

Program EVALPLOT
(Version 2021-1)

by

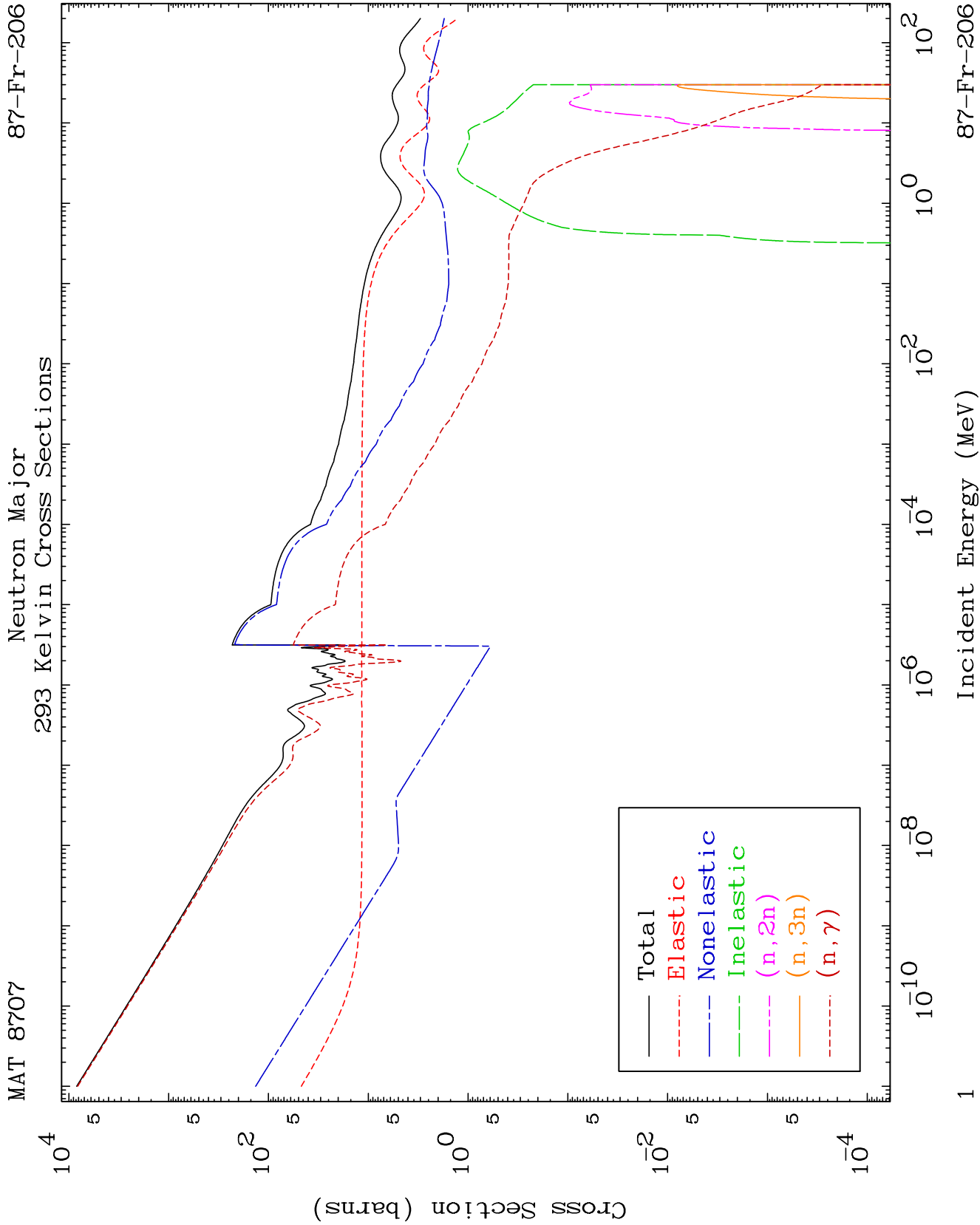
Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

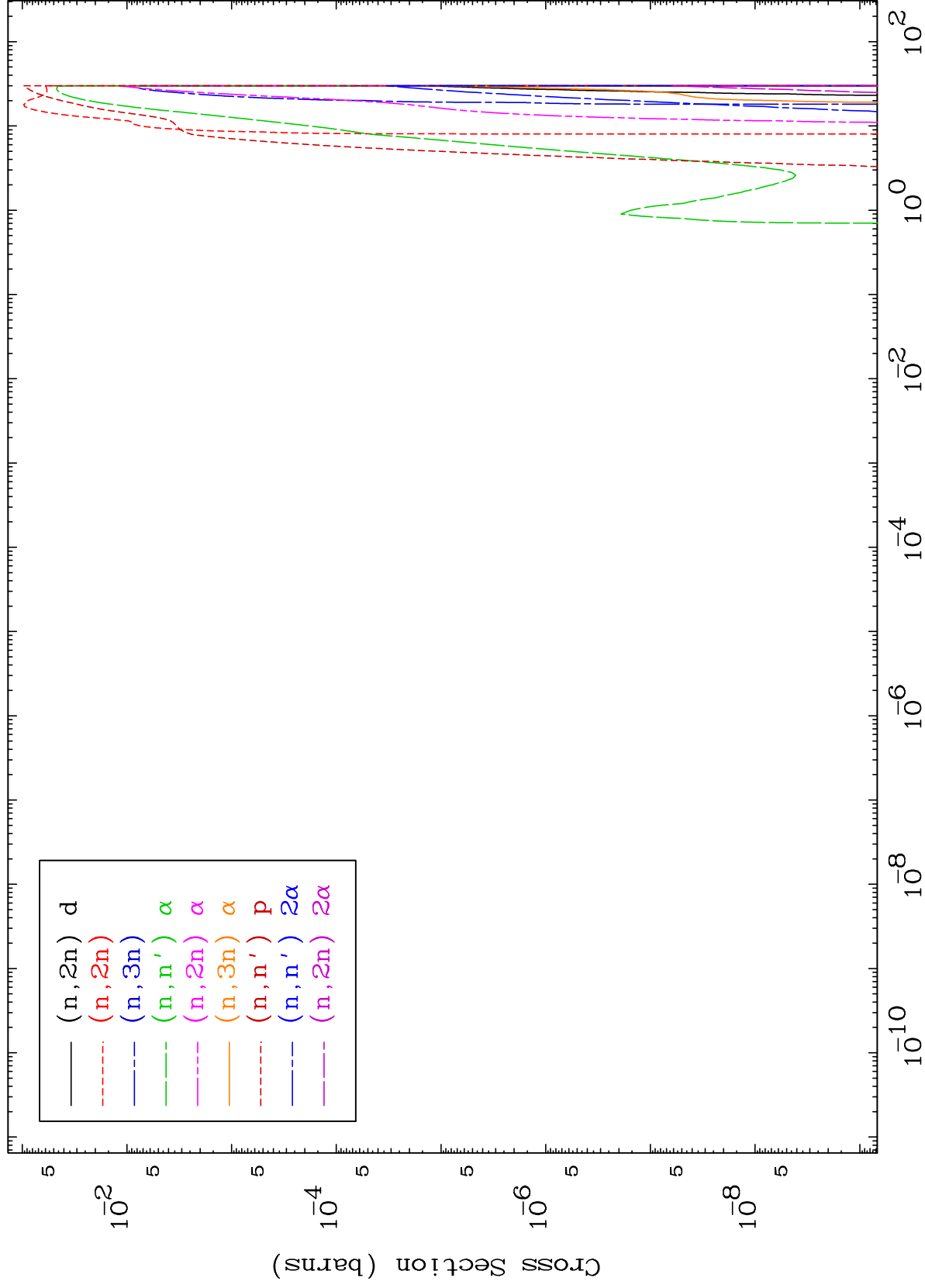
Press Mouse Button to Start

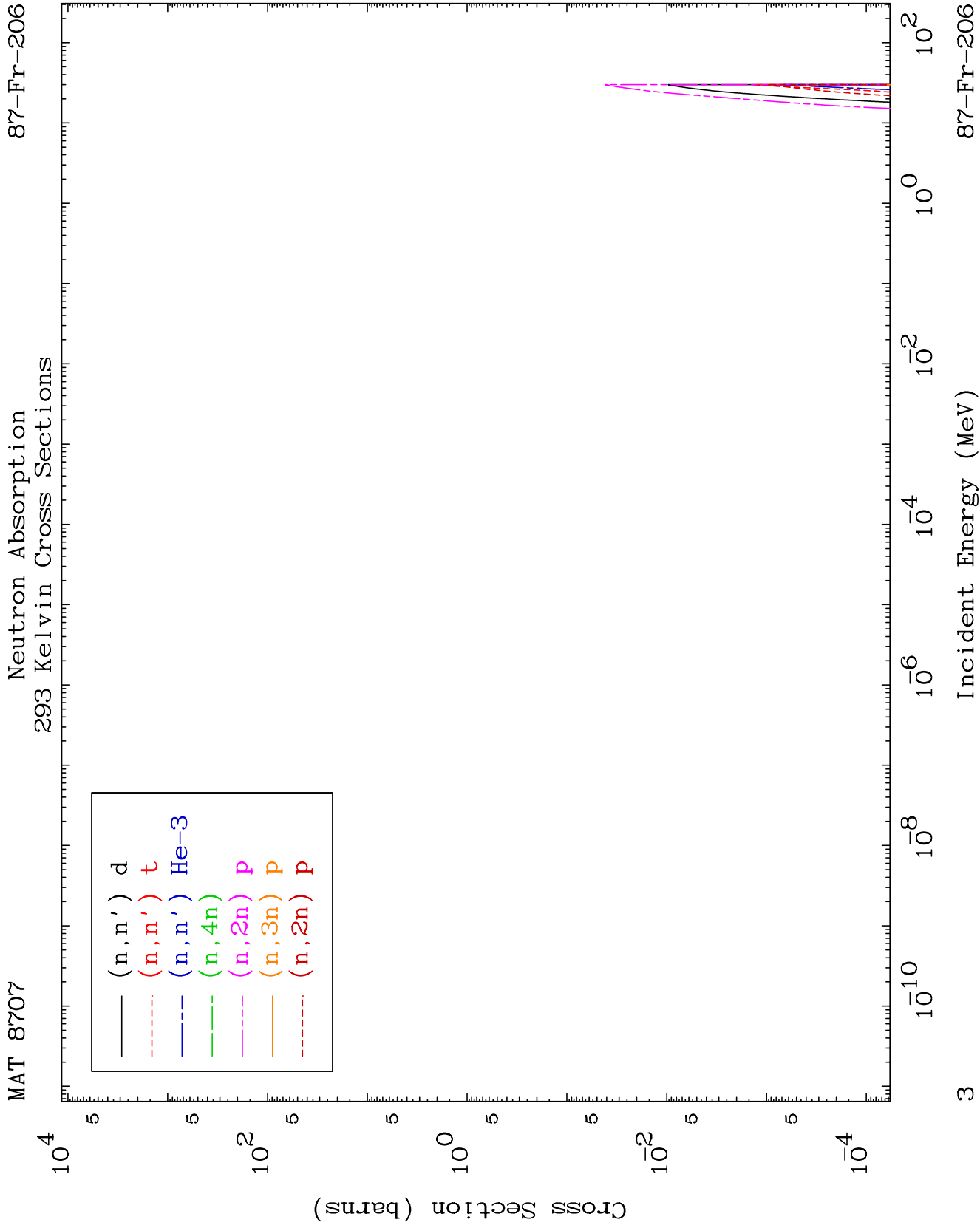


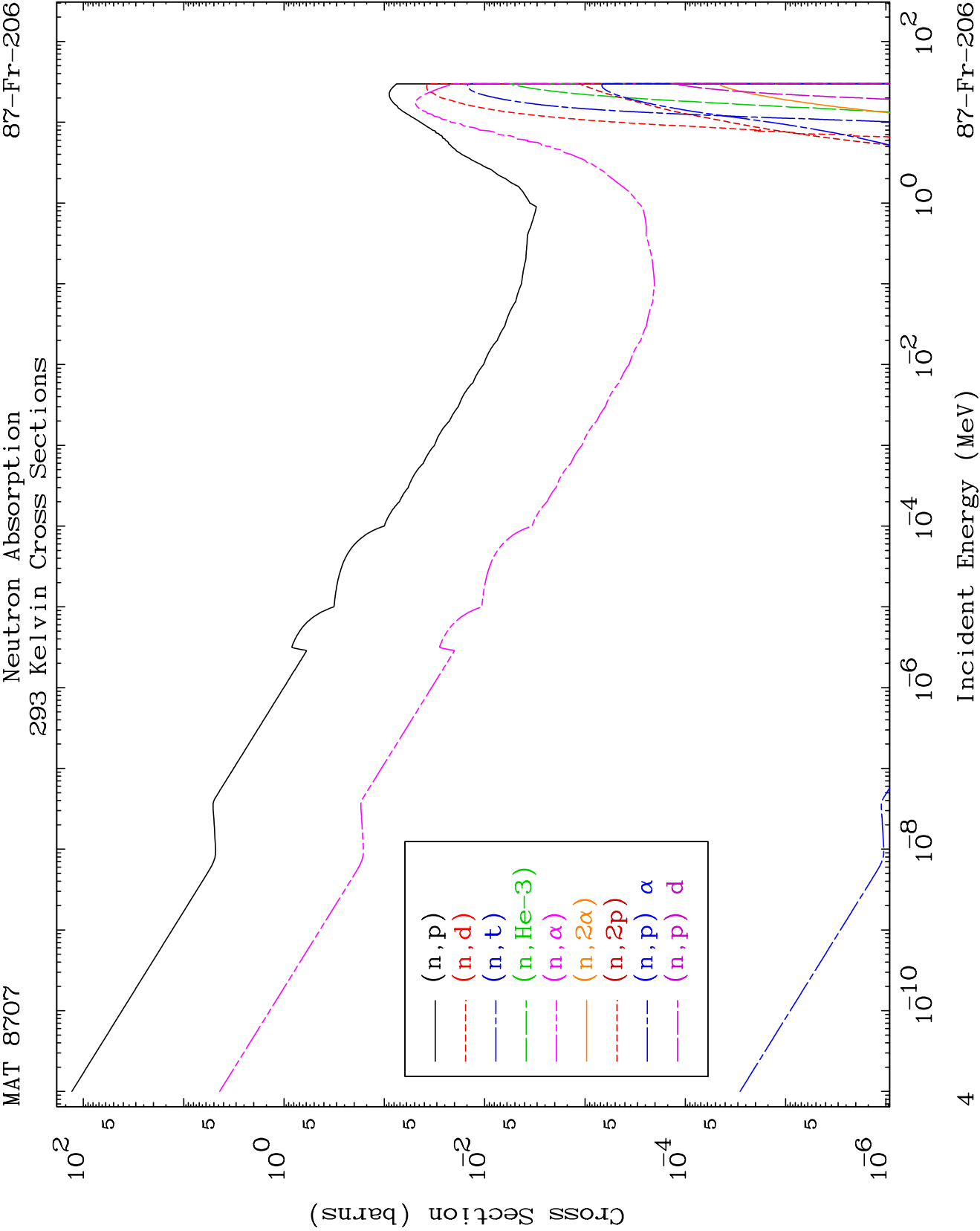
MAT 8707

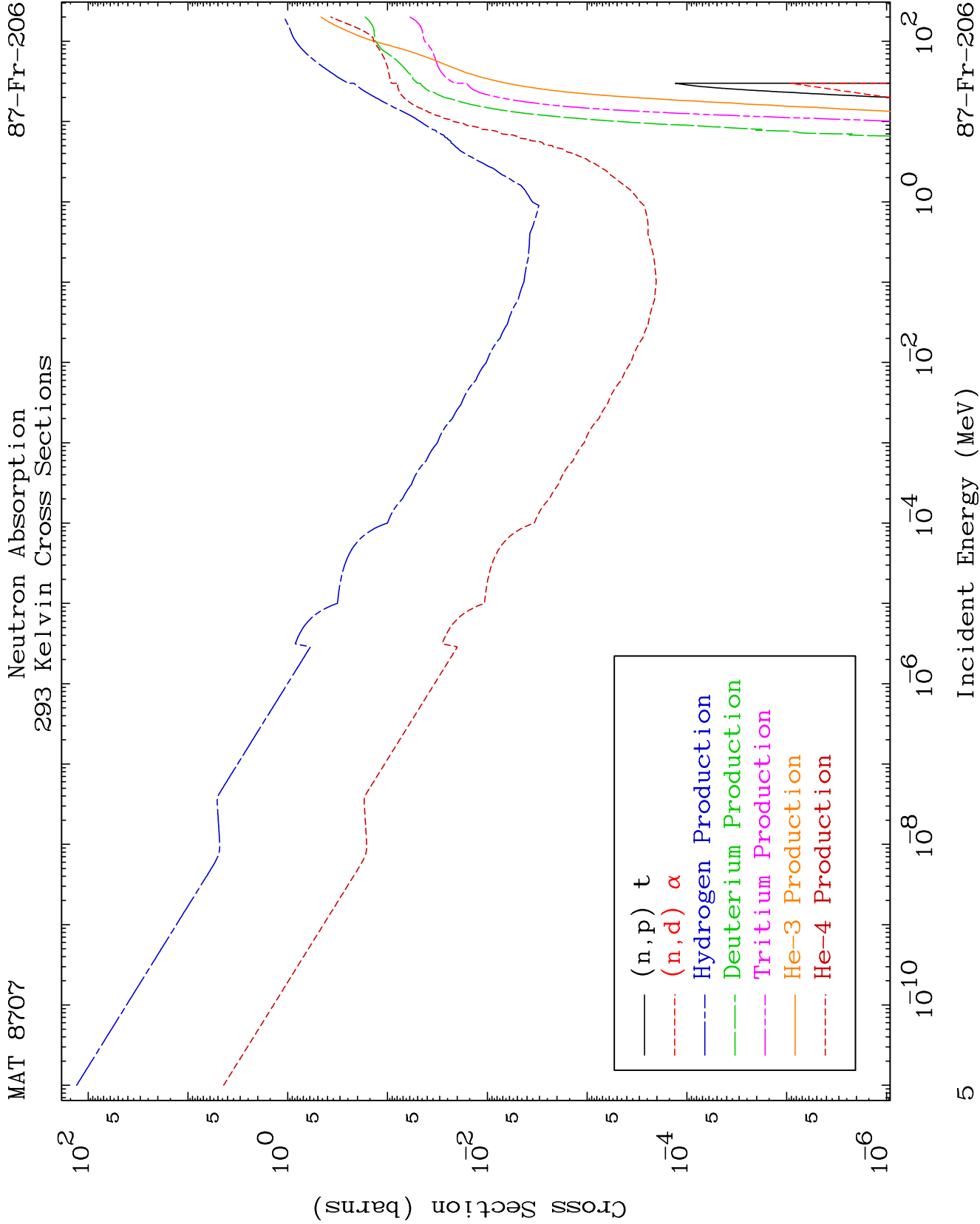
Neutron Absorption
293 Kelvin Cross Sections

