

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

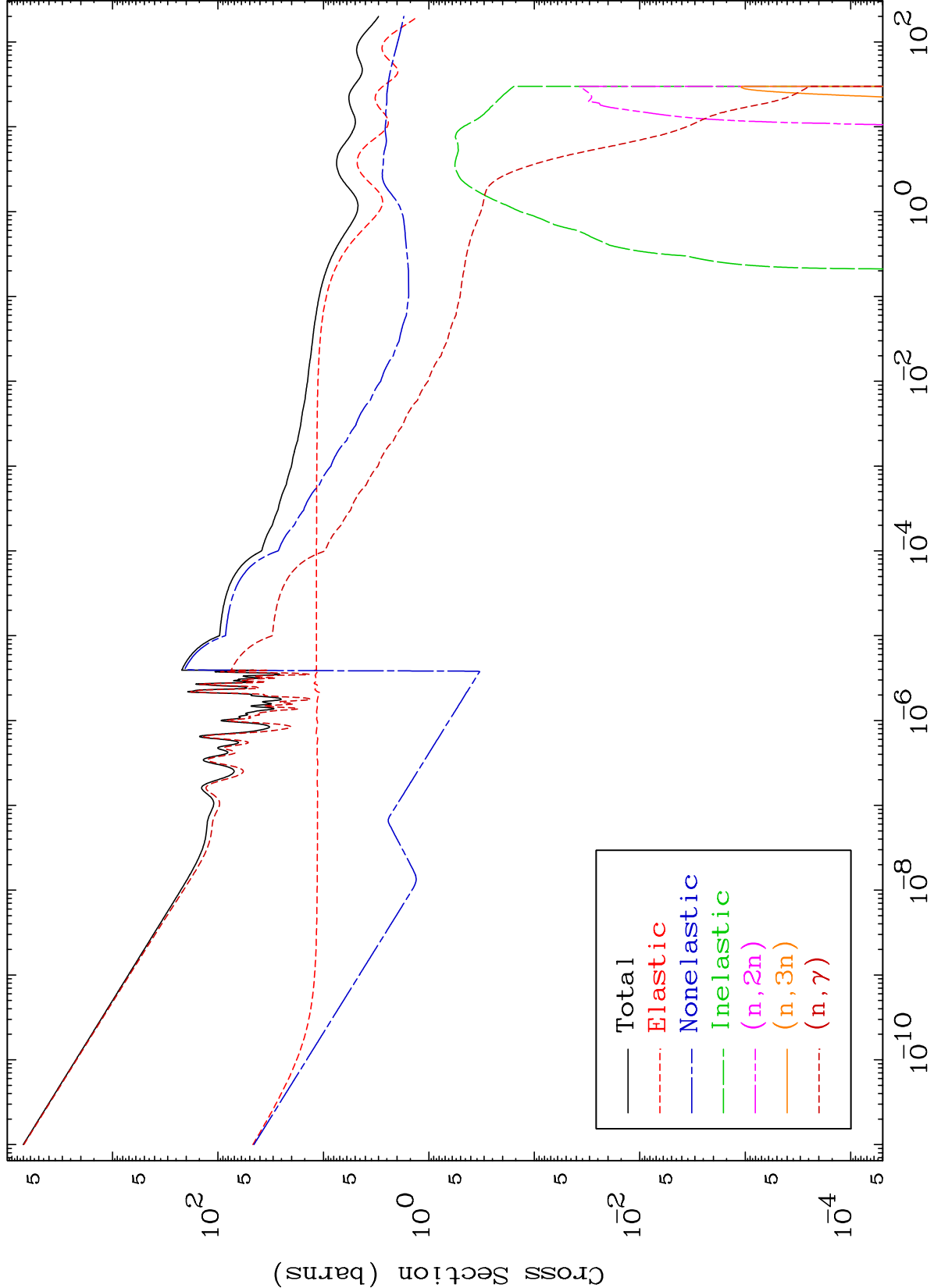
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8704

Neutron Major
293 Kelvin Cross Sections

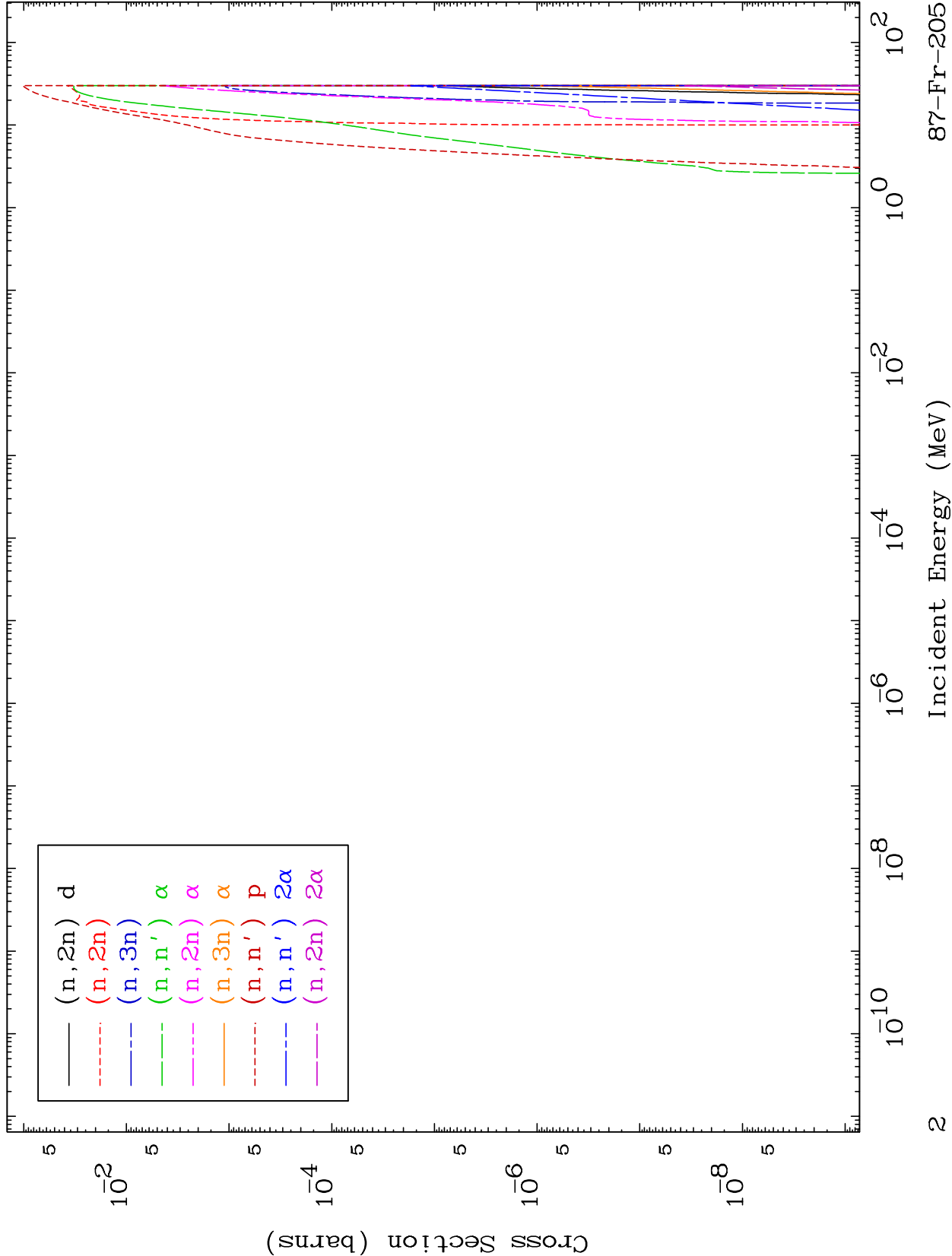
87-Fr-205

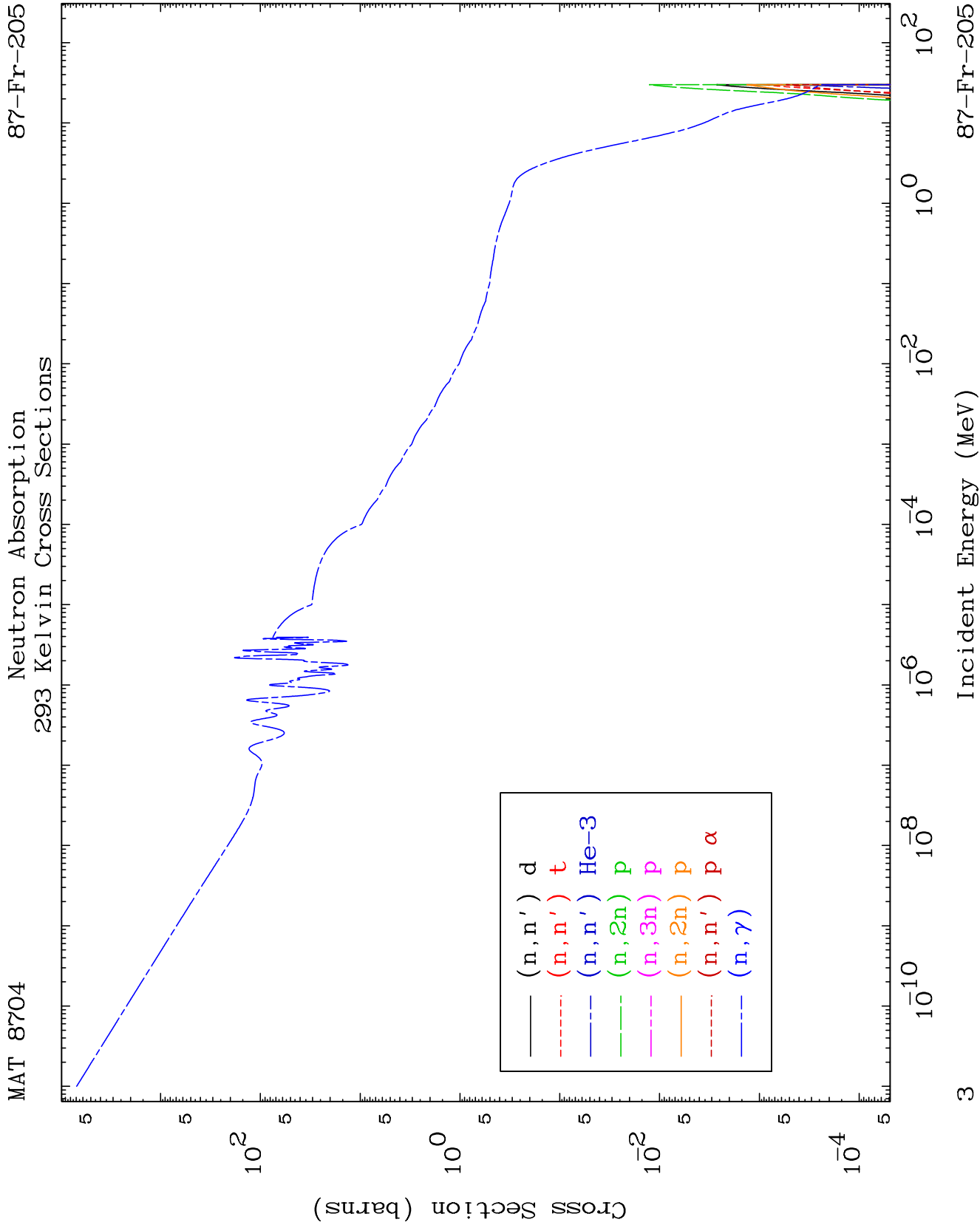


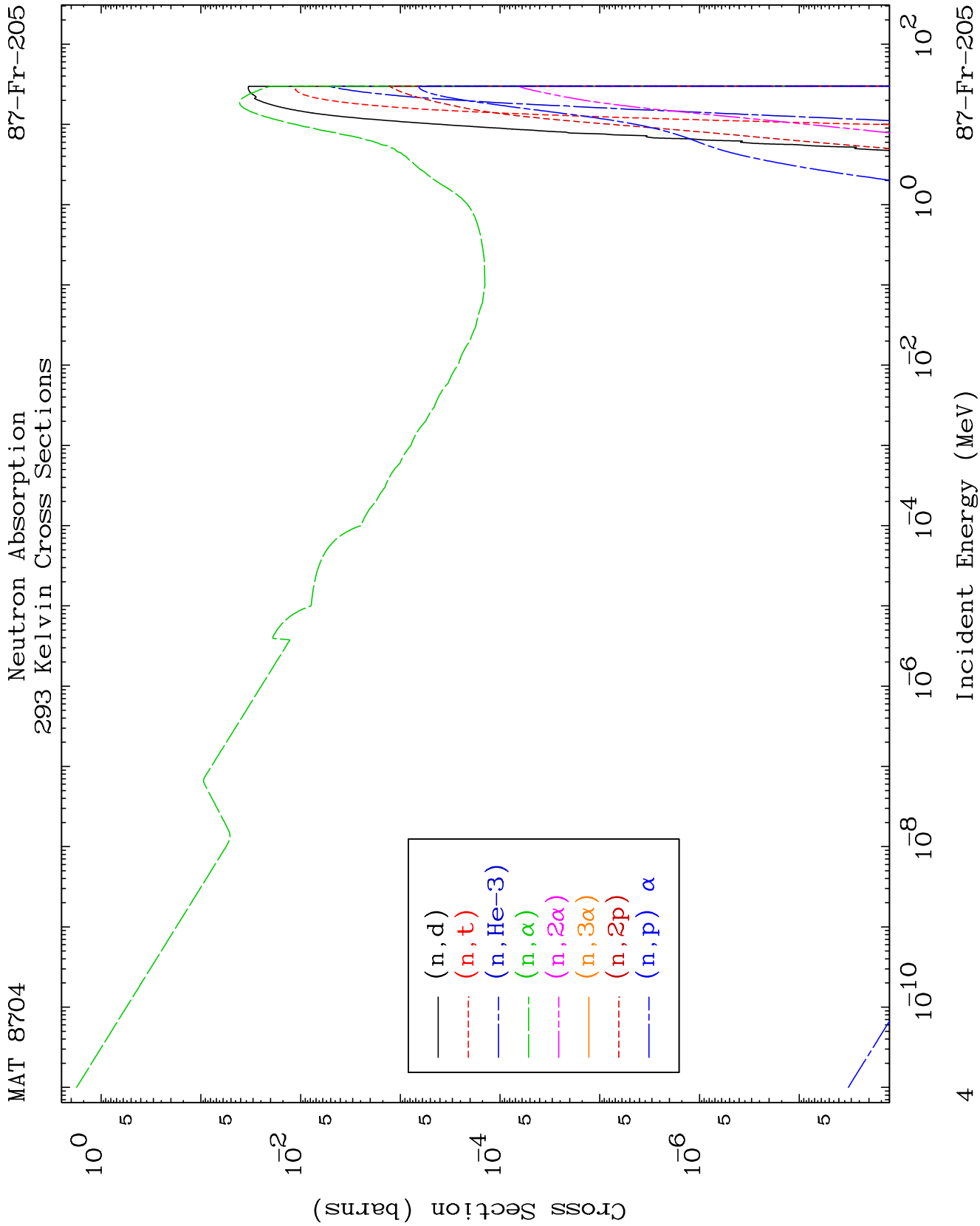
MAT 8704

Neutron Absorption
293 Kelvin Cross Sections

87-Fr-205



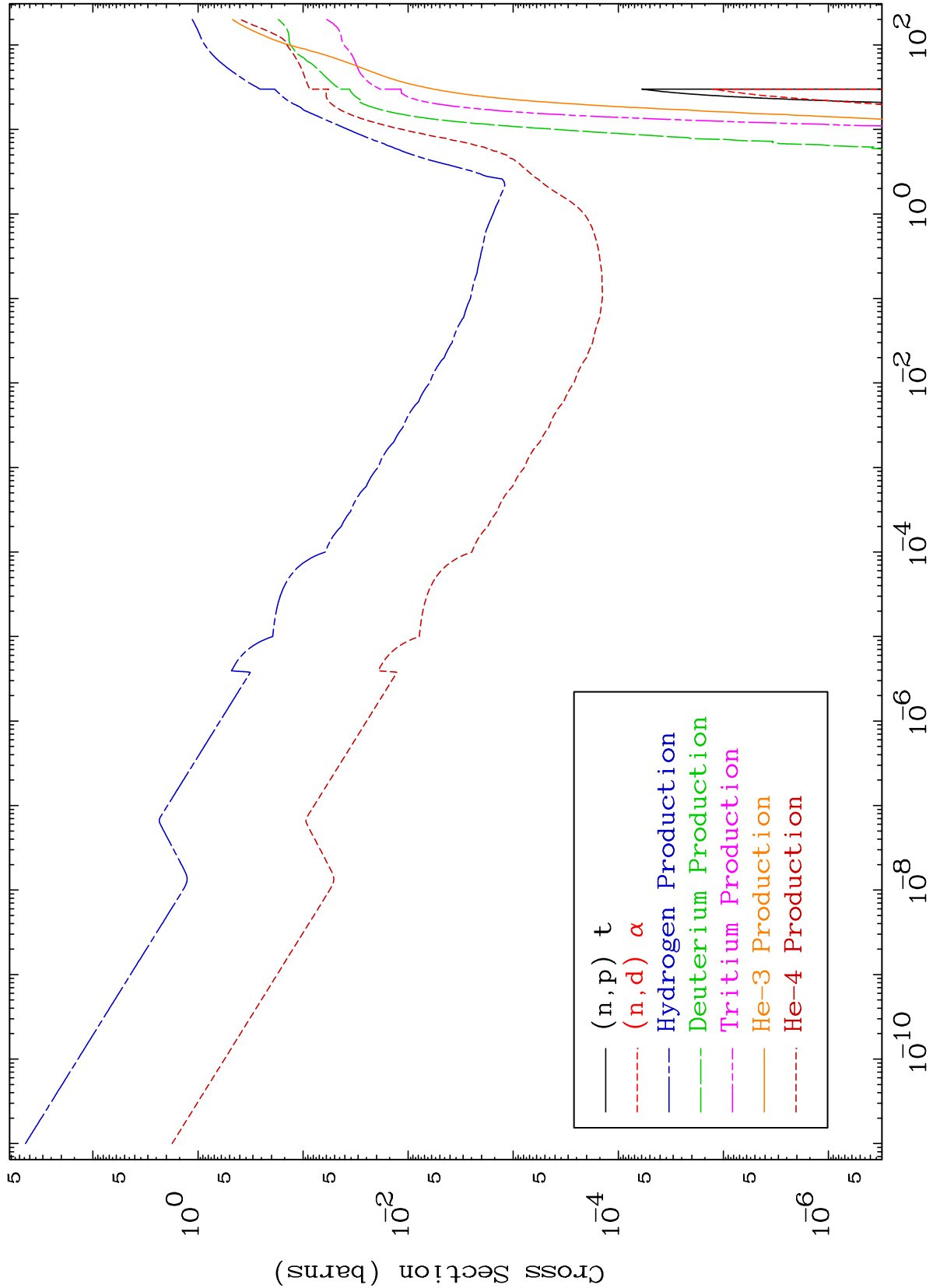




MAT 8704

Neutron Absorption
293 Kelvin Cross Sections

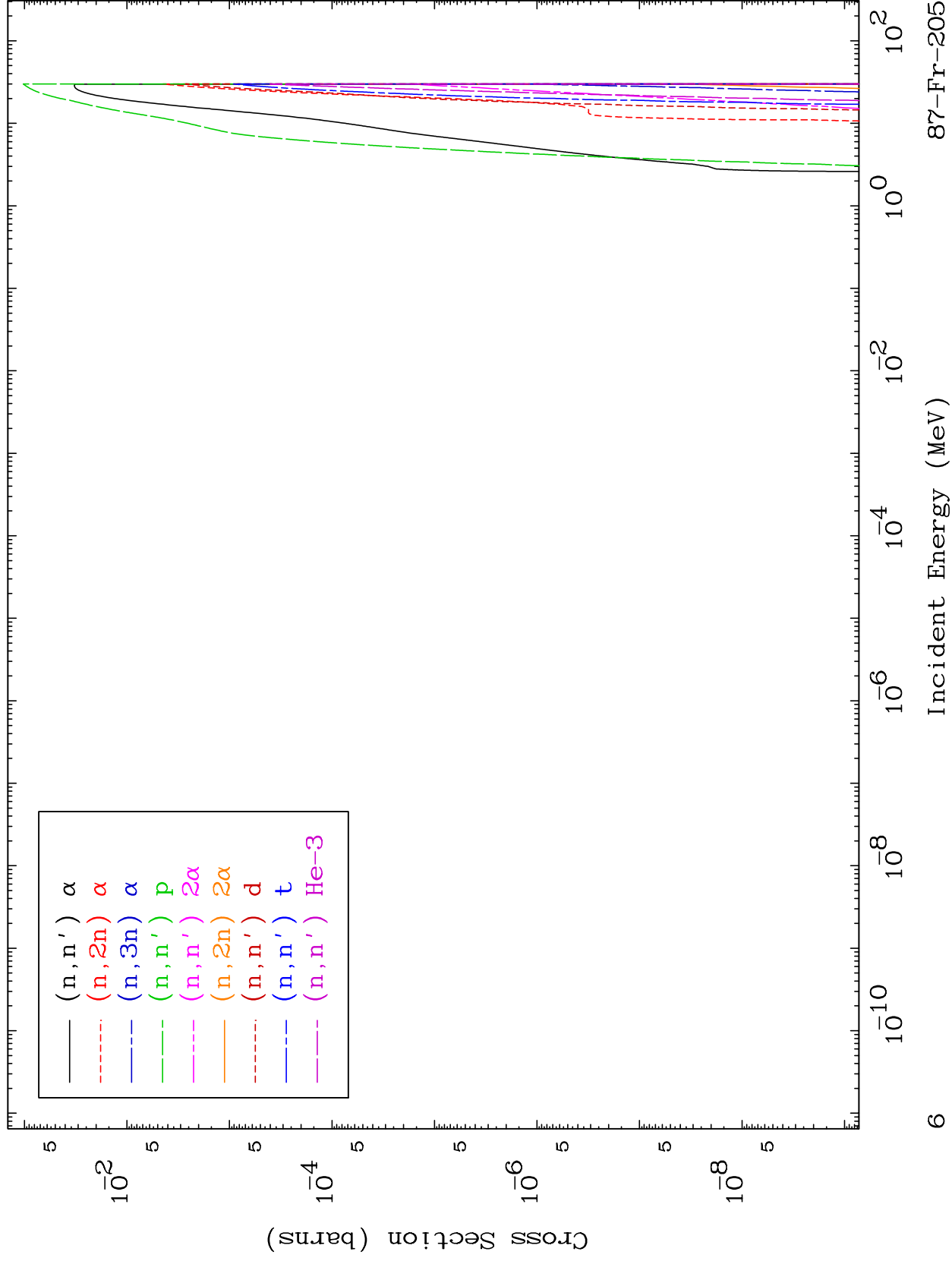
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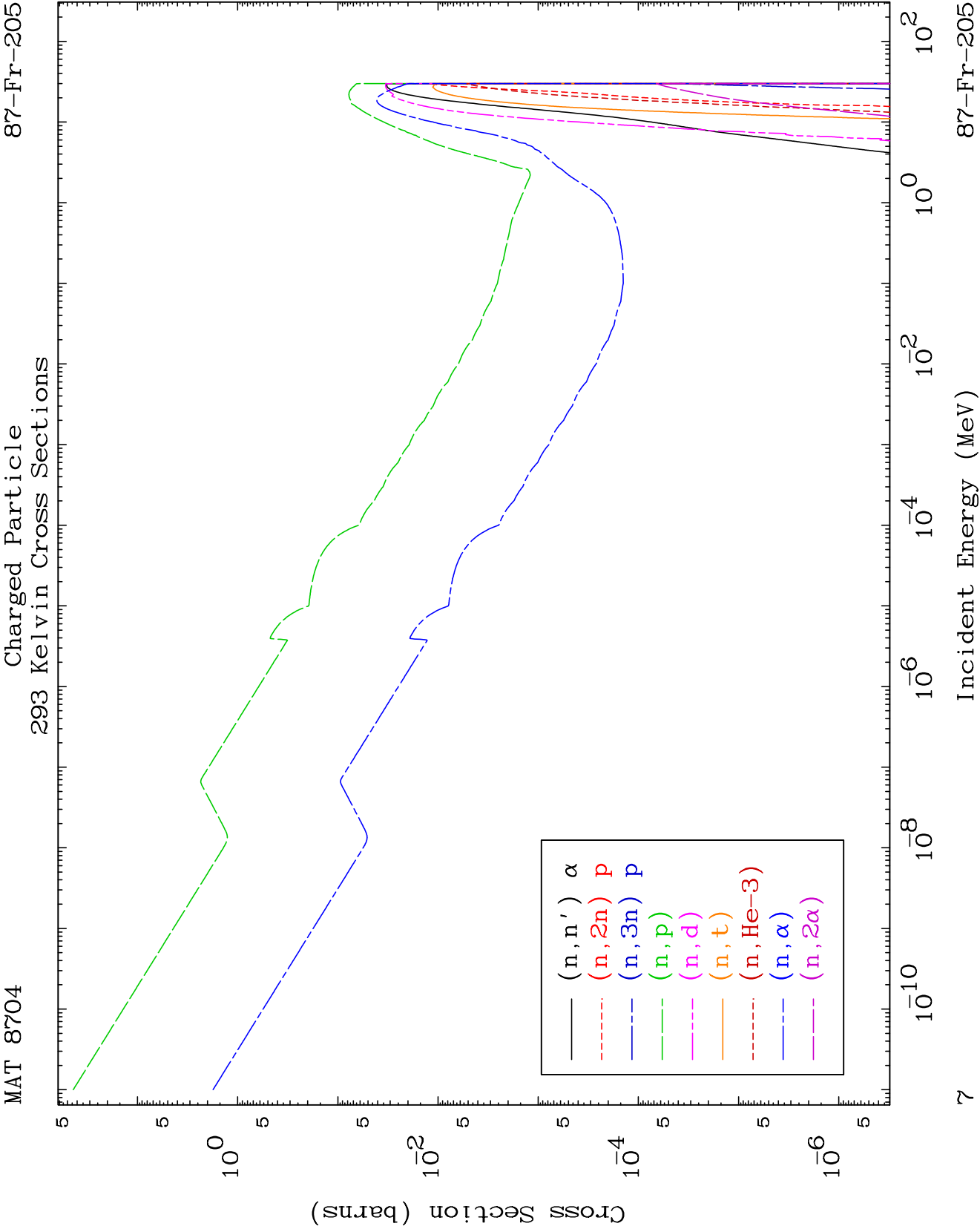


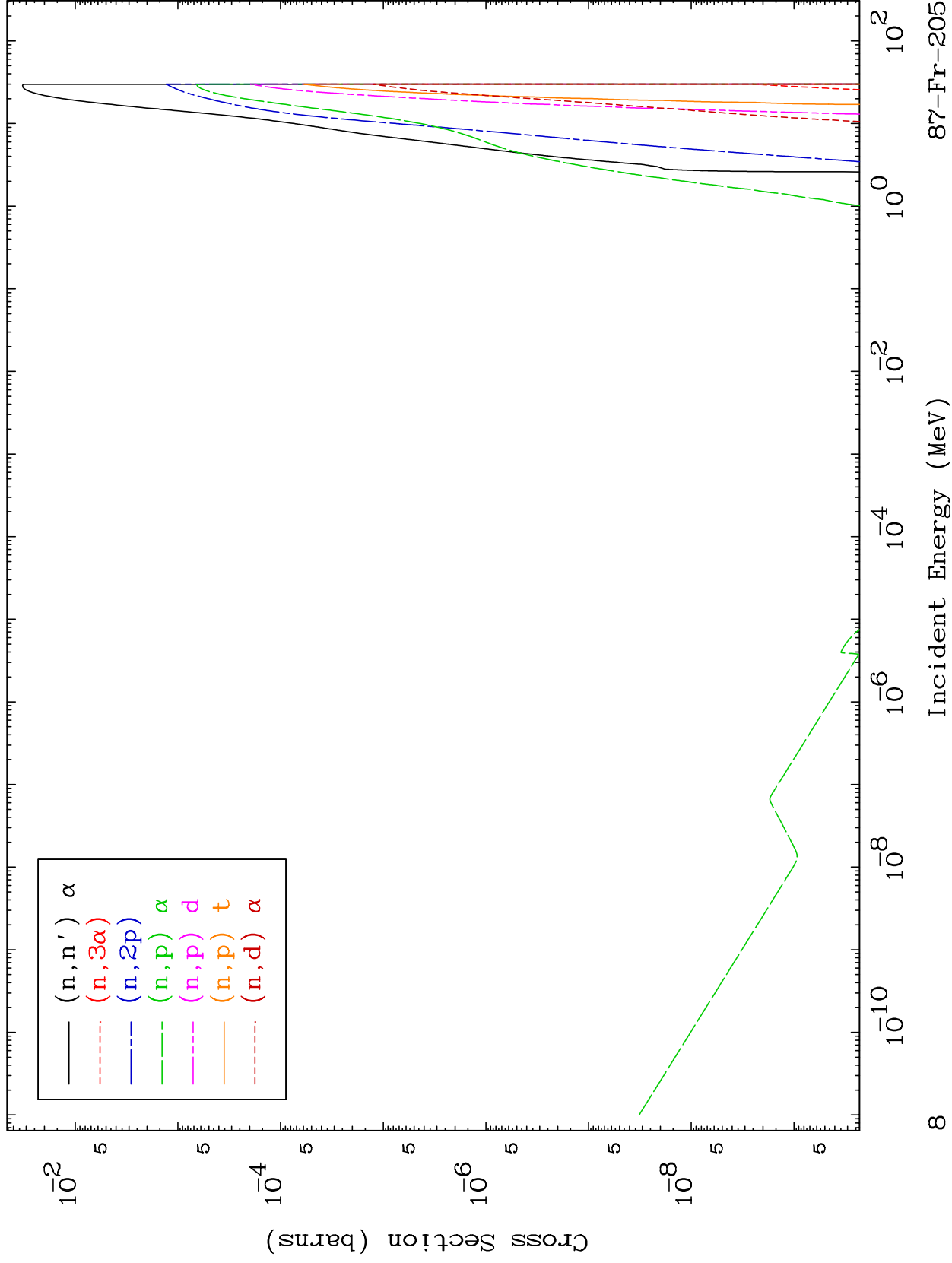
MAT 8704

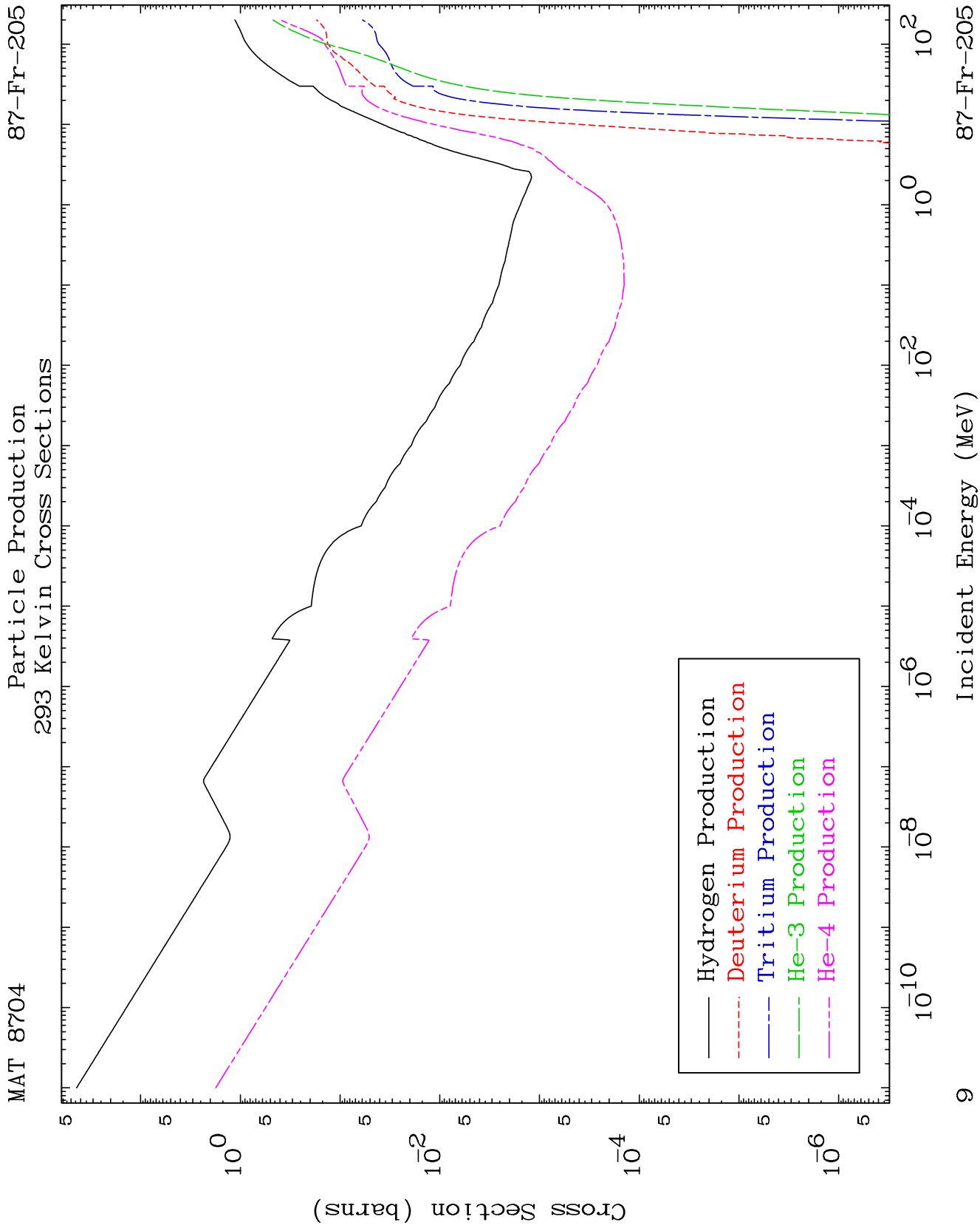
Charged Particle
293 Kelvin Cross Sections

