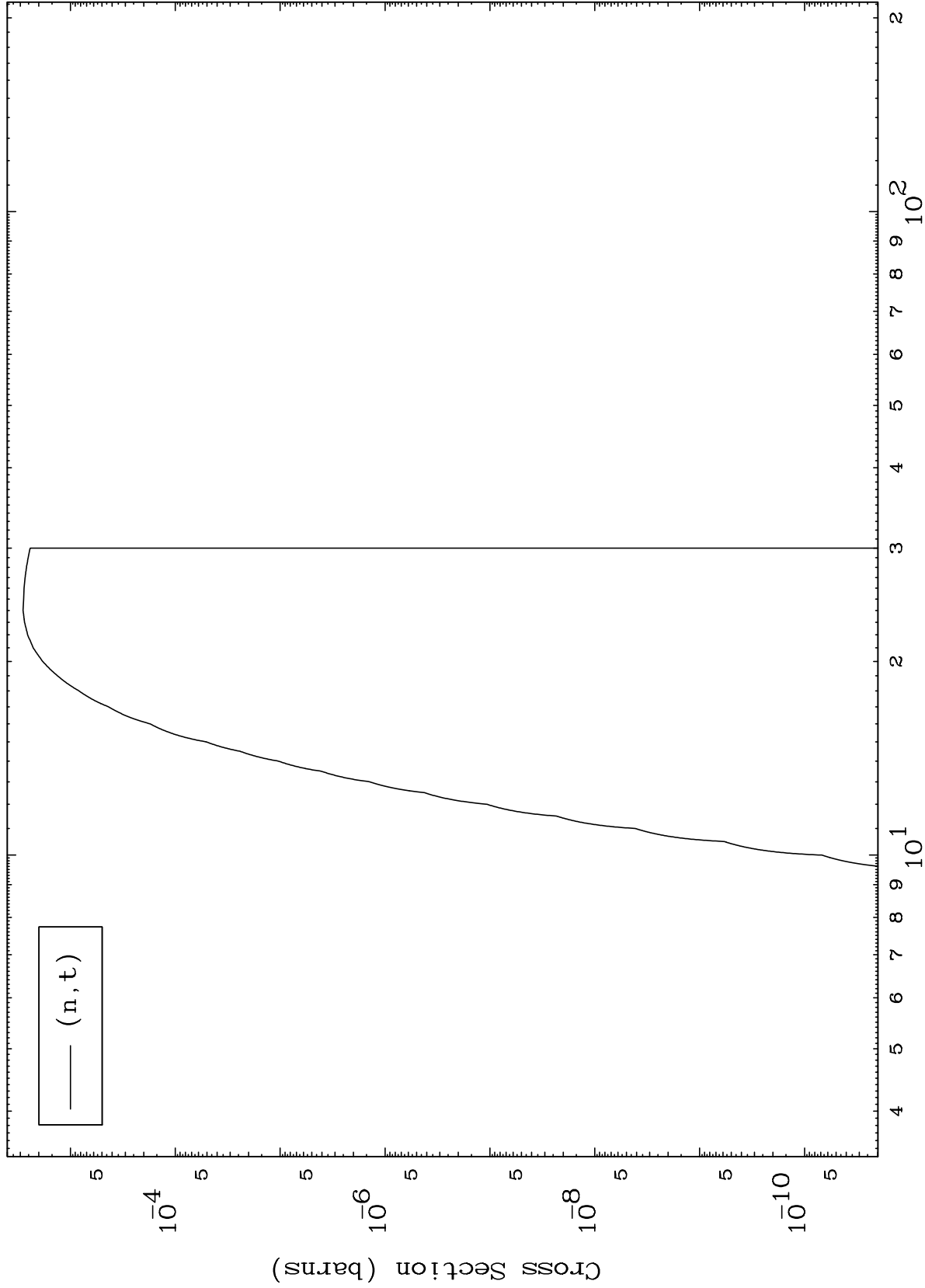




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96-Cm-242

(n,t) Levels
293 Kelvin Cross Sections



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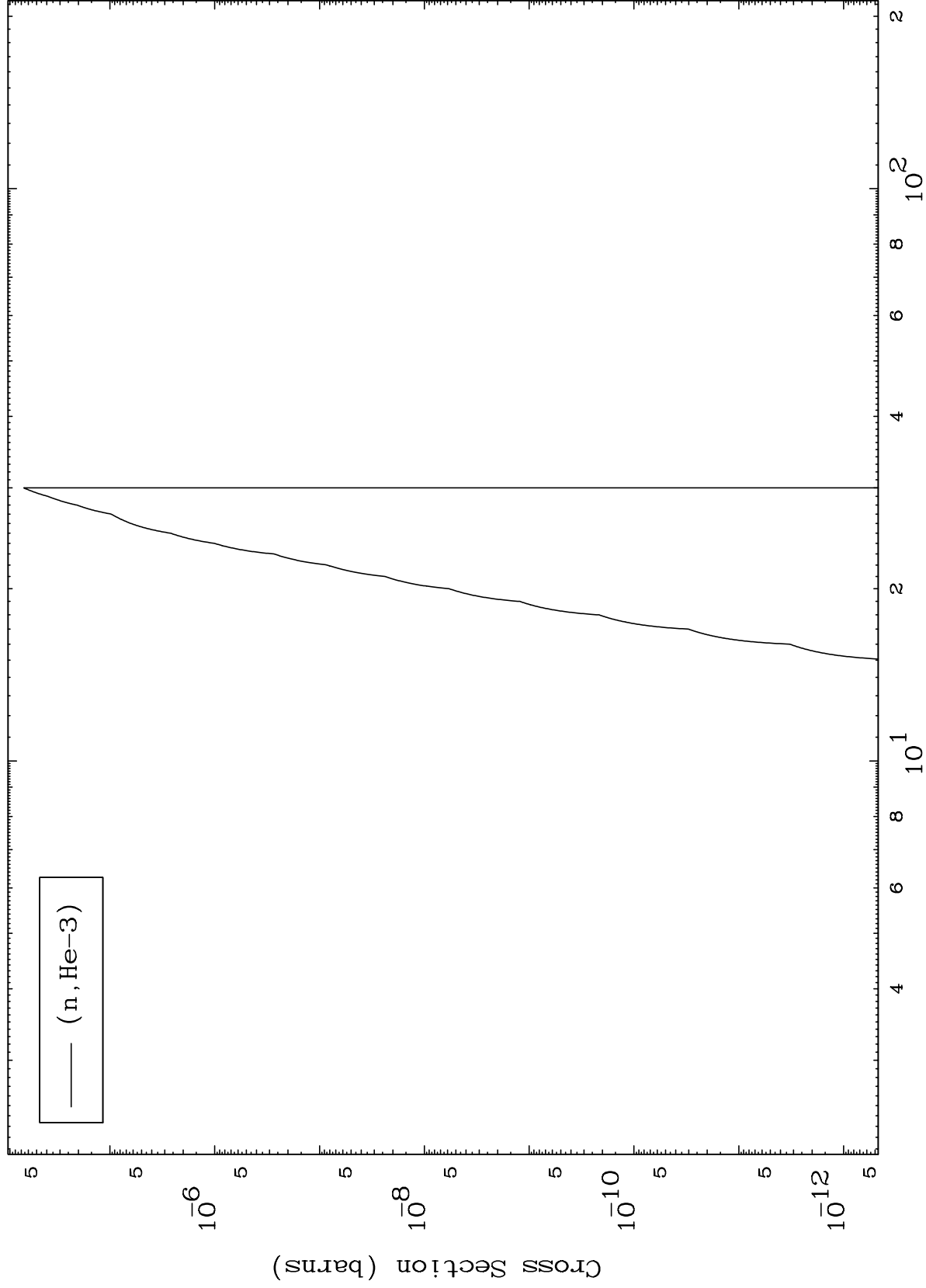
Incident Energy (MeV)