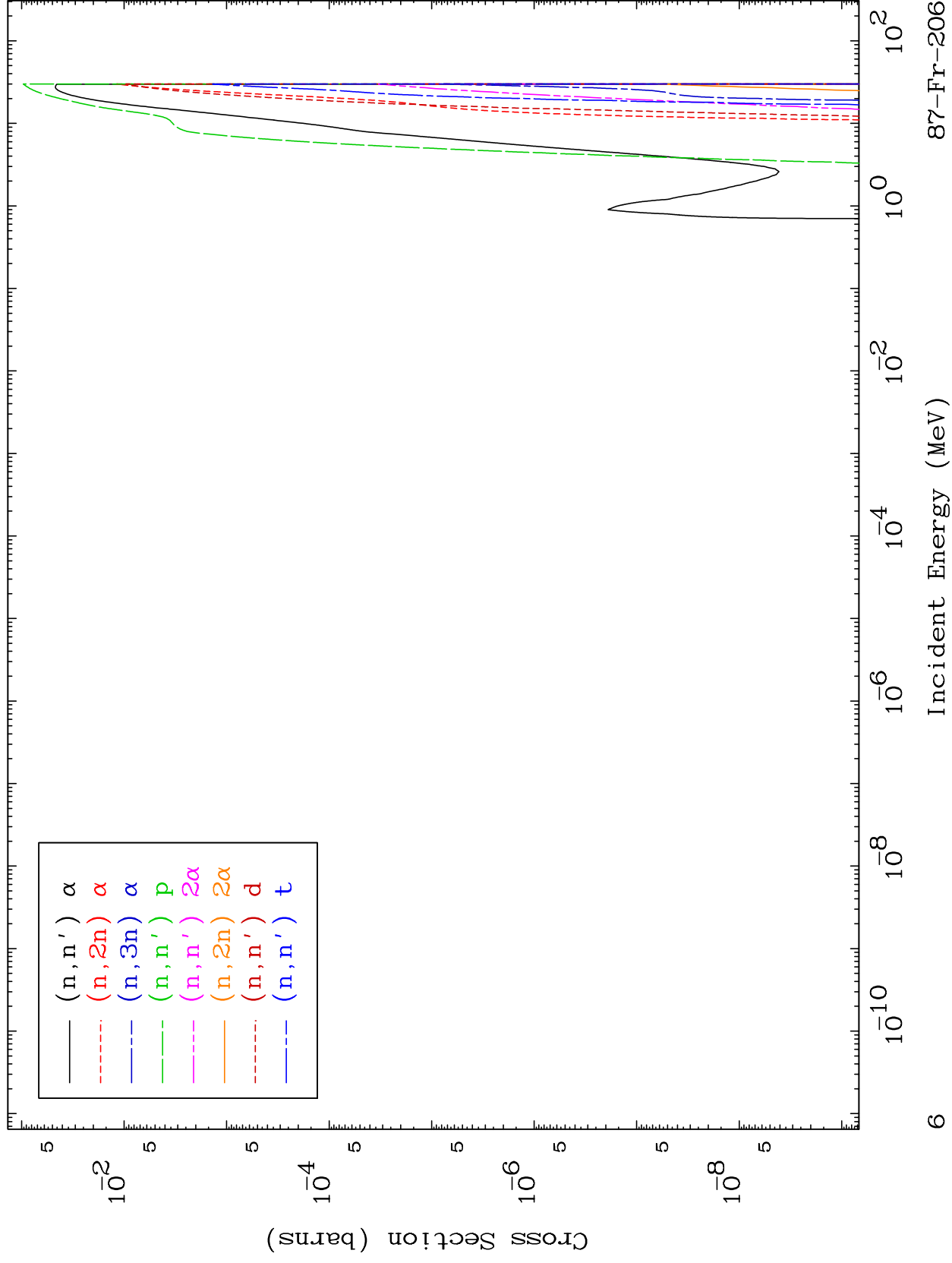
87-Fr-206

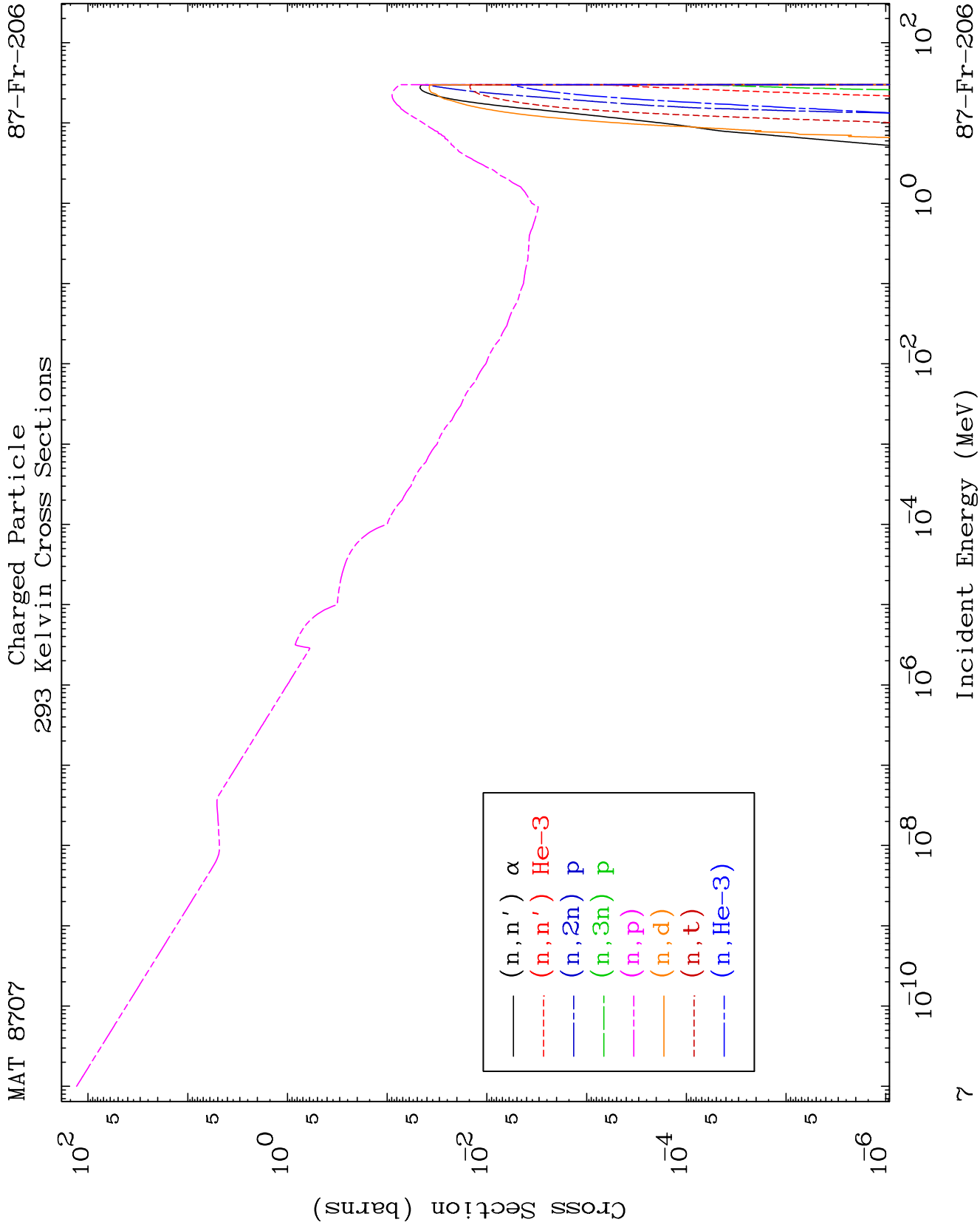


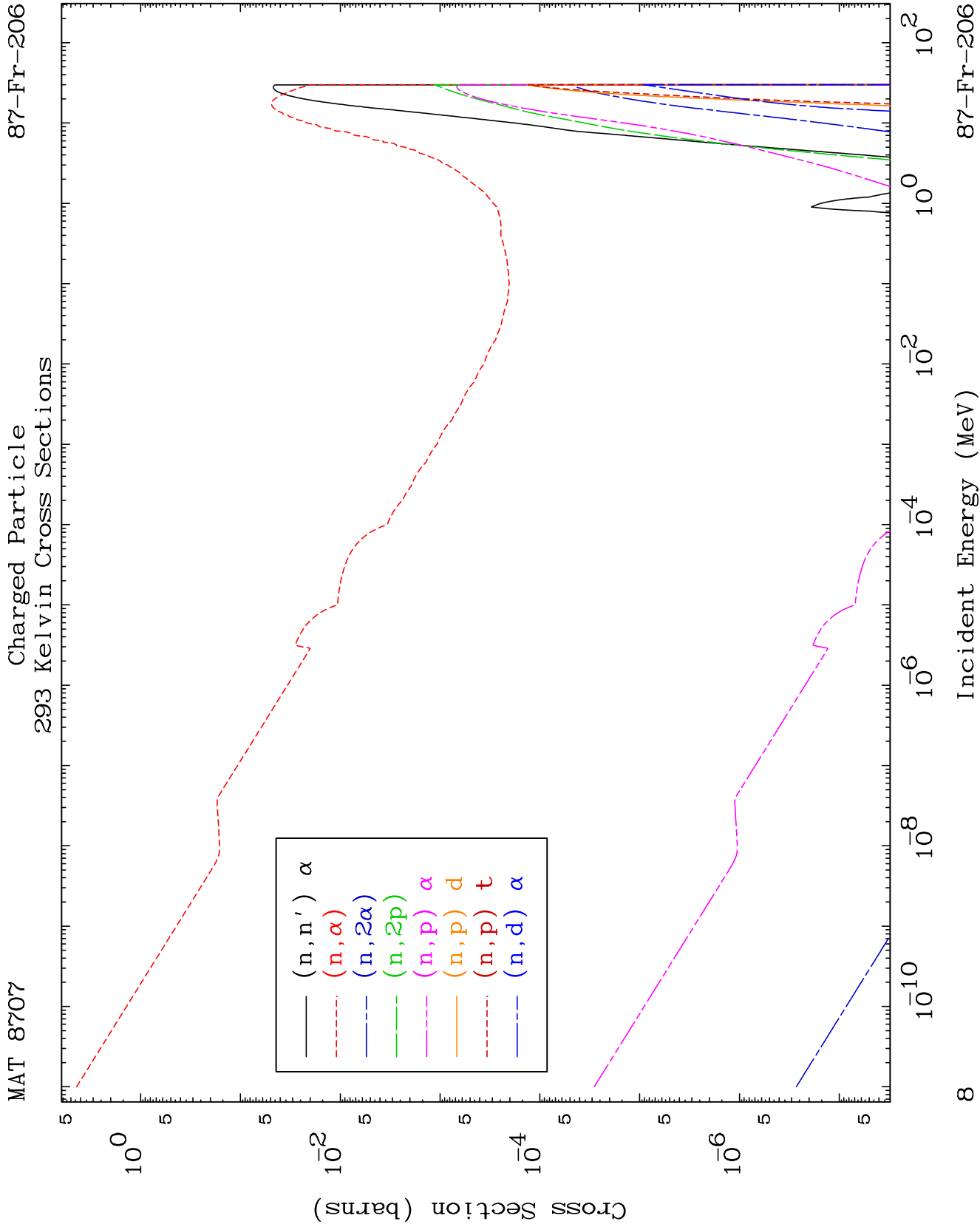


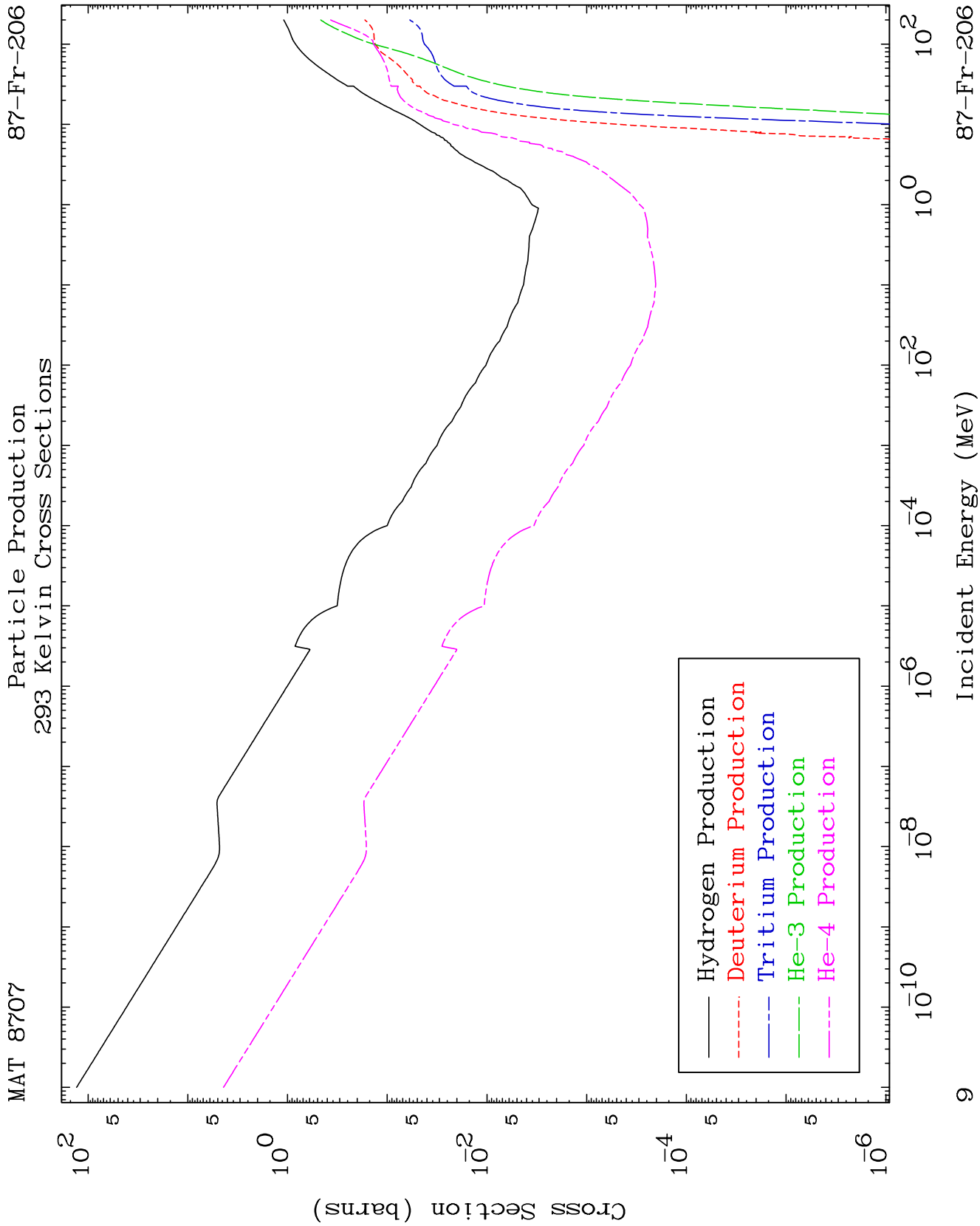








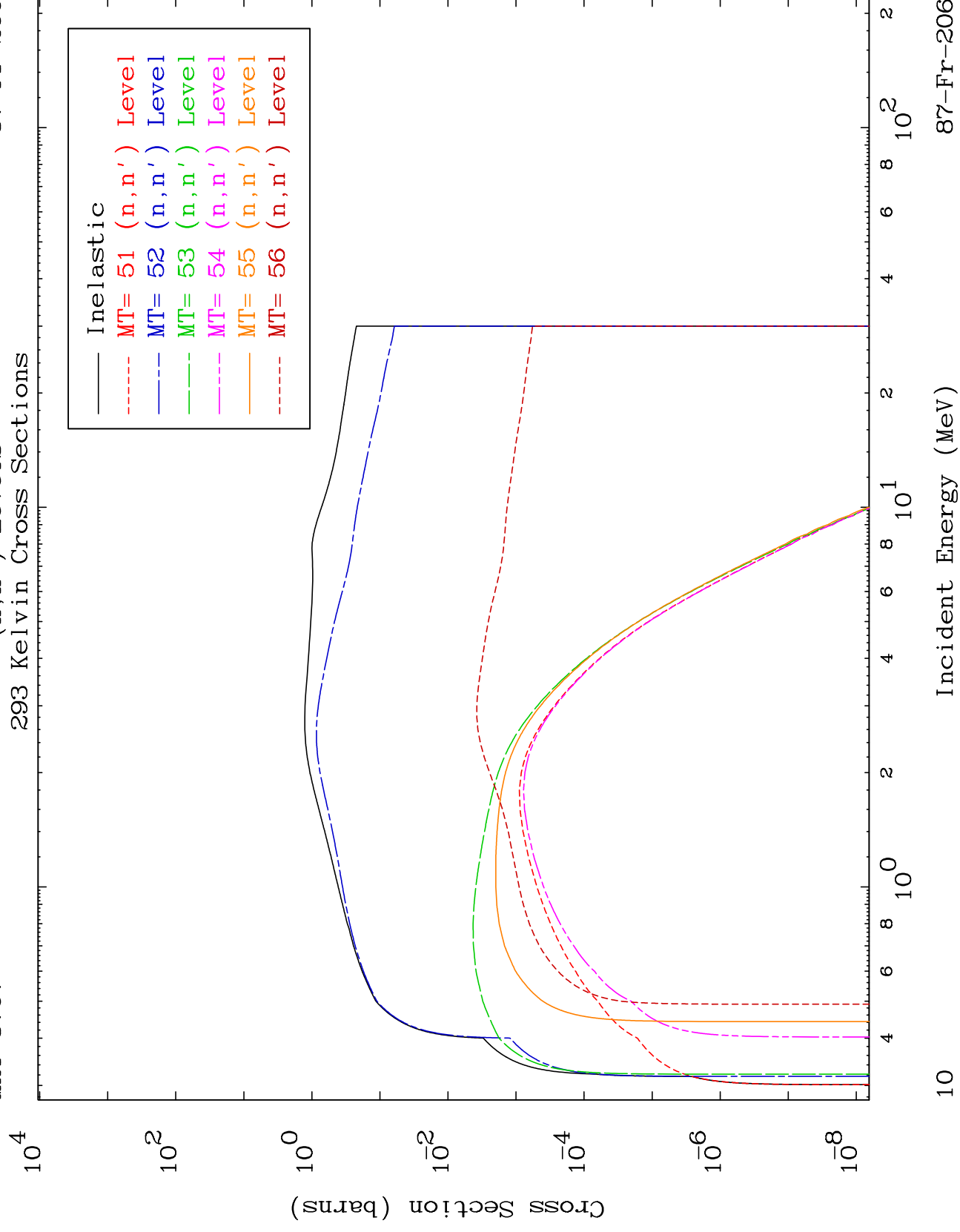




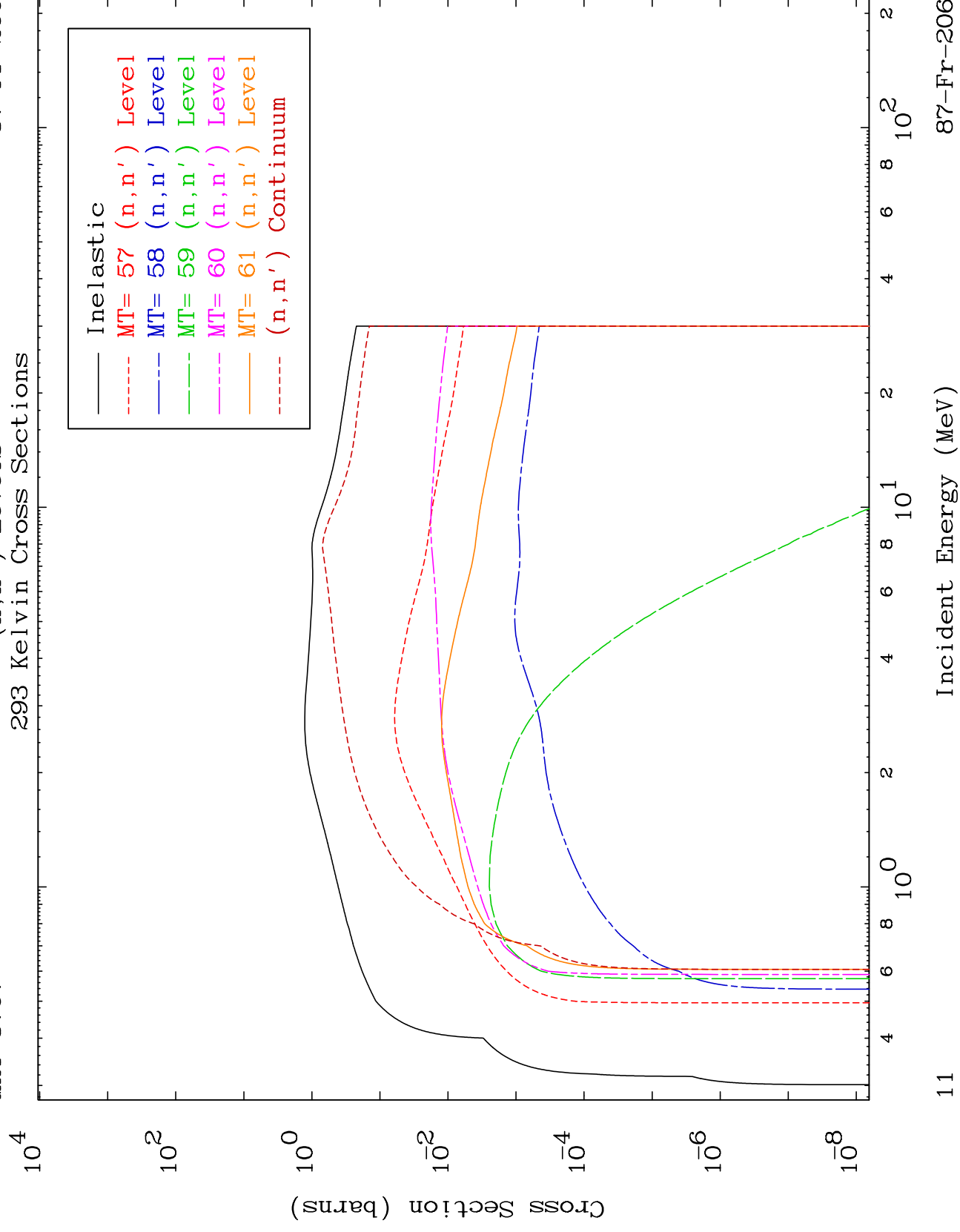
MAT 8707

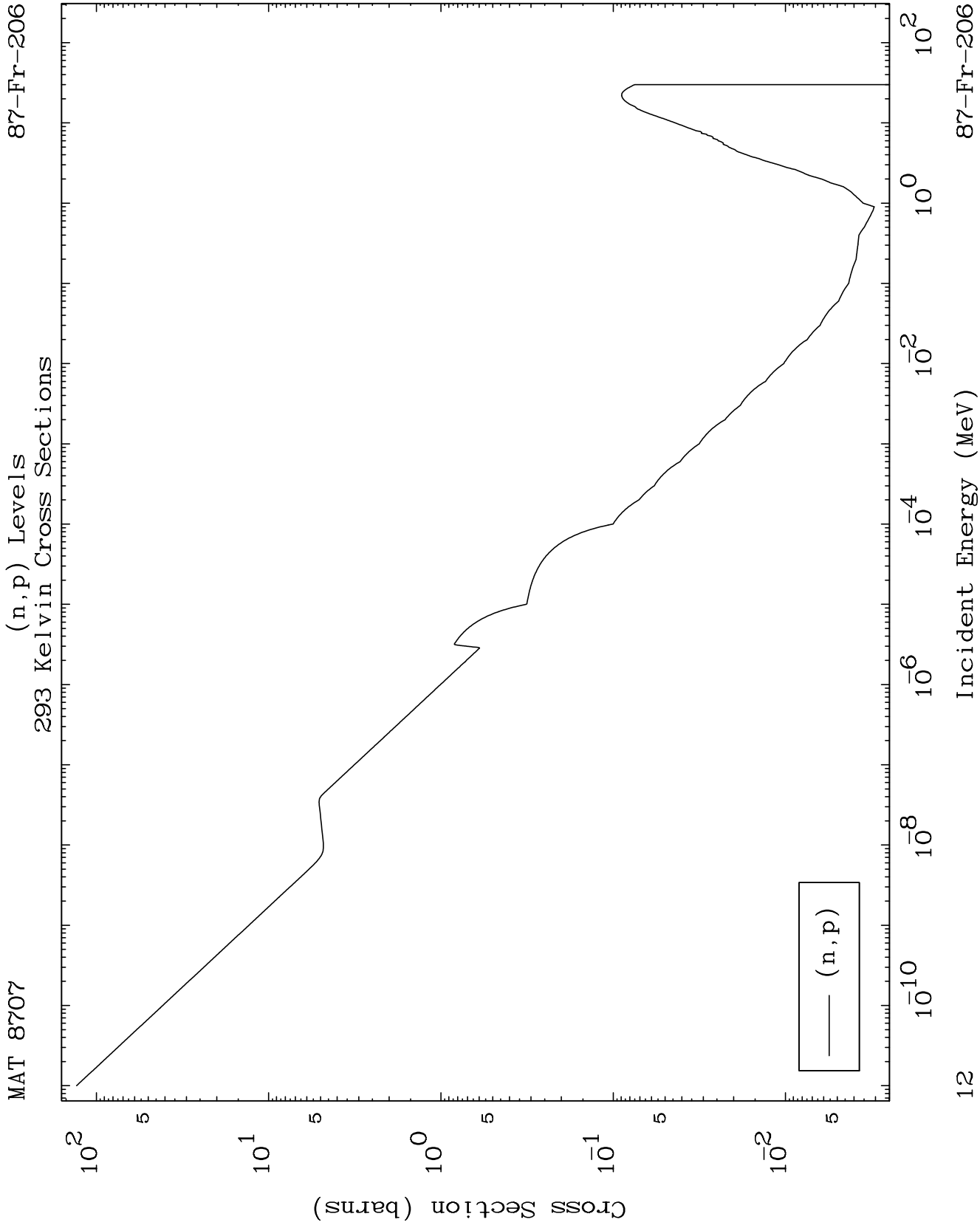
87-Fr-206

(n, n') Levels
293 Kelvin Cross Sections



(n, n') Levels
293 Kelvin Cross Sections

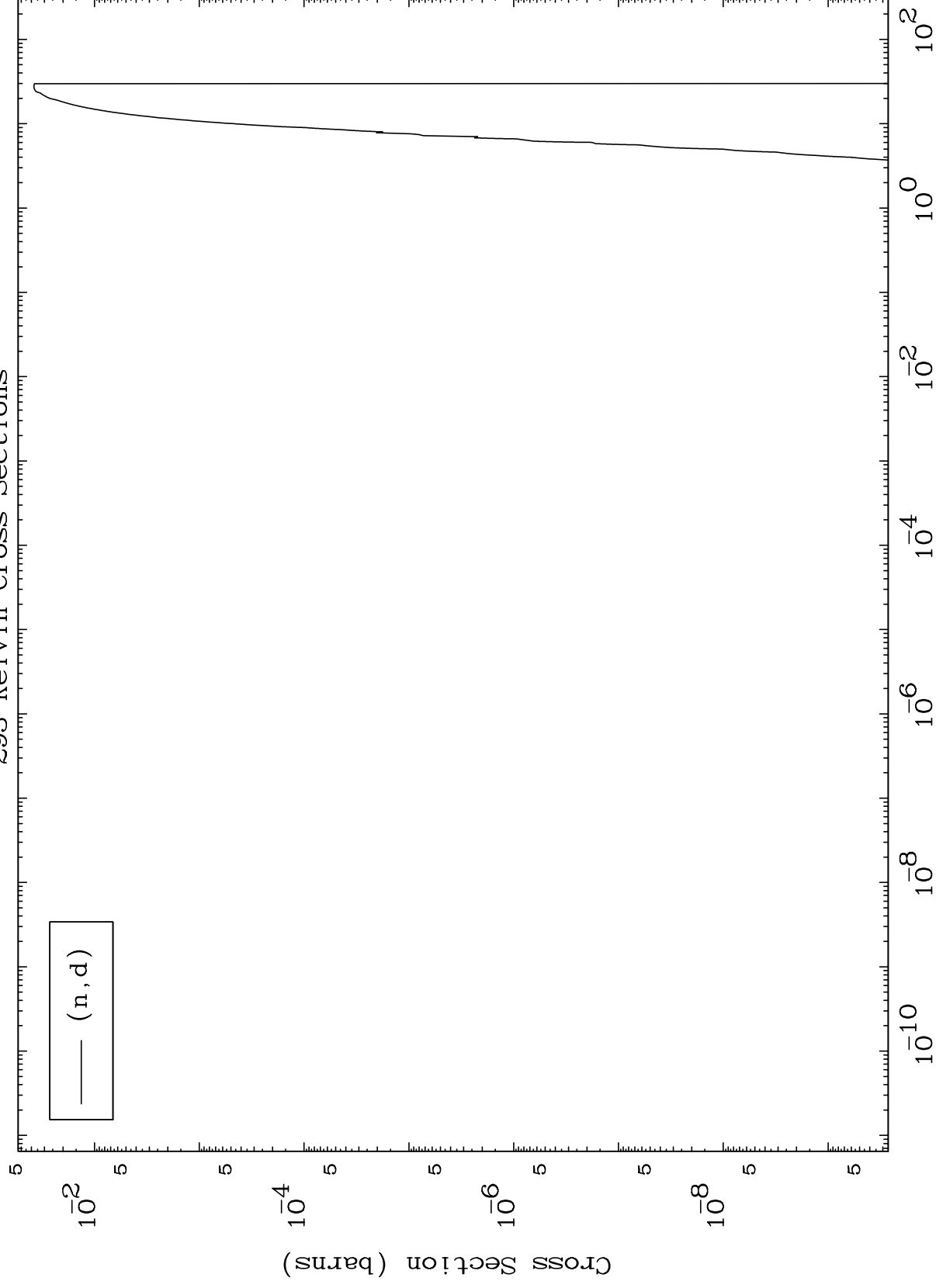




MAT 8707

(n,d) Levels
293 Kelvin Cross Sections

87-Fr-206



13

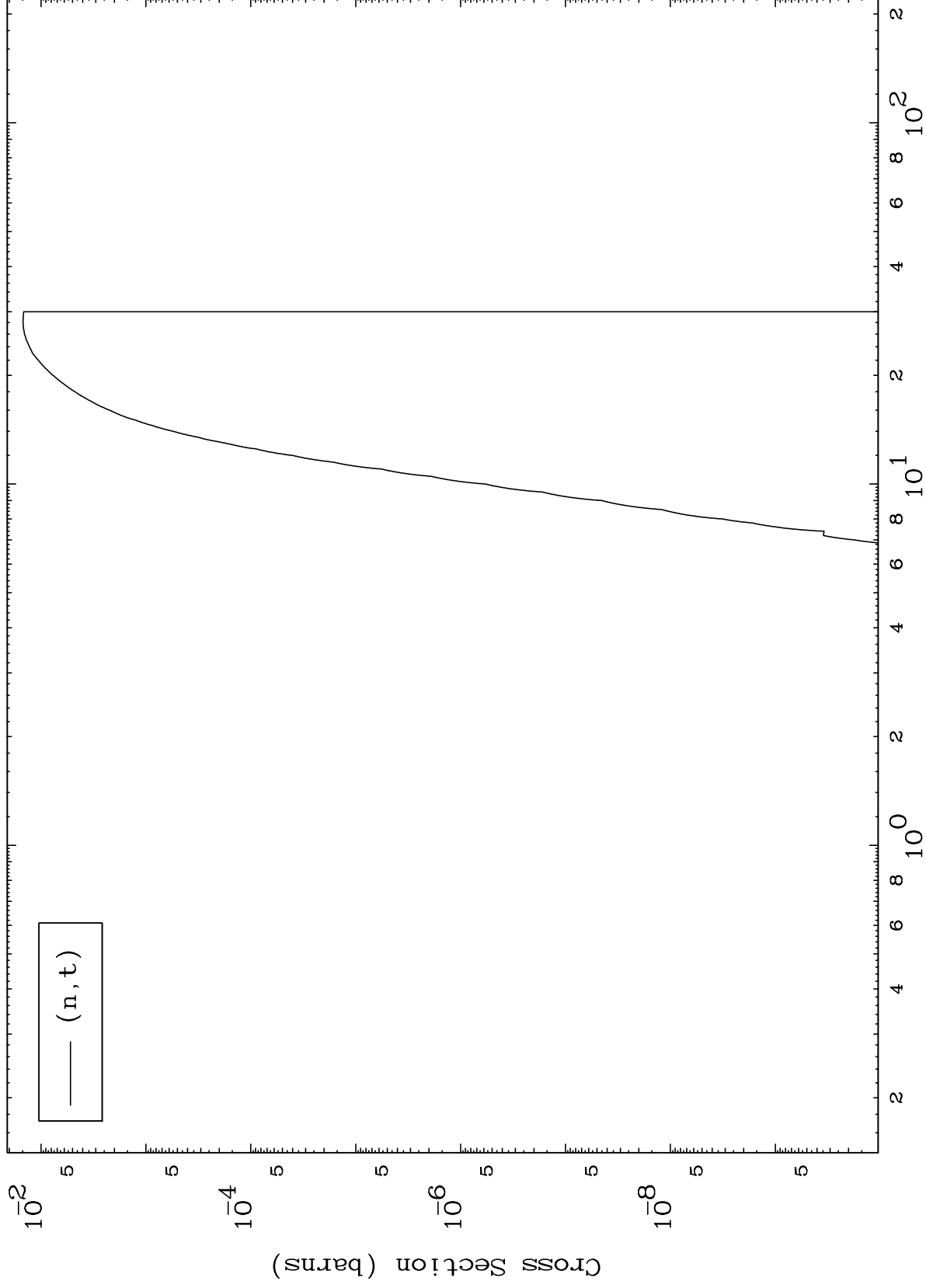
Incident Energy (MeV)

87-Fr-206

MAT 8707

87-Fr-206

(n,t) Levels
293 Kelvin Cross Sections



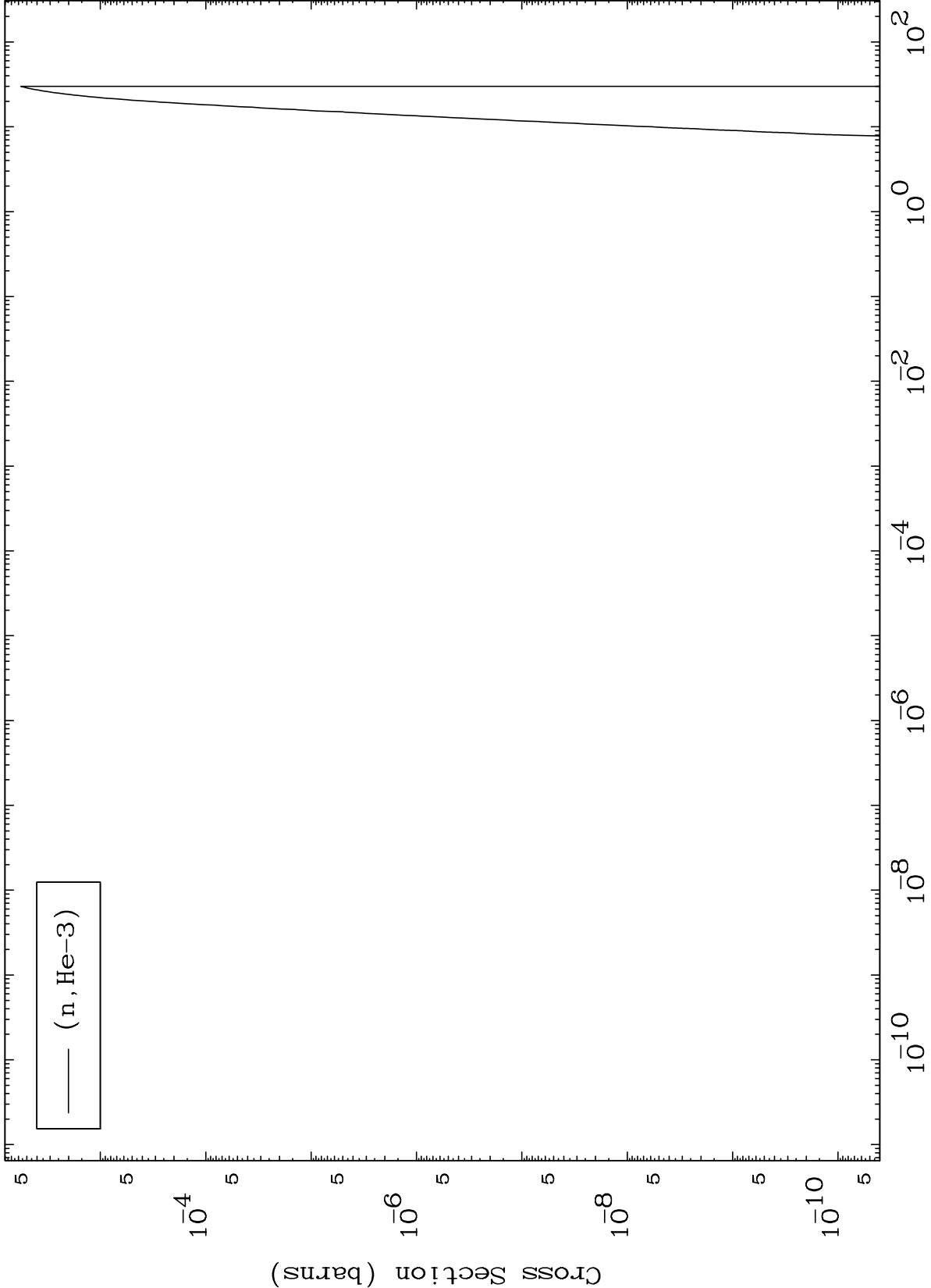
14

87-Fr-206

MAT 8707

(n,He3) Levels
293 Kelvin Cross Sections

87-Fr-206



15

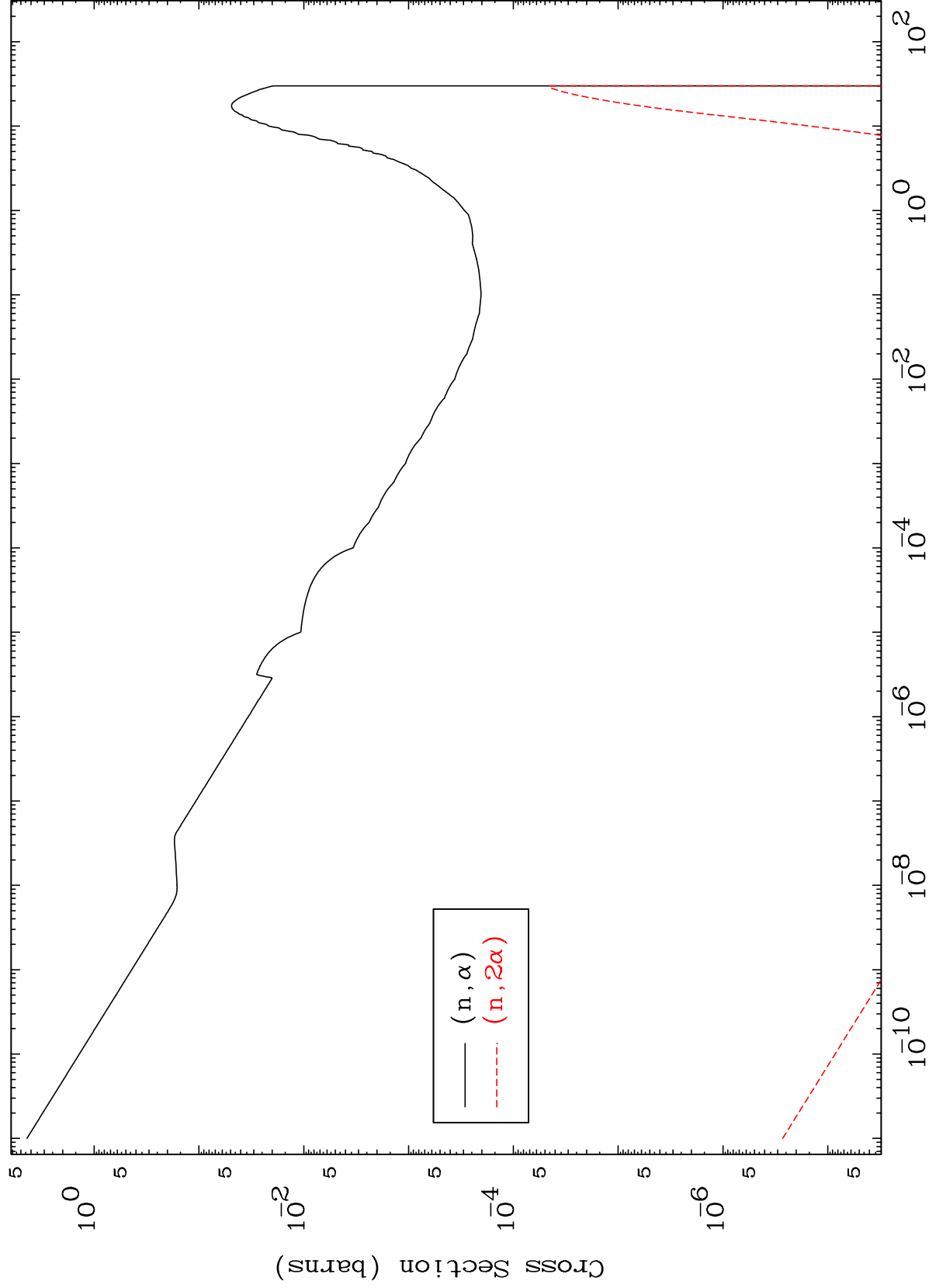
Incident Energy (MeV)

87-Fr-206

MAT 8707

(n, α) Levels
293 Kelvin Cross Sections

87-Fr-206



16

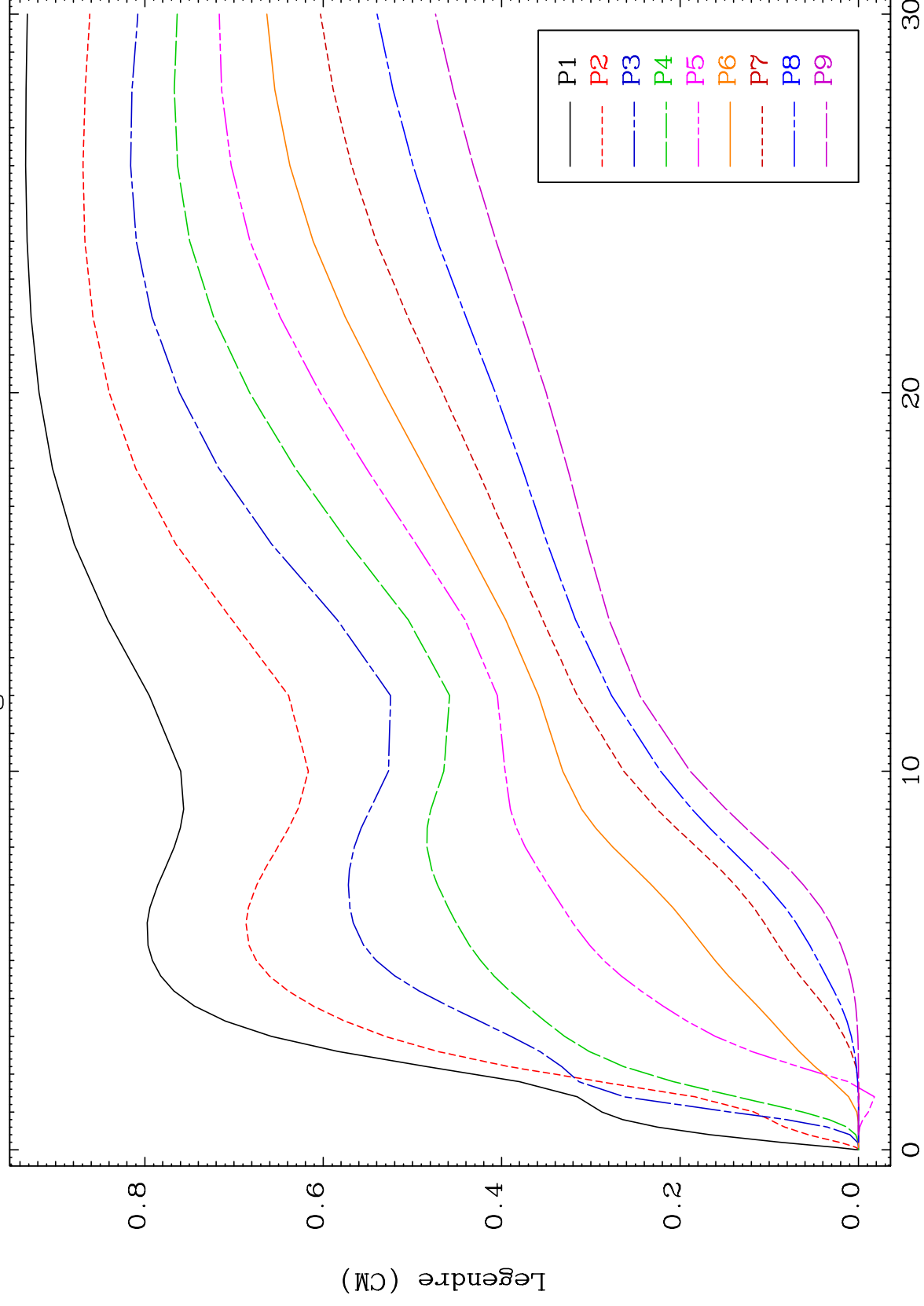
Incident Energy (MeV)