87-Fr-205







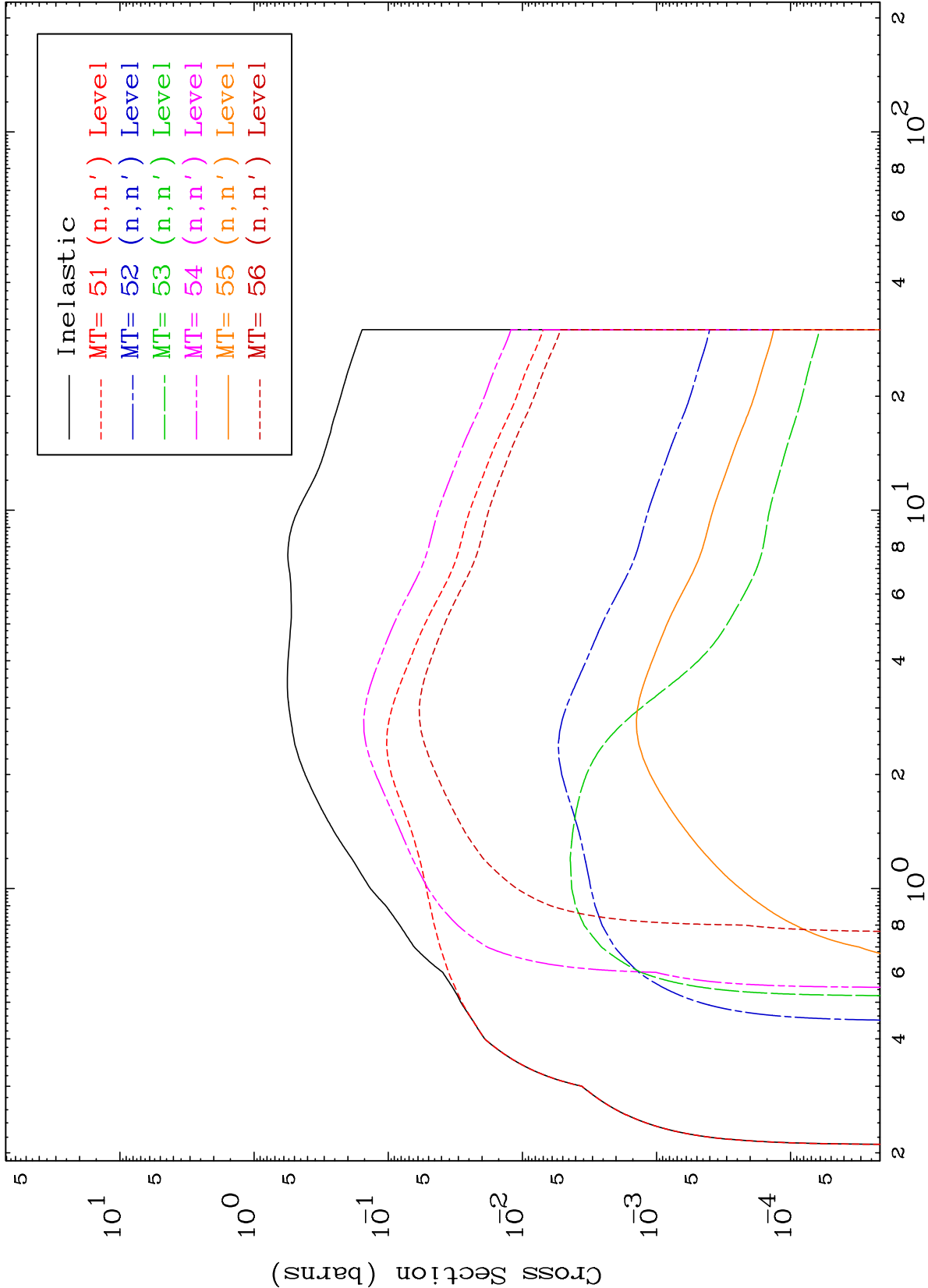


MAT 8704

(n,n') Levels

293 Kelvin Cross Sections

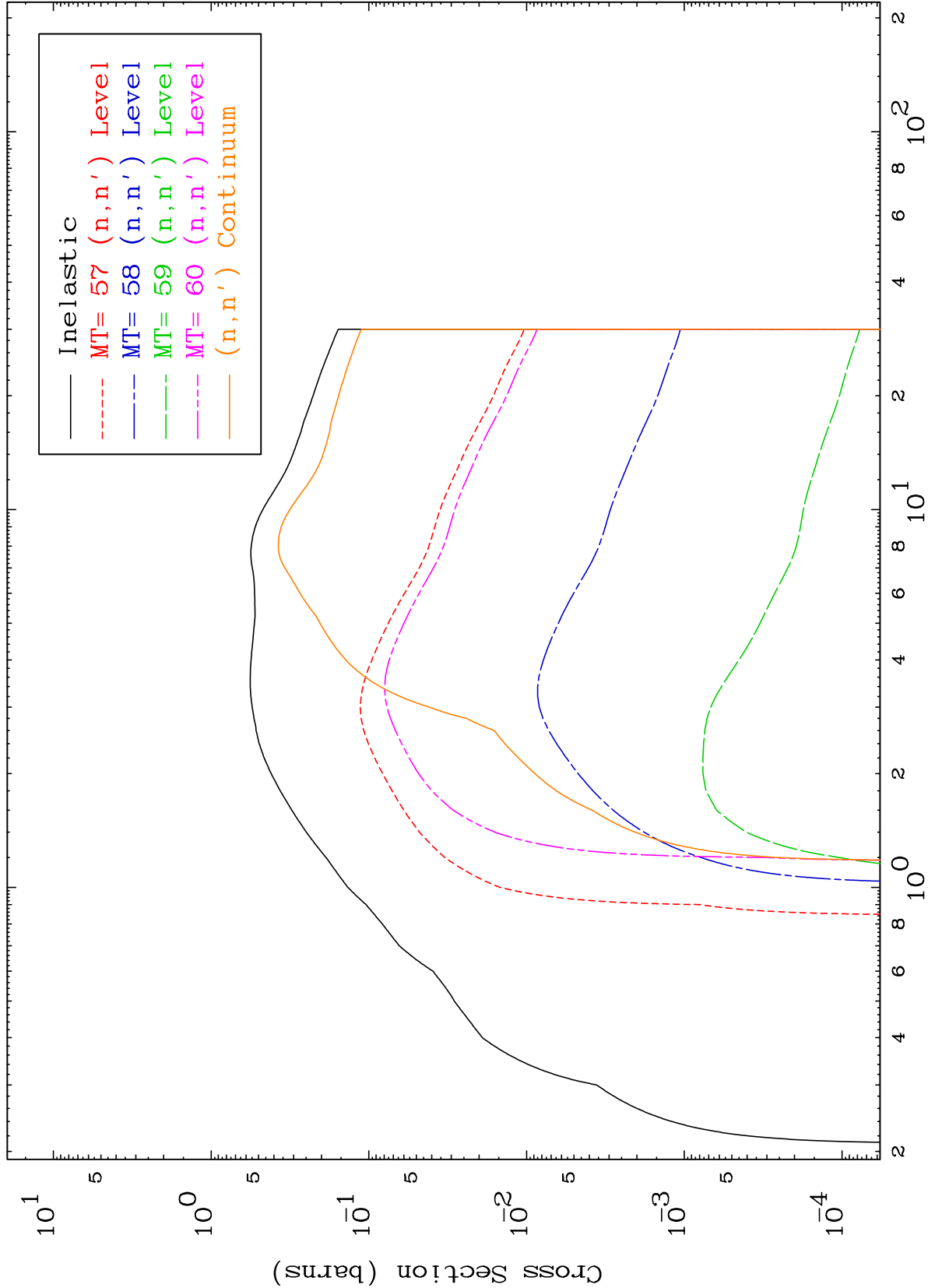
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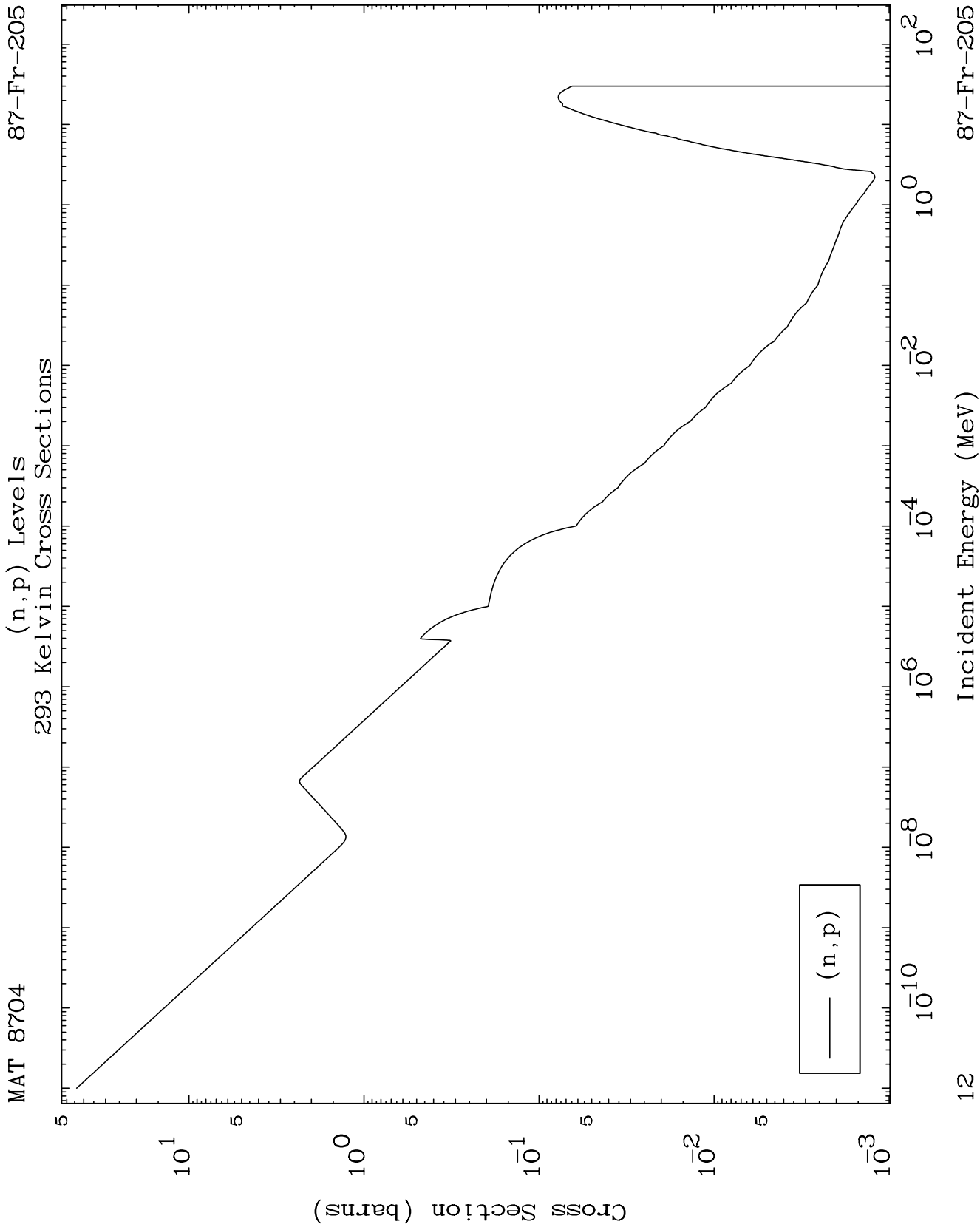


10

Incident Energy (MeV)

87-Fr-205

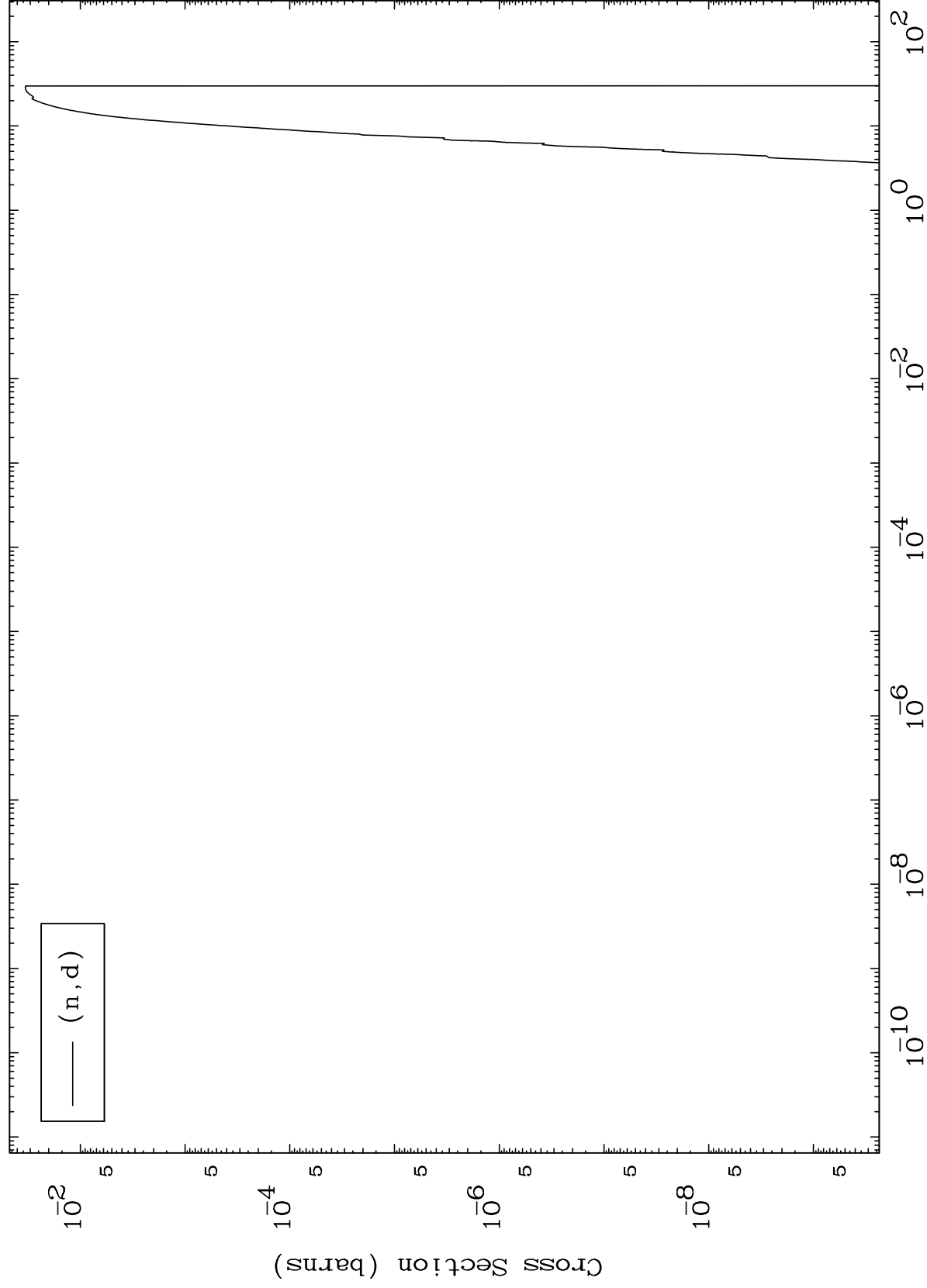




MAT 8704

(n,d) Levels
293 Kelvin Cross Sections

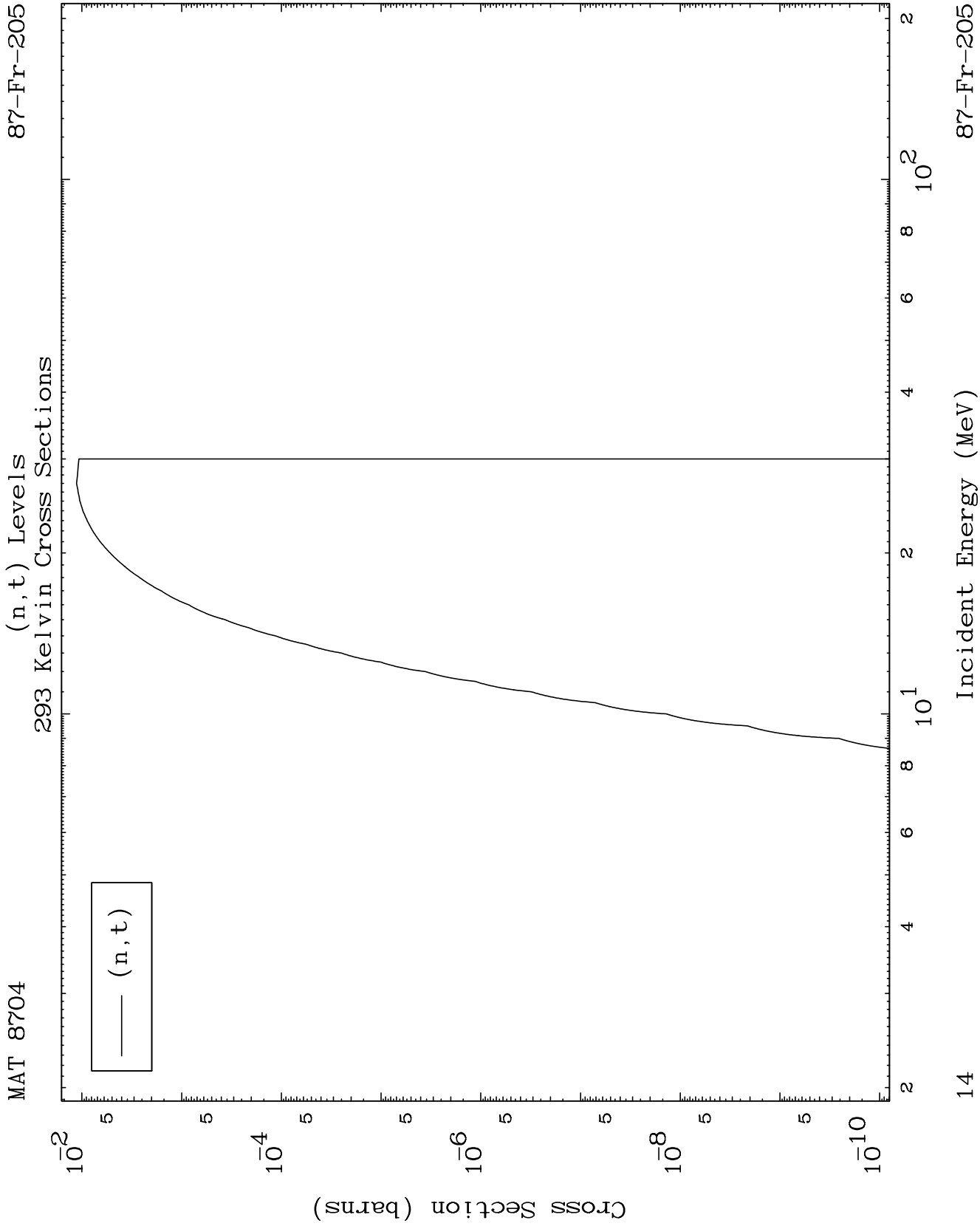
87-Fr-205



13

Incident Energy (MeV)

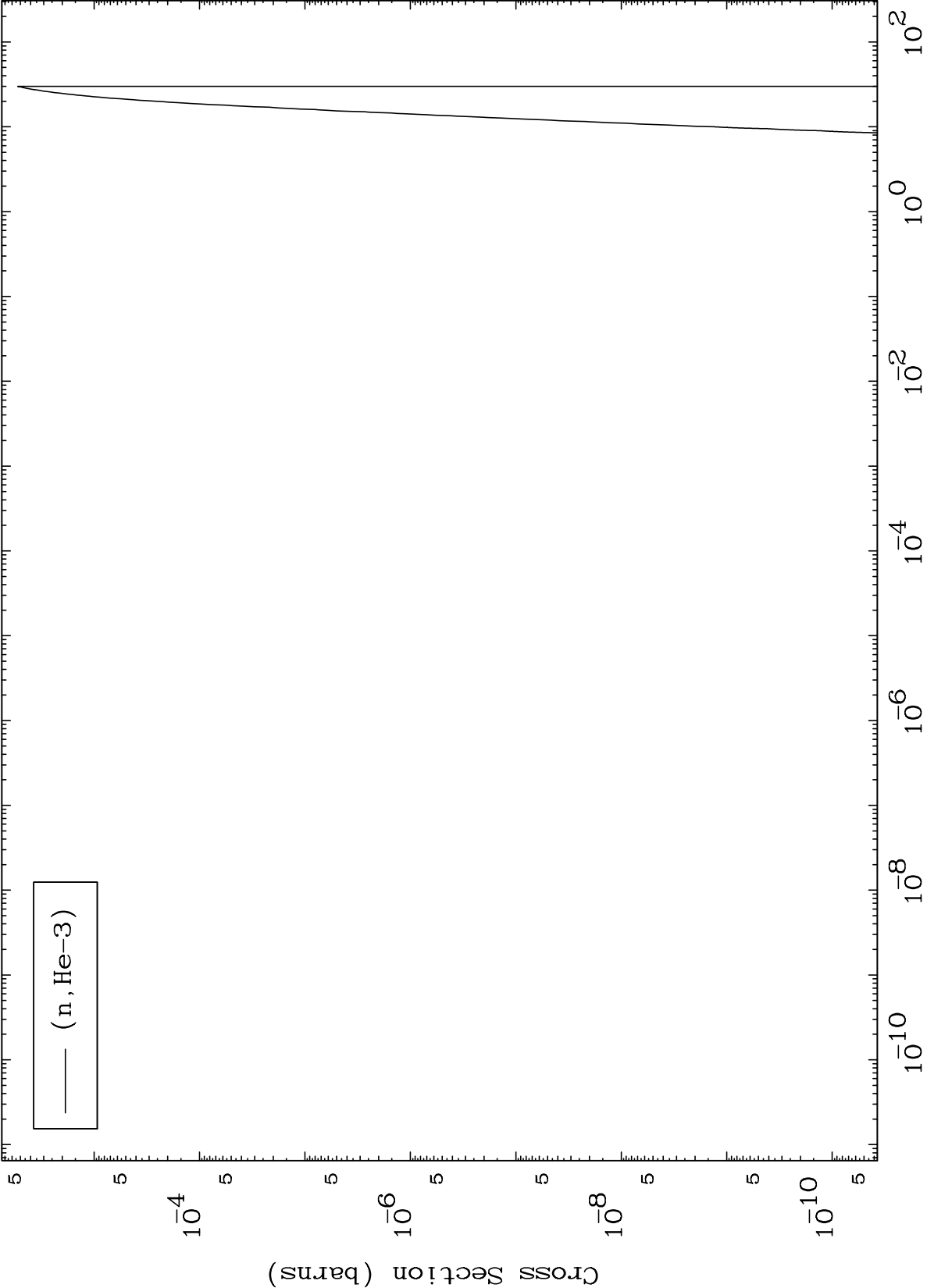
87-Fr-205



MAT 8704

(n,He3) Levels
293 Kelvin Cross Sections

87-Fr-205



15

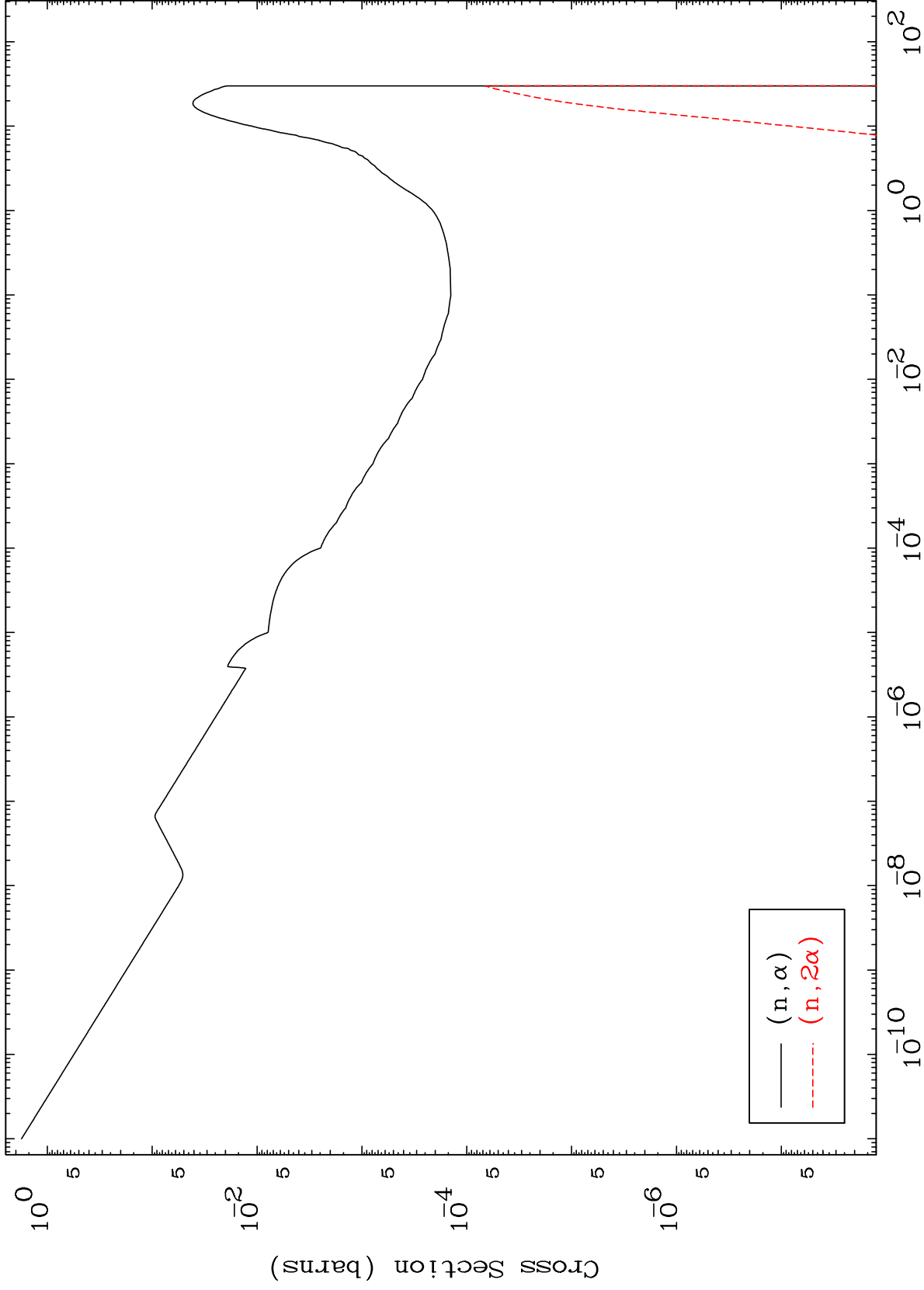
Incident Energy (MeV)

87-Fr-205

MAT 8704

87-Fr-205

(n, α) Levels
293 Kelvin Cross Sections



87-Fr-205

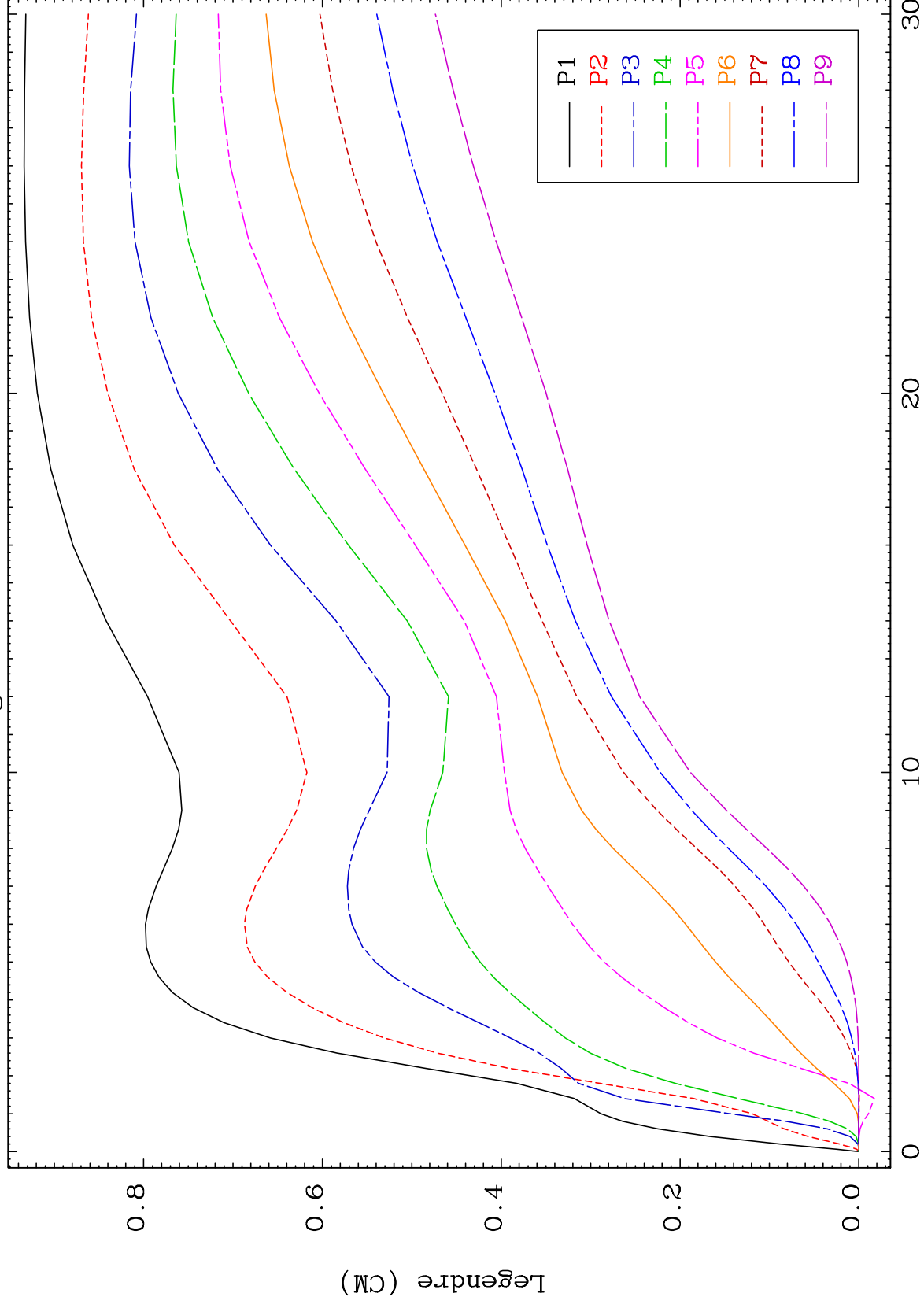
Incident Energy (MeV)

16

MAT 8704

87-Fr-205

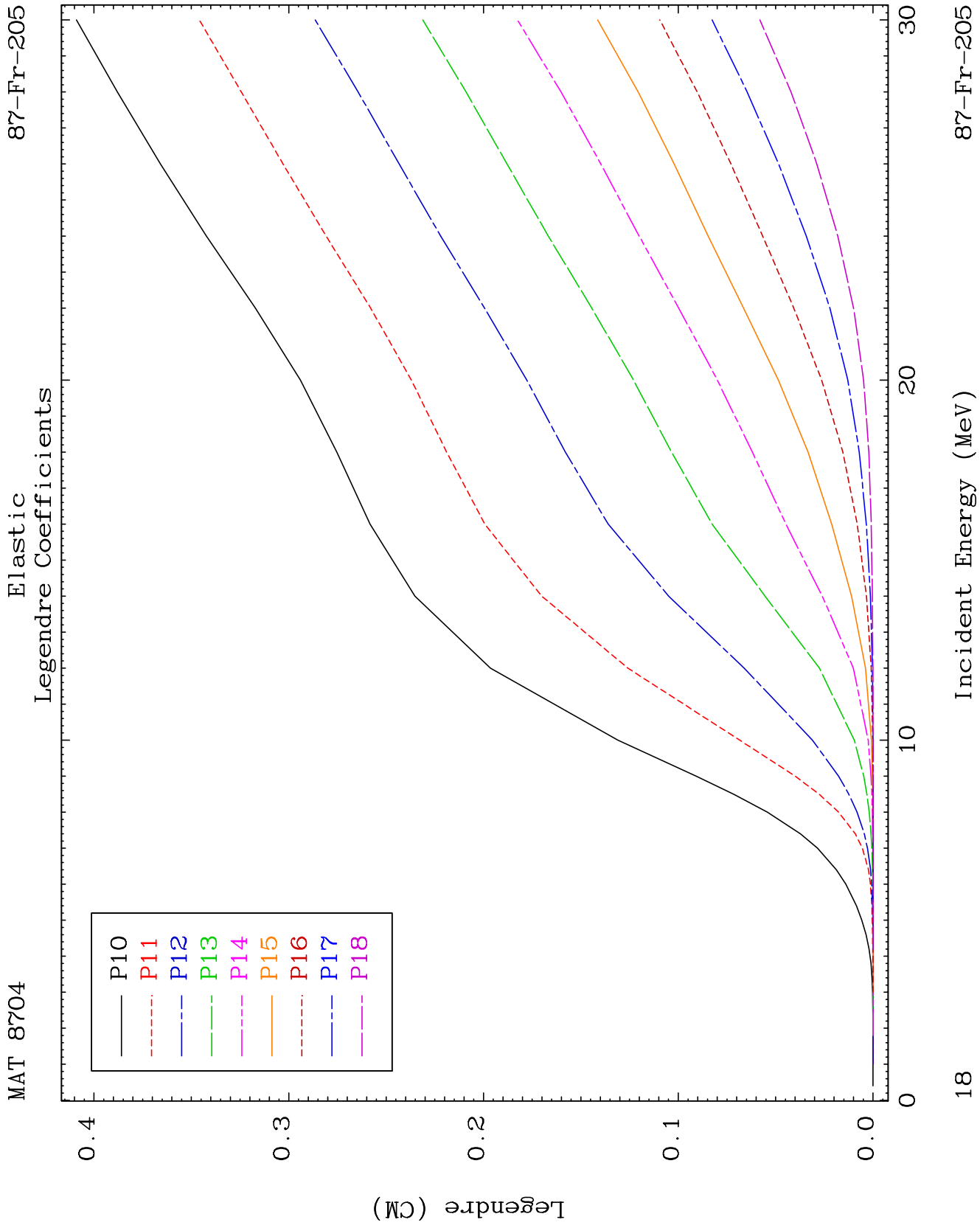
Elastic Legendre Coefficients

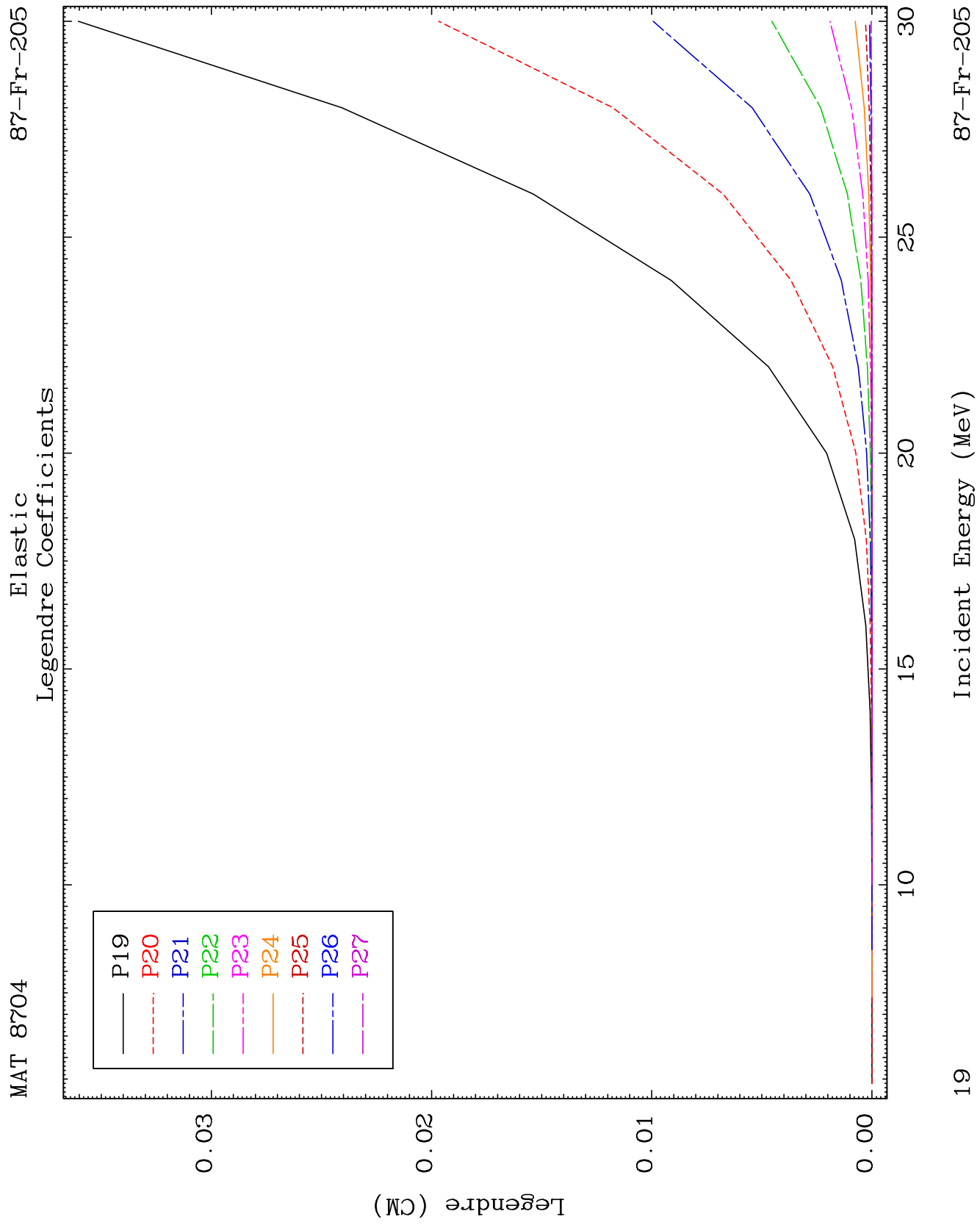


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Incident Energy (MeV)

