

Program EVALPLOT
(Version 2021-1)

by

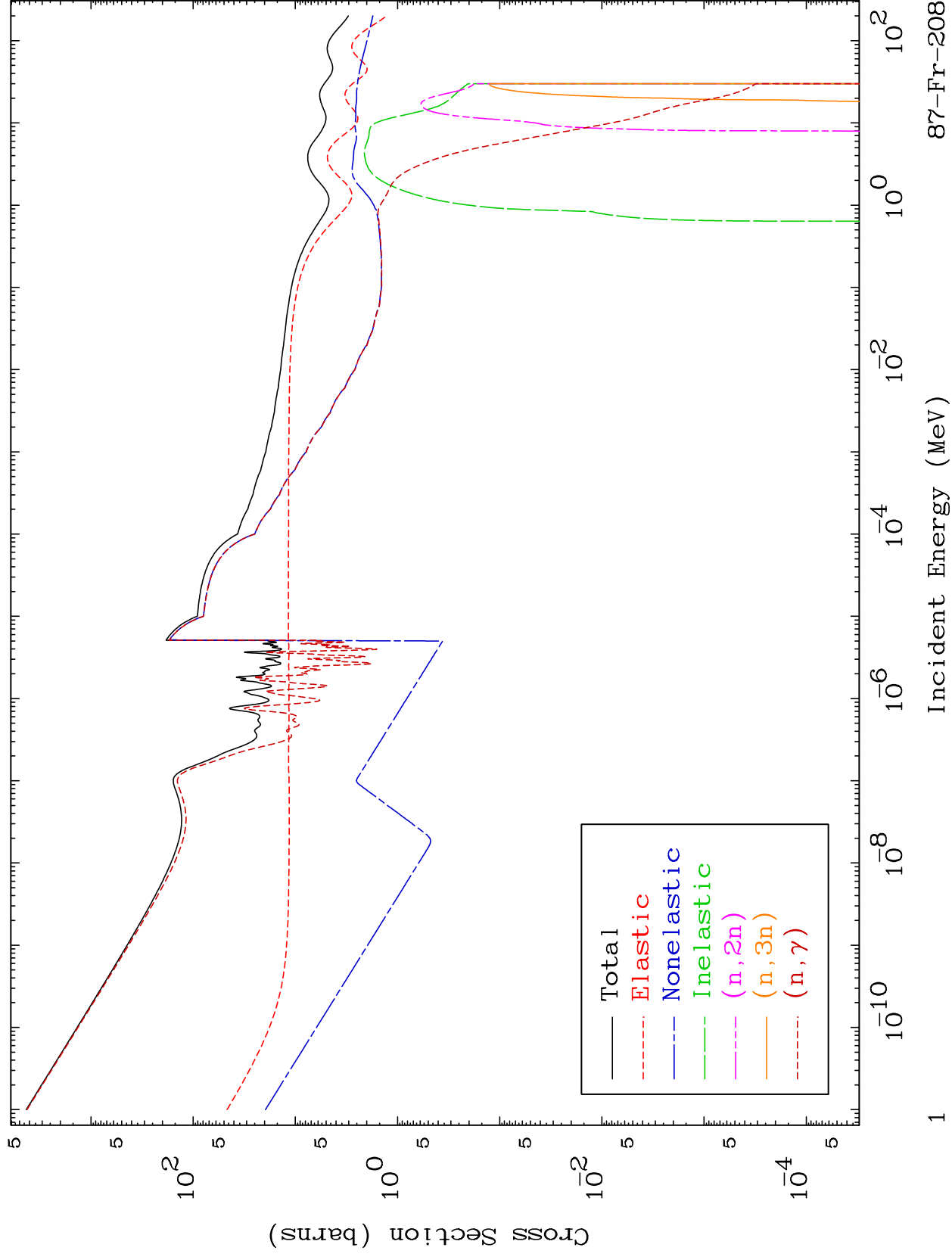
Dermott E. Cullen
(Present Contact Information)

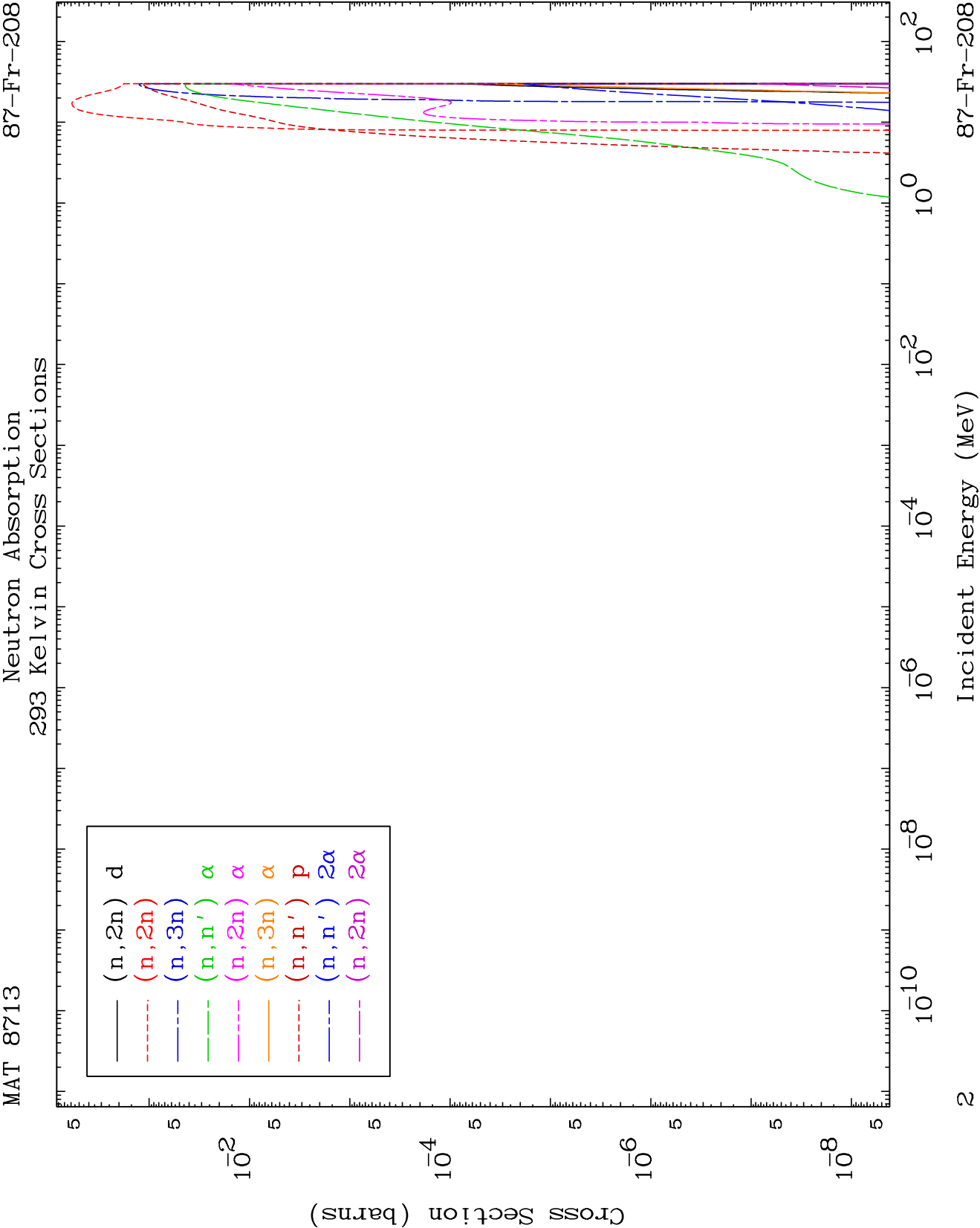
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

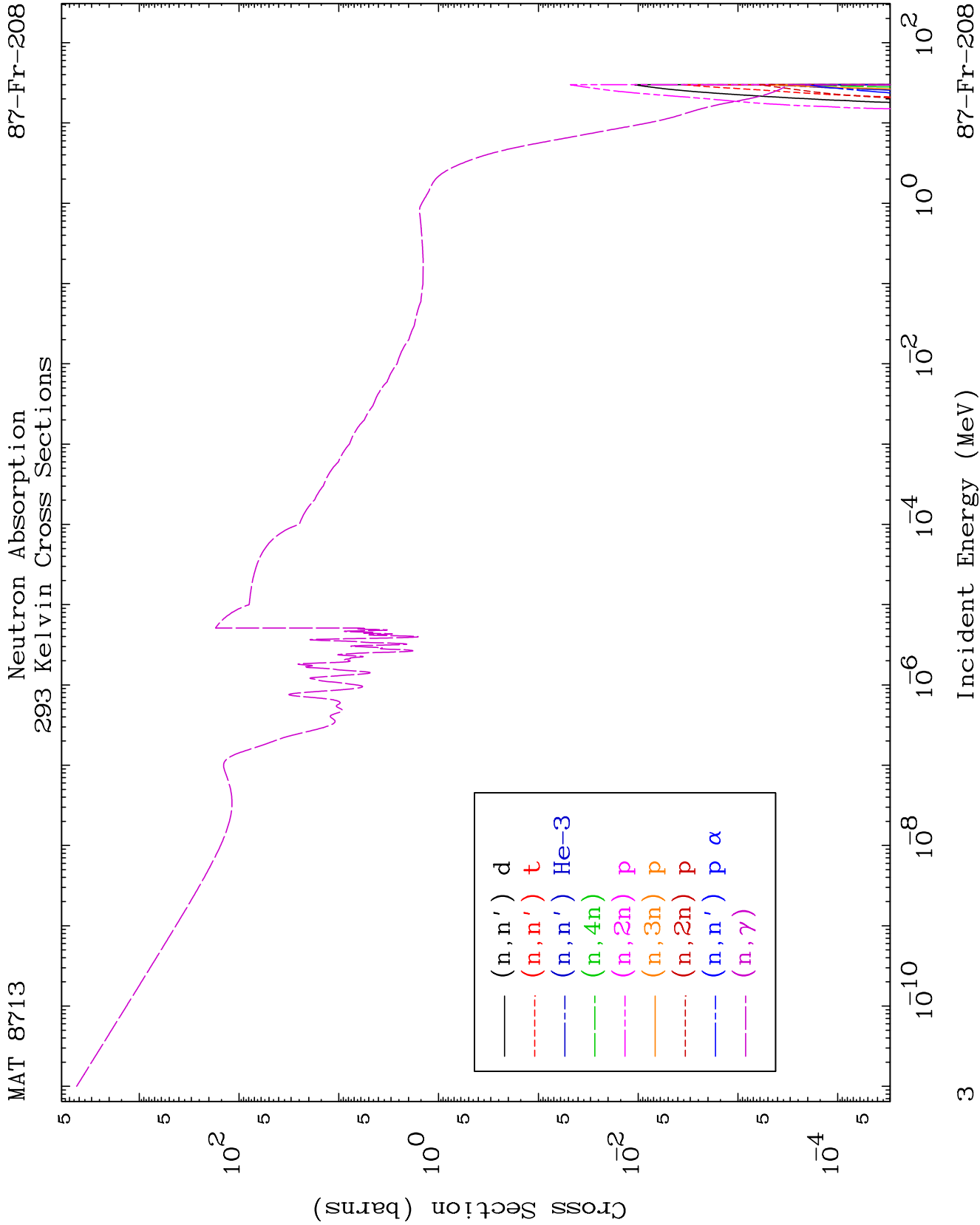
Tele: 925-443-1911

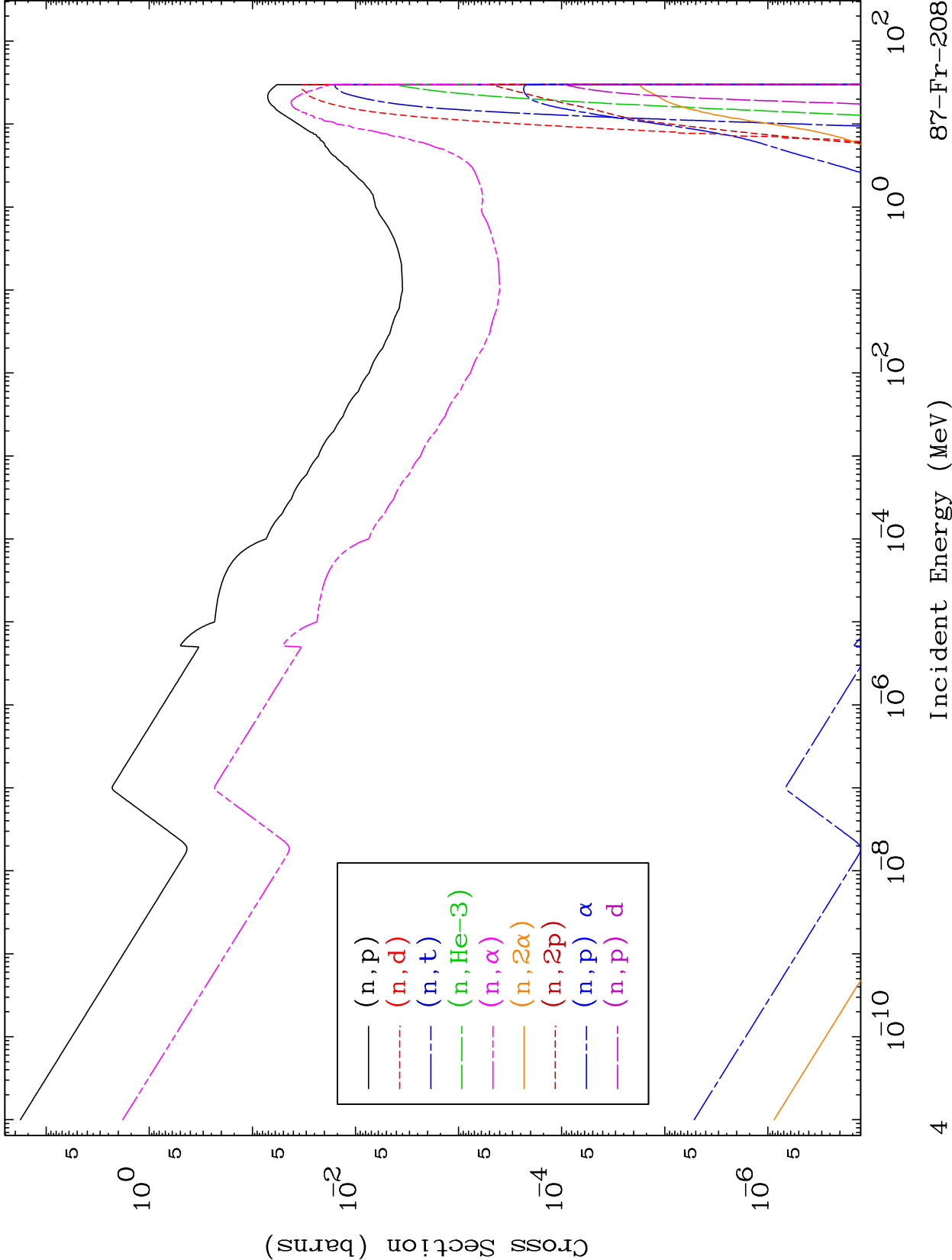
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