87-Fr-206

MAT 8707

87-Fr-206

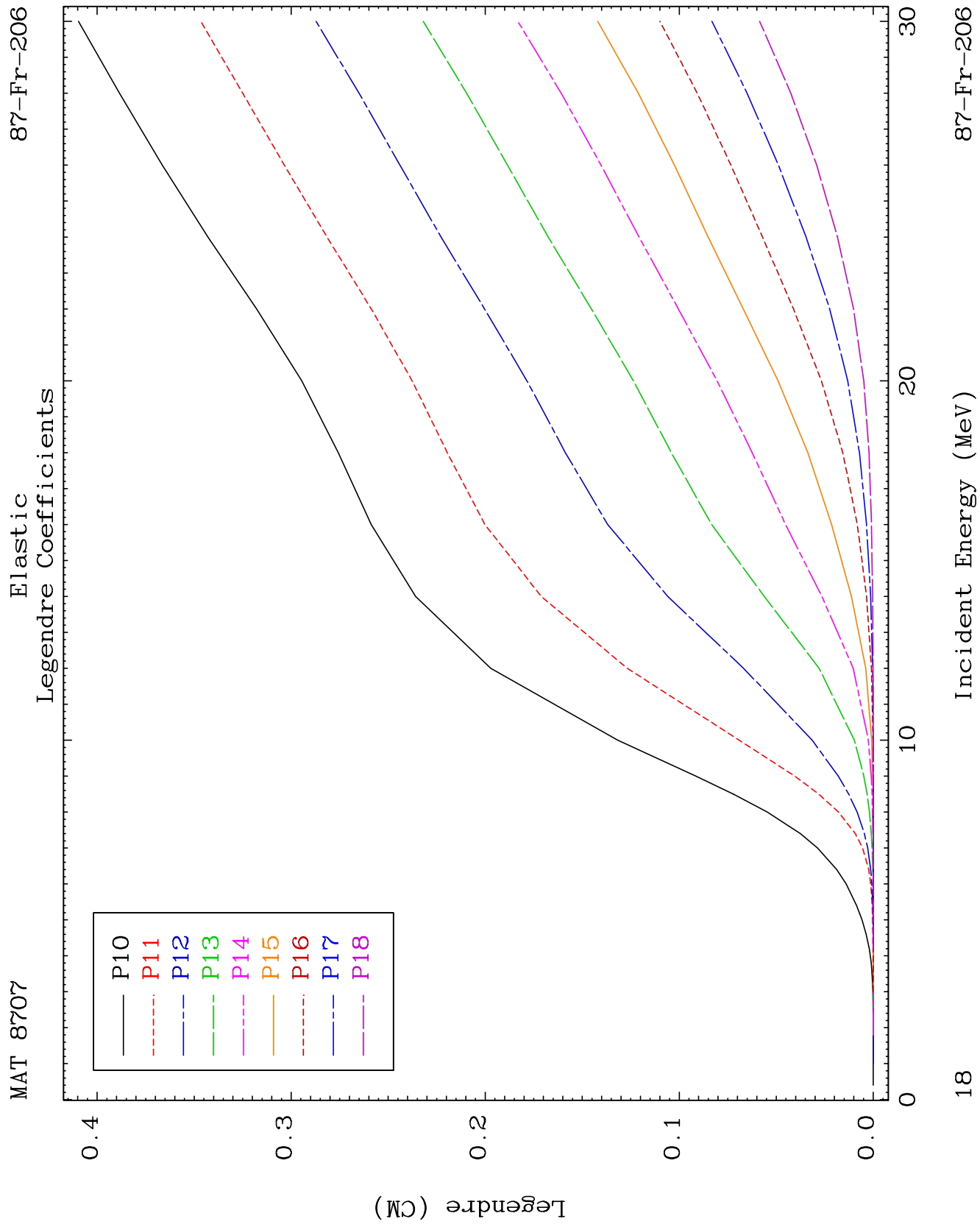
Elastic Legendre Coefficients

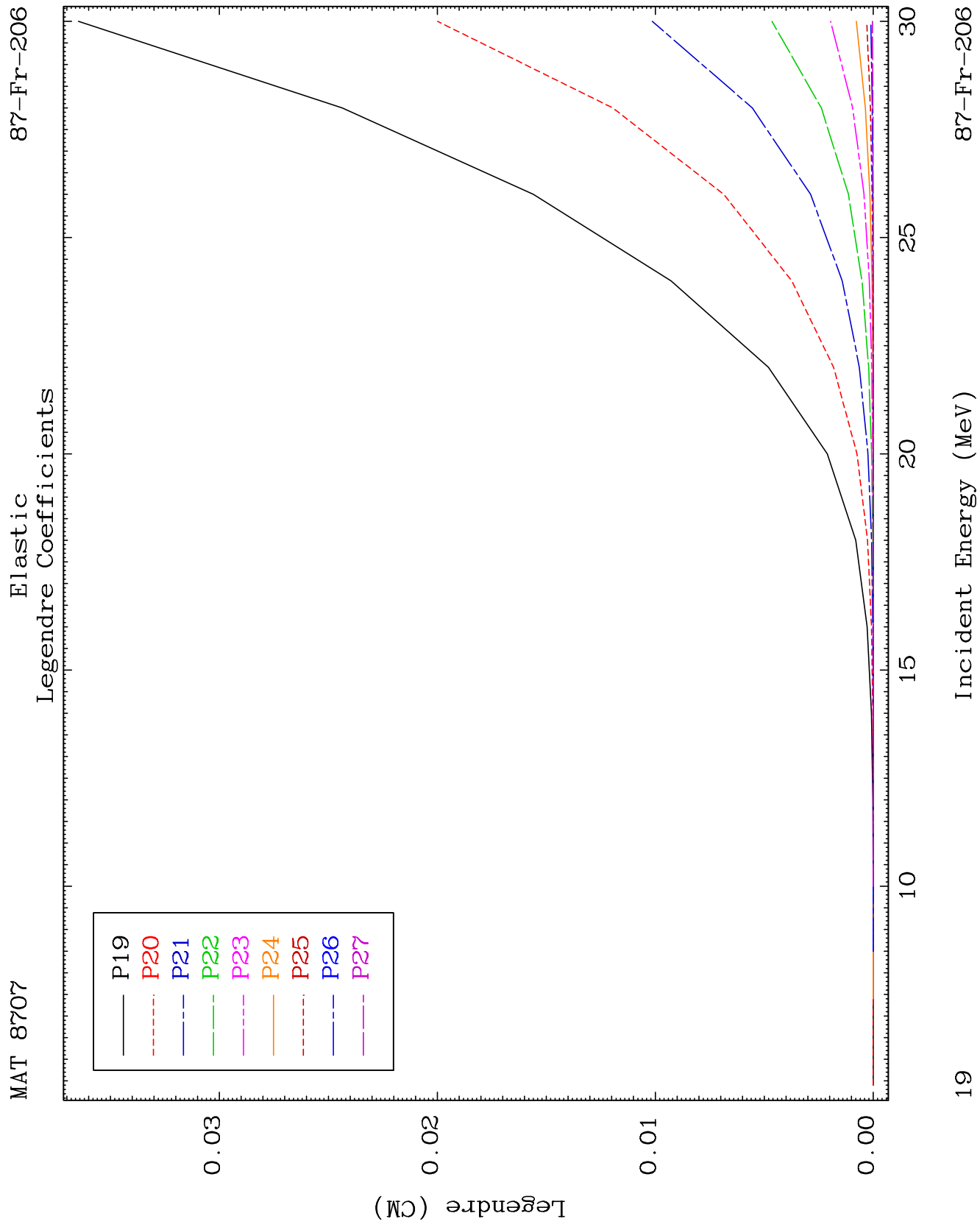


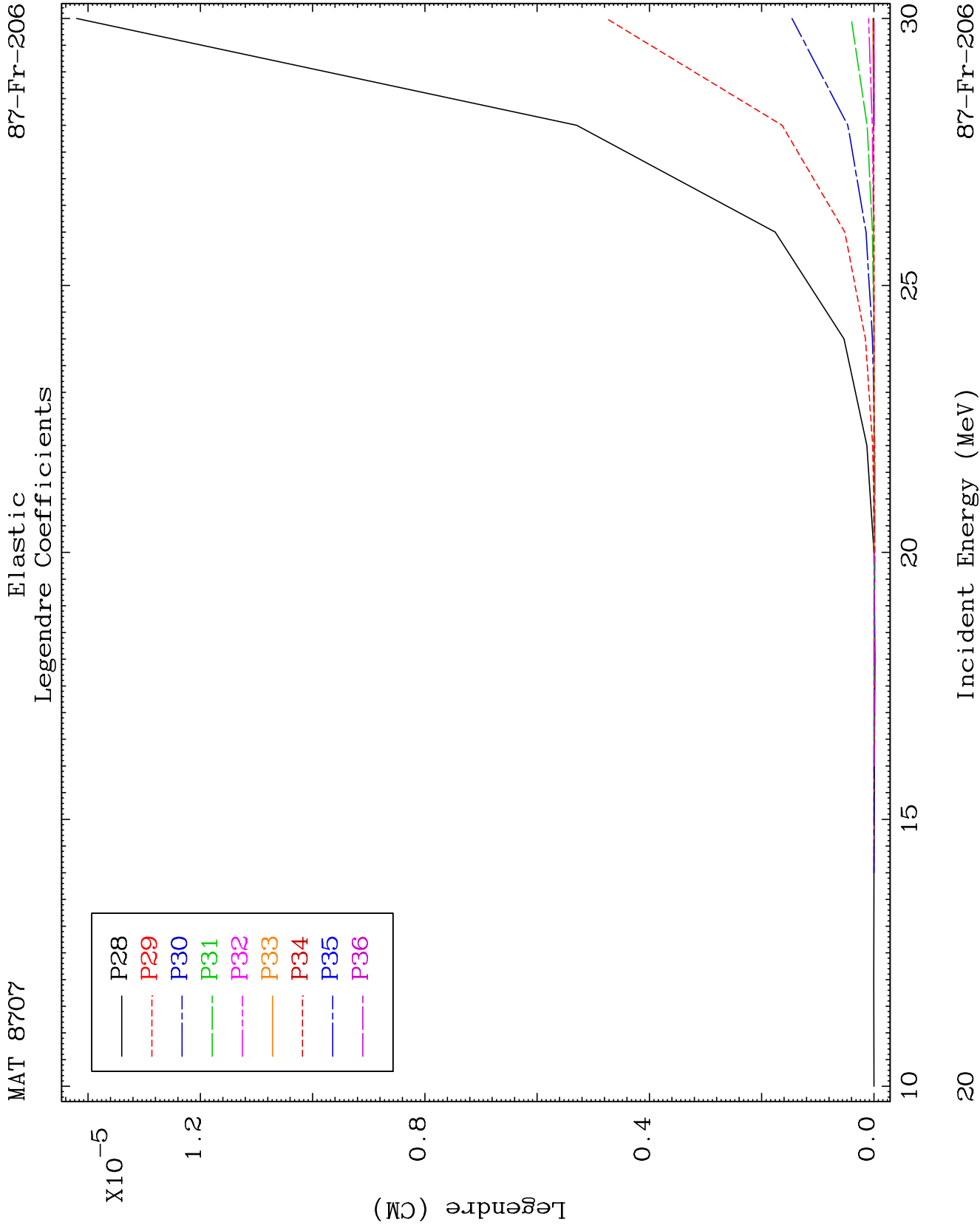
21

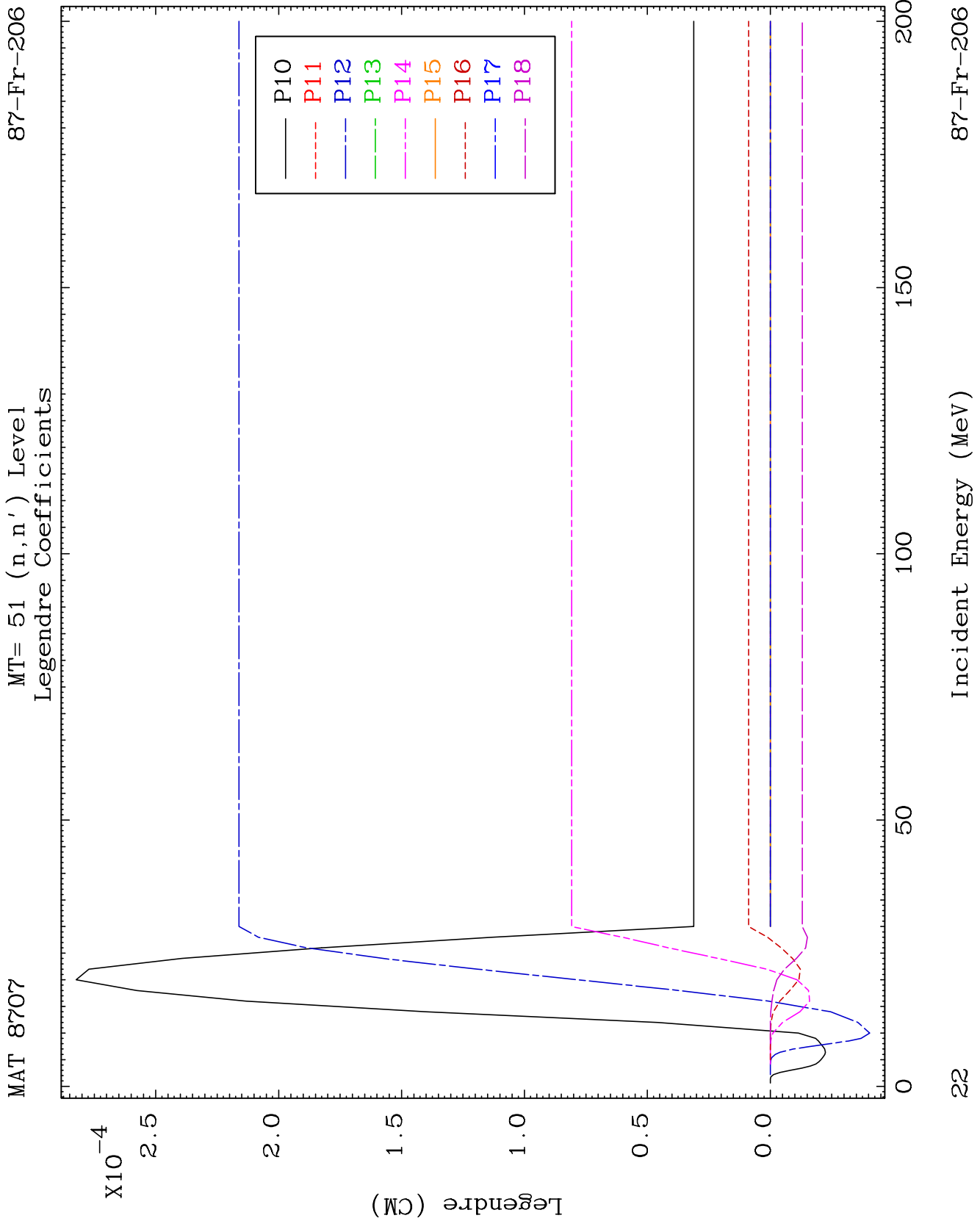
Incident Energy (MeV)

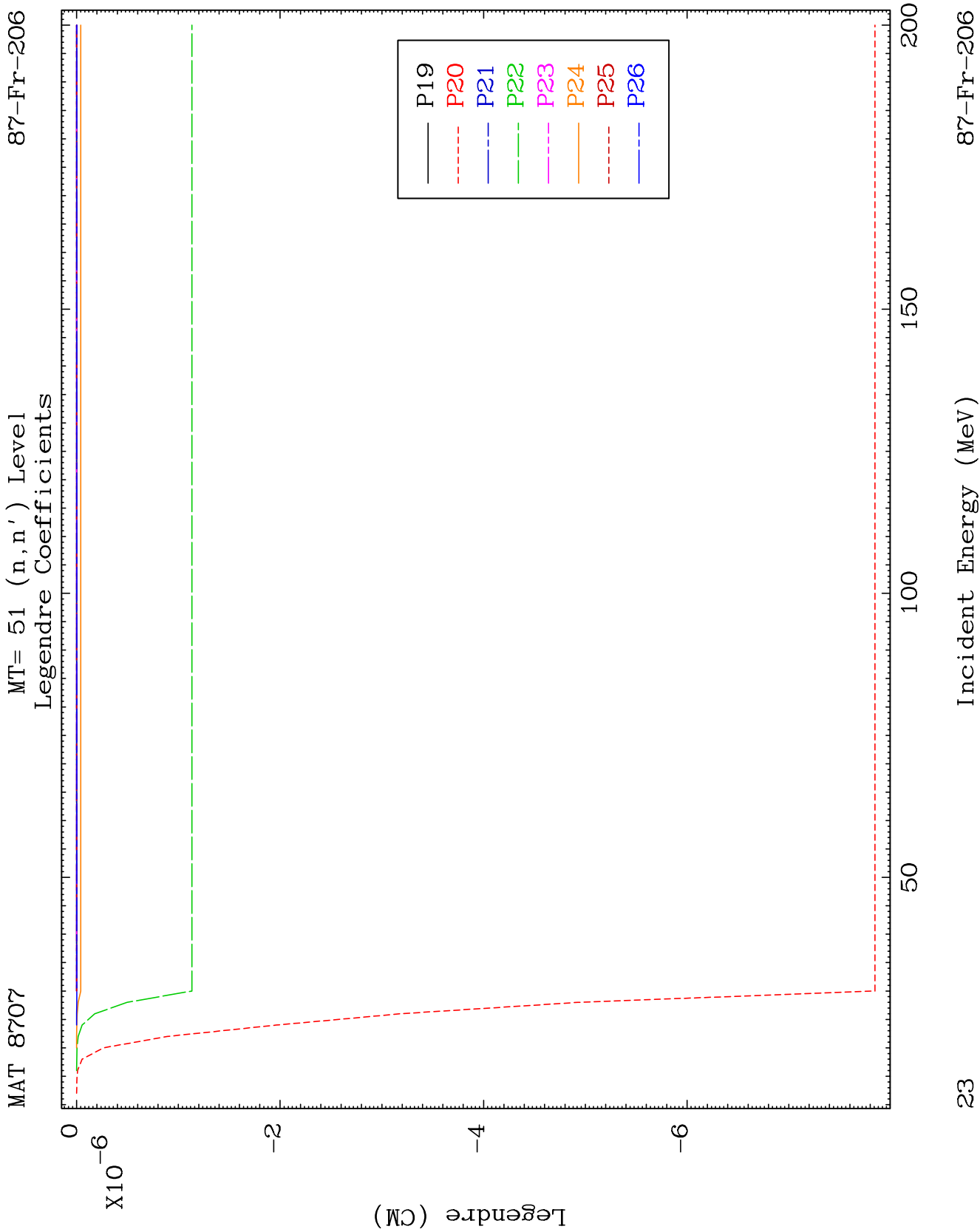
87-FR-206







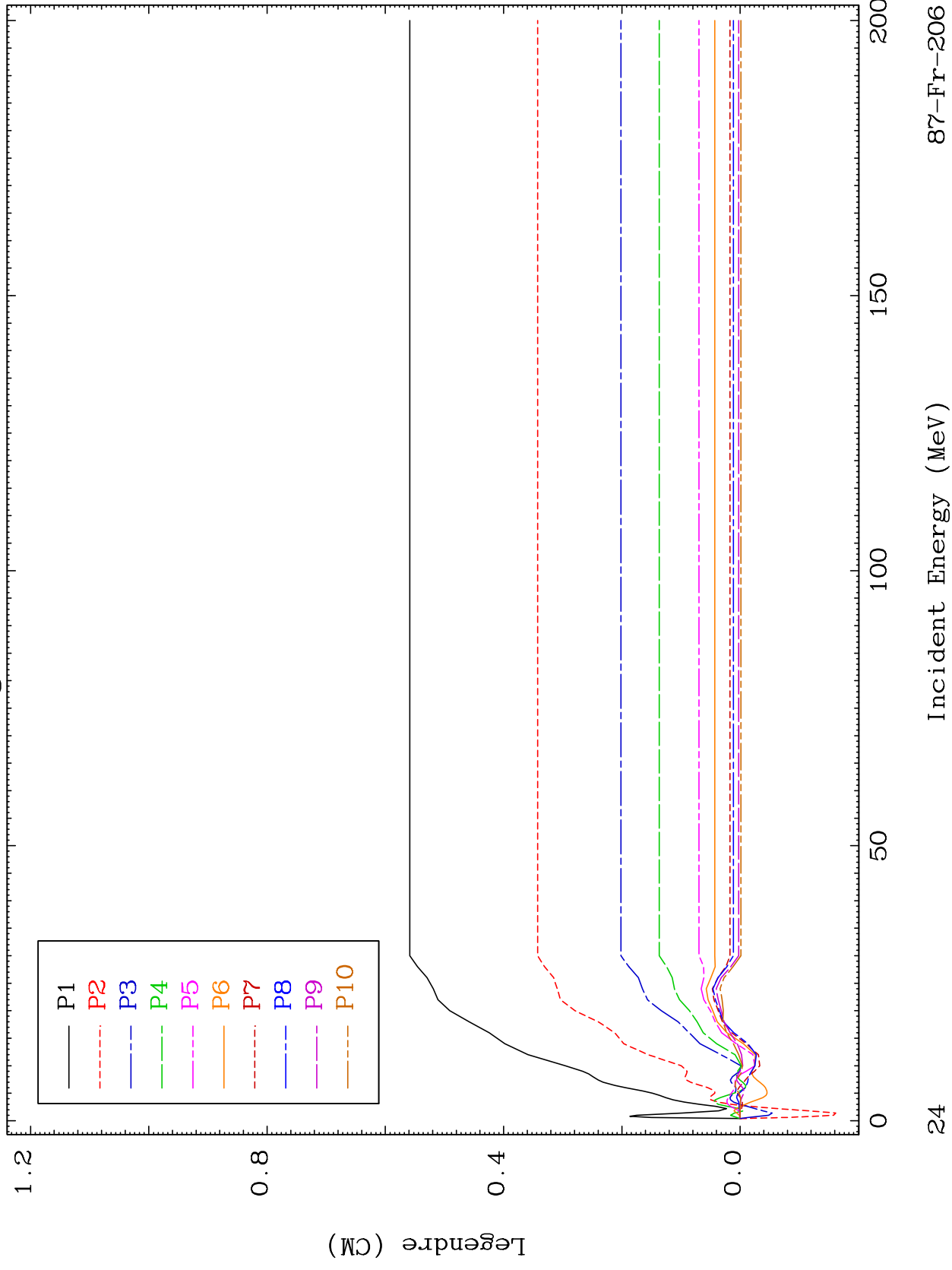


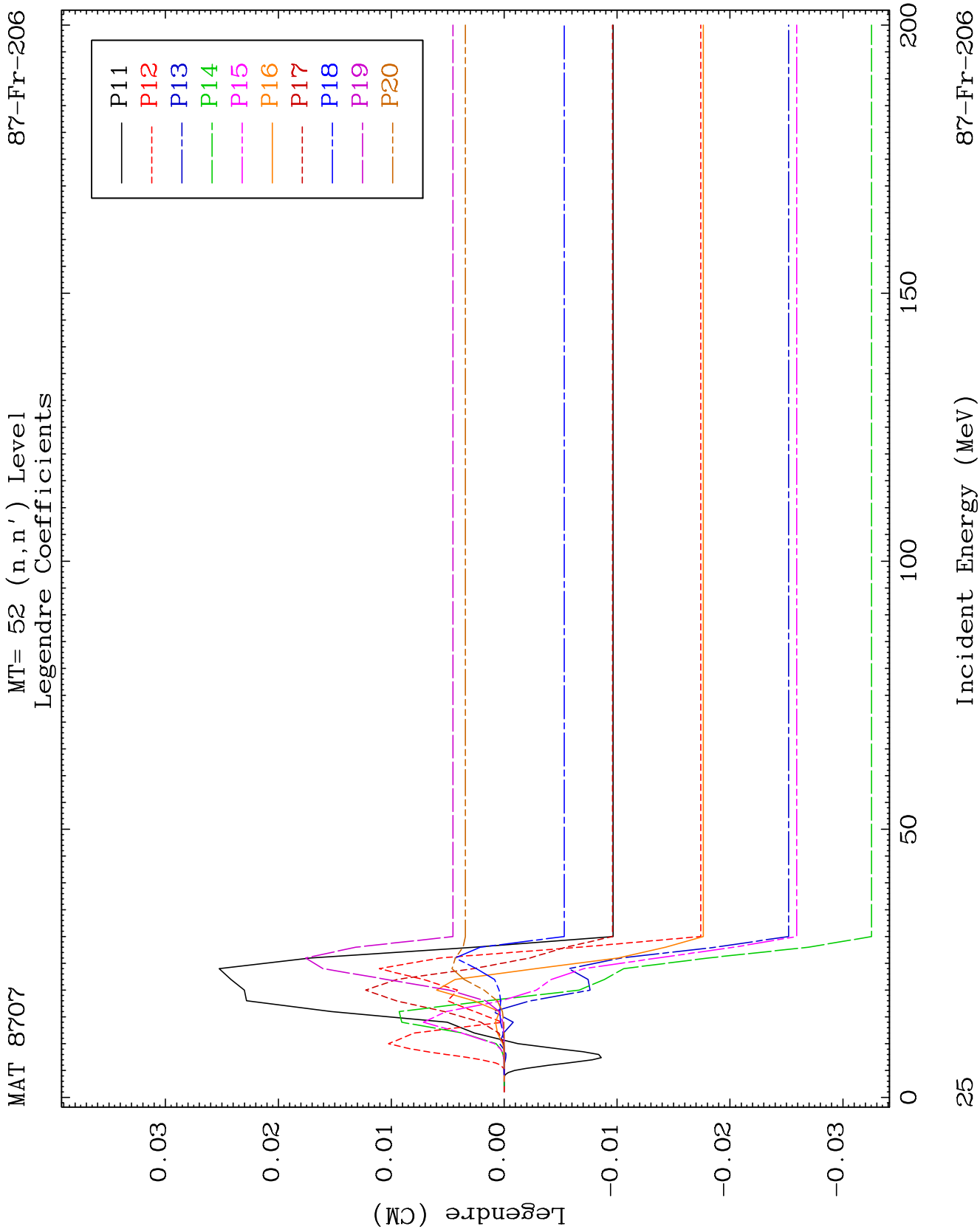


MAT 8707

MT= 52 (n, n') Level
Legendre Coefficients

87-Fr-206

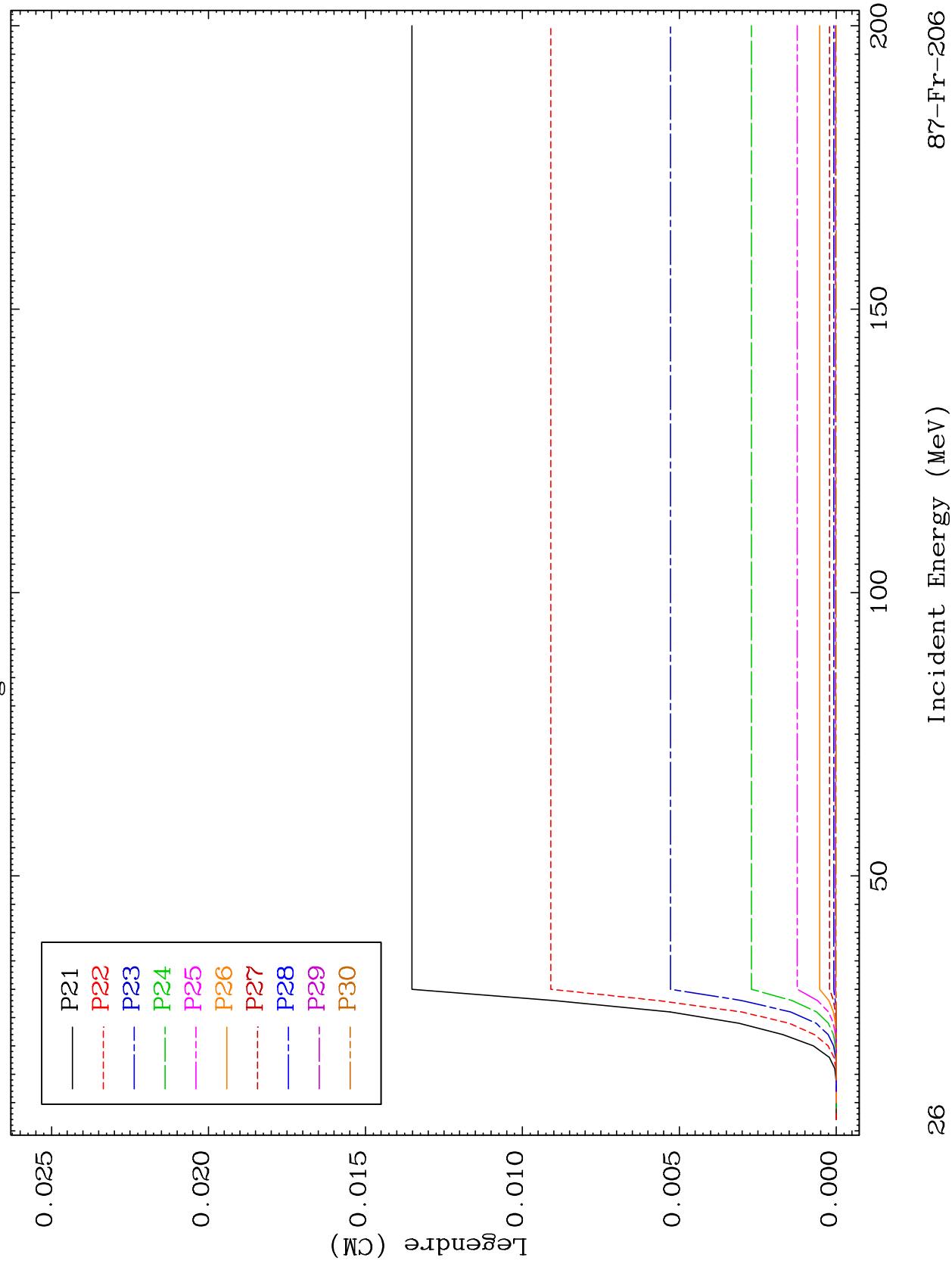




MAT 8707

MT= 52 (n, n') Level
Legendre Coefficients

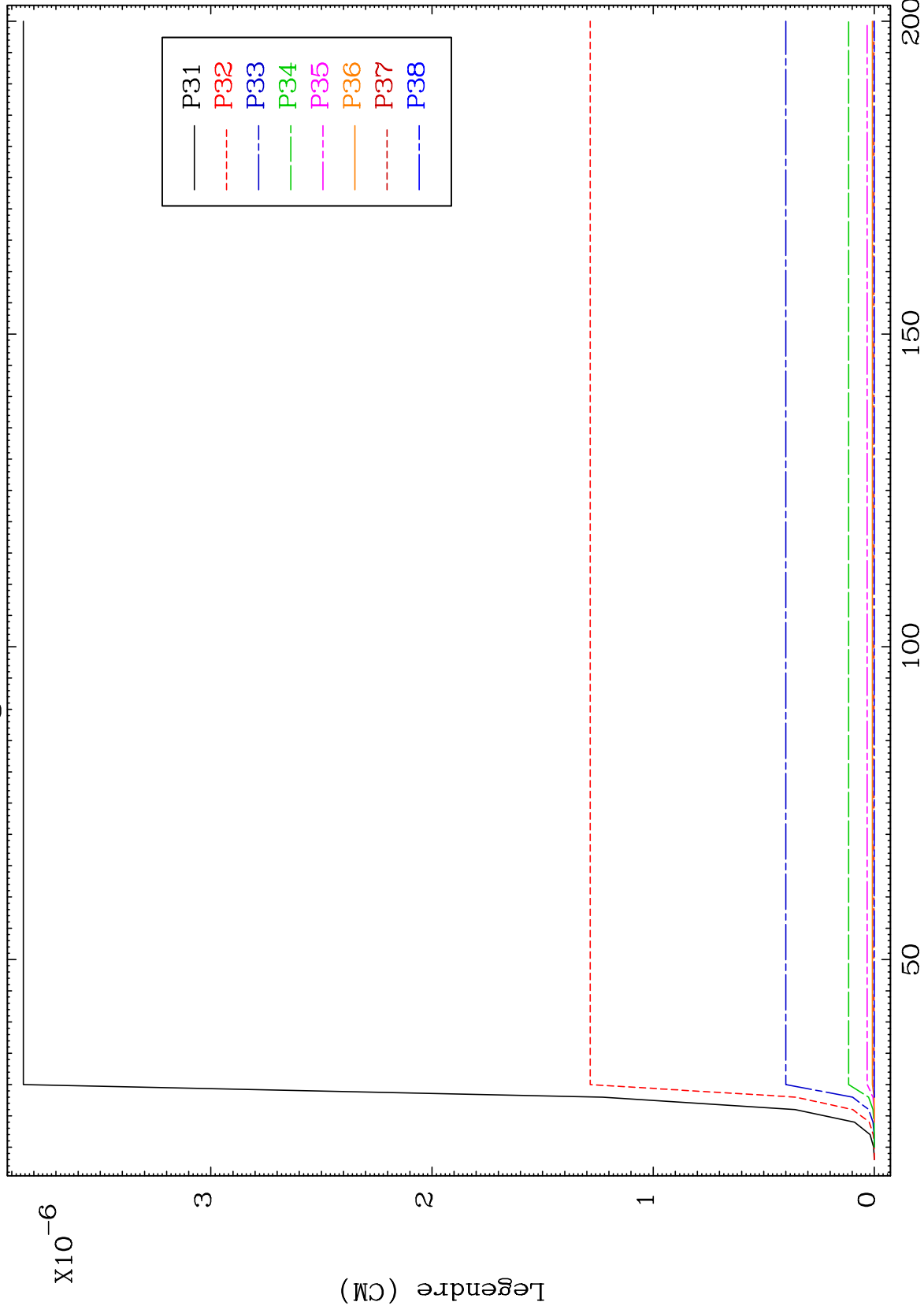
87-Fr-206



MAT 8707

MT= 52 (n,n') Level
Legendre Coefficients

87-Fr-206



27

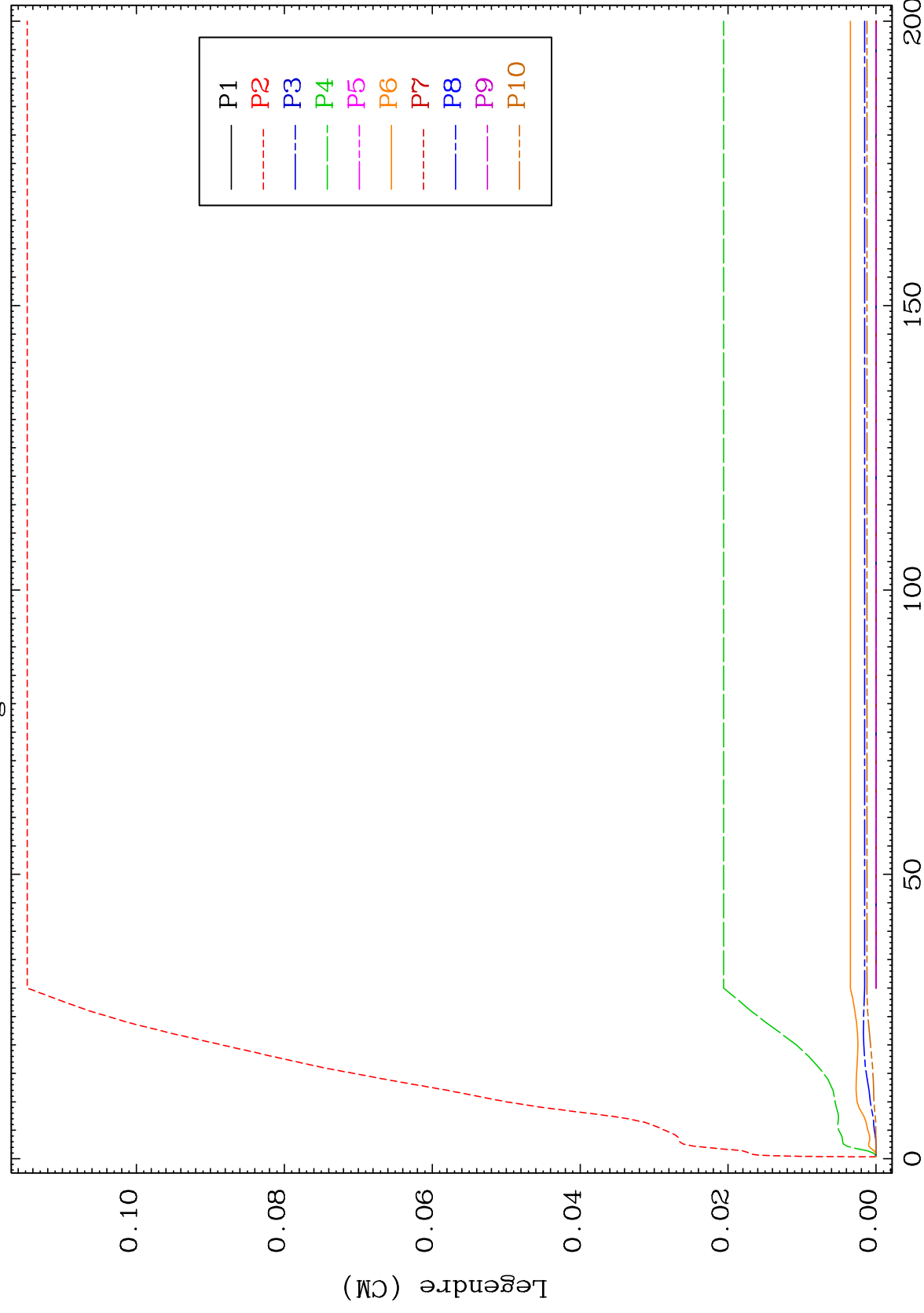
Incident Energy (MeV)

87-Fr-206

MAT 8707

MT= 53 (n, n') Level
Legendre Coefficients

87-Fr-206



87-Fr-206

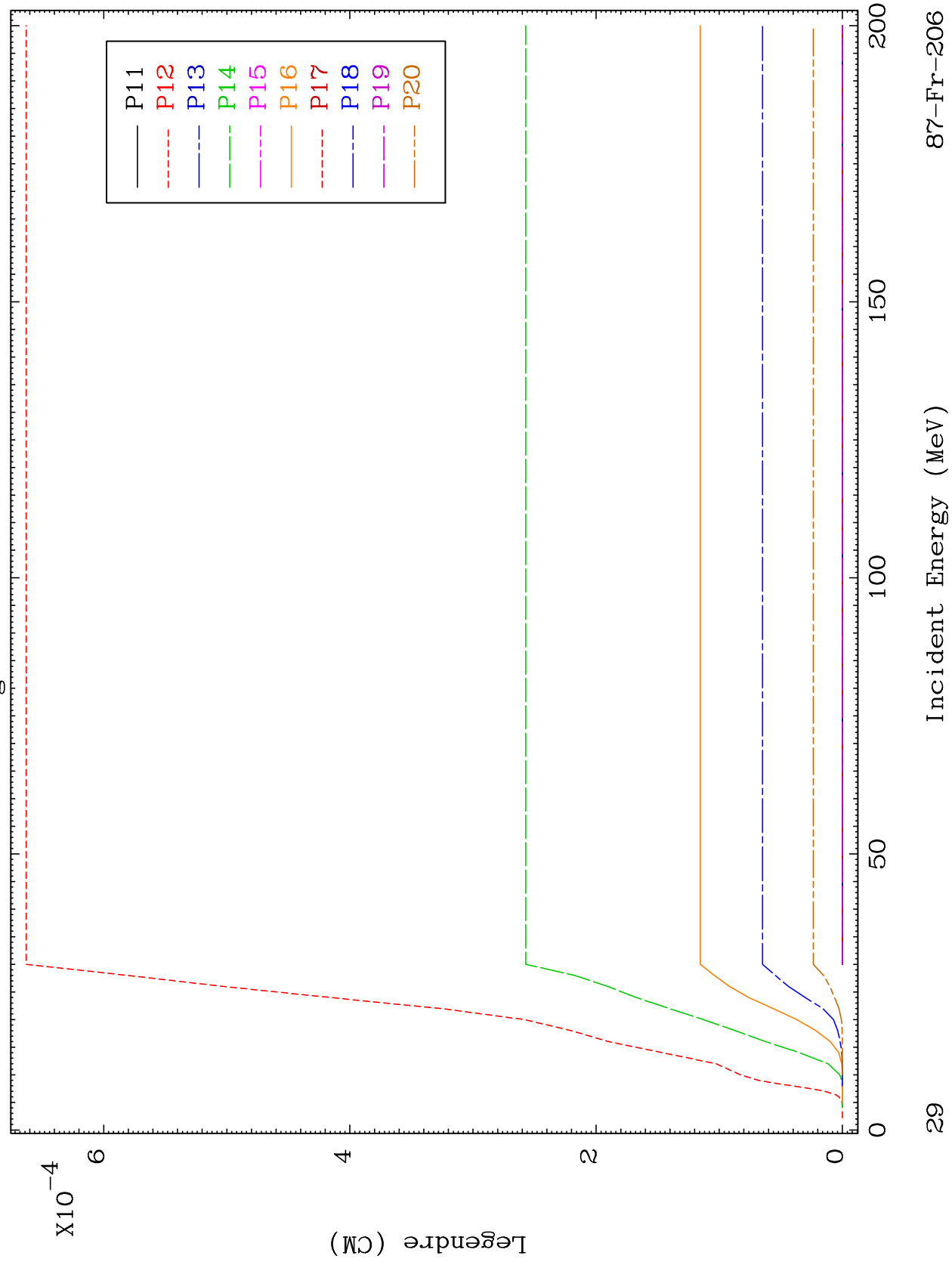
Incident Energy (MeV)