96-Cm-242

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(n,He3) Levels
293 Kelvin Cross Sections

96-Cm-242



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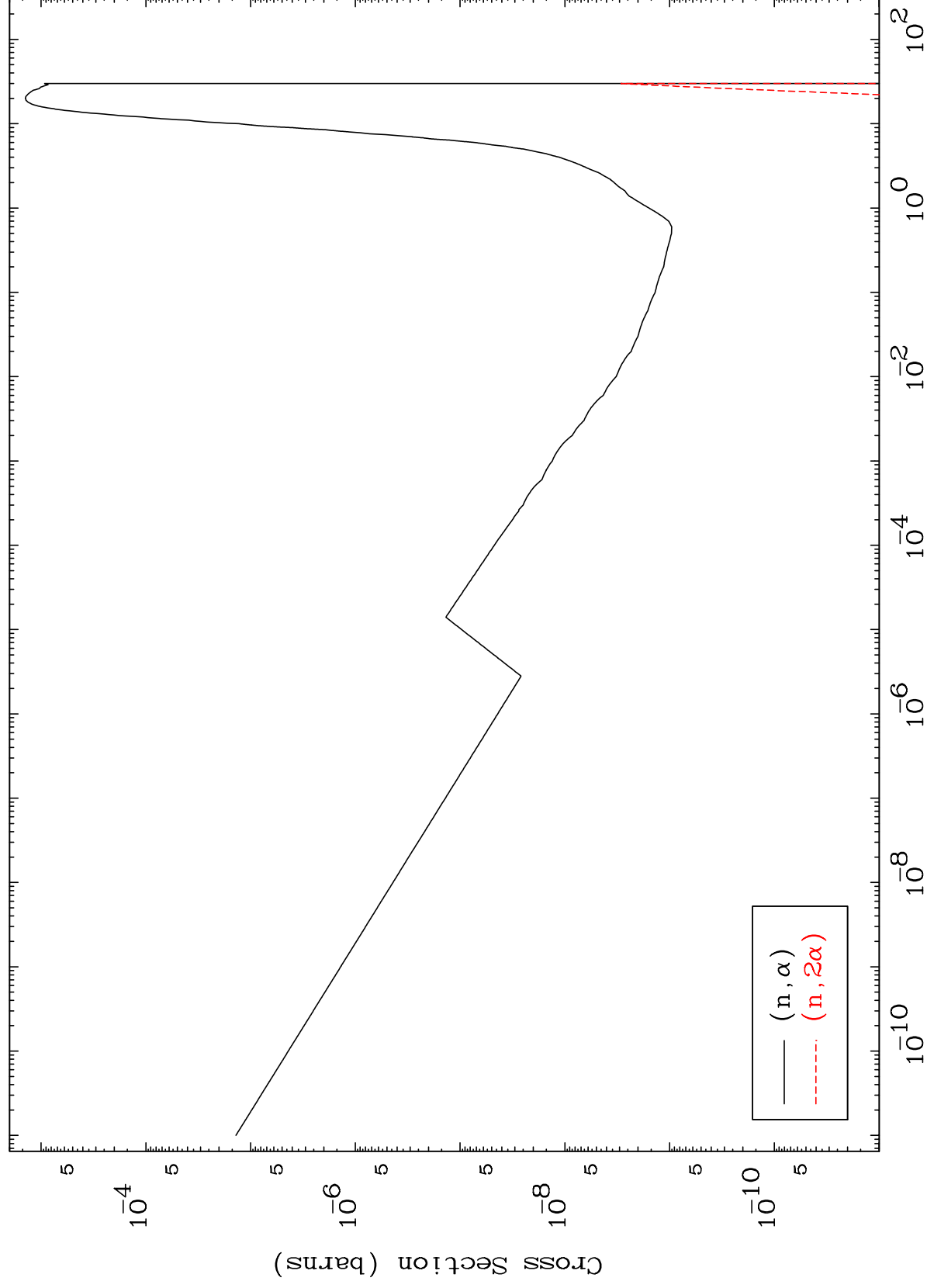
Incident Energy (MeV)

96-Cm-242

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(n, α) Levels
293 Kelvin Cross Sections