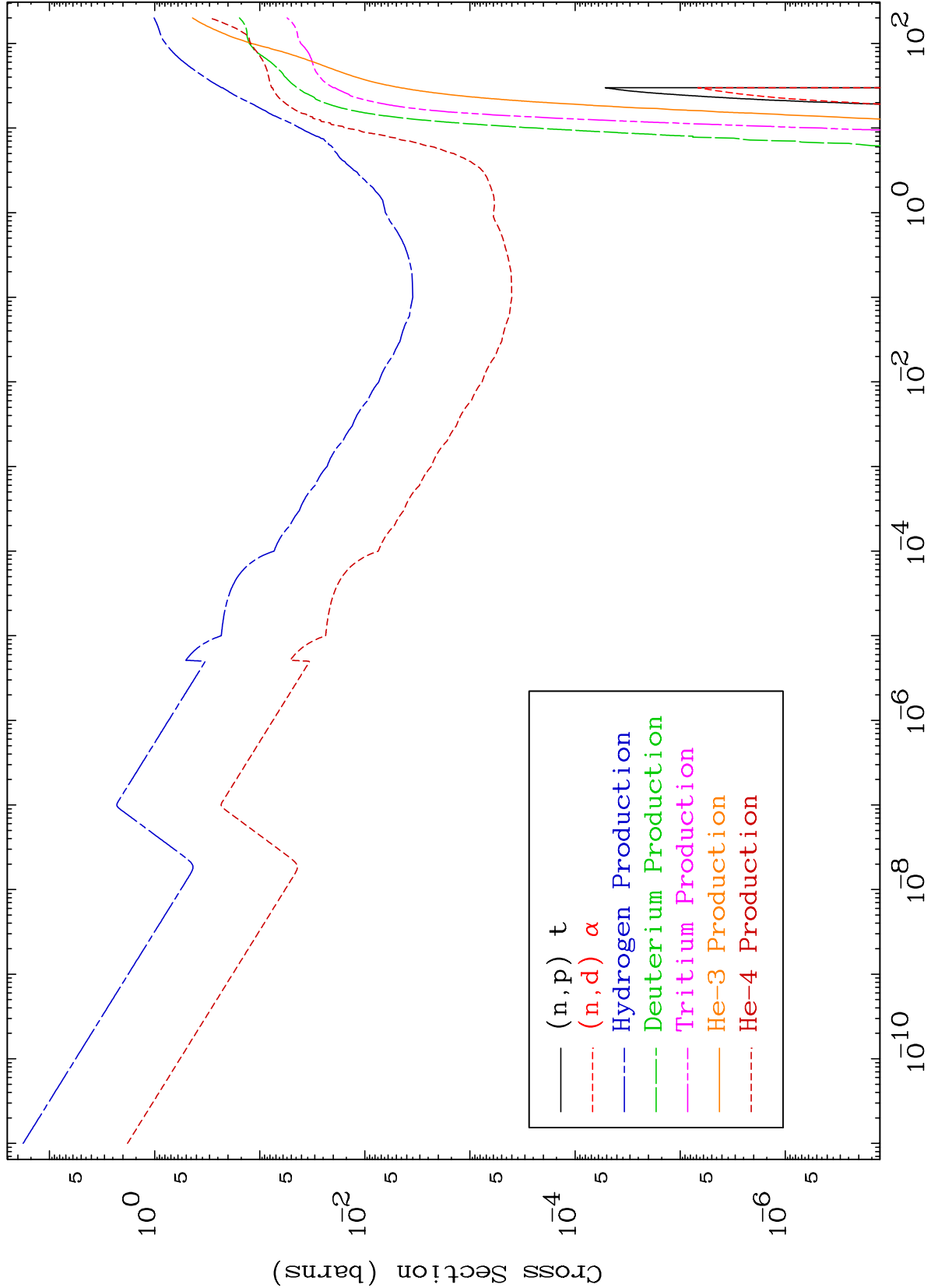
Press Mouse Button to Start

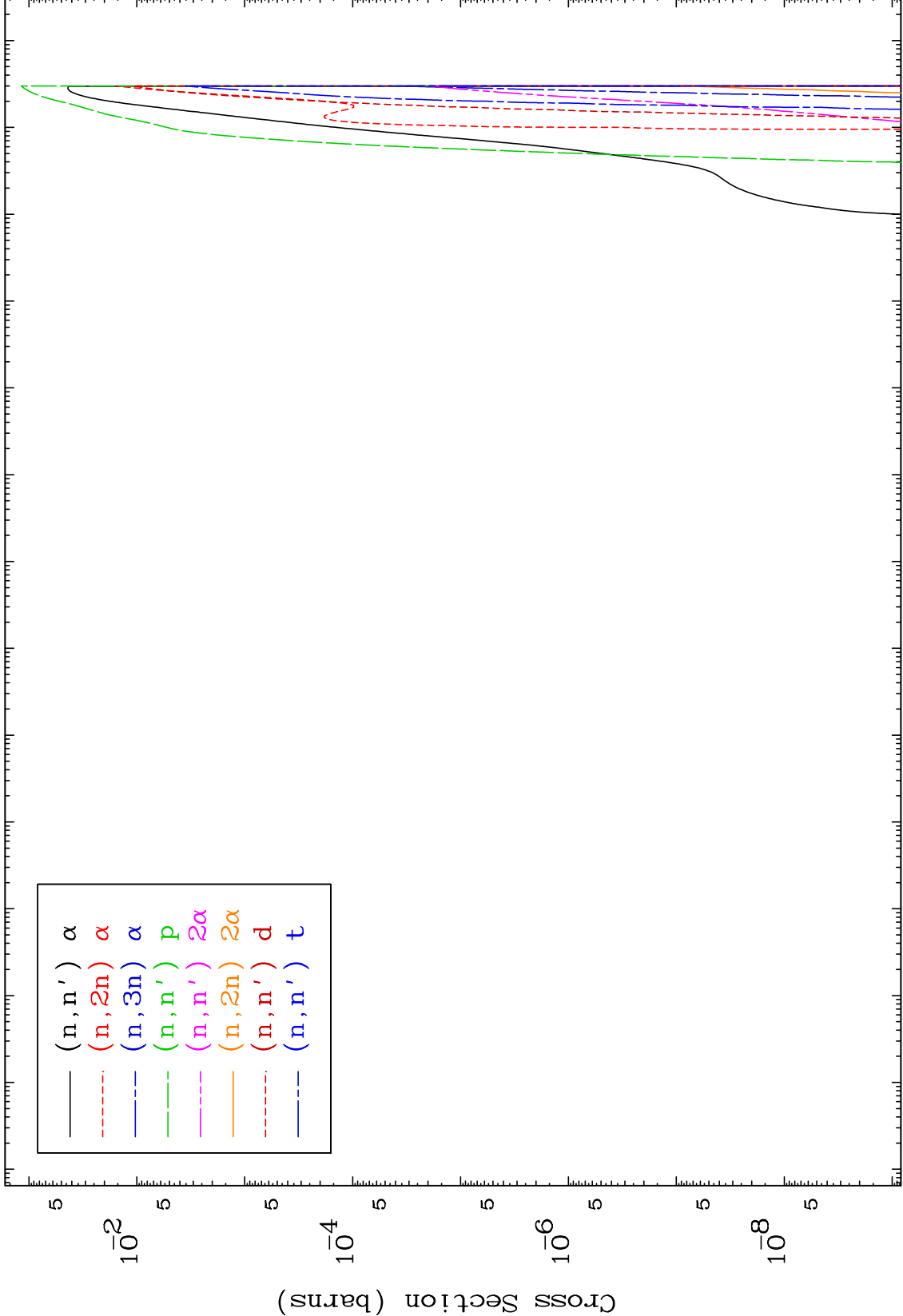


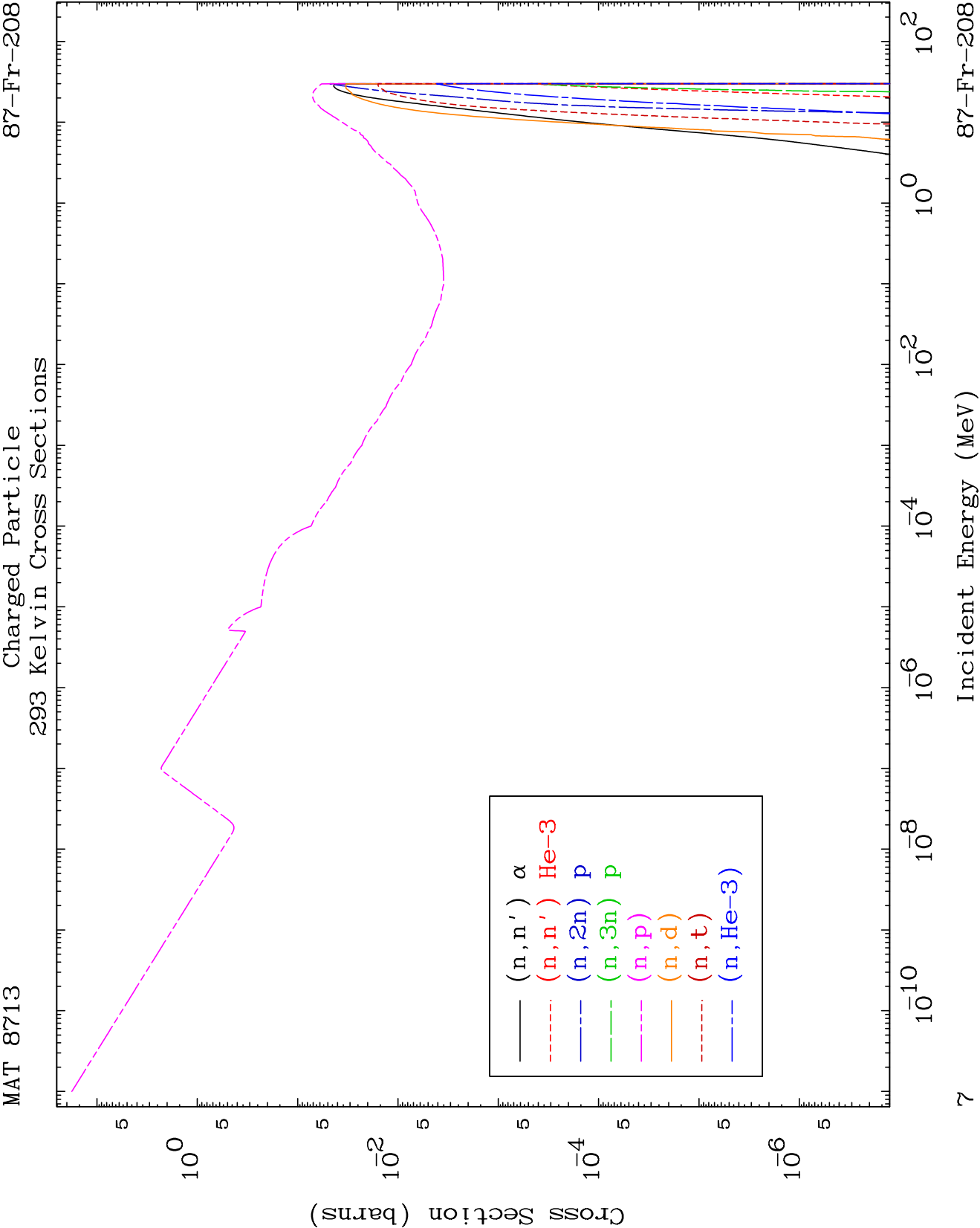


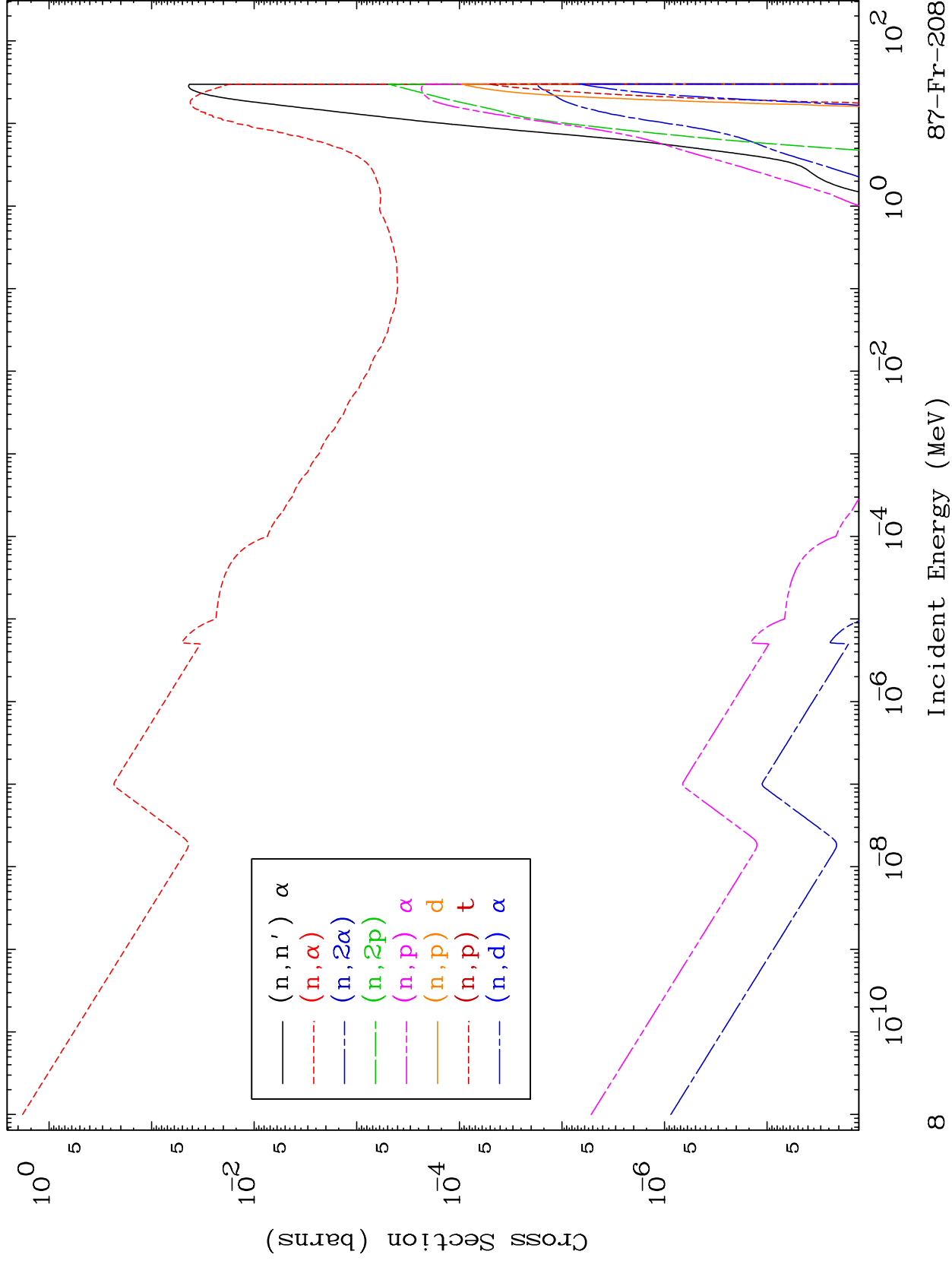


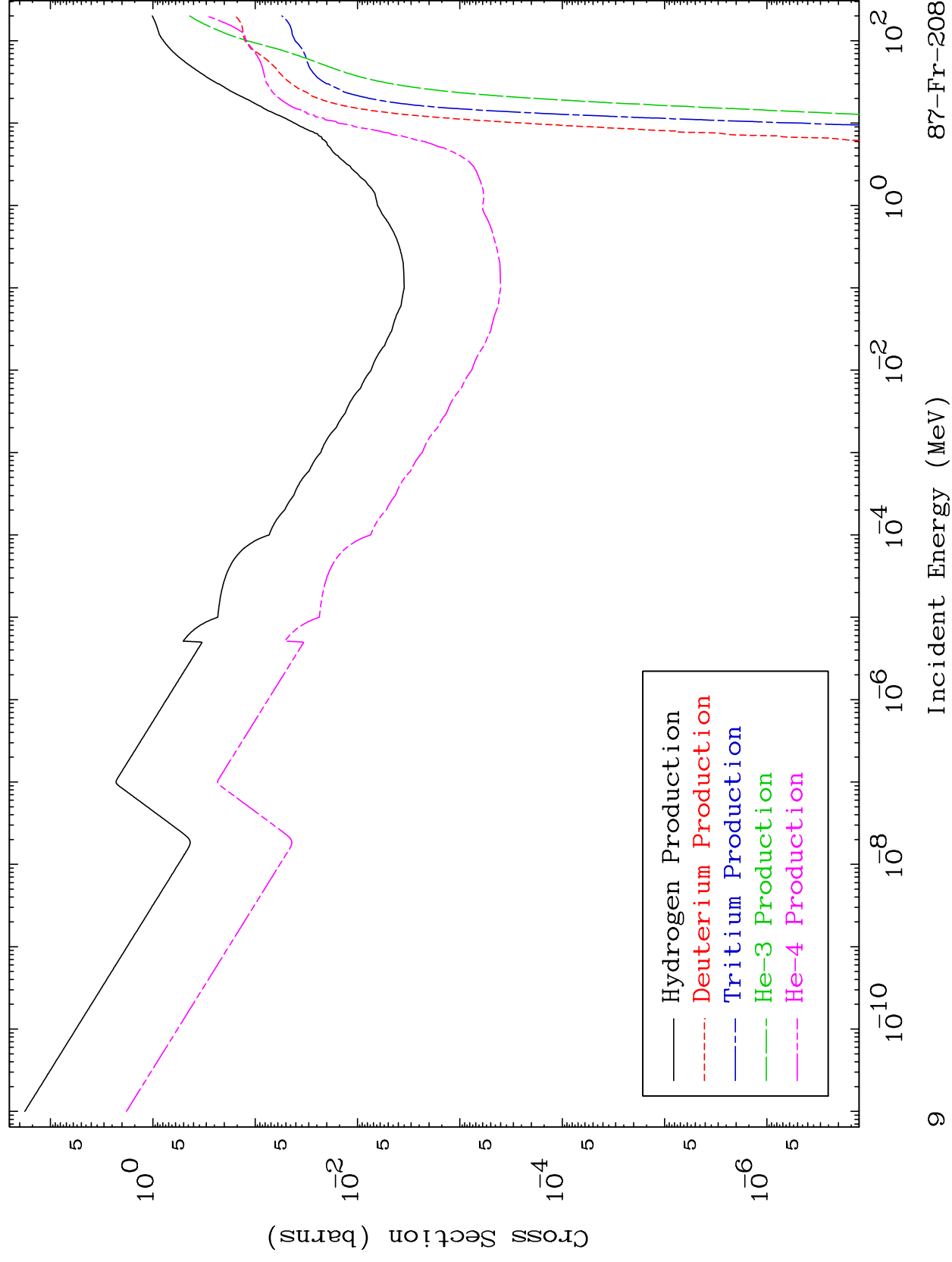


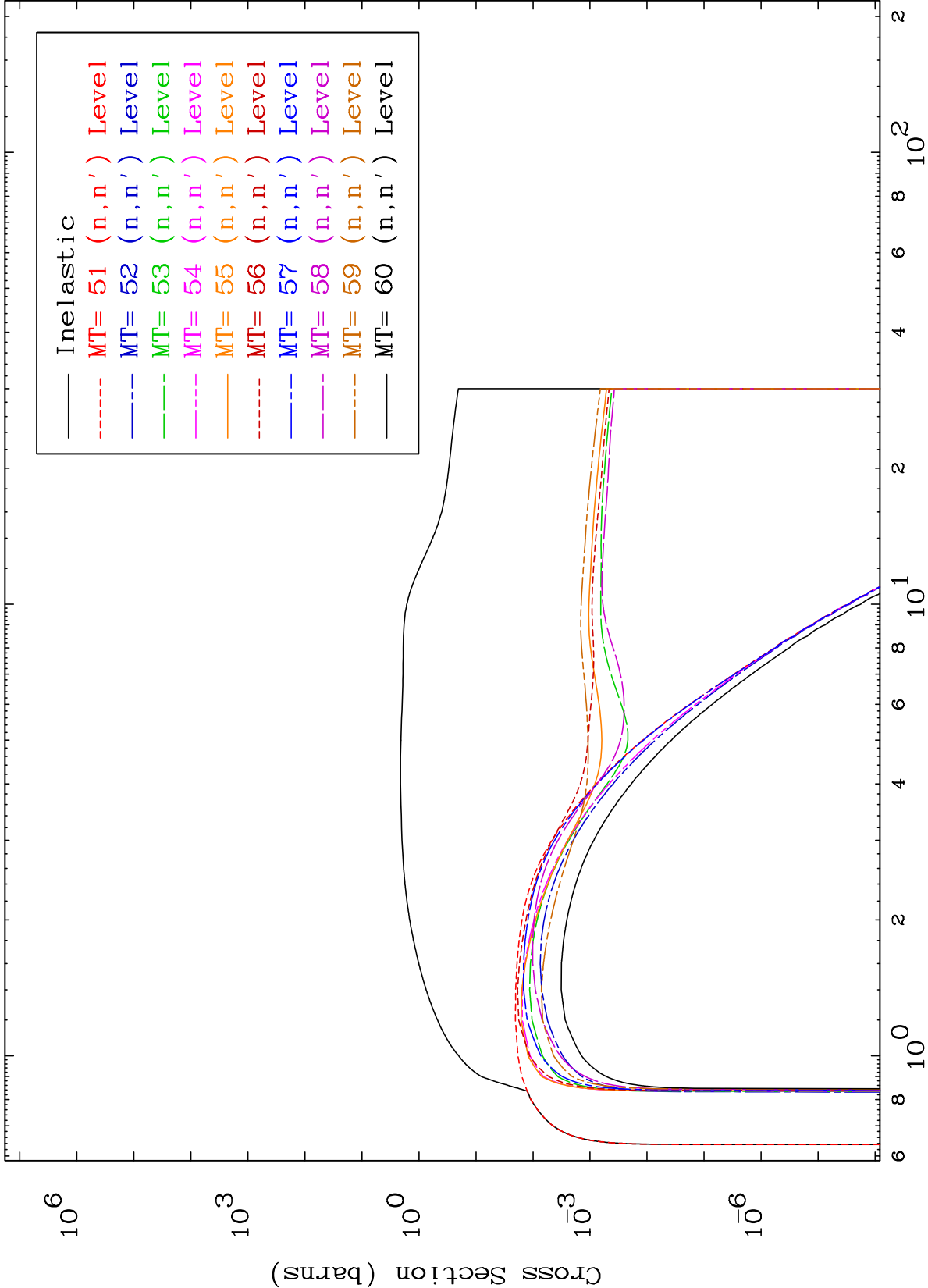


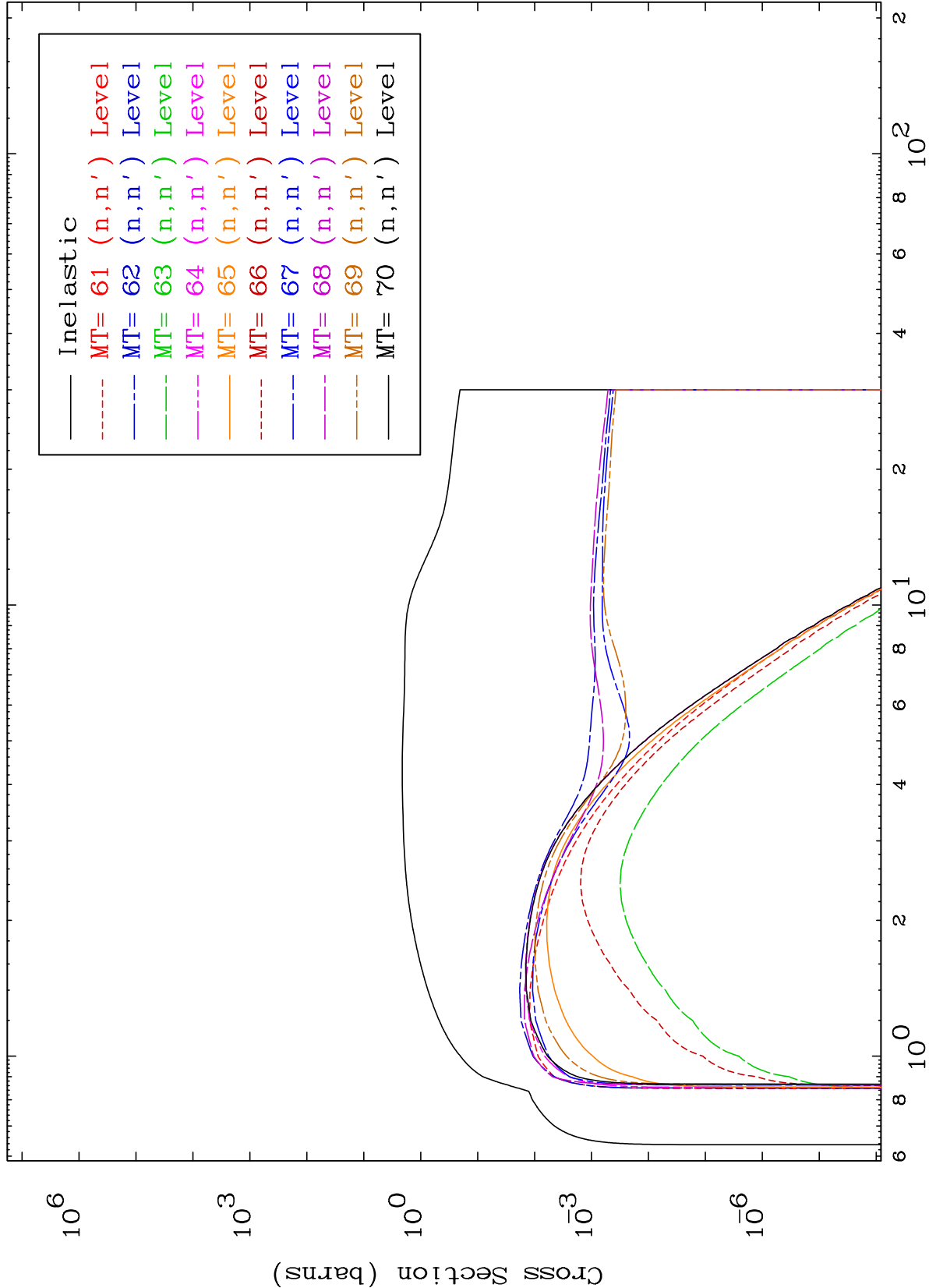


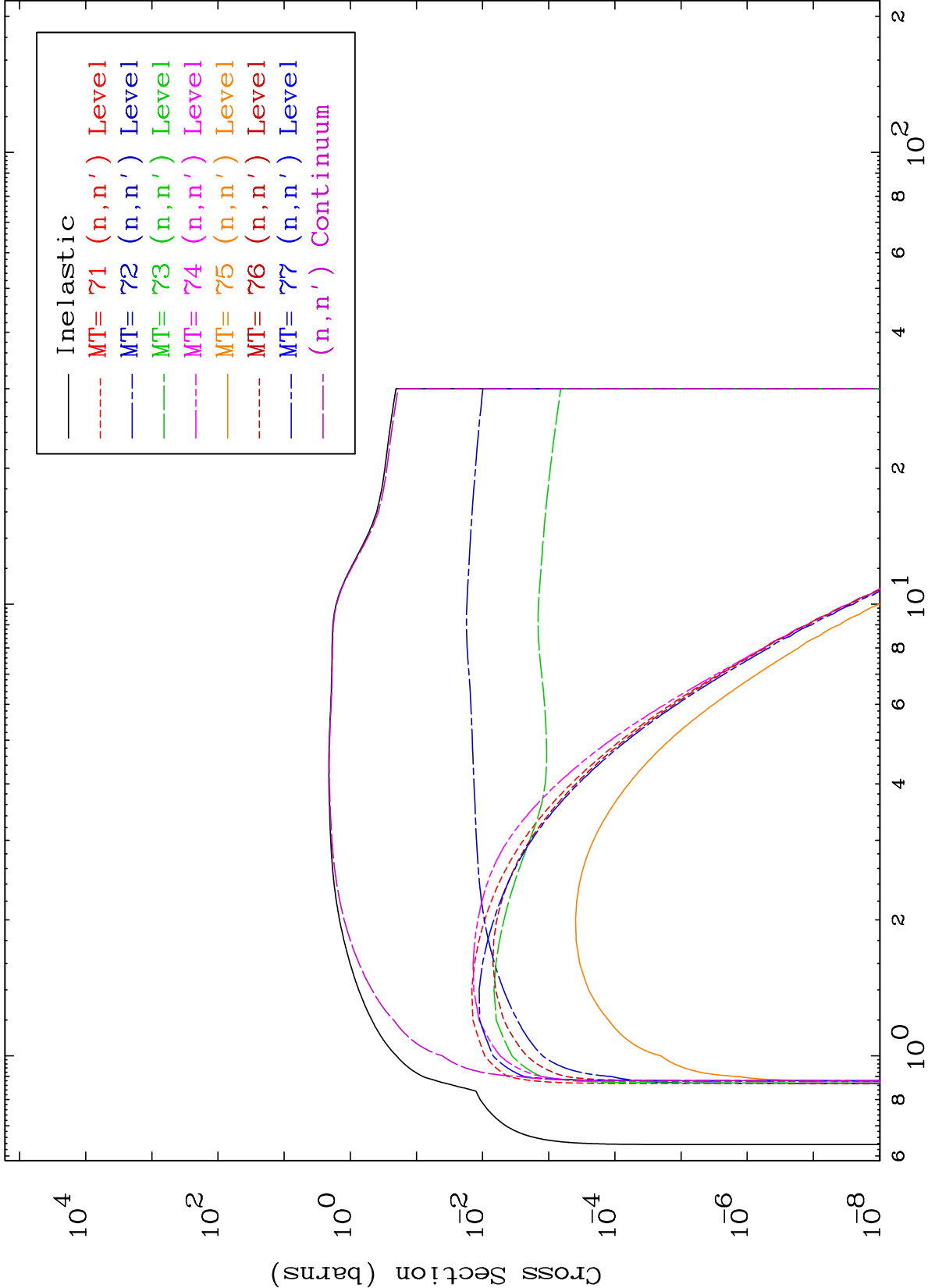








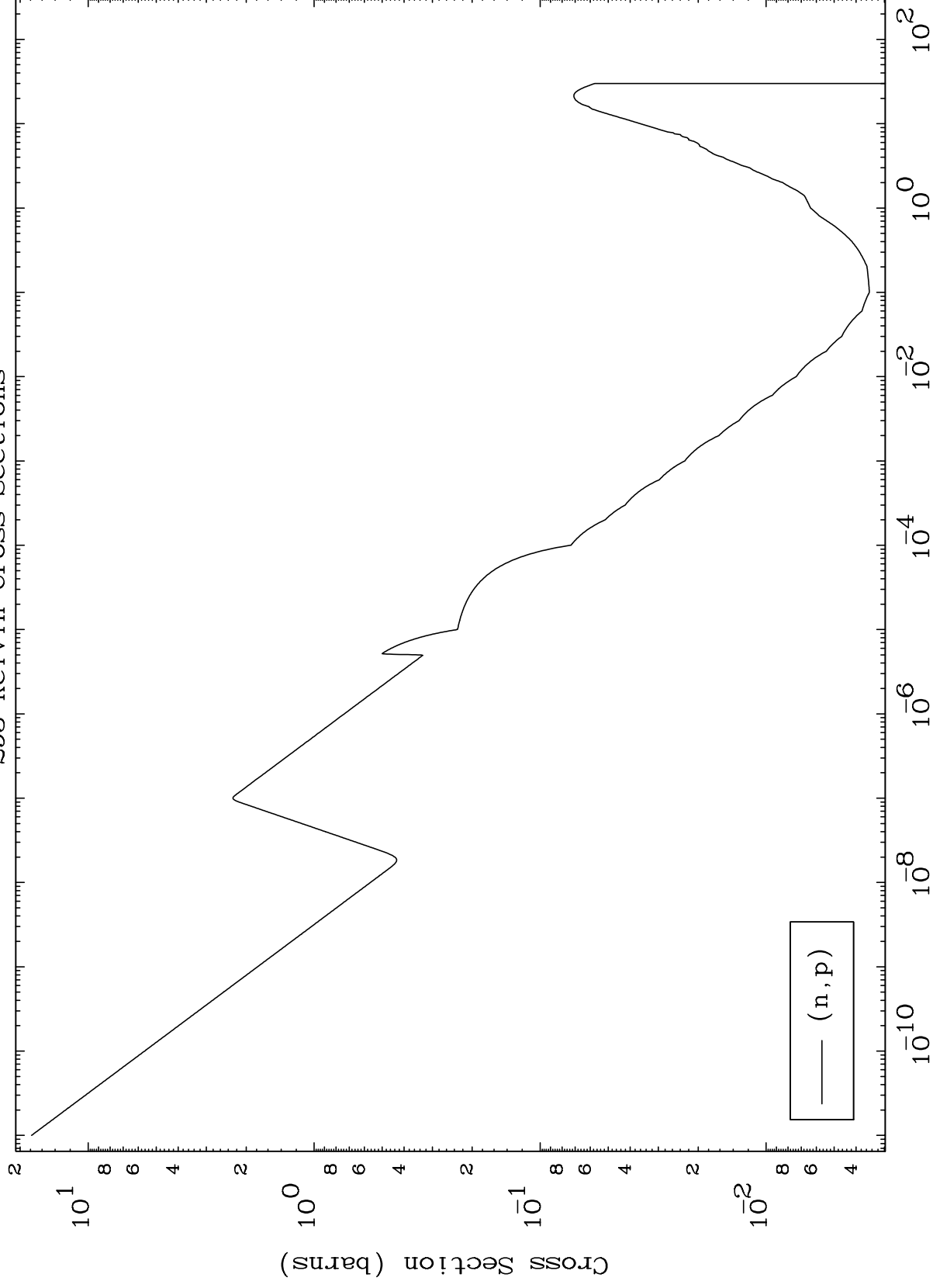




MAT 8713

87-Fr-208

(n,p) Levels
293 Kelvin Cross Sections



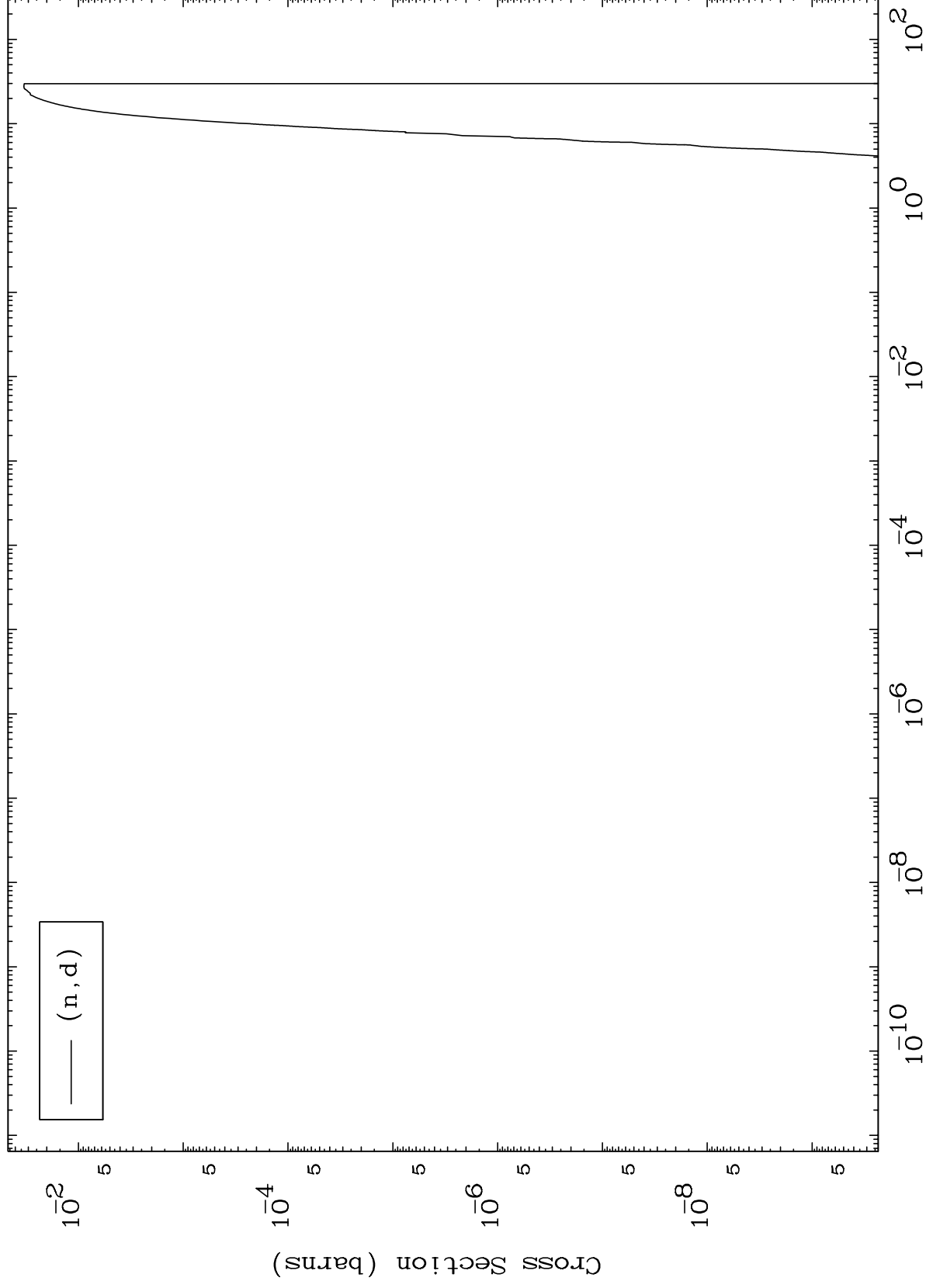
13

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(n,d) Levels
293 Kelvin Cross Sections

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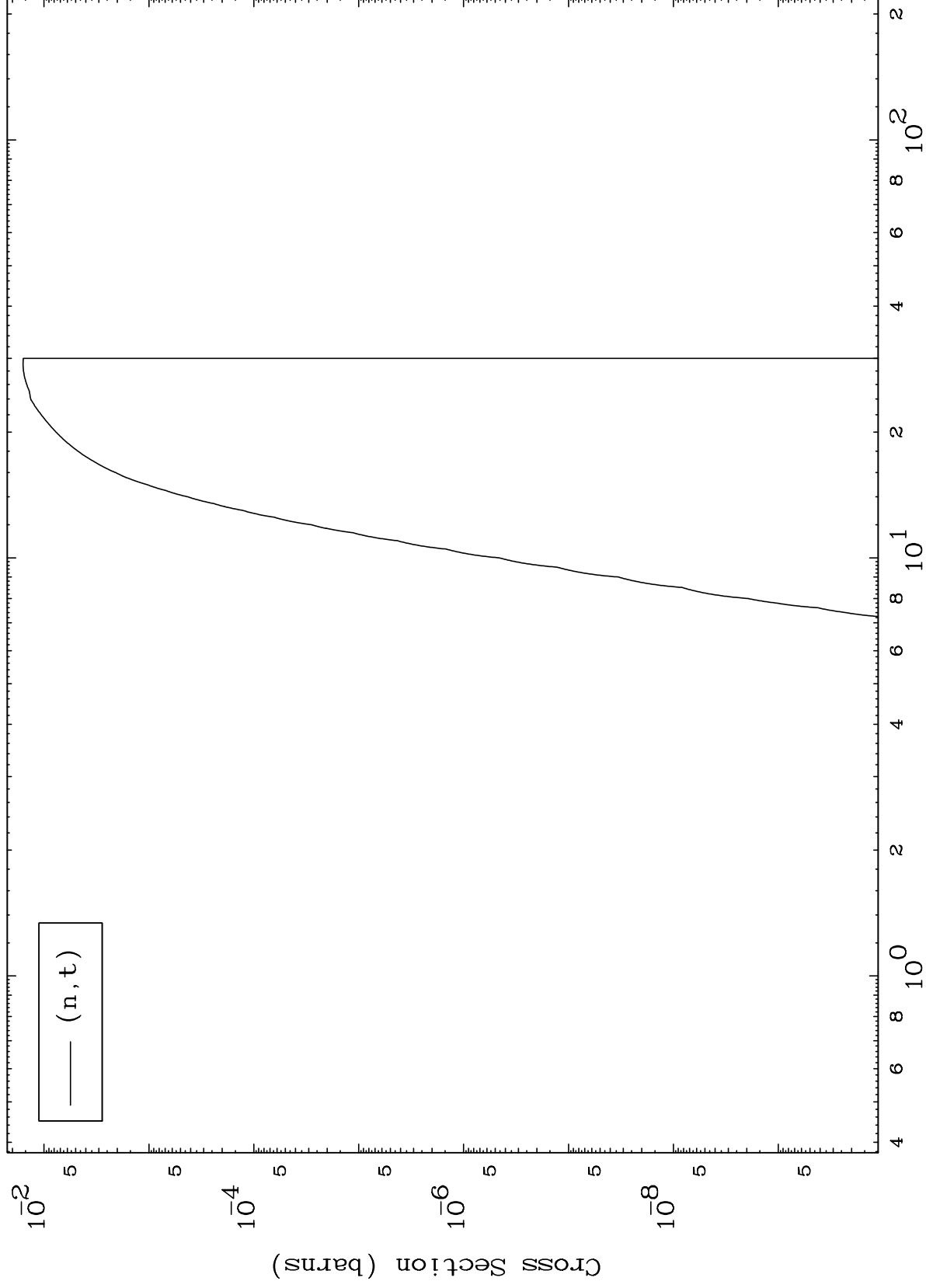
Incident Energy (MeV)

87-Fr-208

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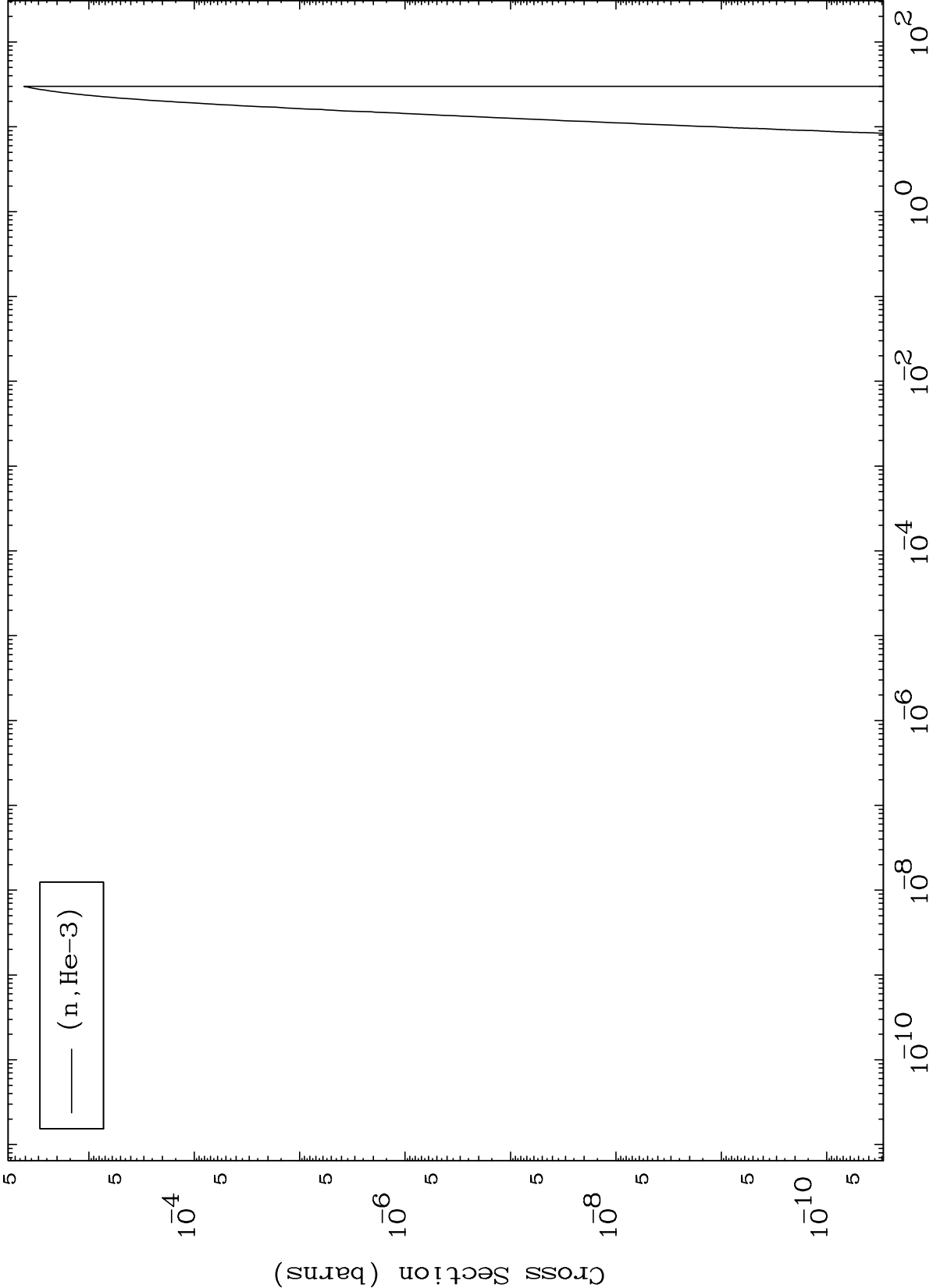
(n,t) Levels
293 Kelvin Cross Sections



Incident Energy (MeV)

87-Fr-208

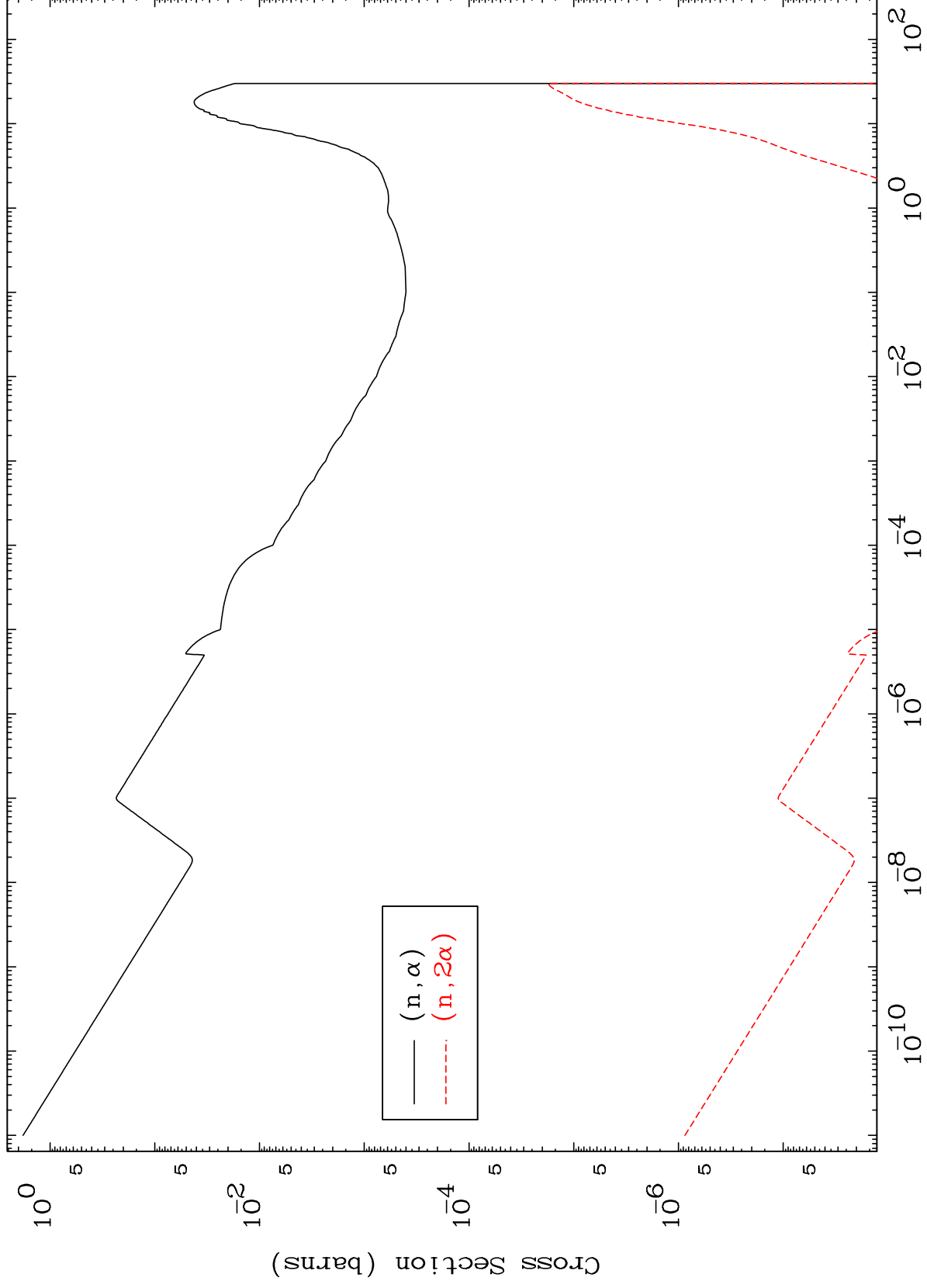
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MAT 8713

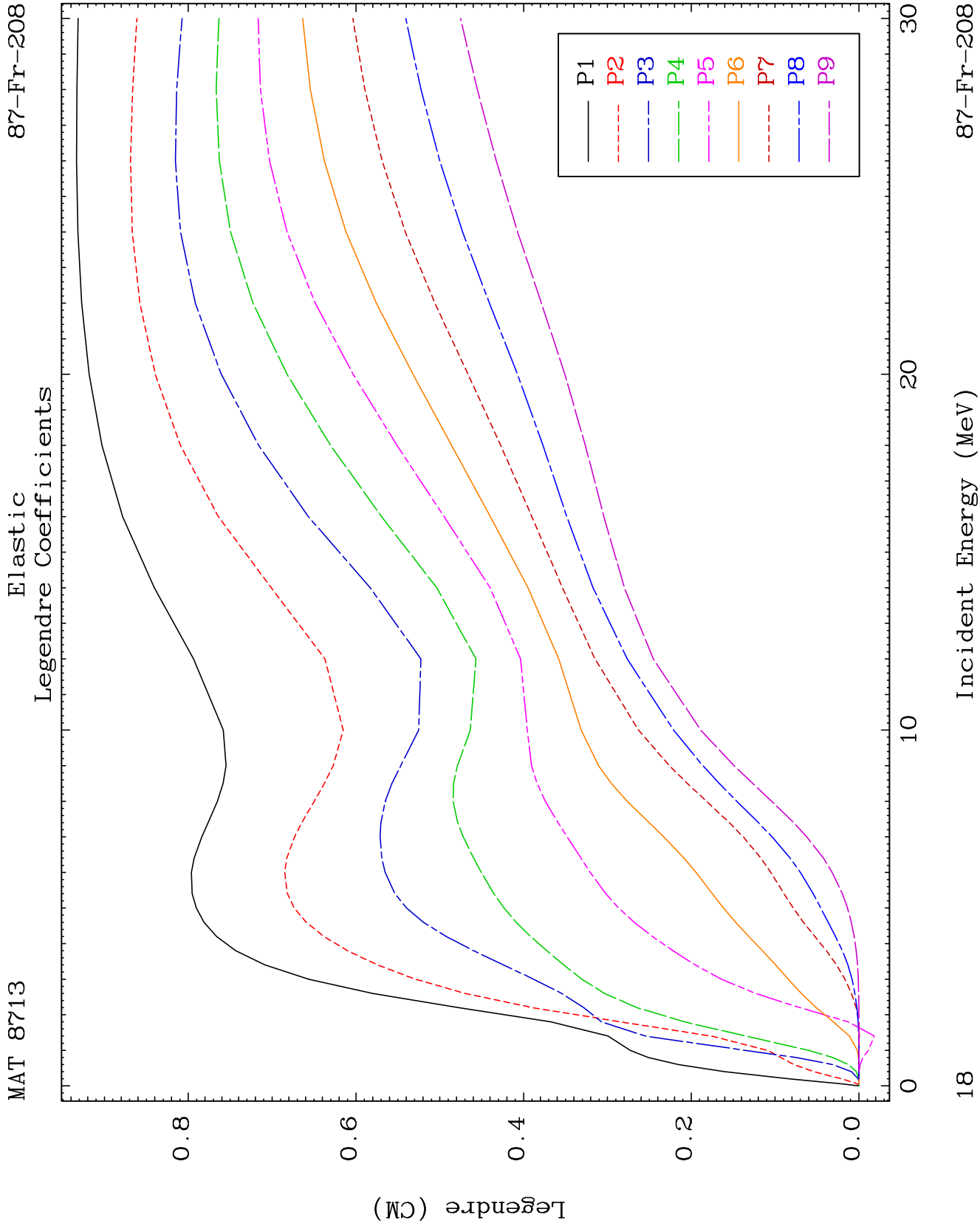
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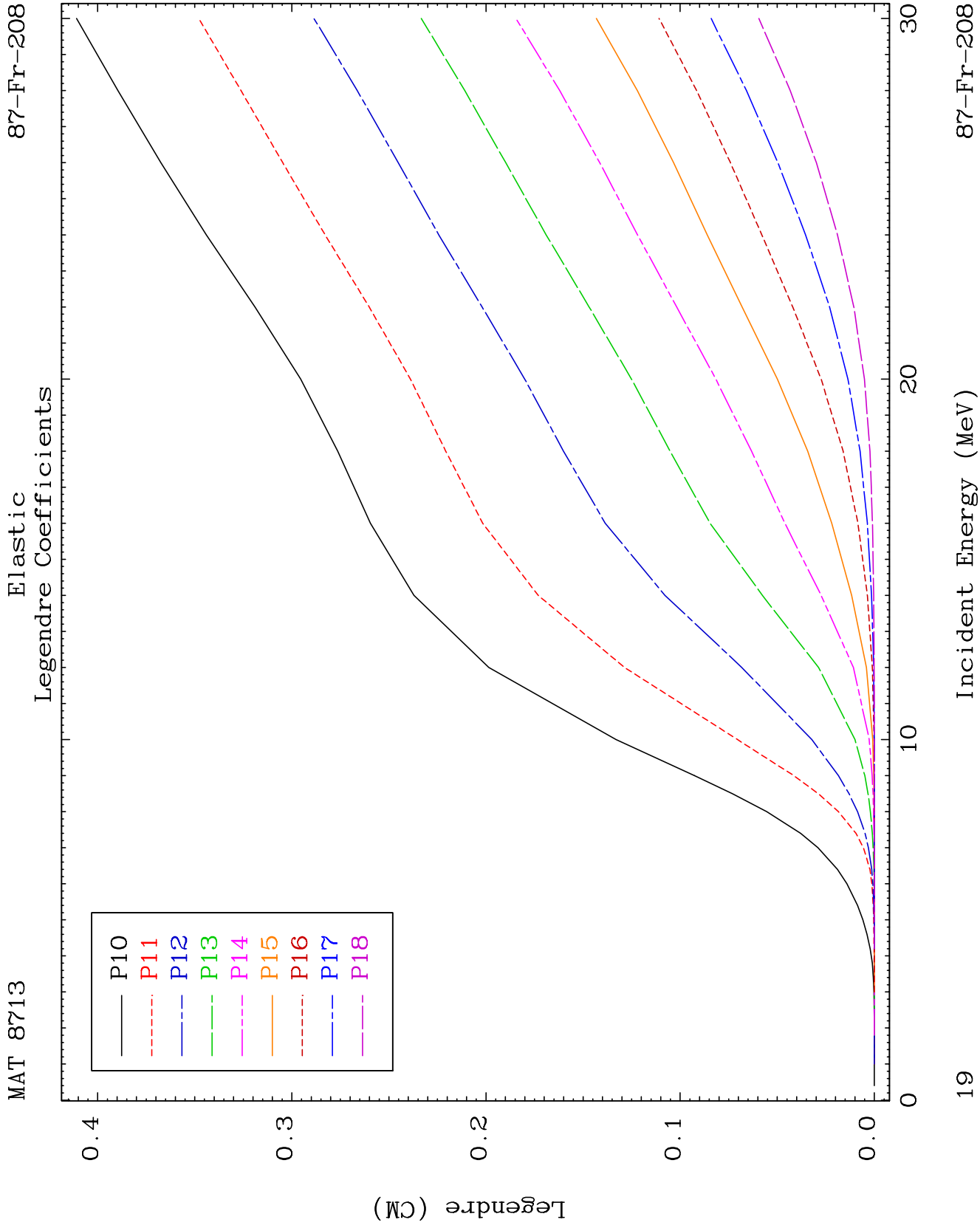
(n, α) Levels
293 Kelvin Cross Sections