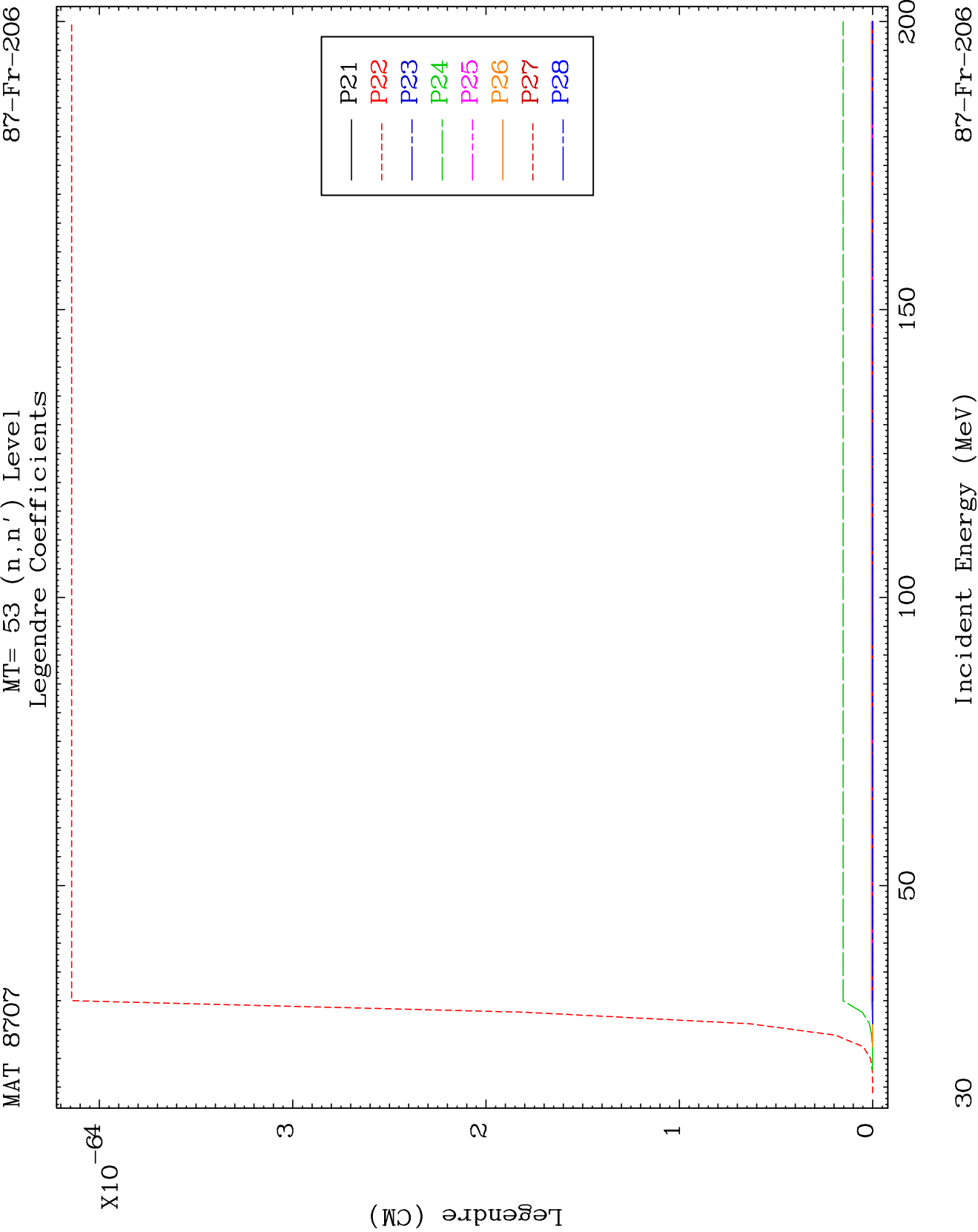
82

MAT 8707

MT= 53 (n, n') Level
Legendre Coefficients

87-Fr-206

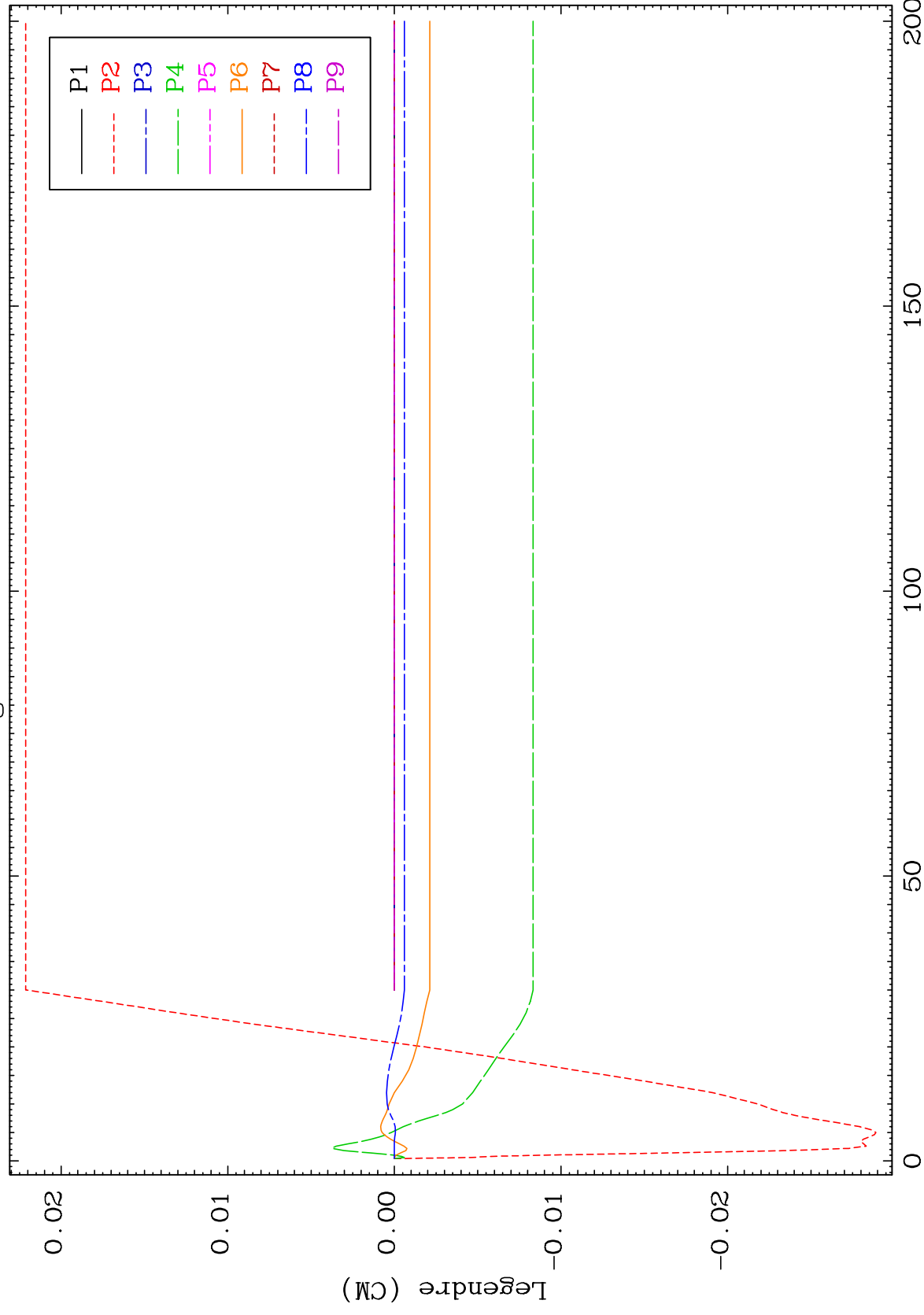




MAT 8707

MT= 54 (n, n') Level
Legendre Coefficients

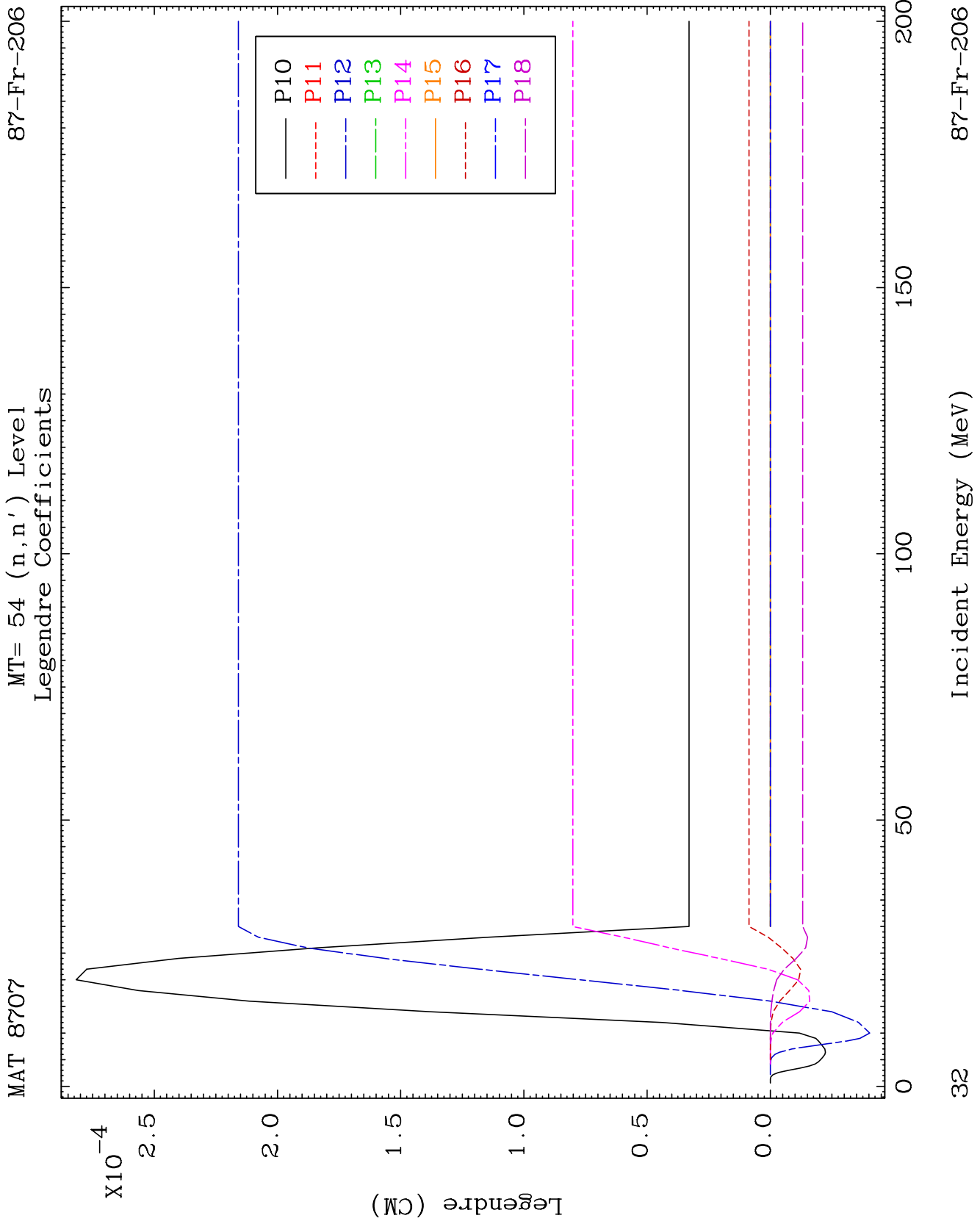
87-Fr-206



31

Incident Energy (MeV)

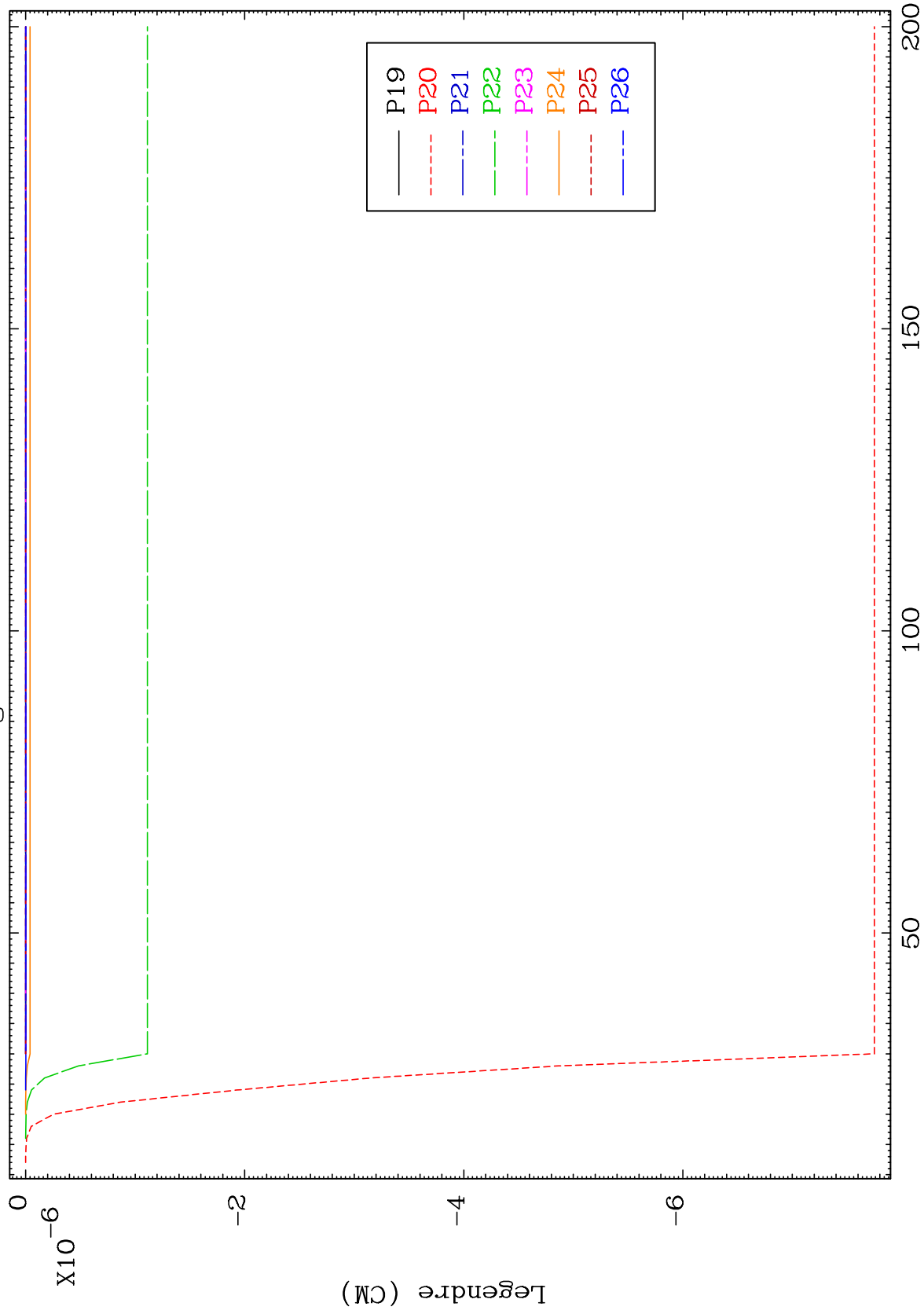
87-FR-206



MAT 8707

MT= 54 (n,n') Level
Legendre Coefficients

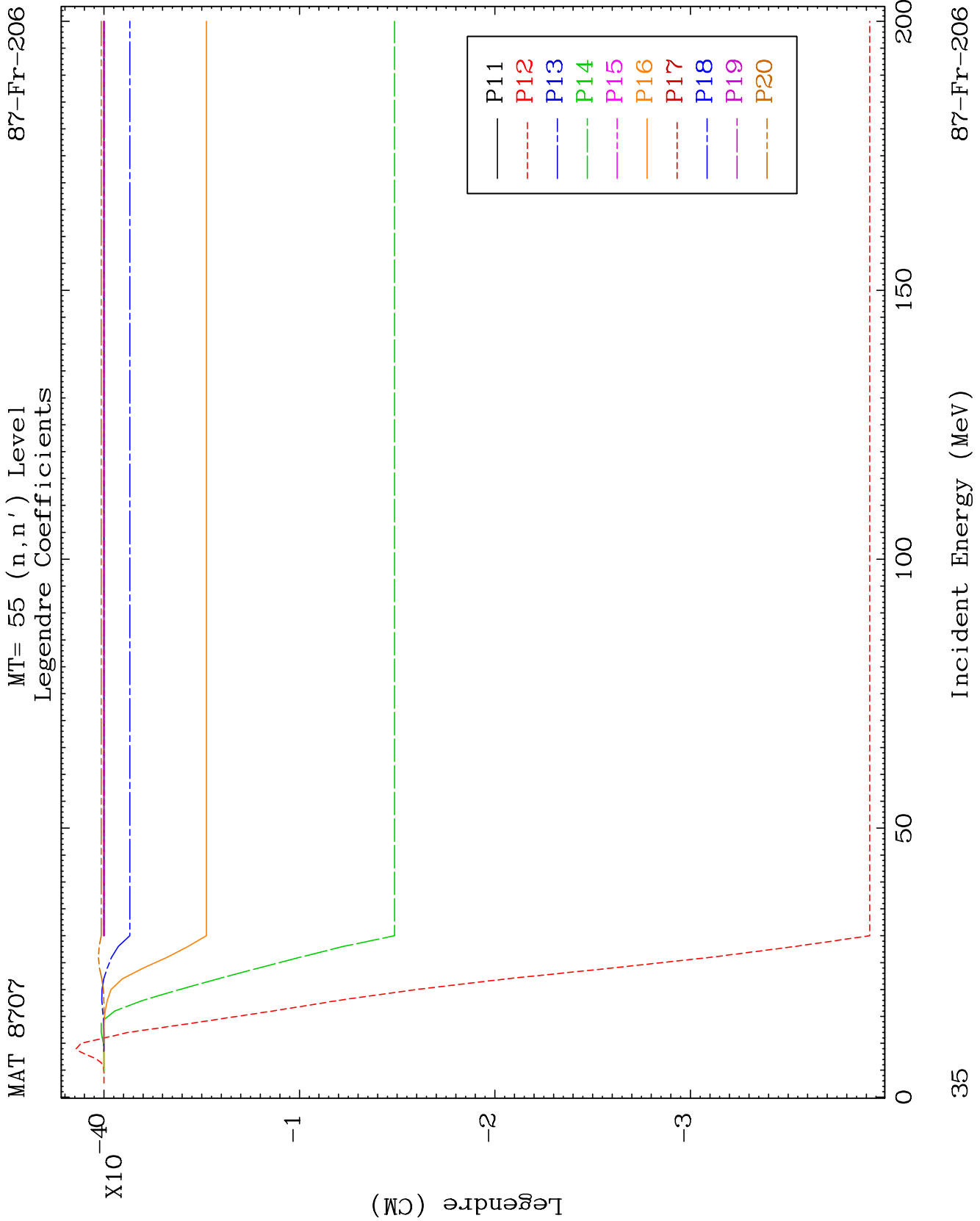
87-Fr-206

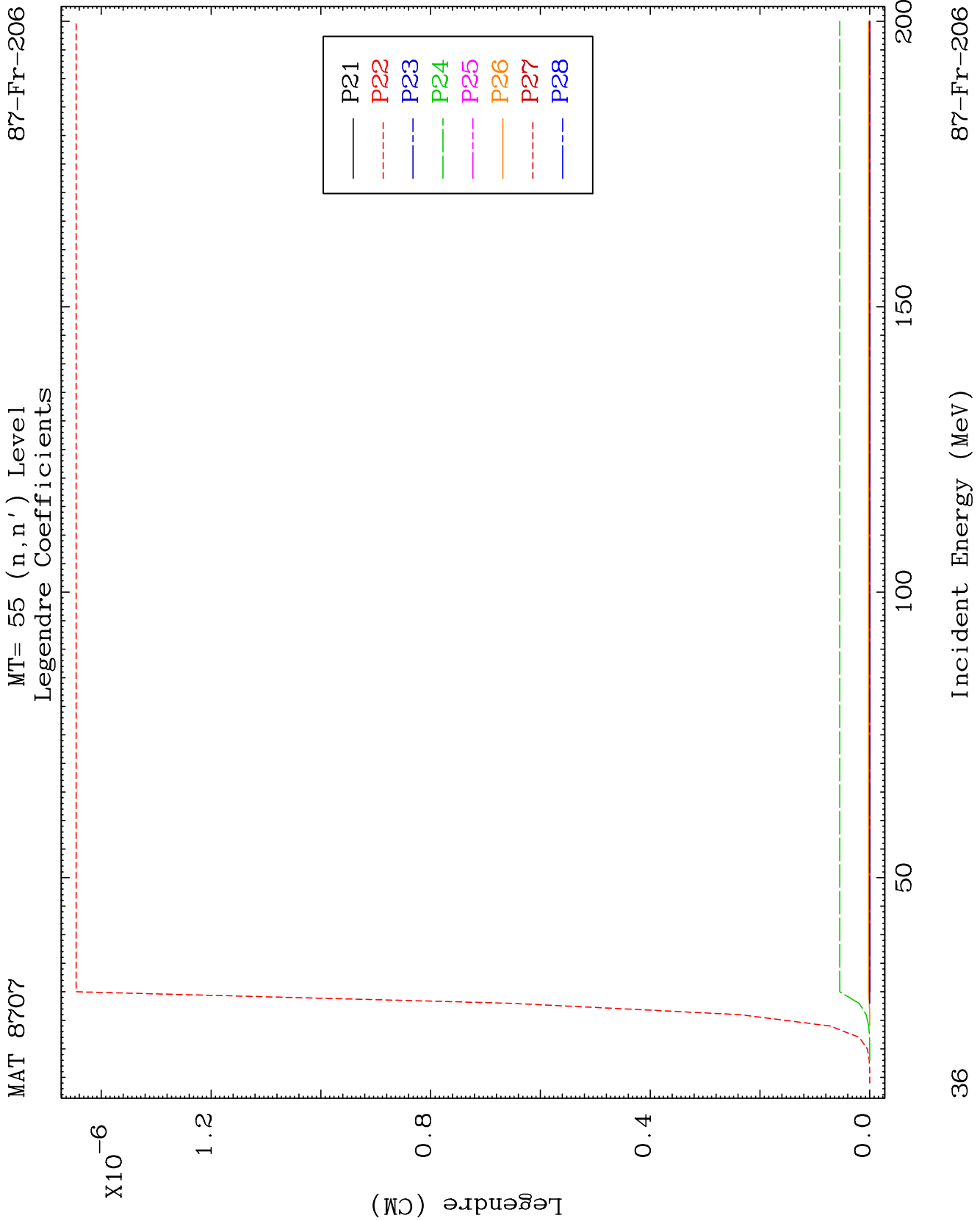


33

Incident Energy (MeV)

87-Fr-206

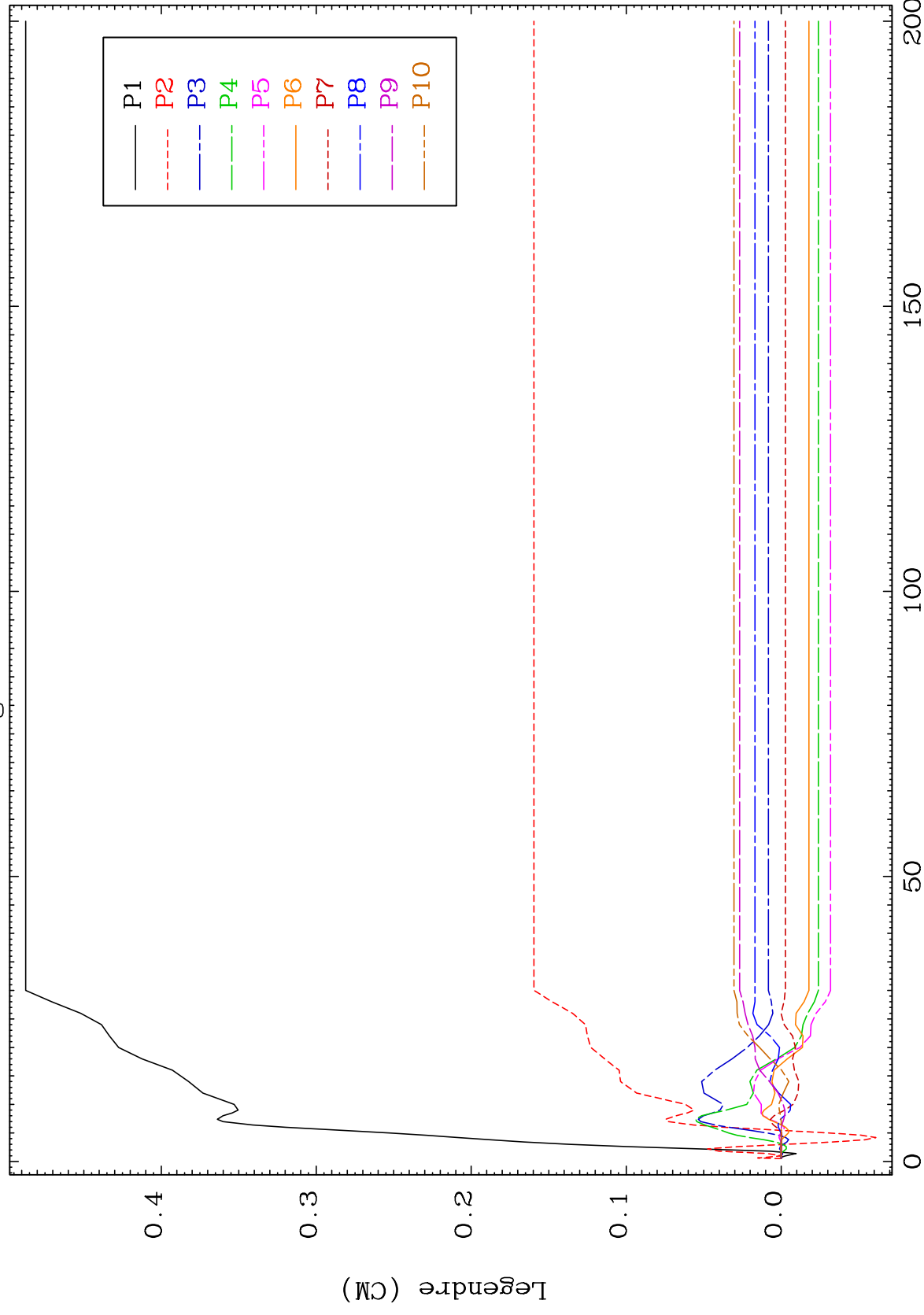




MAT 8707

87-Fr-206

MT= 56 (n, n') Level
Legendre Coefficients



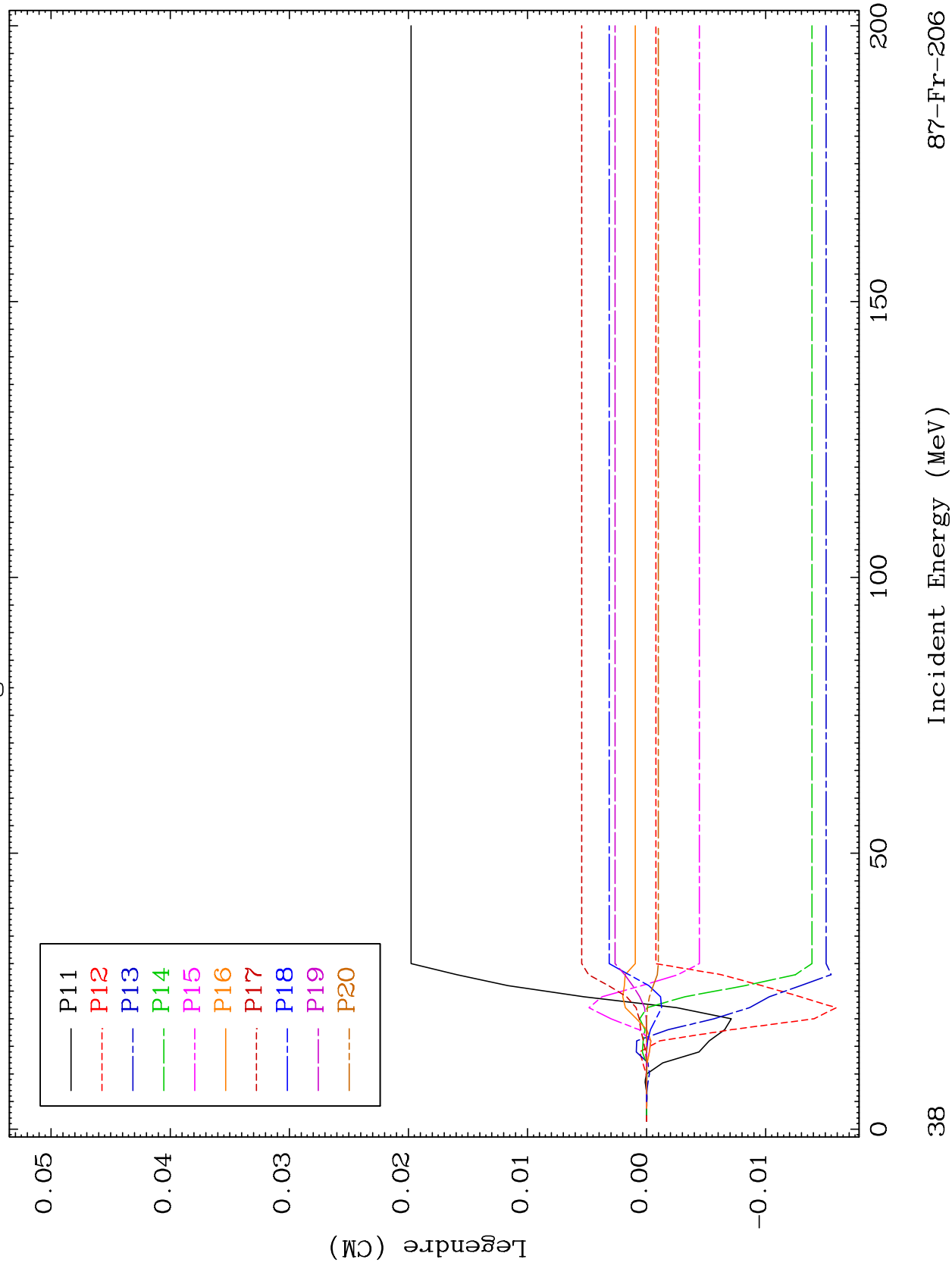
87-Fr-206

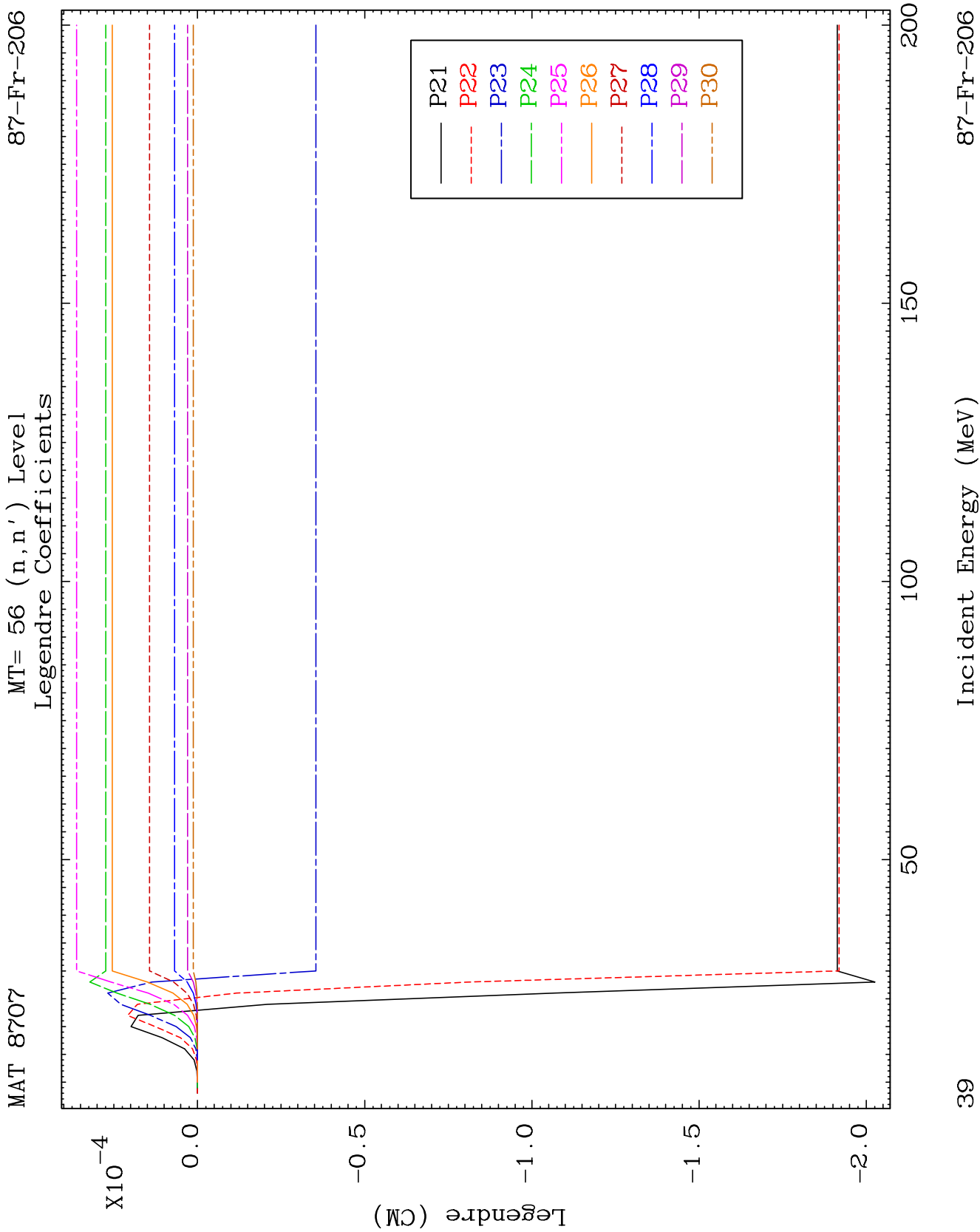
Incident Energy (MeV)