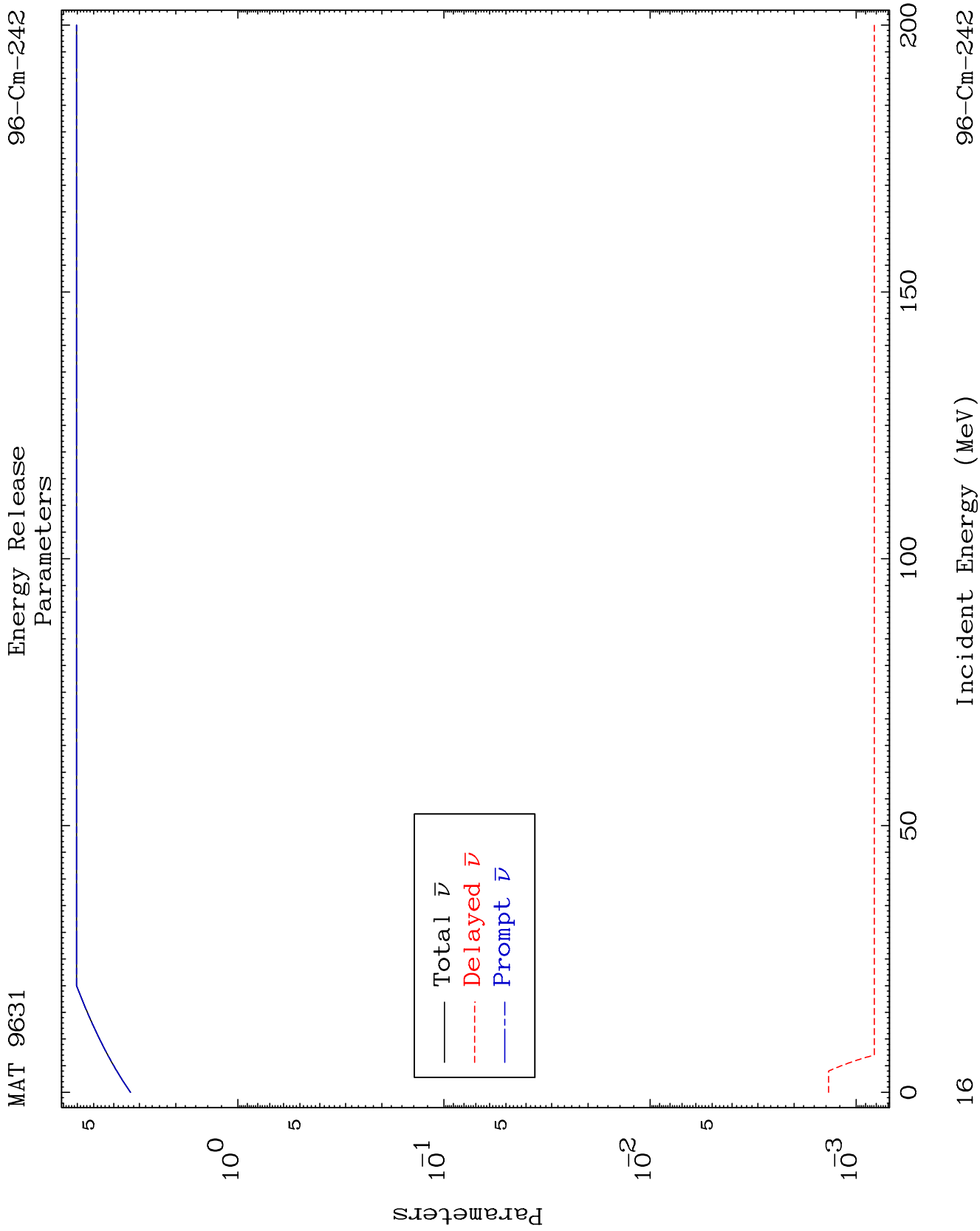
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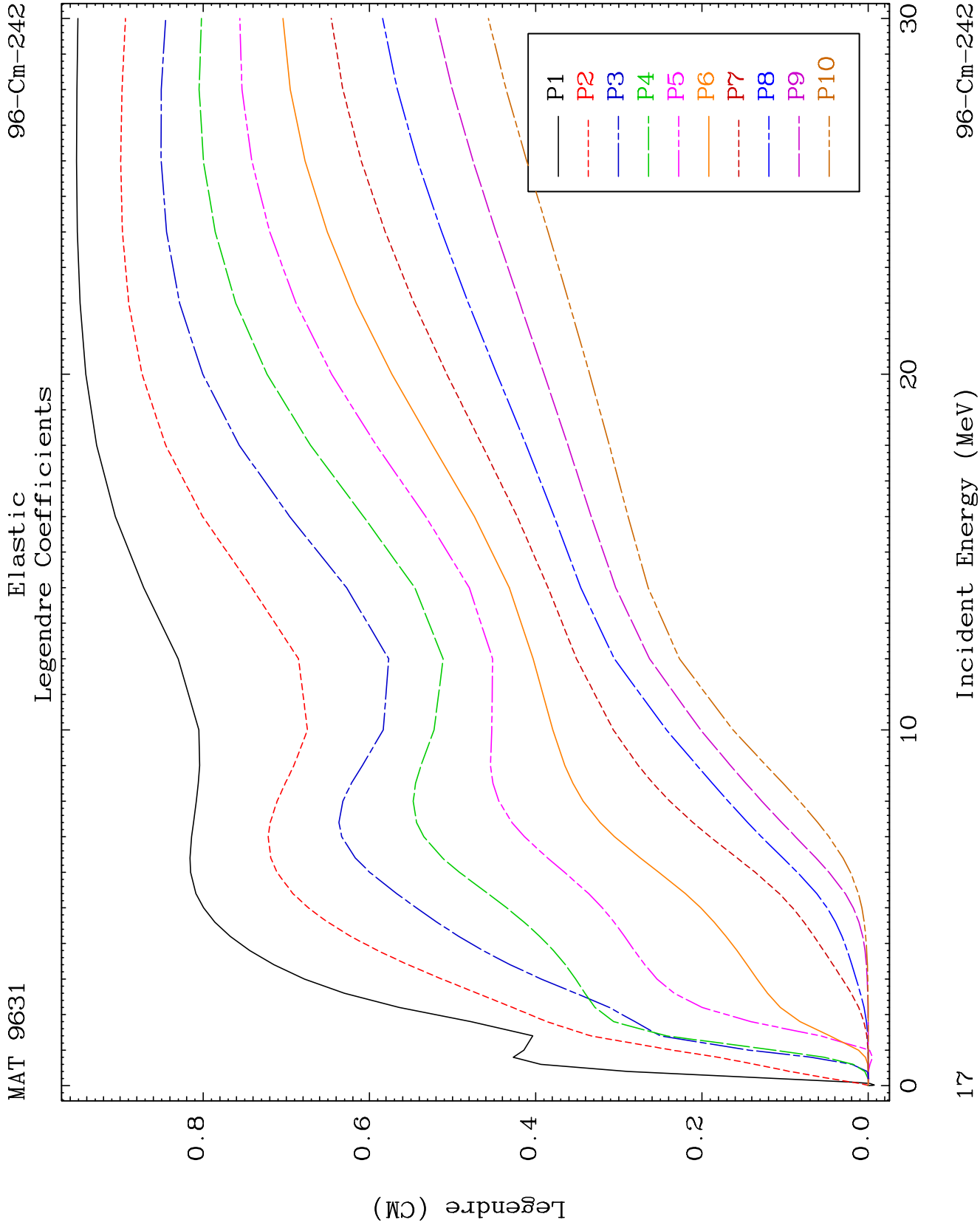