87-FR-205

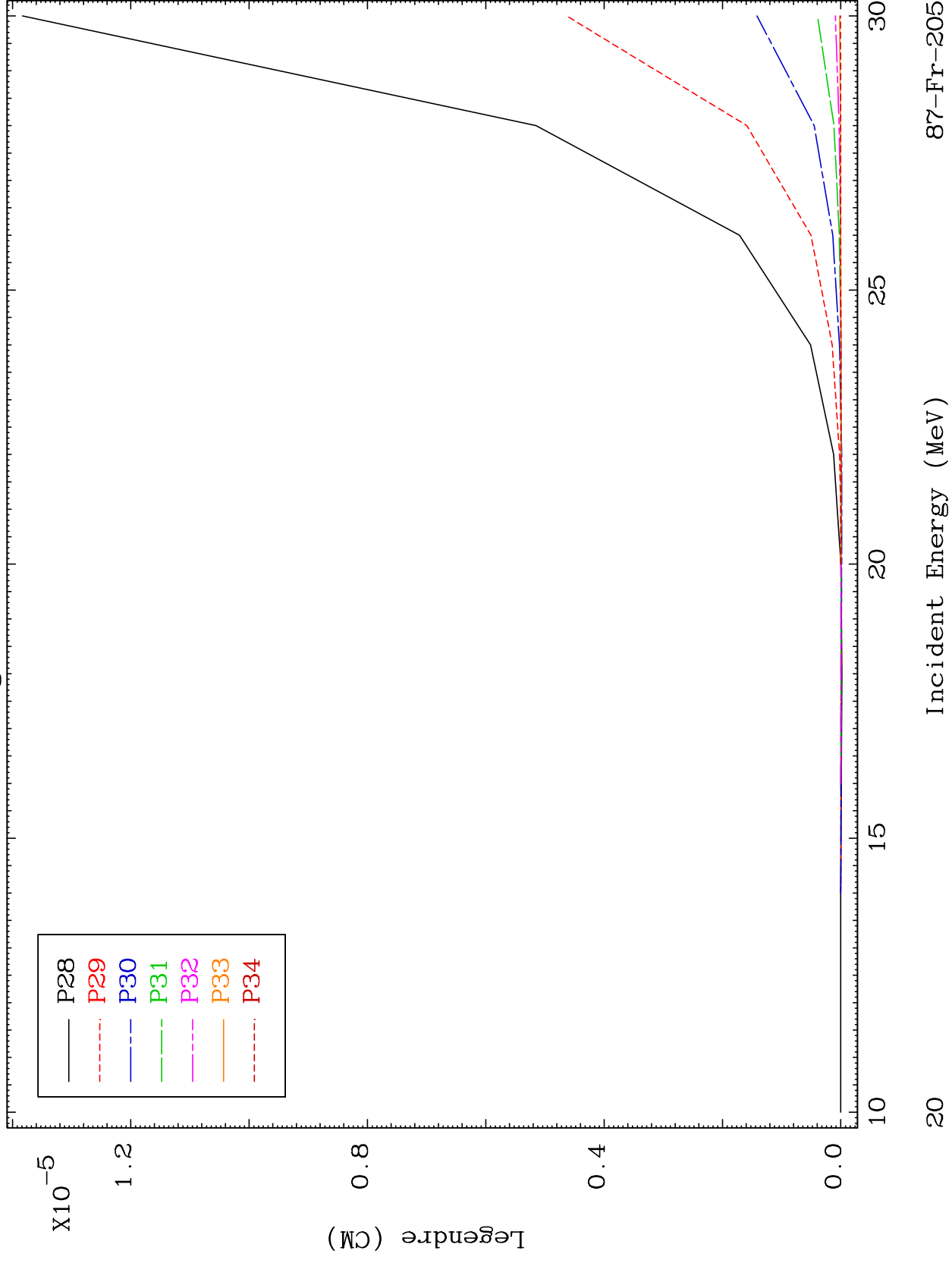


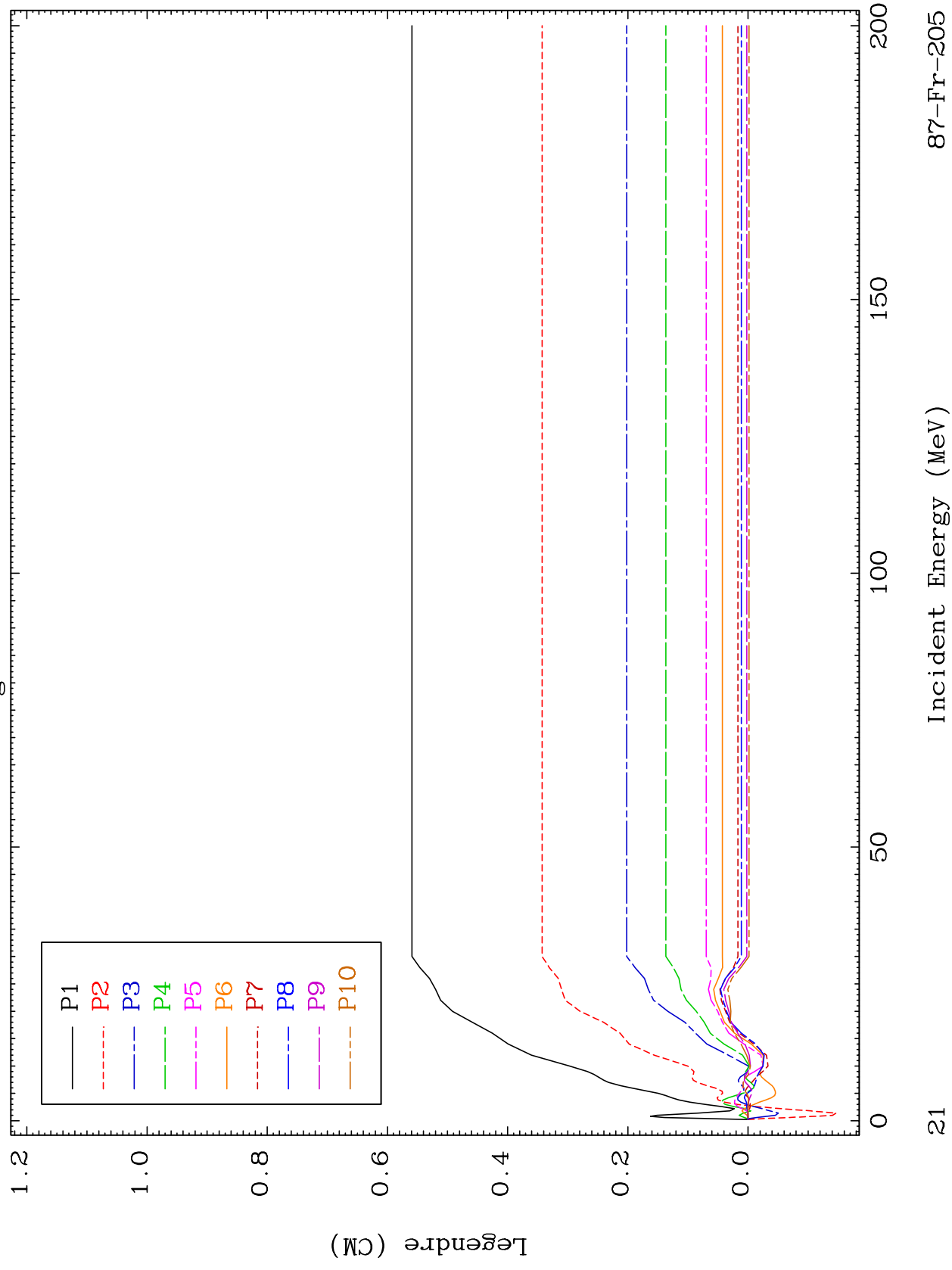


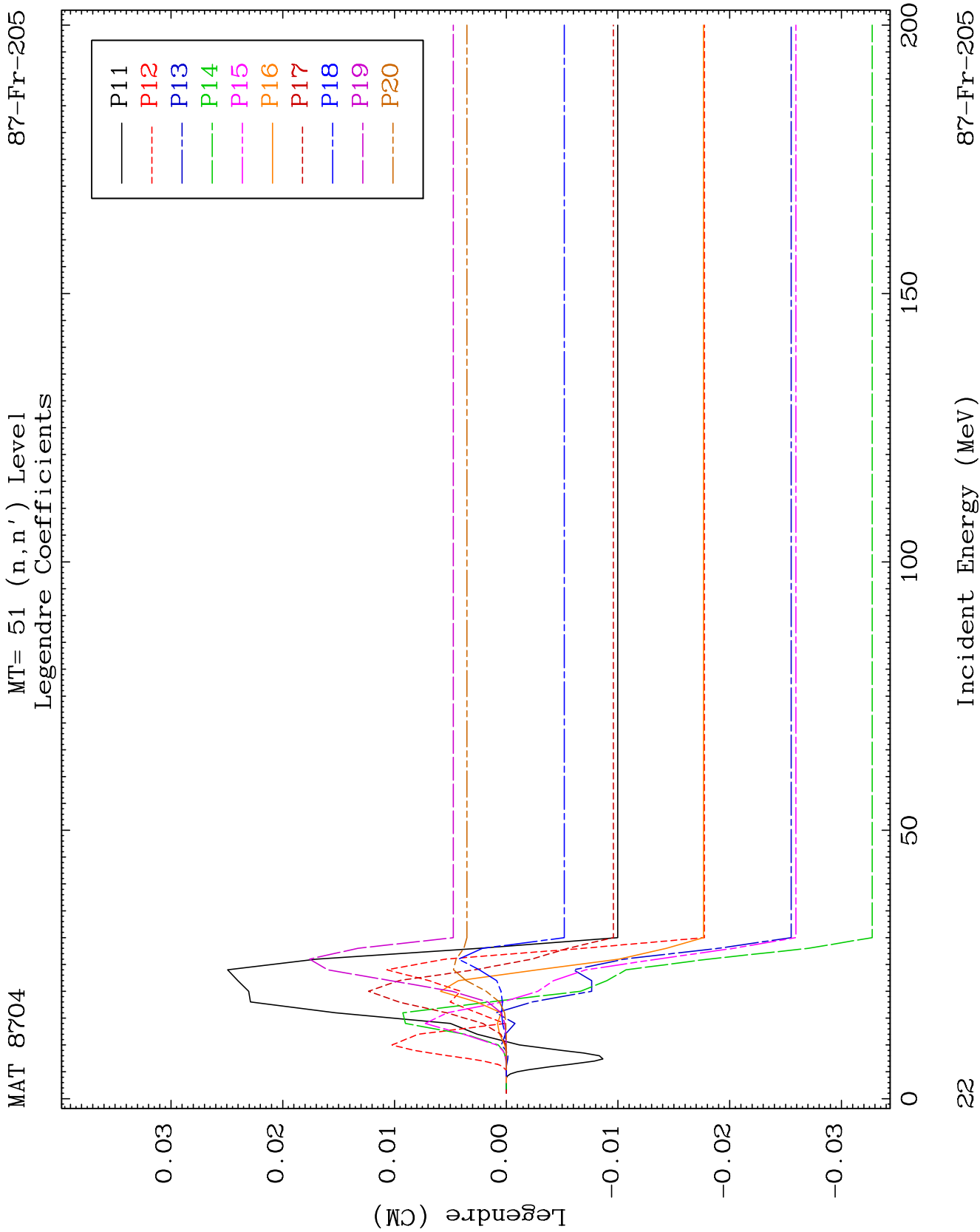
MAT 8704

Elastic Legendre Coefficients

87-Fr-205



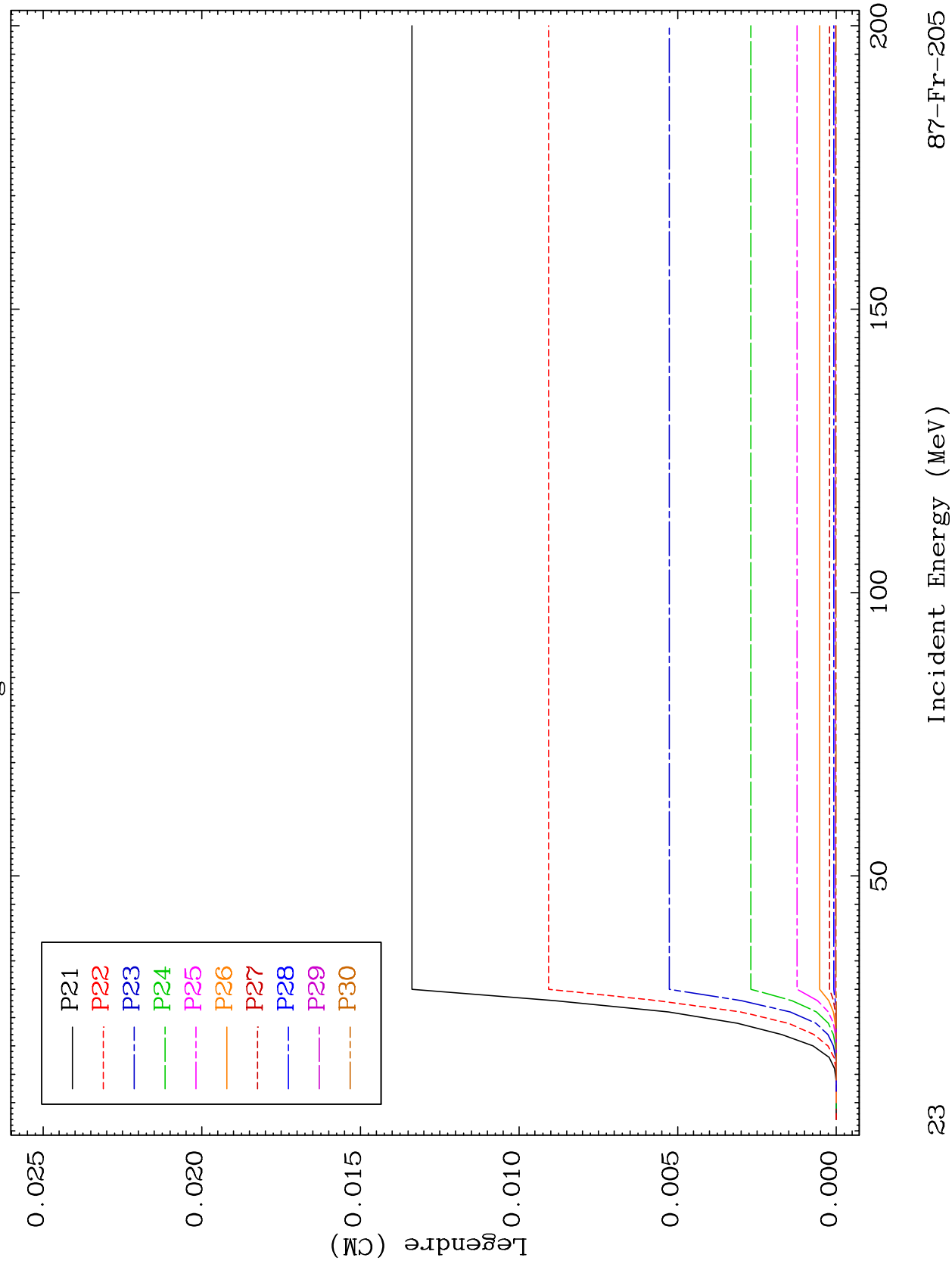




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MT= 51 (n, n') Level
Legendre Coefficients

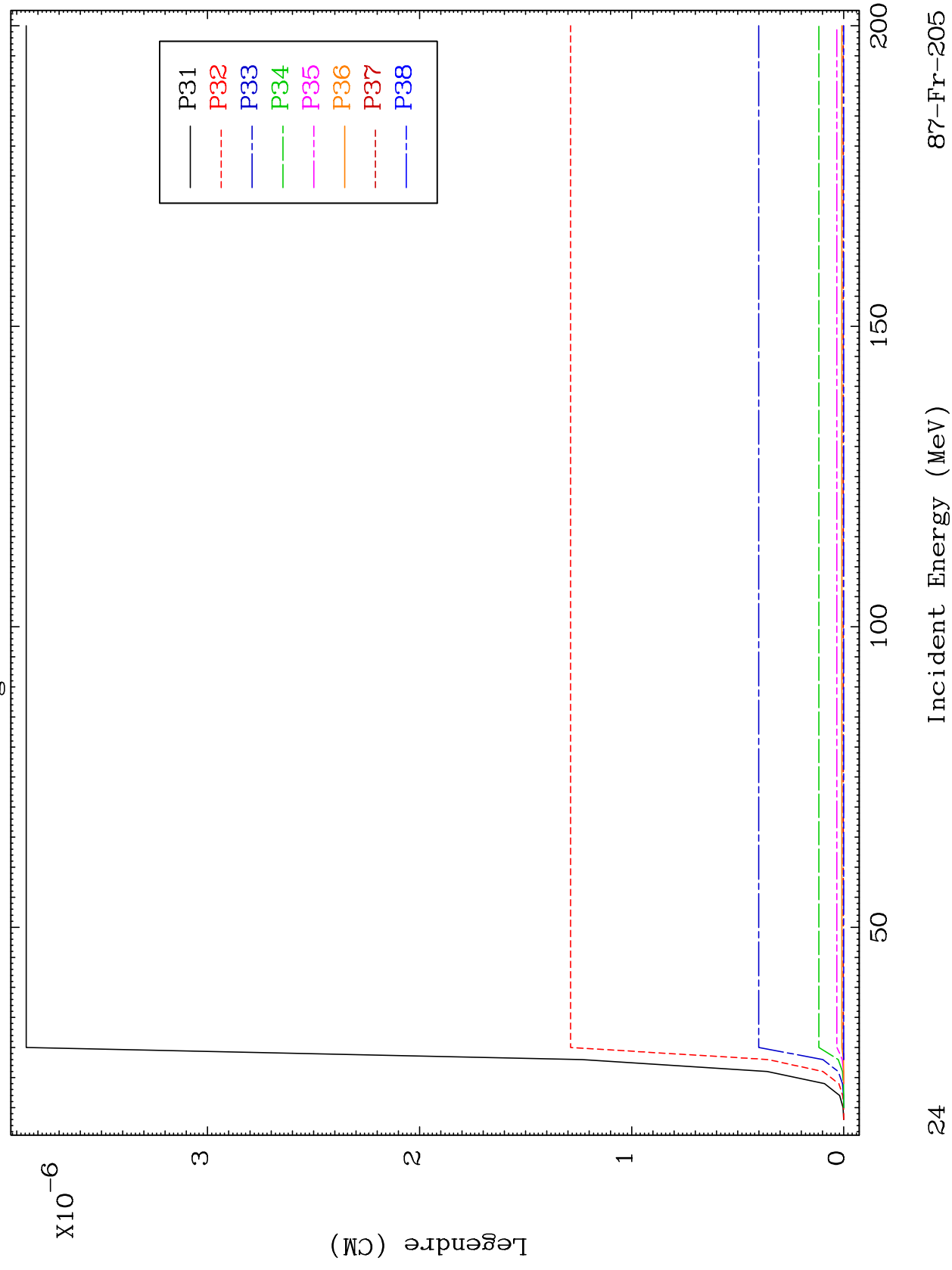
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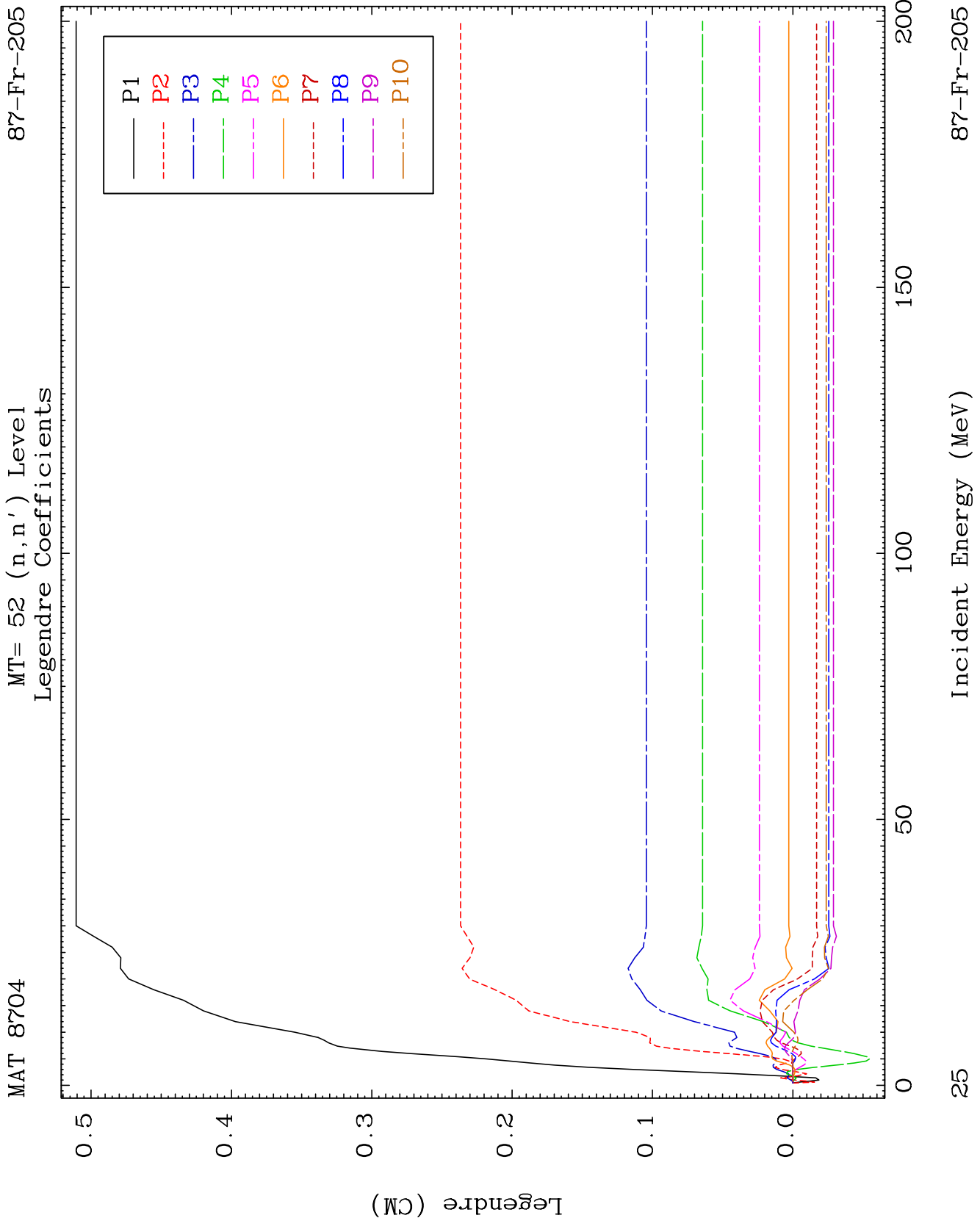


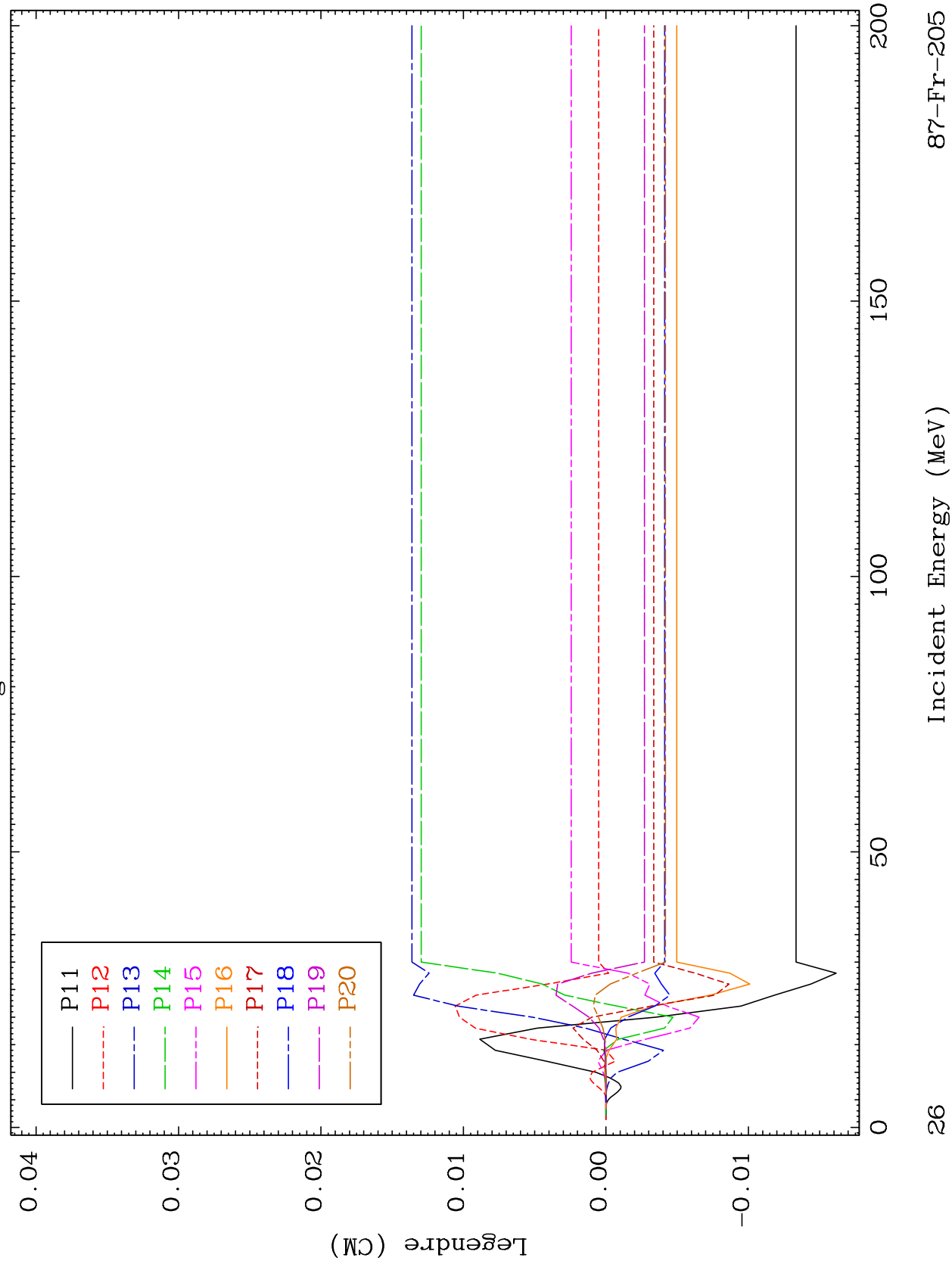
MAT 8704

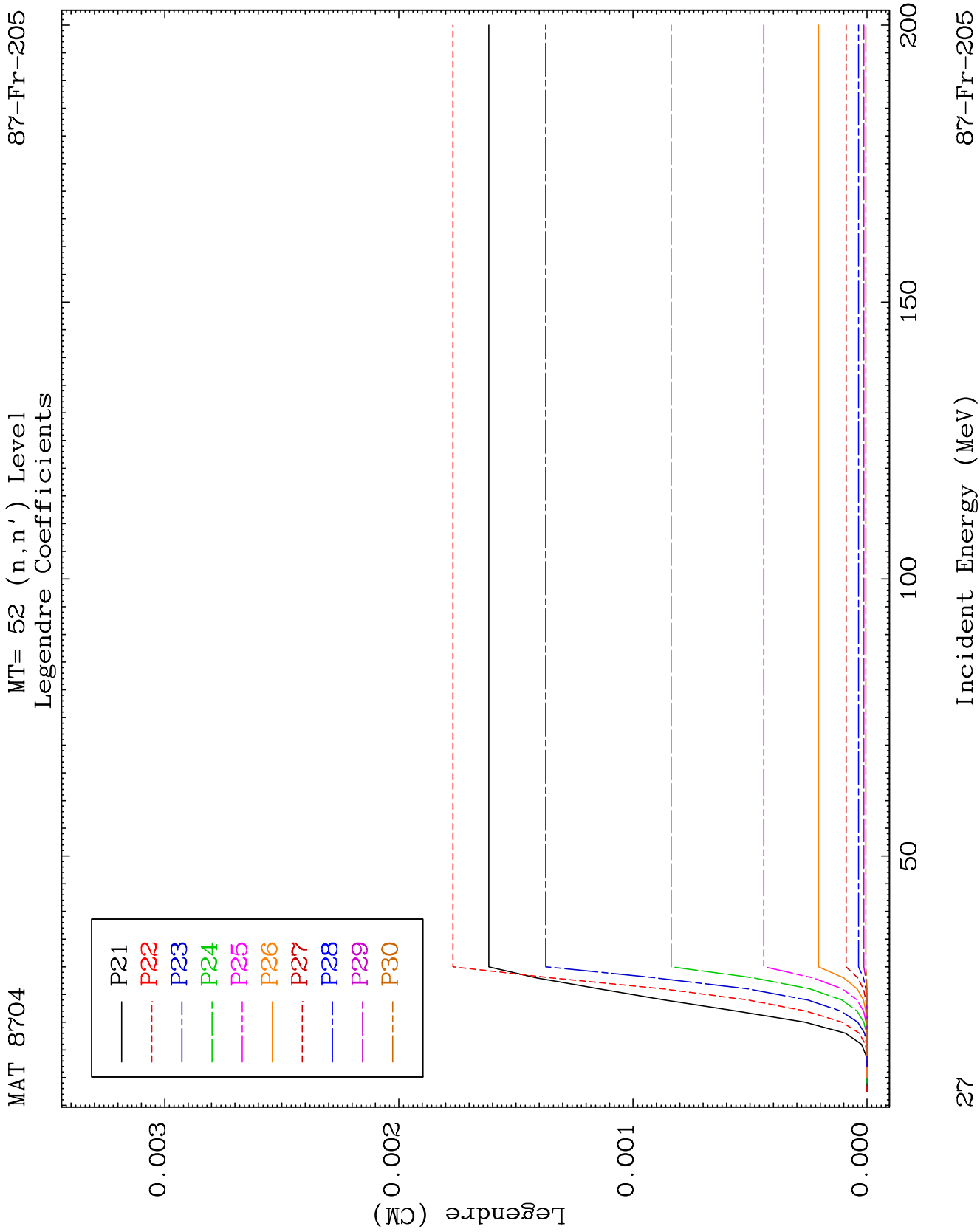
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Legendre Coefficients

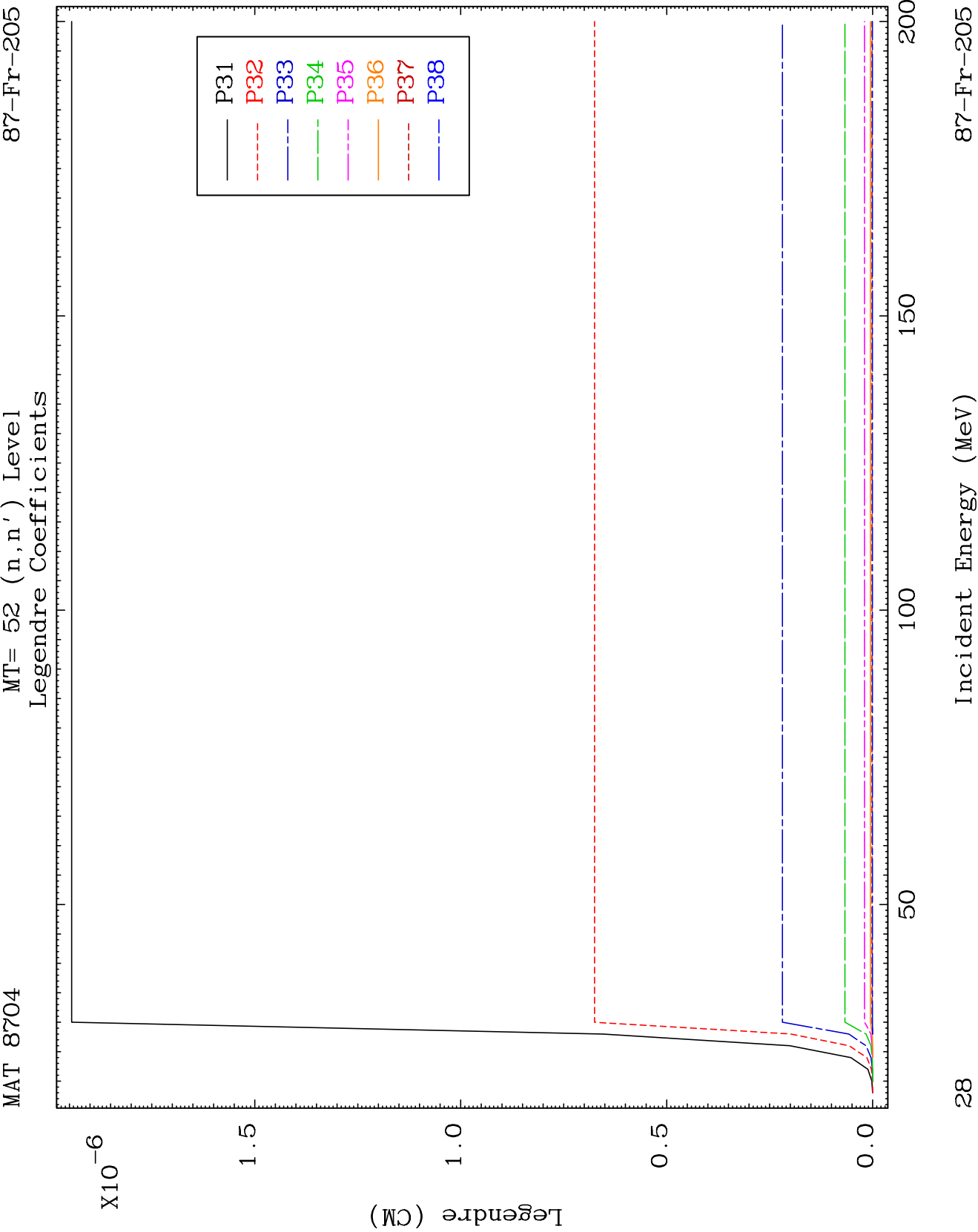
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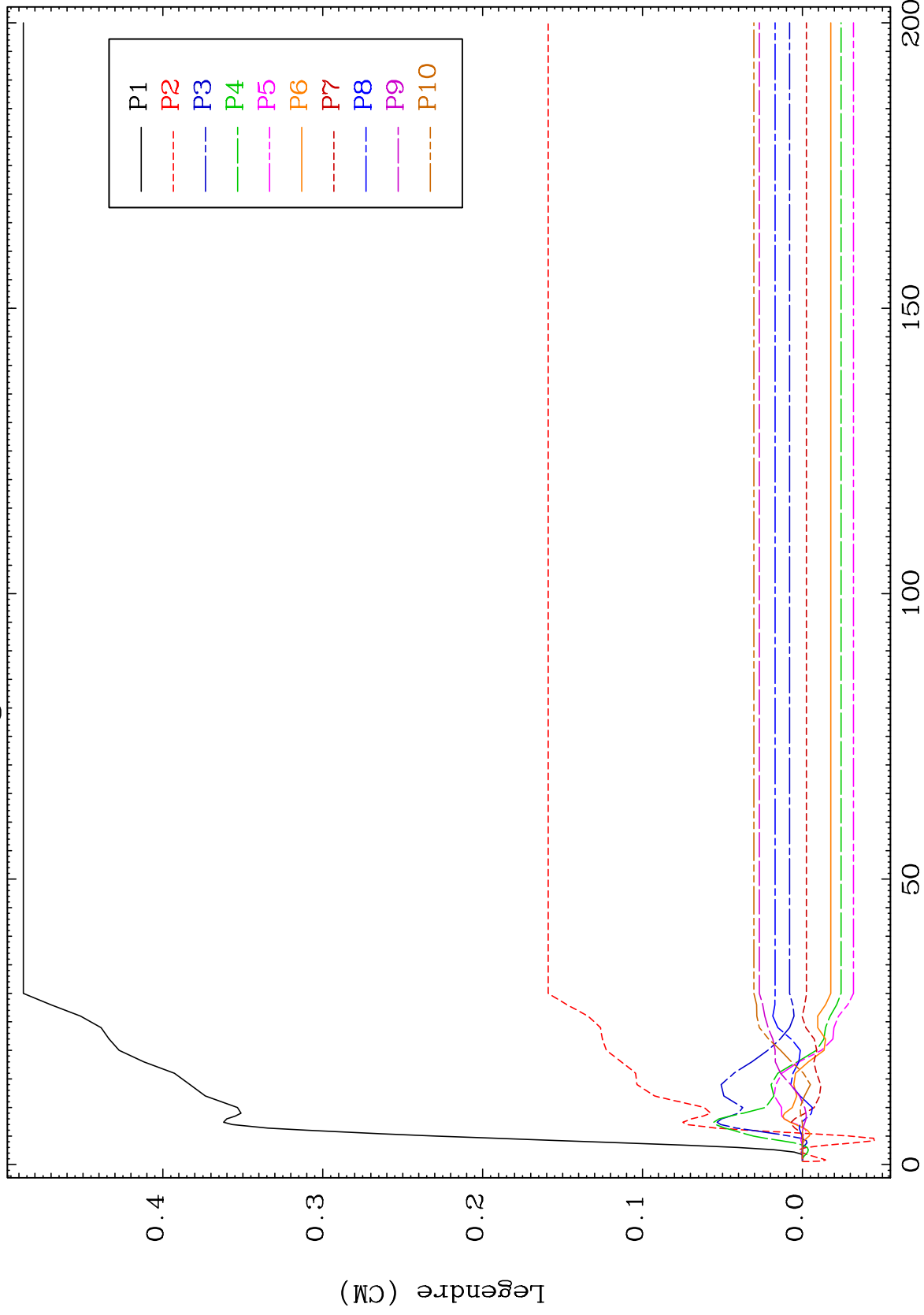