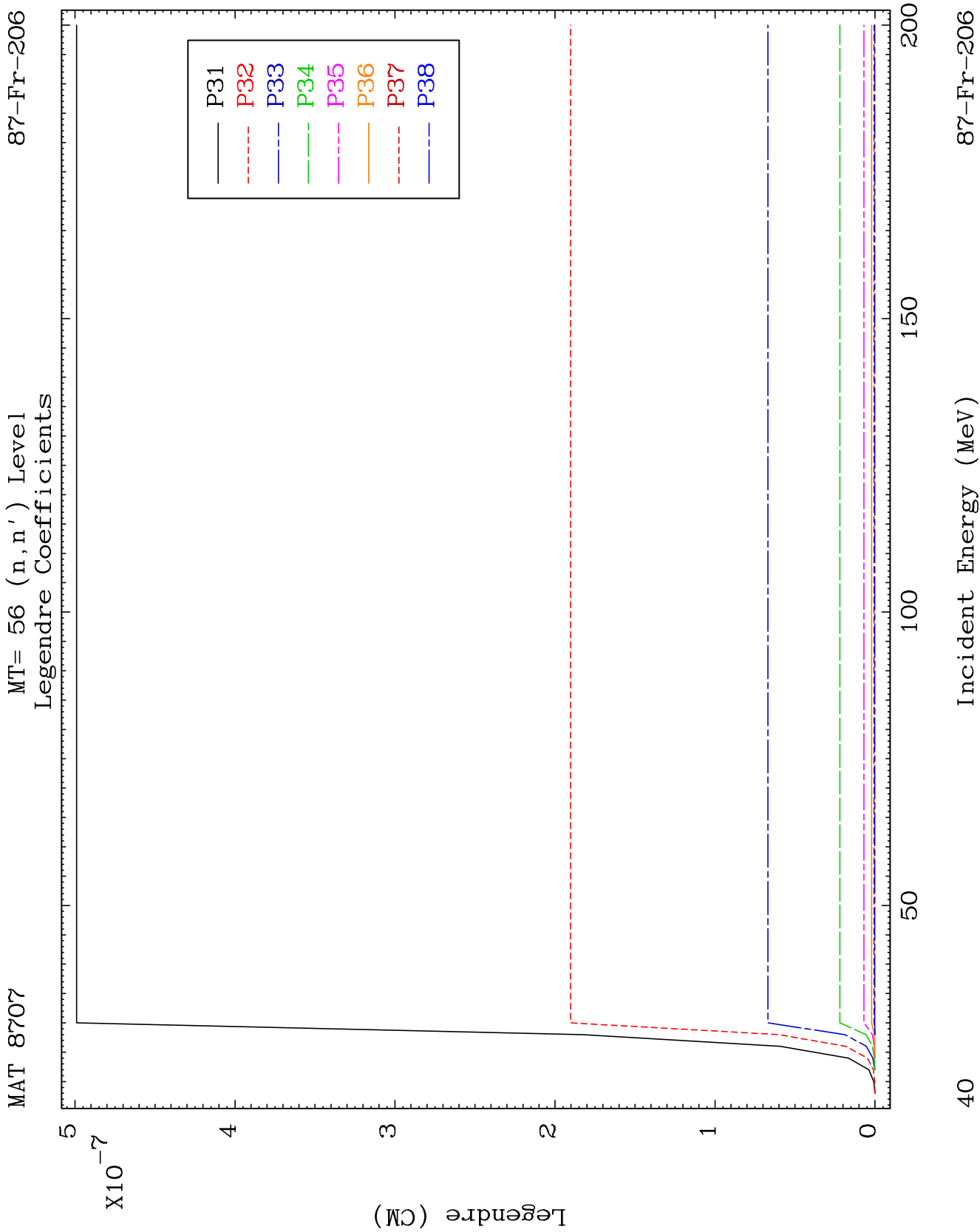
MAT 8707

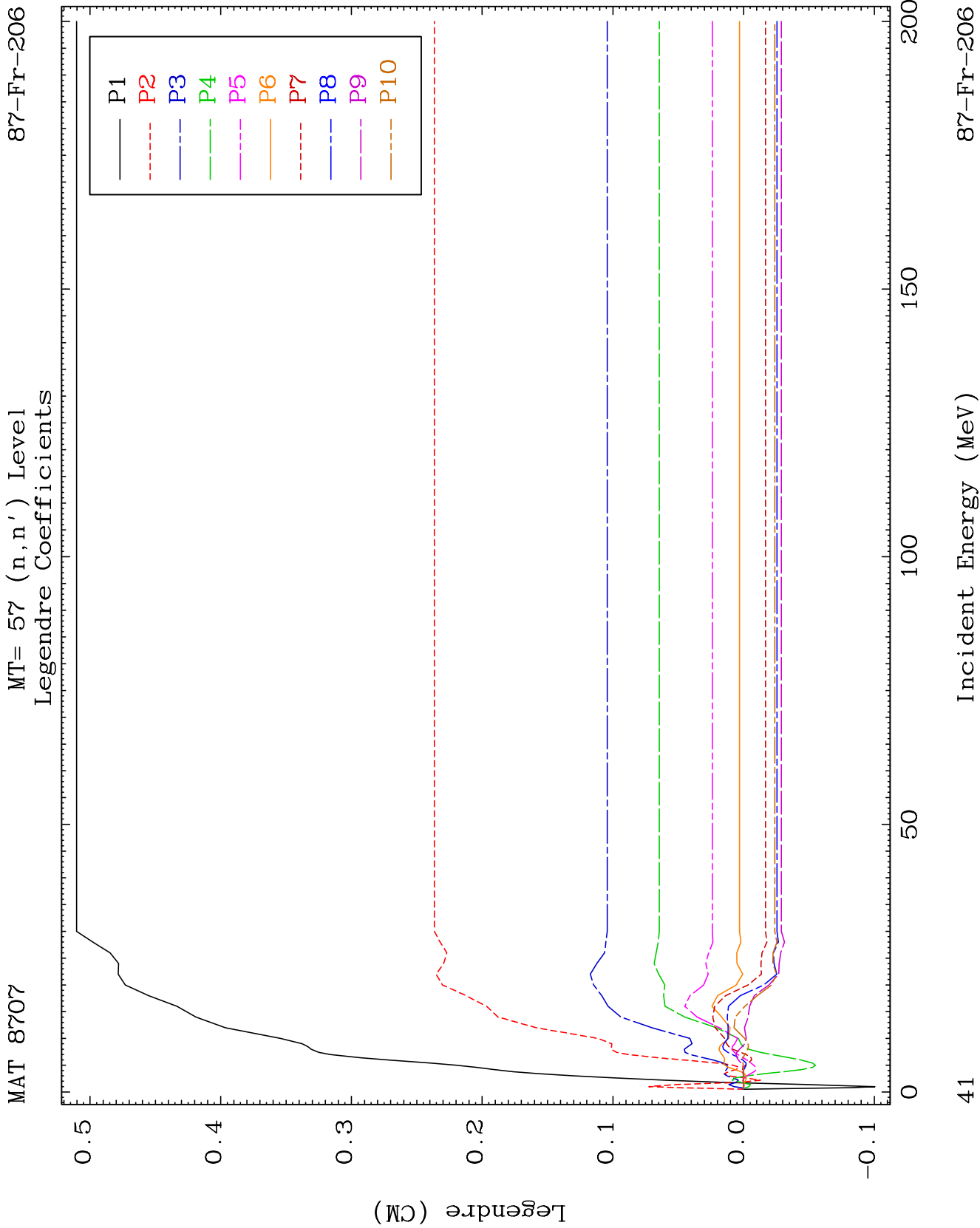
MT= 56 (n, n') Level
Legendre Coefficients