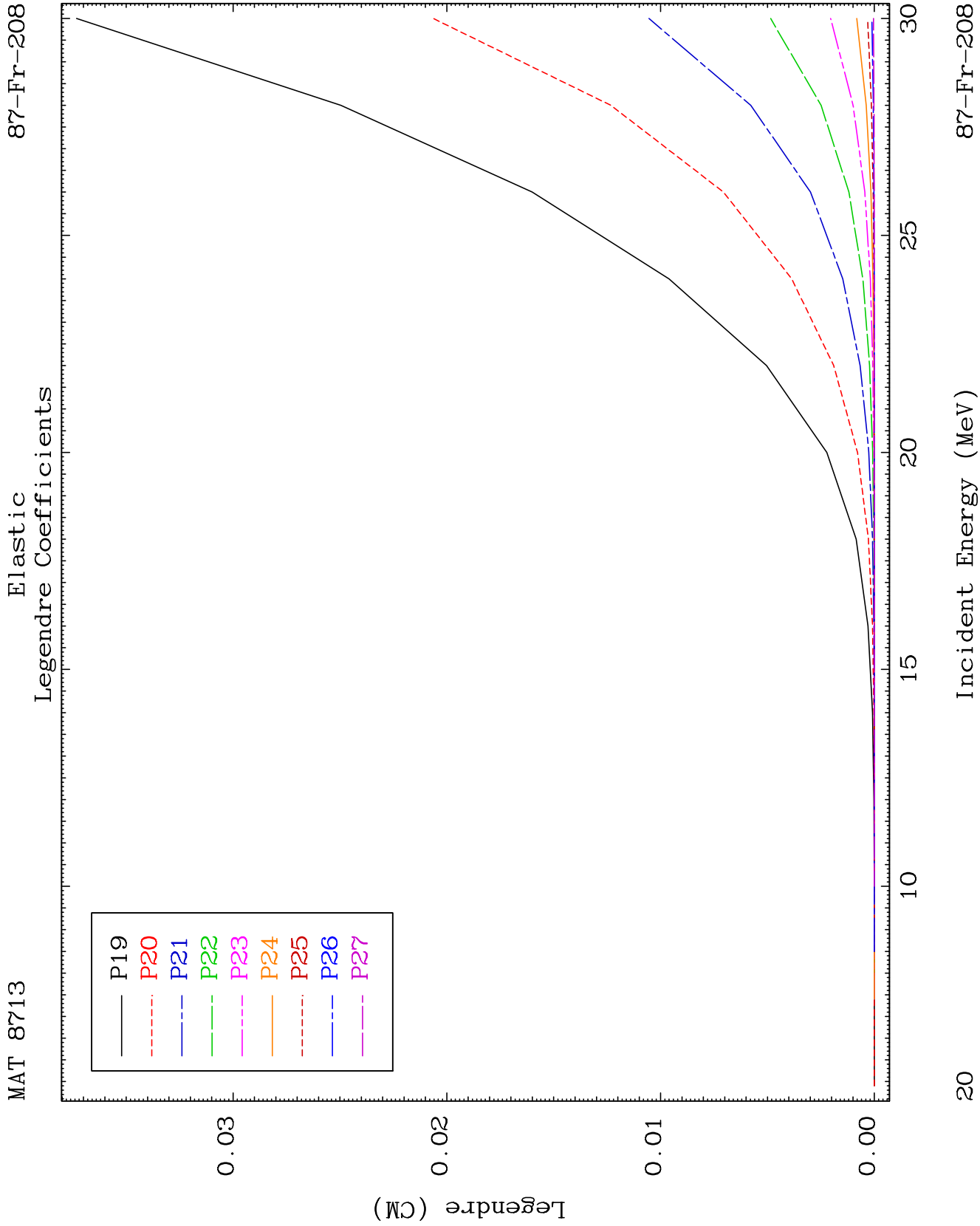


17

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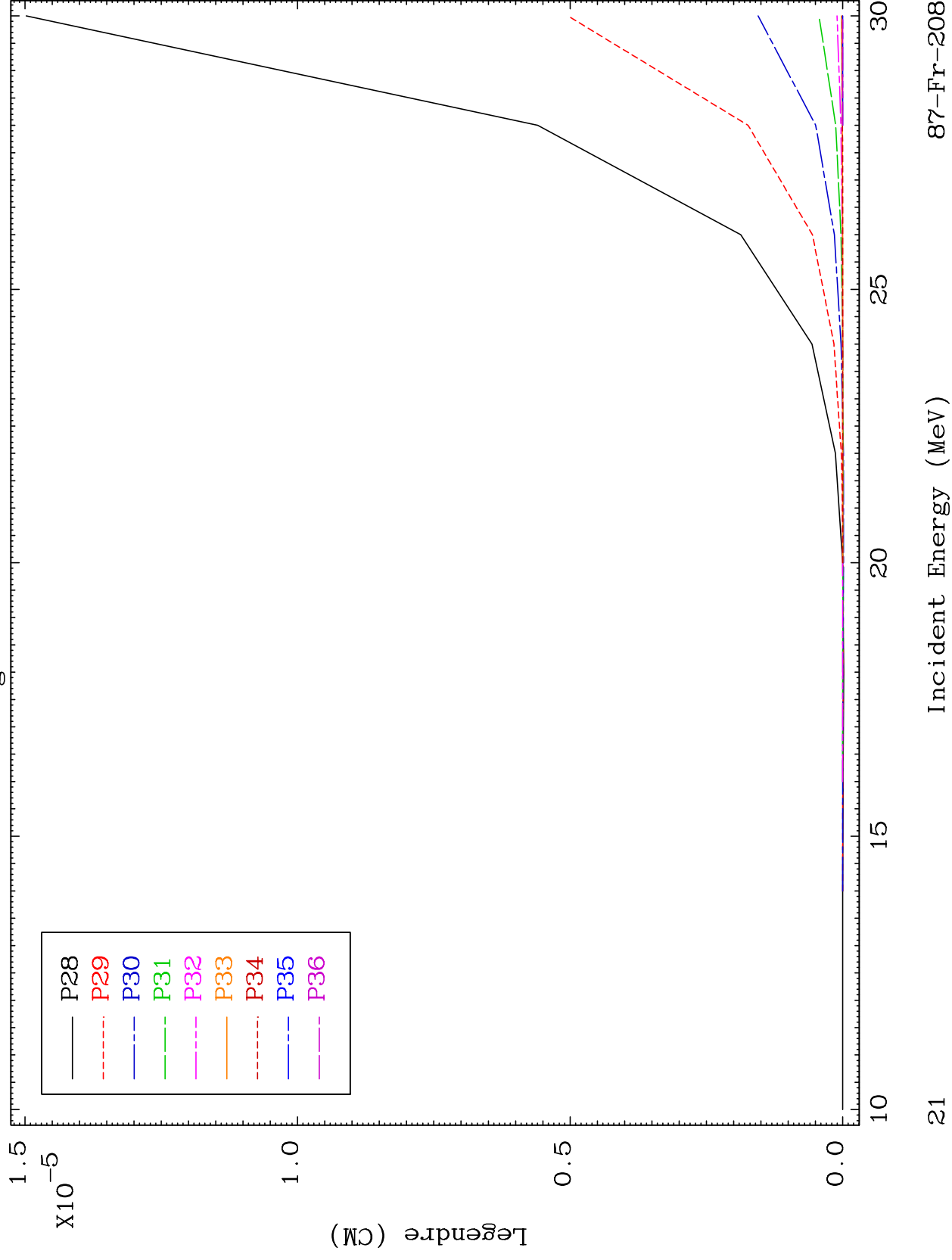




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Elastic Legendre Coefficients

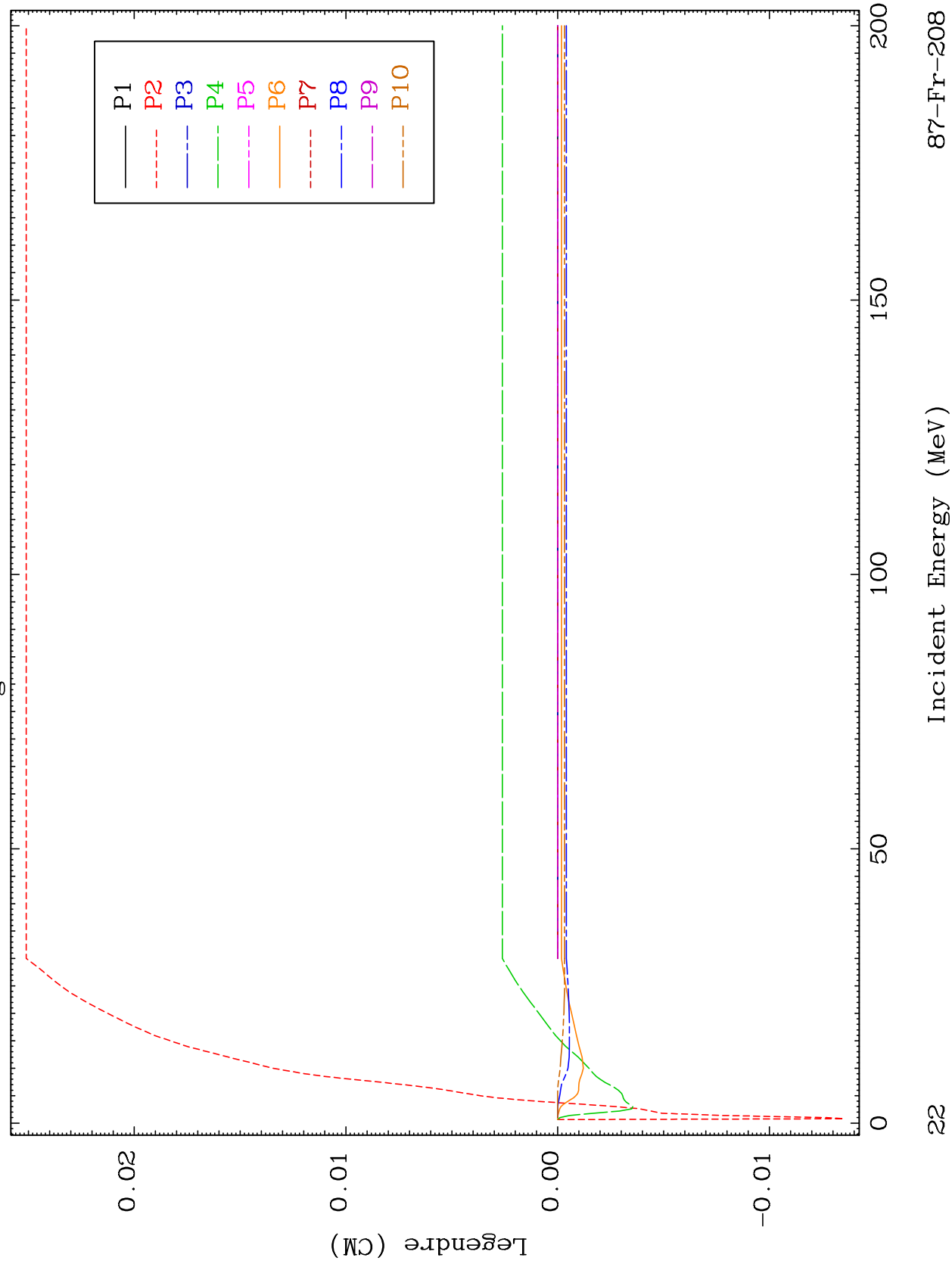
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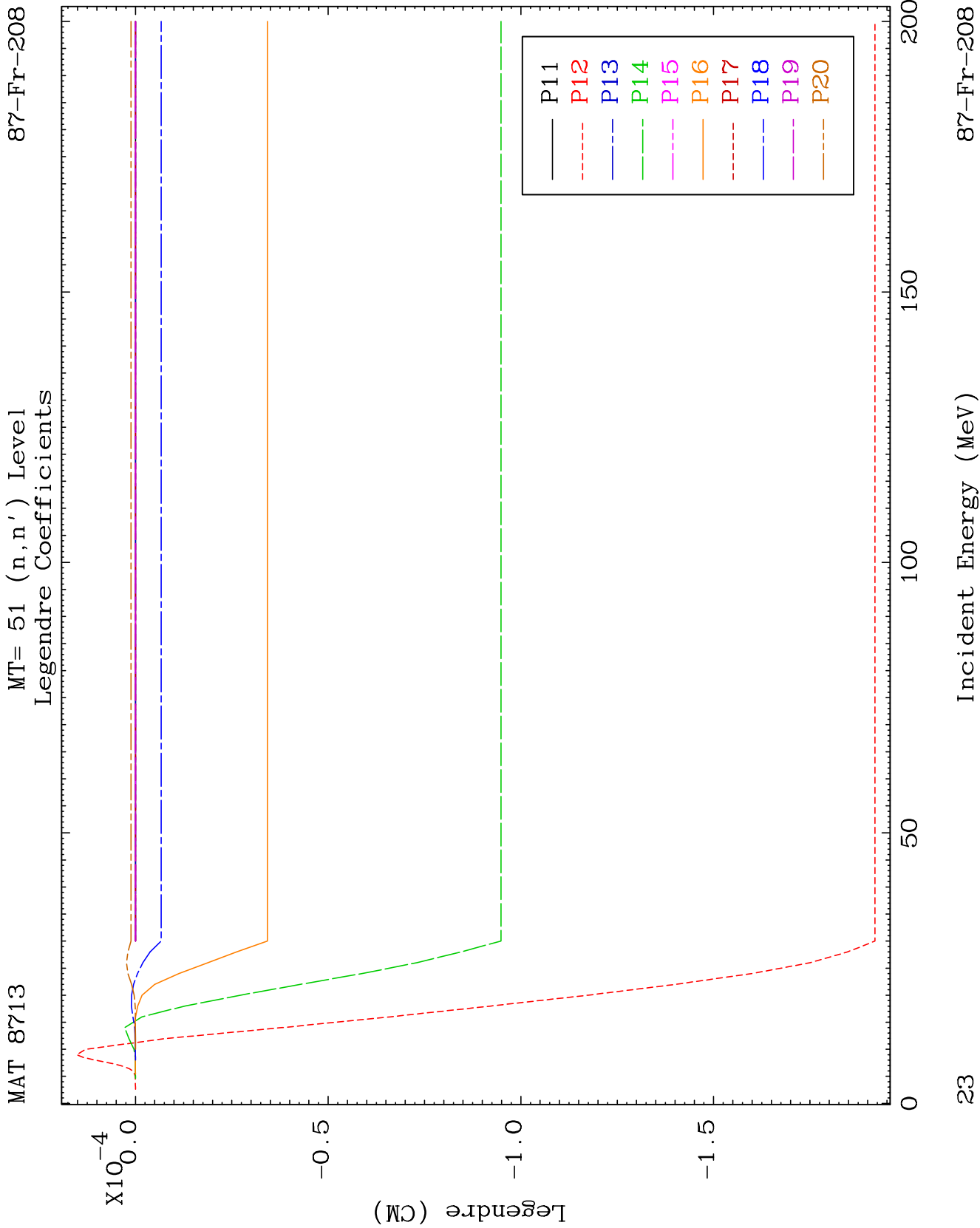


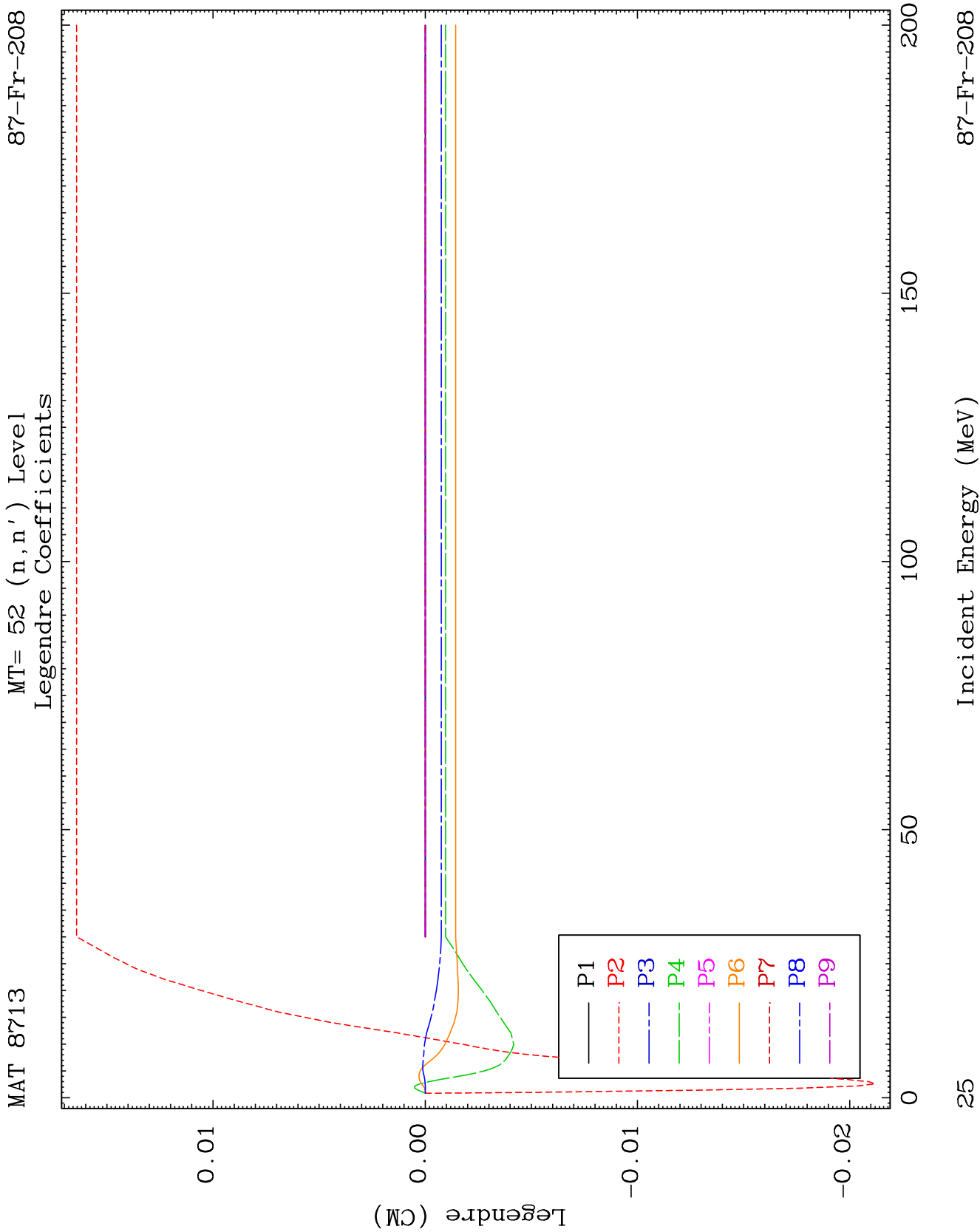
MAT 8713

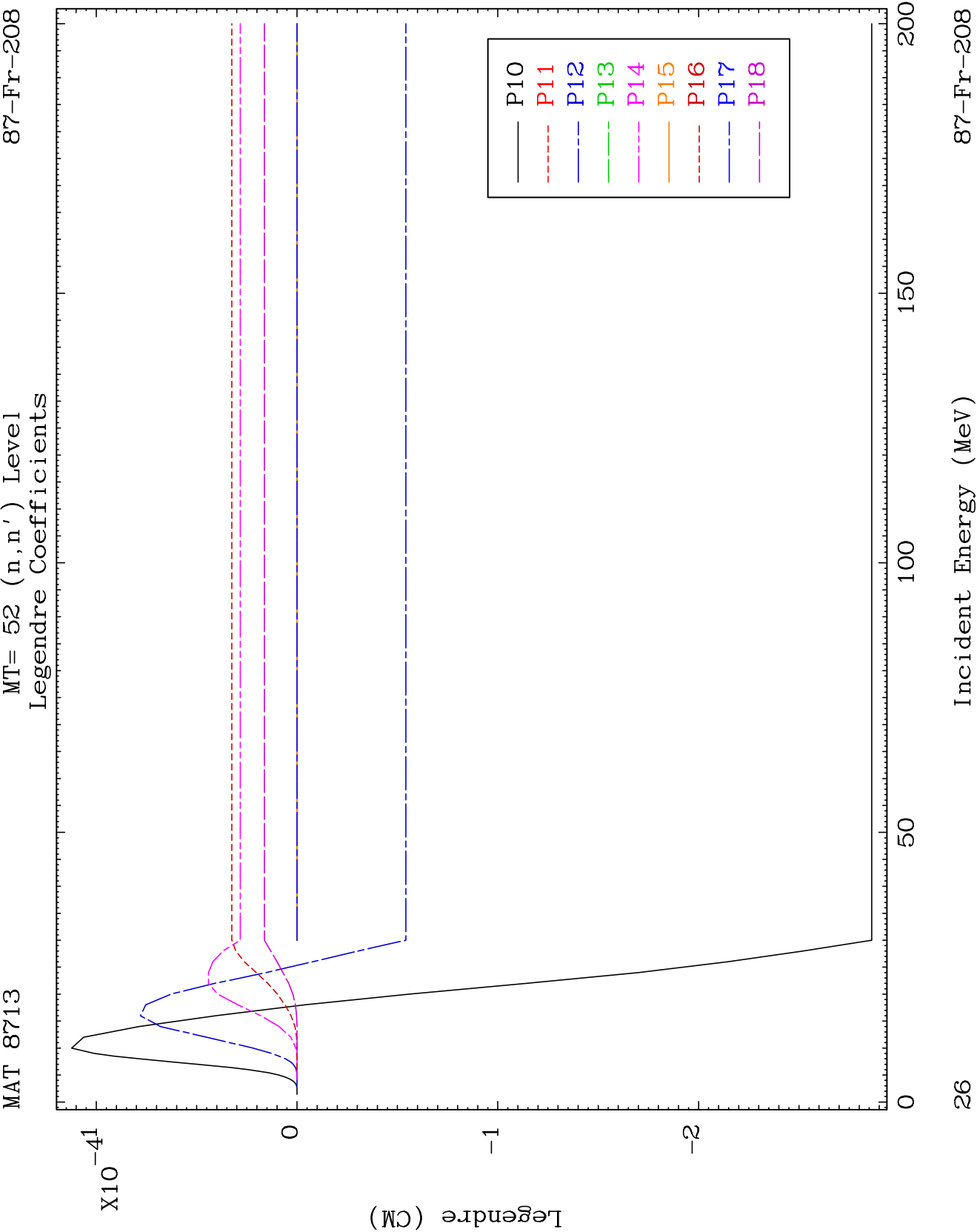
MT= 51 (n, n') Level
Legendre Coefficients

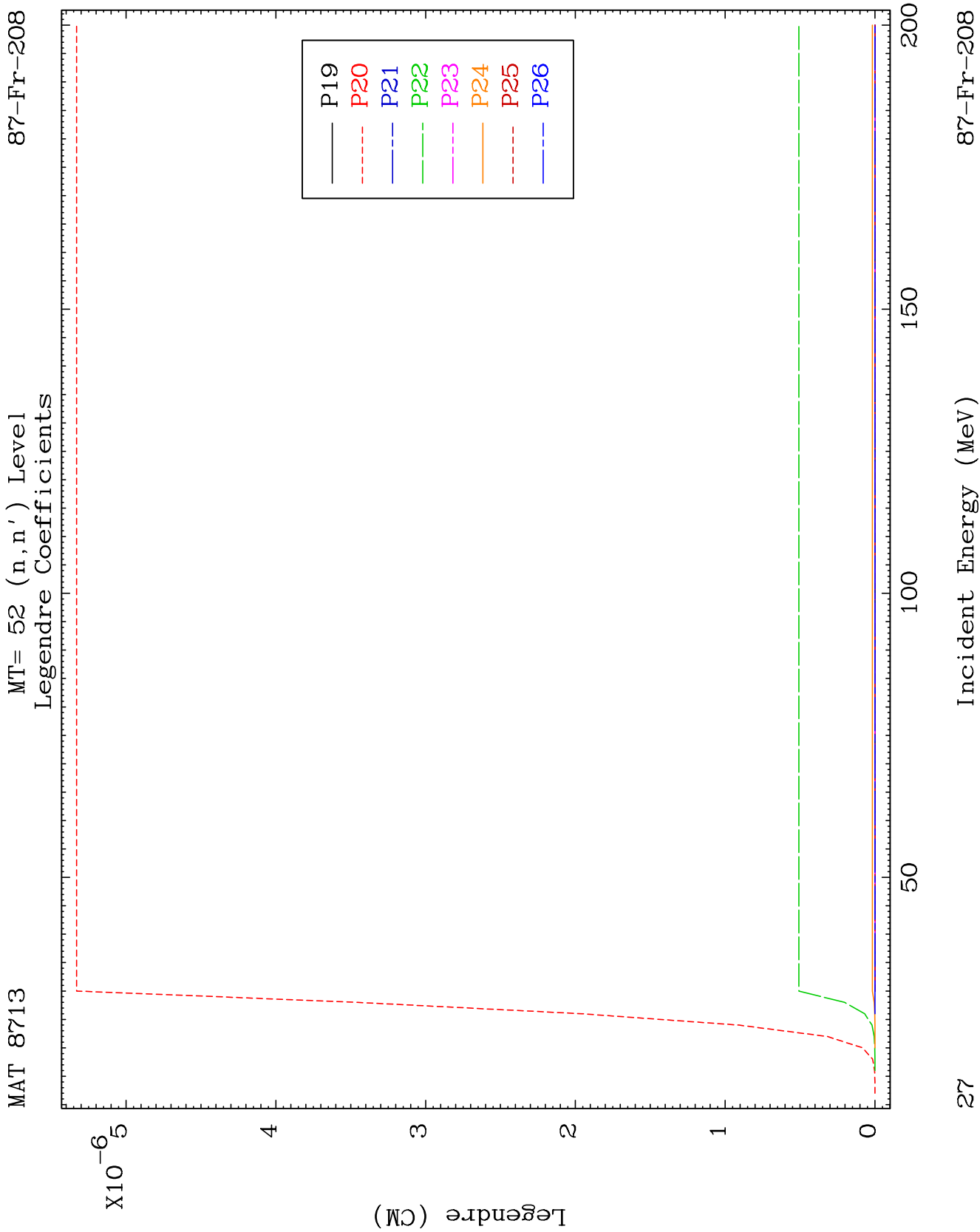
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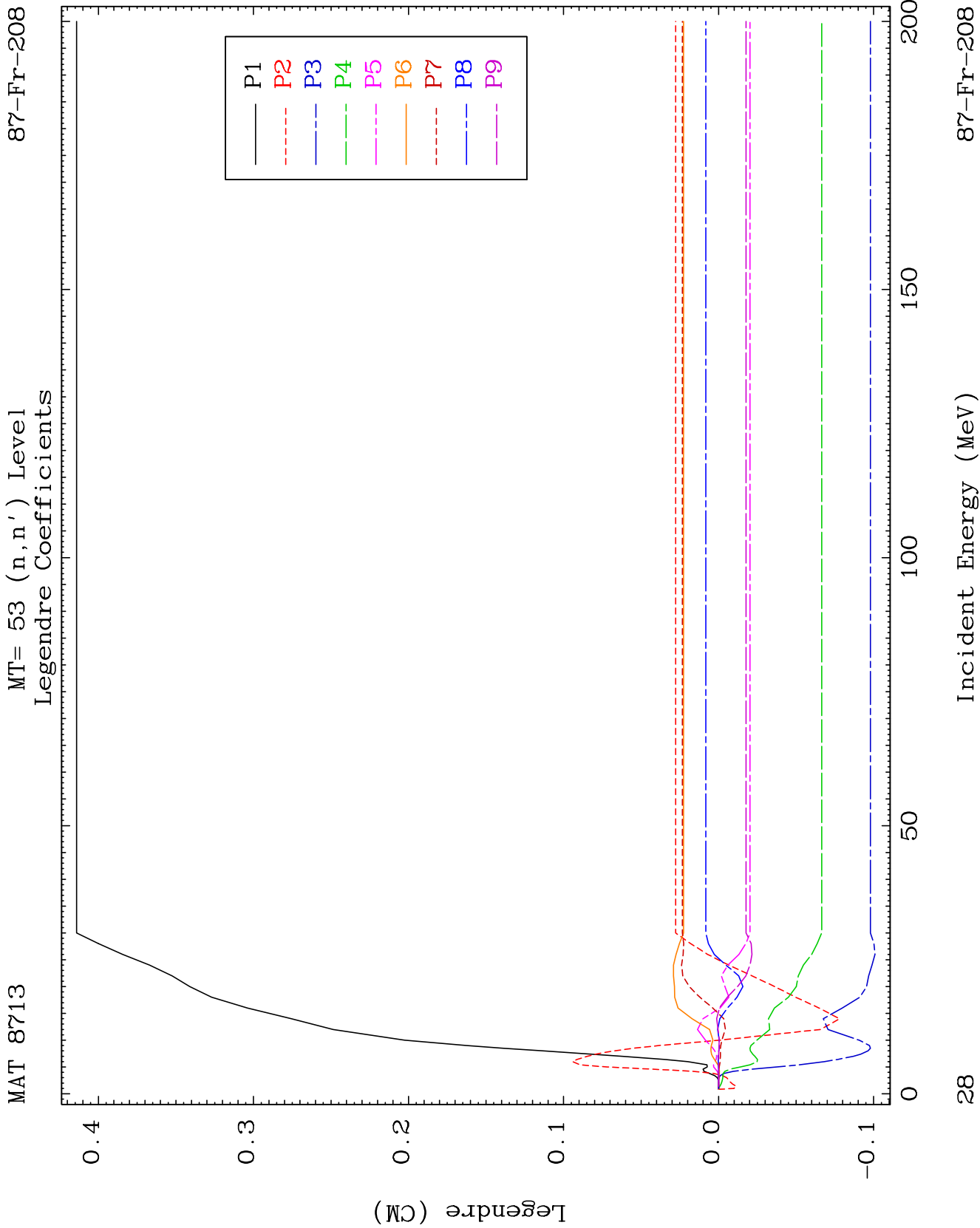