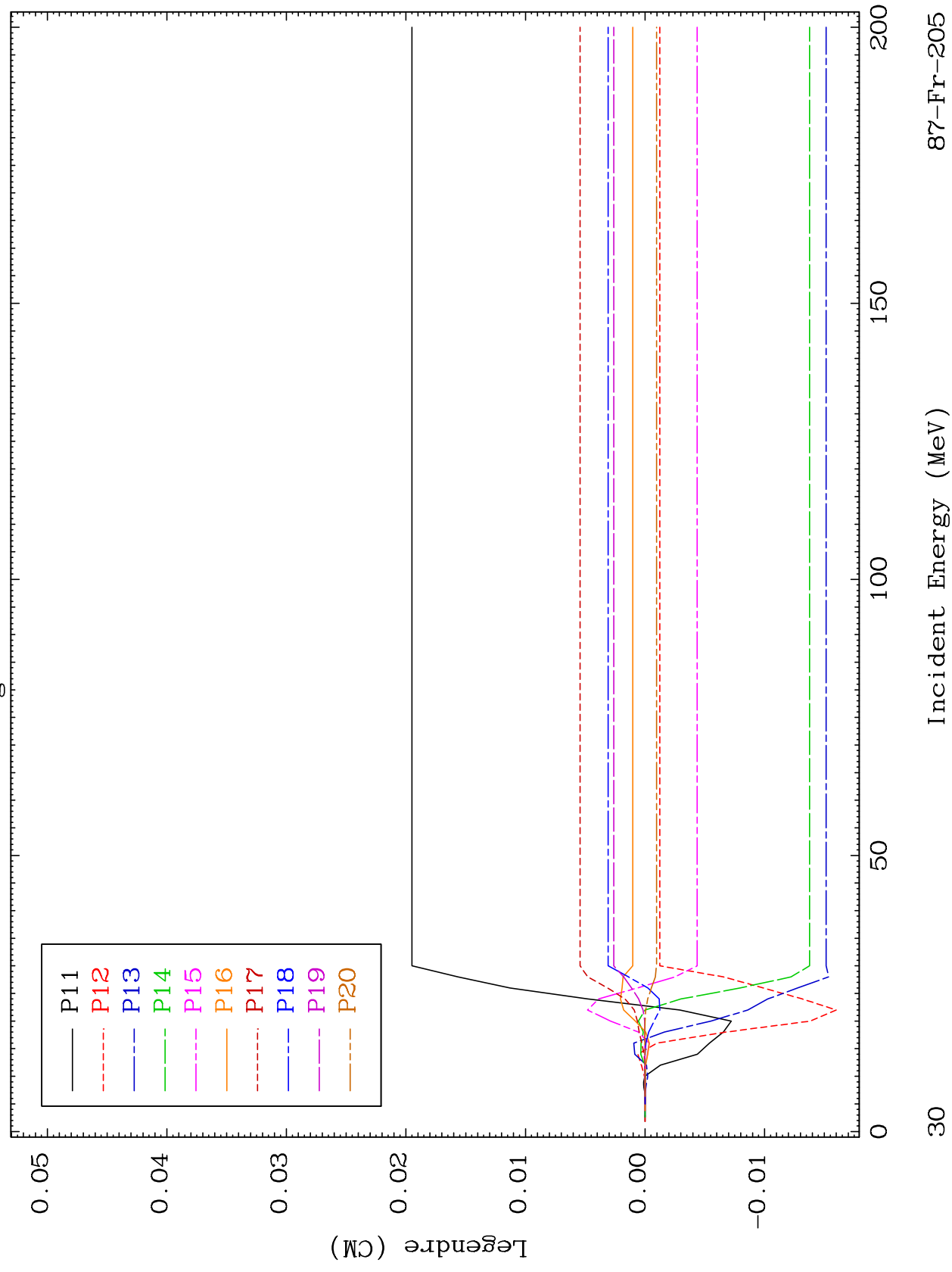


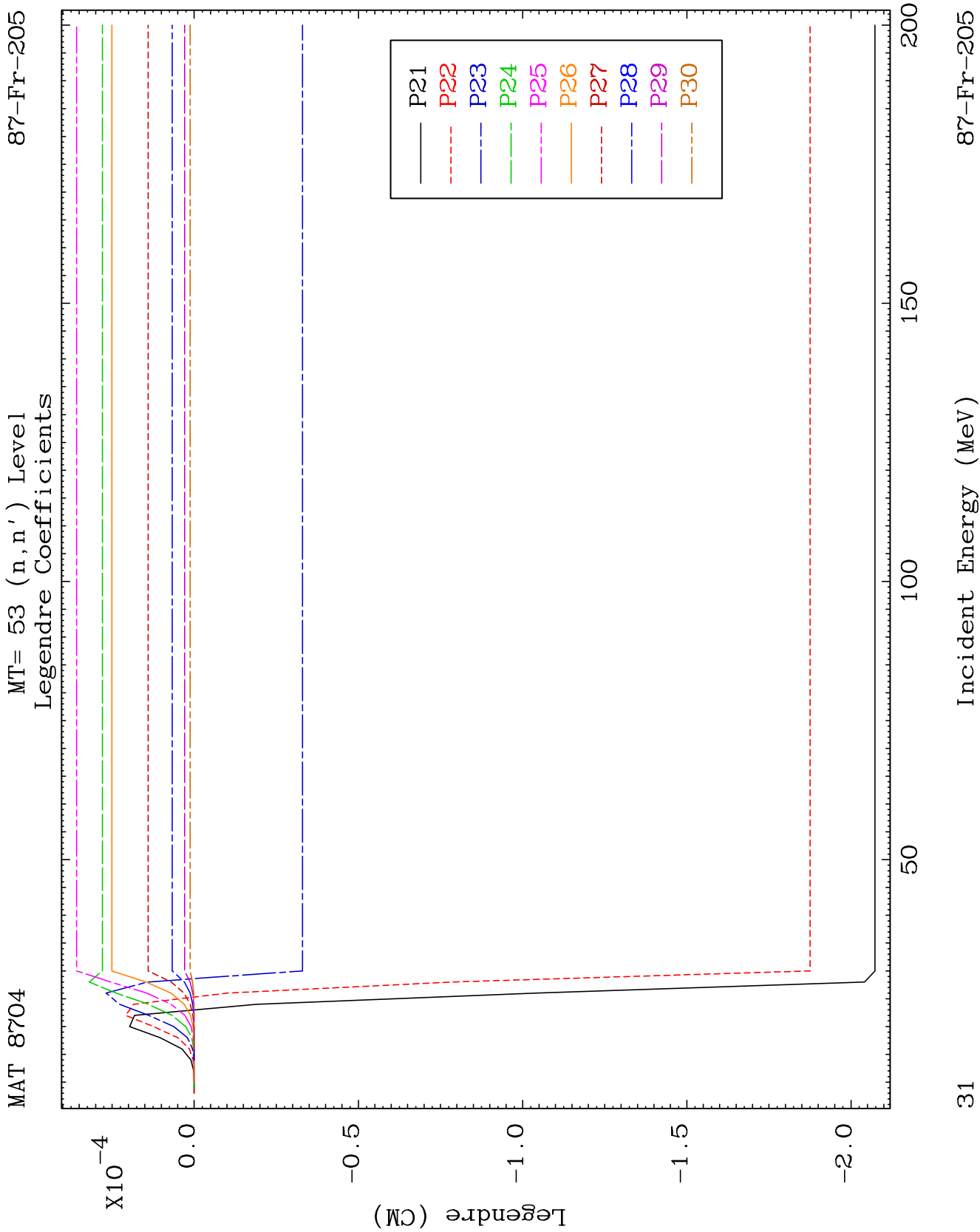


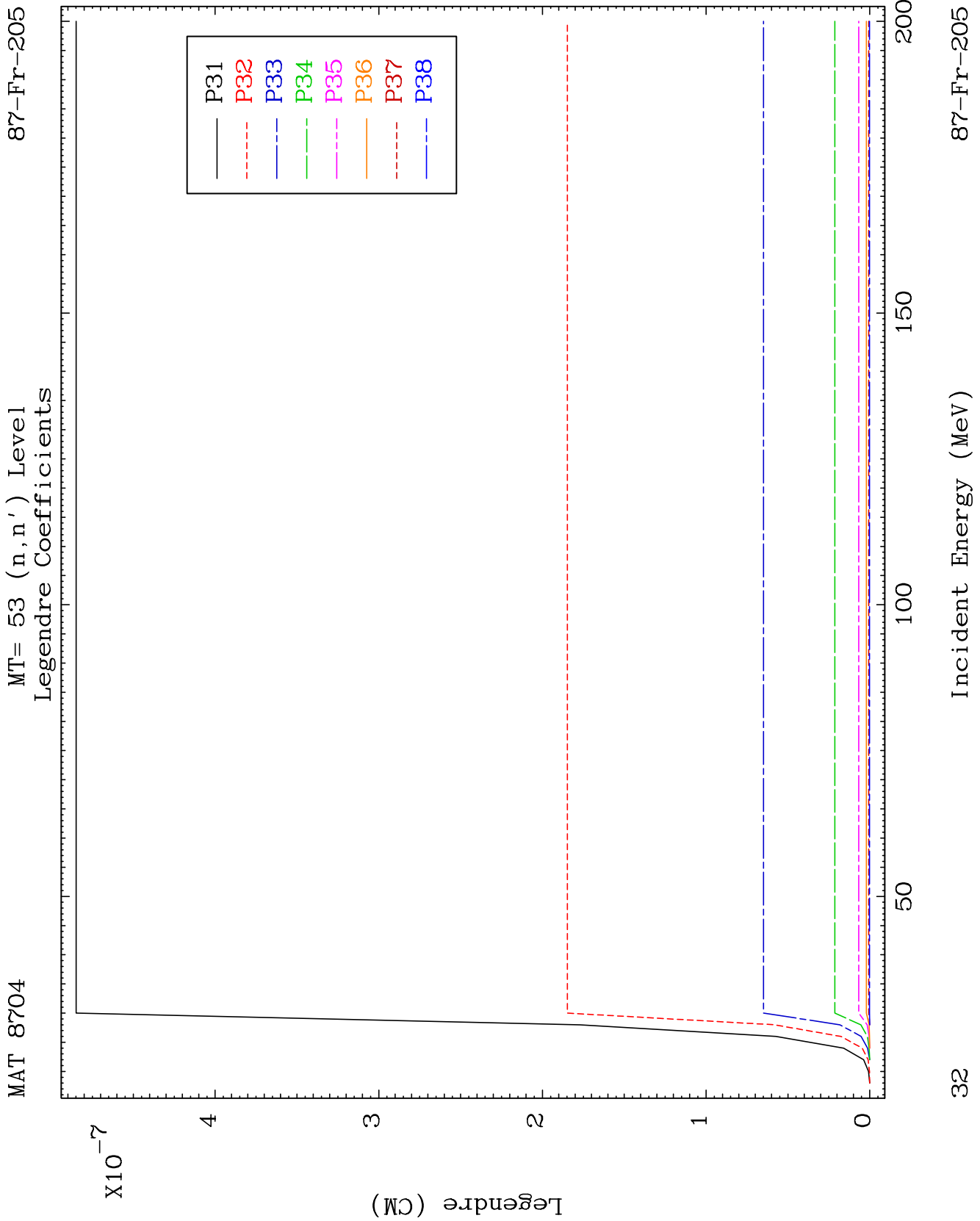
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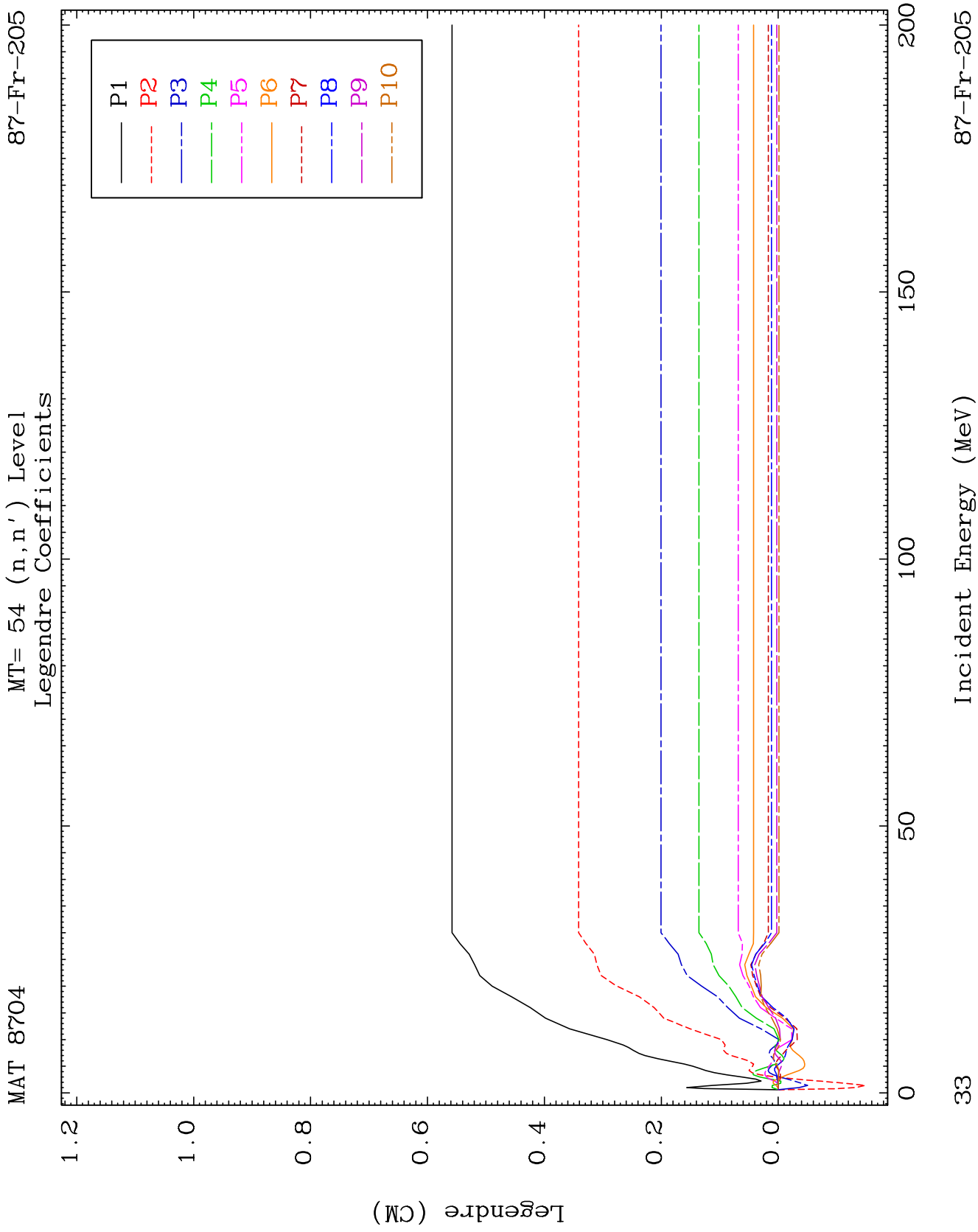
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Legendre Coefficients

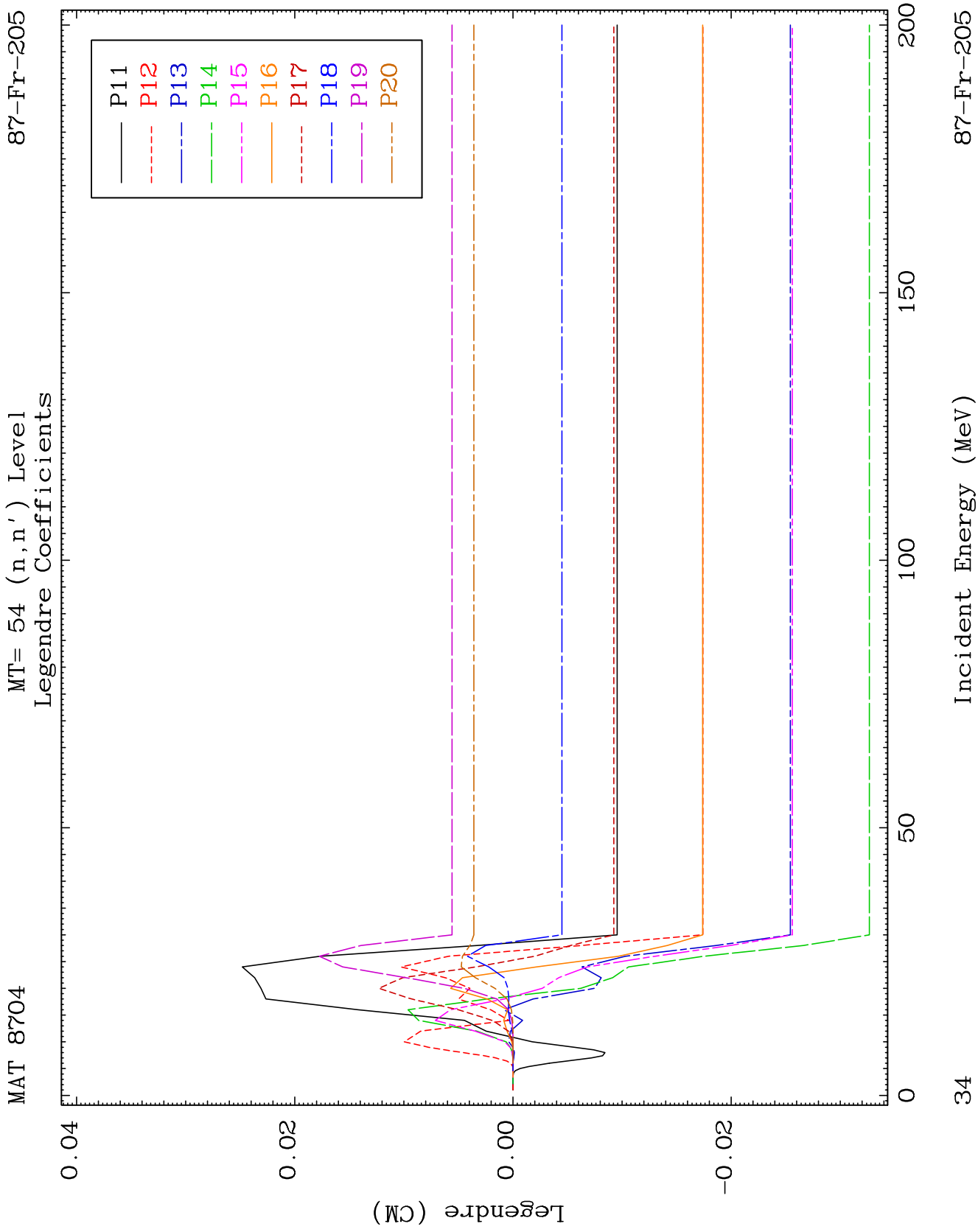
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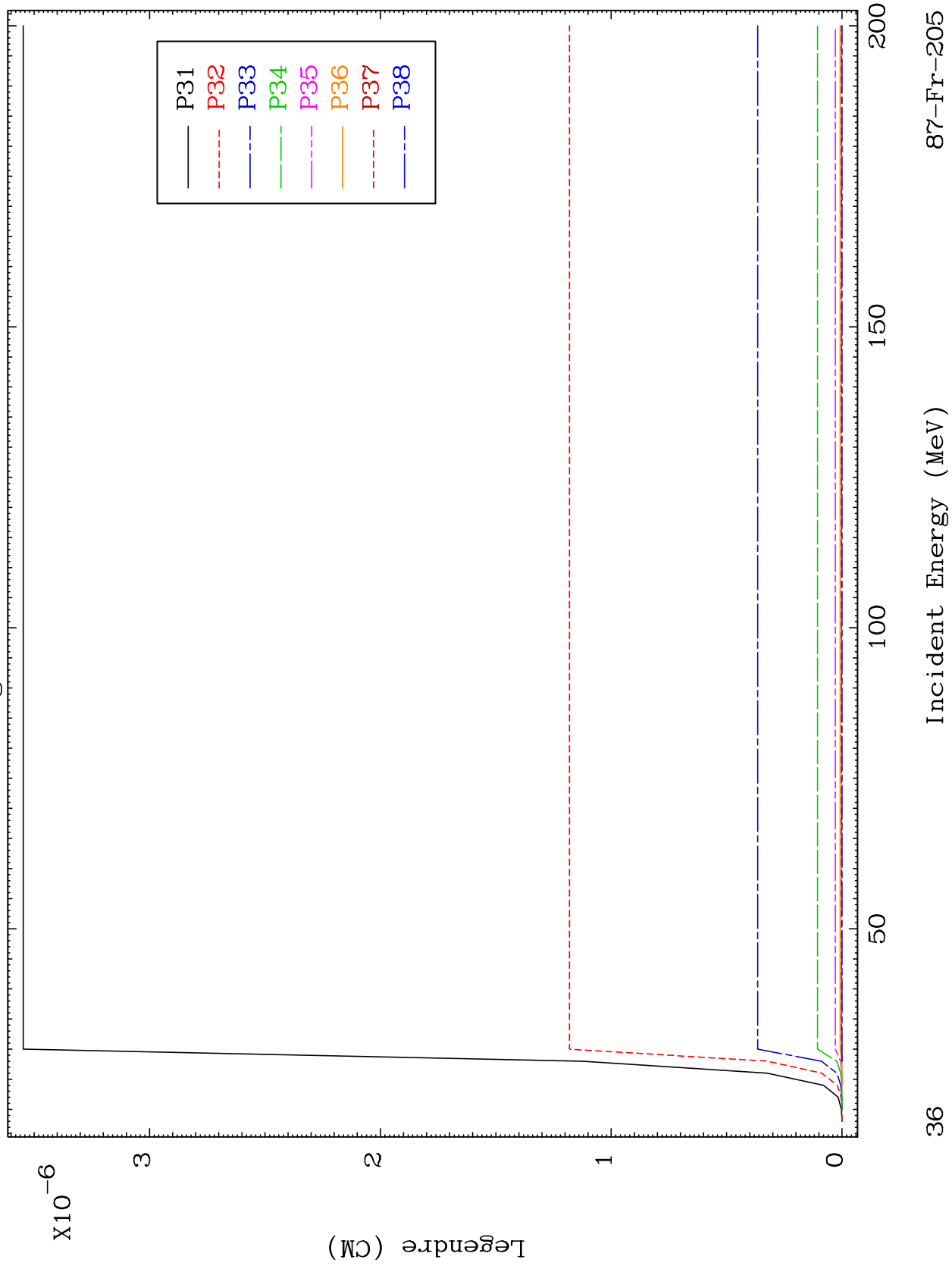


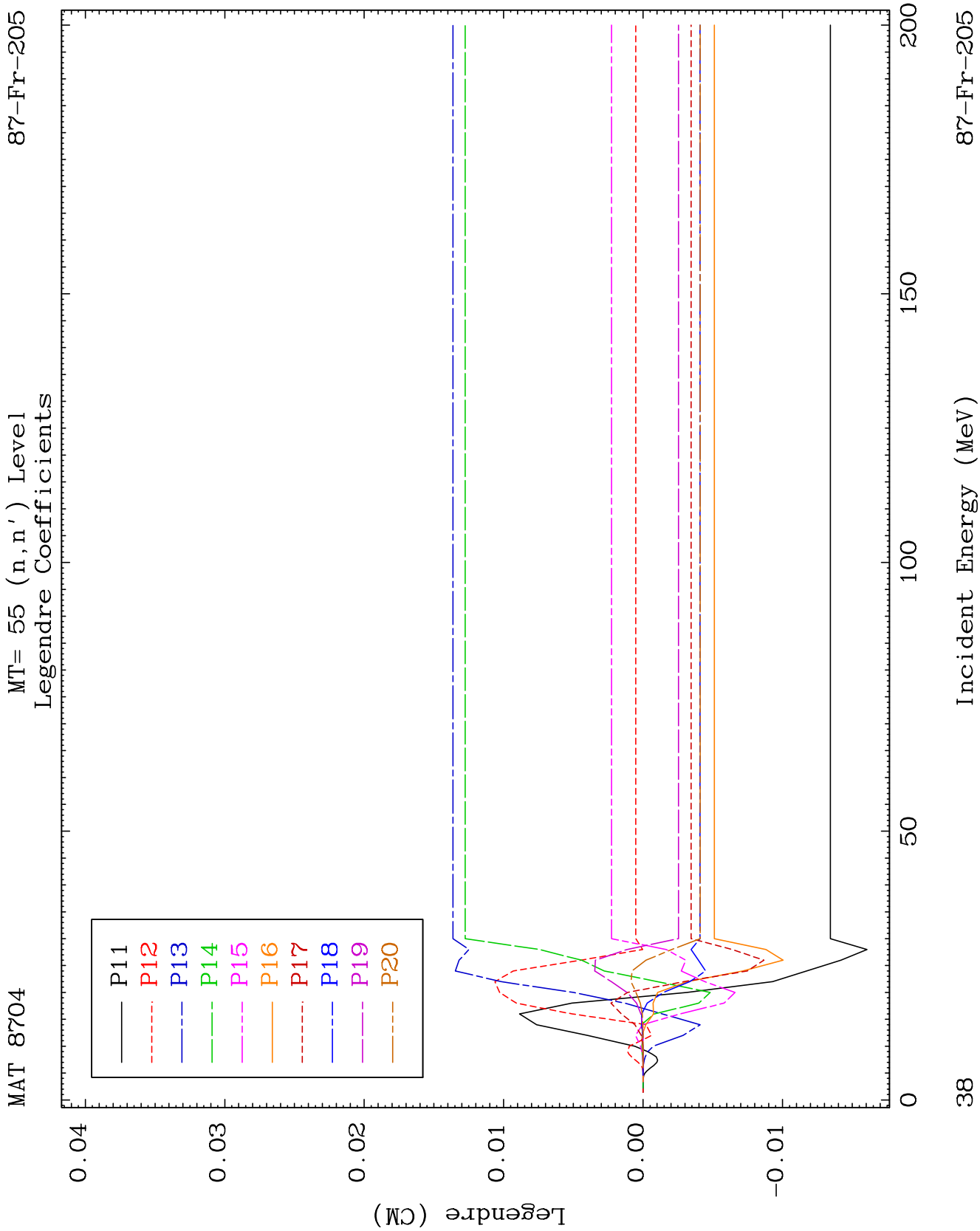


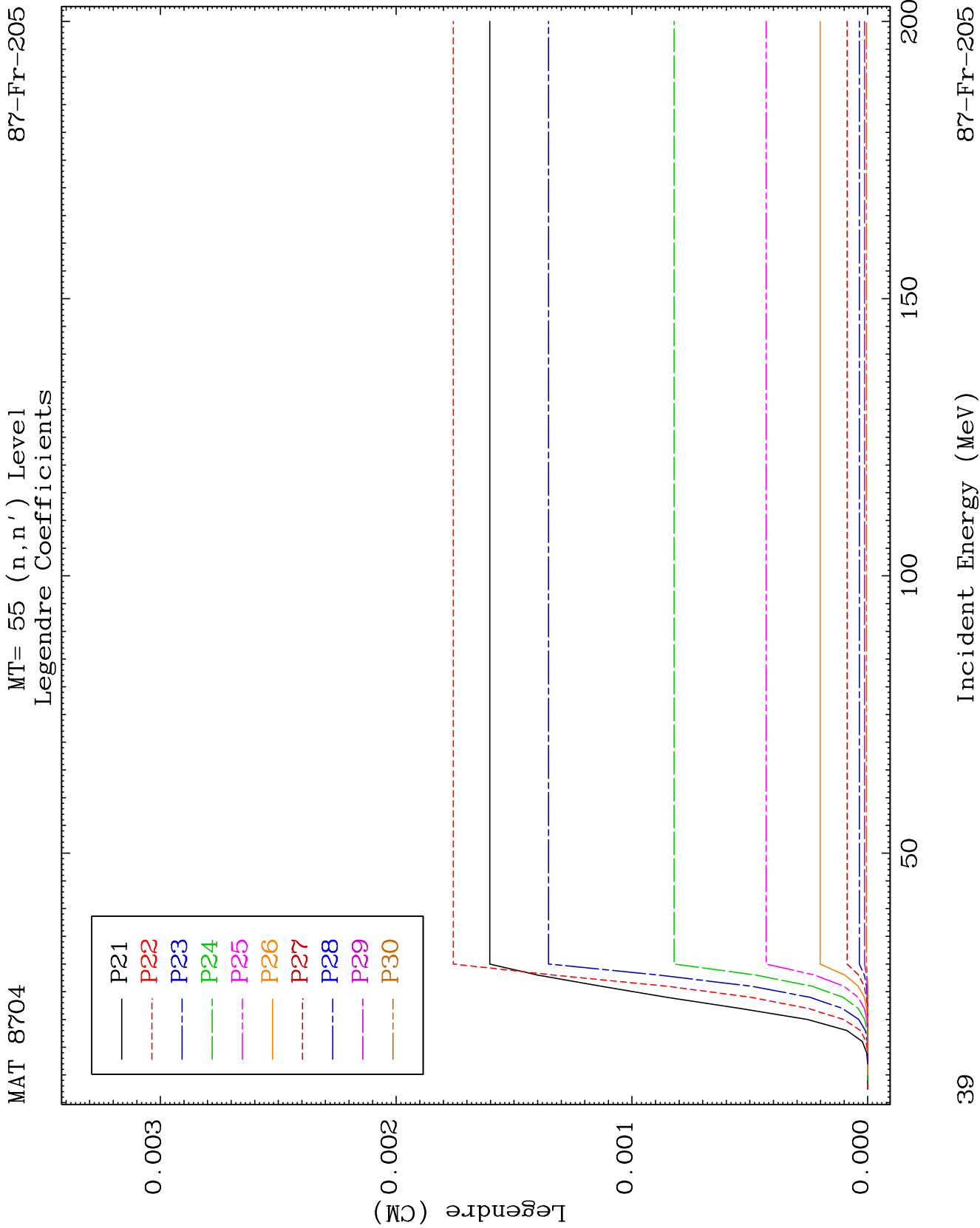
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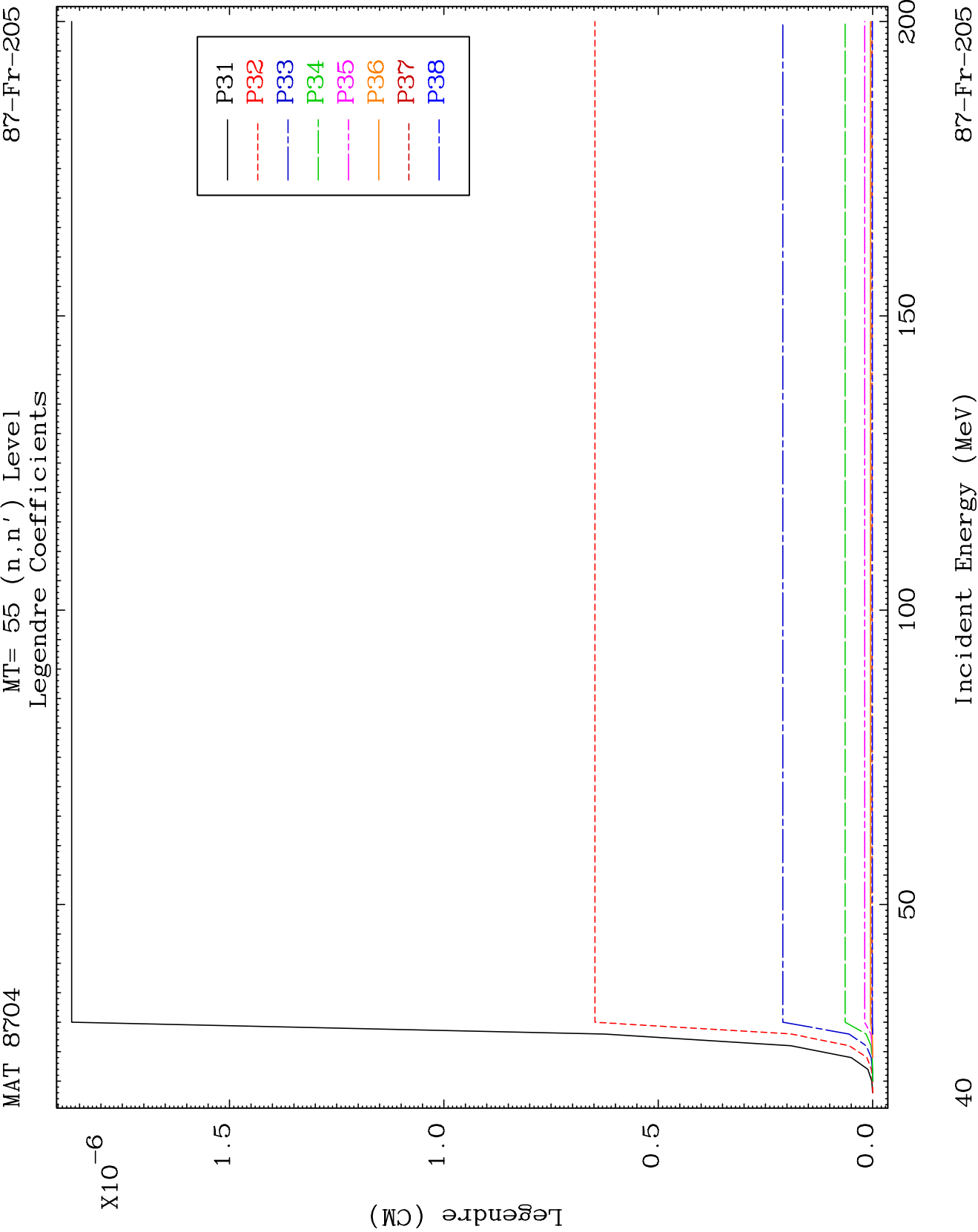
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Legendre Coefficients