87-Fr-206





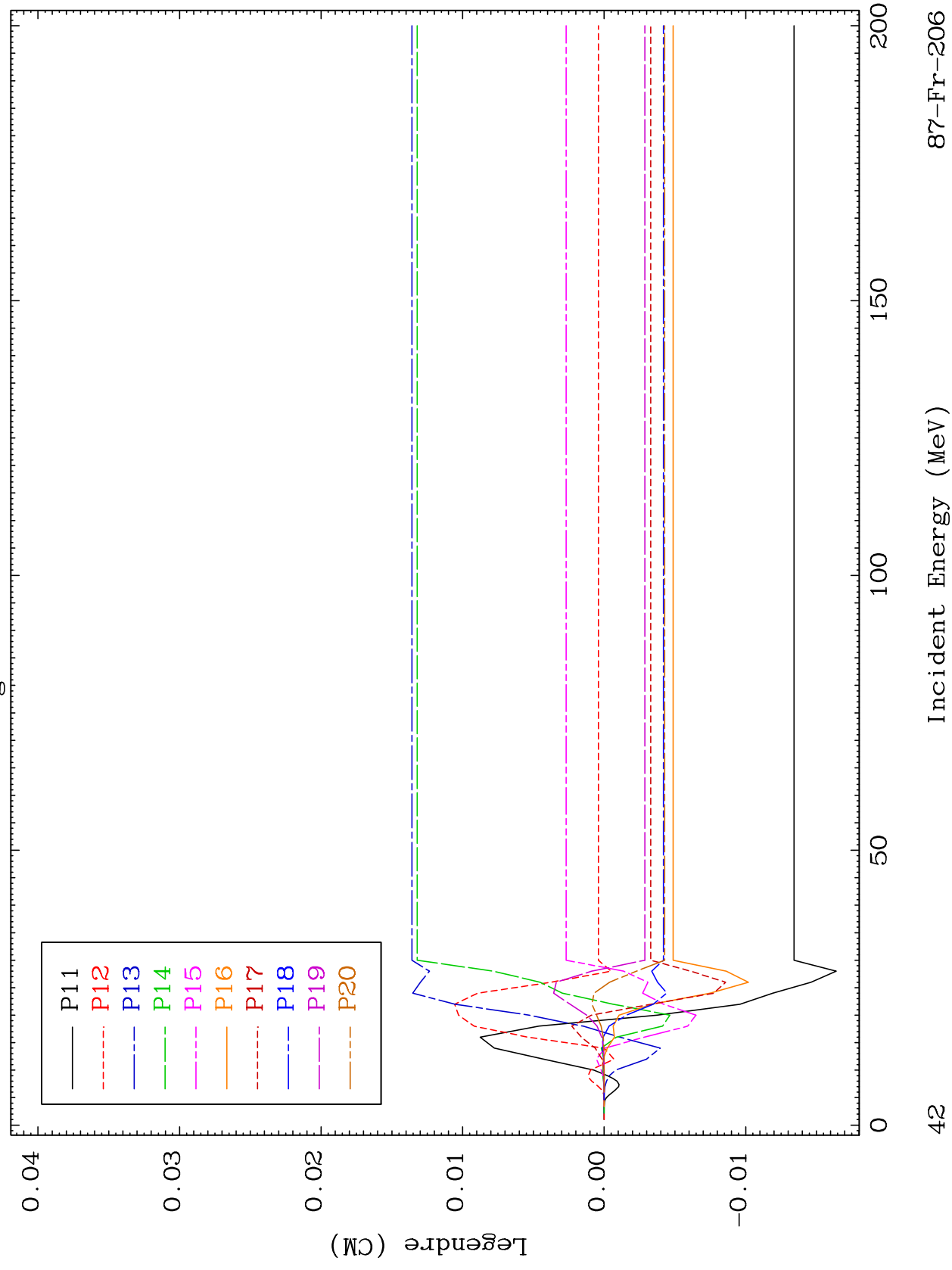




MAT 8707

MT= 57 (n, n') Level
Legendre Coefficients

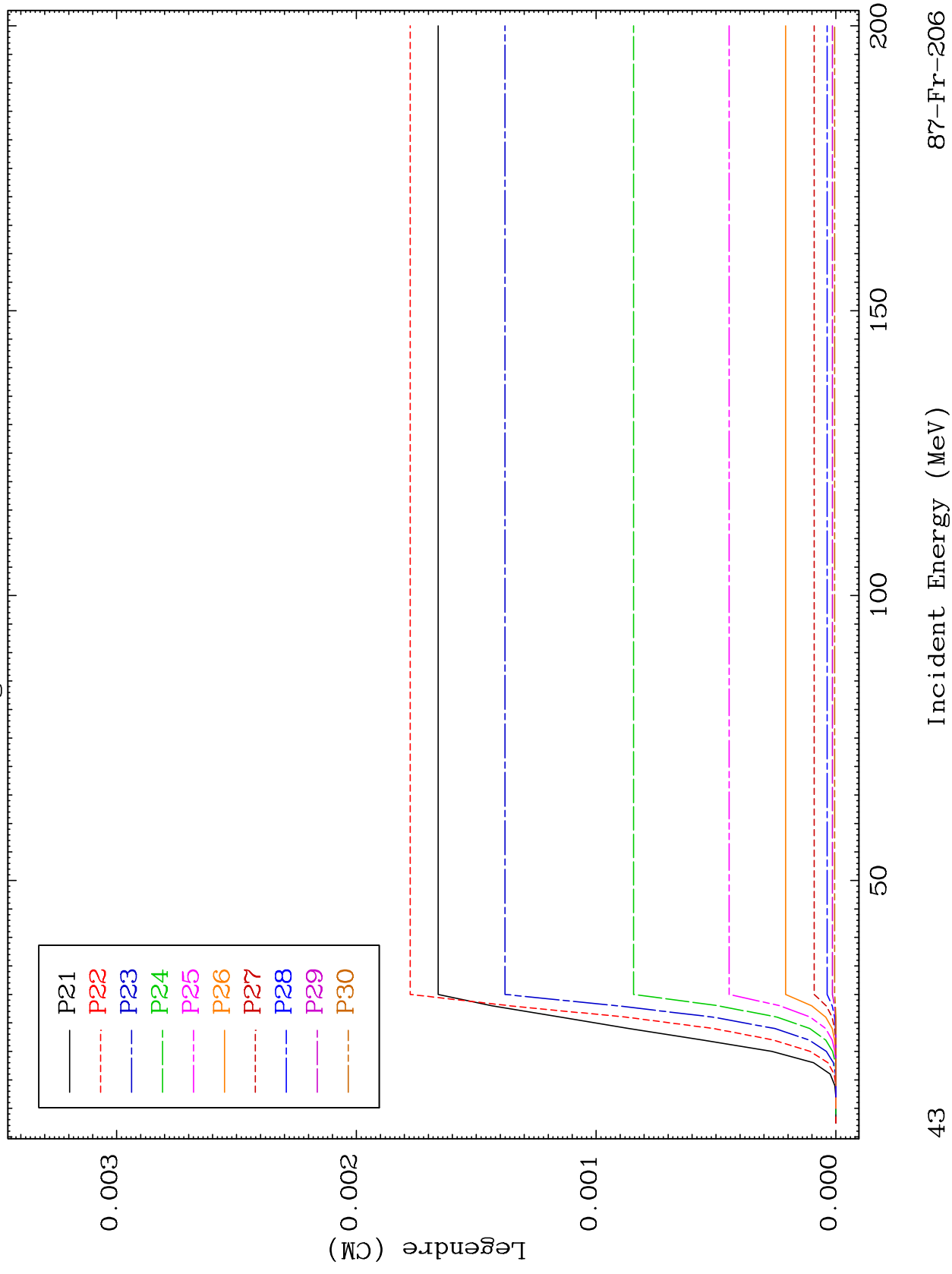
87-Fr-206



MAT 8707

MT= 57 (n, n') Level
Legendre Coefficients

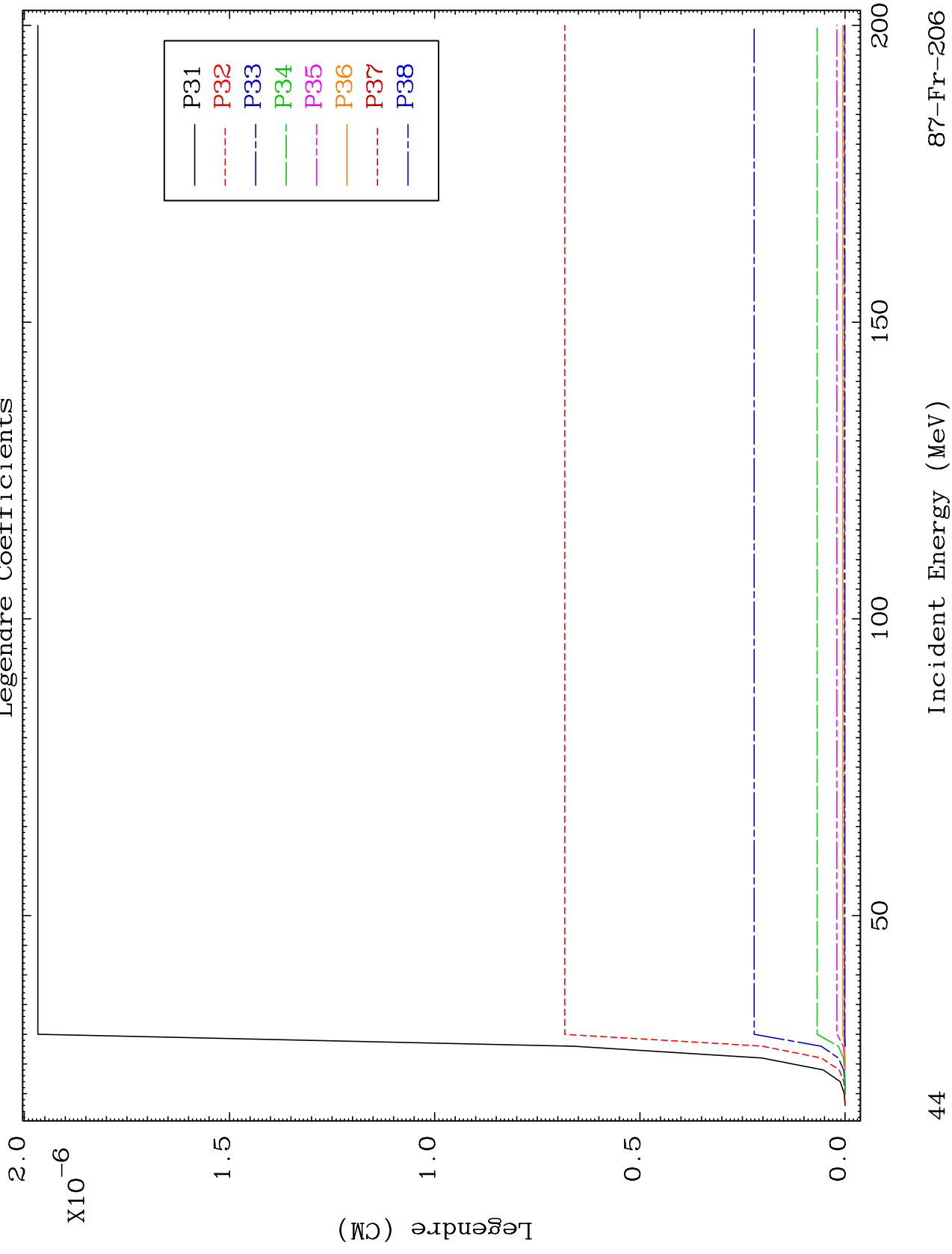
87-Fr-206



MAT 8707

MT= 57 (n, n') Level
Legendre Coefficients

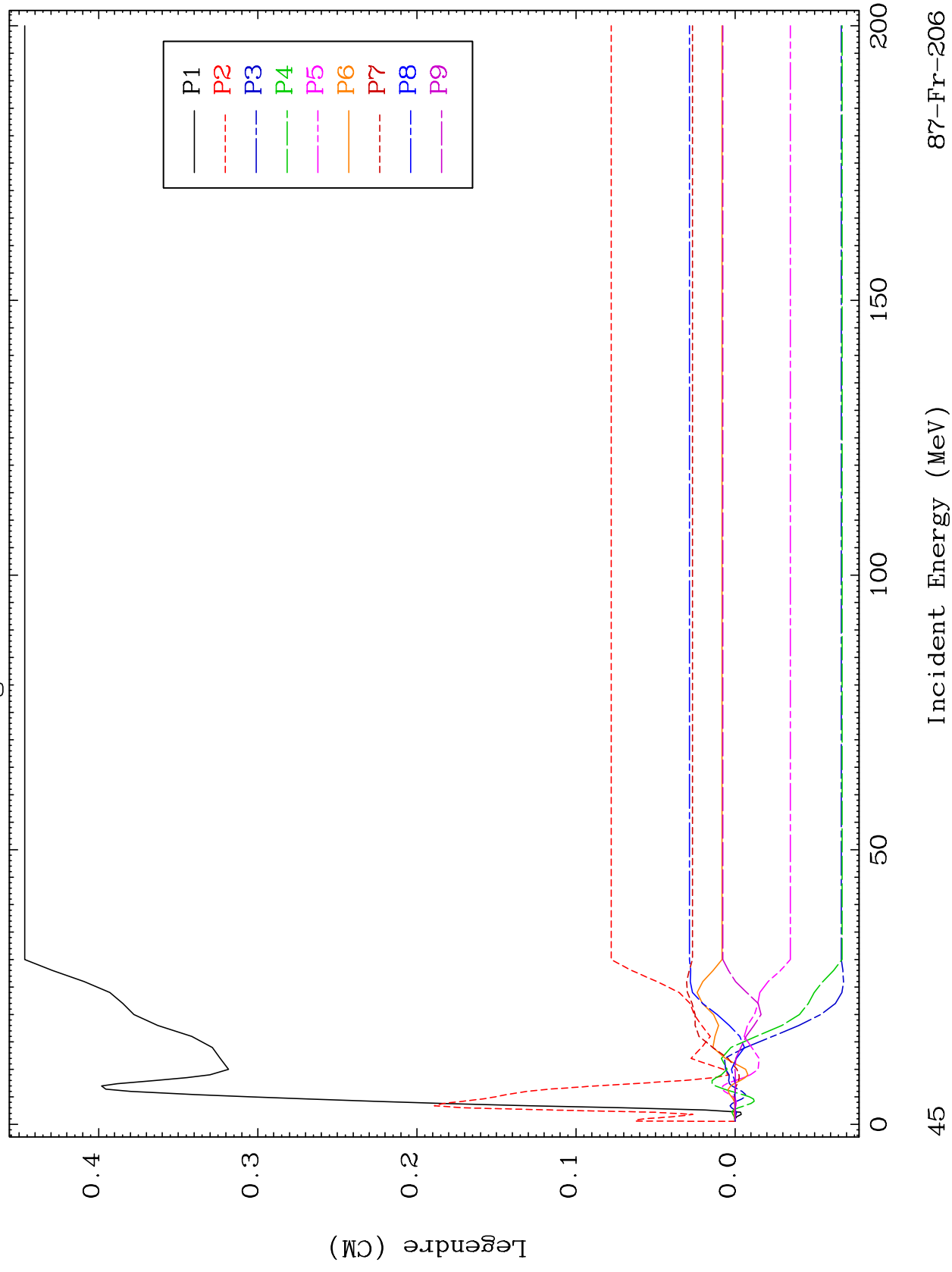
87-Fr-206



MAT 8707

MT= 58 (n, n') Level
Legendre Coefficients

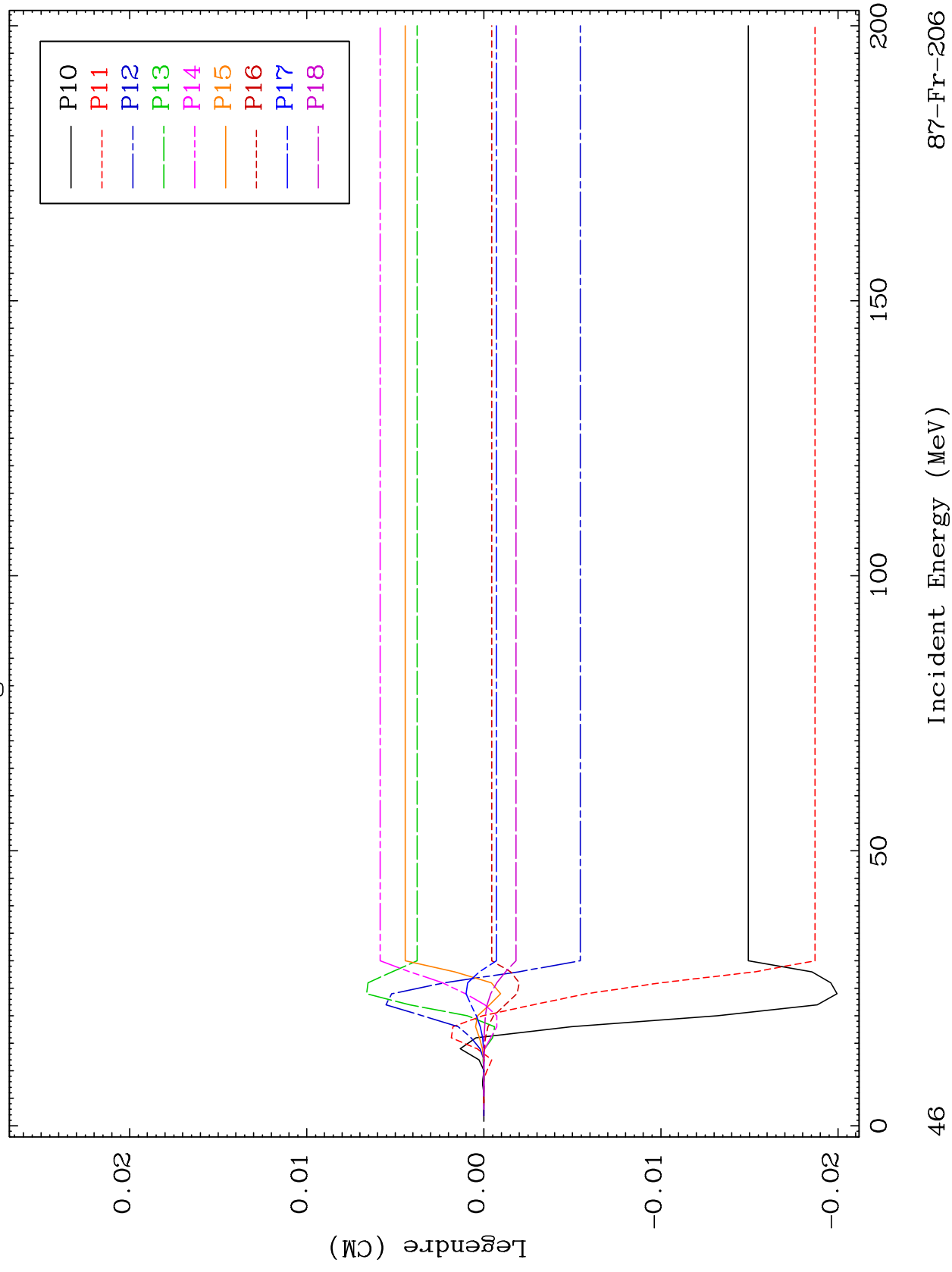
87-Fr-206



MAT 8707

MT= 58 (n, n') Level
Legendre Coefficients

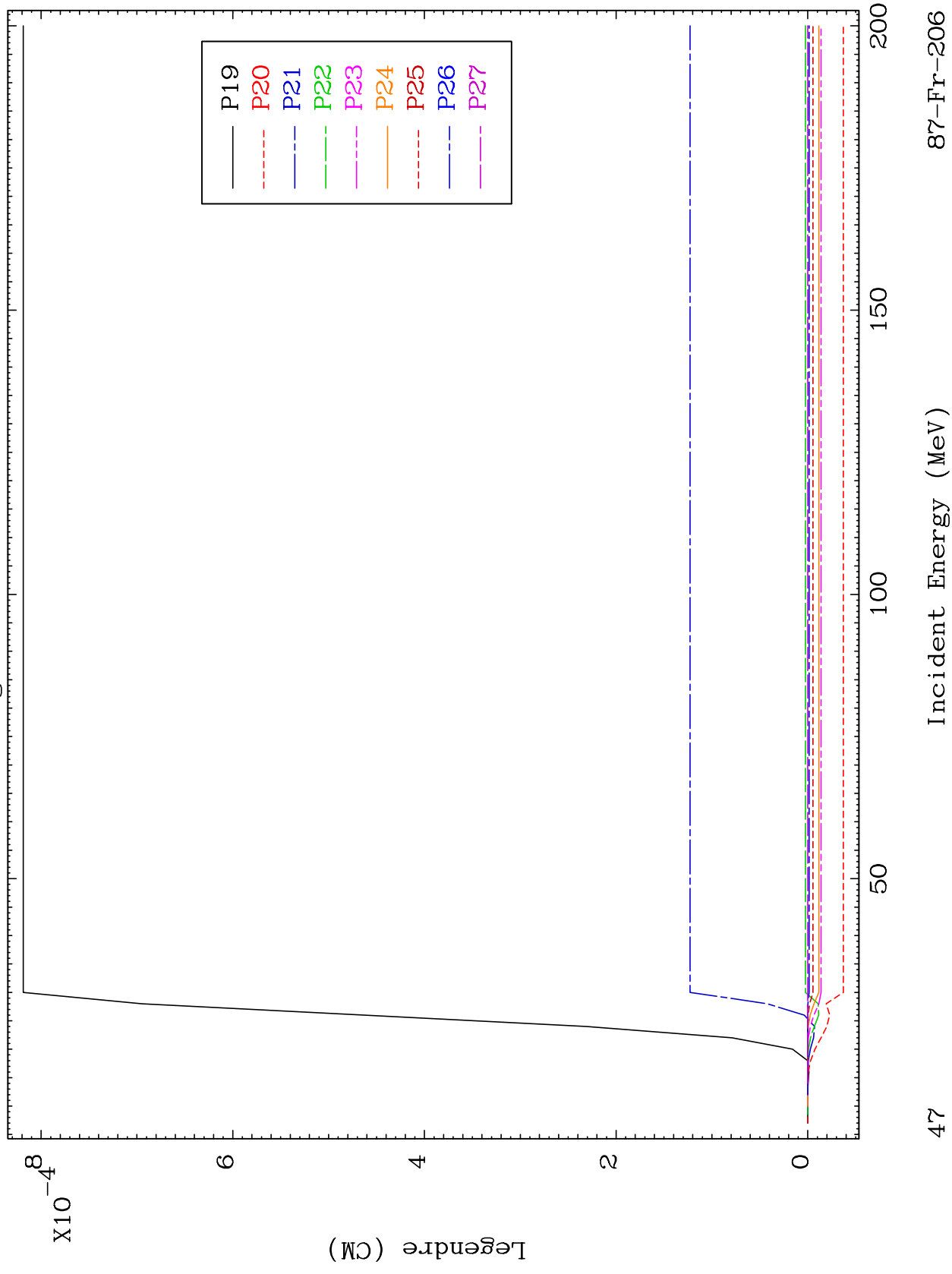
87-Fr-206



MAT 8707

MT= 58 (n, n') Level
Legendre Coefficients

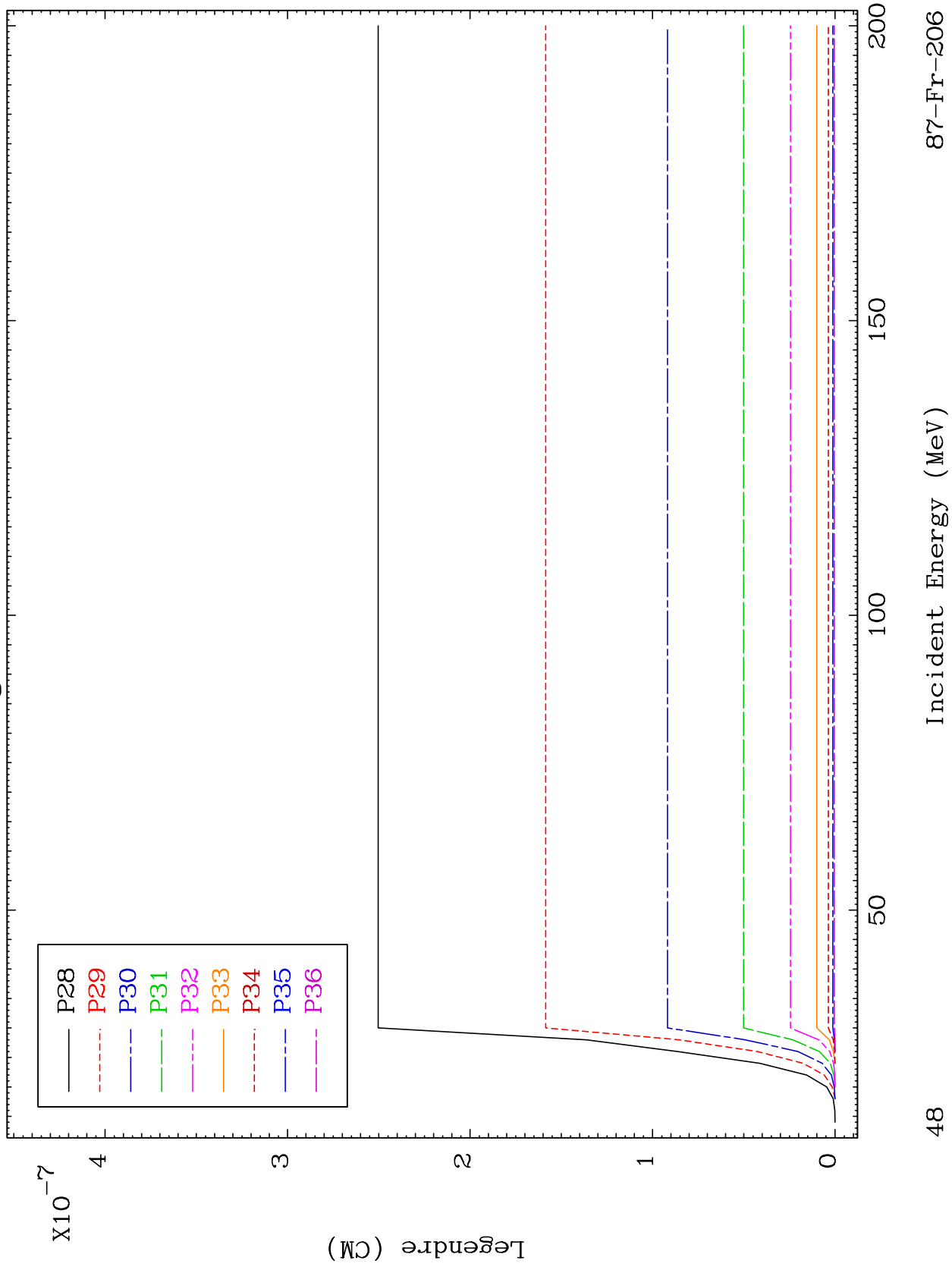
87-Fr-206



MAT 8707

MT= 58 (n, n') Level
Legendre Coefficients

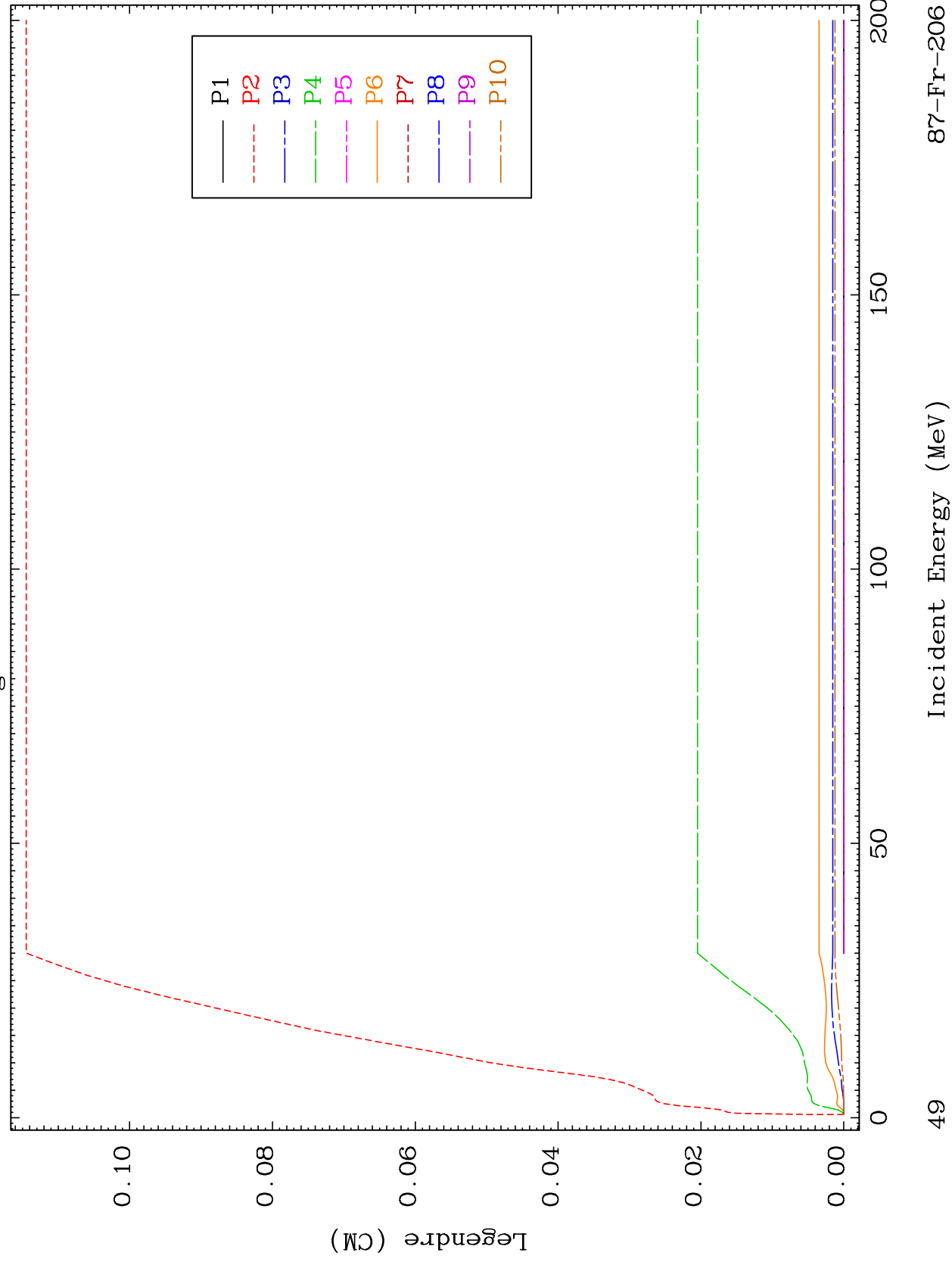
87-Fr-206

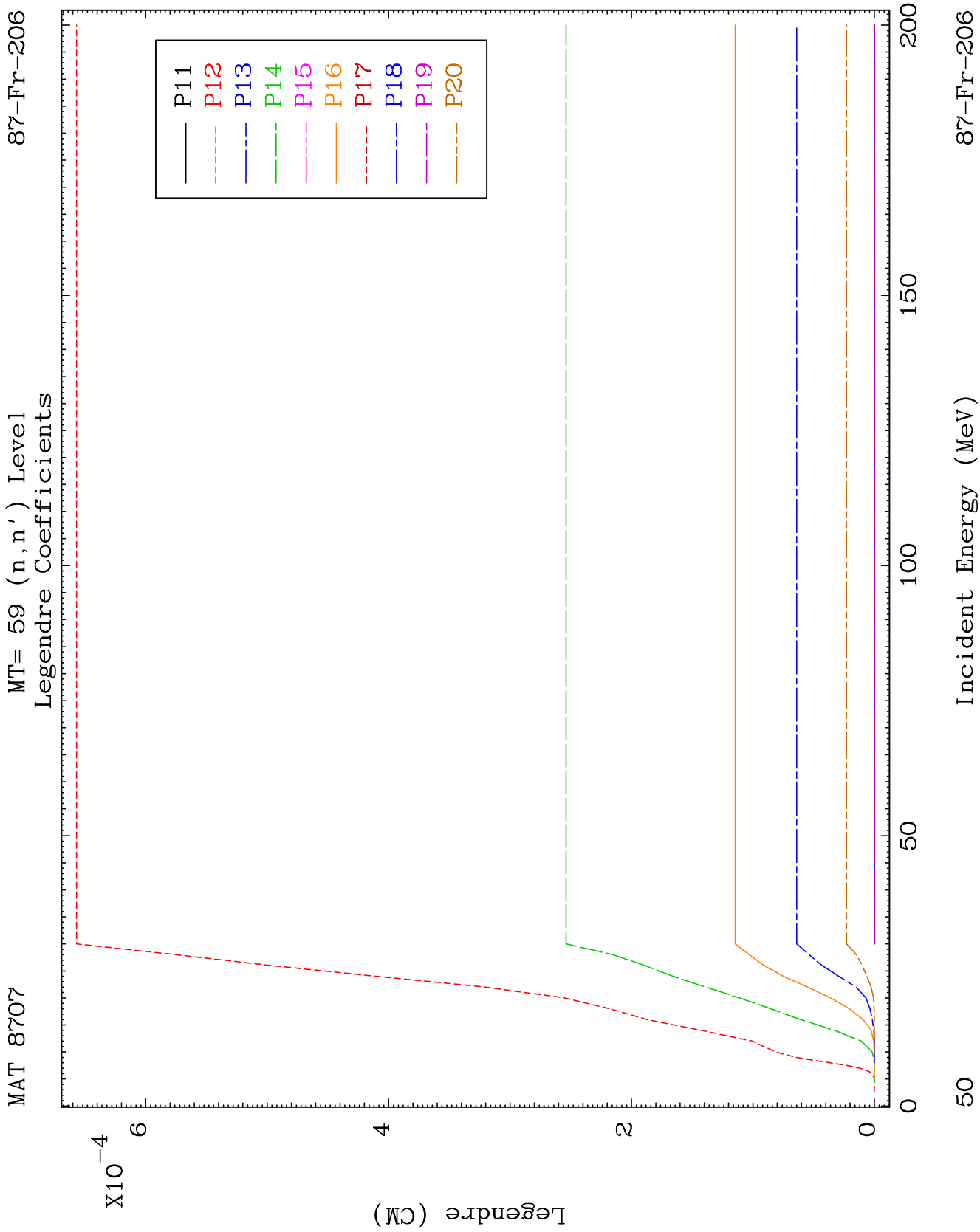


MAT 8707

MT= 59 (n, n') Level
Legendre Coefficients

87-Fr-206

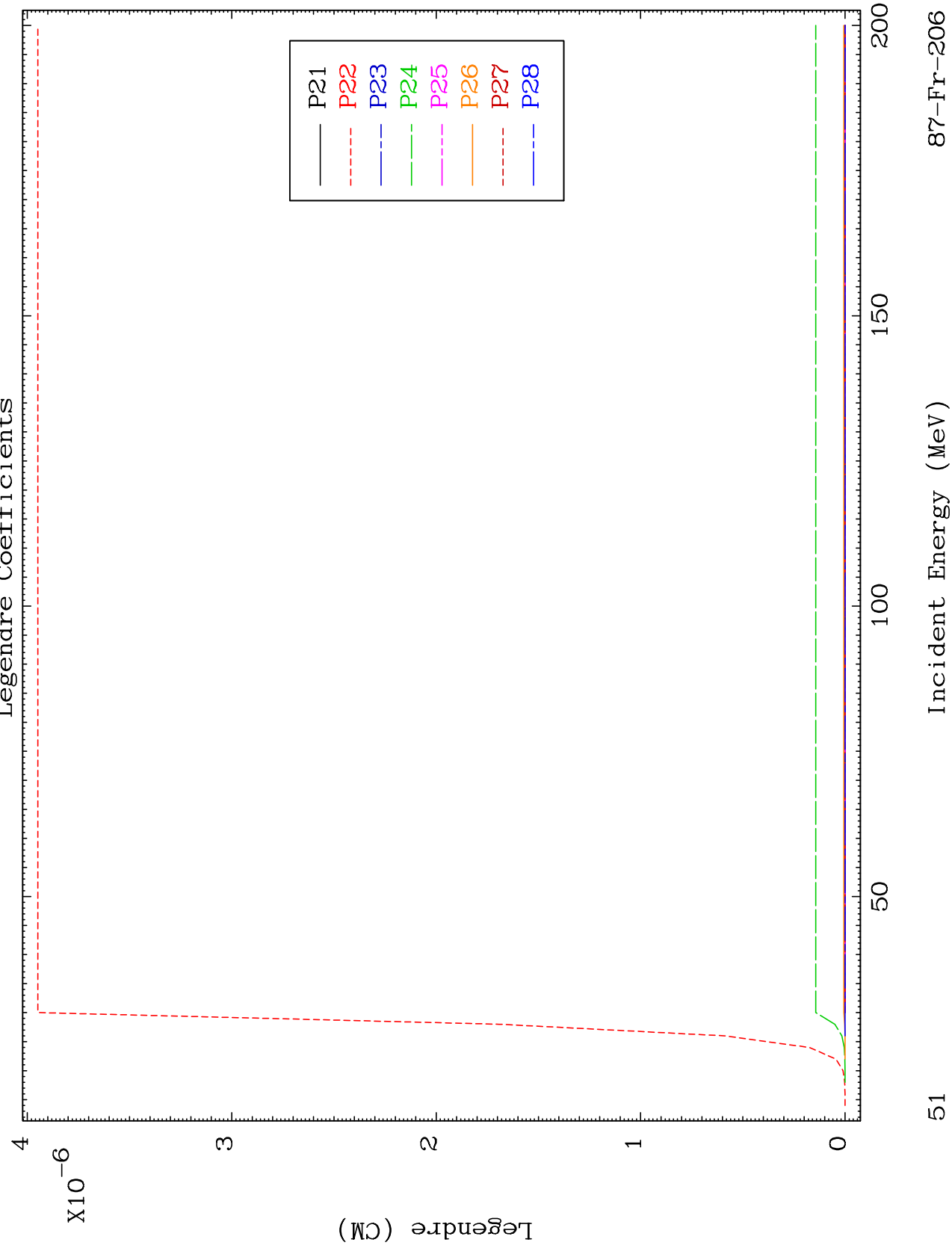


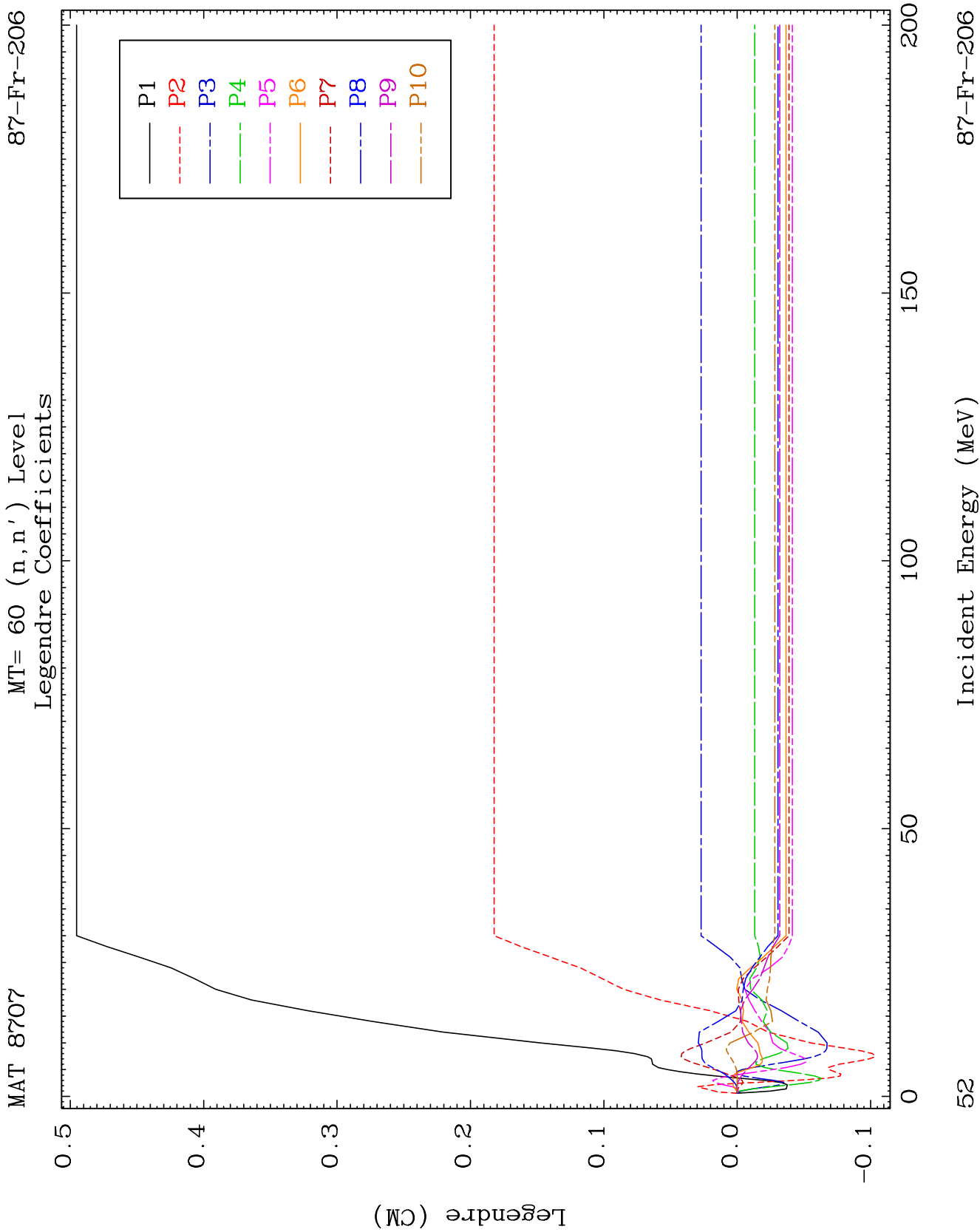


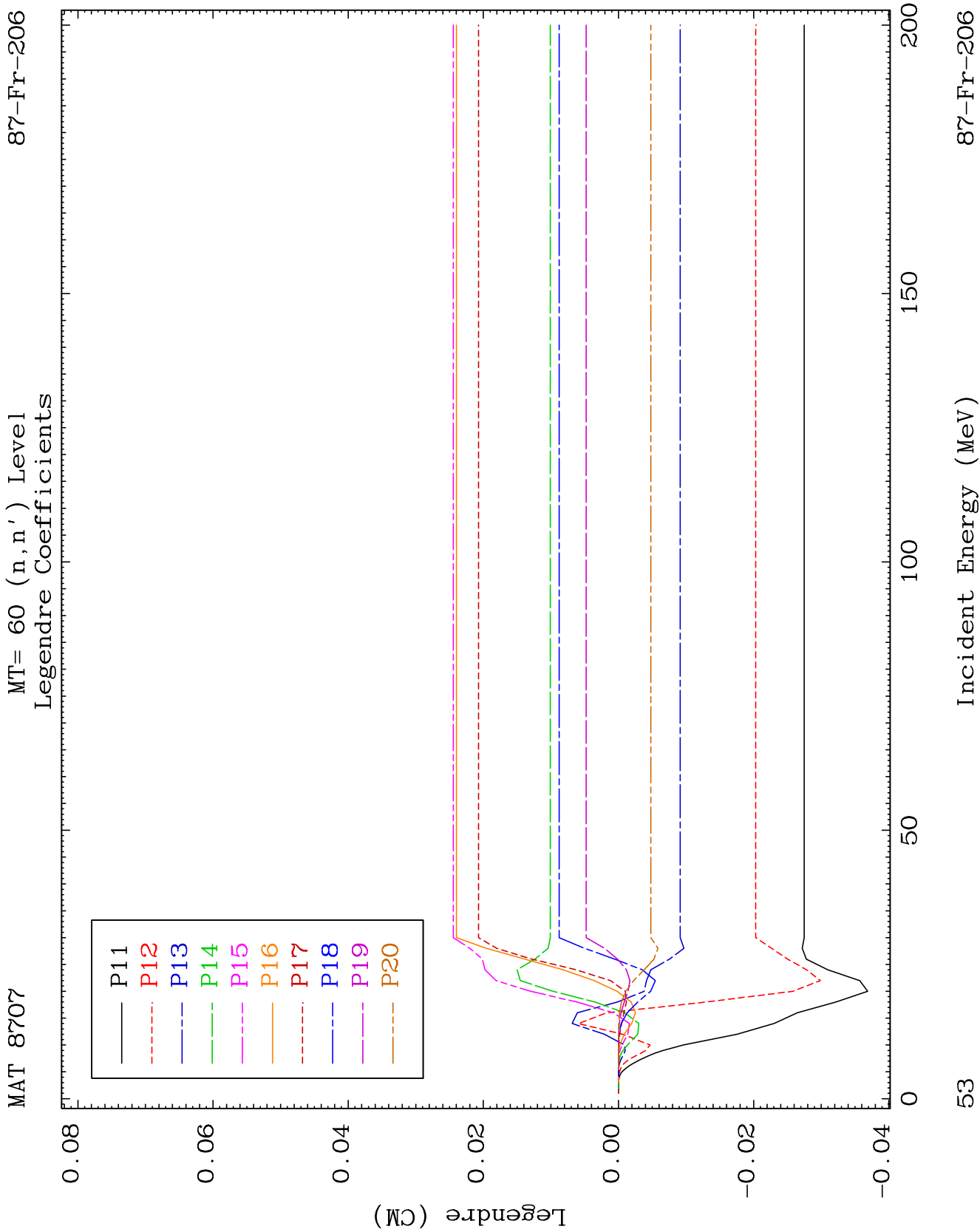
MAT 8707

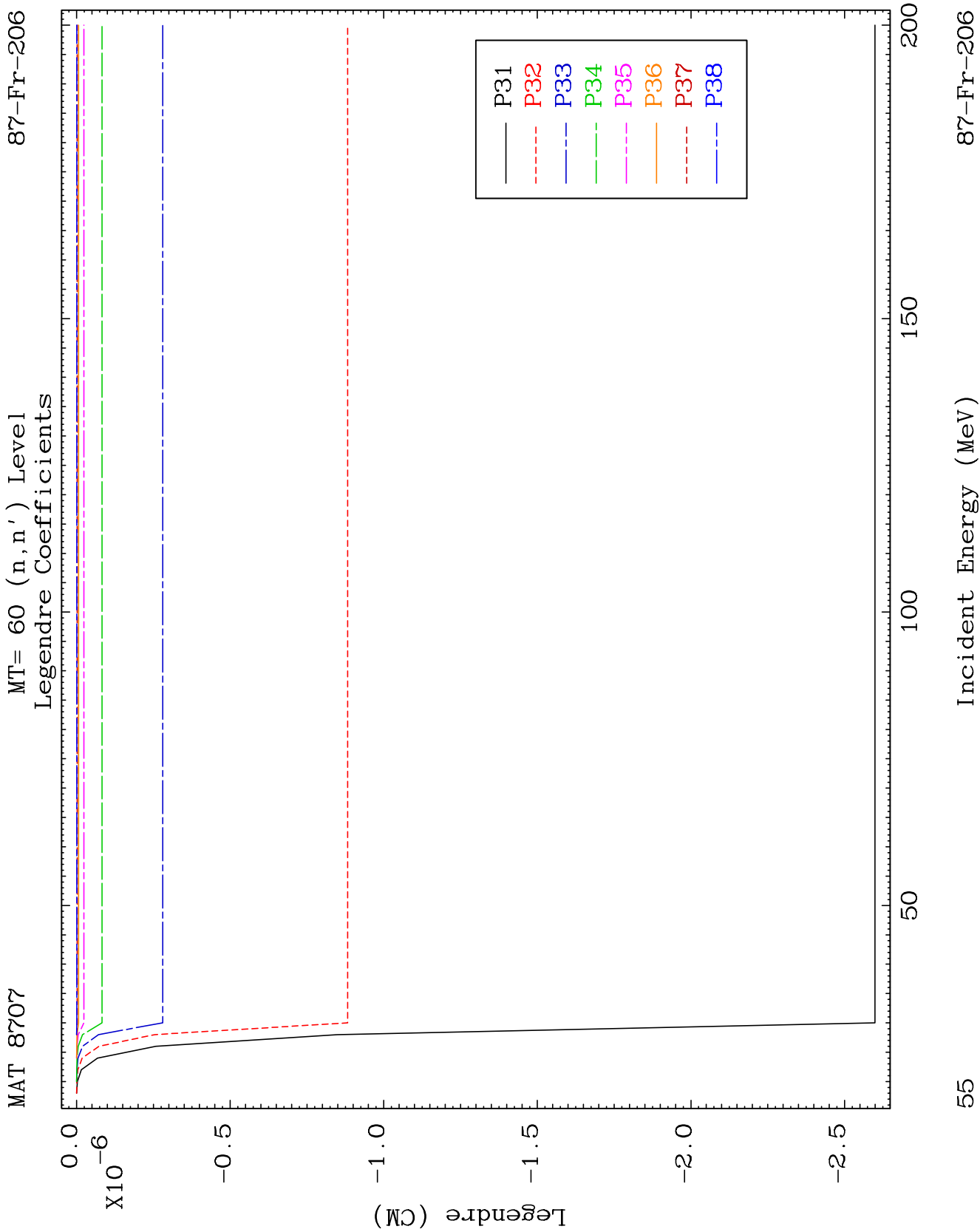
MT= 59 (n, n') Level
Legendre Coefficients

87-Fr-206





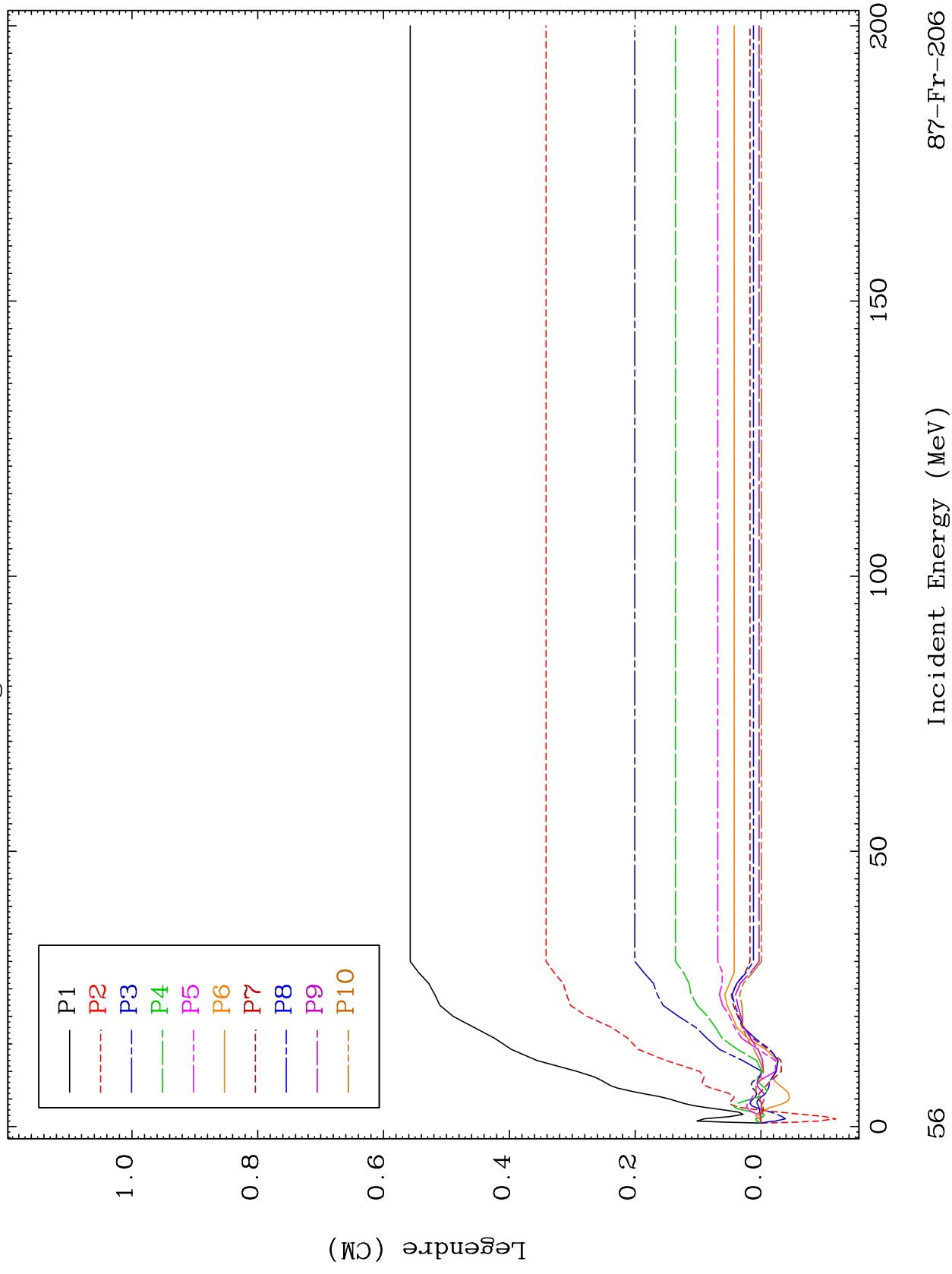


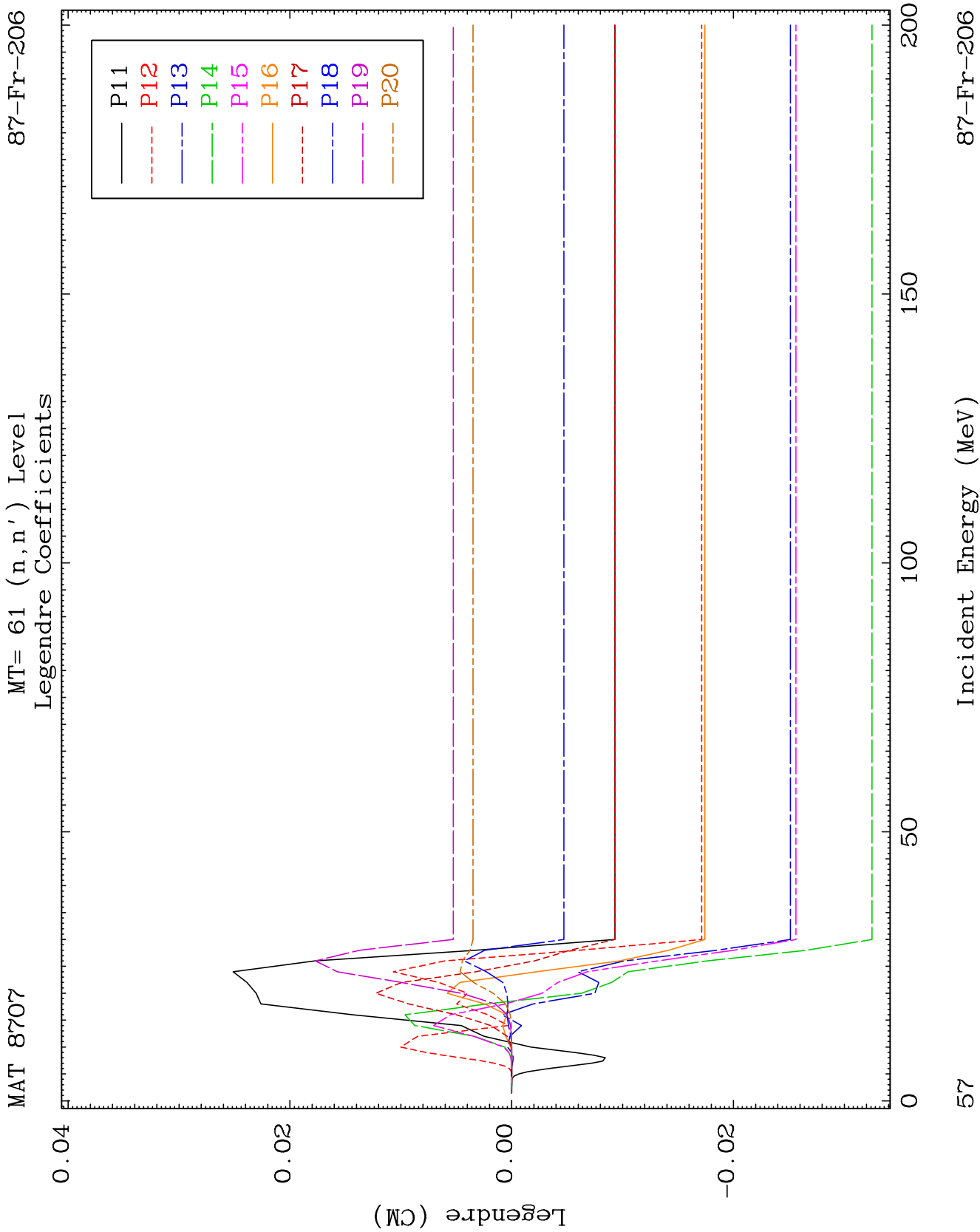


MAT 8707

MT= 61 (n, n') Level
Legendre Coefficients

87-Fr-206

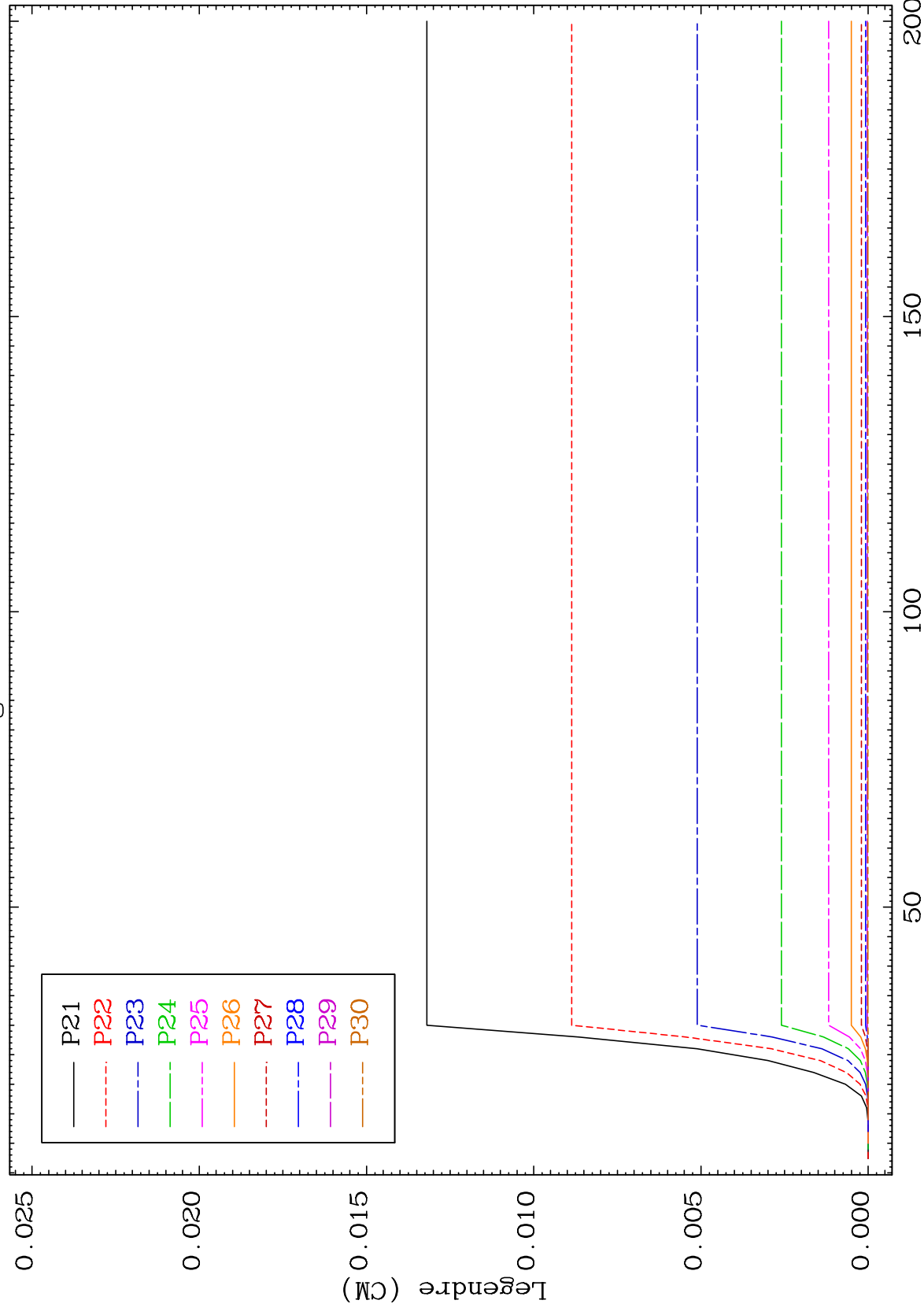




MAT 8707

MT= 61 (n, n') Level
Legendre Coefficients

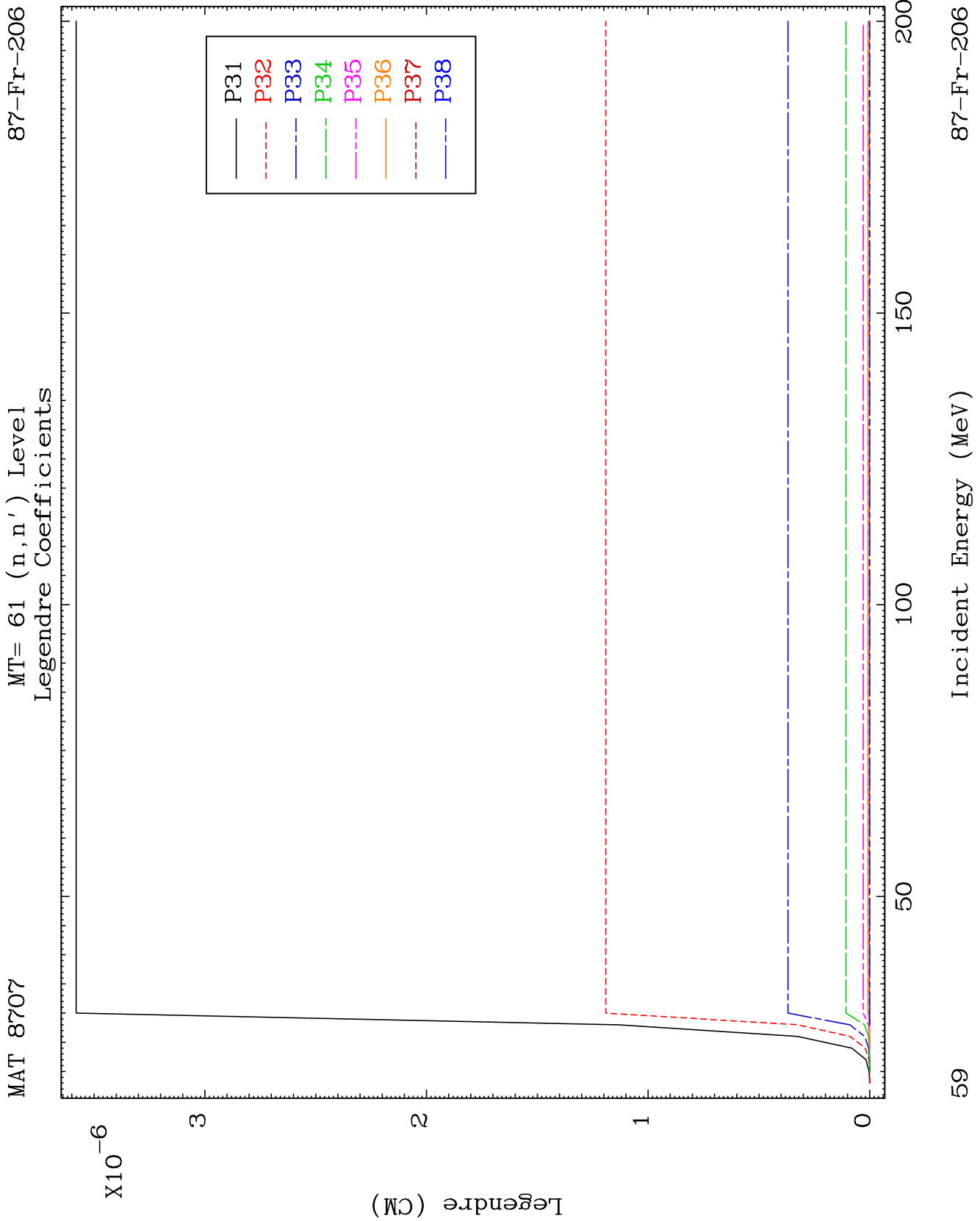
87-Fr-206

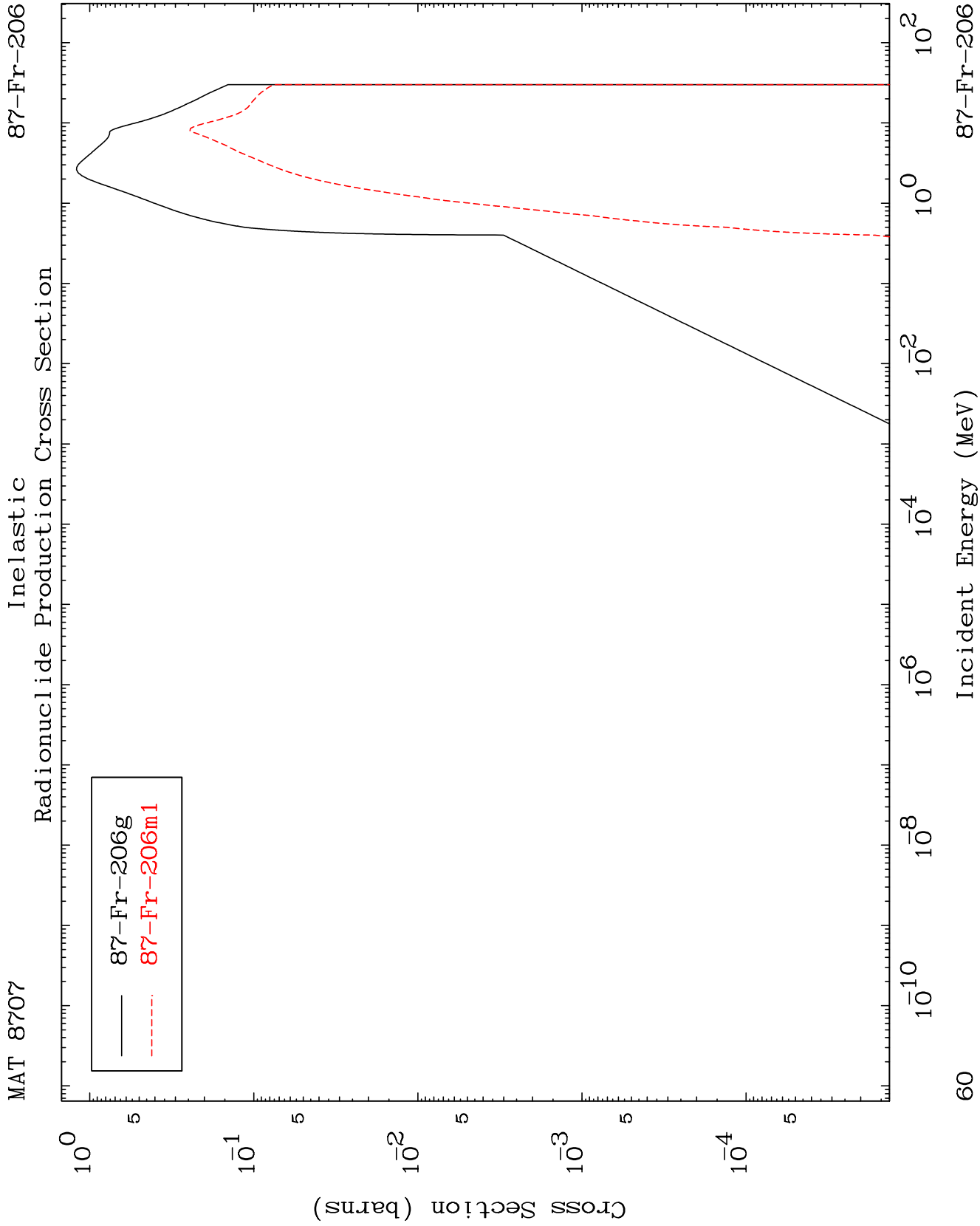


58

Incident Energy (MeV)