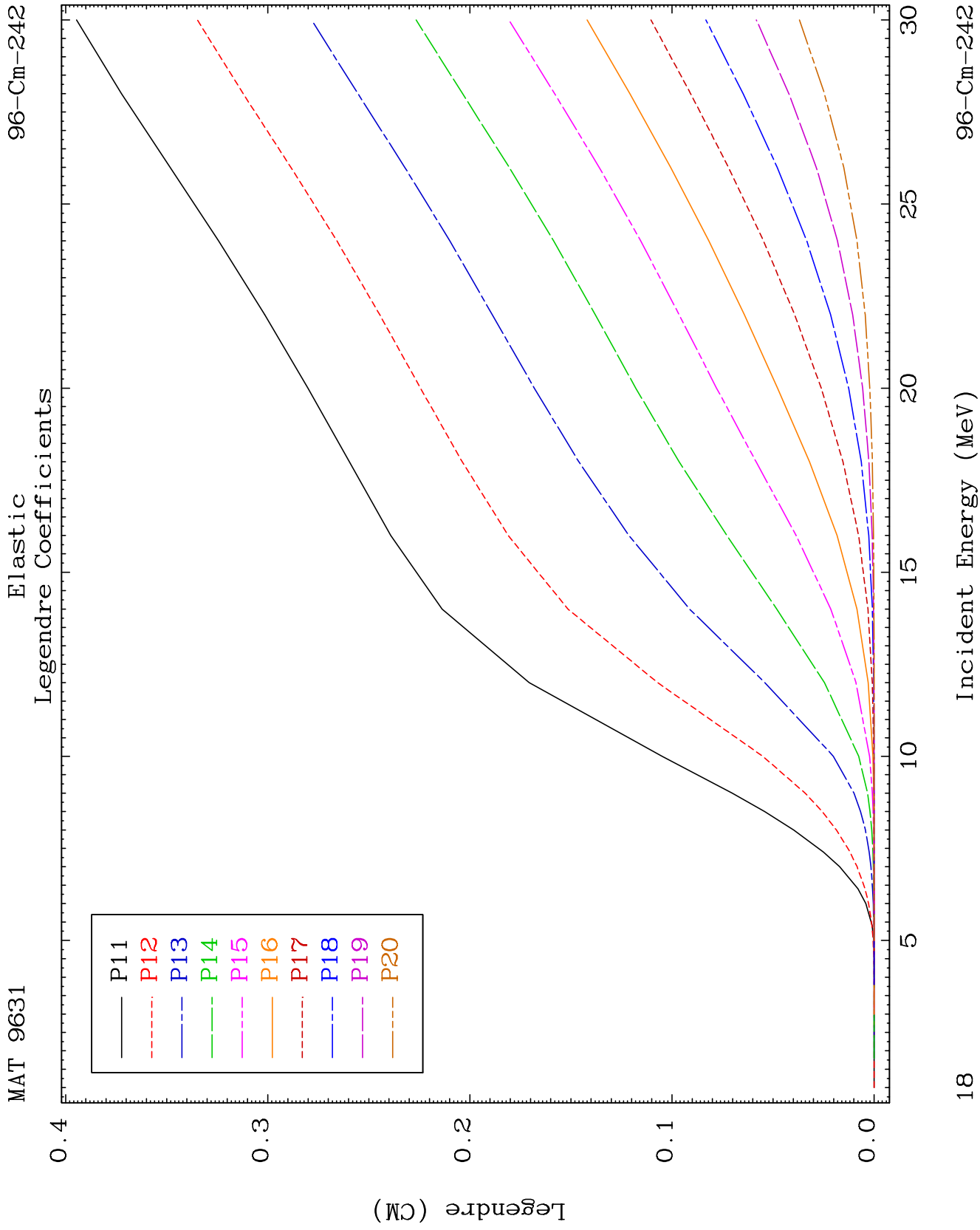
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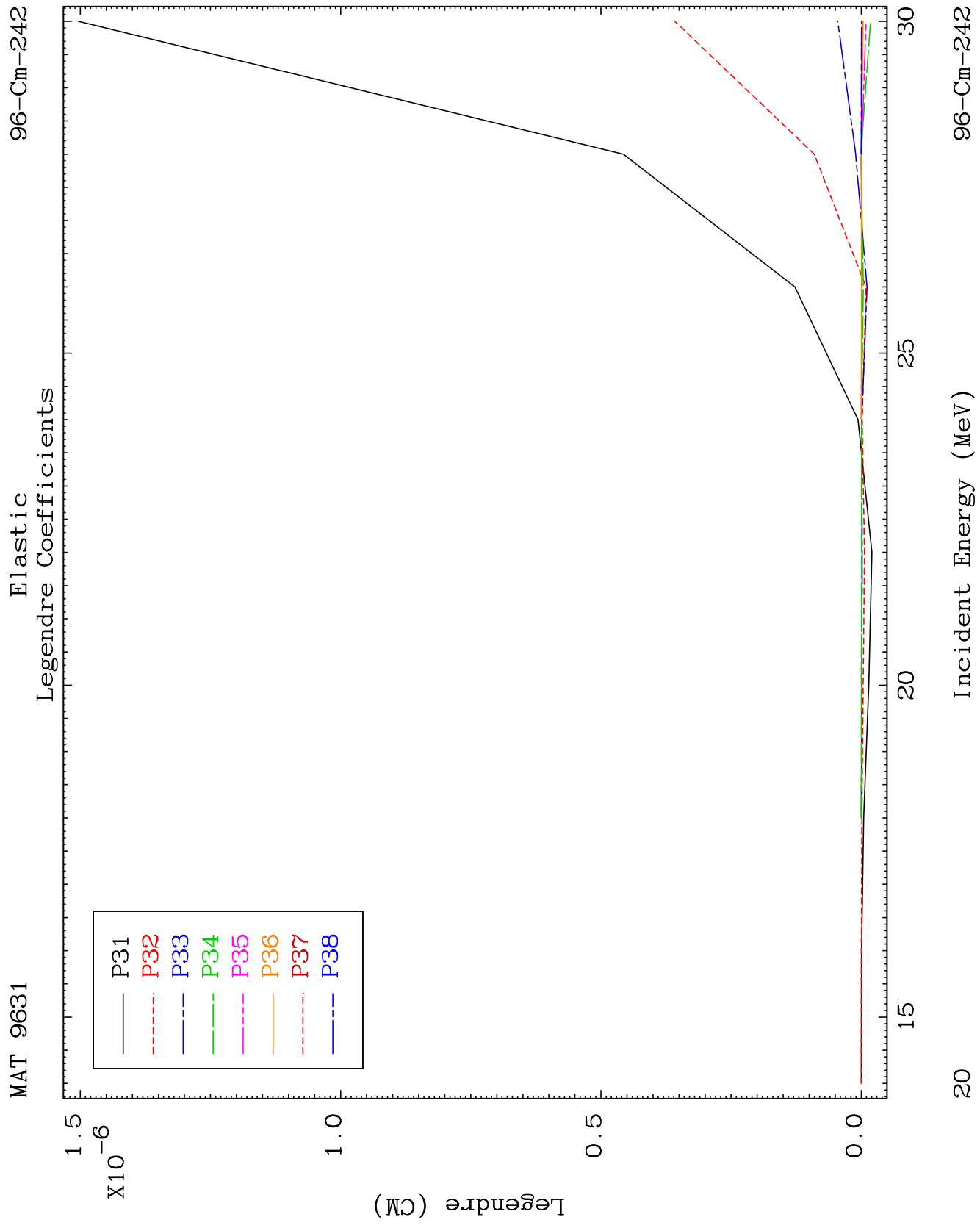
Incident Energy (MeV)