87-Fr-205





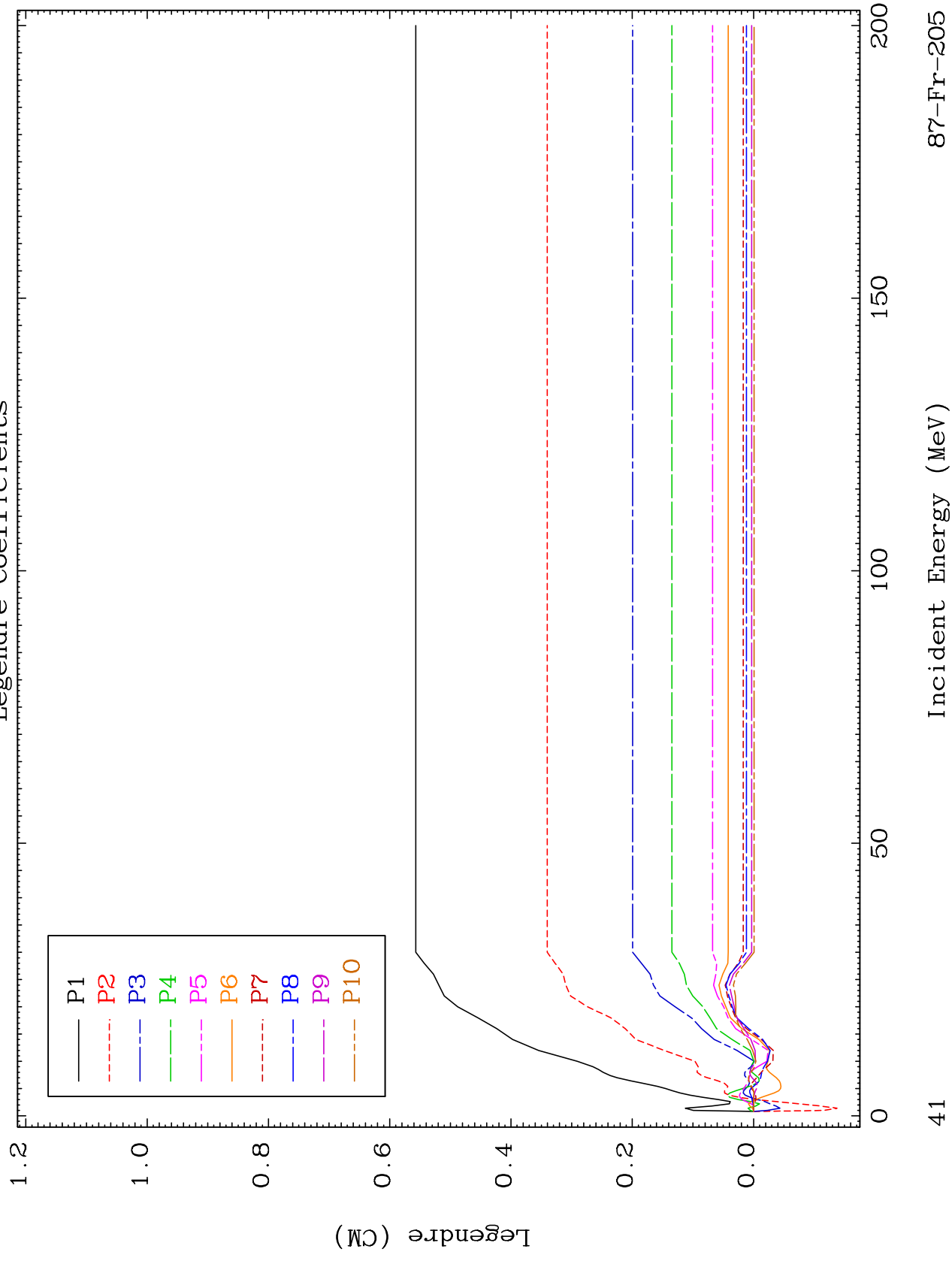


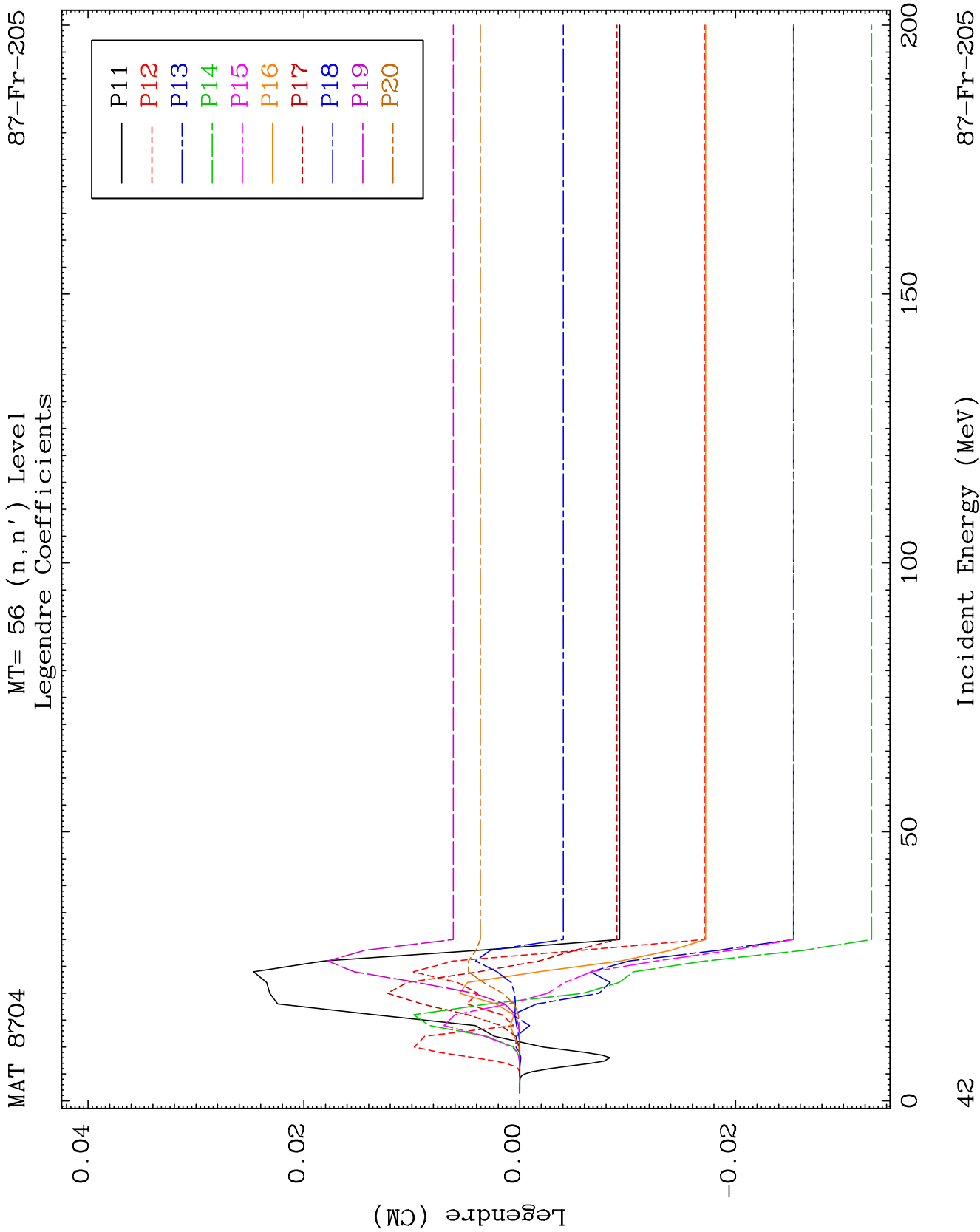


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MT= 56 (n, n') Level
Legendre Coefficients

87-Fr-205

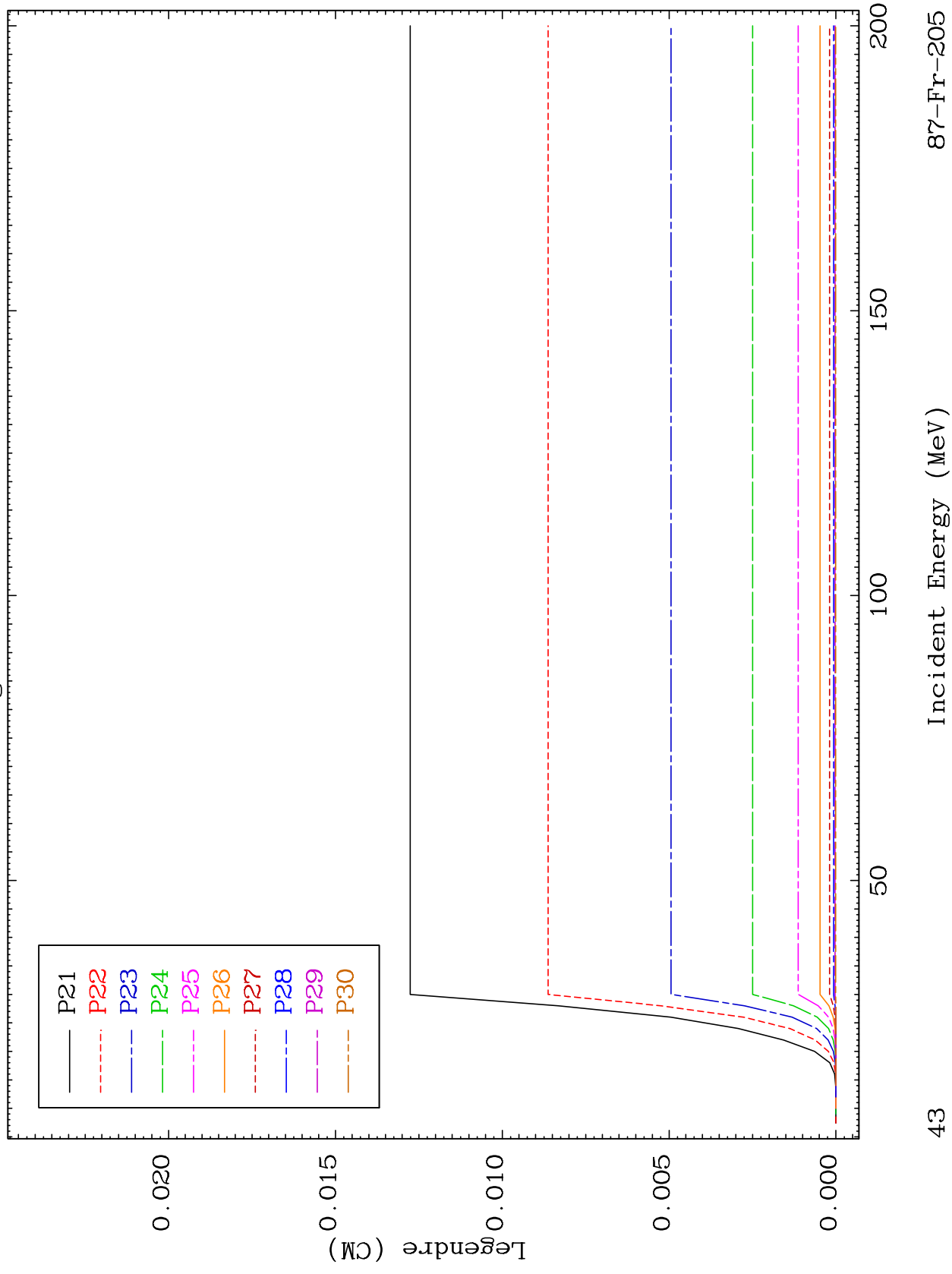


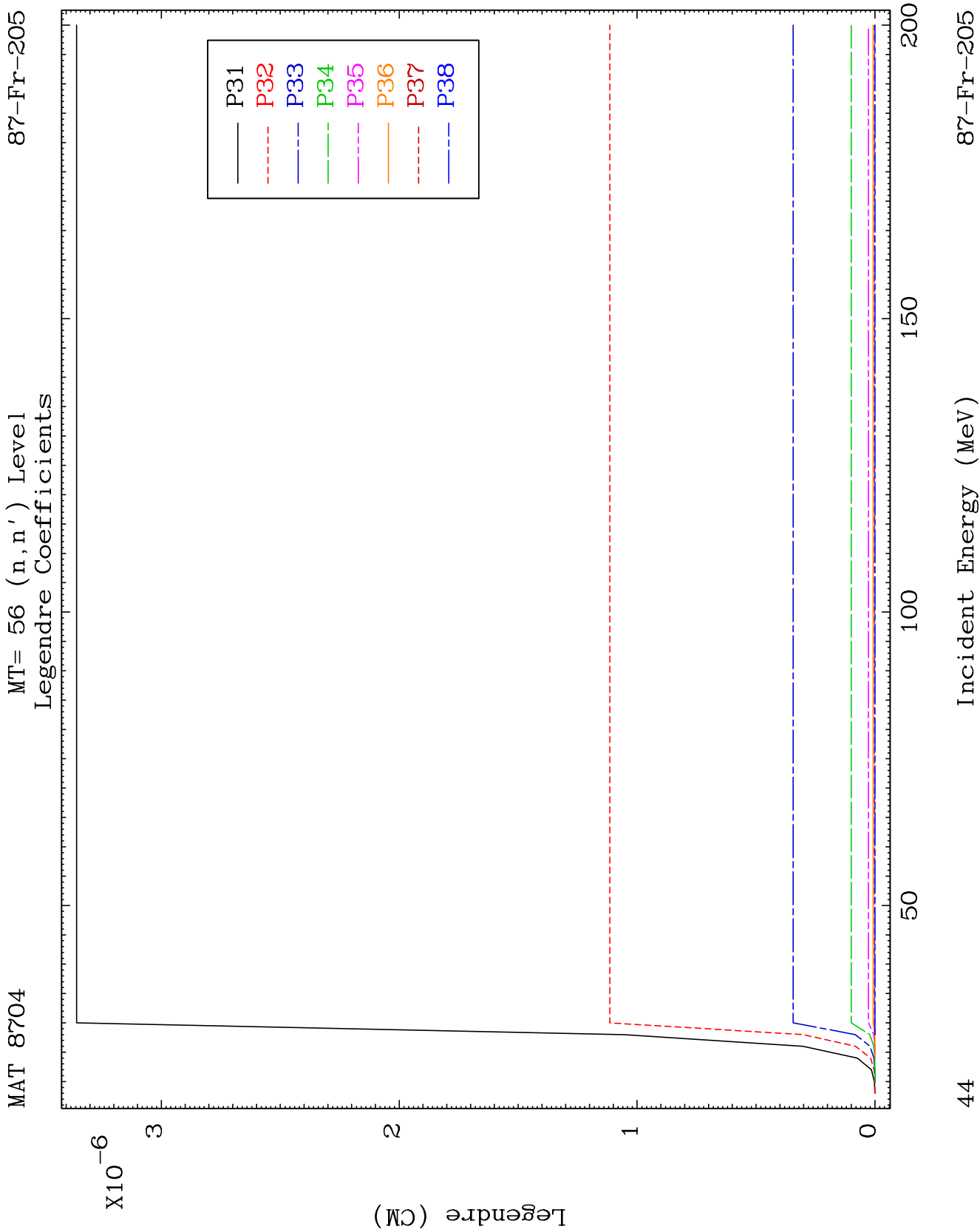


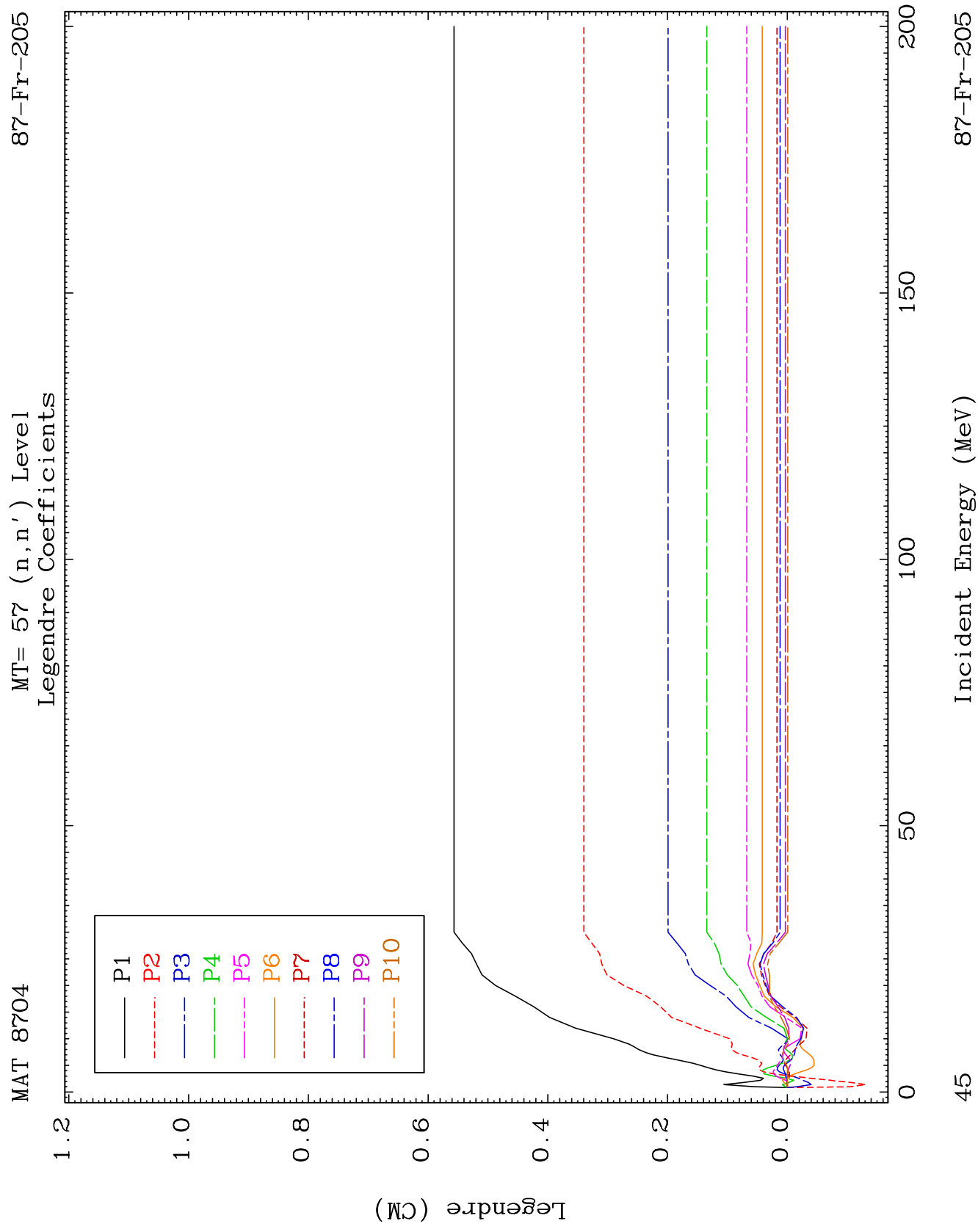
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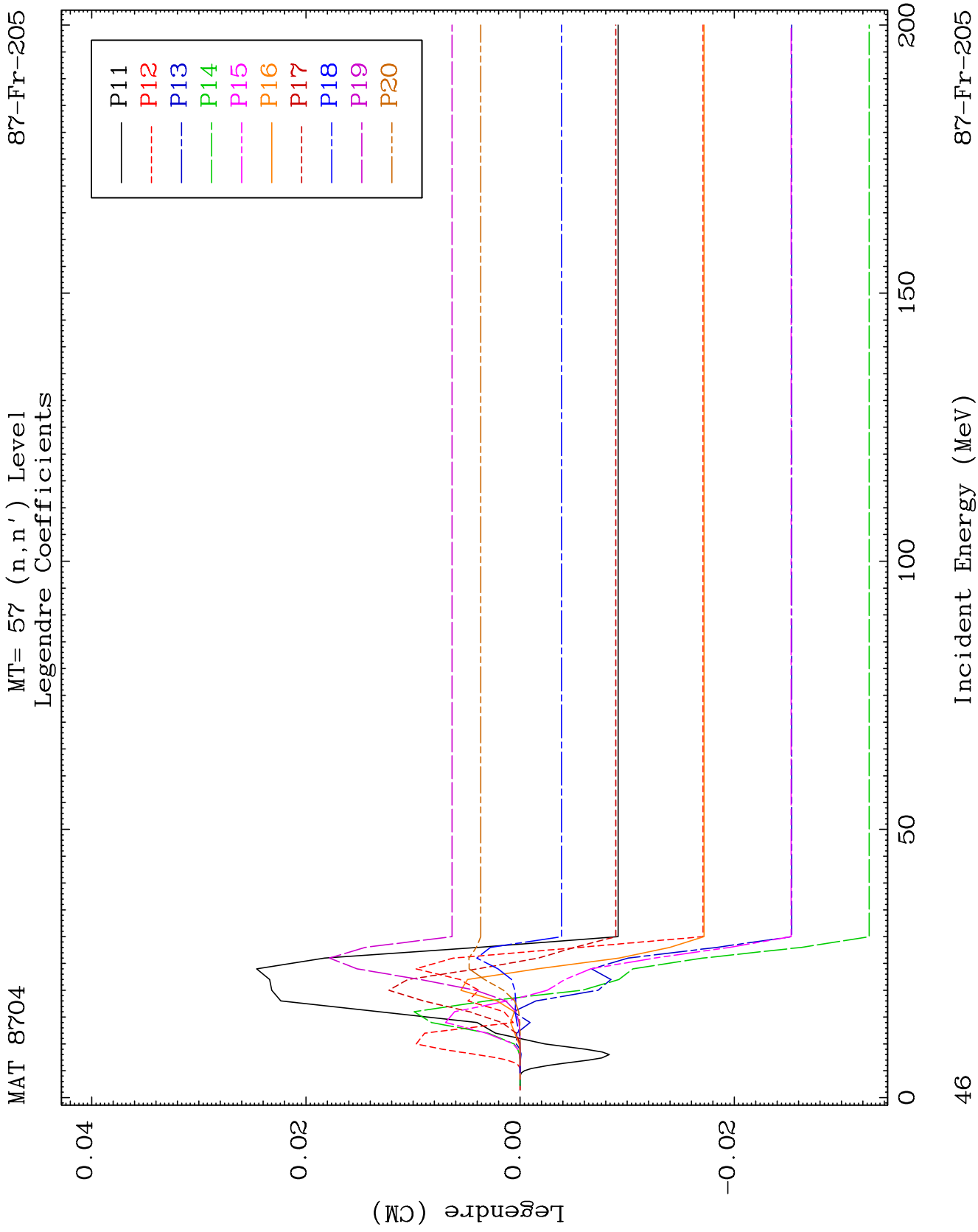
MT= 56 (n, n') Level
Legendre Coefficients

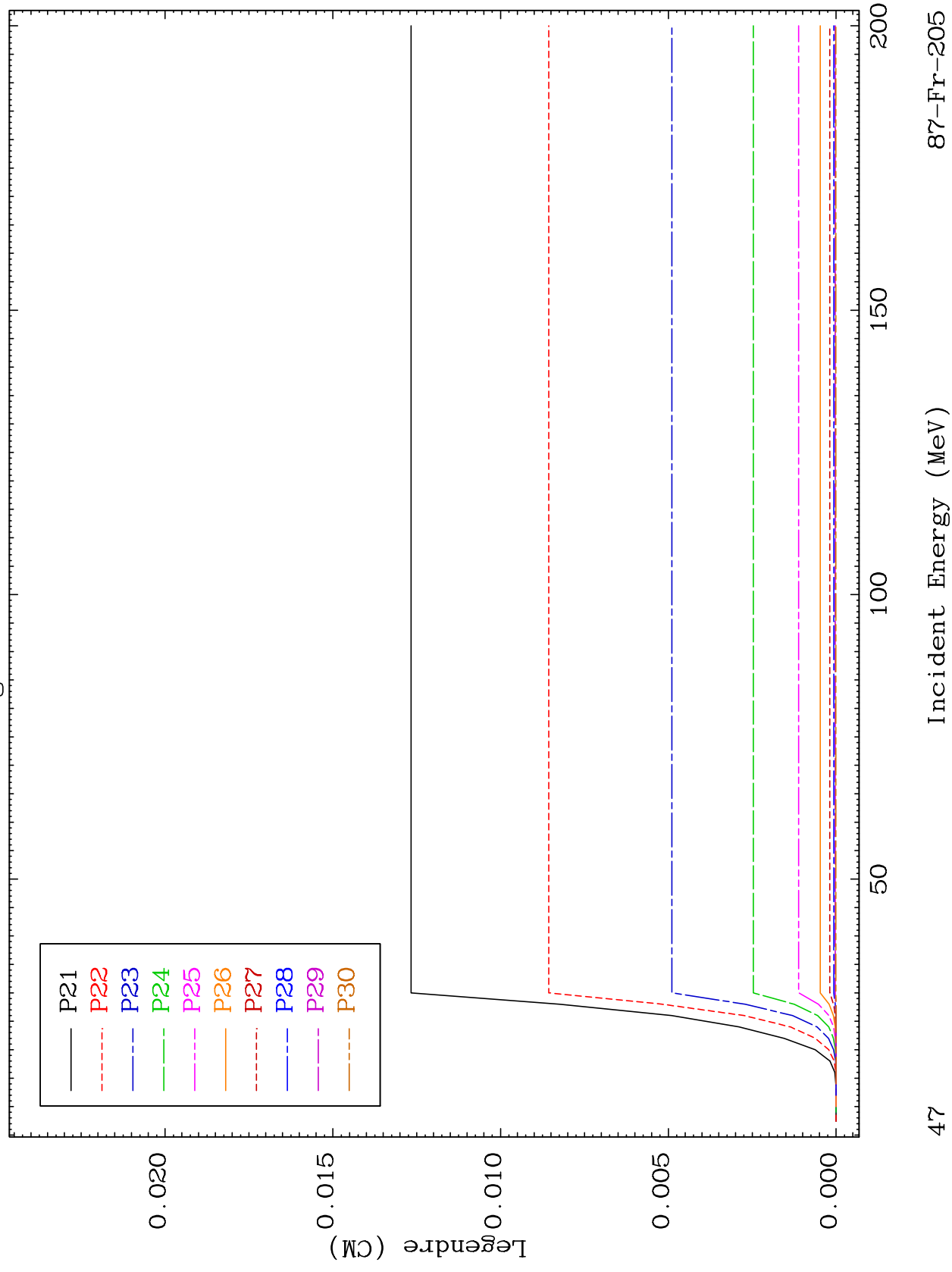
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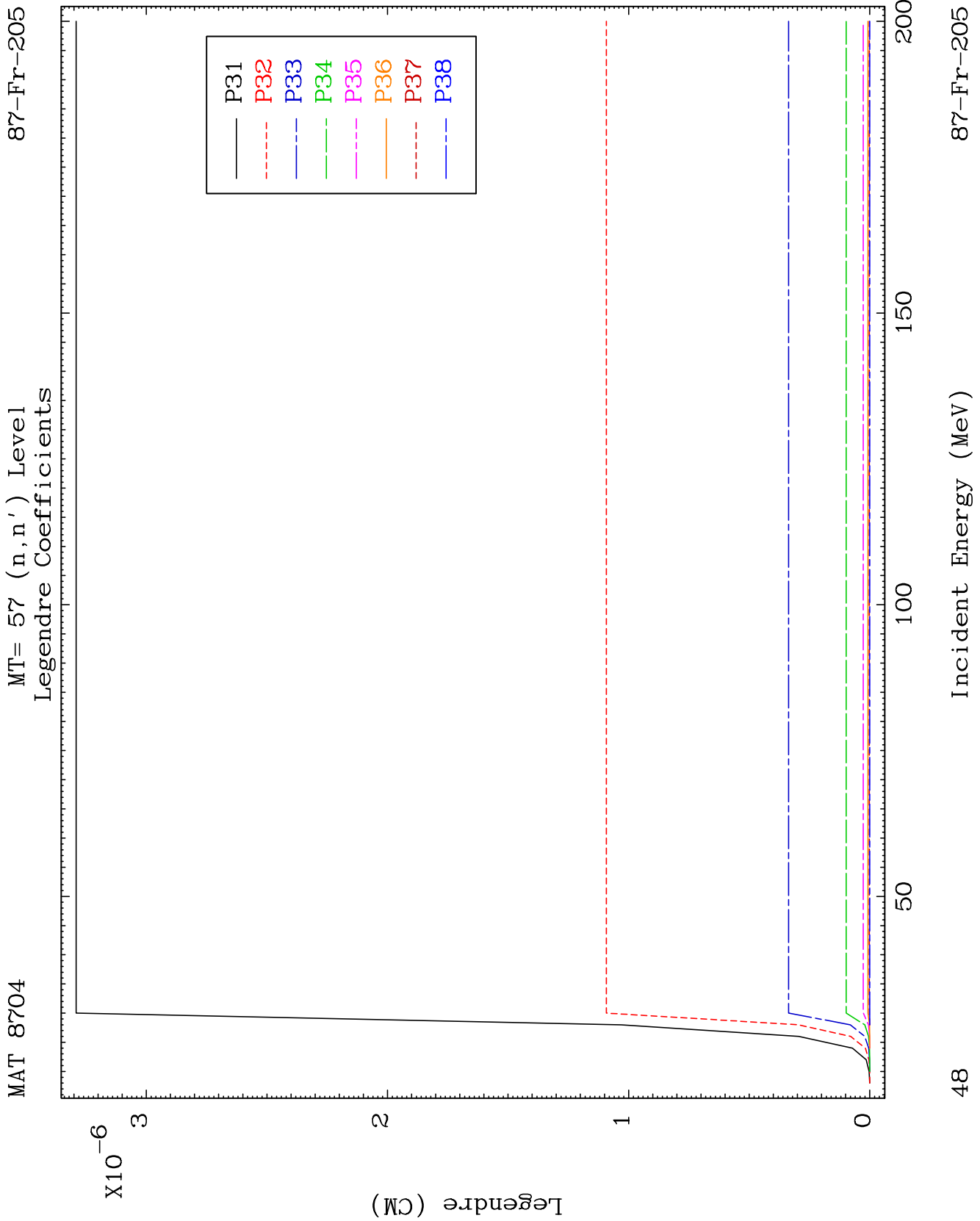