87-FR-206

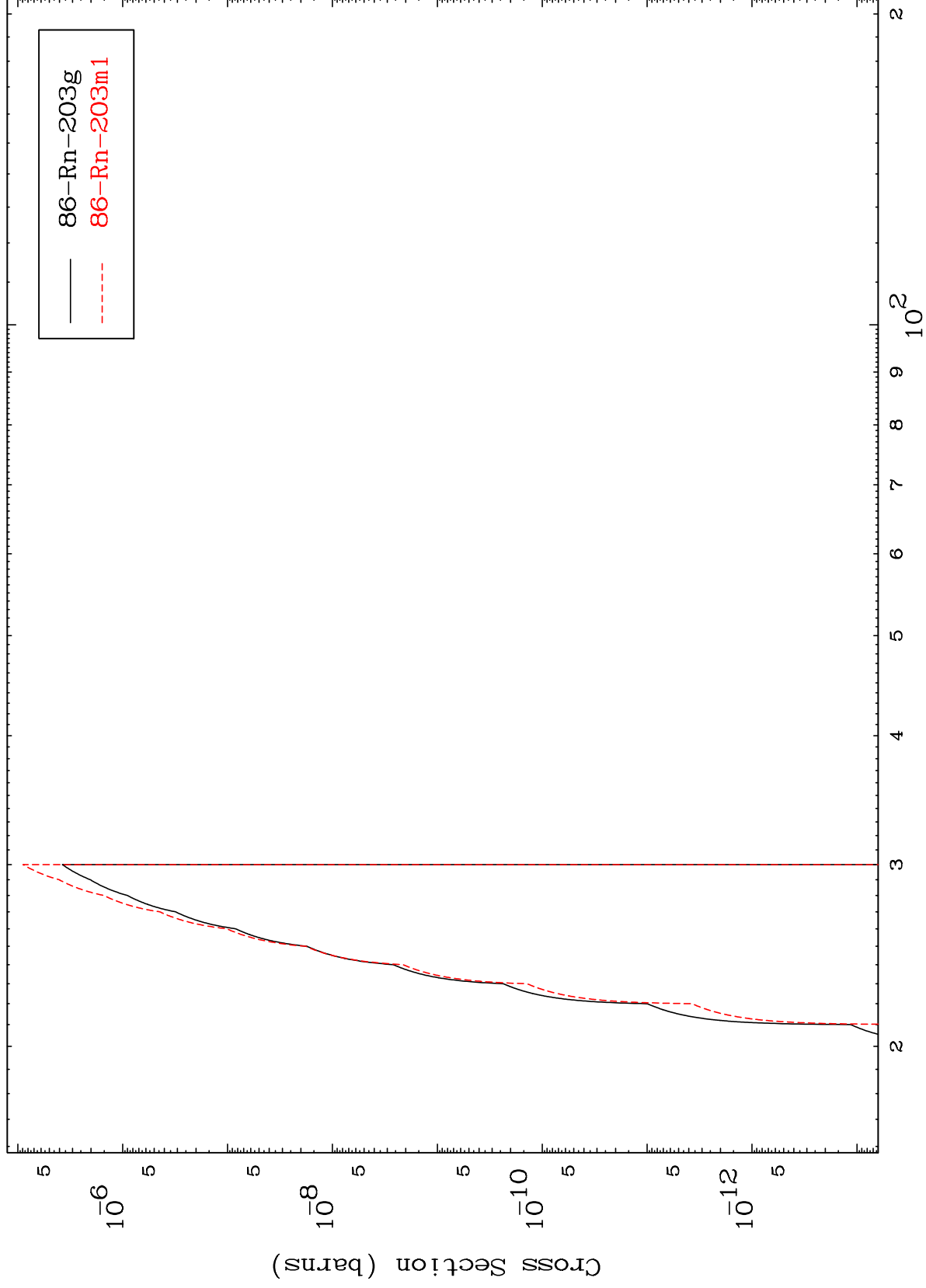




MAT 8707

87-Fr-206

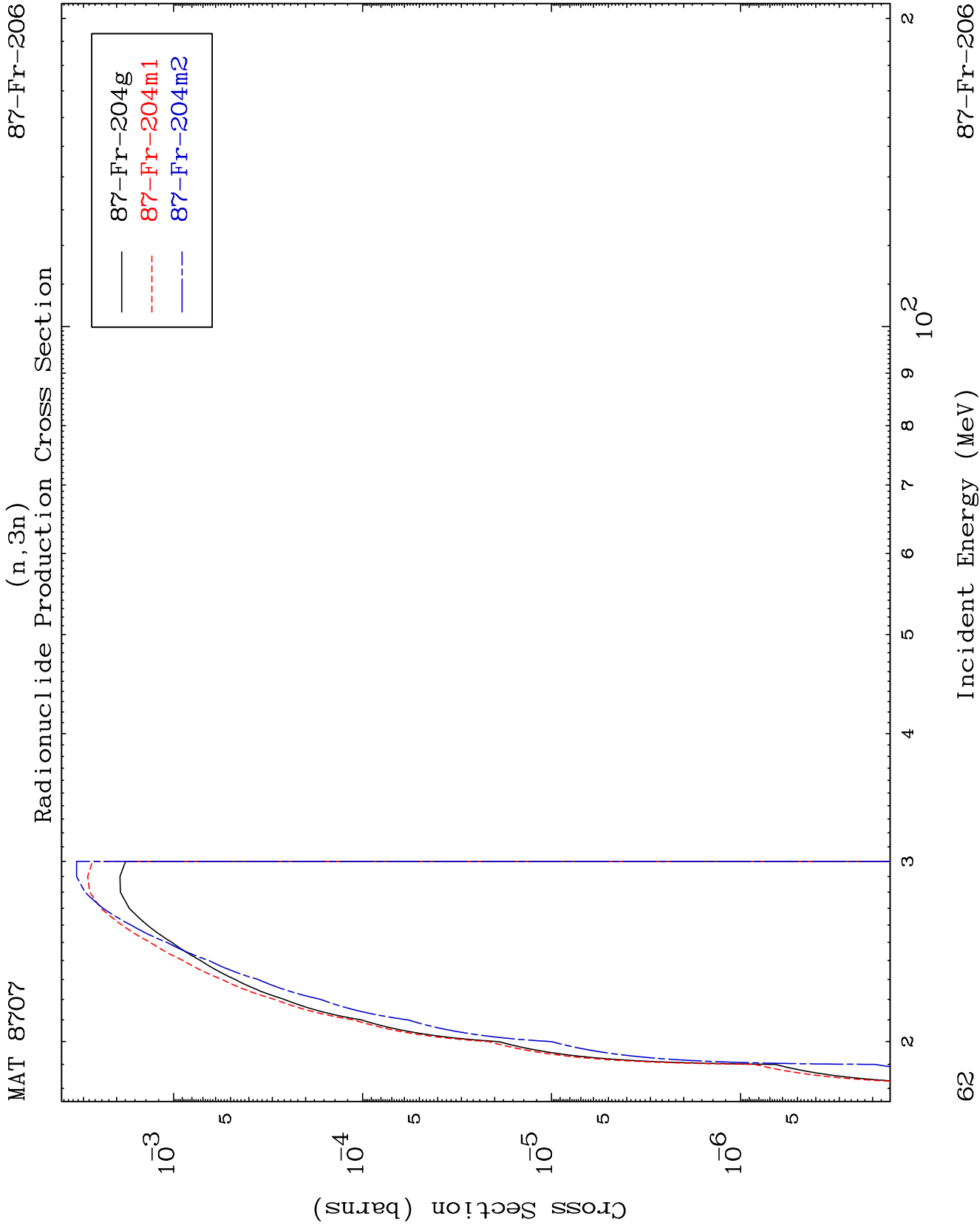
(n,2n) d
Radionuclide Production Cross Section



61

Incident Energy (MeV)

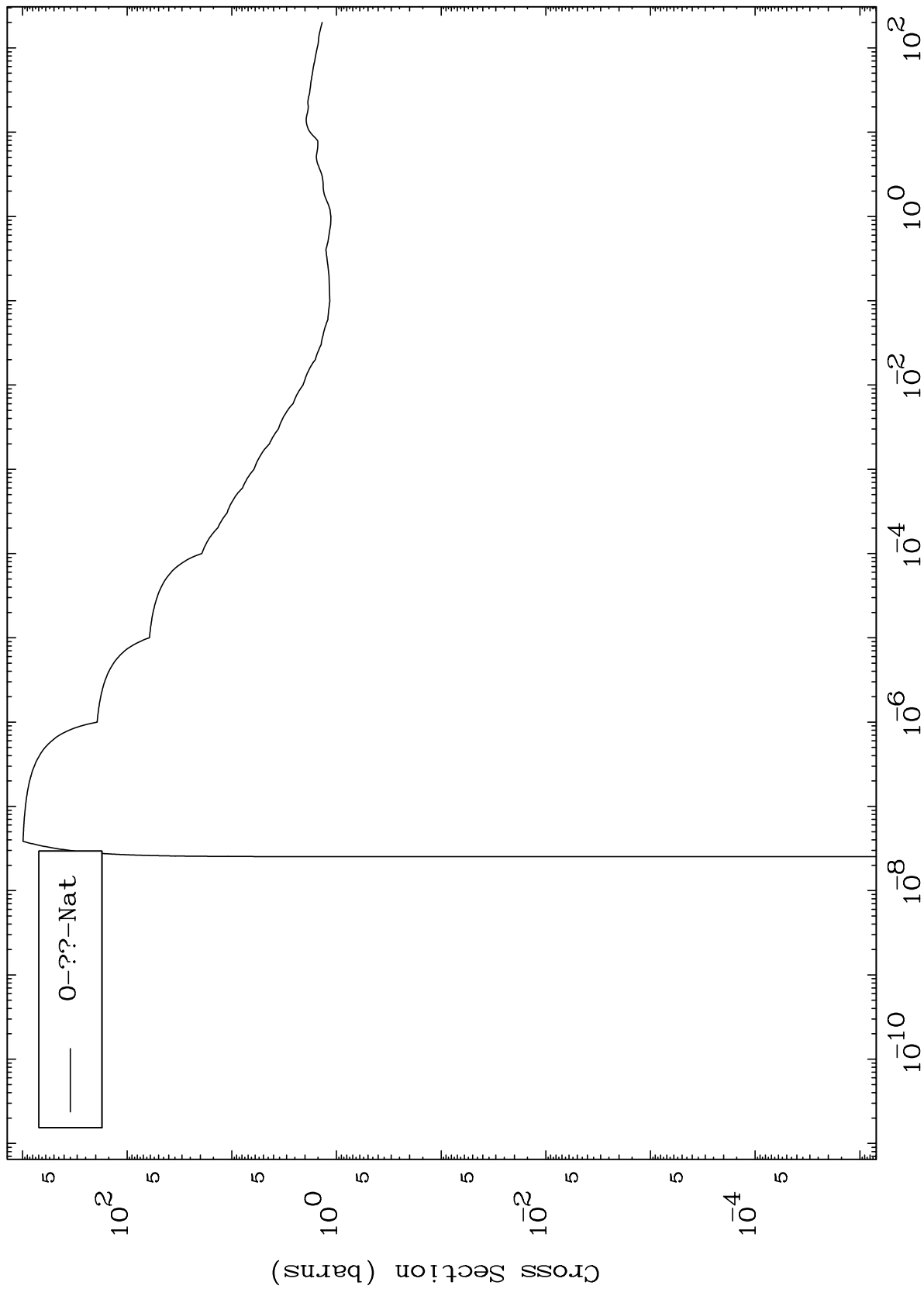
87-Fr-206



MAT 8707

87-Fr-206

Fission
Radionuclide Production Cross Section



63

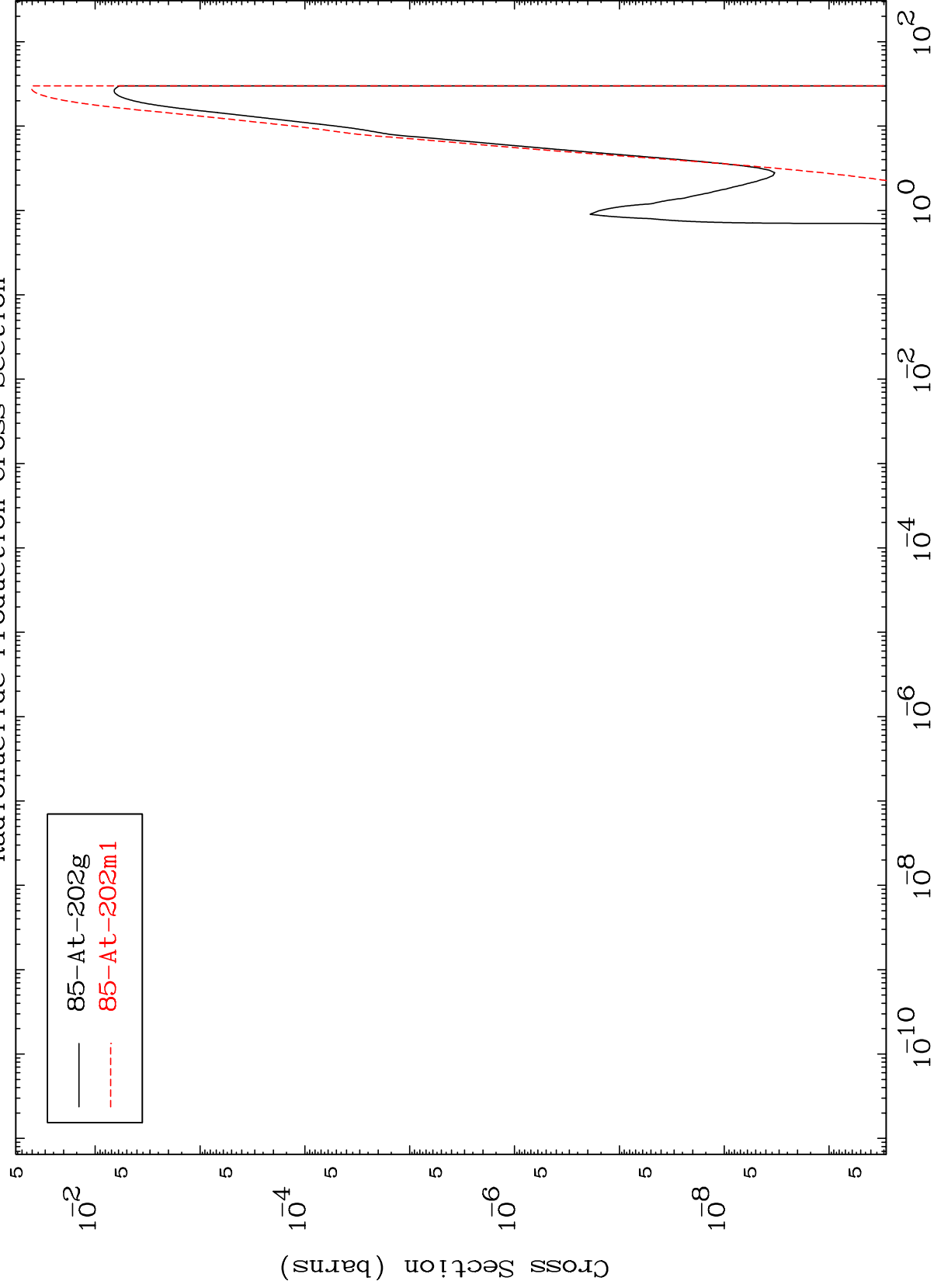
Incident Energy (MeV)

87-Fr-206

MAT 8707

87-Fr-206

$(n,n') \alpha$
Radionuclide Production Cross Section

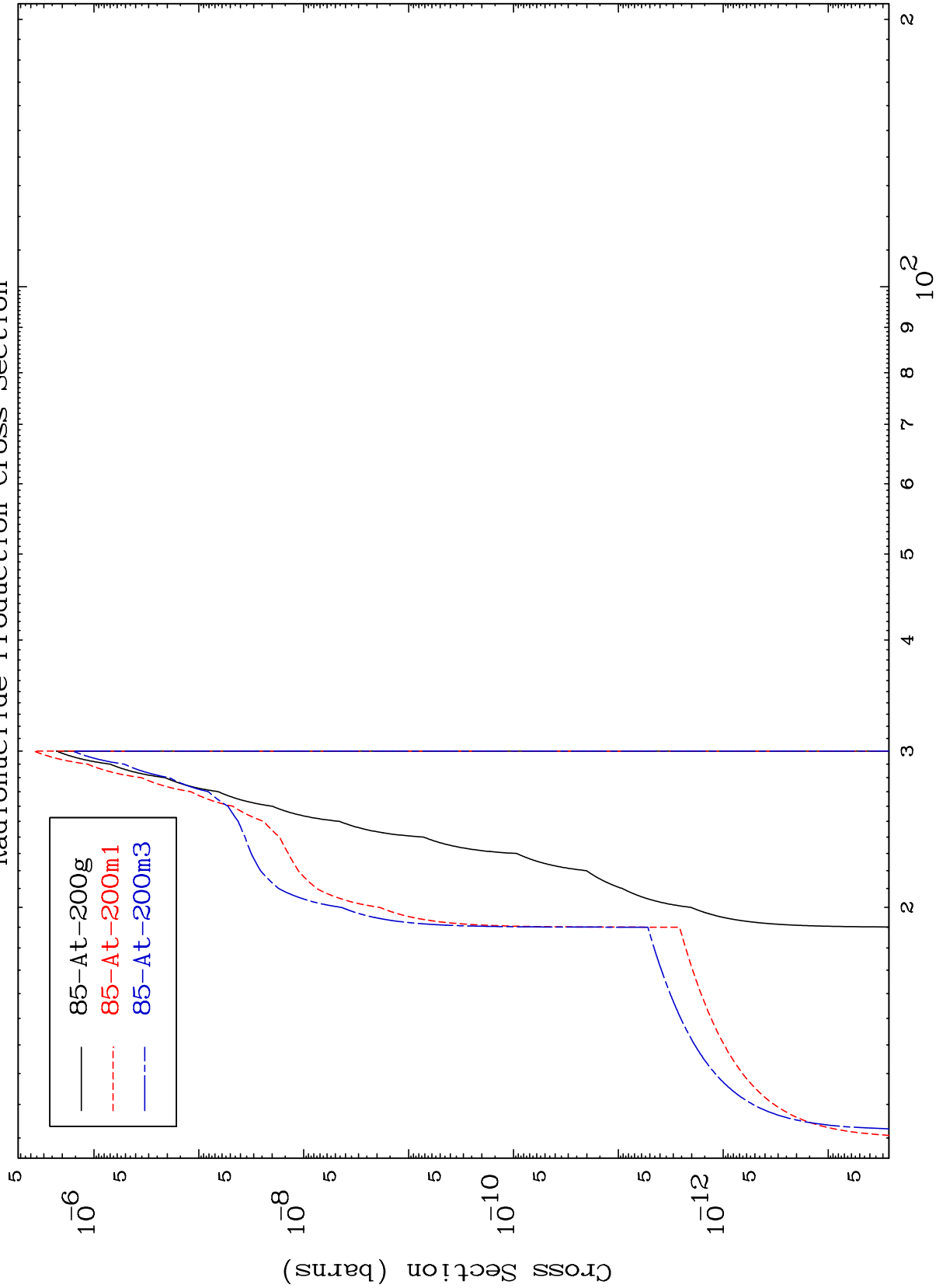


64

Incident Energy (MeV)

87-Fr-206

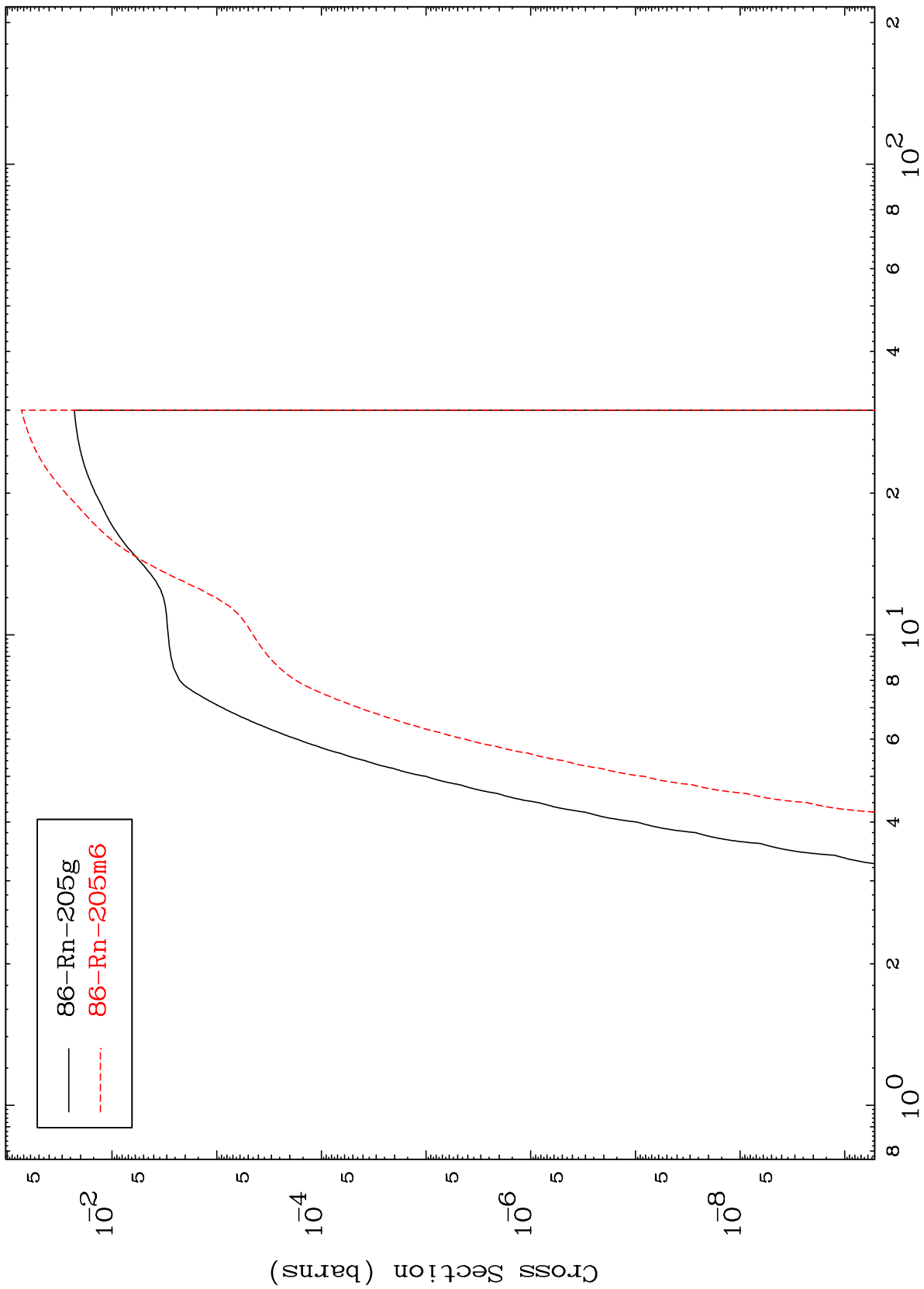
Radionuclide Production Cross Section
(n,3n) α



MAT 8707

87-Fr-206

(n,n') p
Radionuclide Production Cross Section



Incident Energy (MeV)

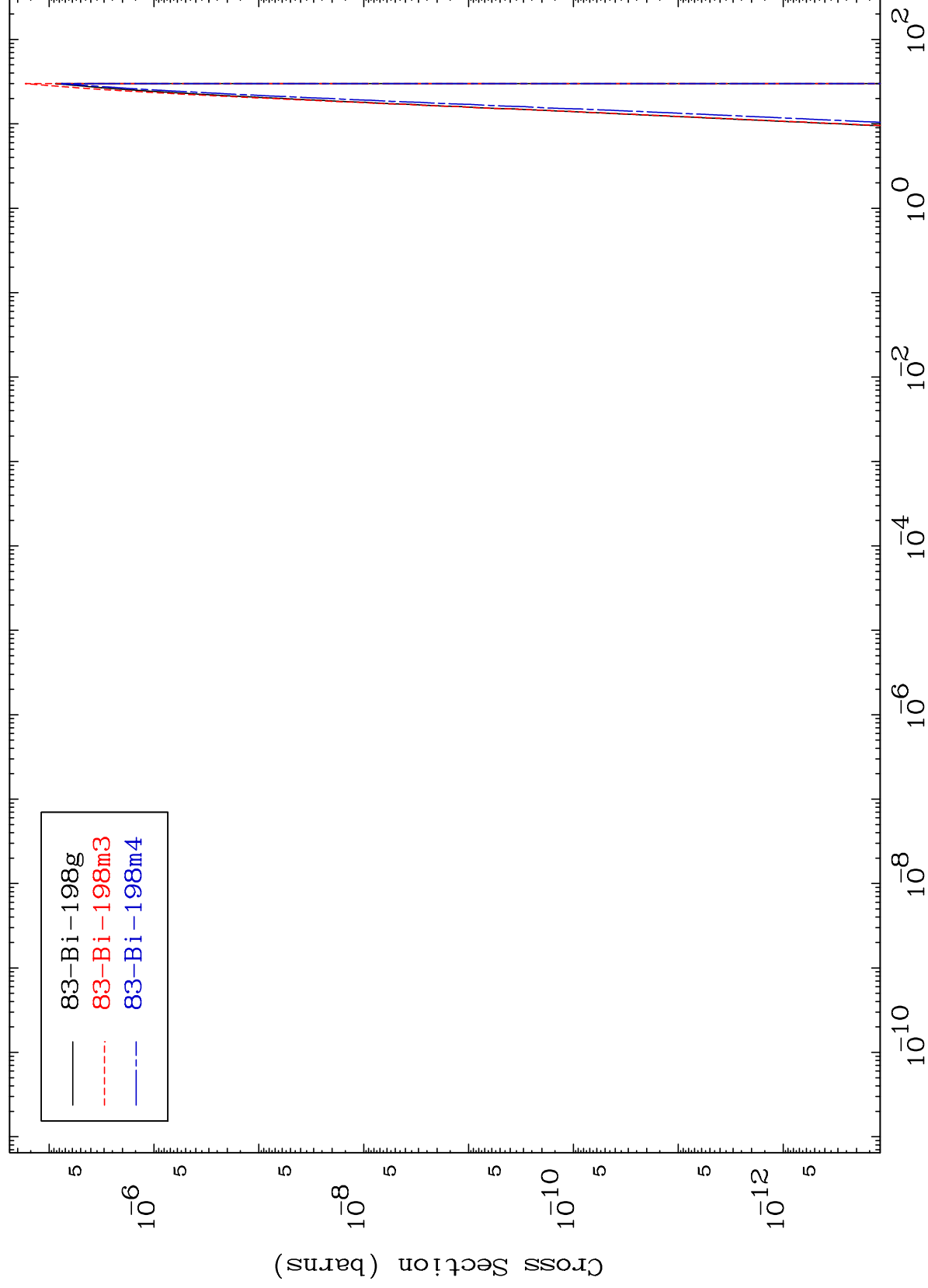
87-Fr-206

66

MAT 8707

87-Fr-206

(n,n') 2 α
Radionuclide Production Cross Section



67

Incident Energy (MeV)