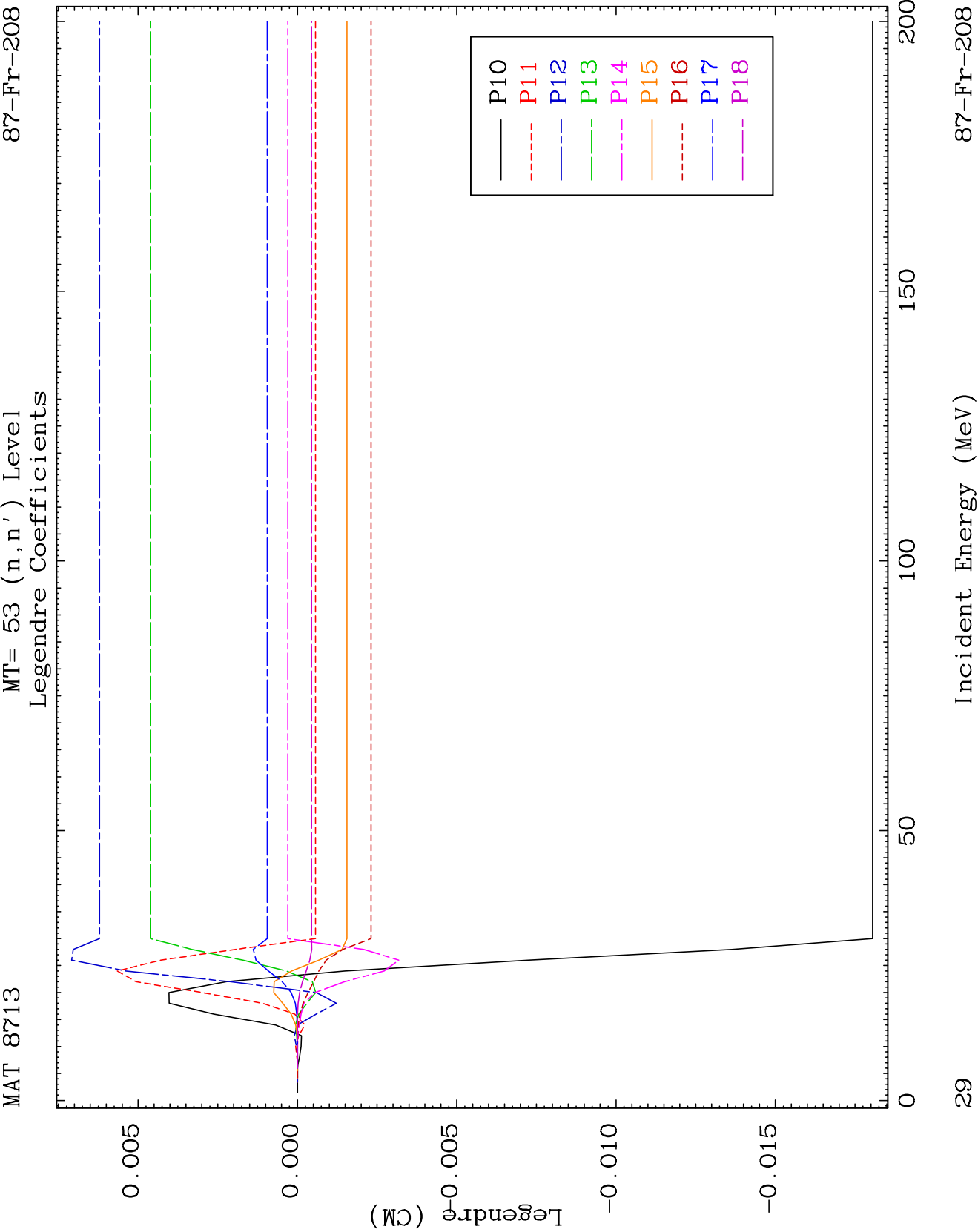








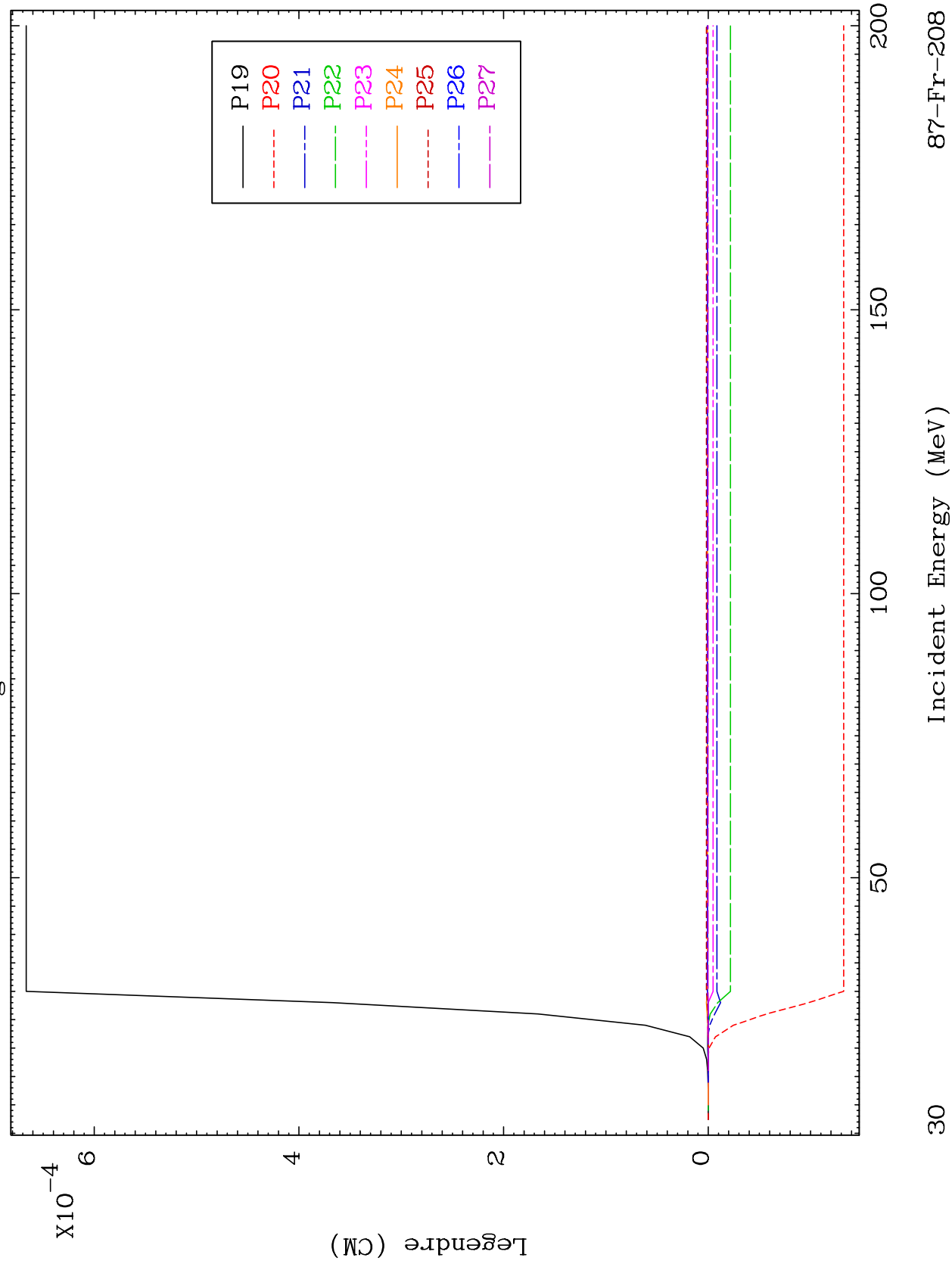


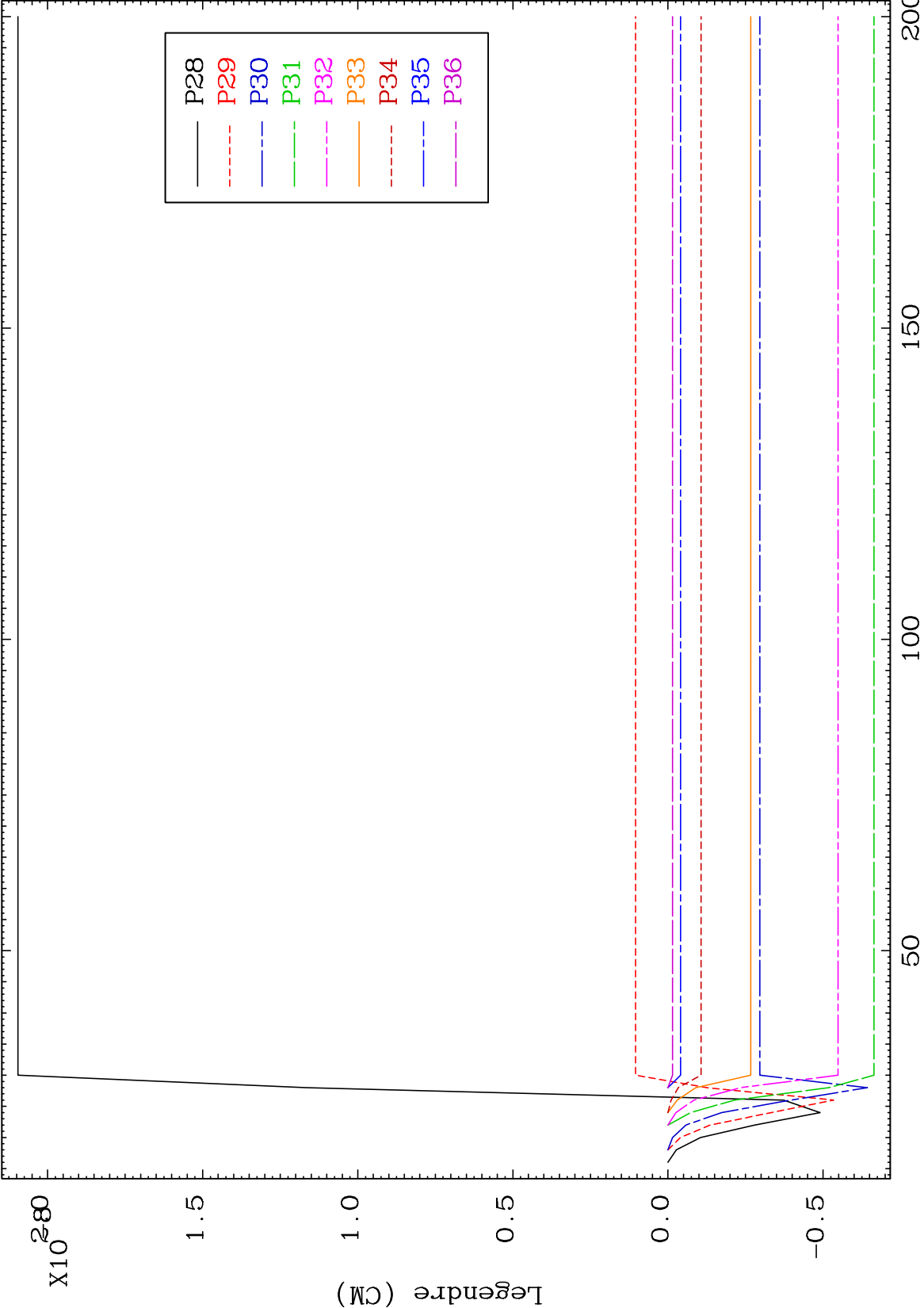


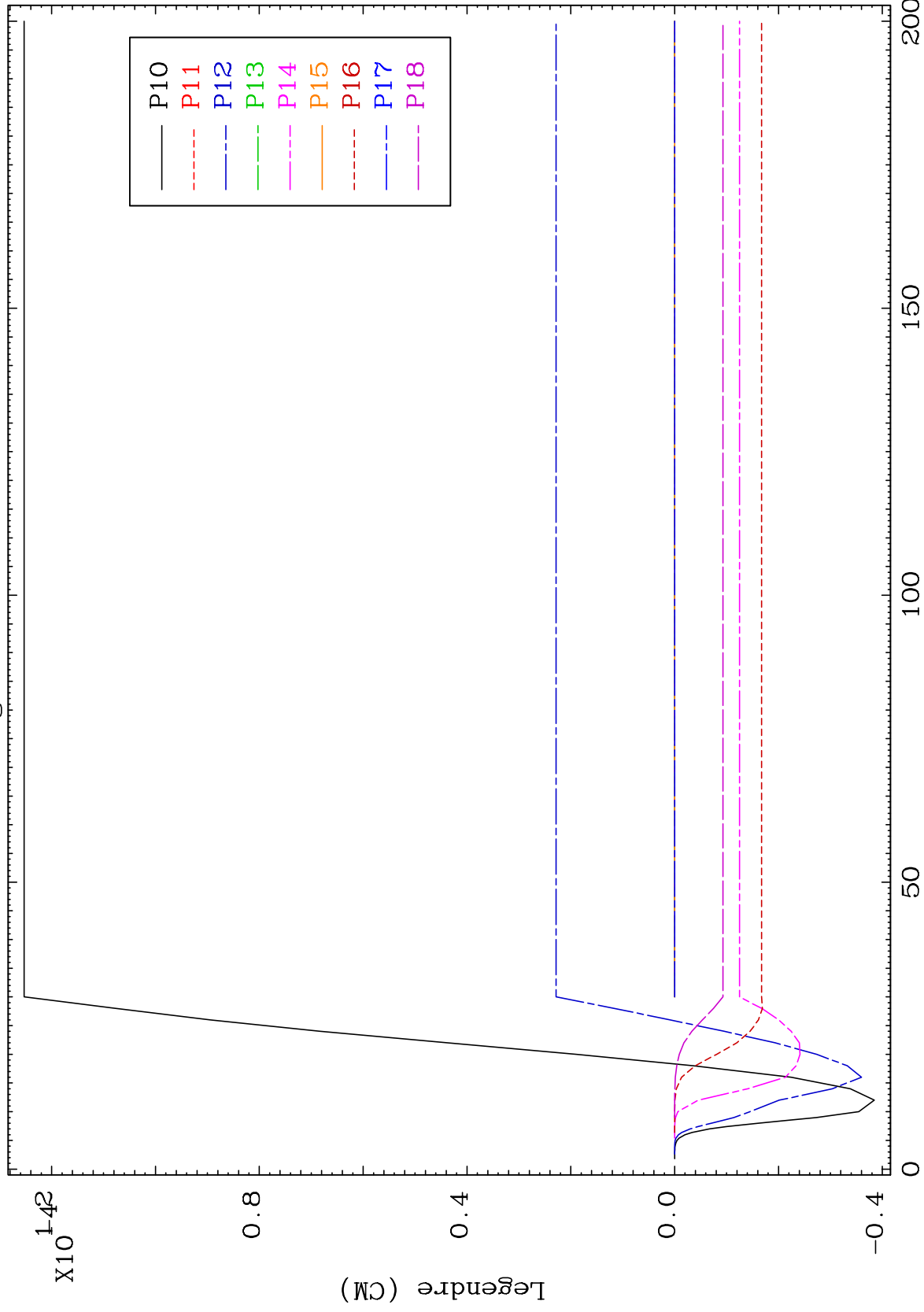
MAT 8713

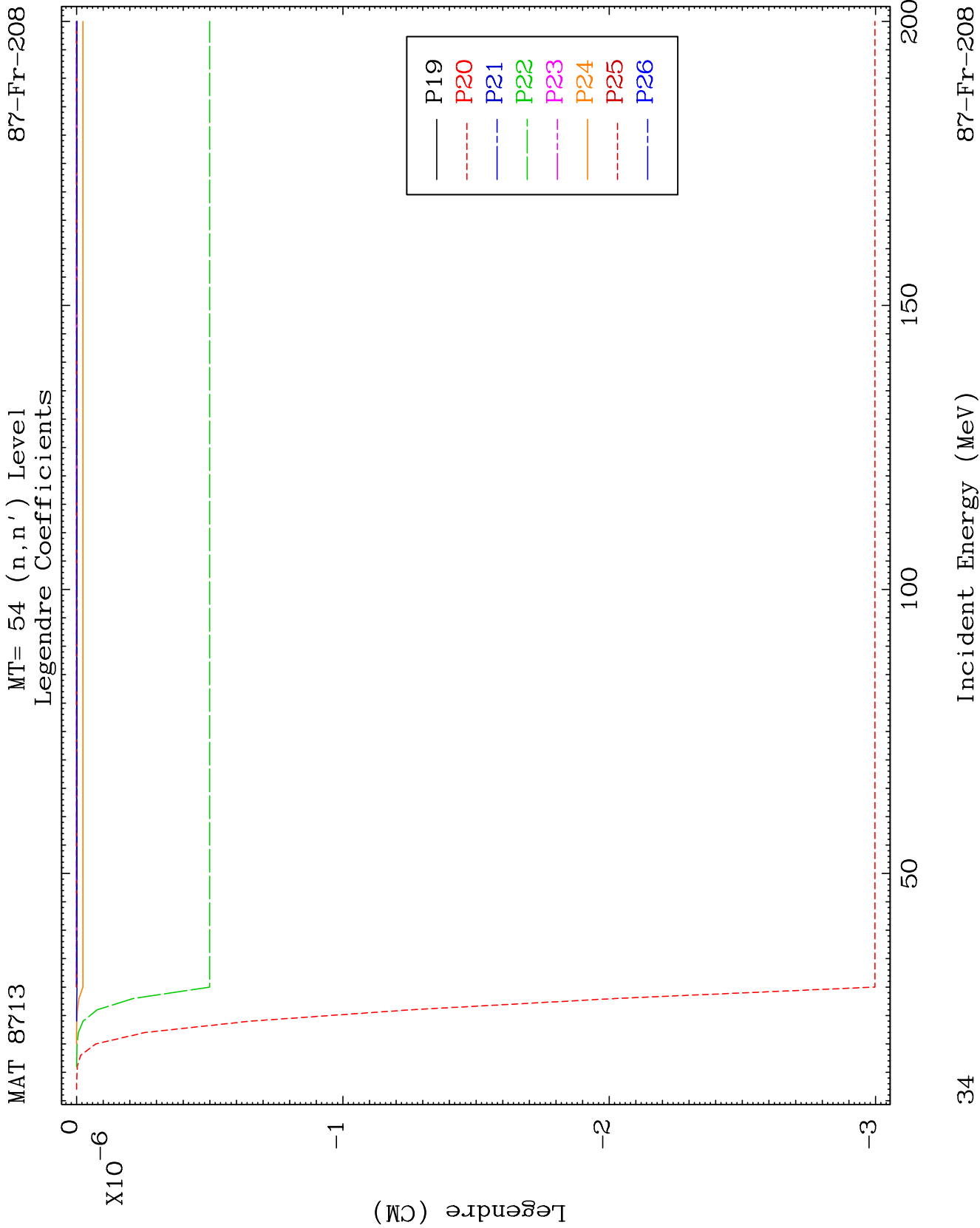
MT= 53 (n, n') Level
Legendre Coefficients

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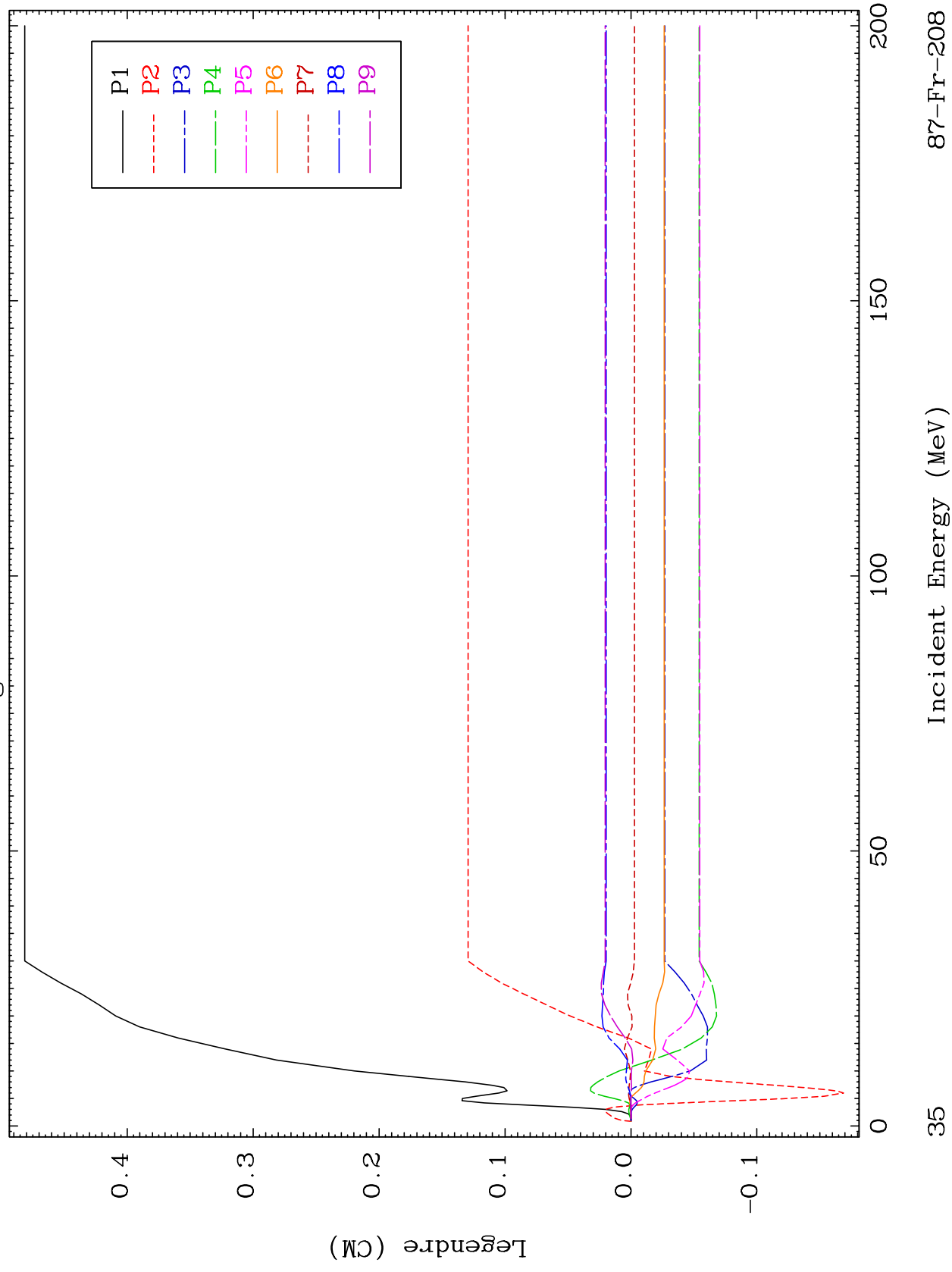




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MT= 55 (n, n') Level
Legendre Coefficients

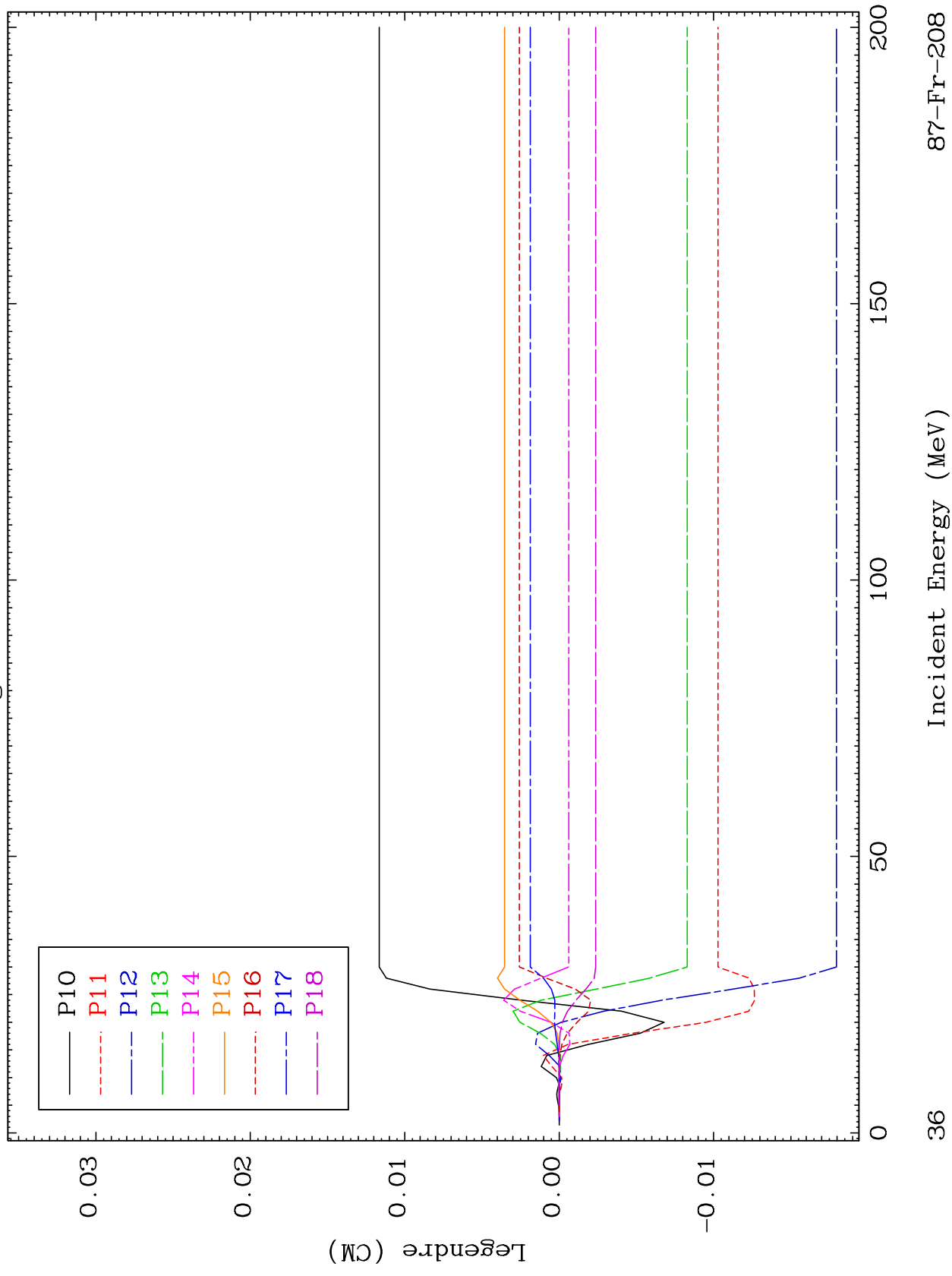
87-Fr-208



MAT 8713

87-Fr-208

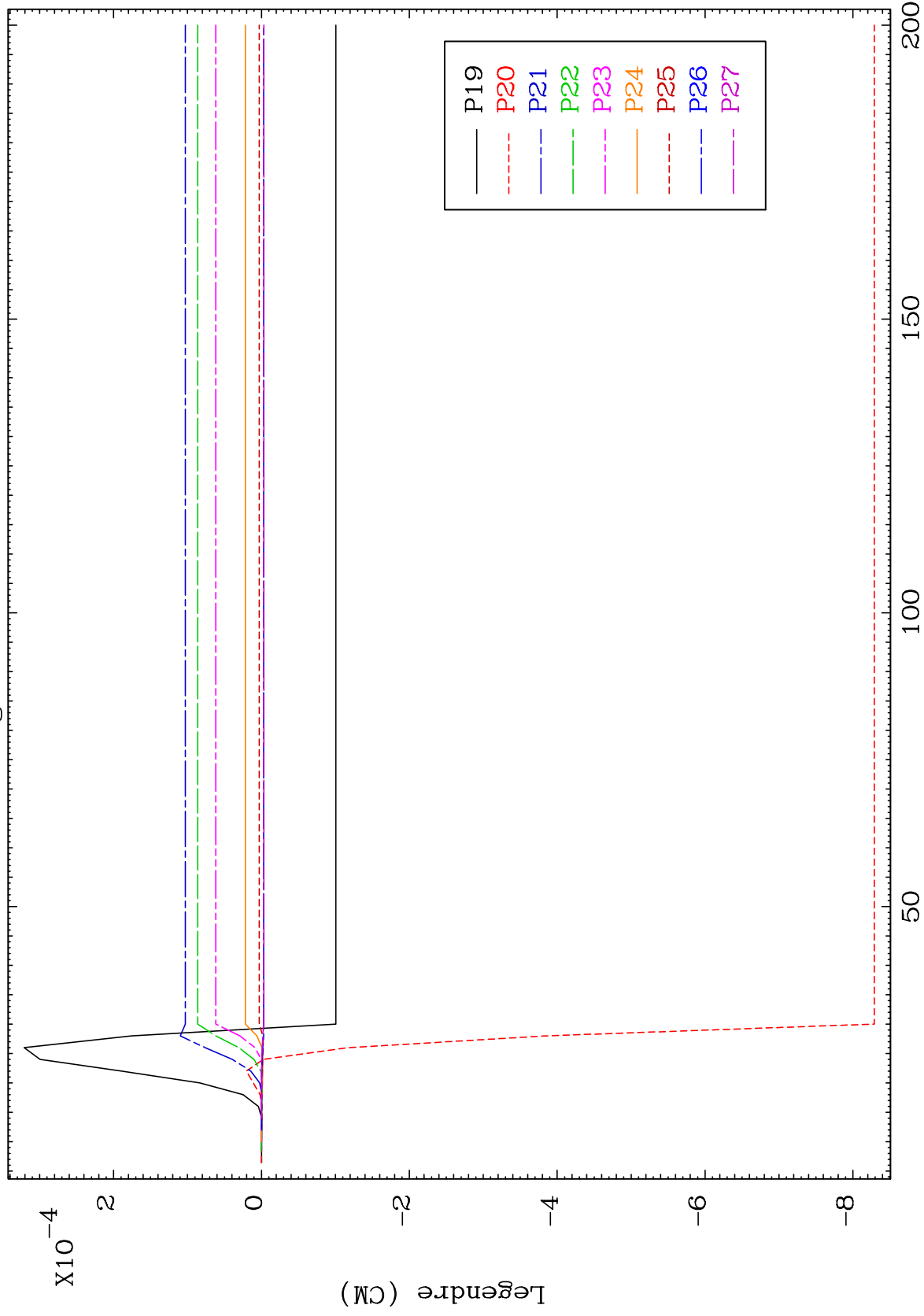
MT= 55 (n, n') Level
Legendre Coefficients



MAT 8713

MT= 55 (n,n') Level
Legendre Coefficients

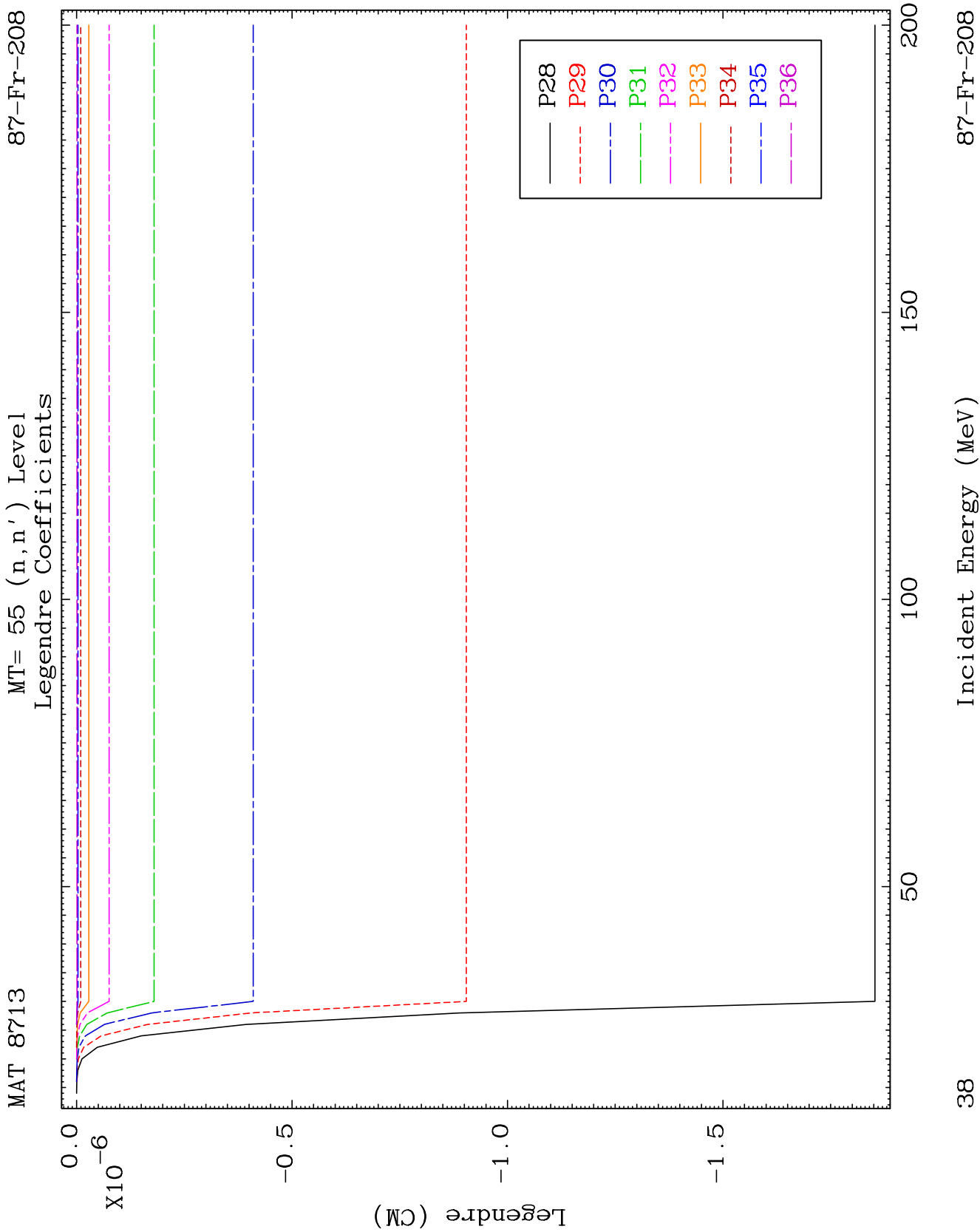
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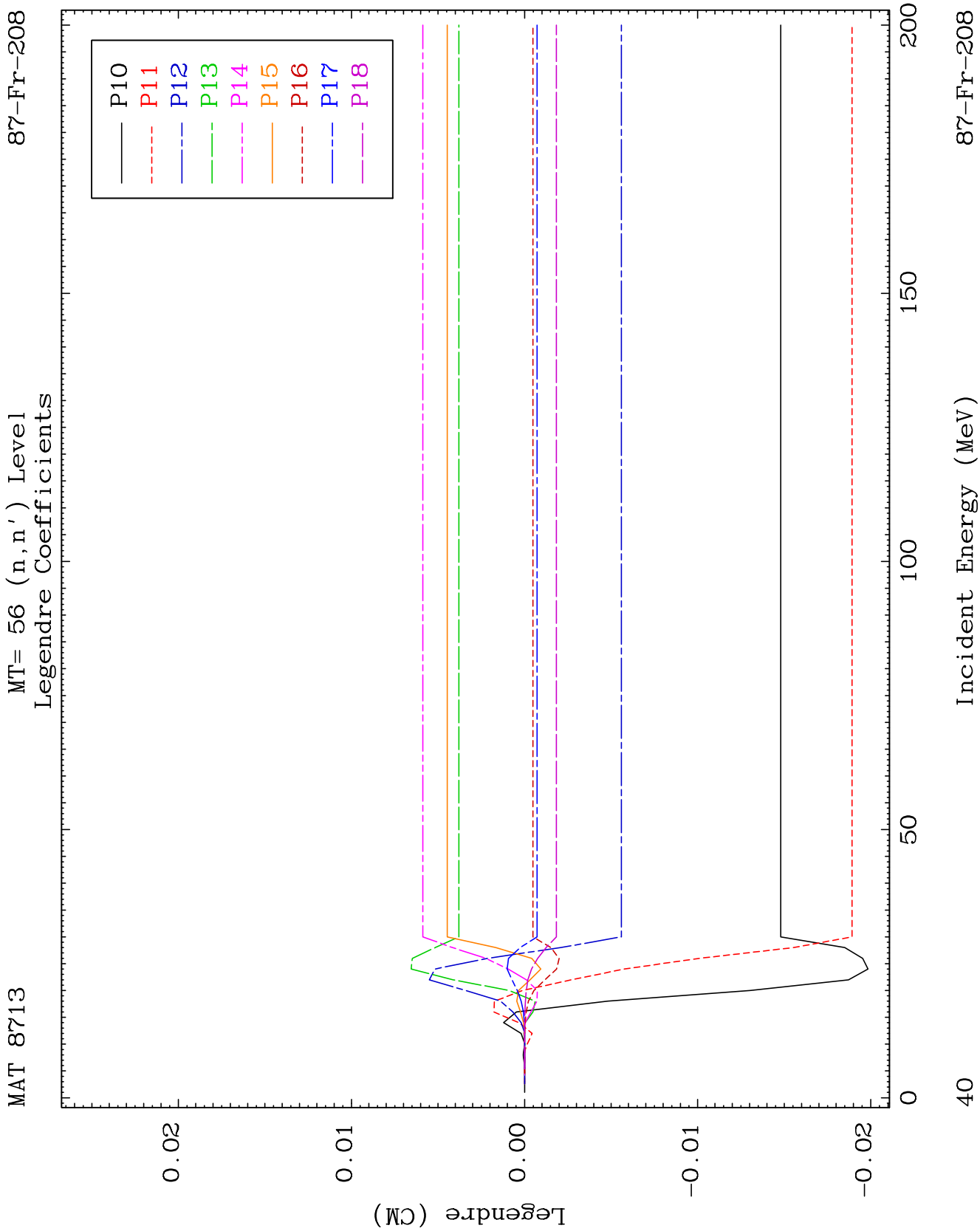


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Incident Energy (MeV)

87-Fr-208

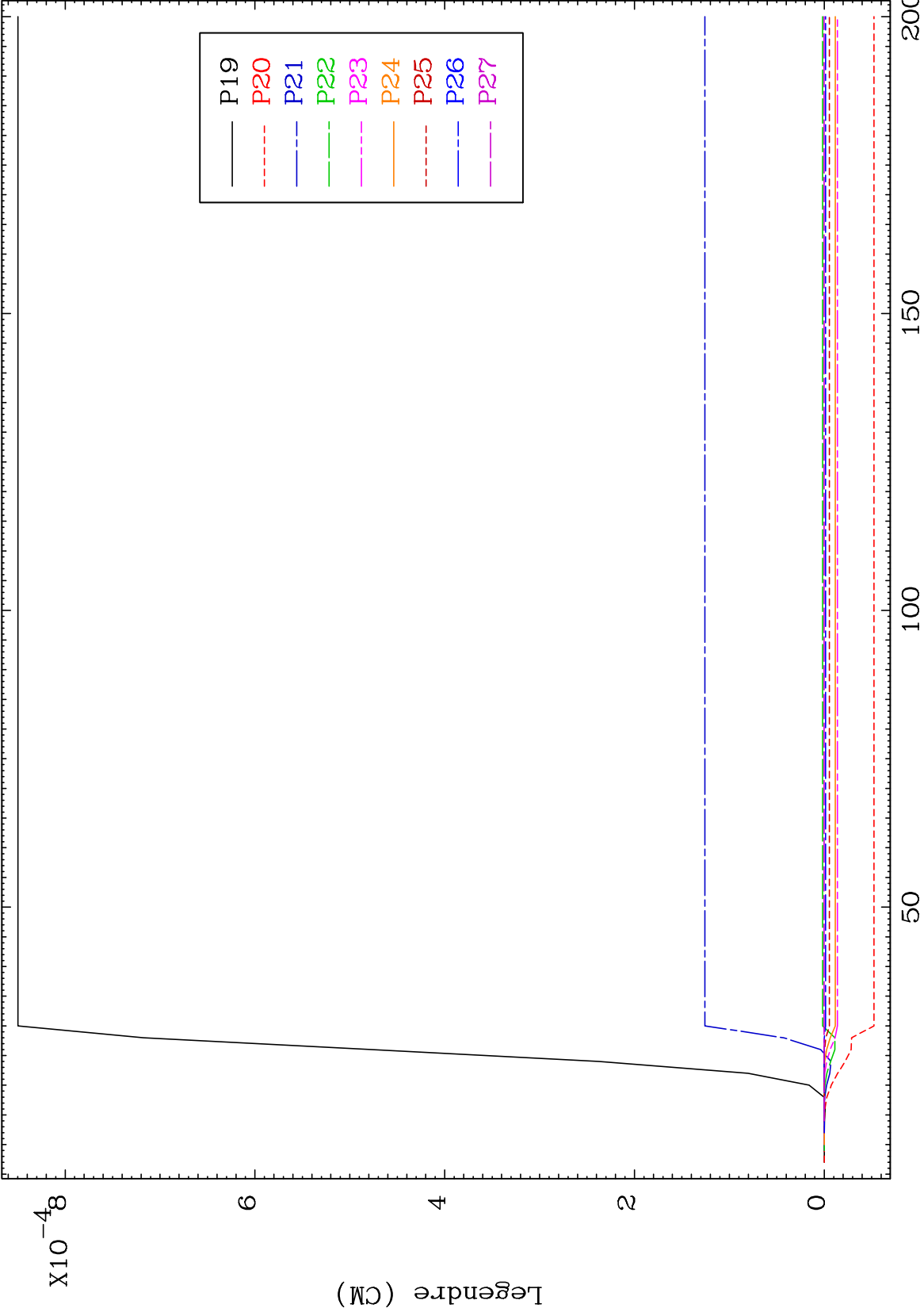




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MT= 56 (n,n') Level
Legendre Coefficients