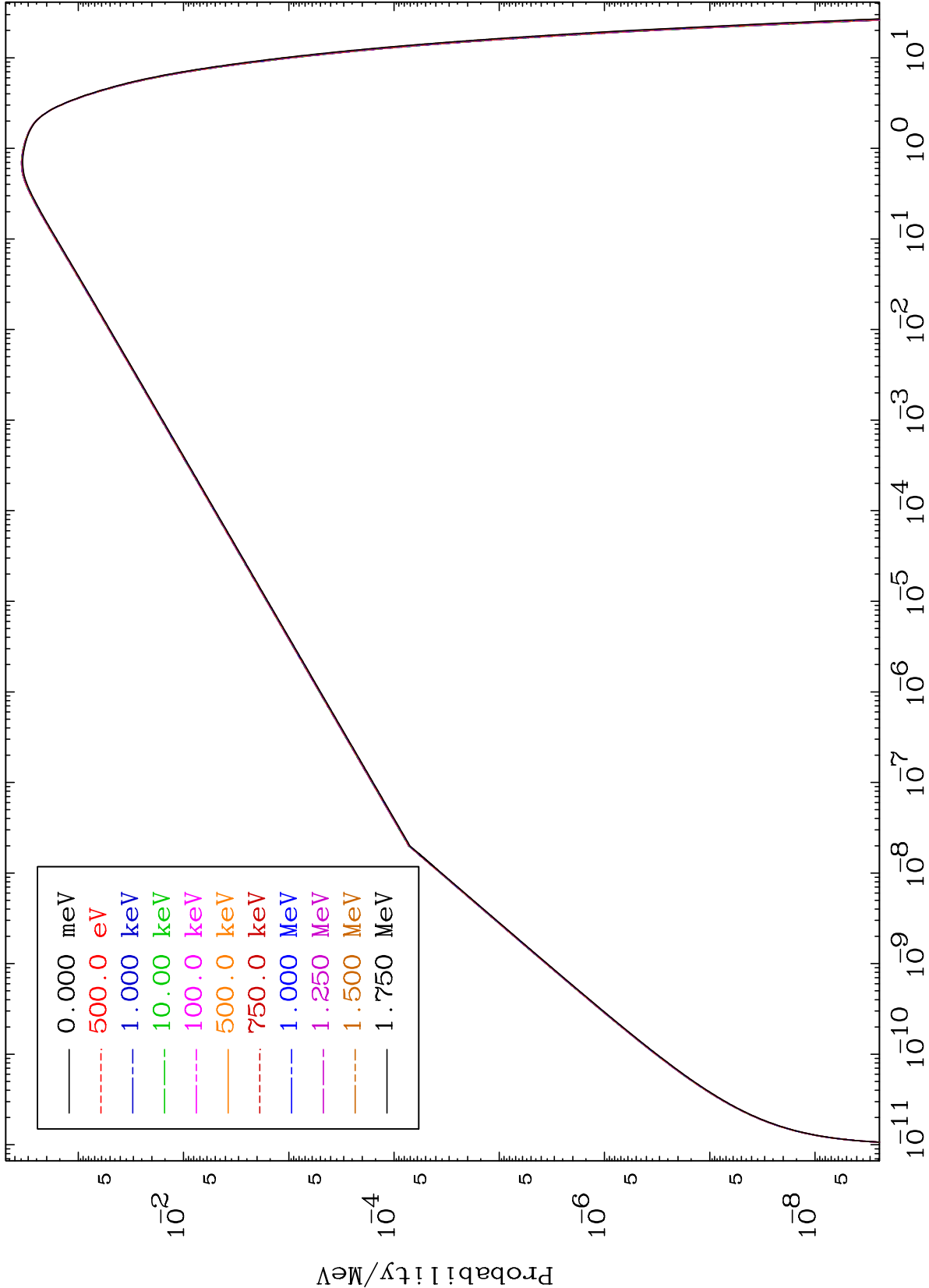
96-Cm-242







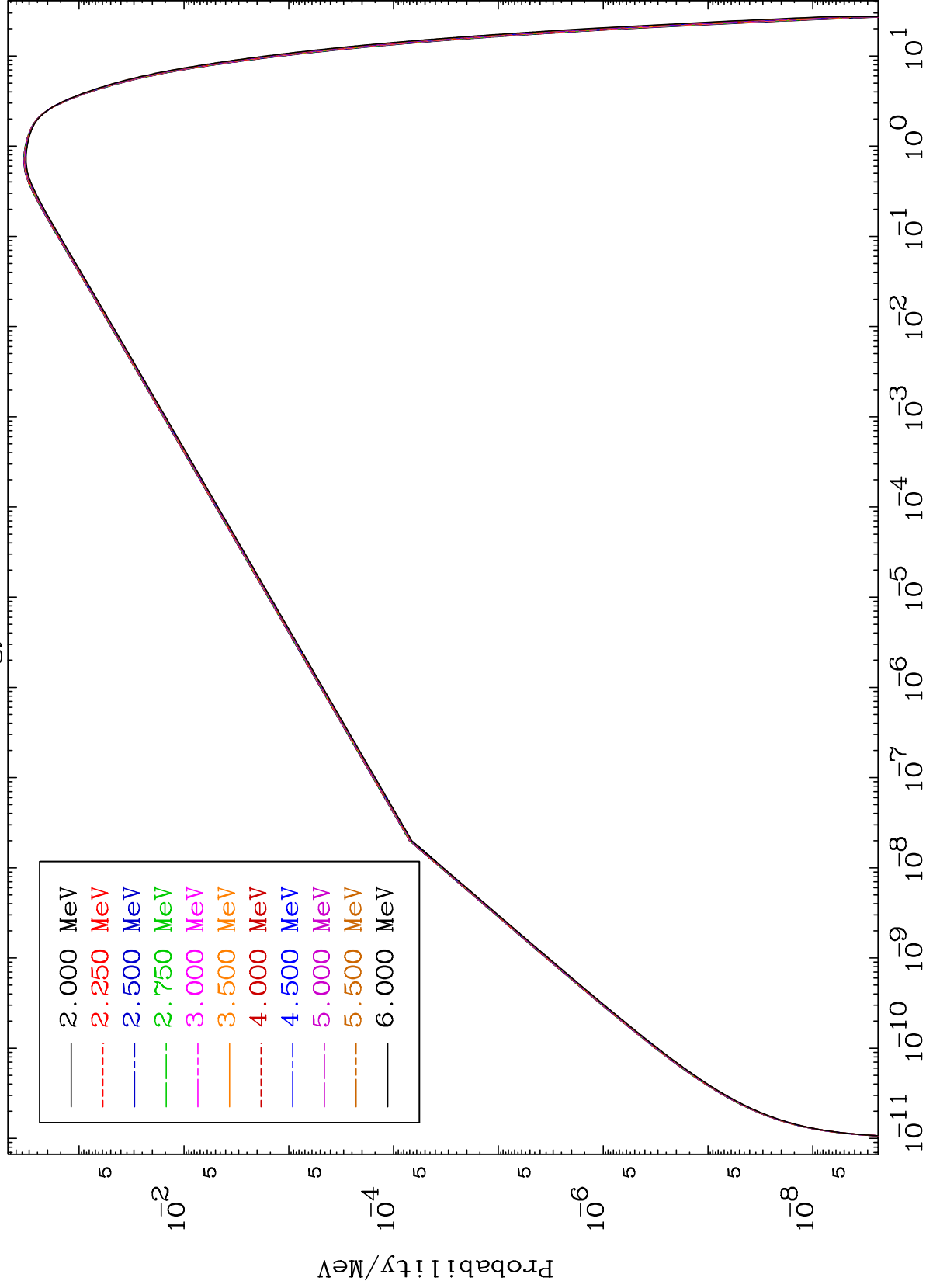




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Fission
Energy Distributions