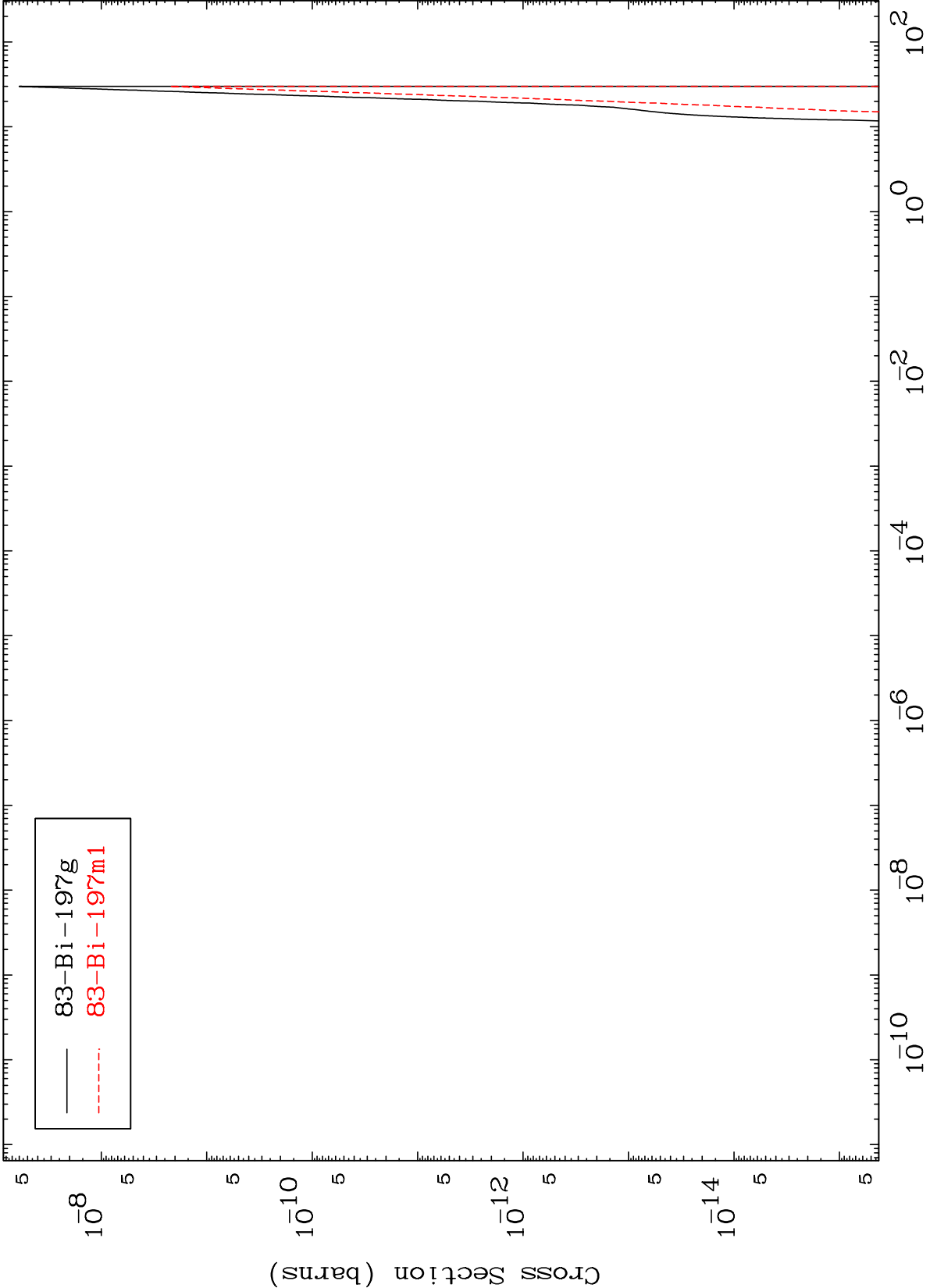
87-Fr-206

MAT 8707

(n,2n) 2 α

87-Fr-206

Radionuclide Production Cross Section



68

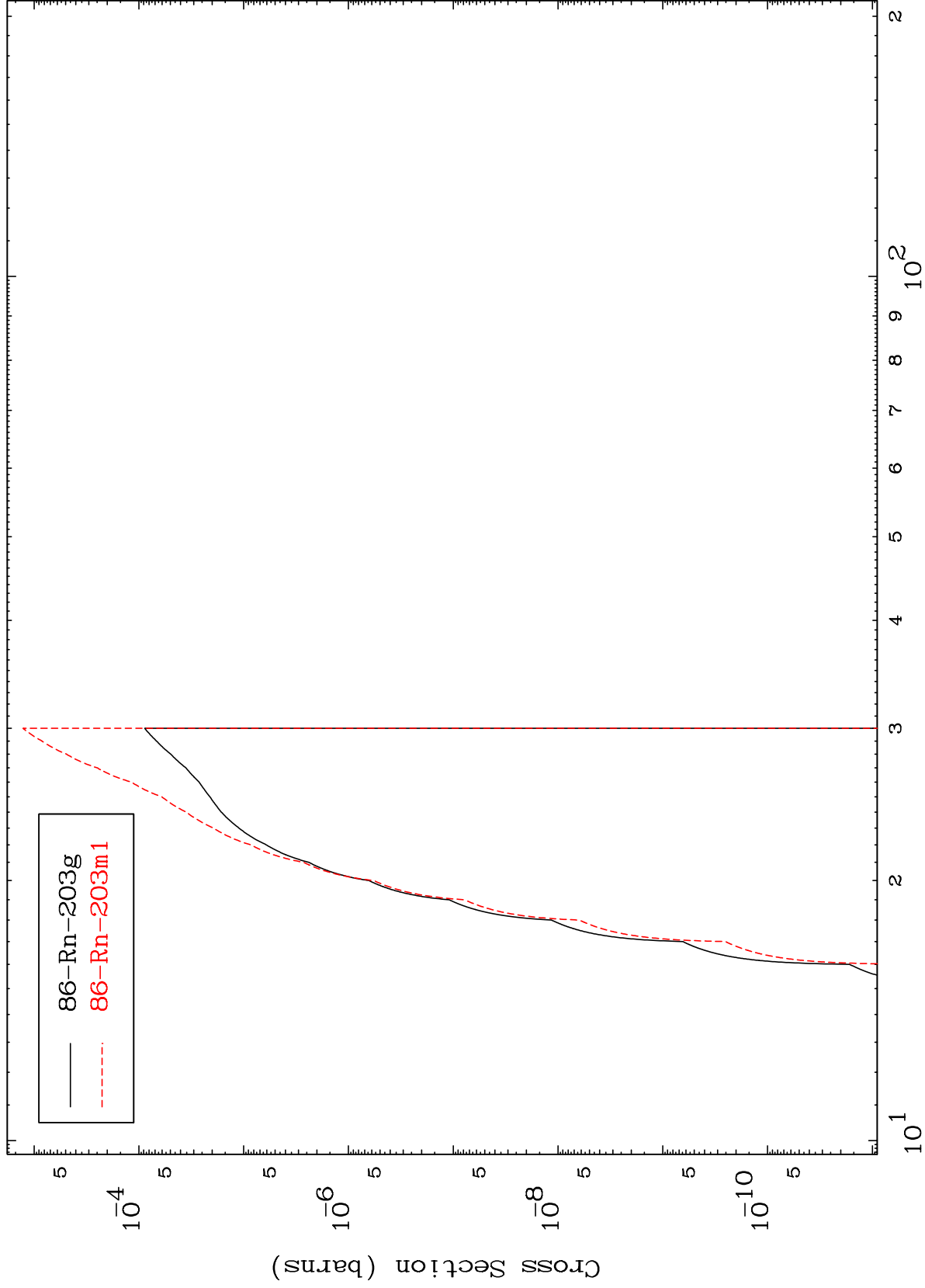
Incident Energy (MeV)

87-Fr-206

MAT 8707

87-Fr-206

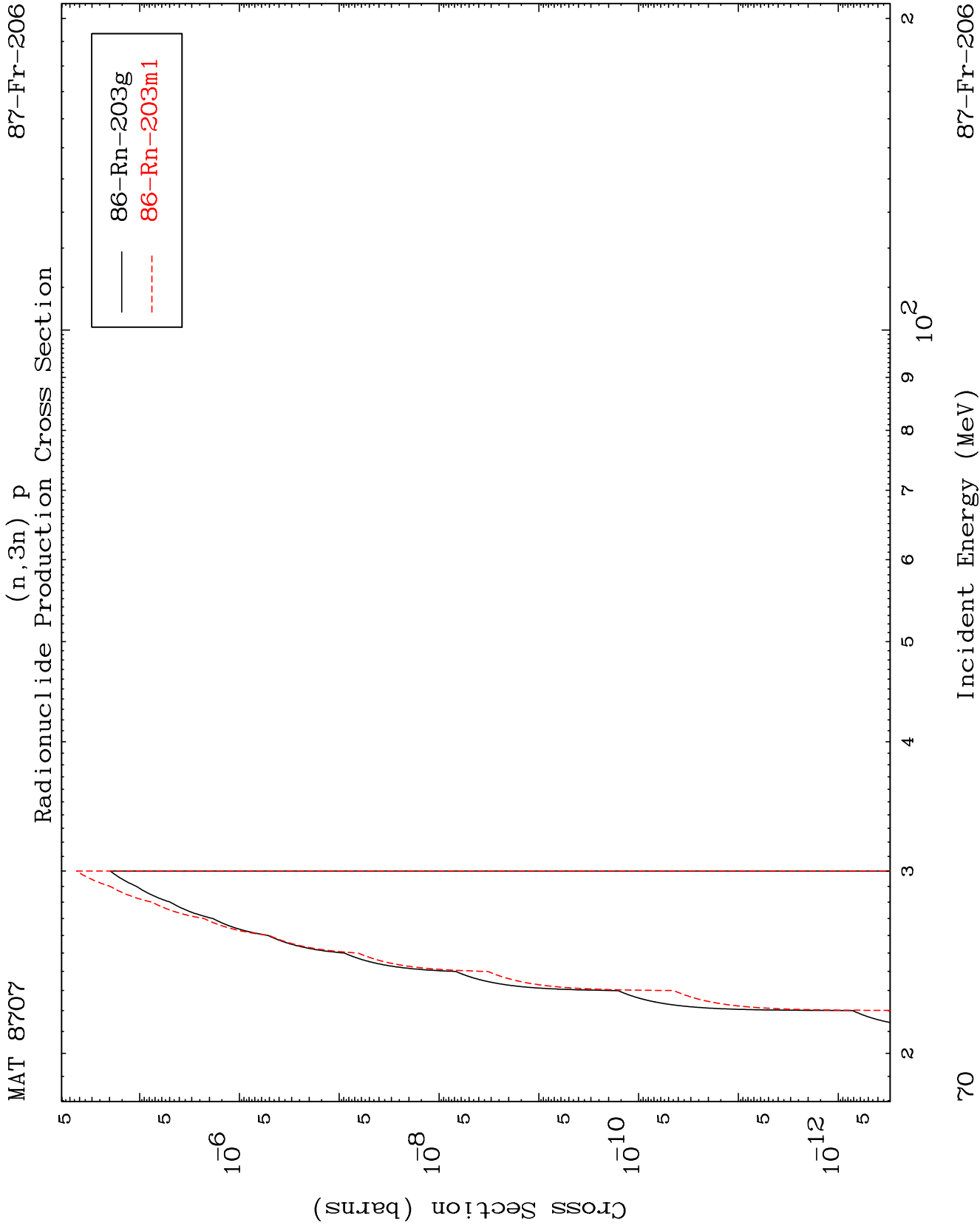
(n,n') t
Radionuclide Production Cross Section



87-Fr-206

Incident Energy (MeV)

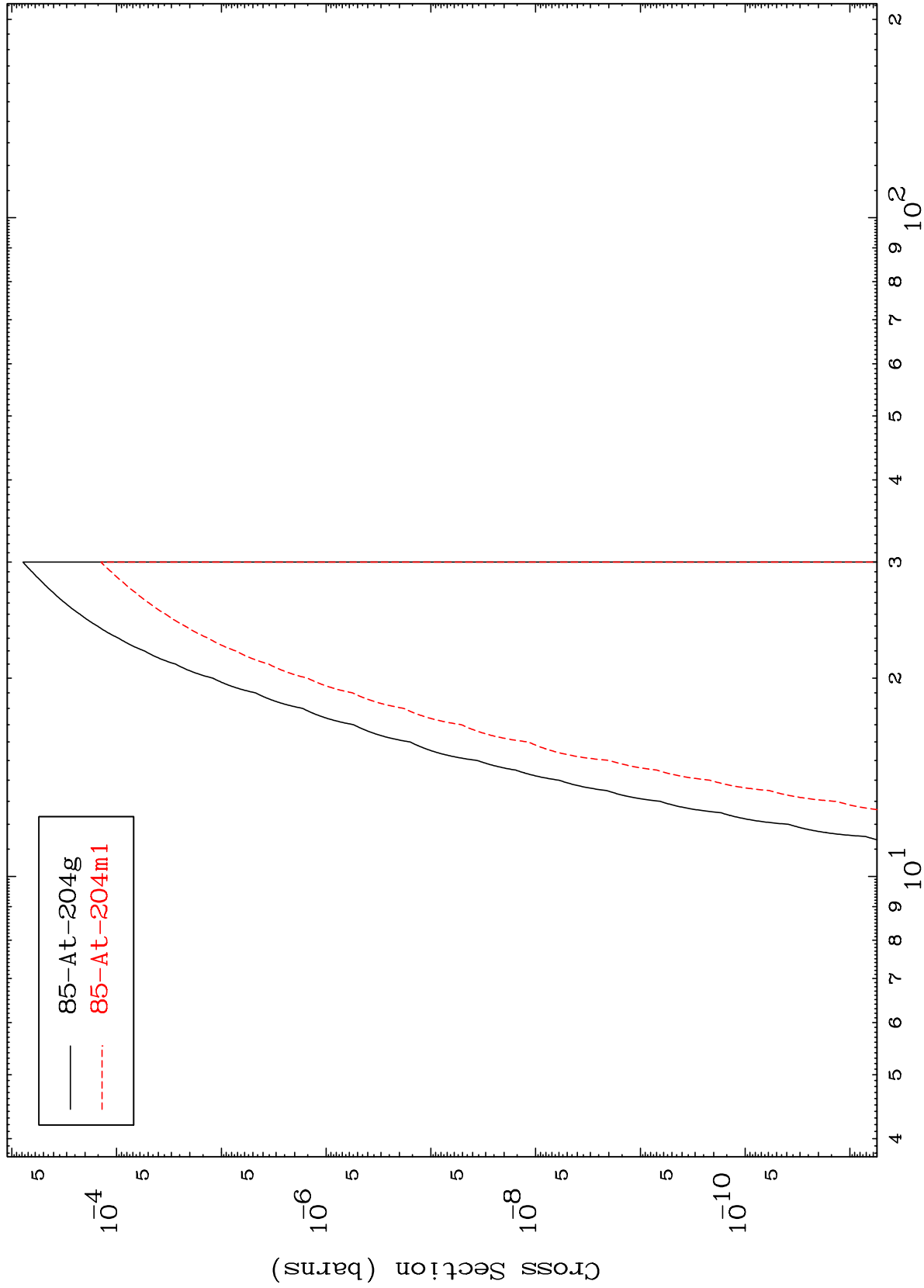
69



MAT 8707

87-Fr-206

(n,2n) p
Radionuclide Production Cross Section



71

Incident Energy (MeV)

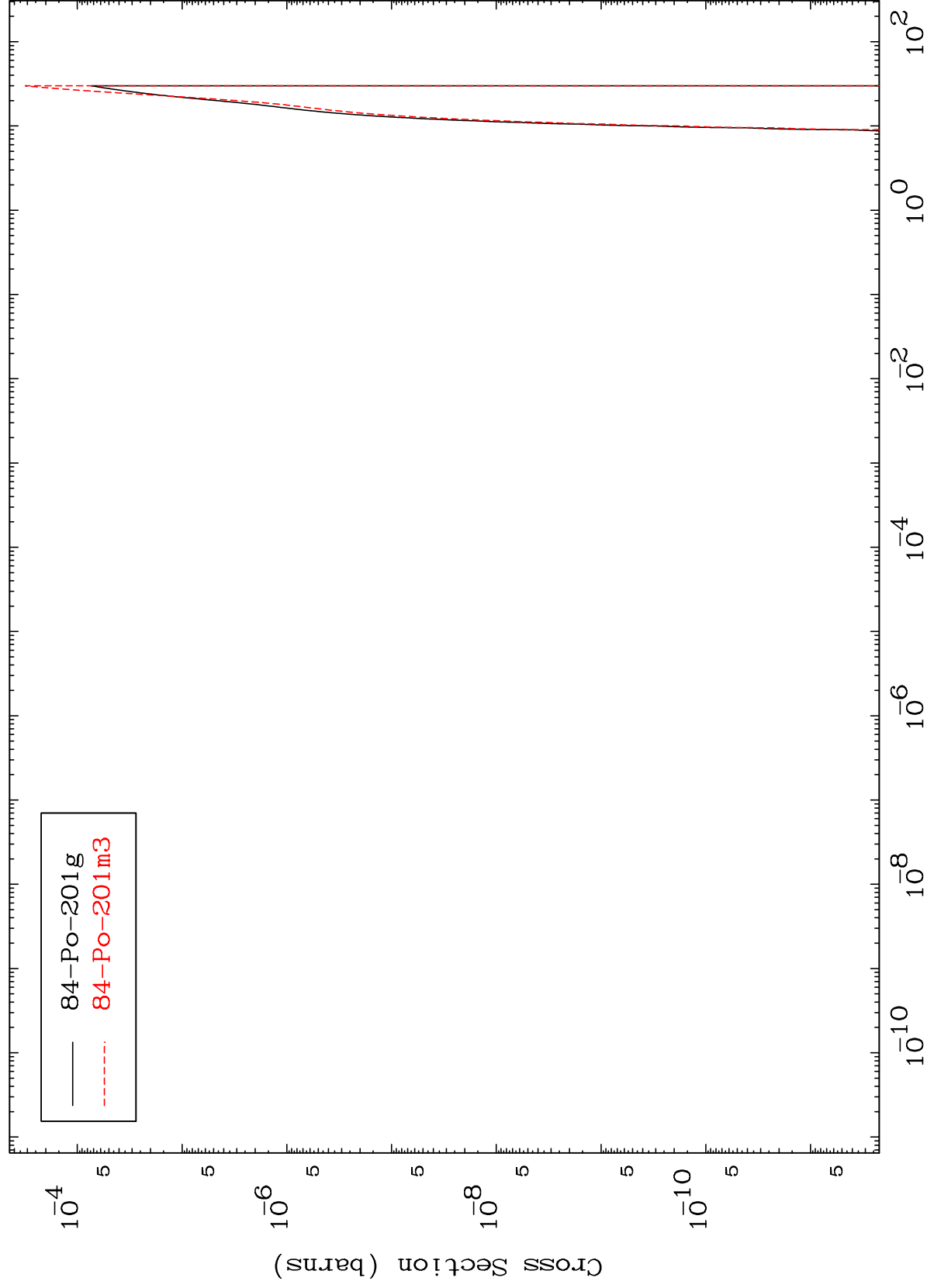
87-Fr-206

MAT 8707

(n,n') p α

87-Fr-206

Radionuclide Production Cross Section

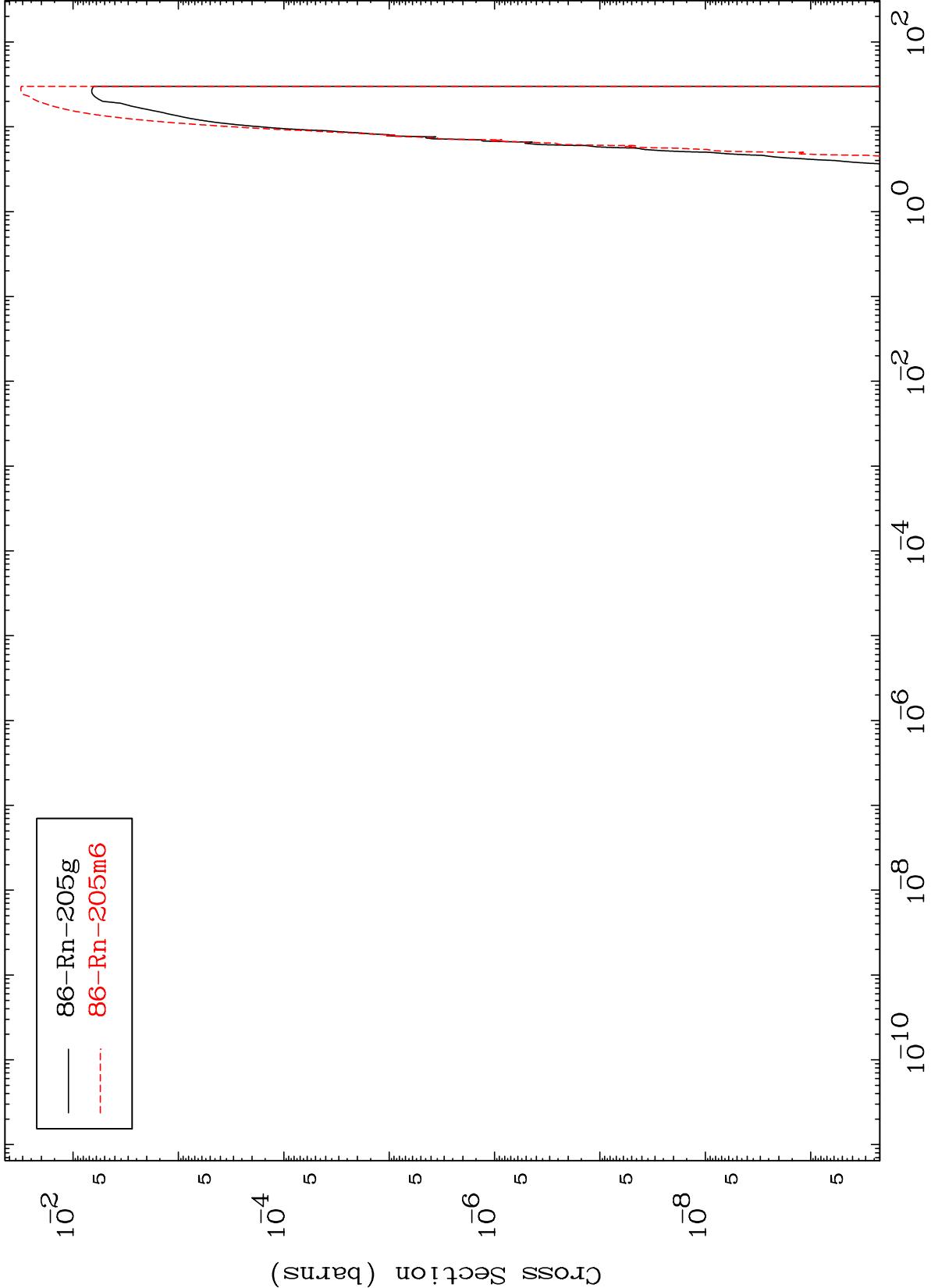


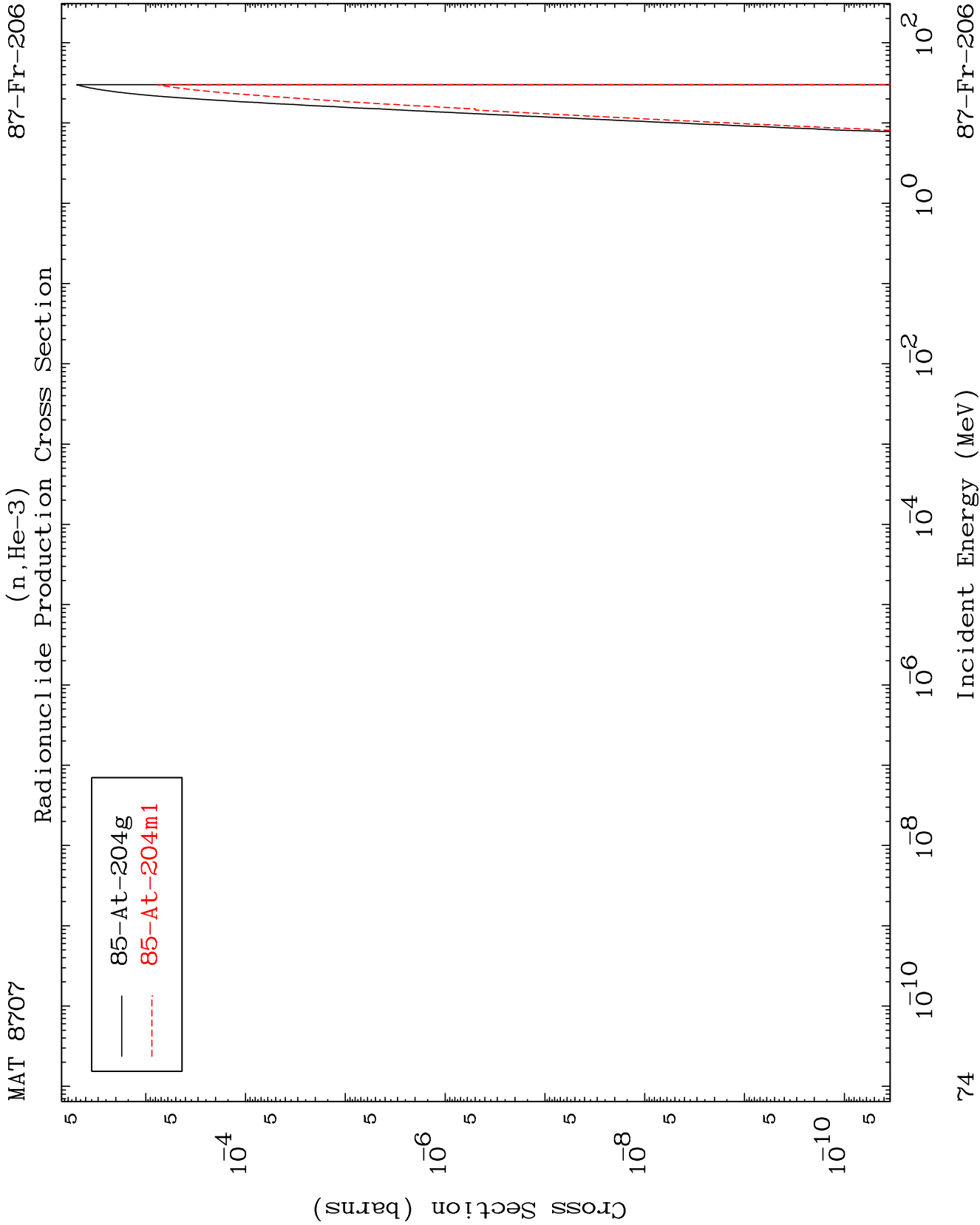
72

Incident Energy (MeV)

87-Fr-206

Radionuclide Production Cross Section



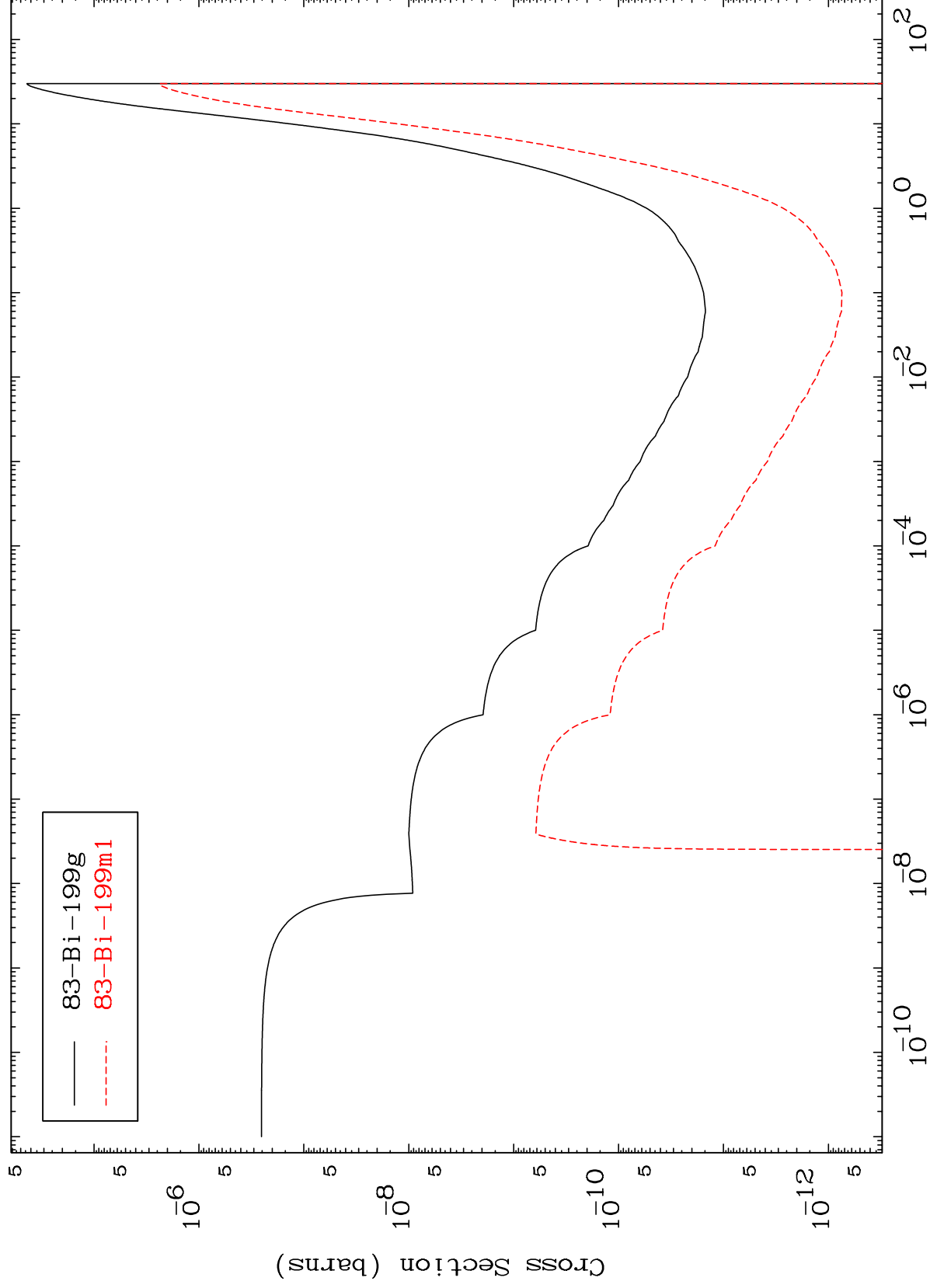


MAT 8707

(n,2α)

87-Fr-206

Radionuclide Production Cross Section



75

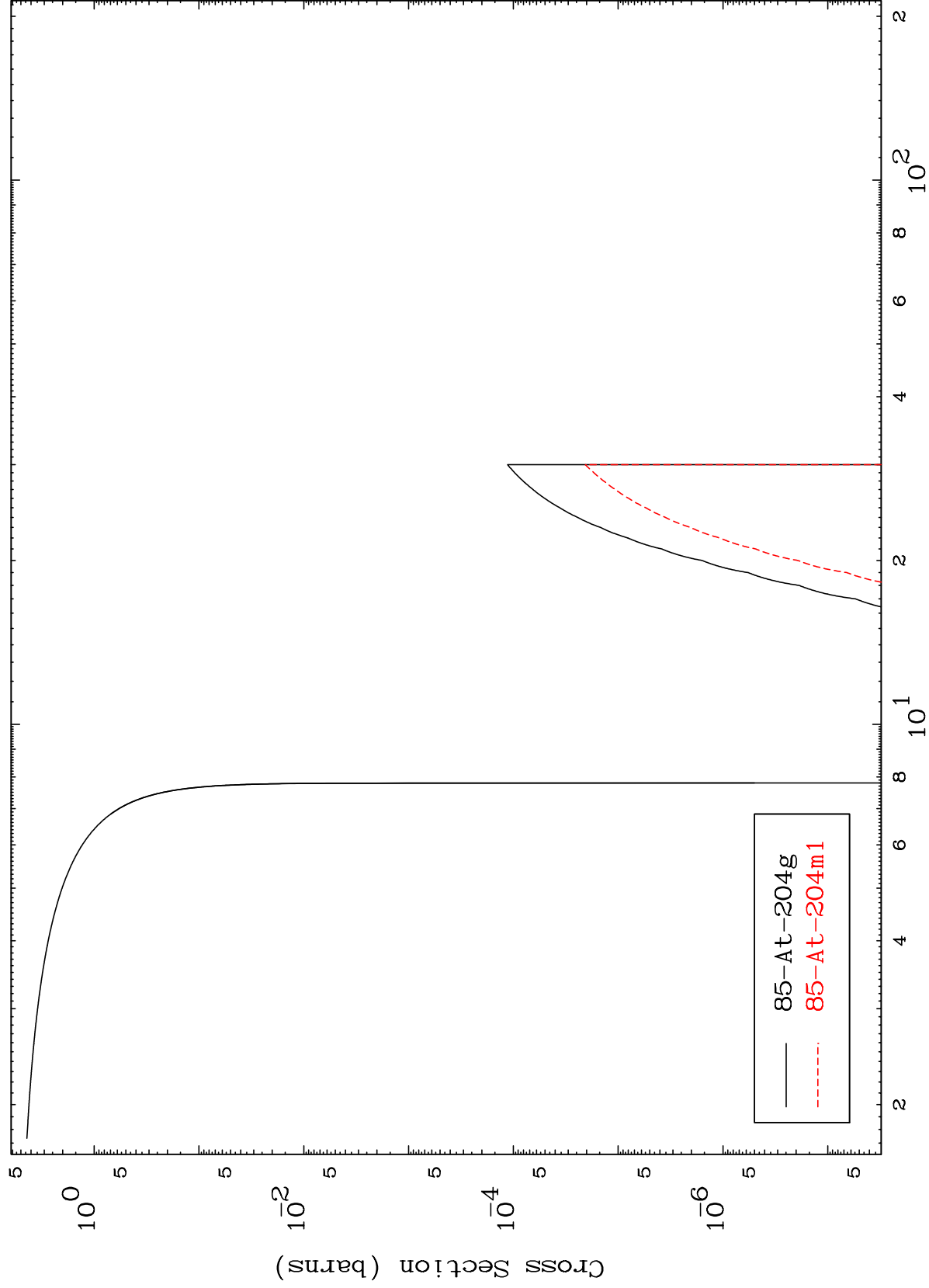
Incident Energy (MeV)

87-Fr-206

MAT 8707

87-Fr-206

(n,p) d
Radionuclide Production Cross Section



76

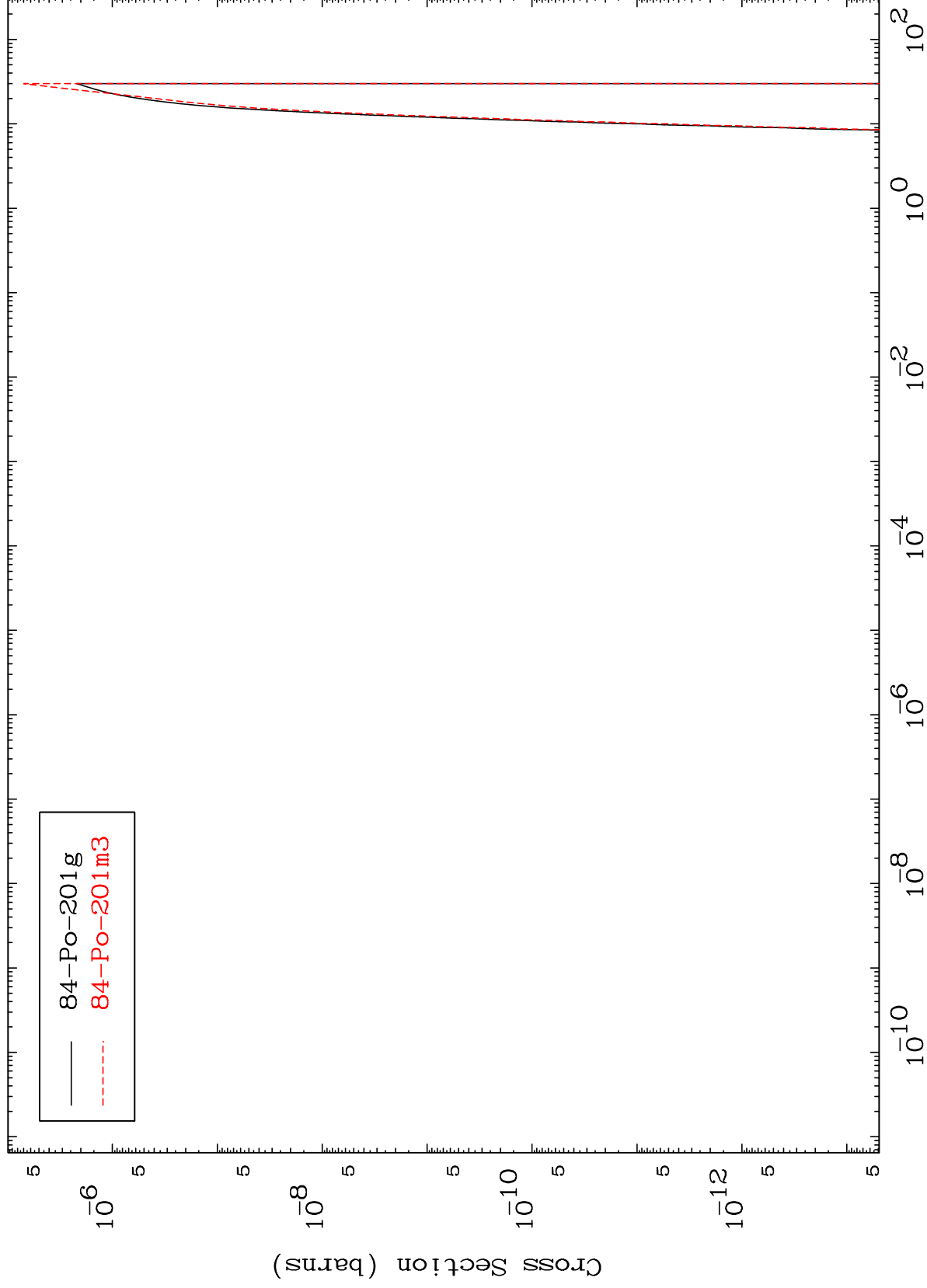
Incident Energy (MeV)

87-Fr-206

MAT 8707

87-Fr-206

(n,d) α
Radionuclide Production Cross Section



77

Incident Energy (MeV)

87-Fr-206