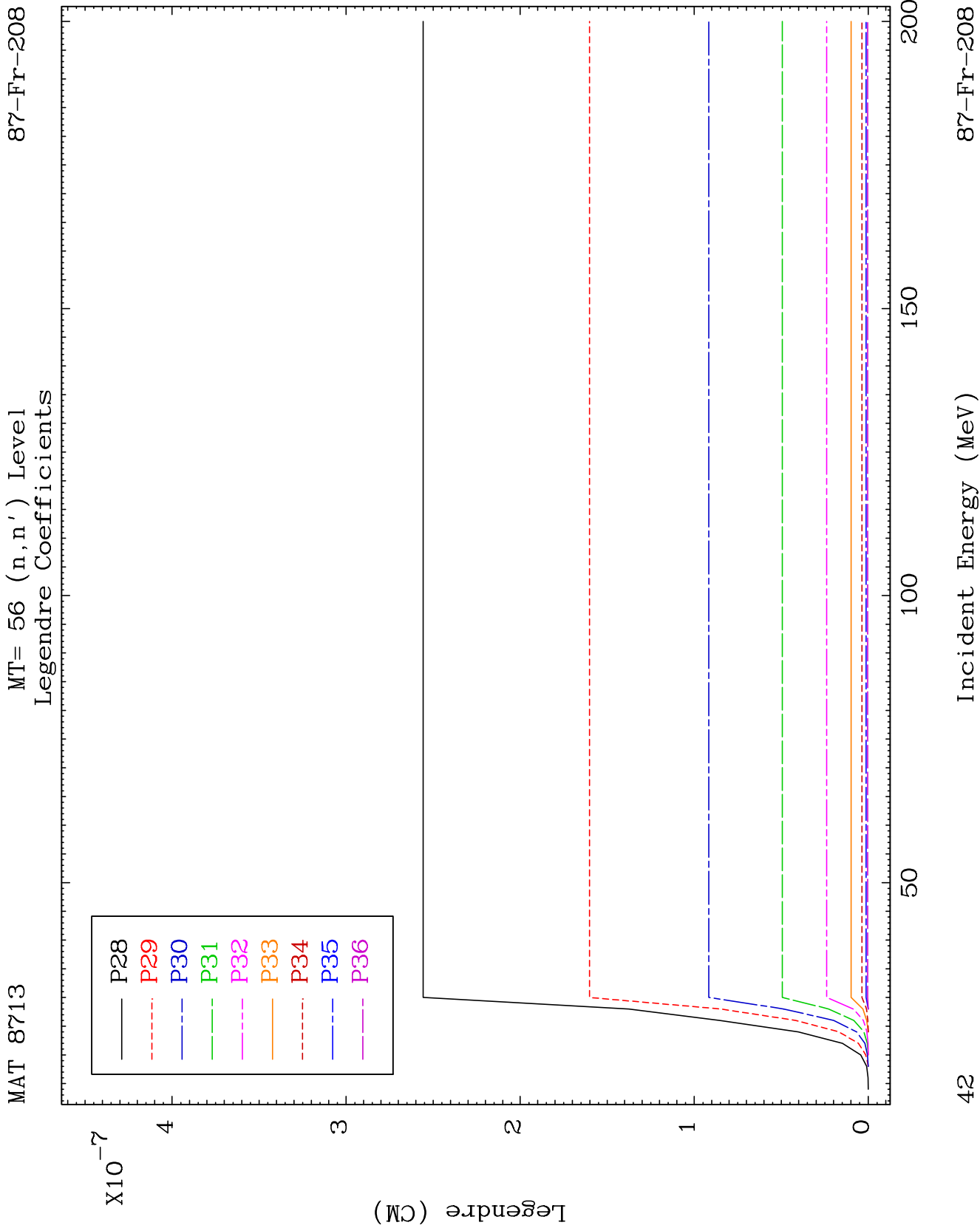
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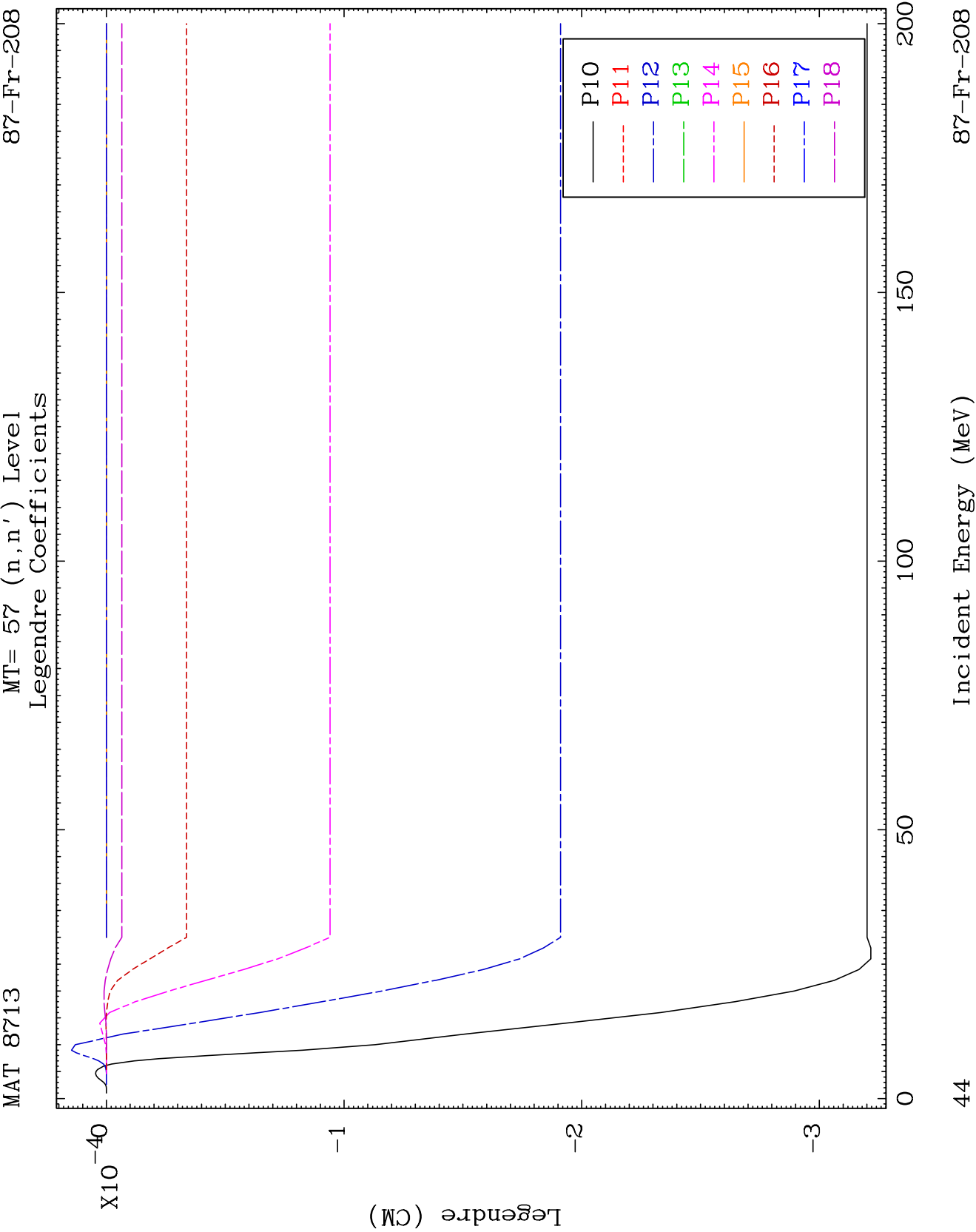


41

Incident Energy (MeV)

87-Fr-208

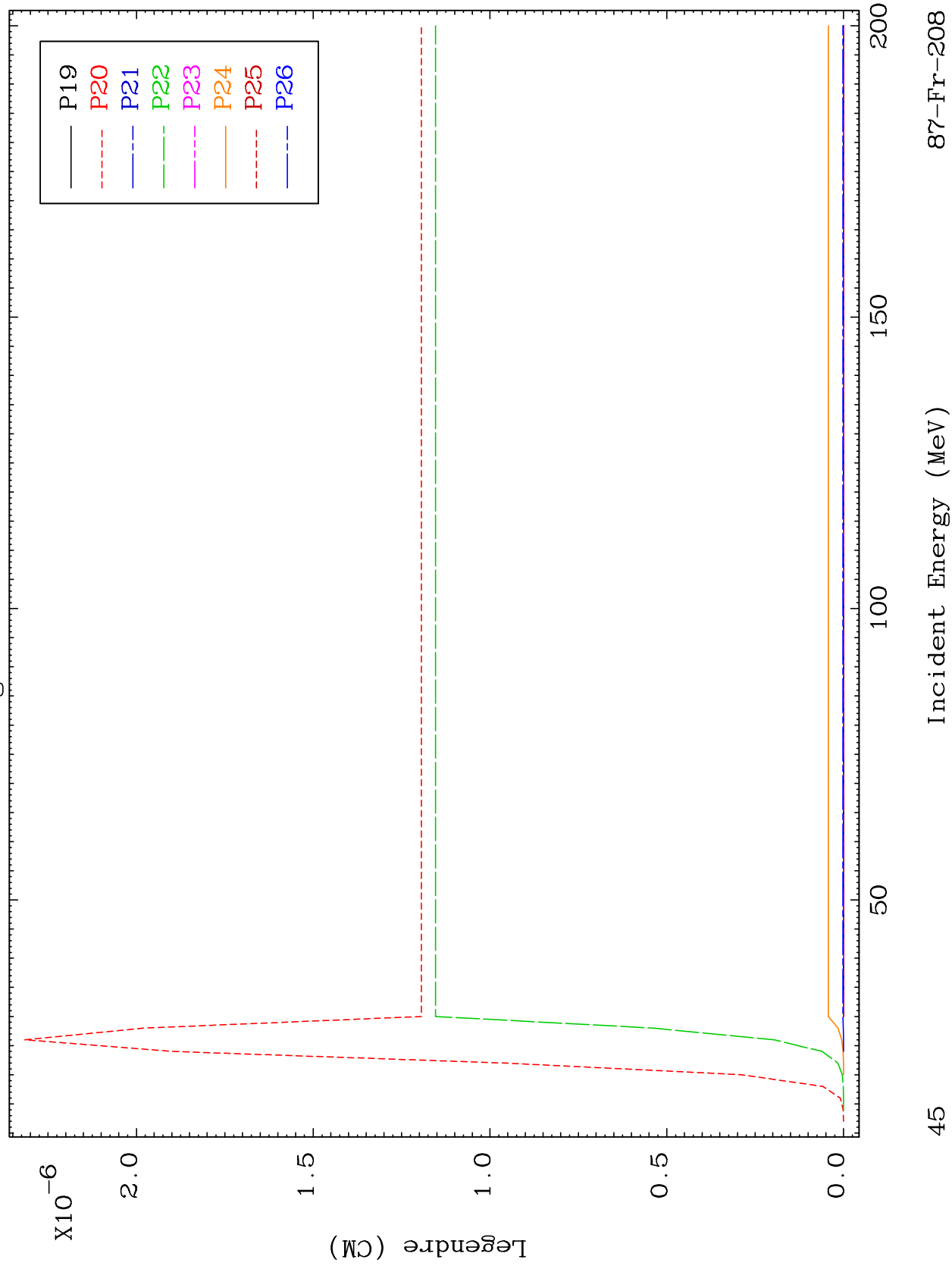




MAT 8713

MT= 57 (n, n') Level
Legendre Coefficients

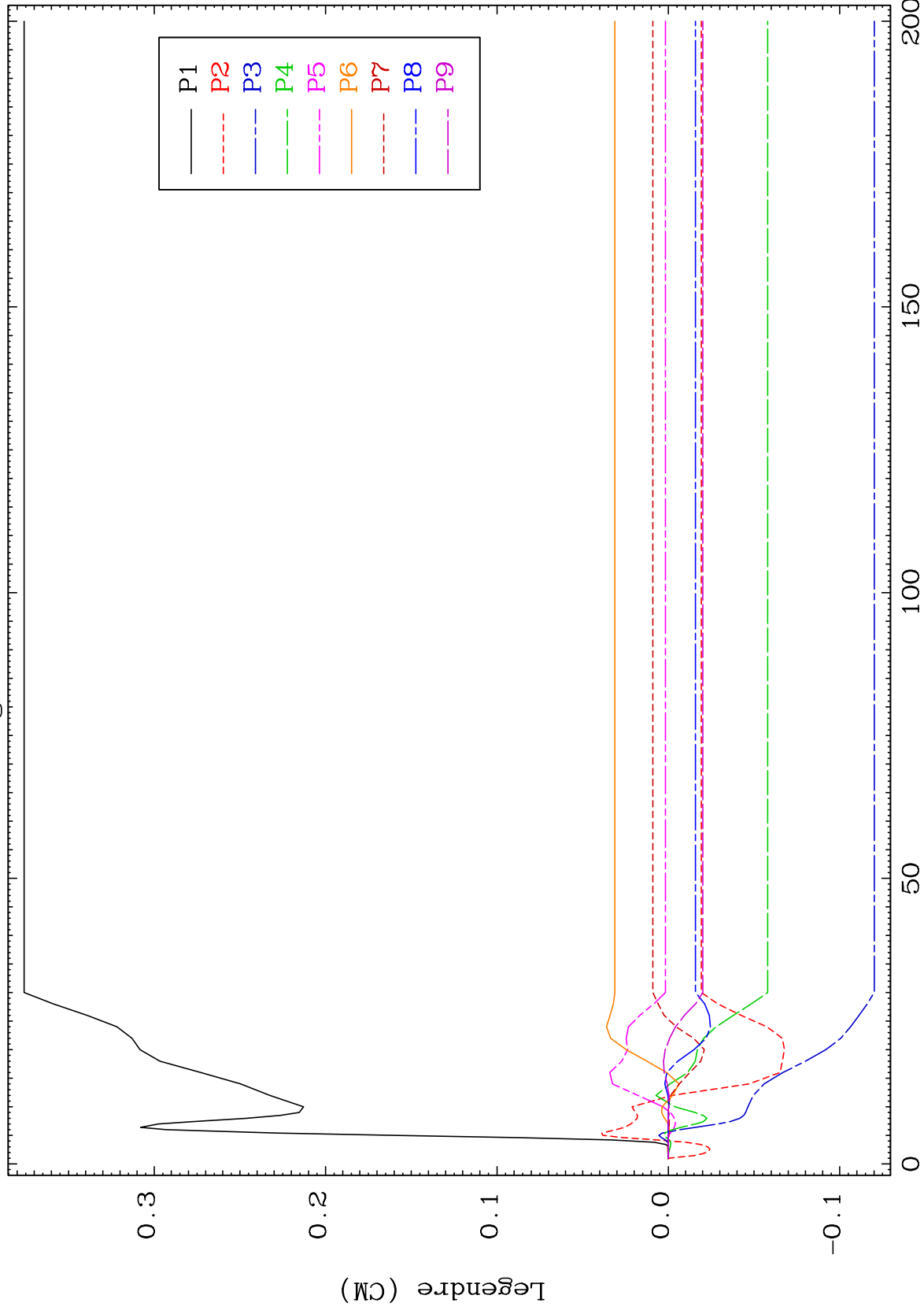
87-Fr-208



MAT 8713

MT= 58 (n,n') Level
Legendre Coefficients

87-Fr-208



87-Fr-208

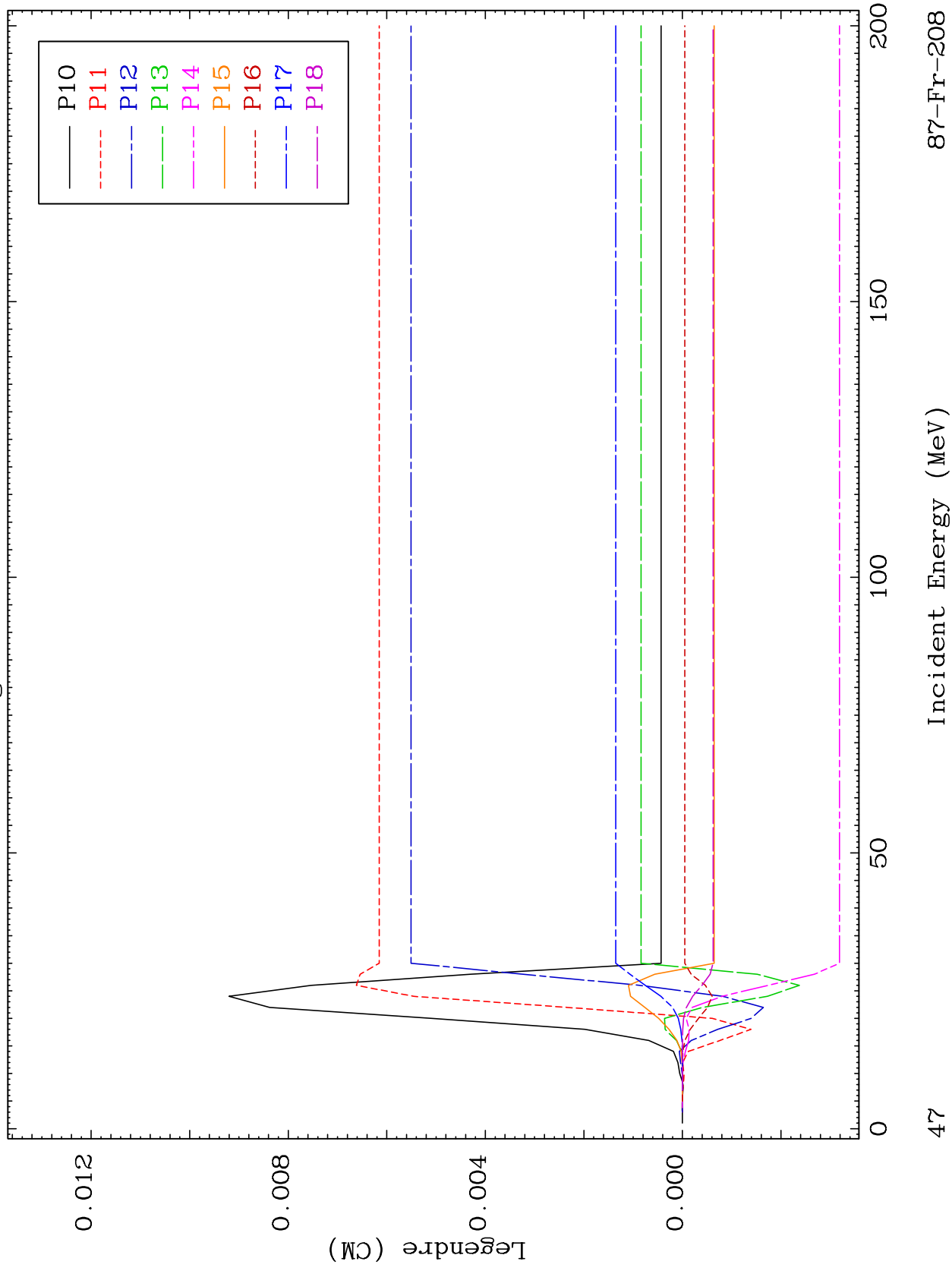
Incident Energy (MeV)

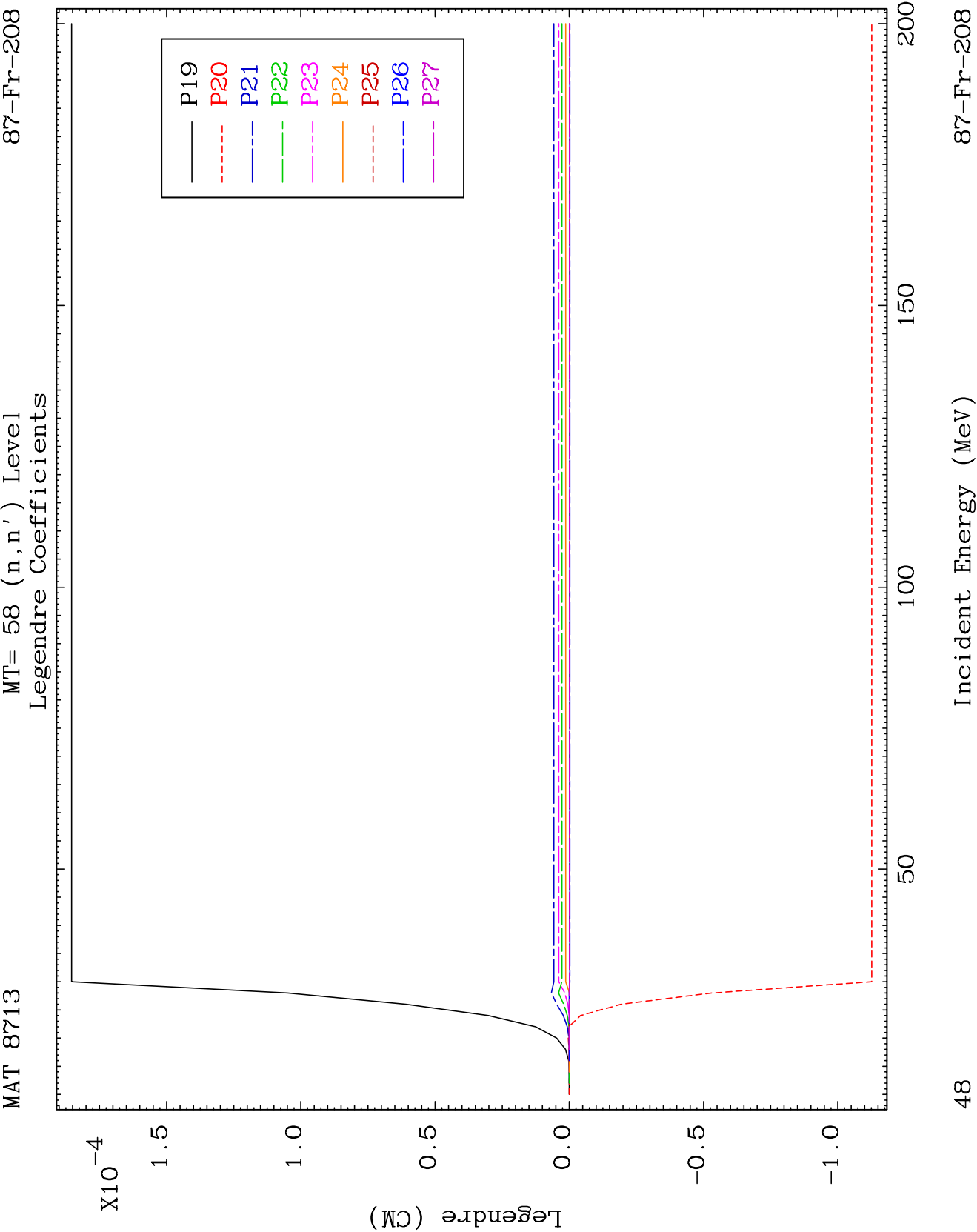
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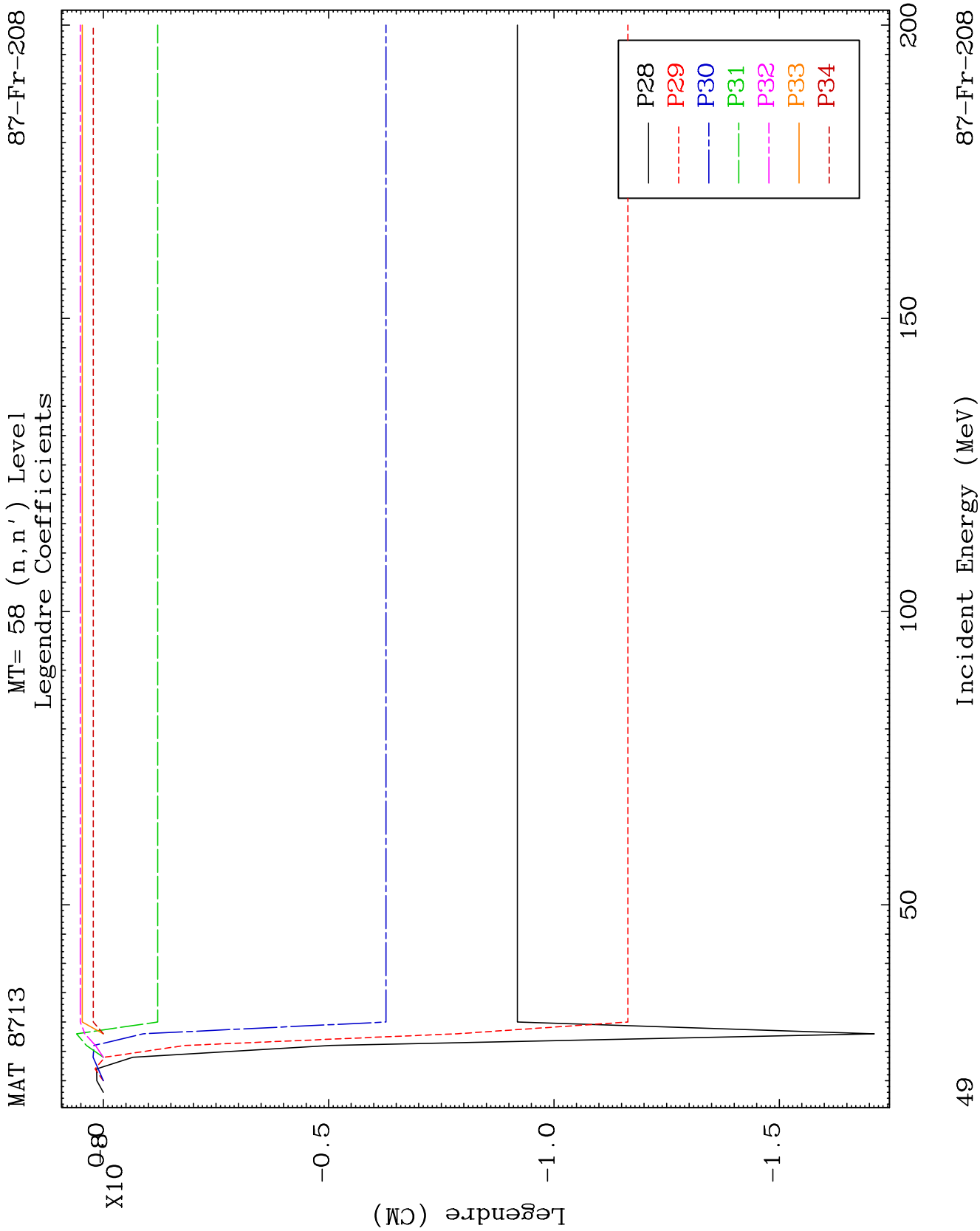
MAT 8713

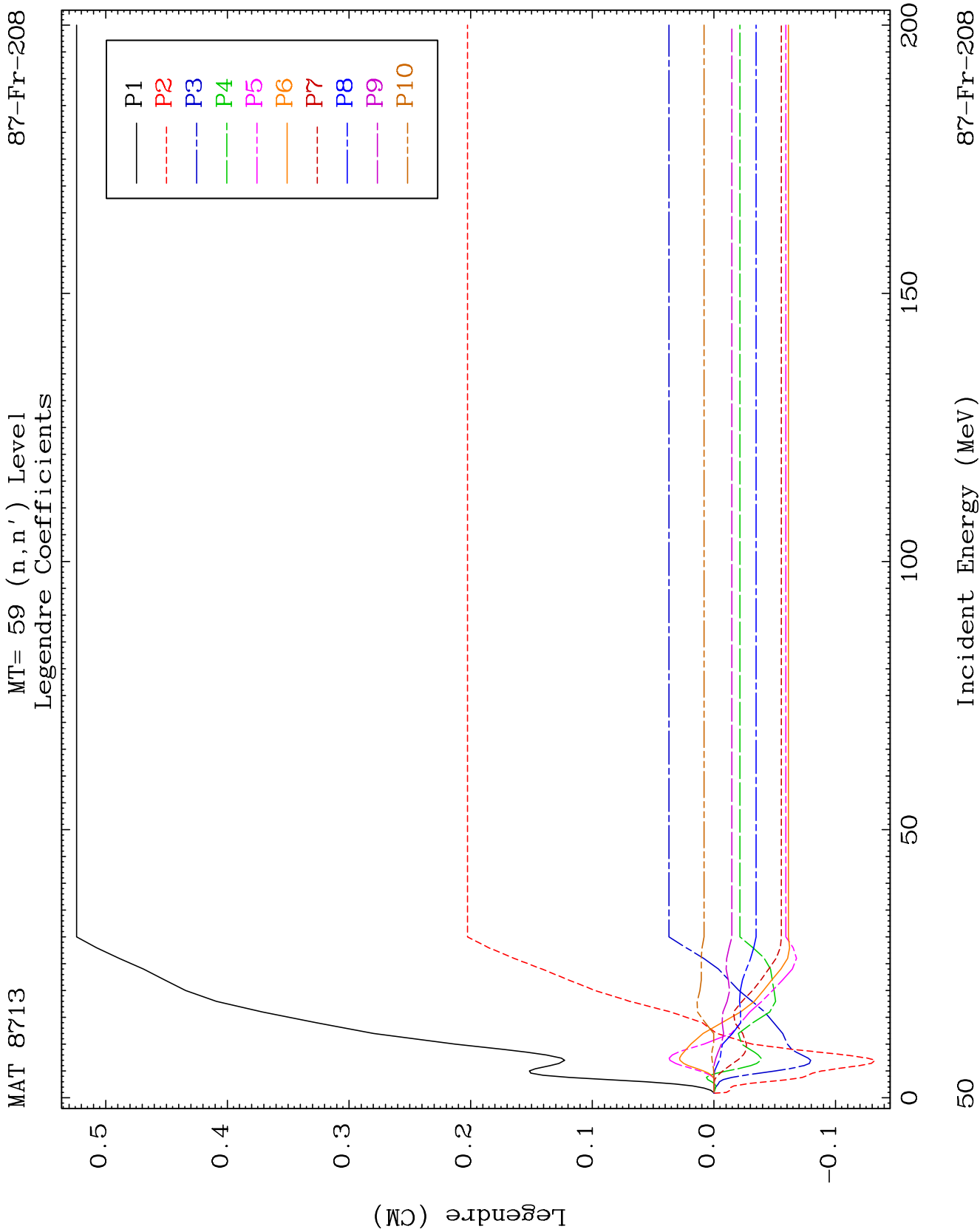
MT= 58 (n, n') Level
Legendre Coefficients

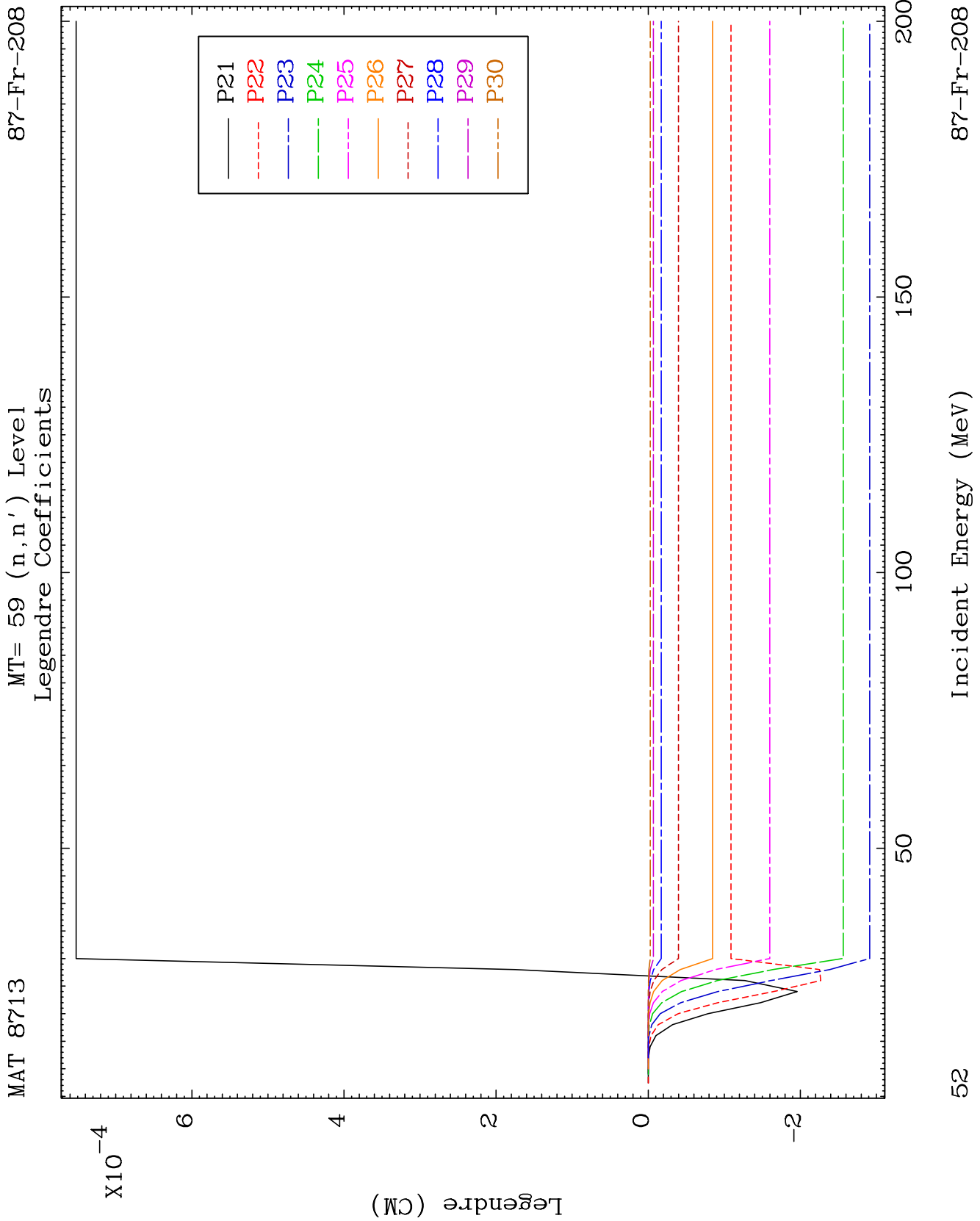
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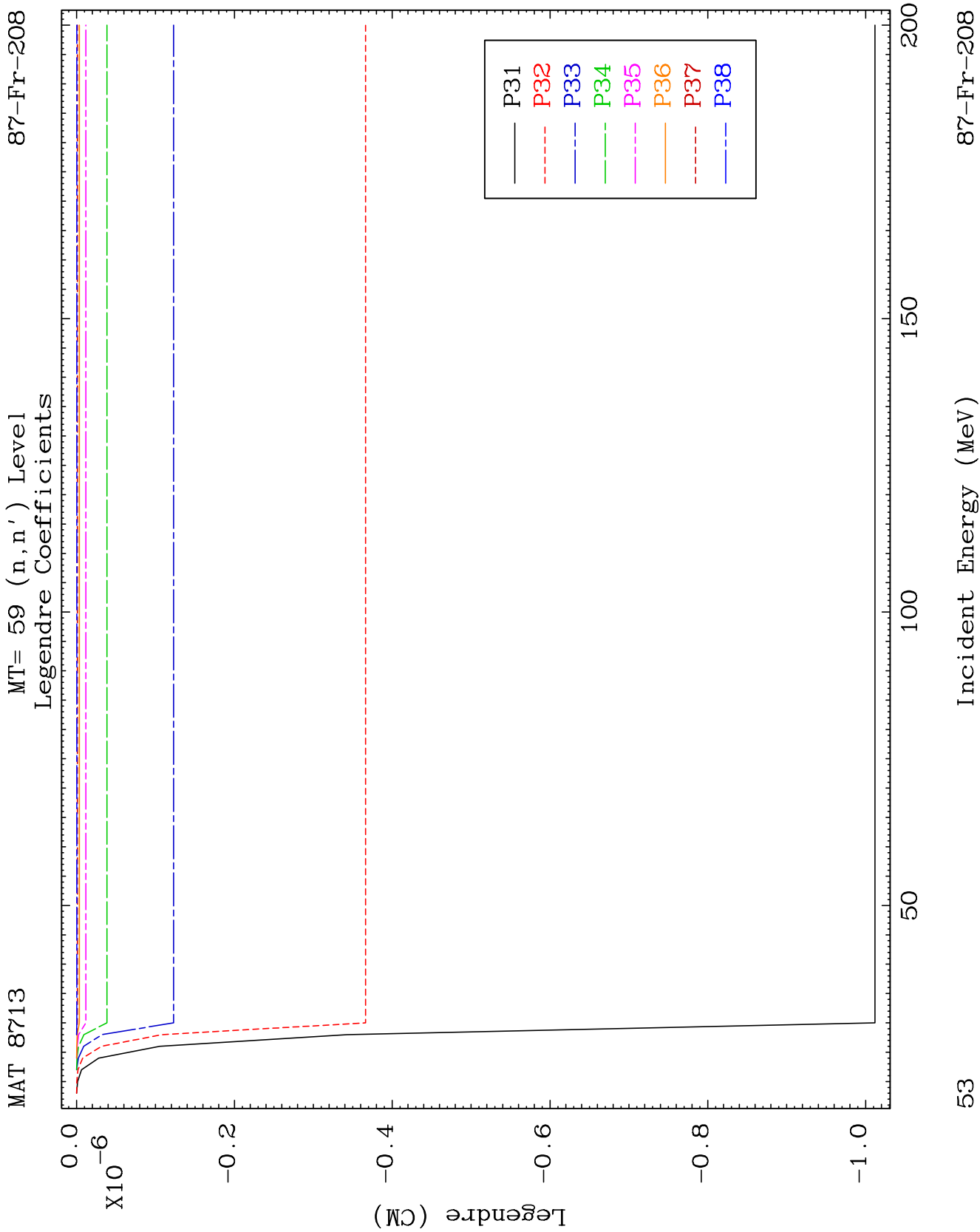