96-Cm-242



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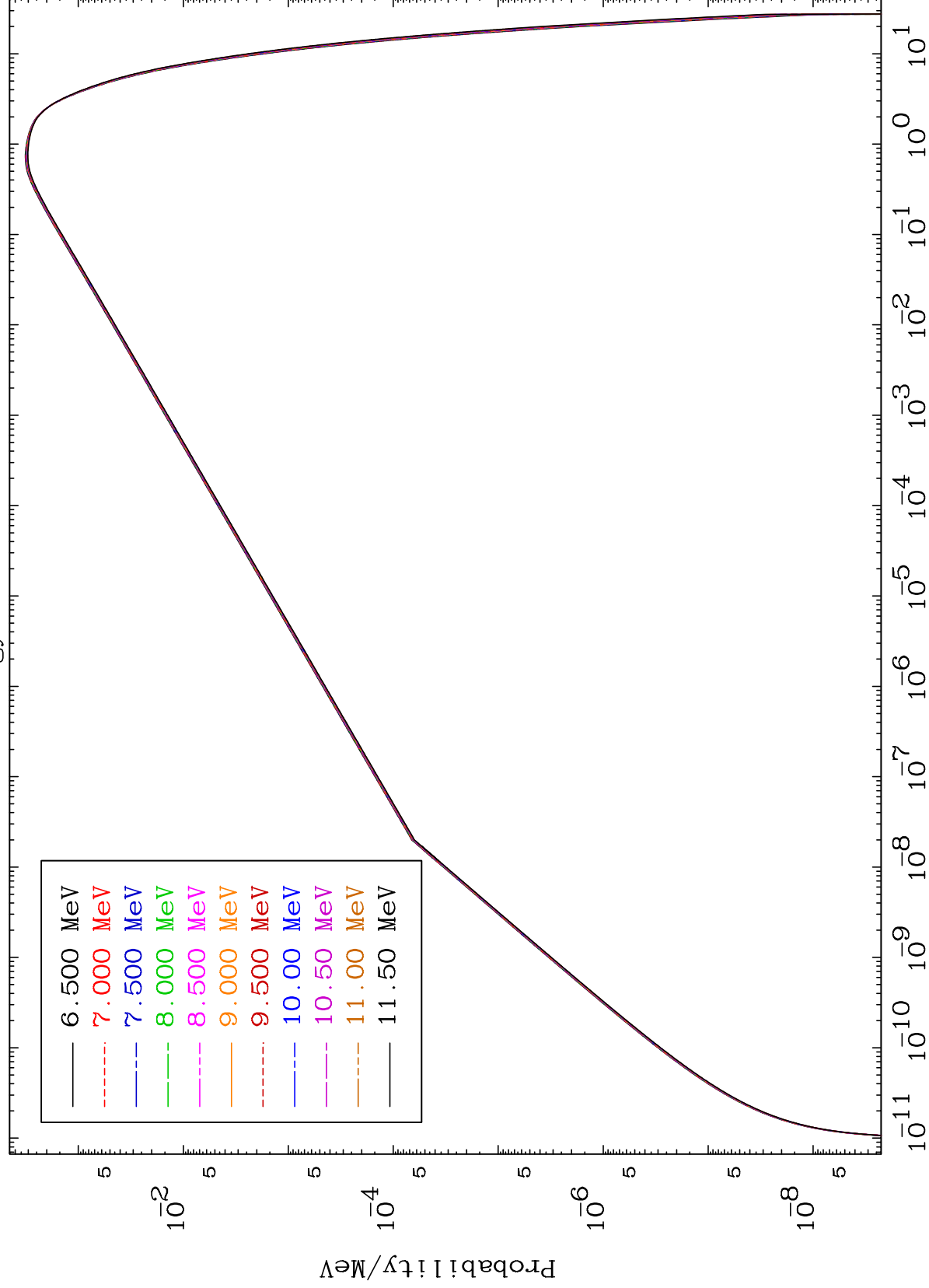
Secondary Energy (MeV)

96-Cm-242

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Fission Energy Distributions