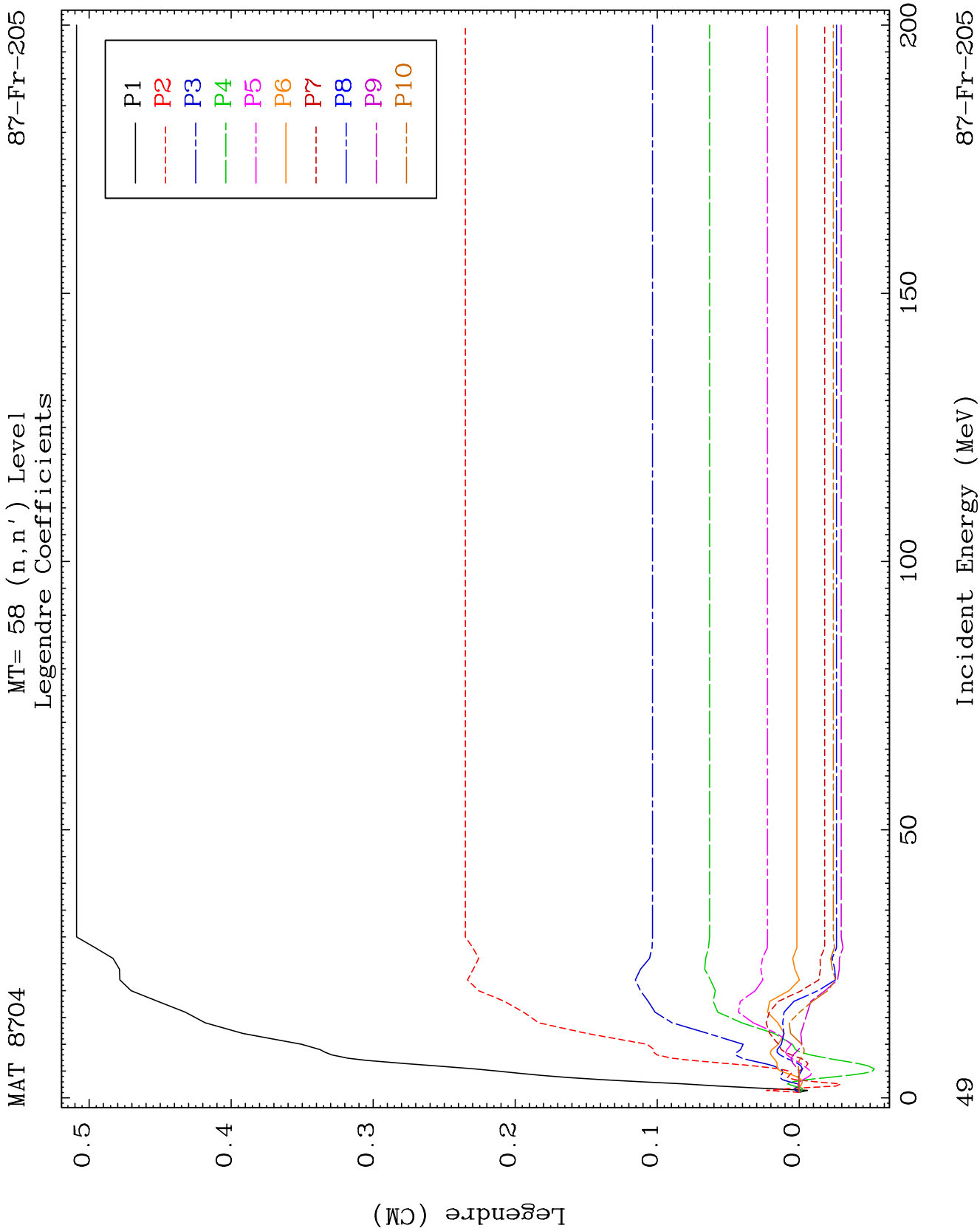


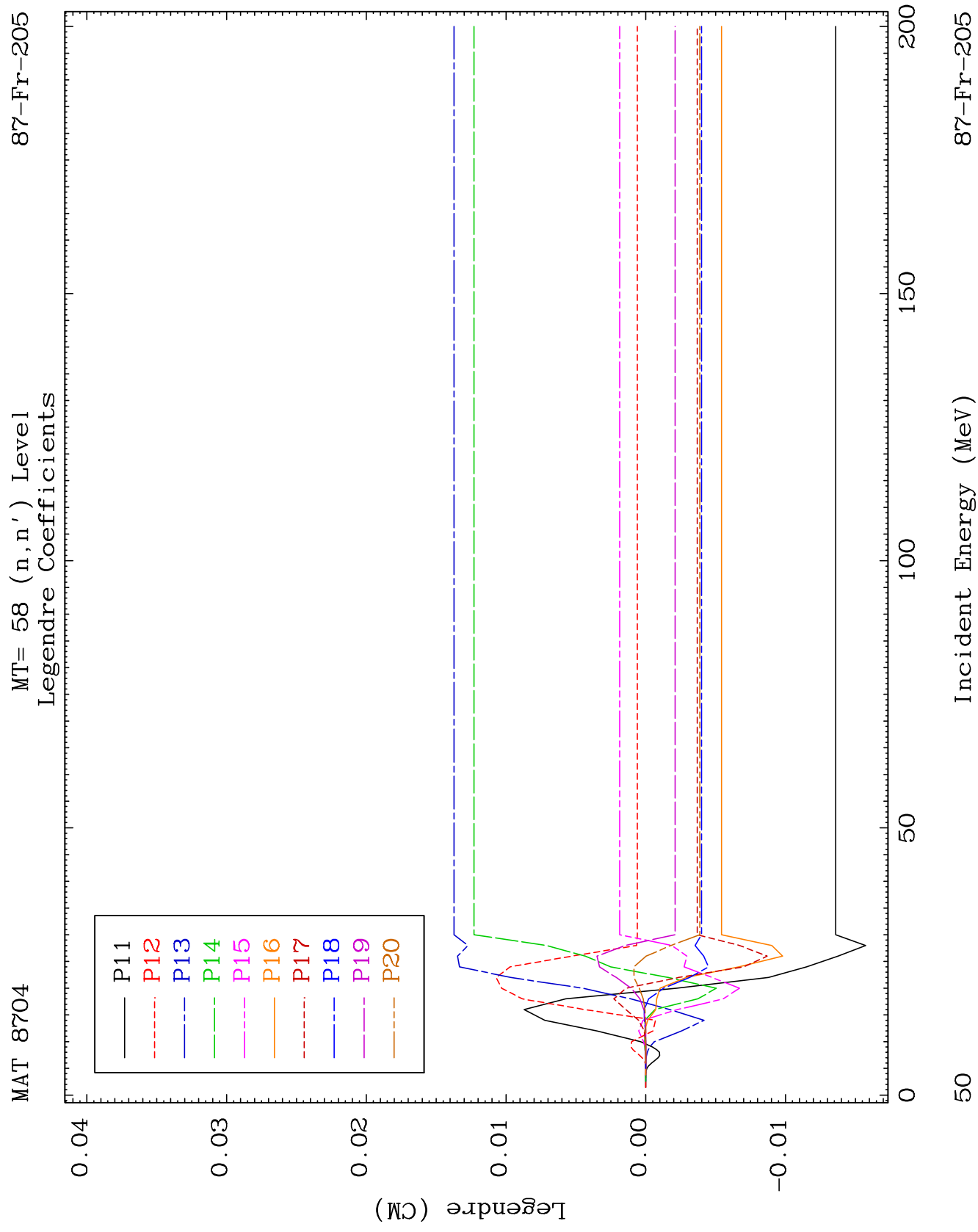


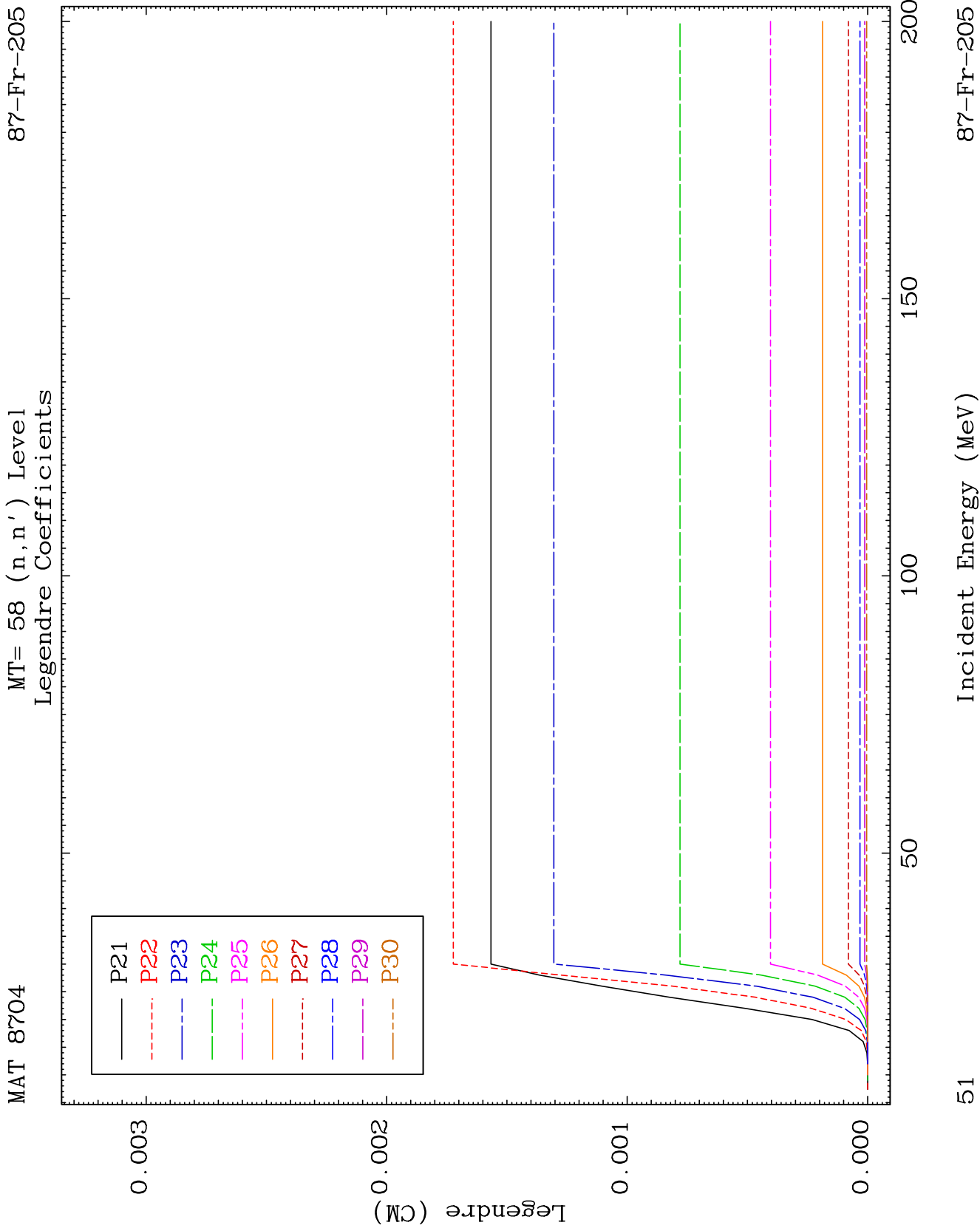


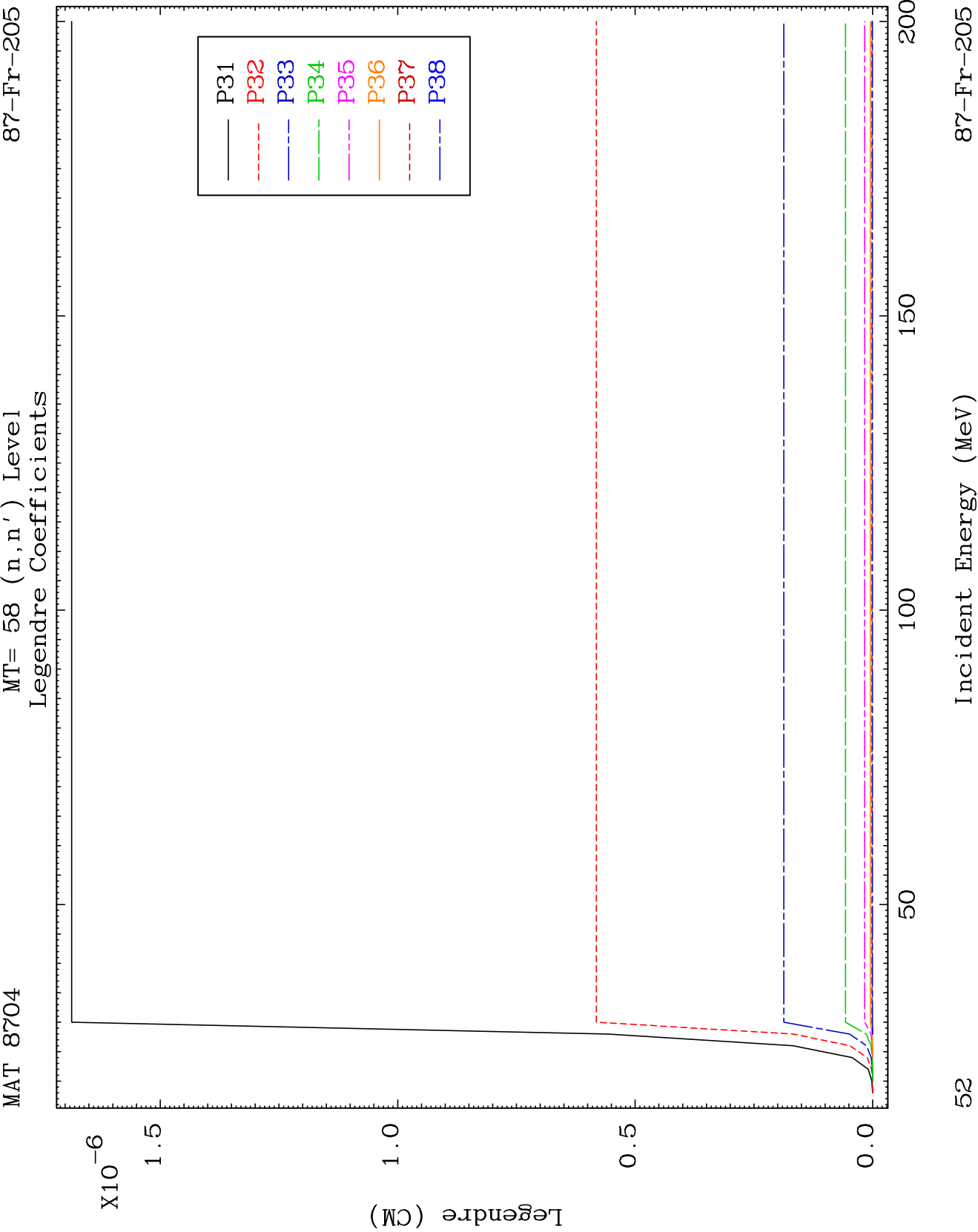


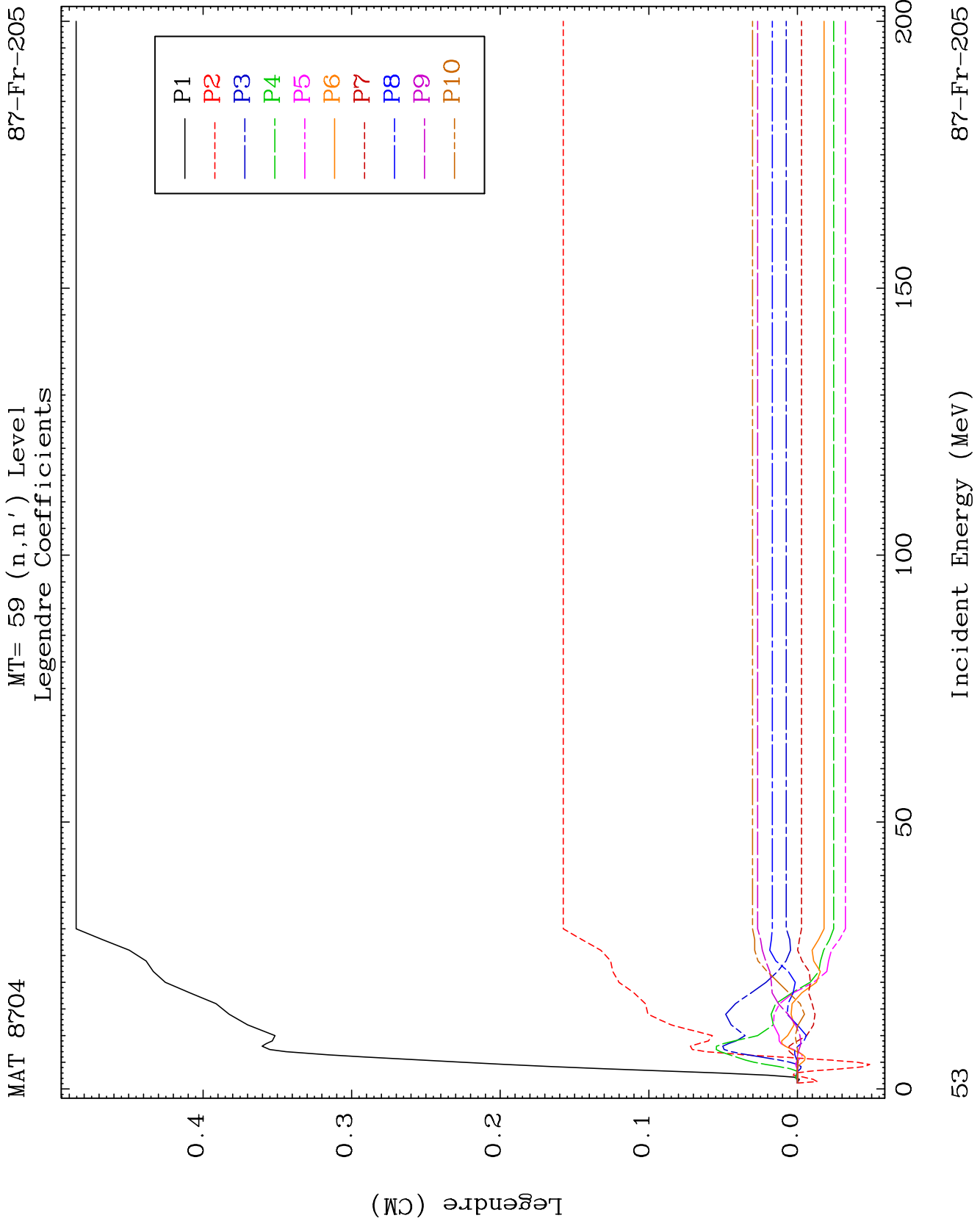








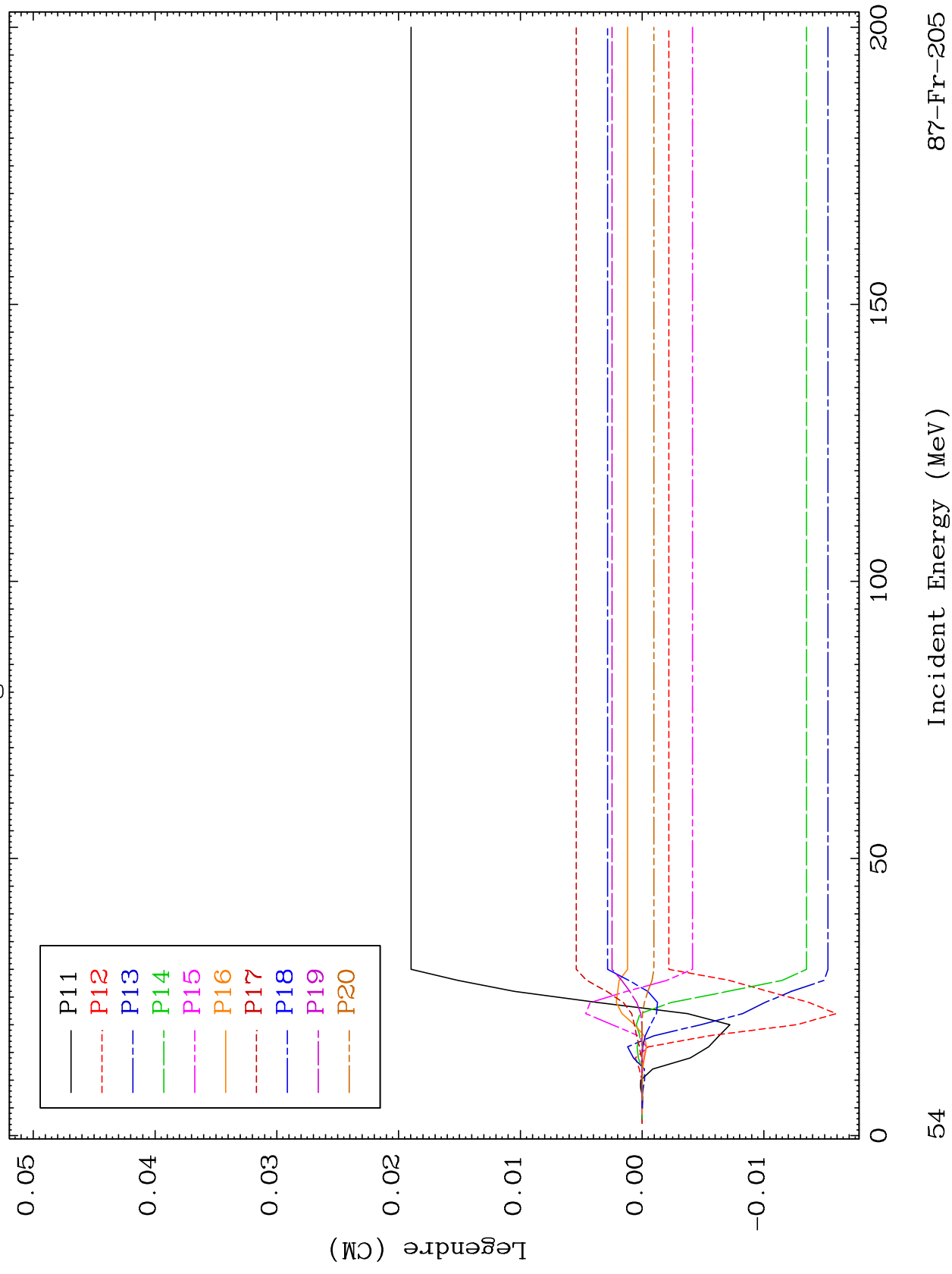


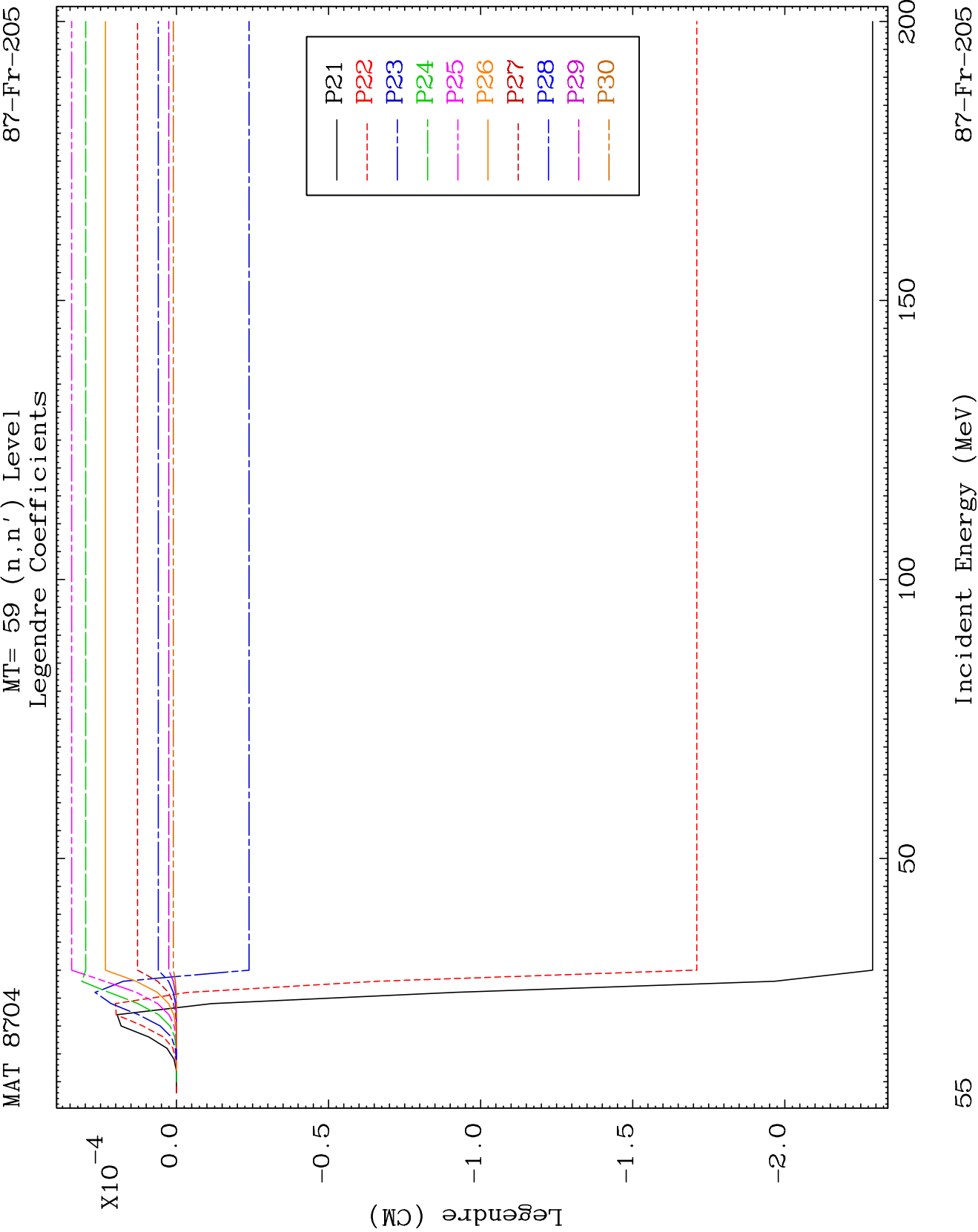


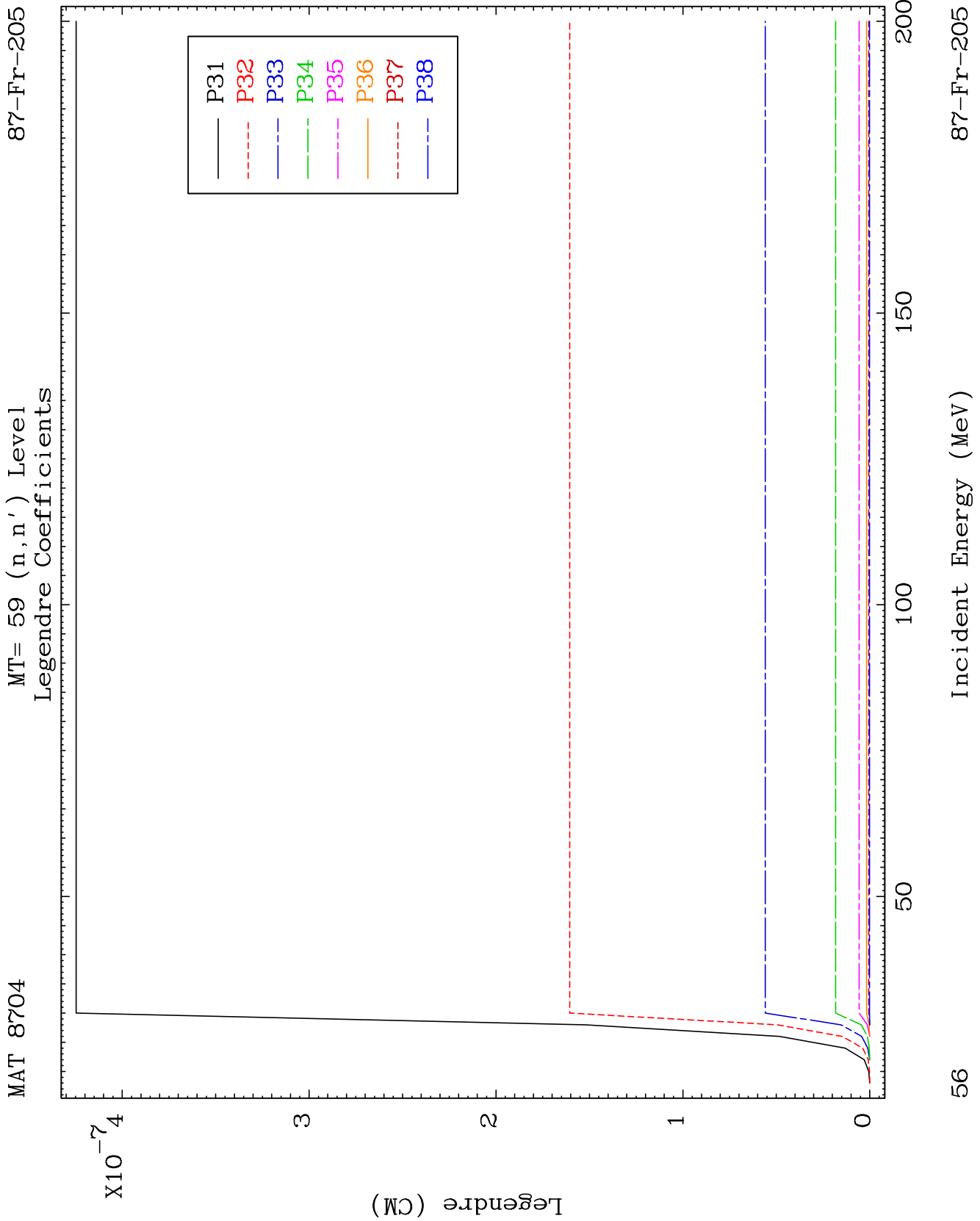
MAT 8704

MT= 59 (n, n') Level
Legendre Coefficients

87-Fr-205



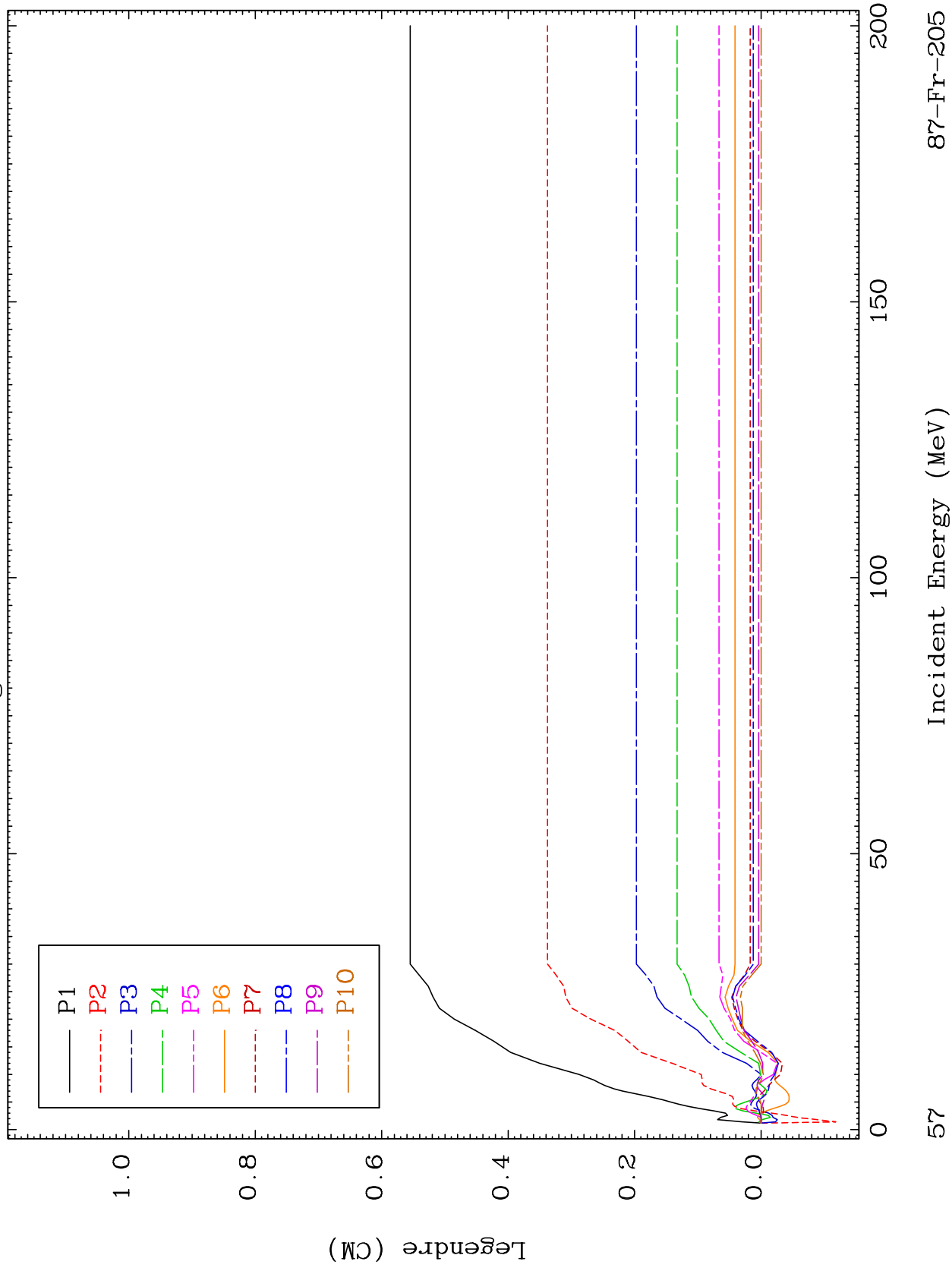




MAT 8704

MT= 60 (n, n') Level
Legendre Coefficients

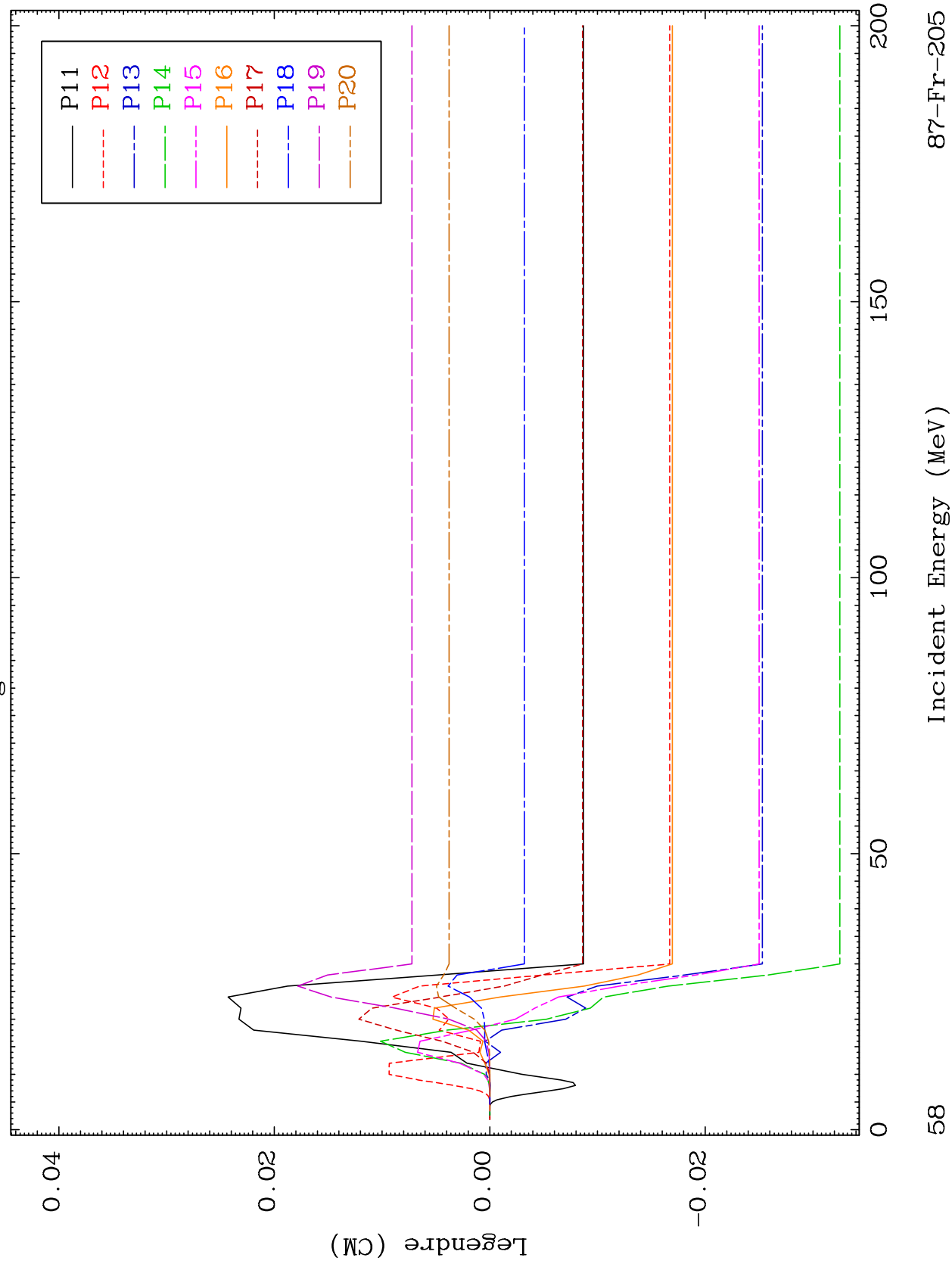
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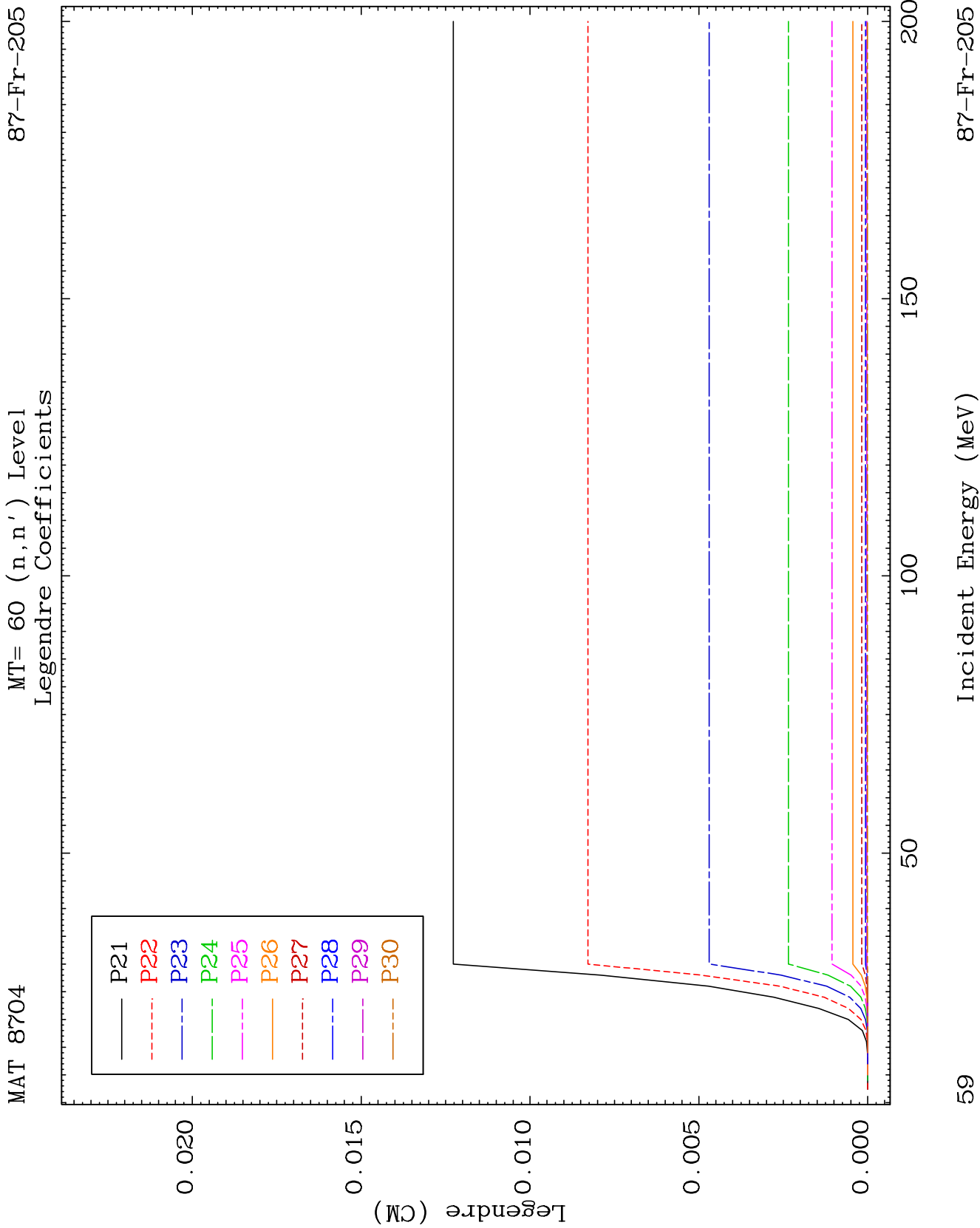