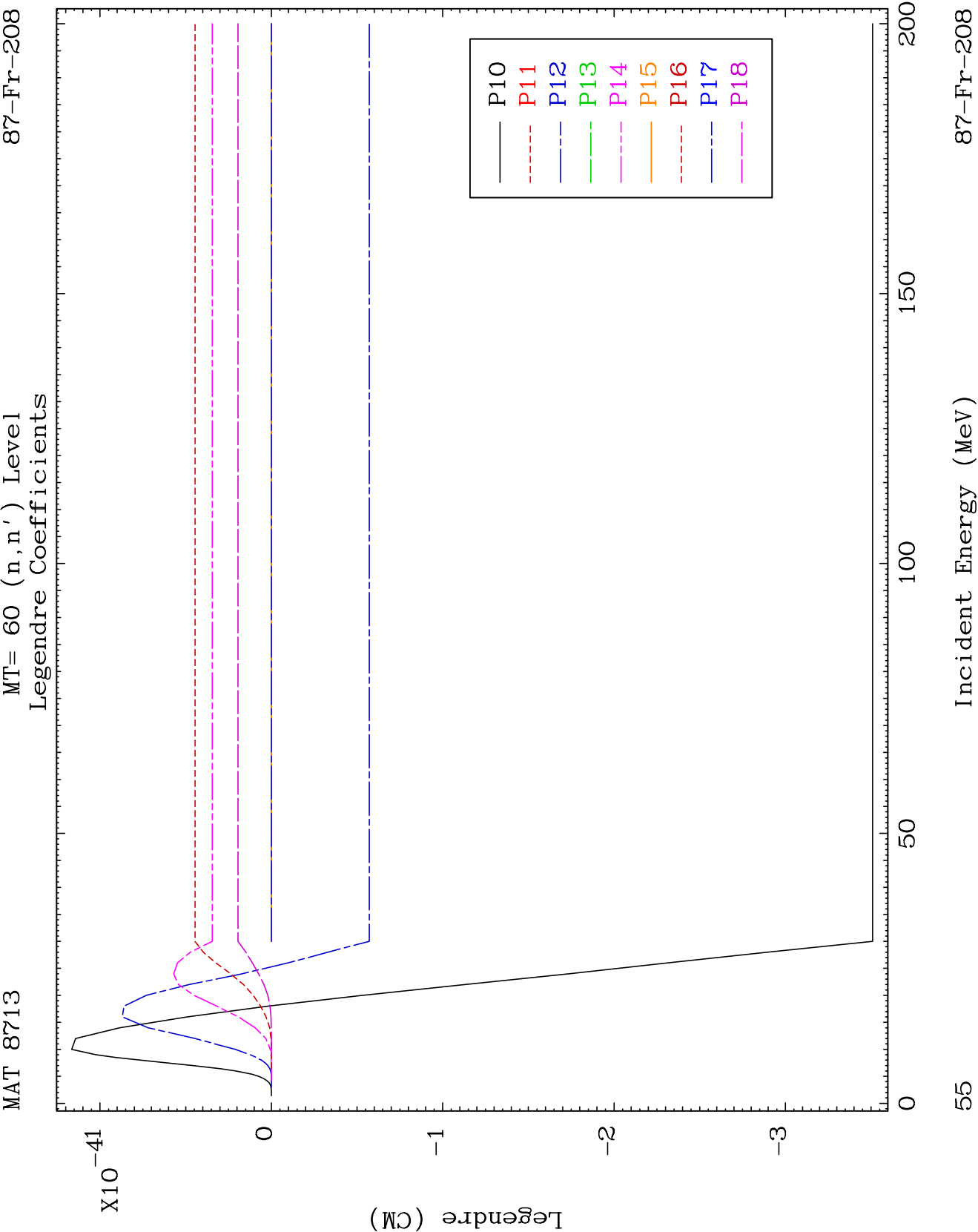


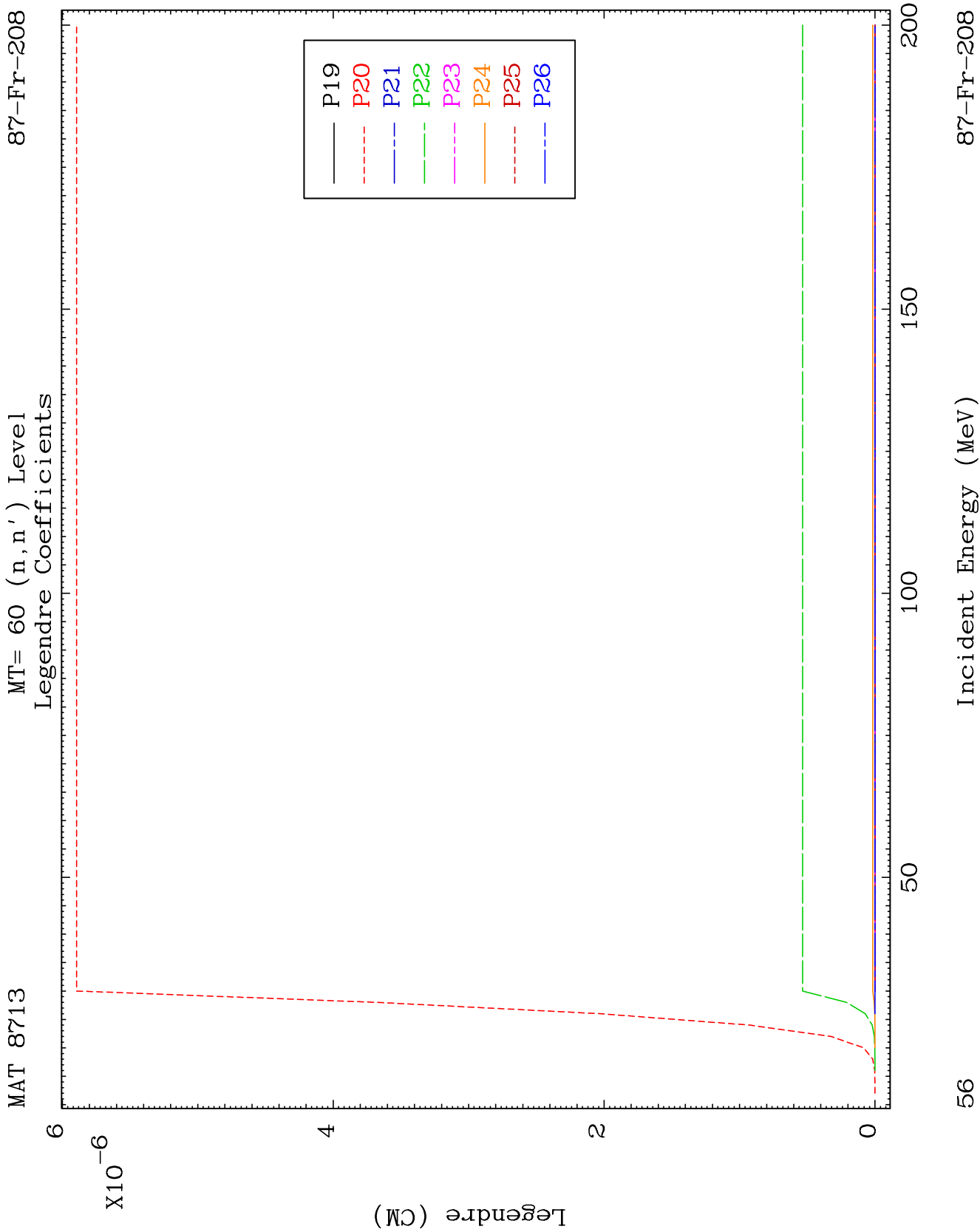


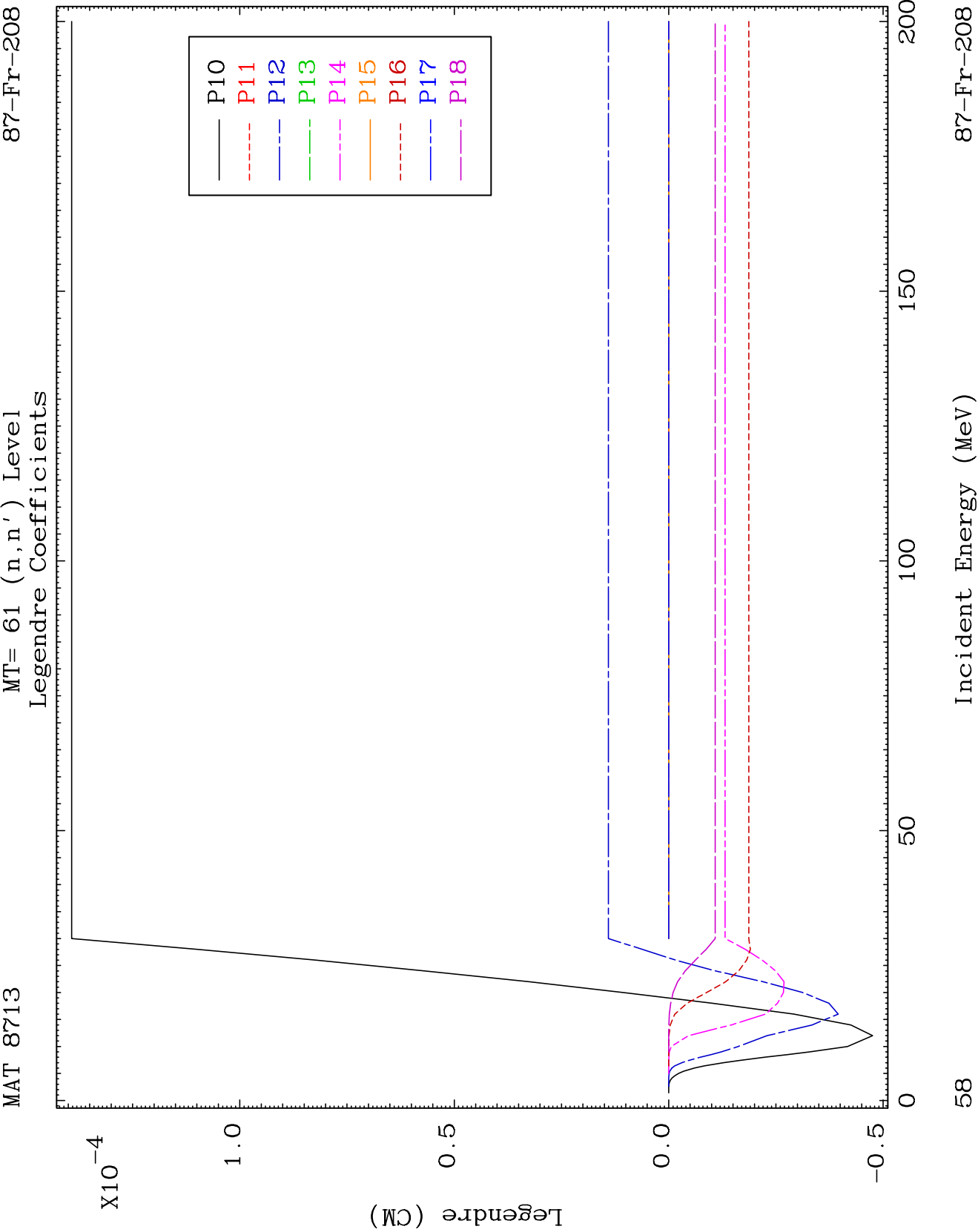








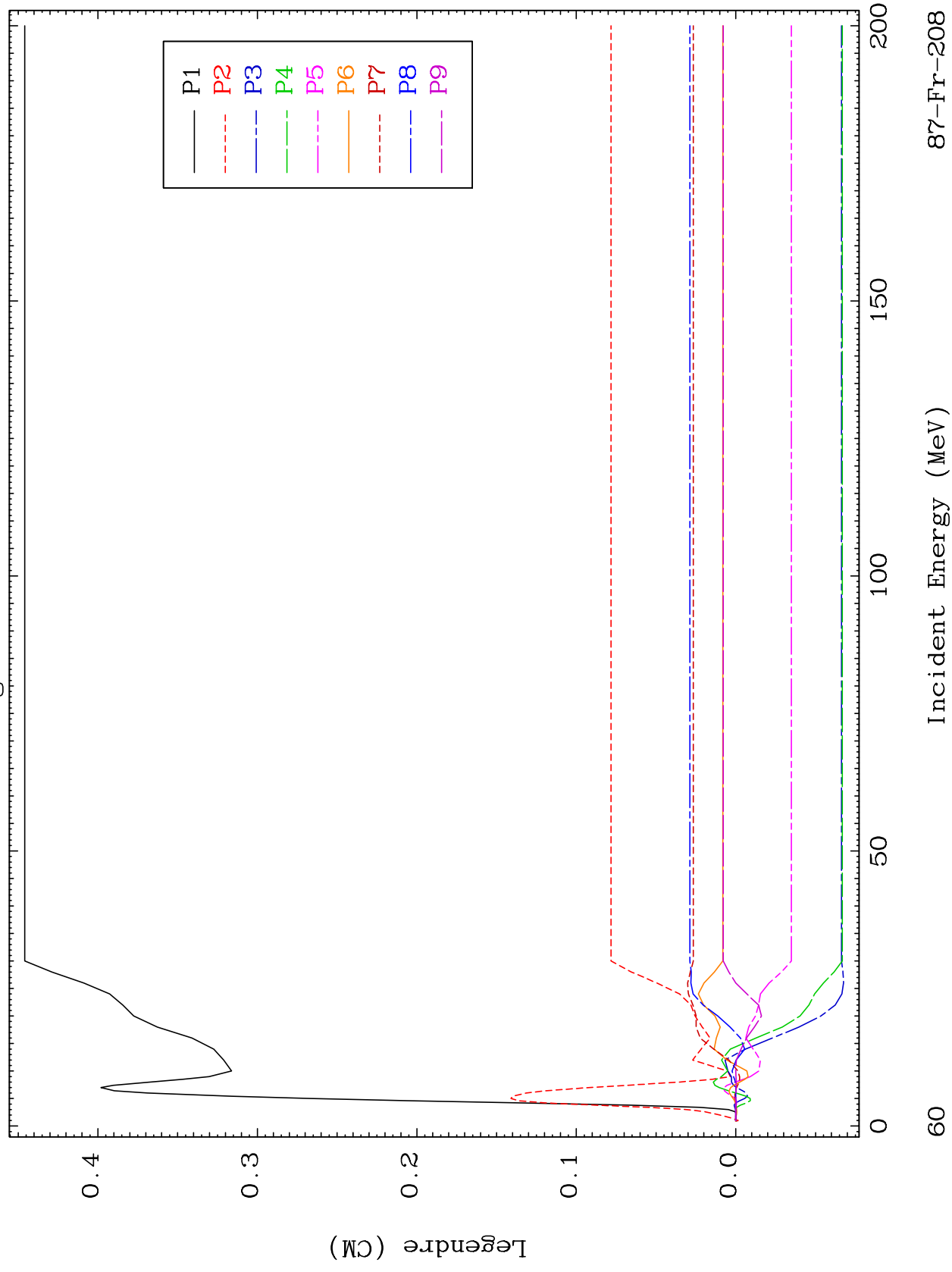




MAT 8713

MT= 62 (n, n') Level
Legendre Coefficients

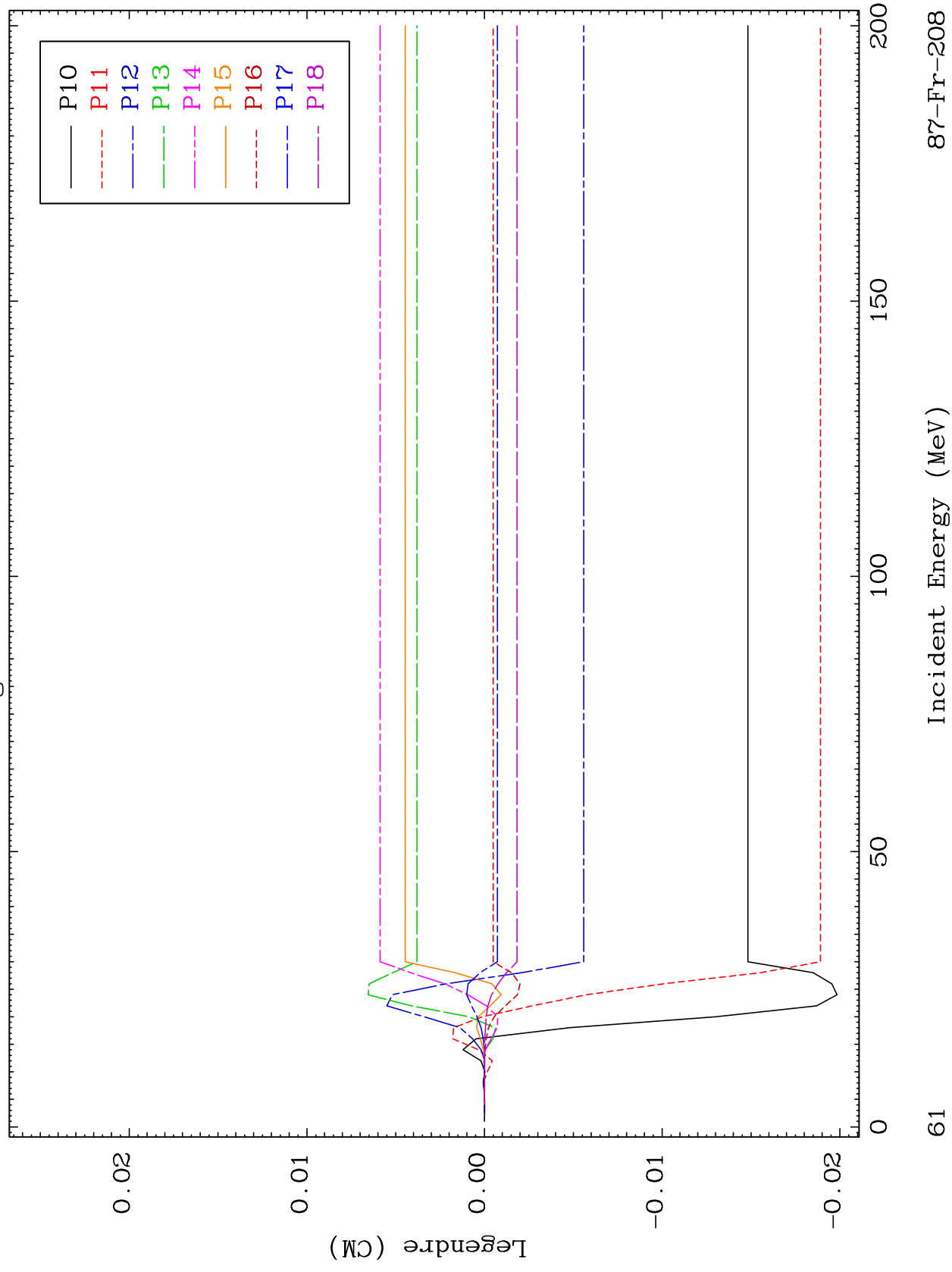
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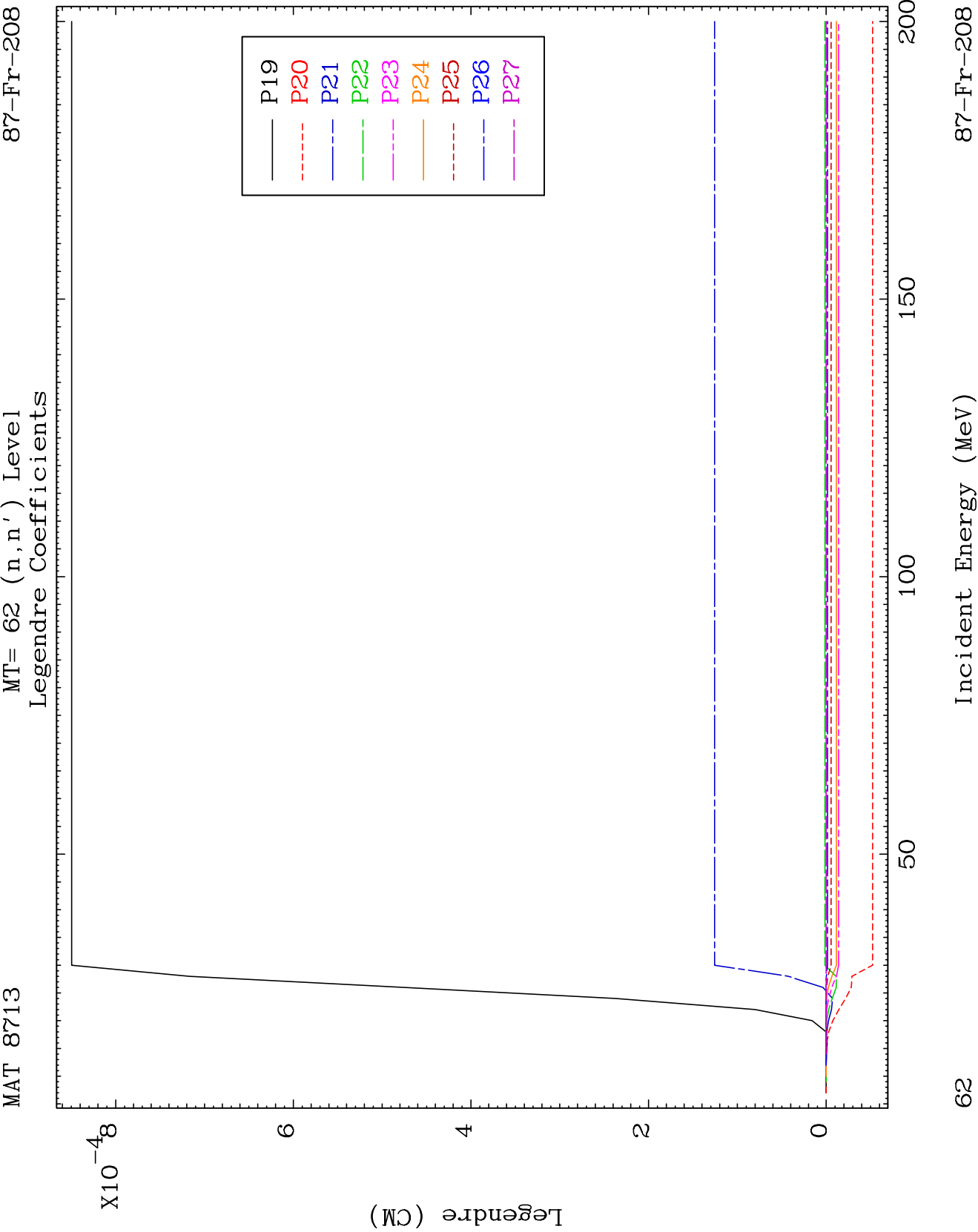