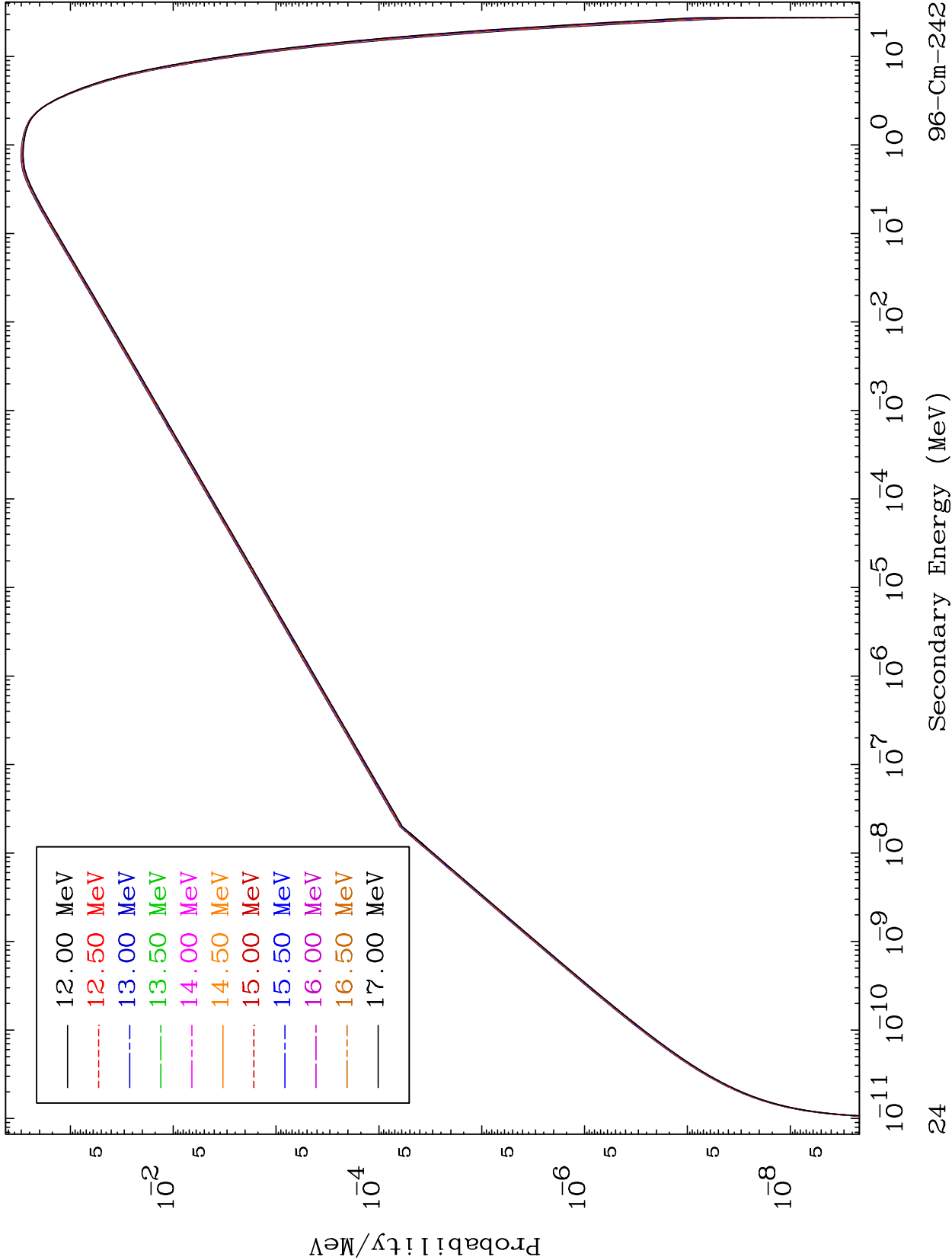
96-Cm-242



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Fission
Energy Distributions

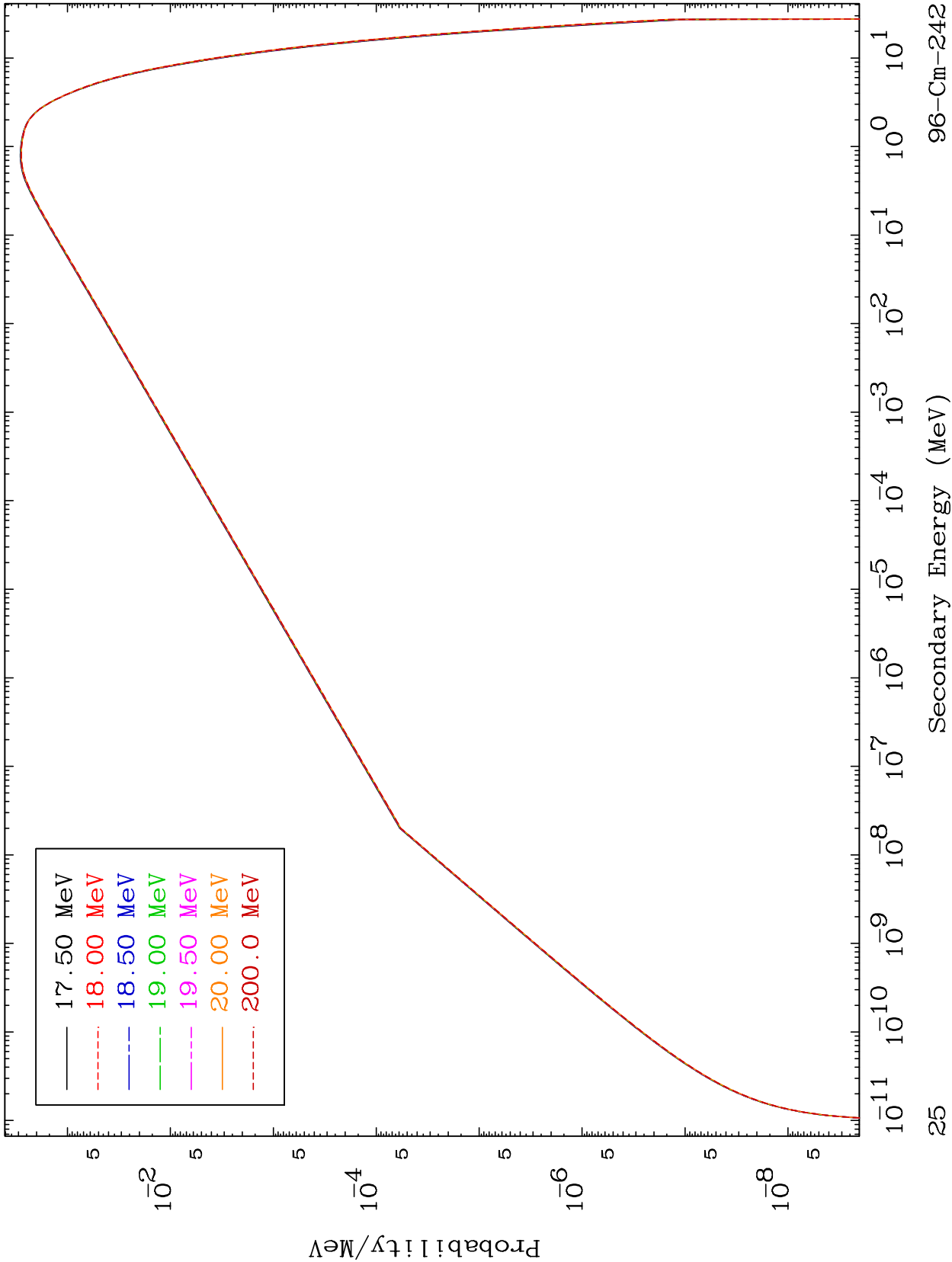
96-Cm-242



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Fission
Energy Distributions