MAT 8704

MT= 60 (n, n') Level
Legendre Coefficients

87-Fr-205

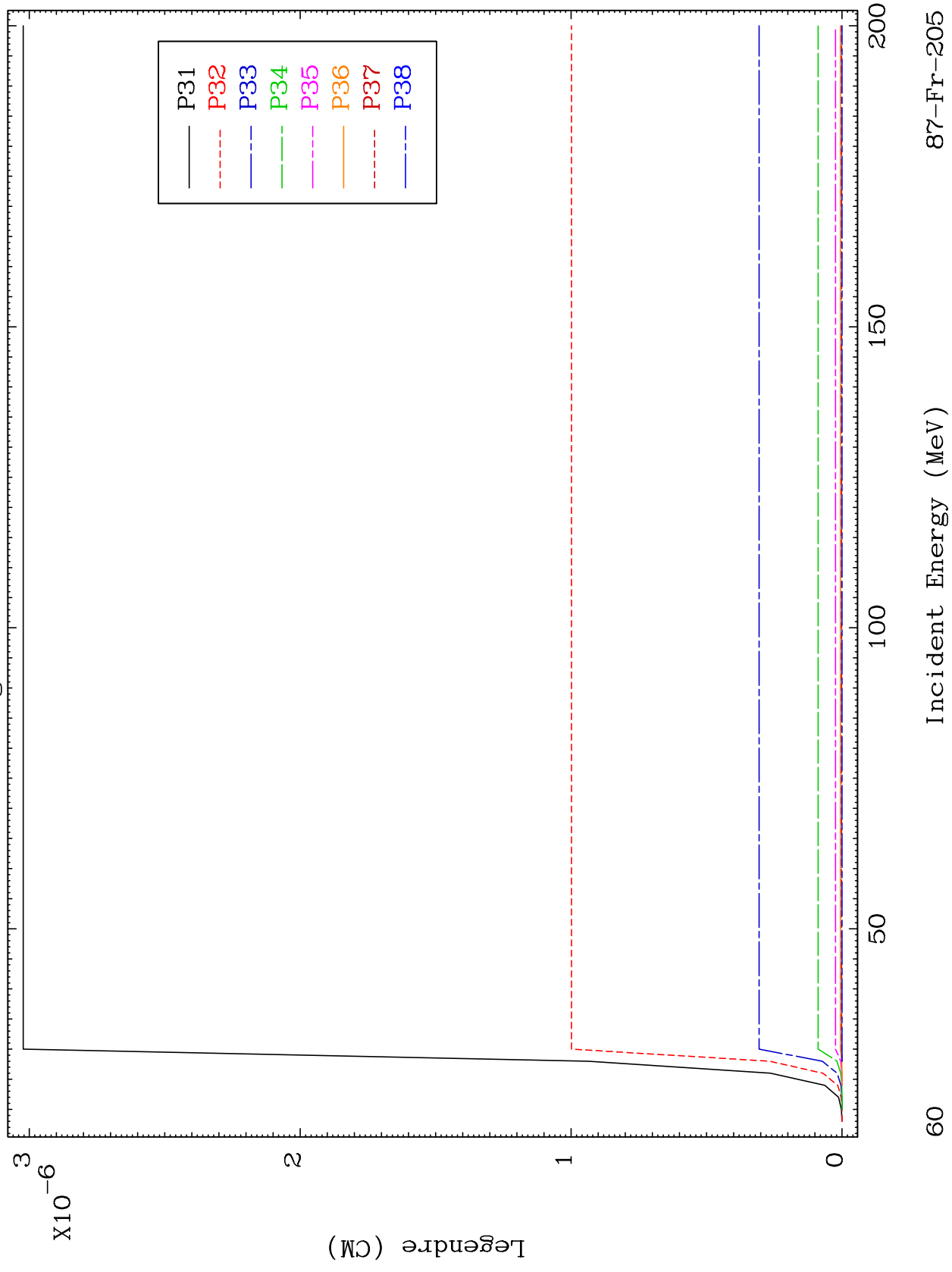




MAT 8704

MT= 60 (n, n') Level
Legendre Coefficients

87-Fr-205

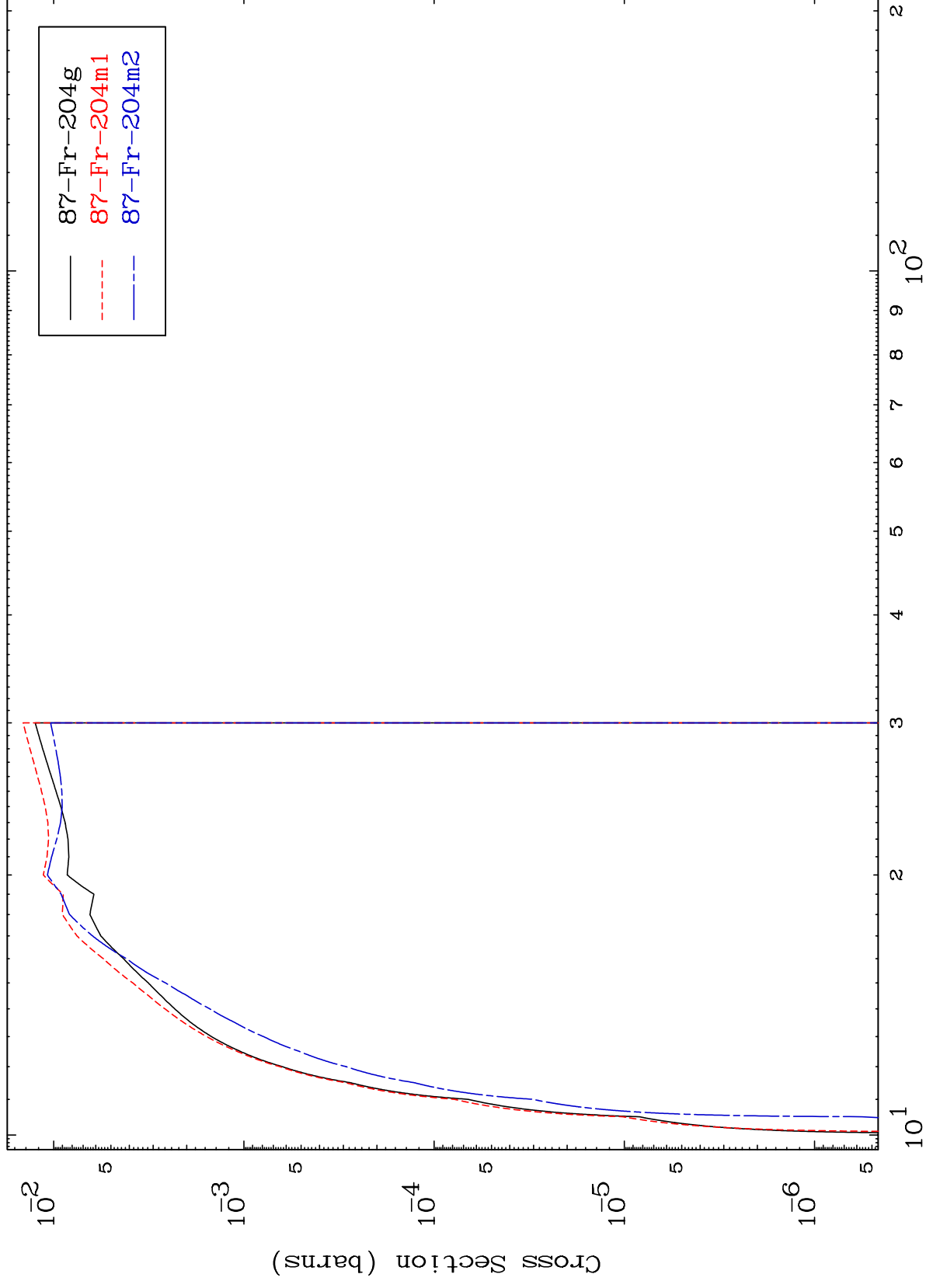


MAT 8704

$$(n, 2n)$$

87-Fr-205

Radionuclide Production Cross Section



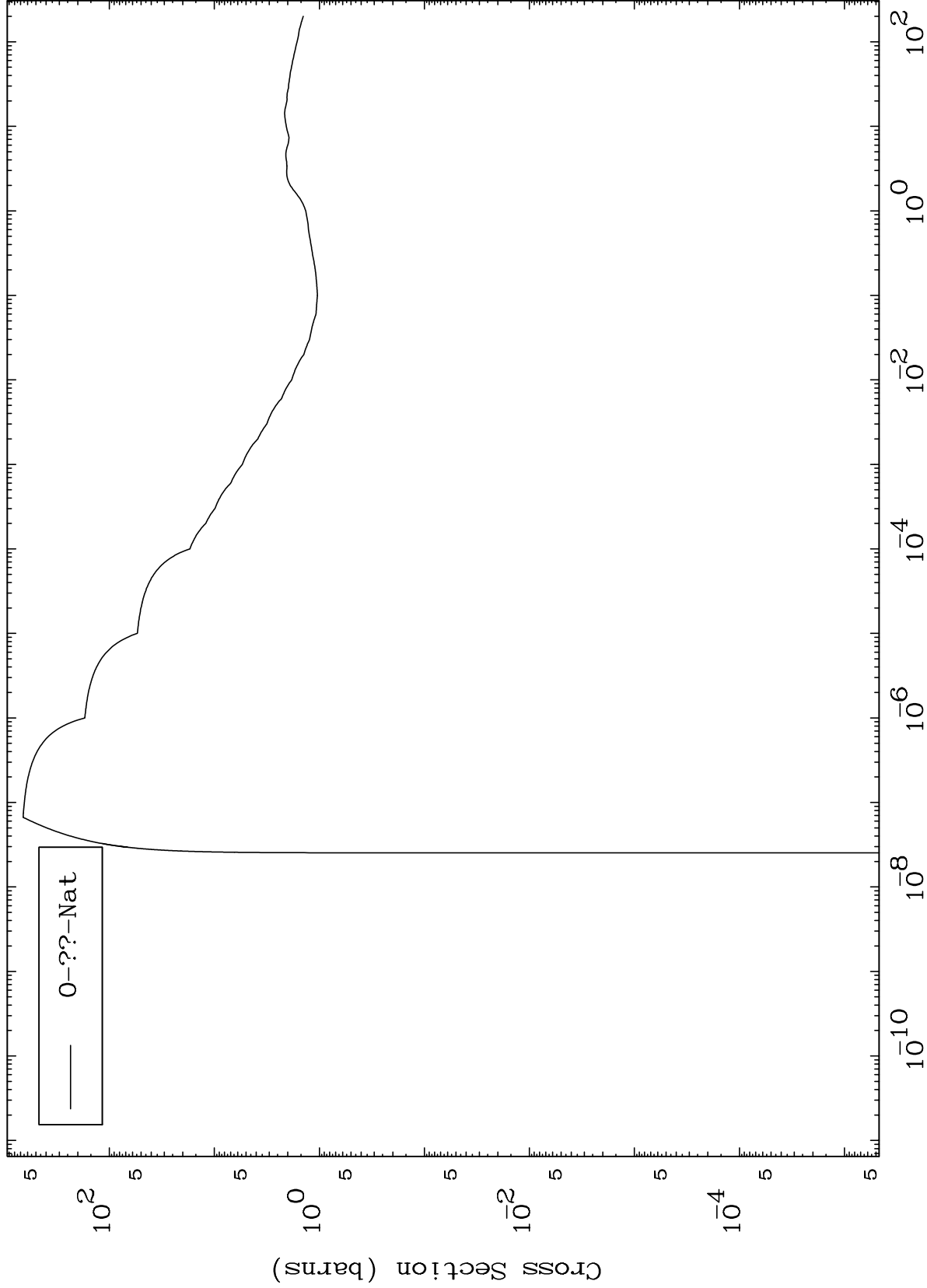
Incident Energy (MeV)

87-Fr-205

MAT 8704

87-Fr-205

Fission
Radionuclide Production Cross Section



0-??-Nat

62

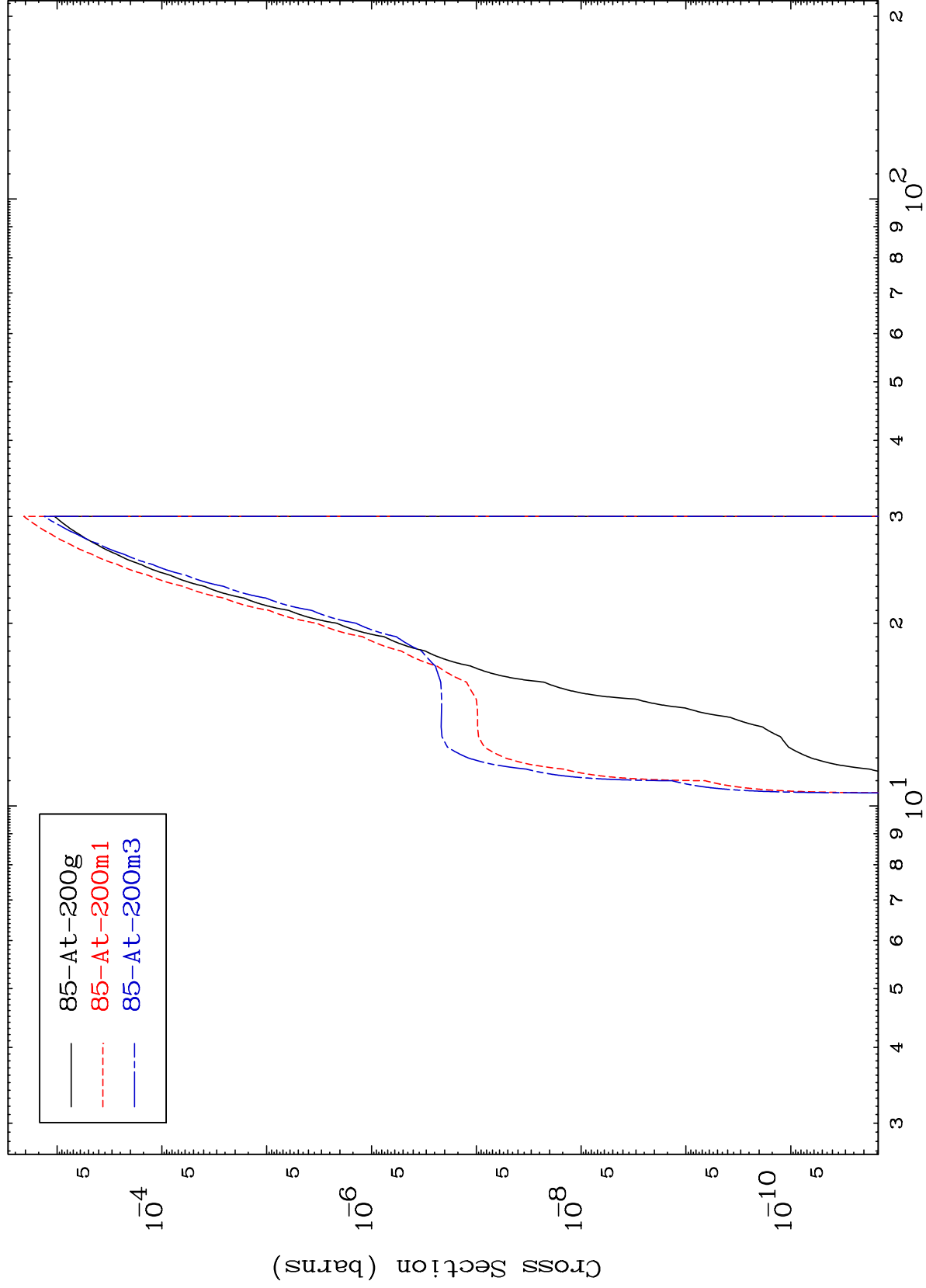
87-Fr-205

Incident Energy (MeV)

MAT 8704

87-Fr-205

$(n,2n) \alpha$
Radionuclide Production Cross Section



63

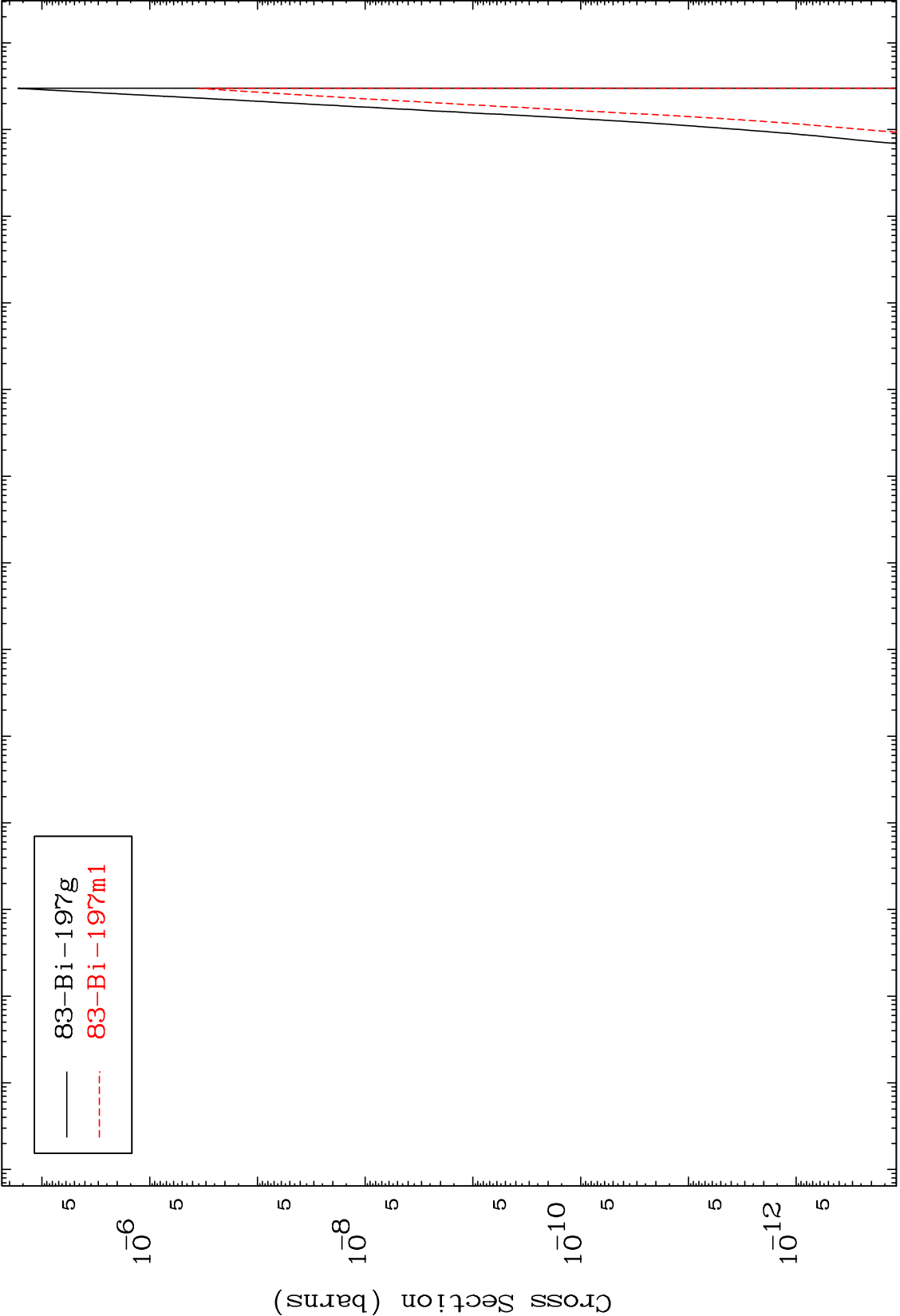
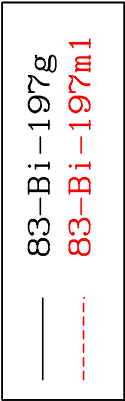
87-Fr-205

MAT 8704

(n,n') 2 α

87-Fr-205

Radionuclide Production Cross Section



64

Incident Energy (MeV)

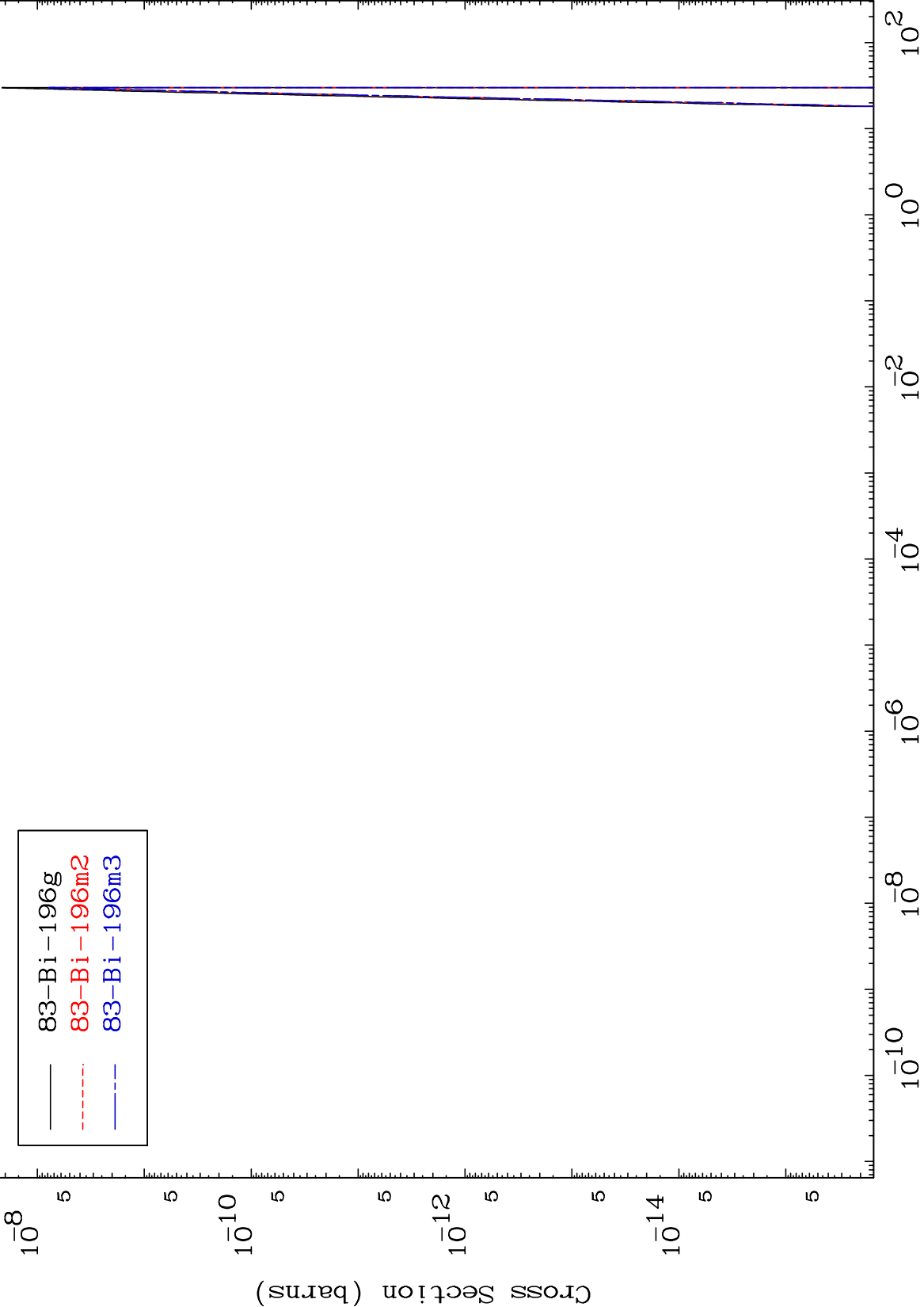
87-Fr-205

MAT 8704

(n,2n) 2 α

87-Fr-205

Radionuclide Production Cross Section



65

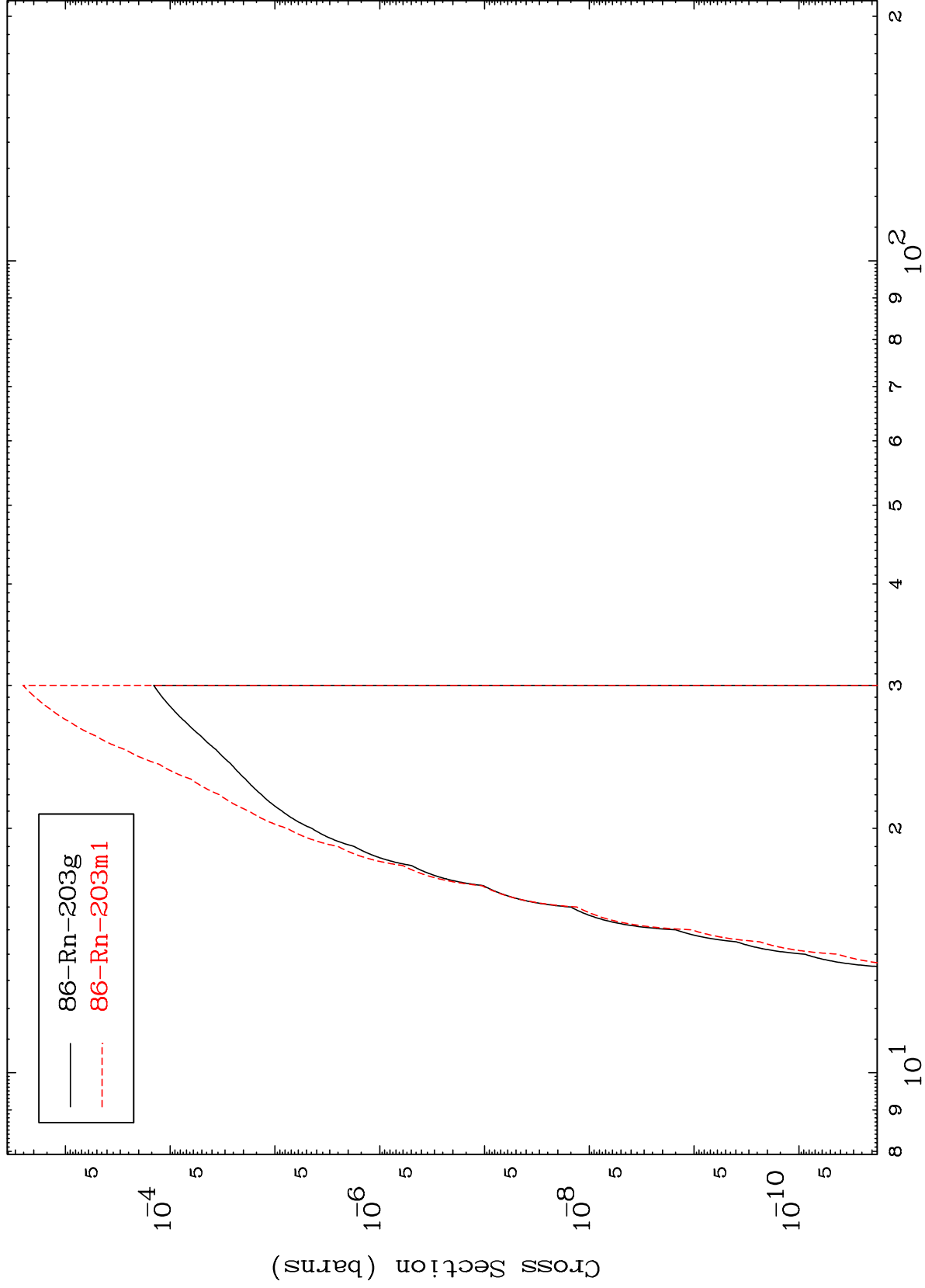
Incident Energy (MeV)

87-Fr-205

MAT 8704

87-Fr-205

(n,n') d
Radionuclide Production Cross Section



66

Incident Energy (MeV)

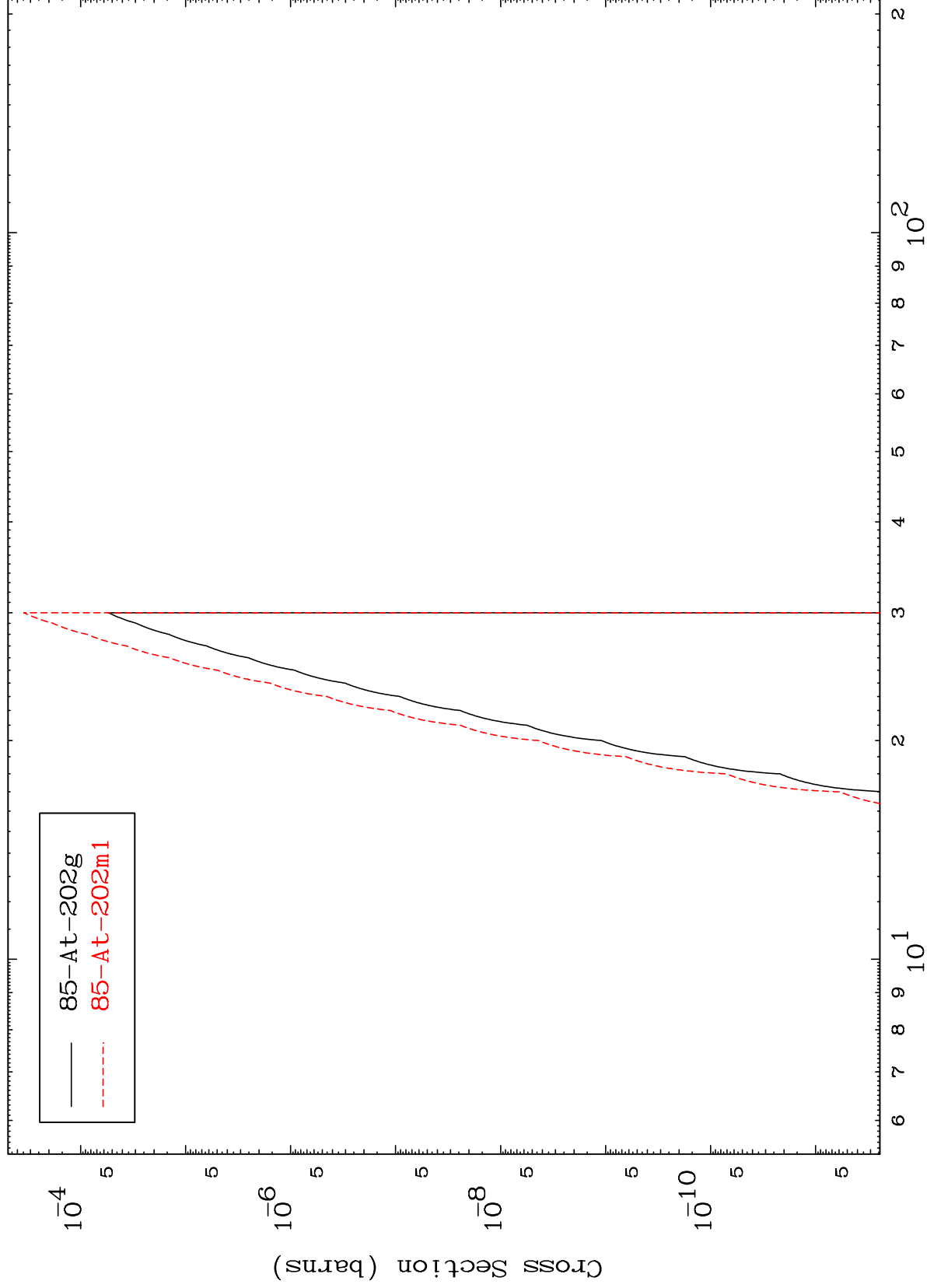
87-Fr-205

MAT 8704

(n,n') He-3

87-Fr-205

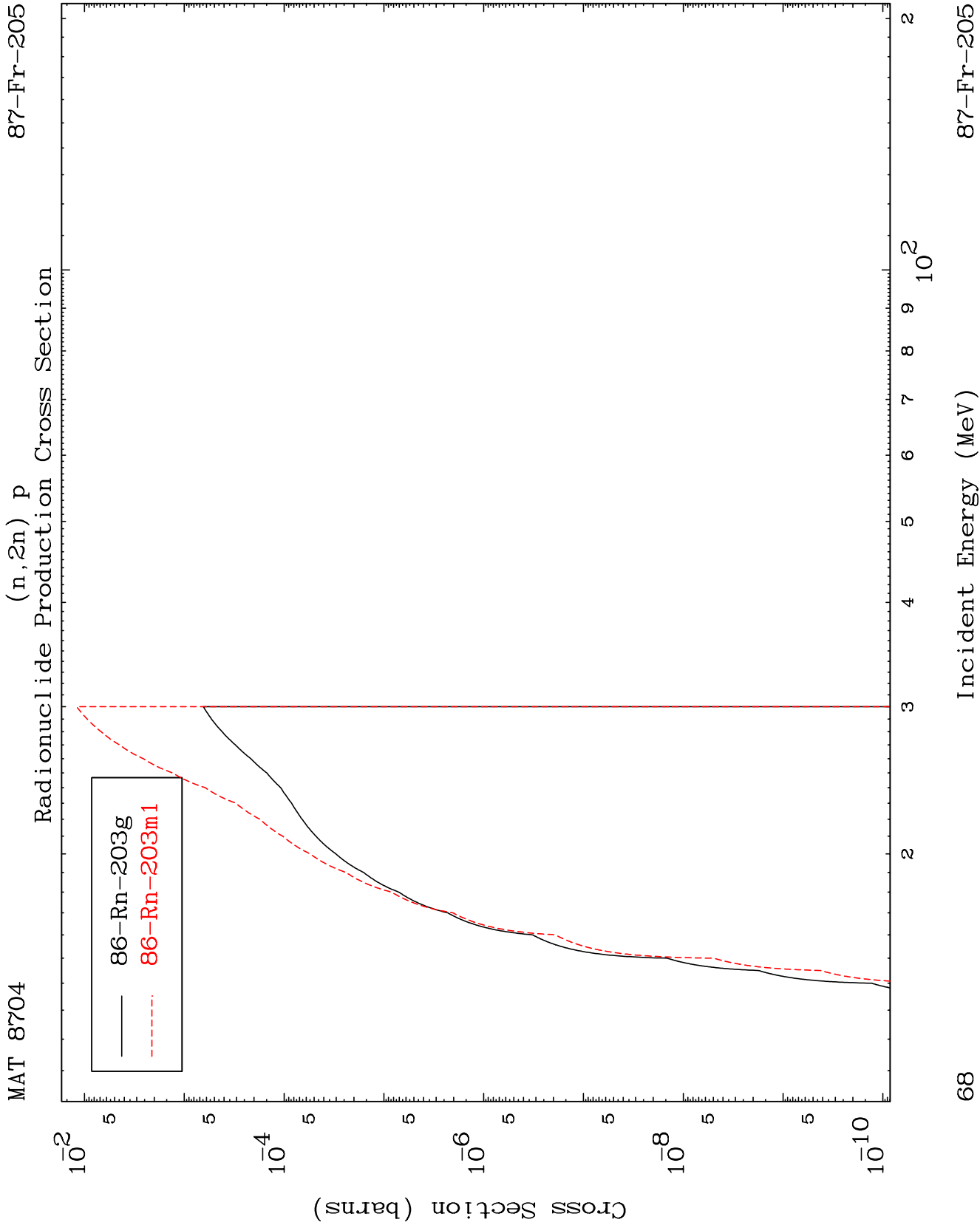
Radionuclide Production Cross Section



67

Incident Energy (MeV)