MAT 8713

MT= 62 (n, n') Level
Legendre Coefficients

87-Fr-208

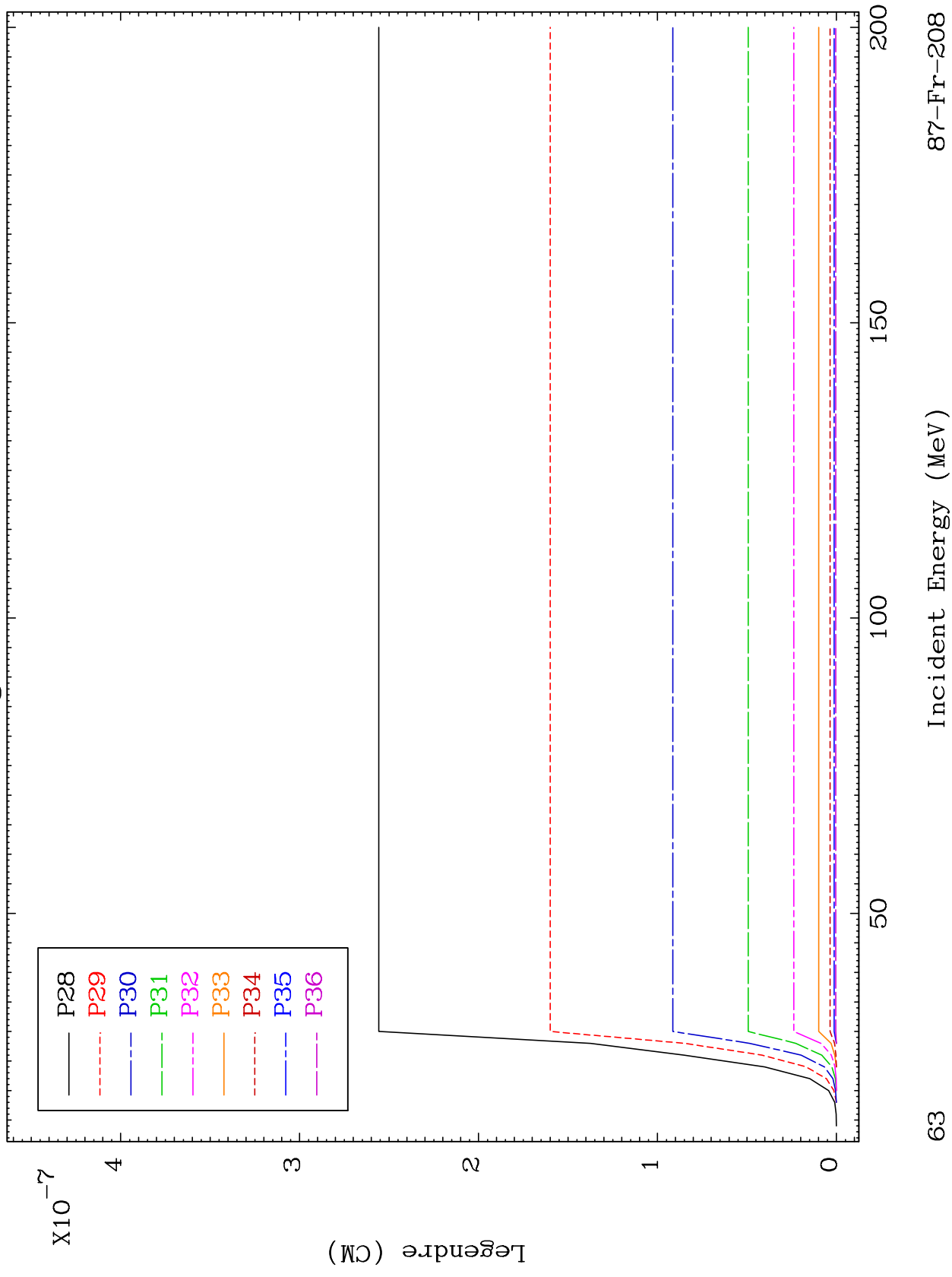


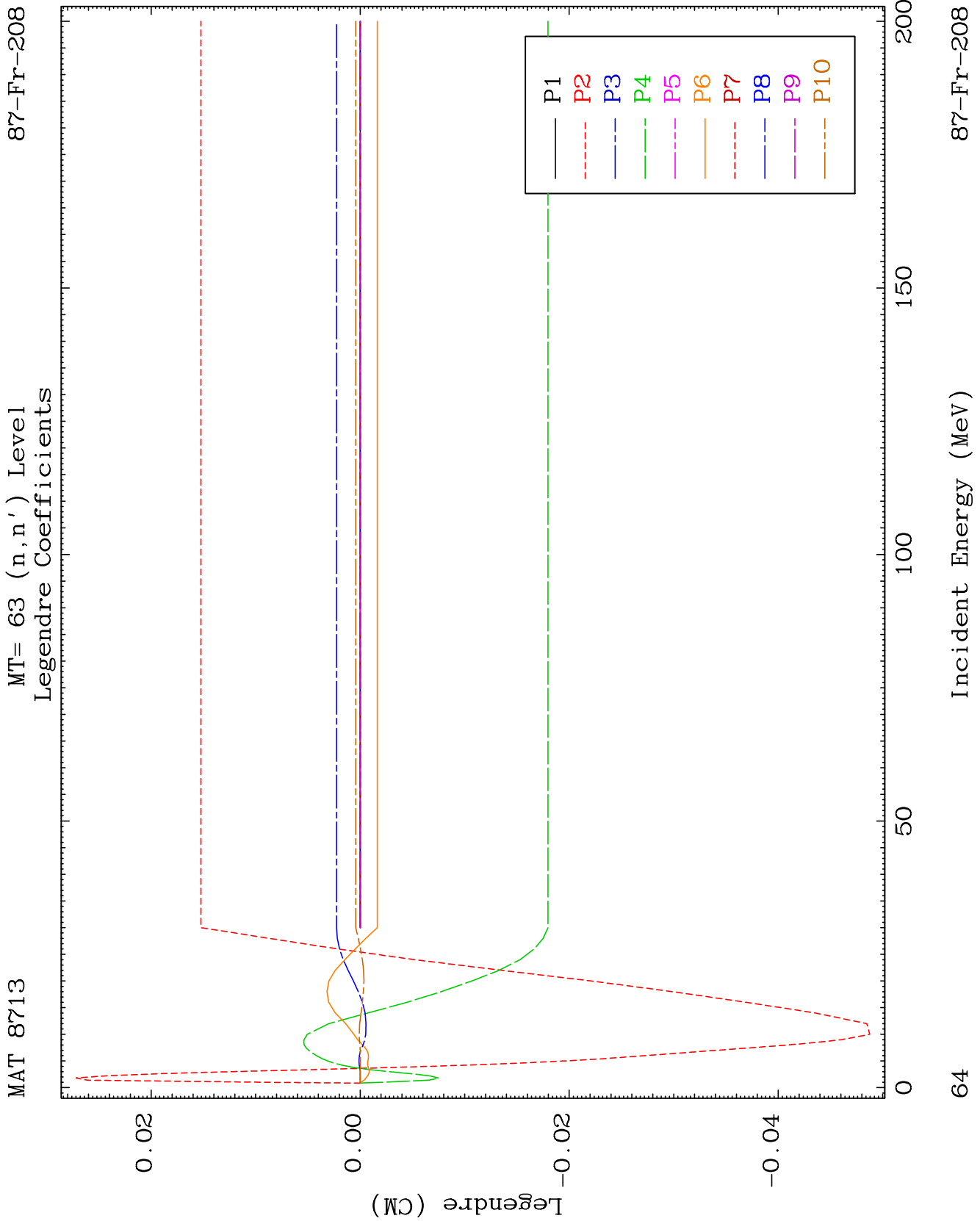


MAT 8713

MT= 62 (n, n') Level
Legendre Coefficients

87-Fr-208

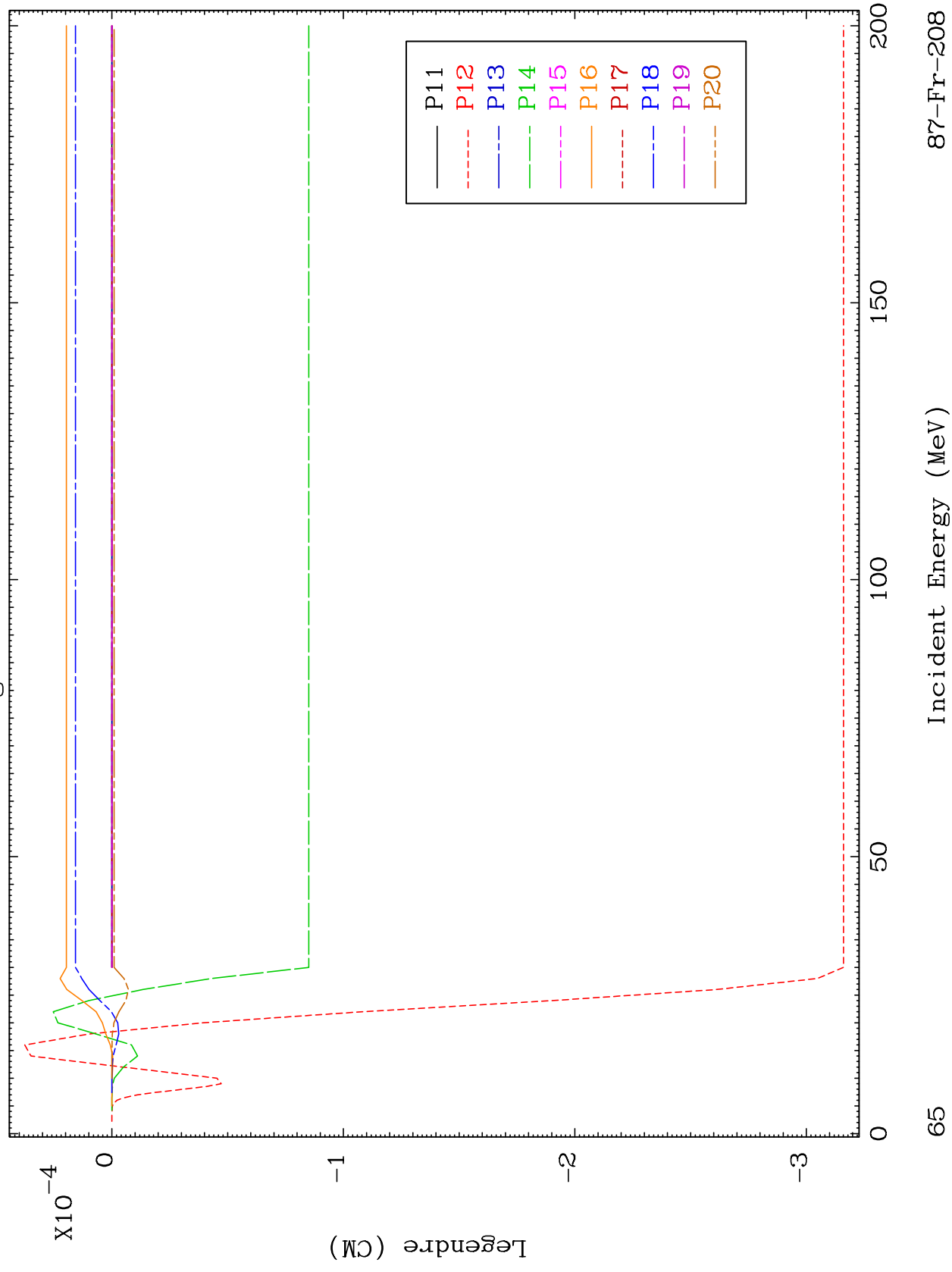


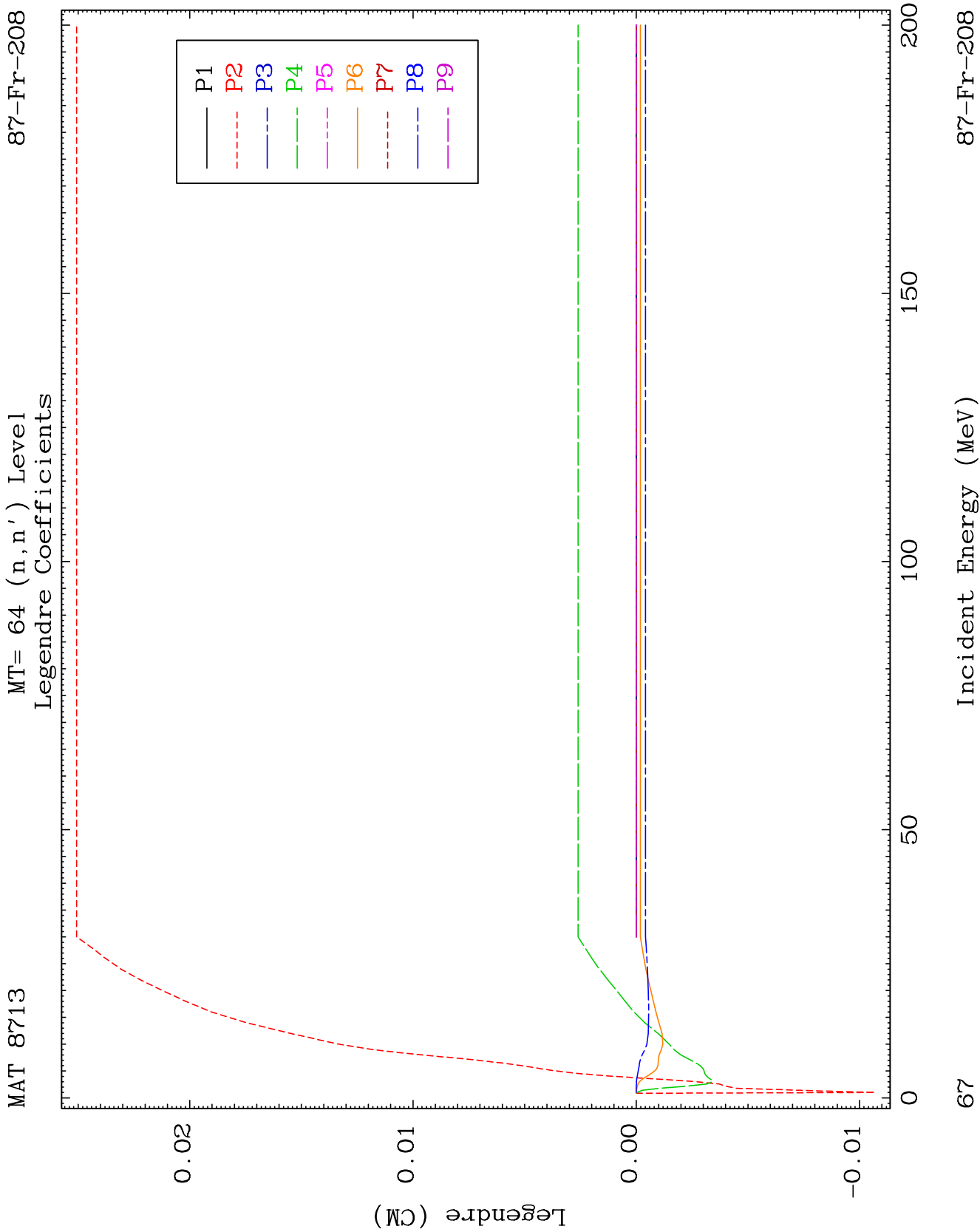


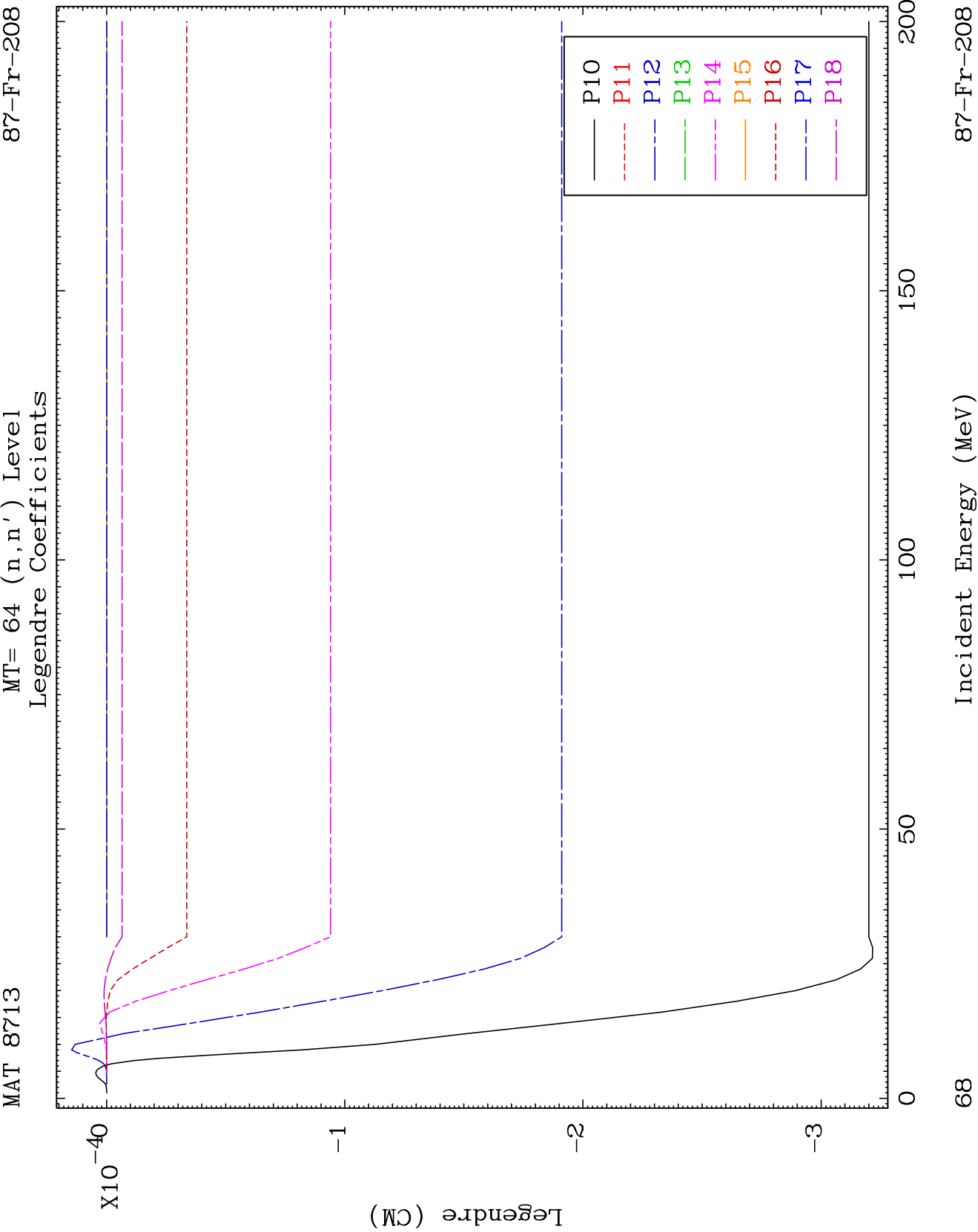
MAT 8713

MT= 63 (n, n') Level
Legendre Coefficients

87-Fr-208



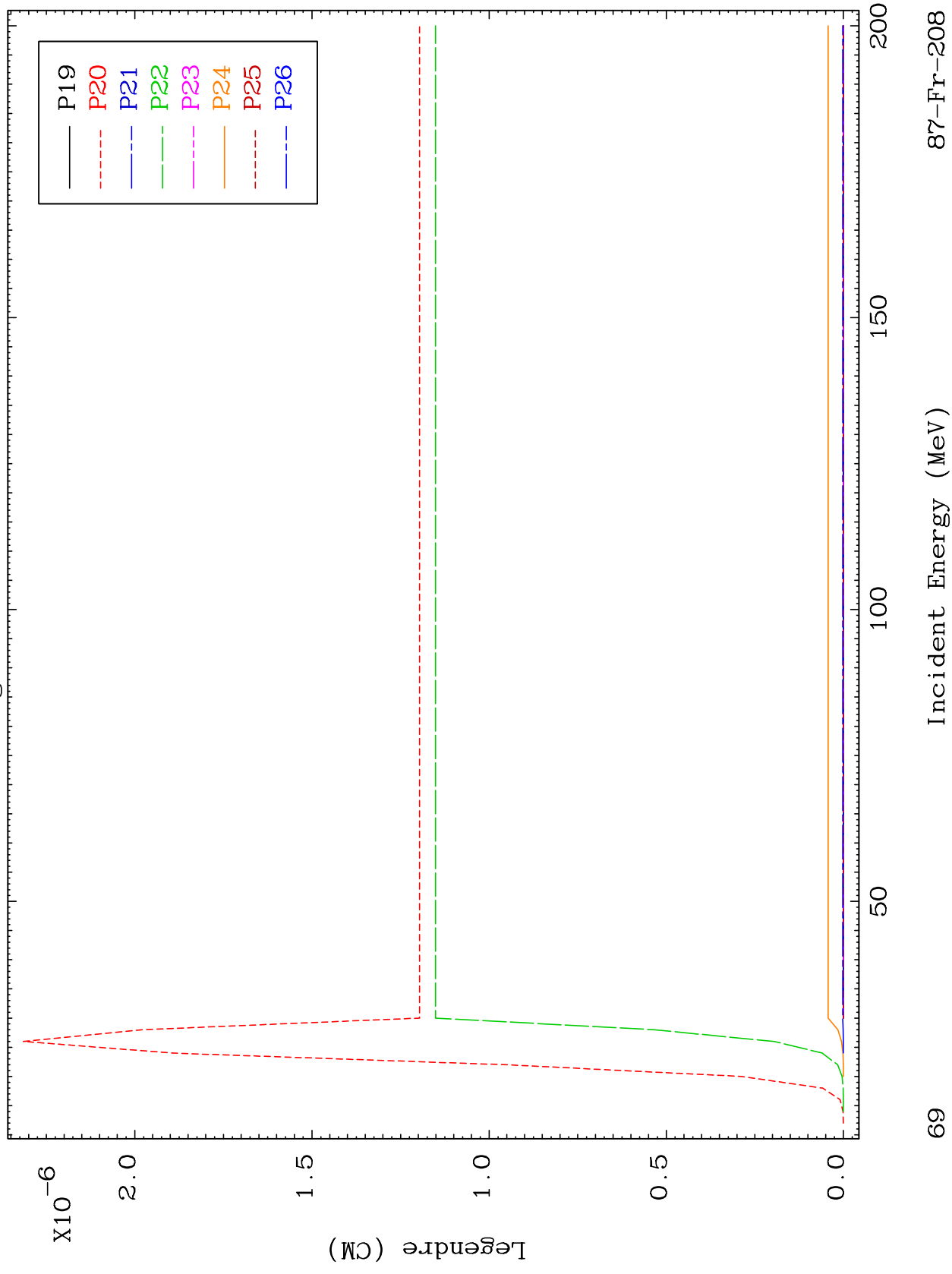


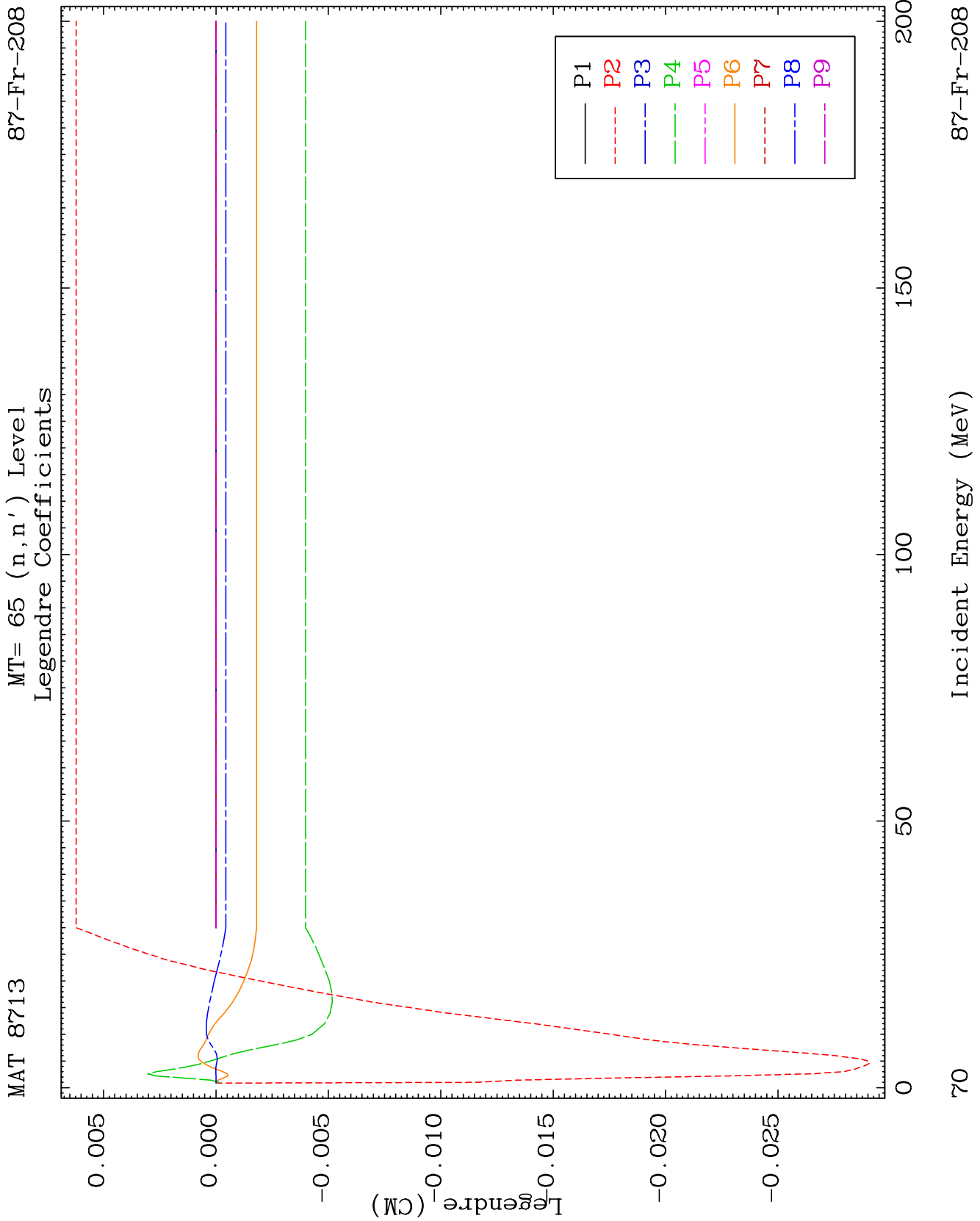


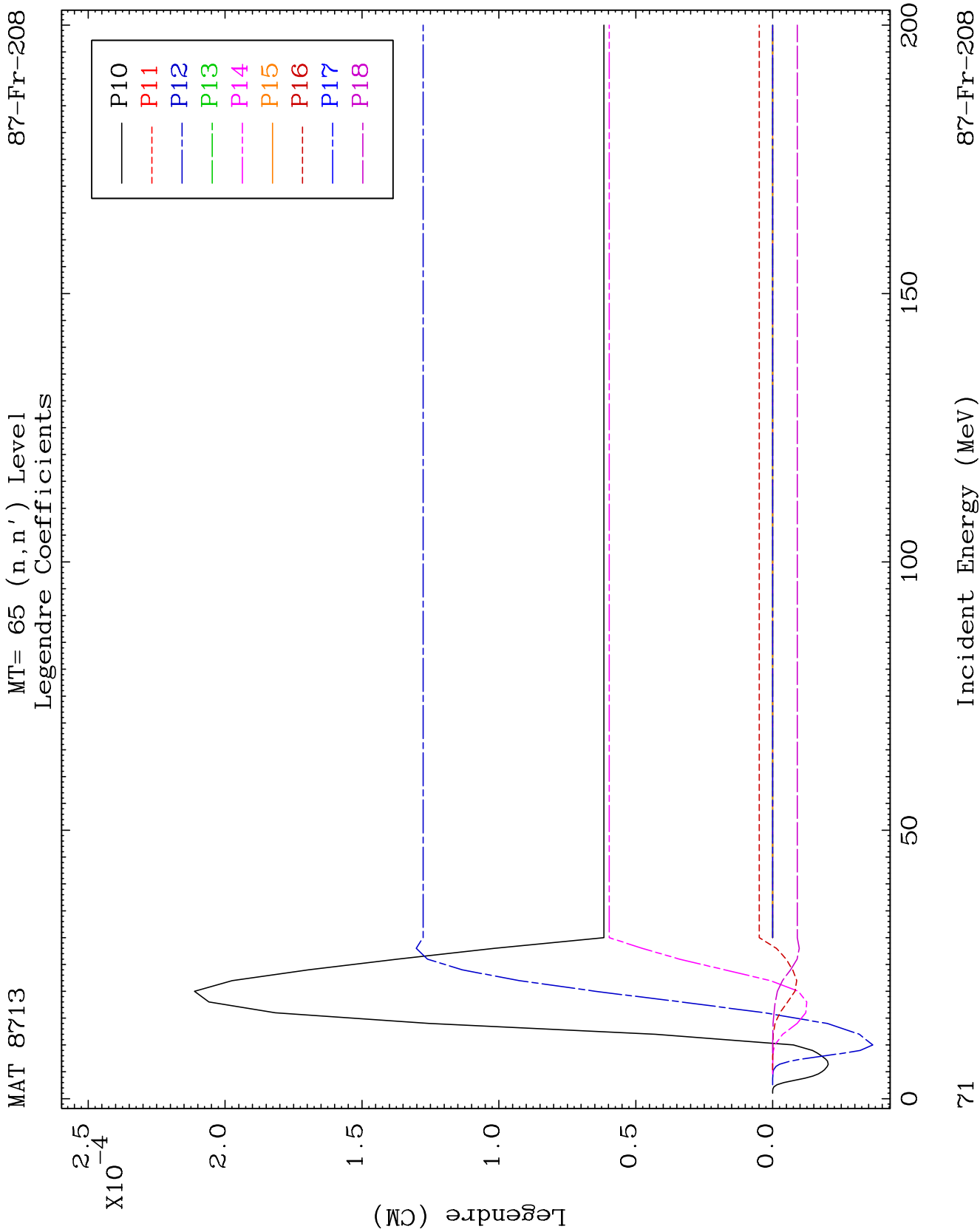
MAT 8713

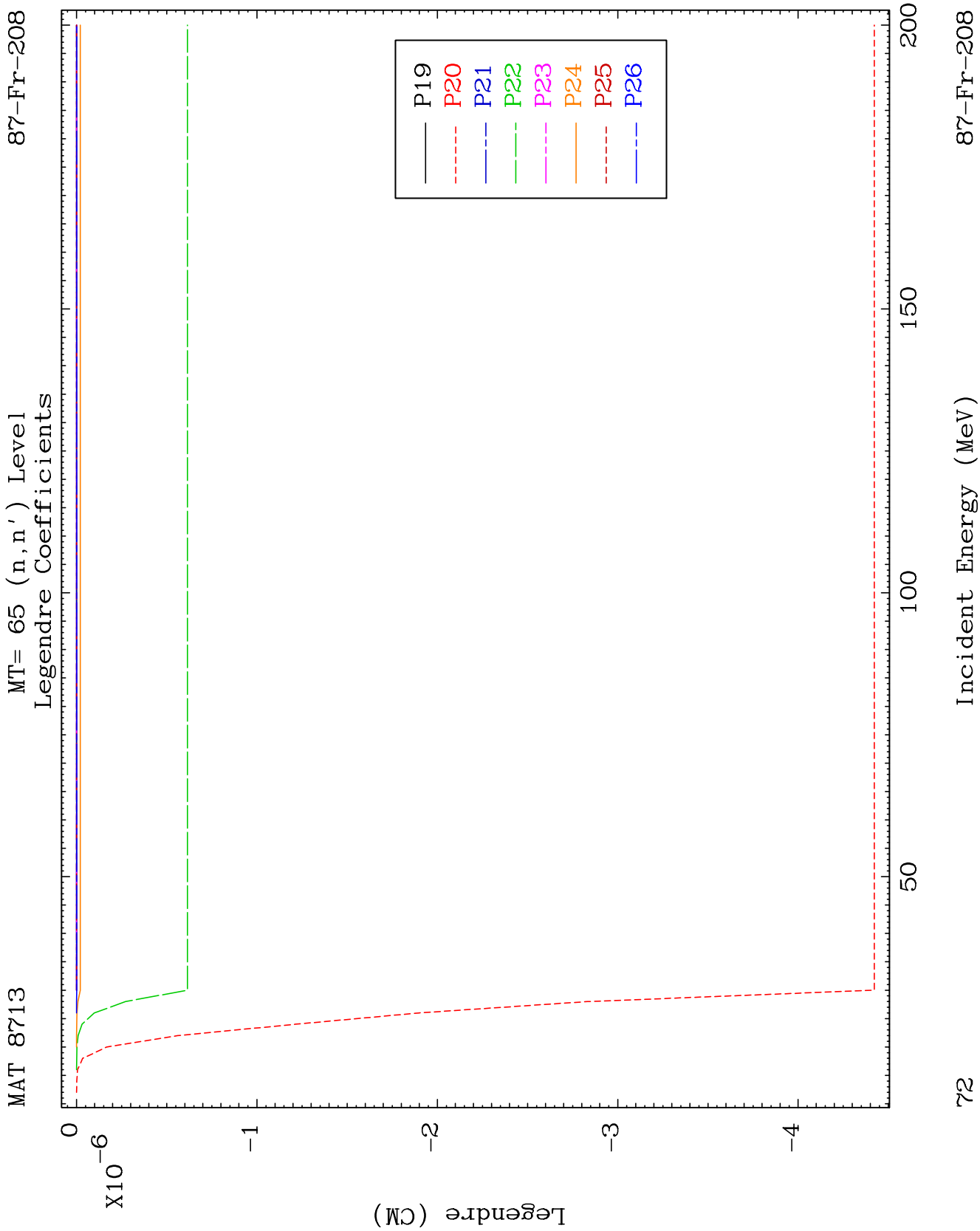
MT= 64 (n, n') Level
Legendre Coefficients