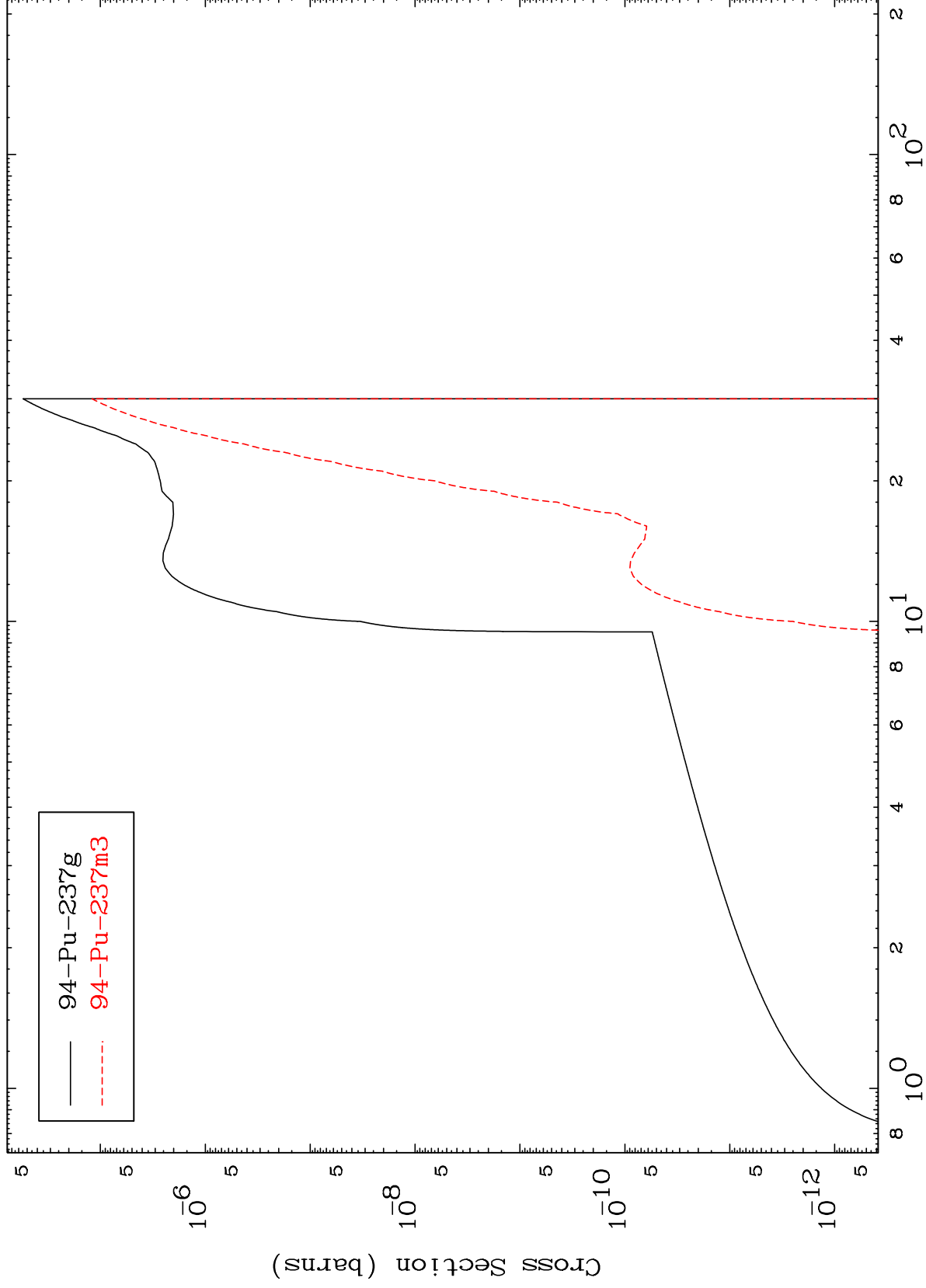
96-Cm-242



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96-Cm-242

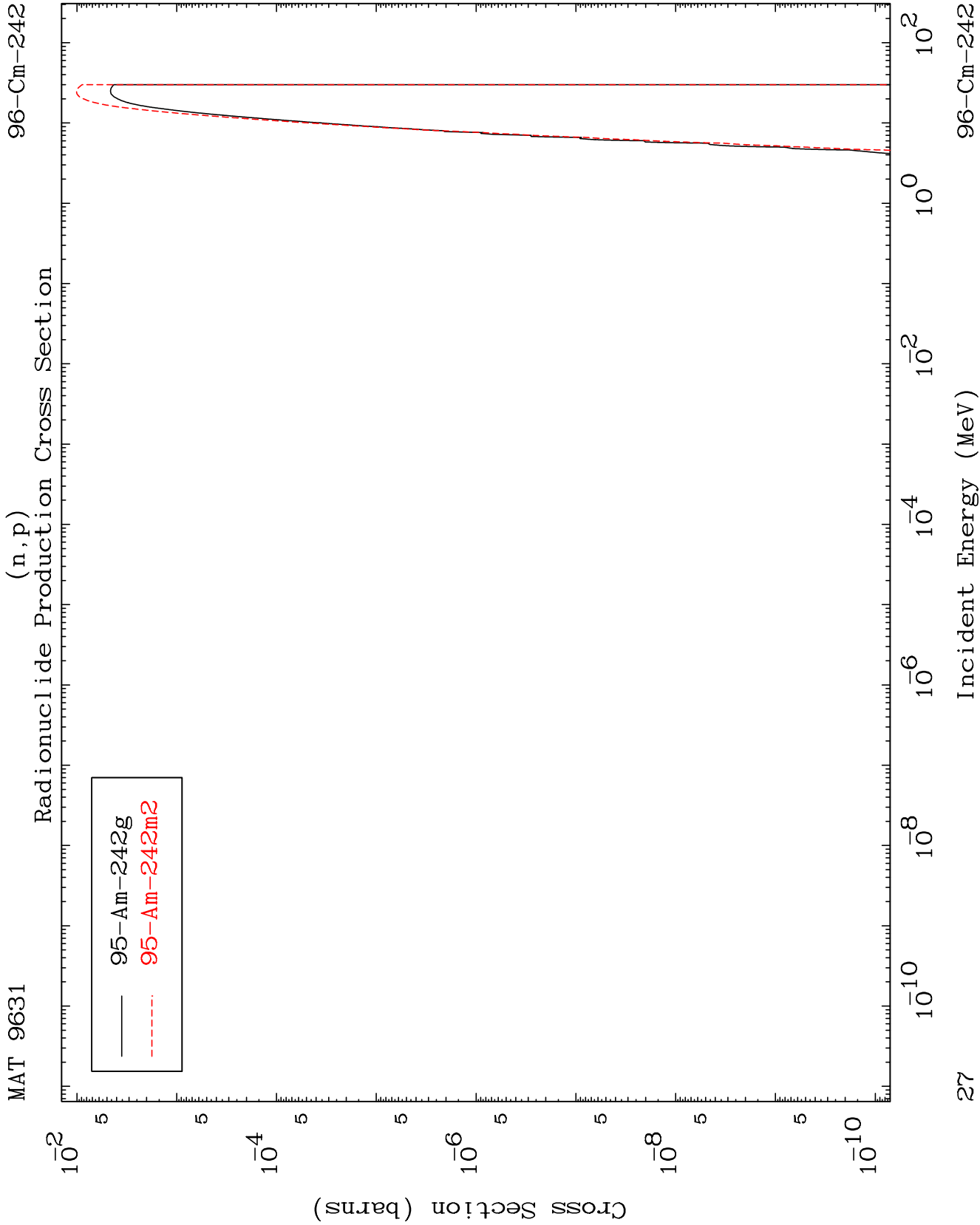
Radionuclide Production Cross Section
(n,2n) α



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Incident Energy (MeV)

96-Cm-242



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(n,2α)

96-Cm-242

Radionuclide Production Cross Section