87-Fr-205

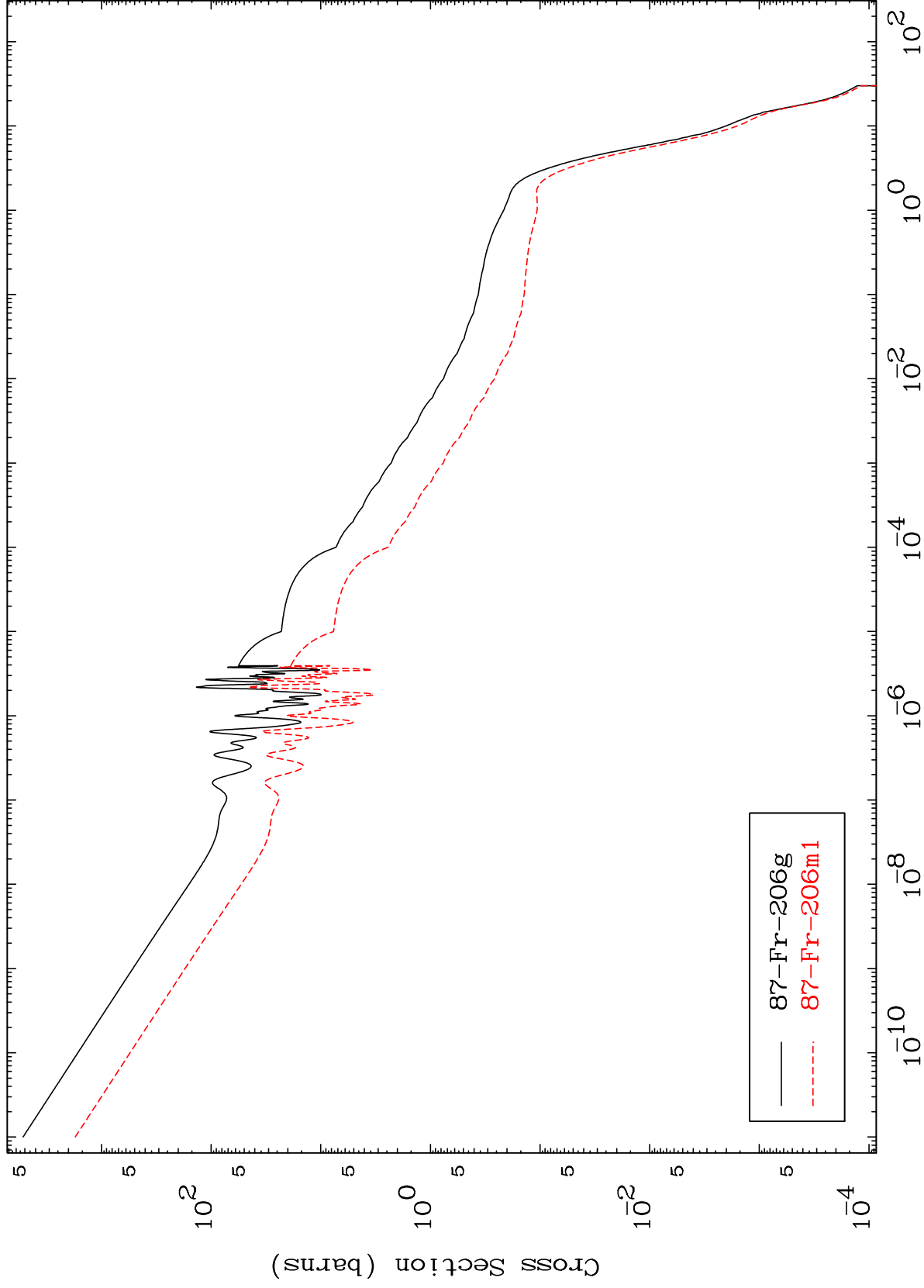


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87-Fr-205

Radionuclide Production Cross Section

(n, γ)



69

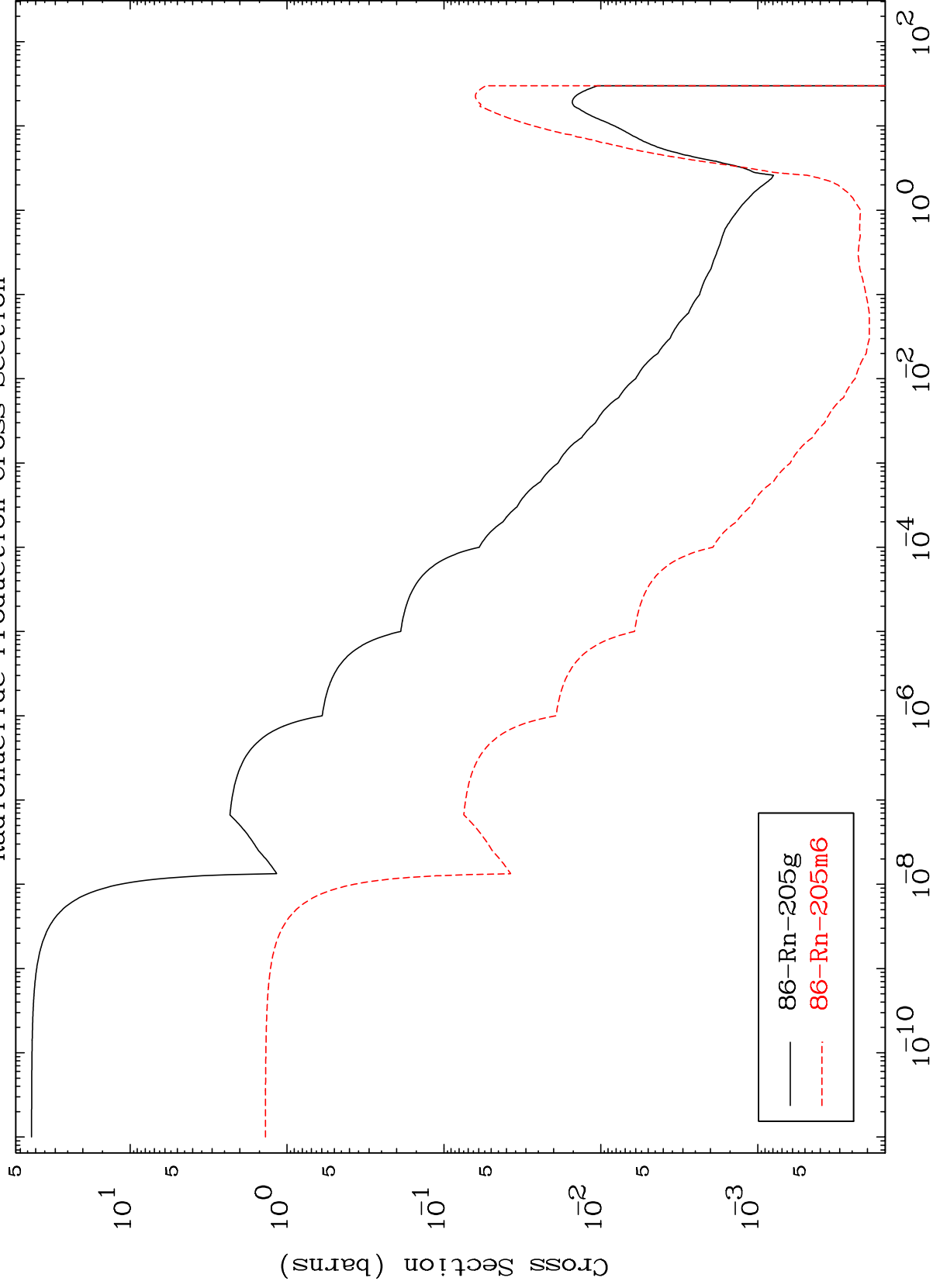
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MAT 8704

87-Fr-205

Radionuclide Production Cross Section

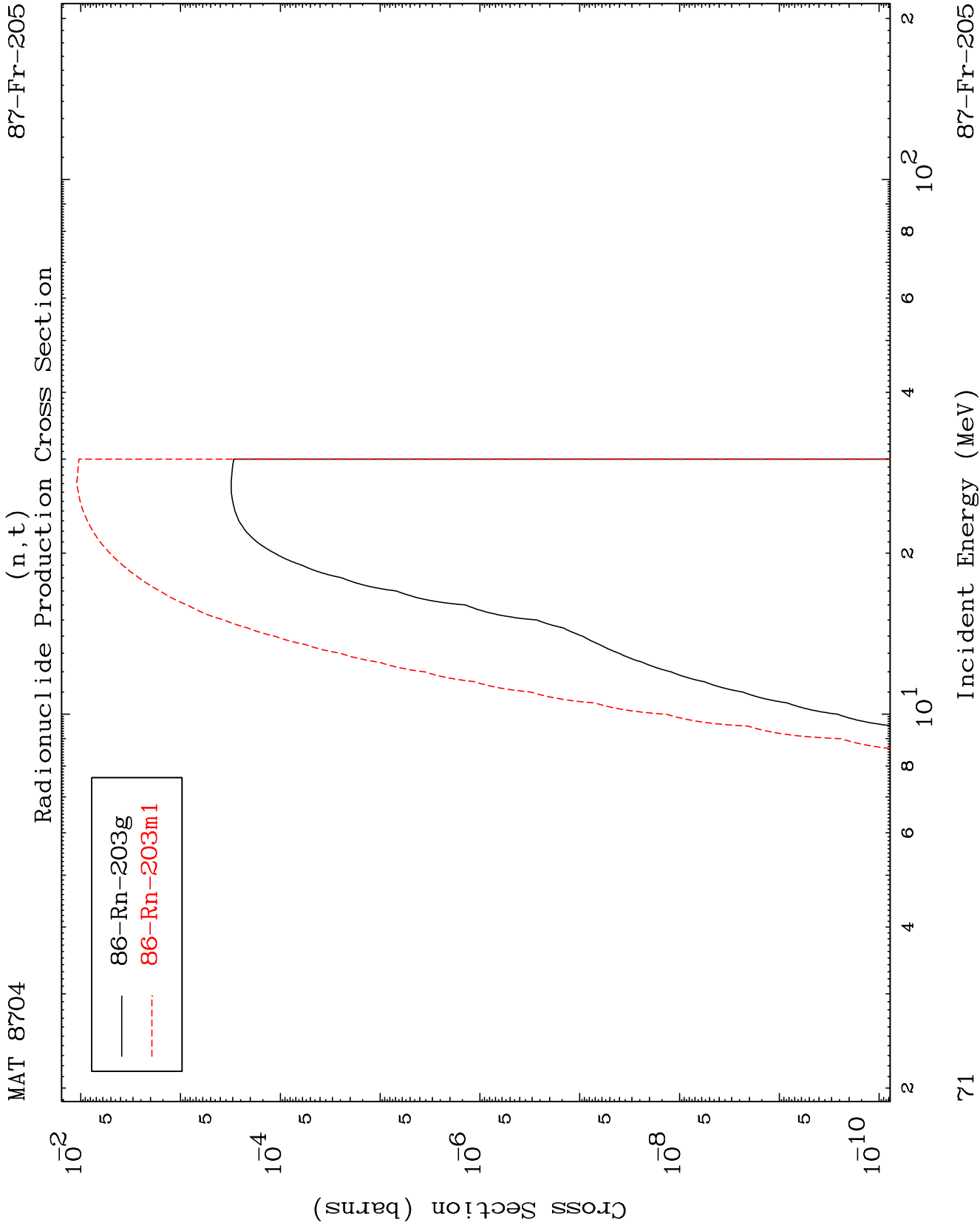
(n,p)

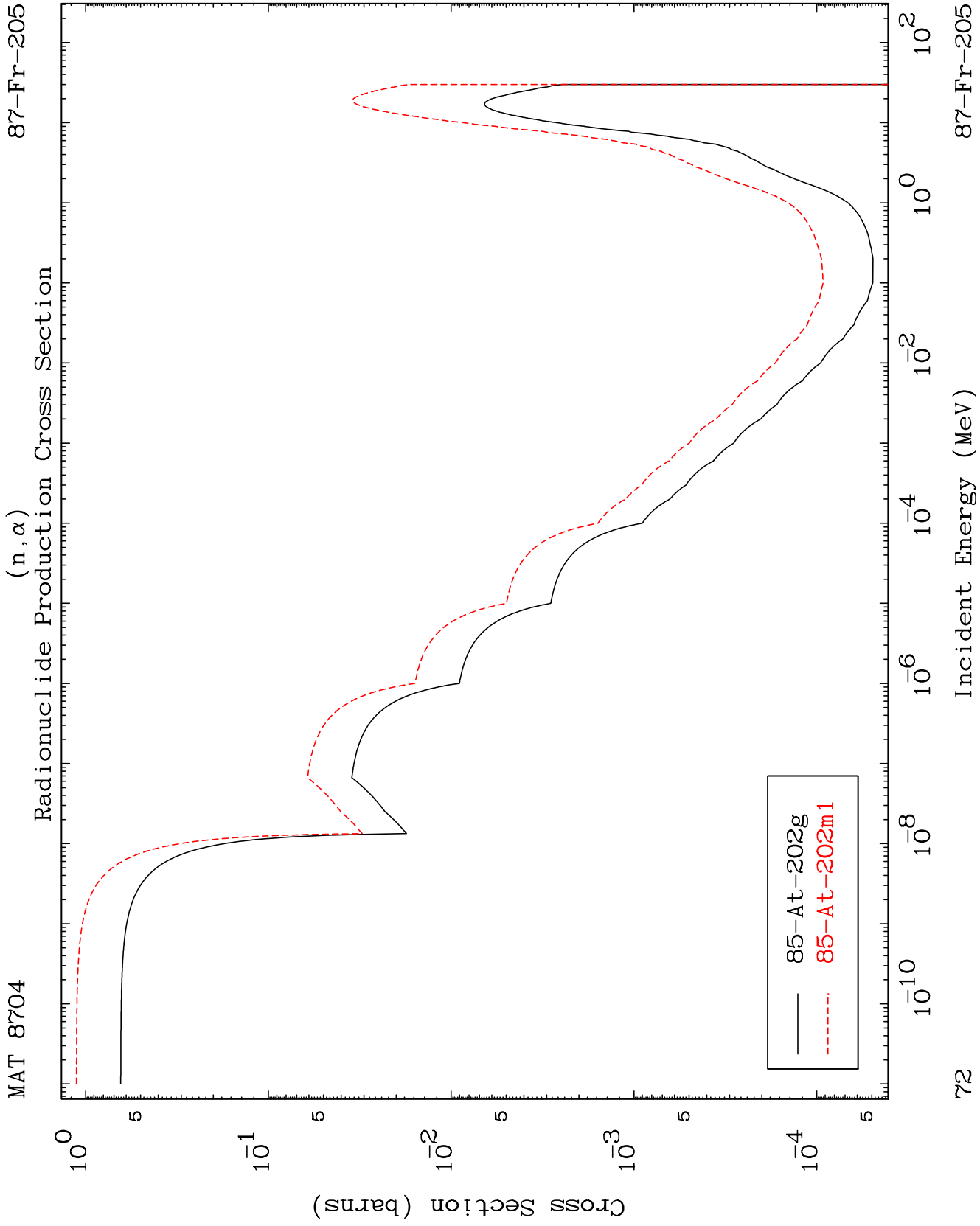


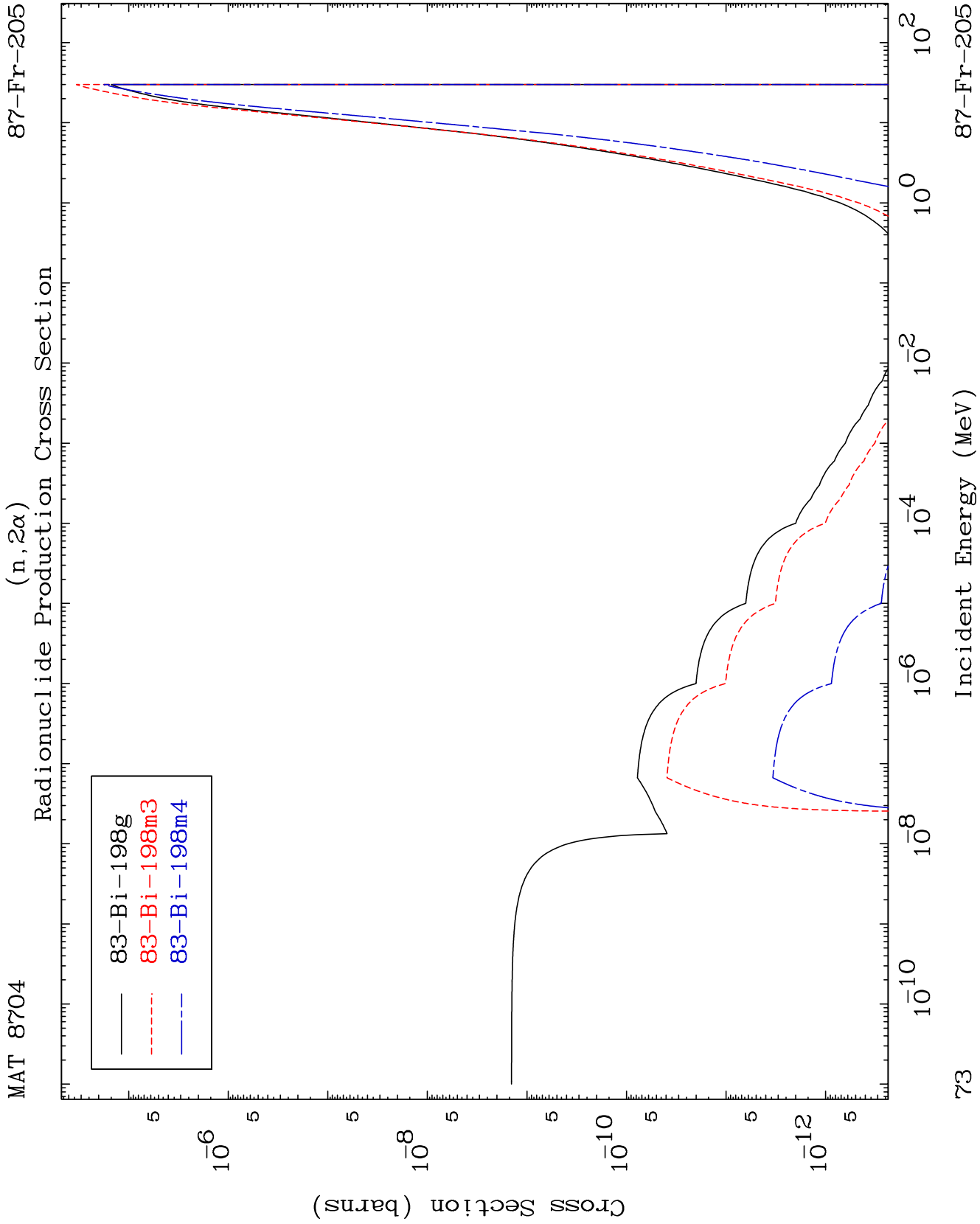
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Incident Energy (MeV)

87-Fr-205



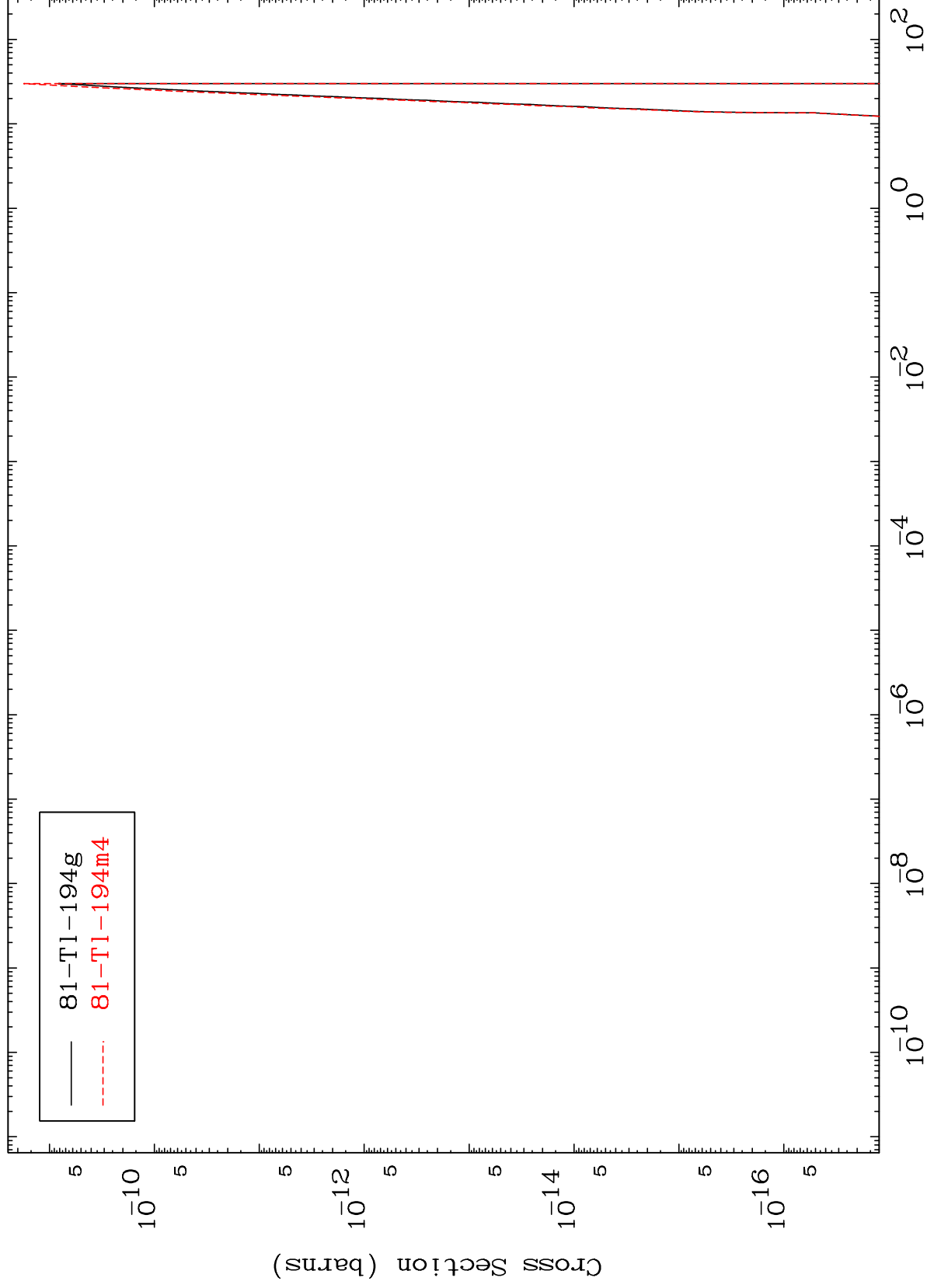




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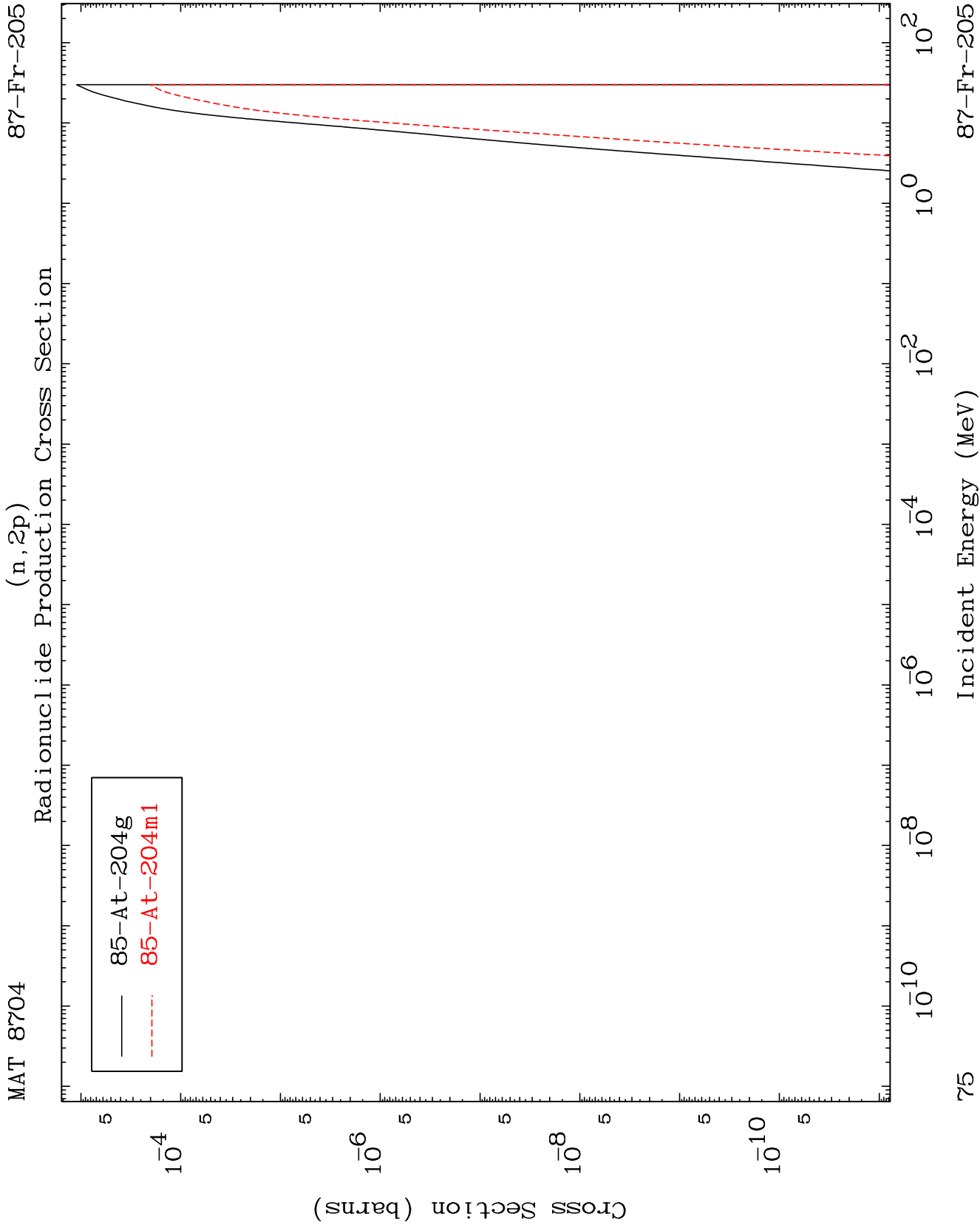
87-Fr-205

(n,3 α)
Radionuclide Production Cross Section



74

87-Fr-205

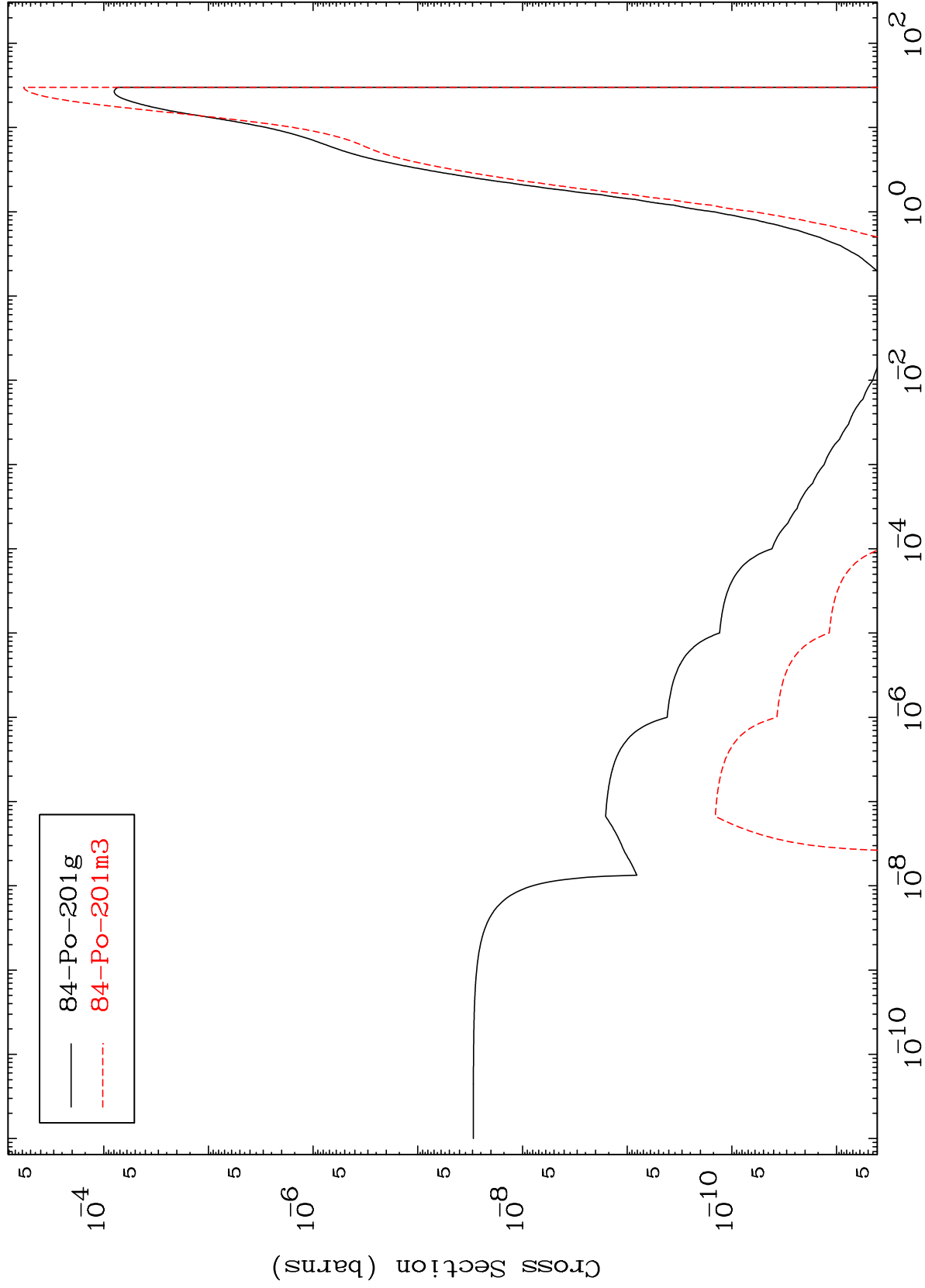


MAT 8704

(n,p) α

87-Fr-205

Radionuclide Production Cross Section



76

Incident Energy (MeV)

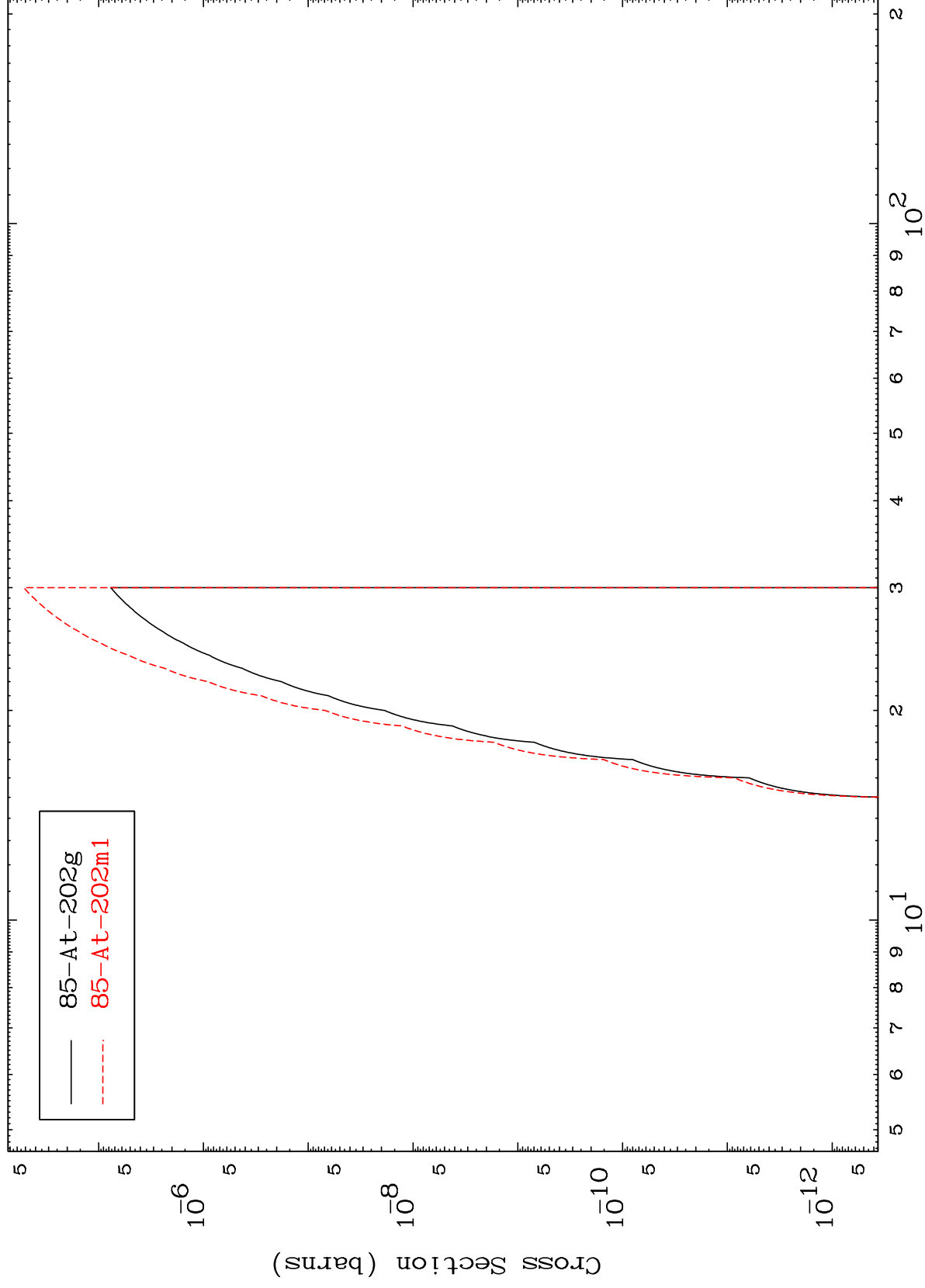
87-Fr-205

MAT 8704

(n,p) t

87-Fr-205

Radionuclide Production Cross Section



77

Incident Energy (MeV)

87-Fr-205