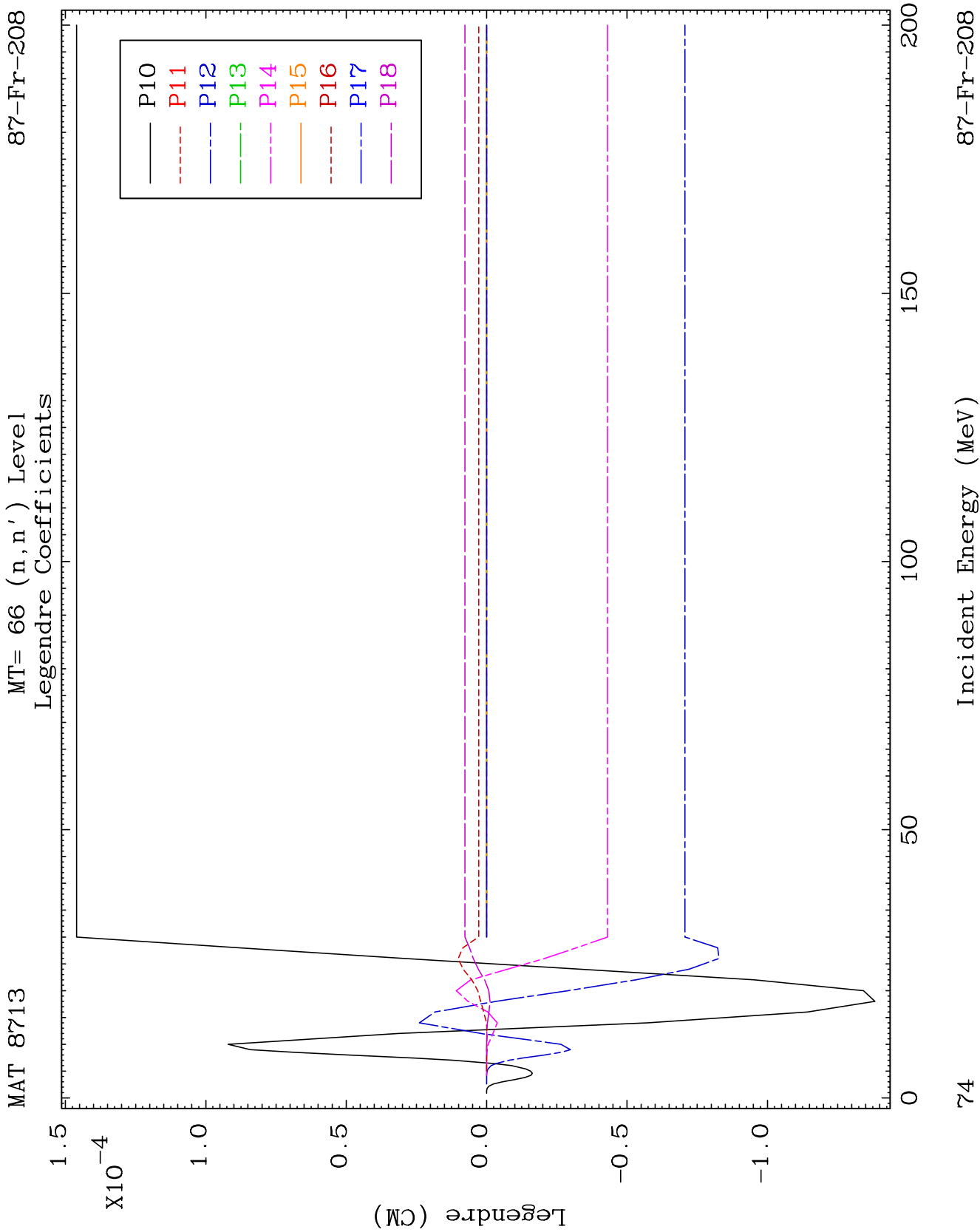
87-Fr-208

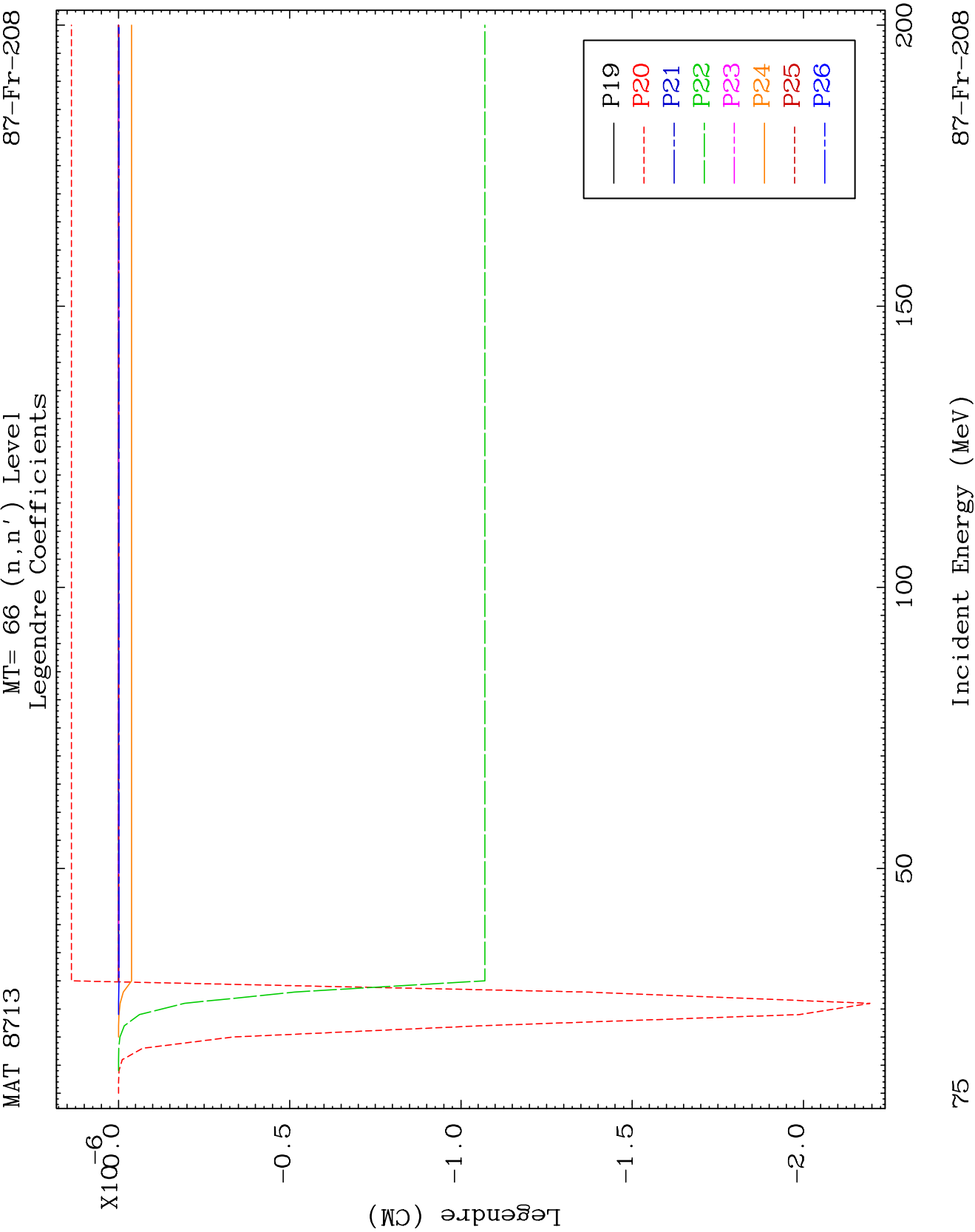


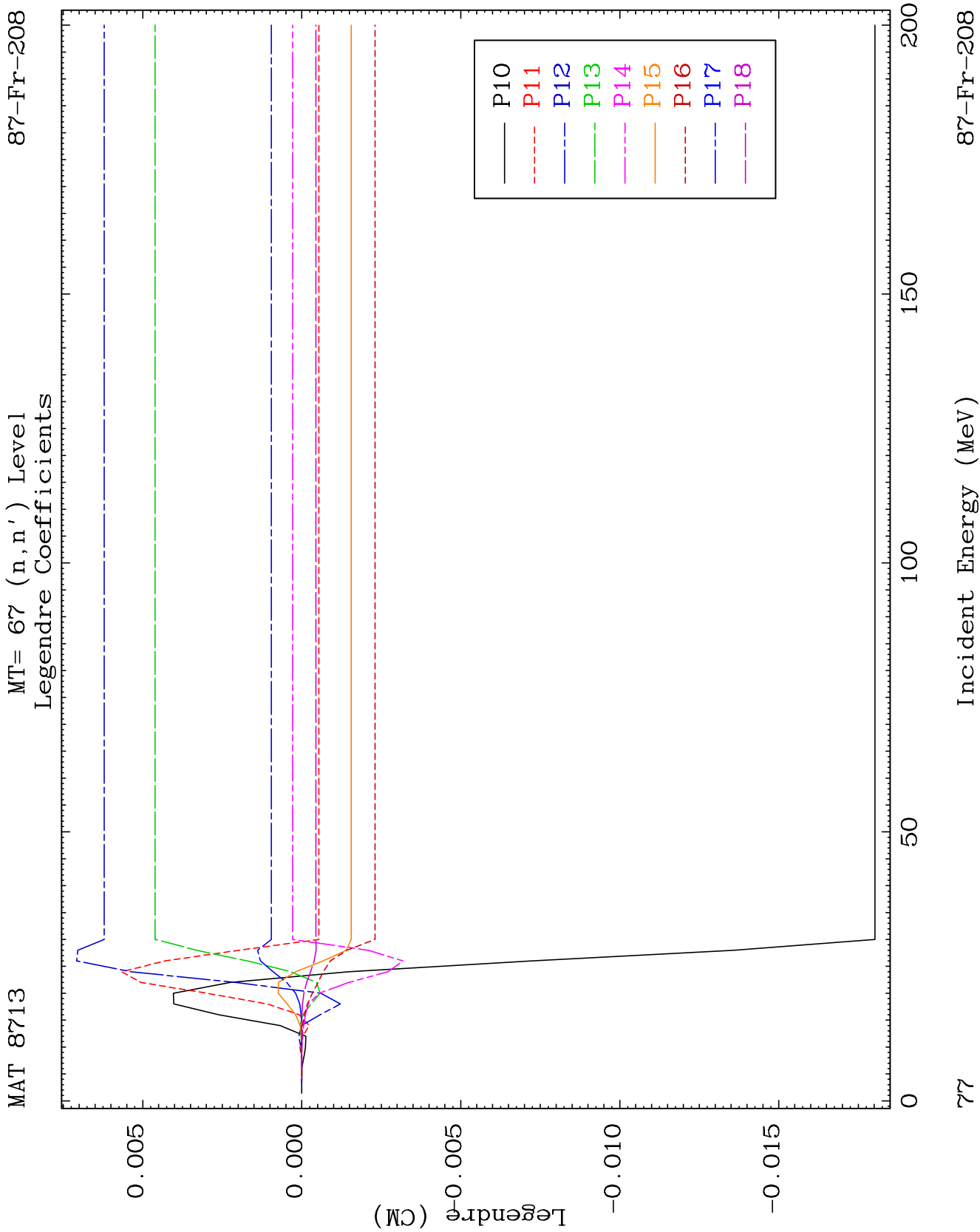


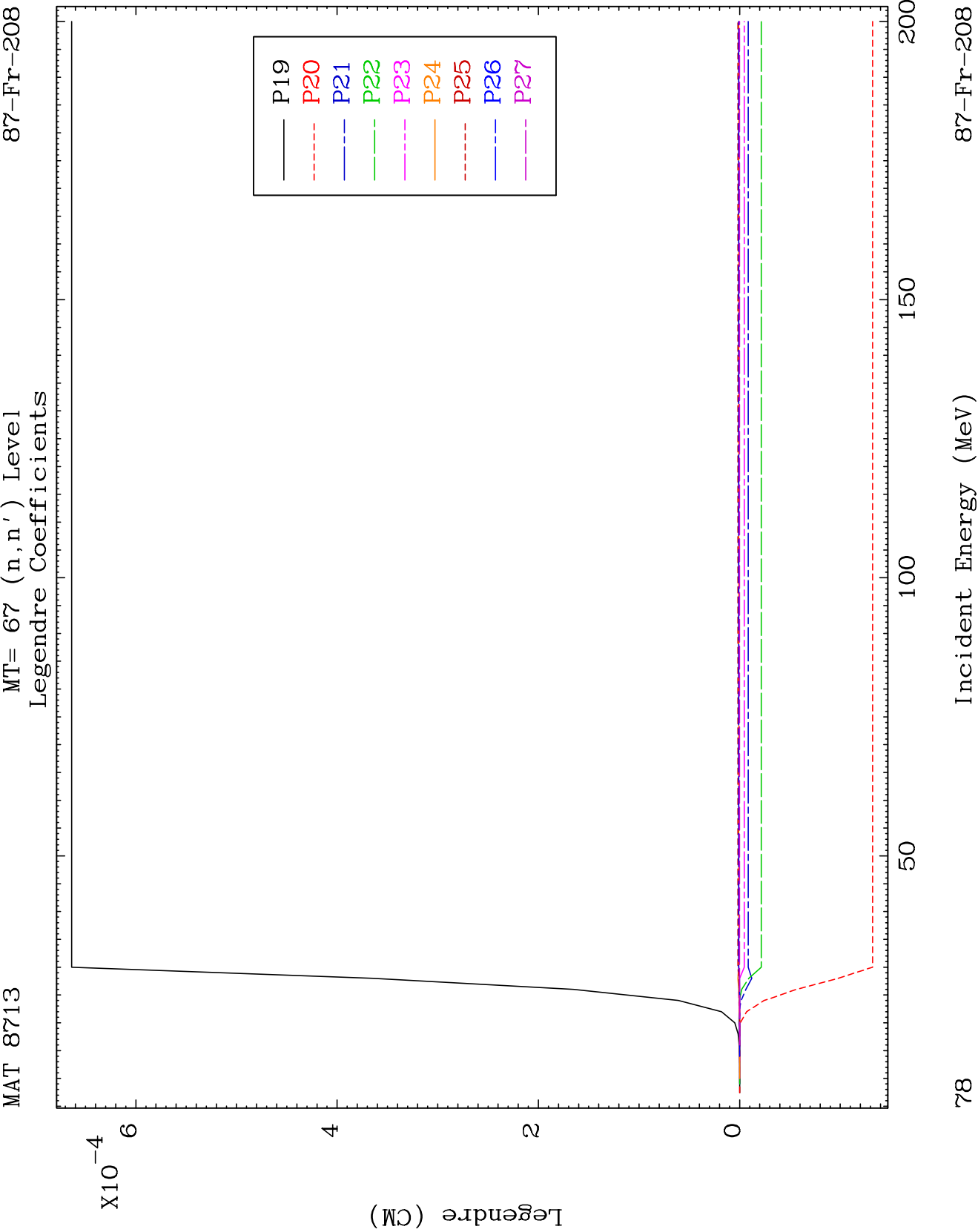


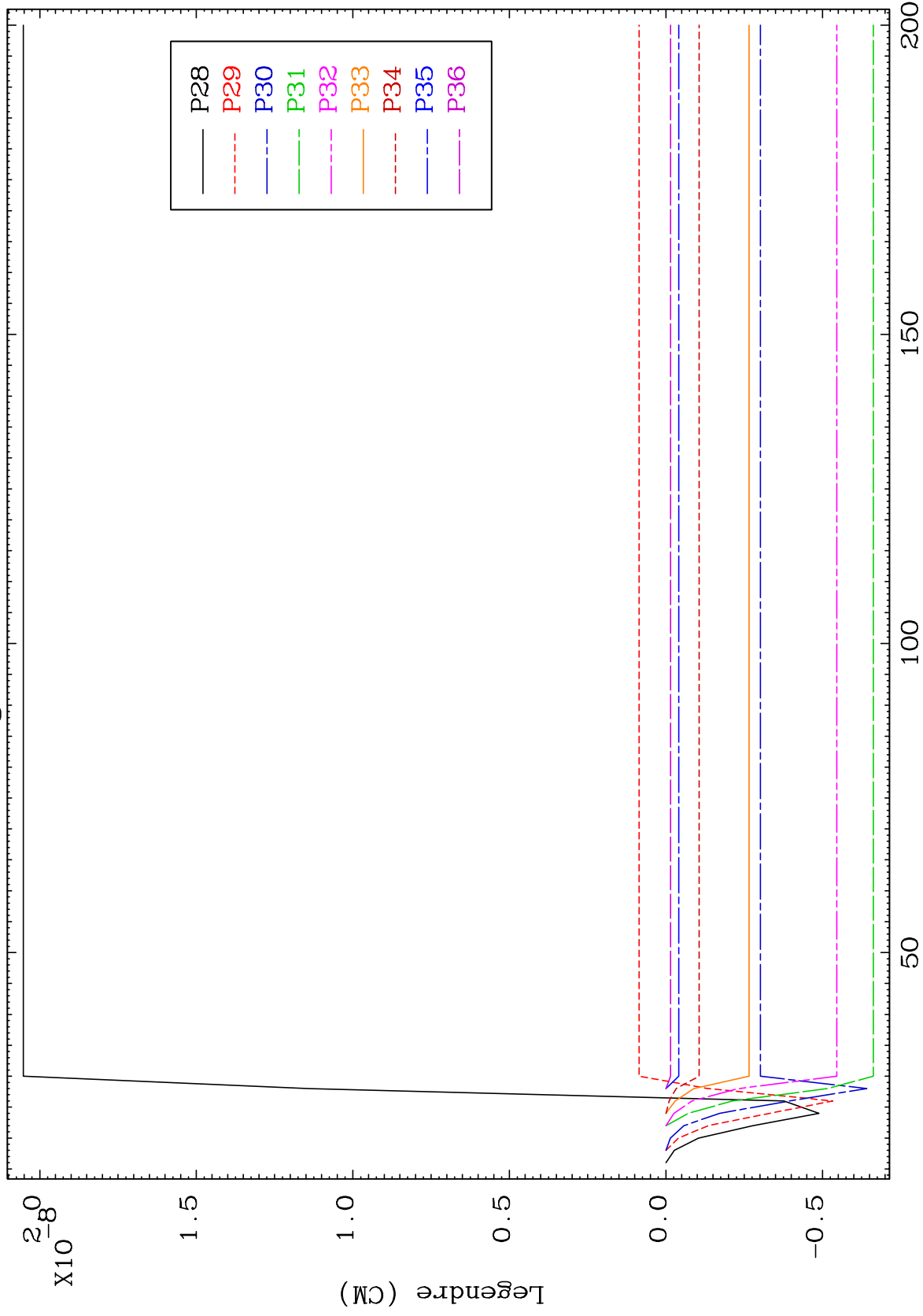


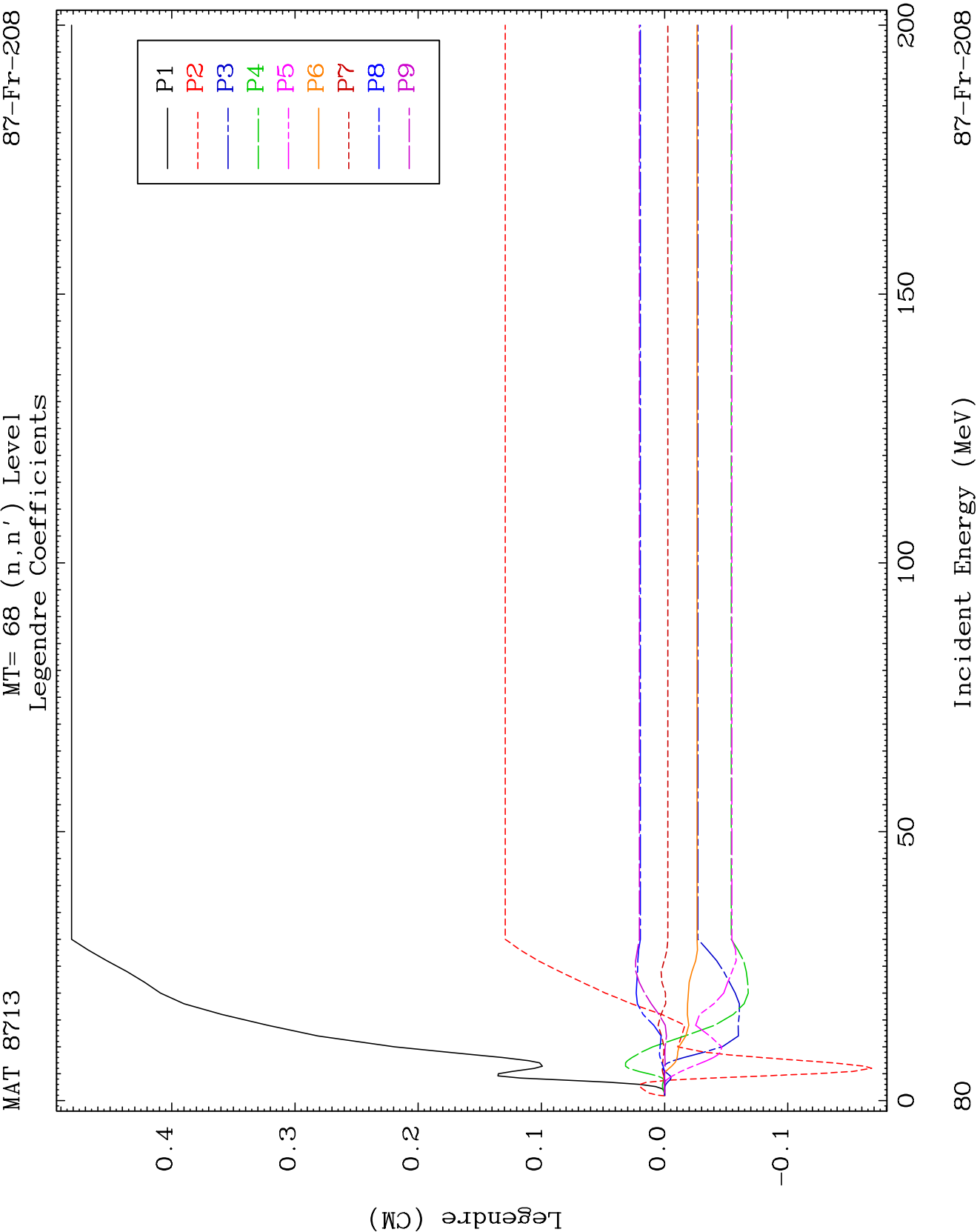


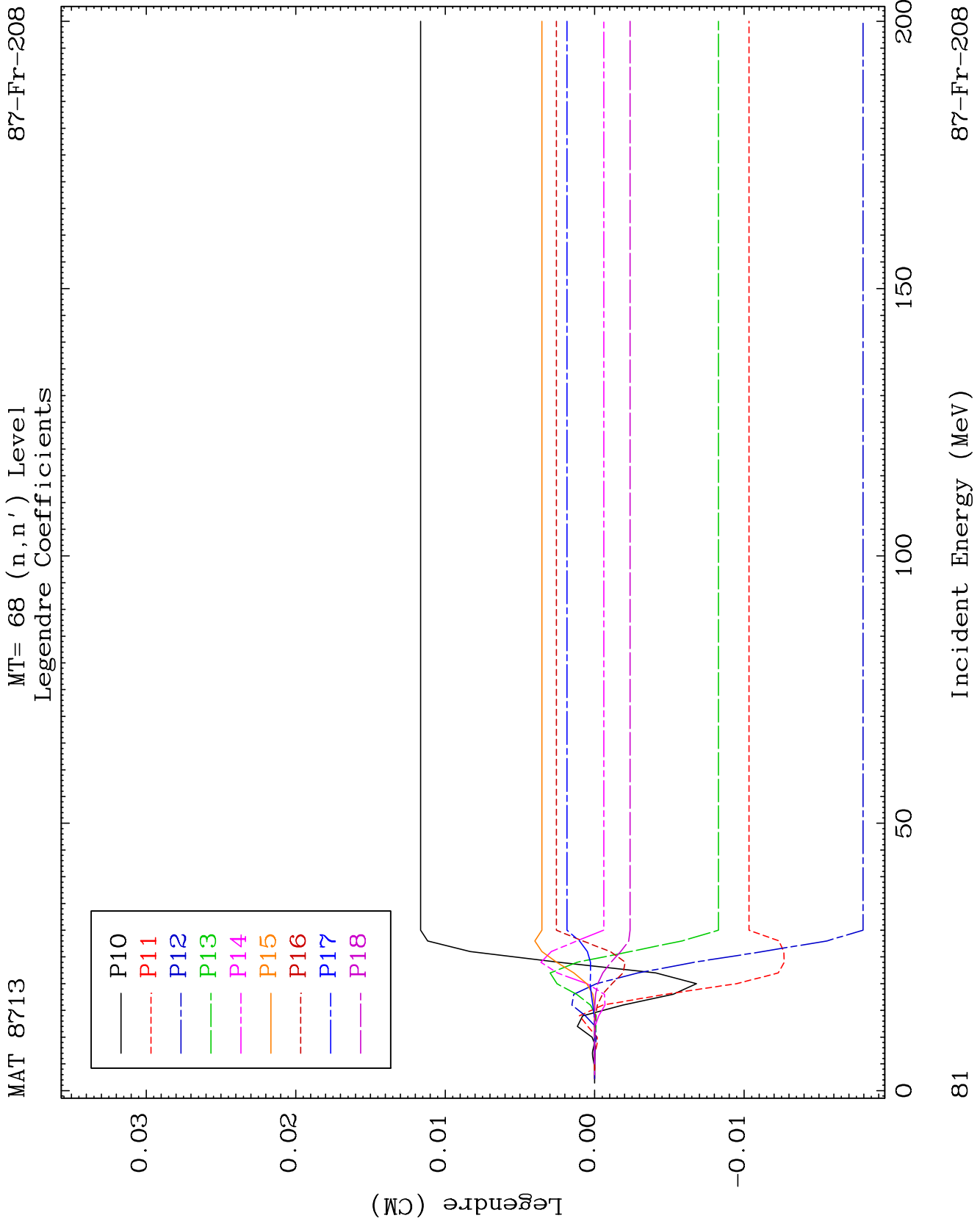


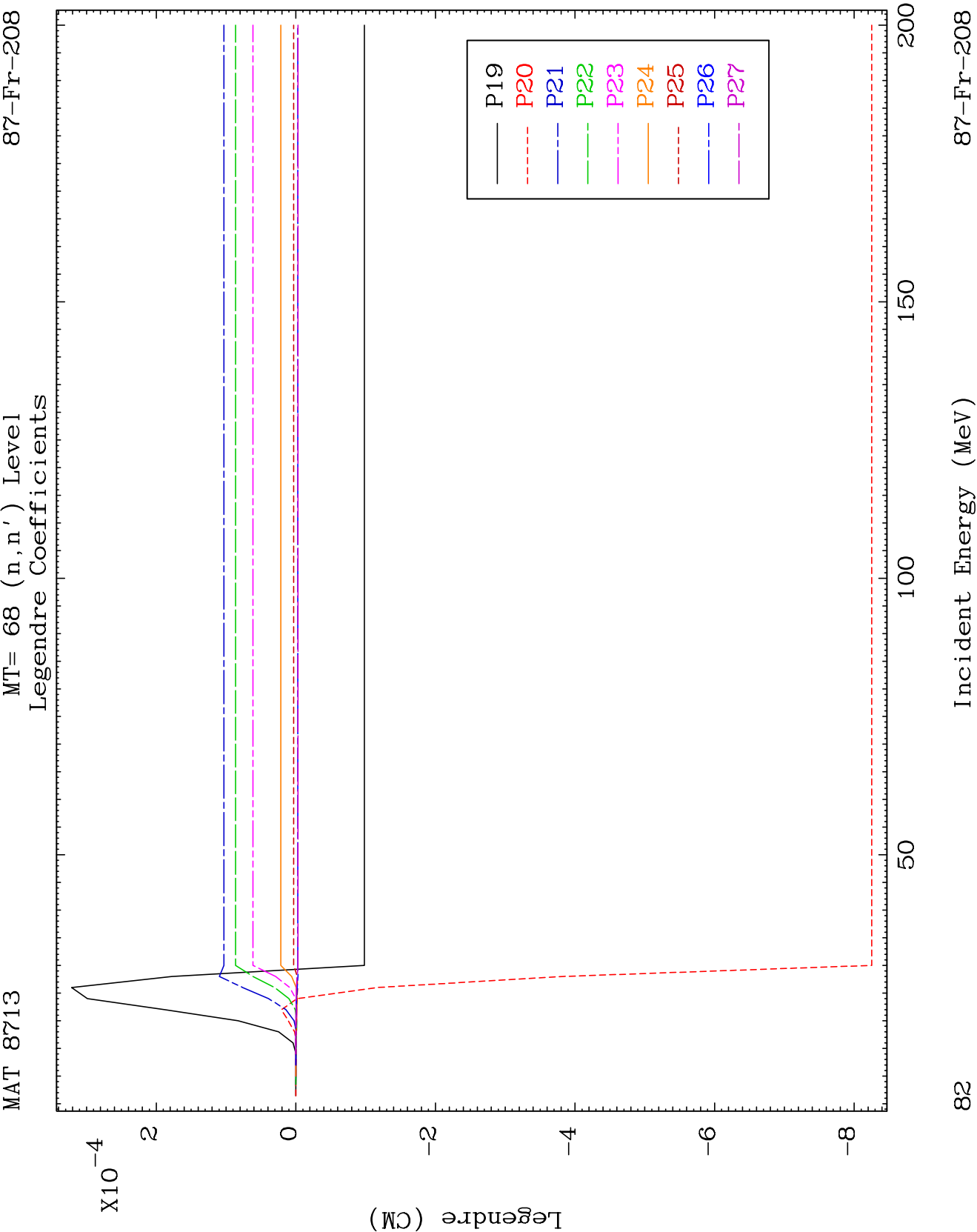


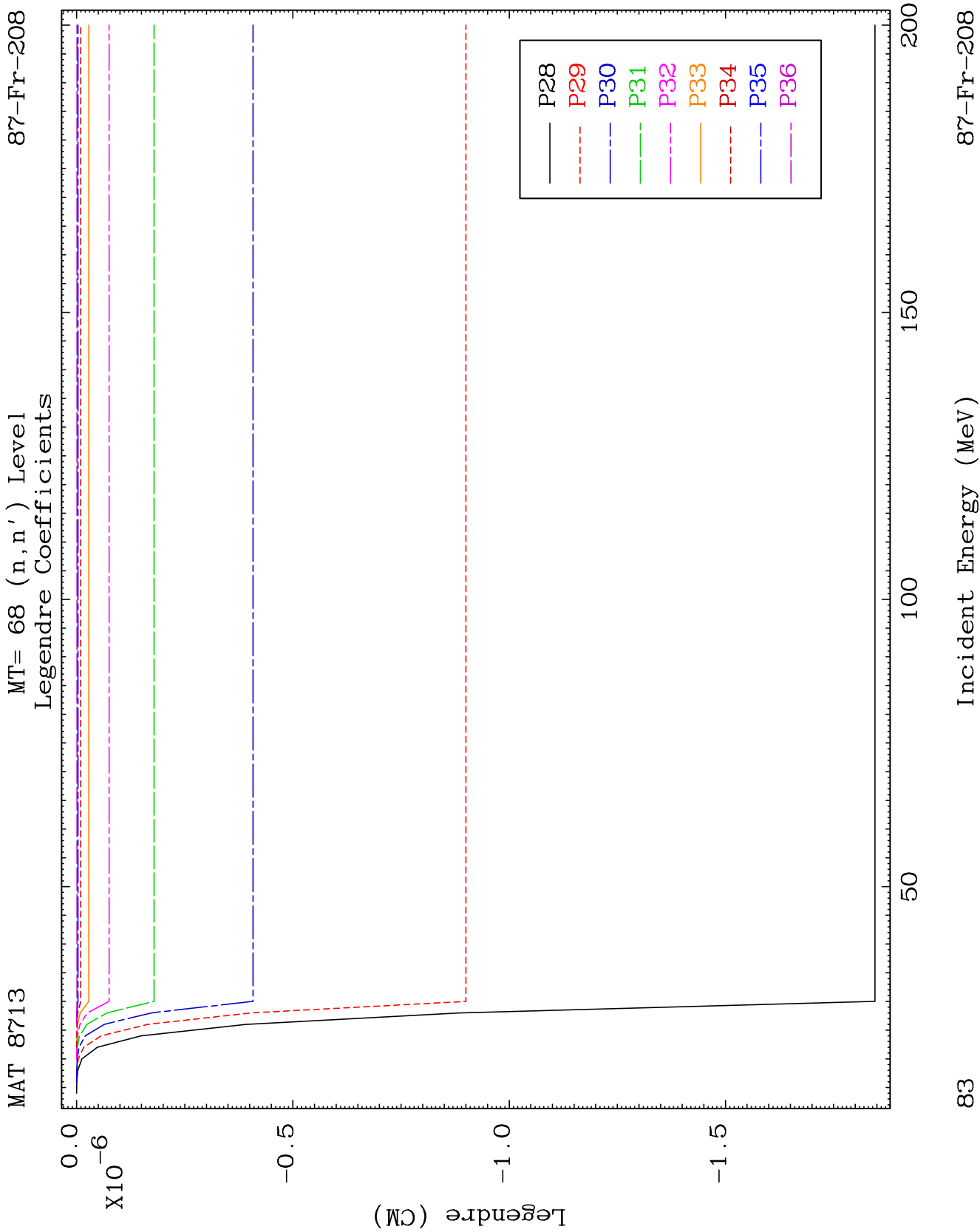


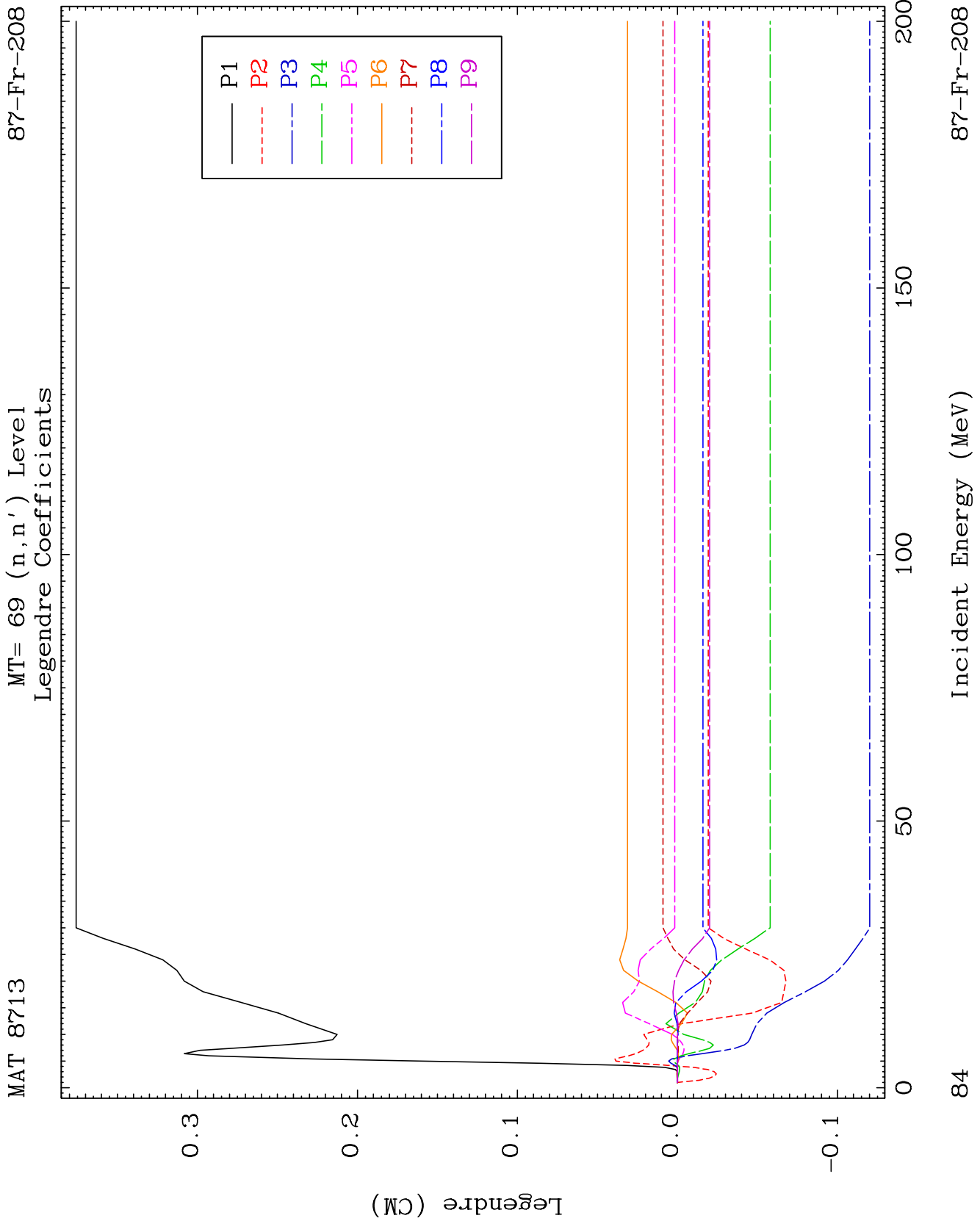


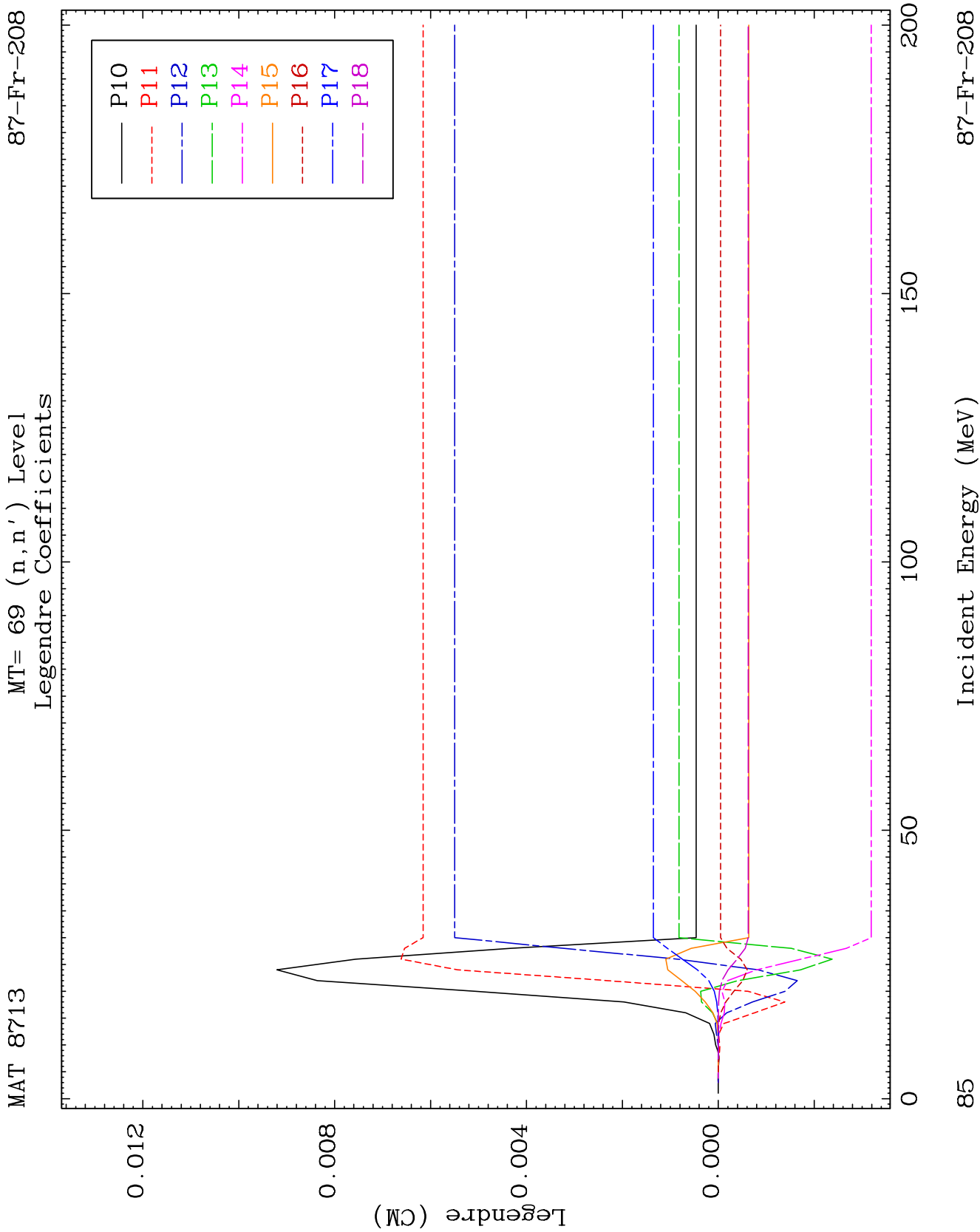


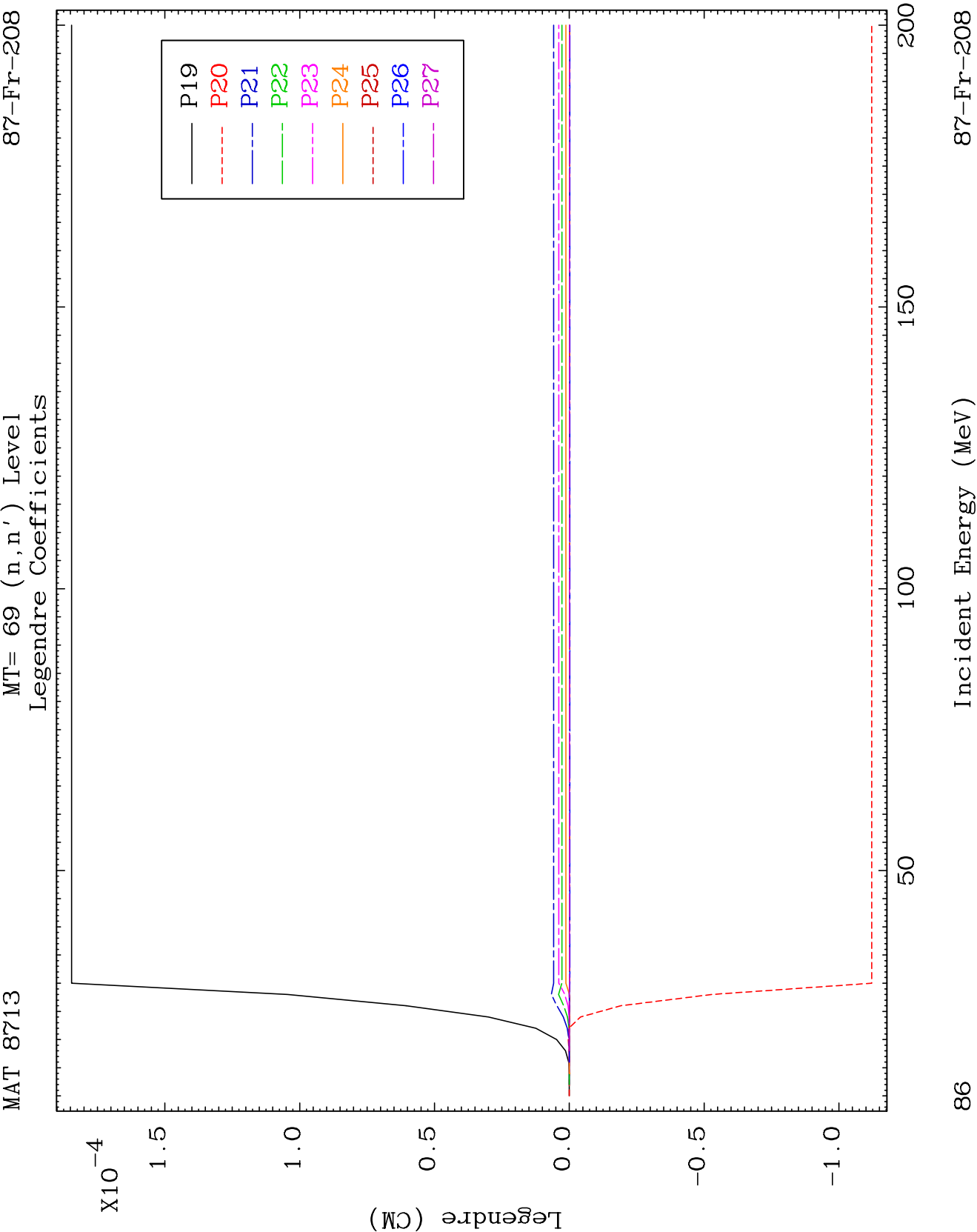


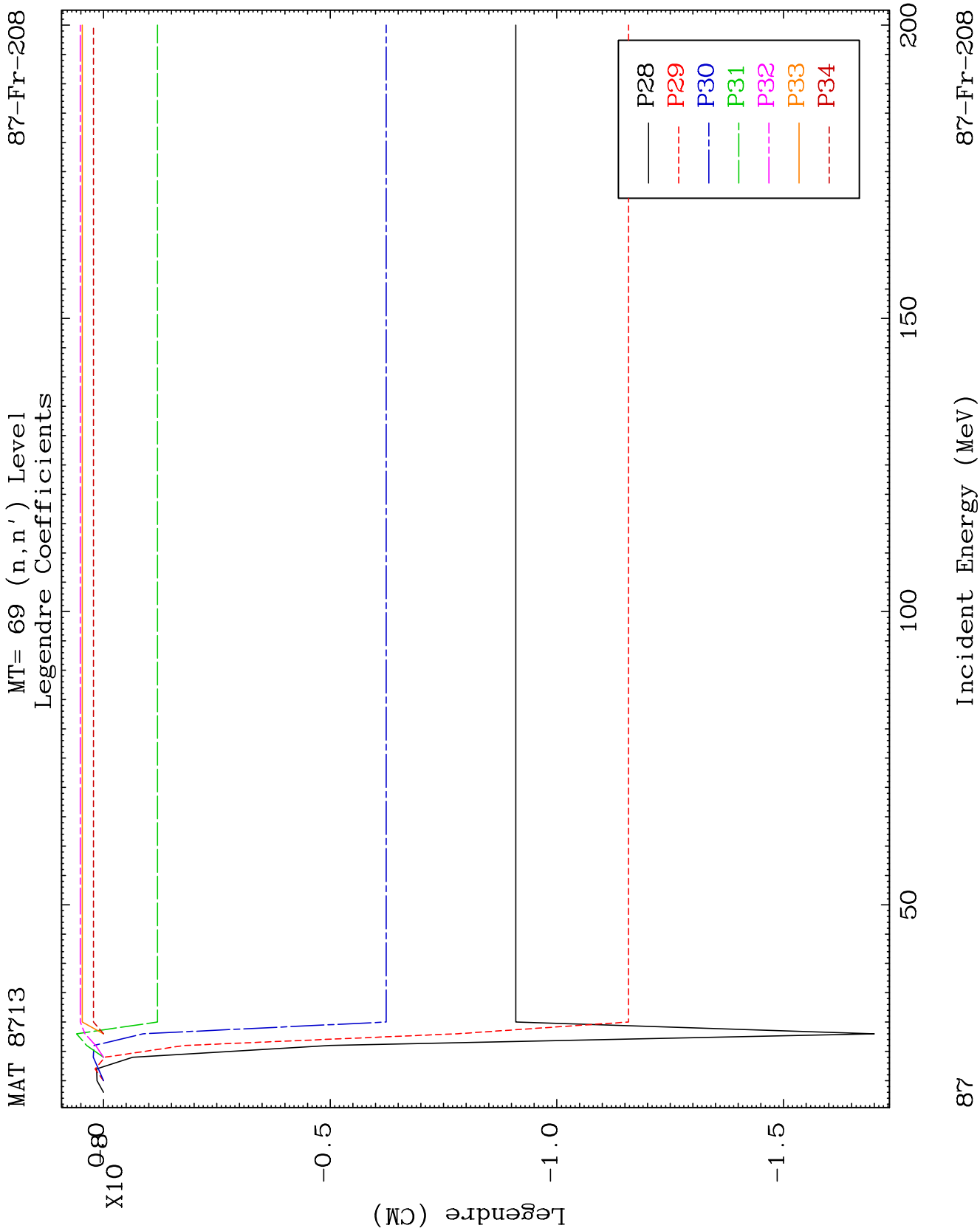


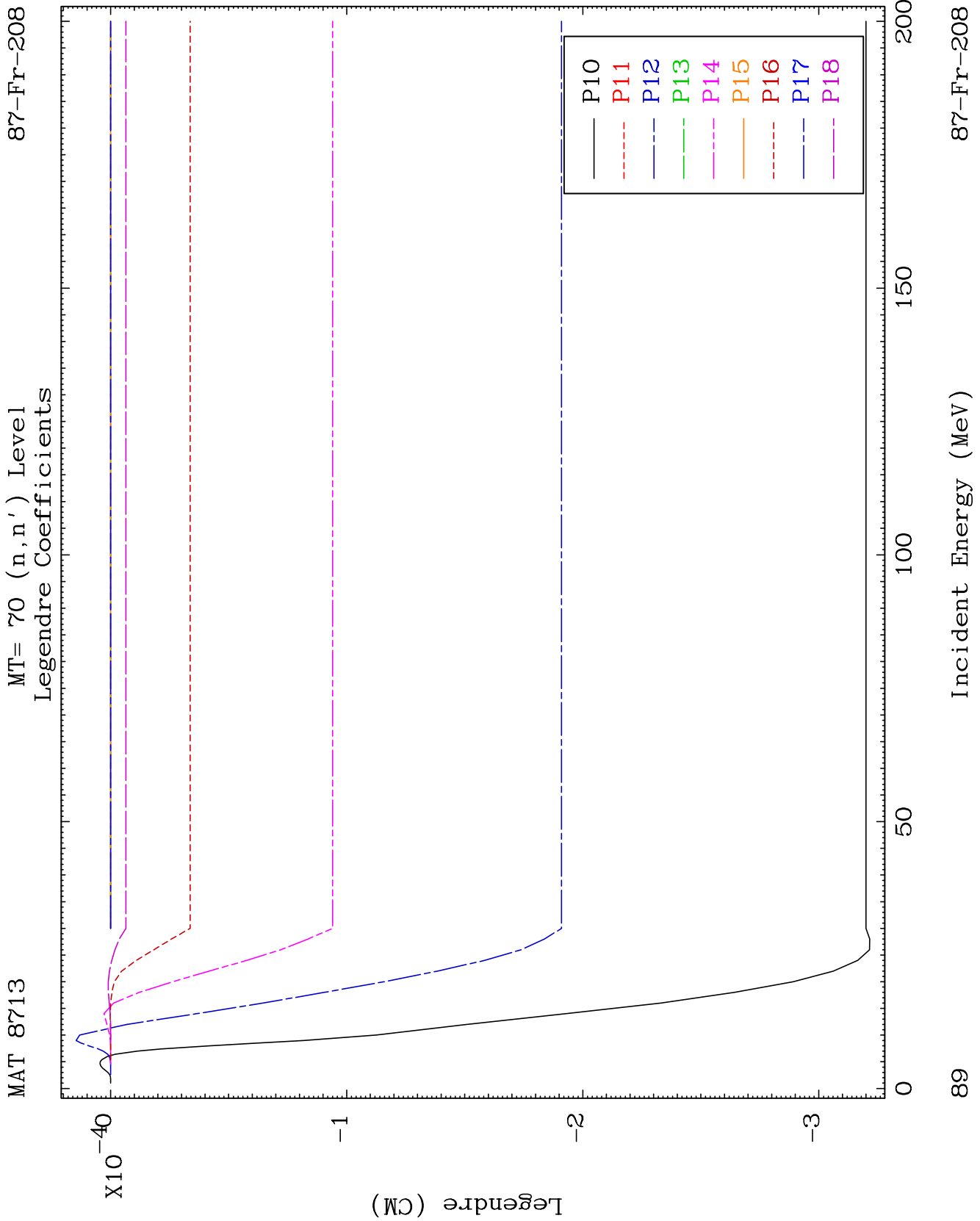


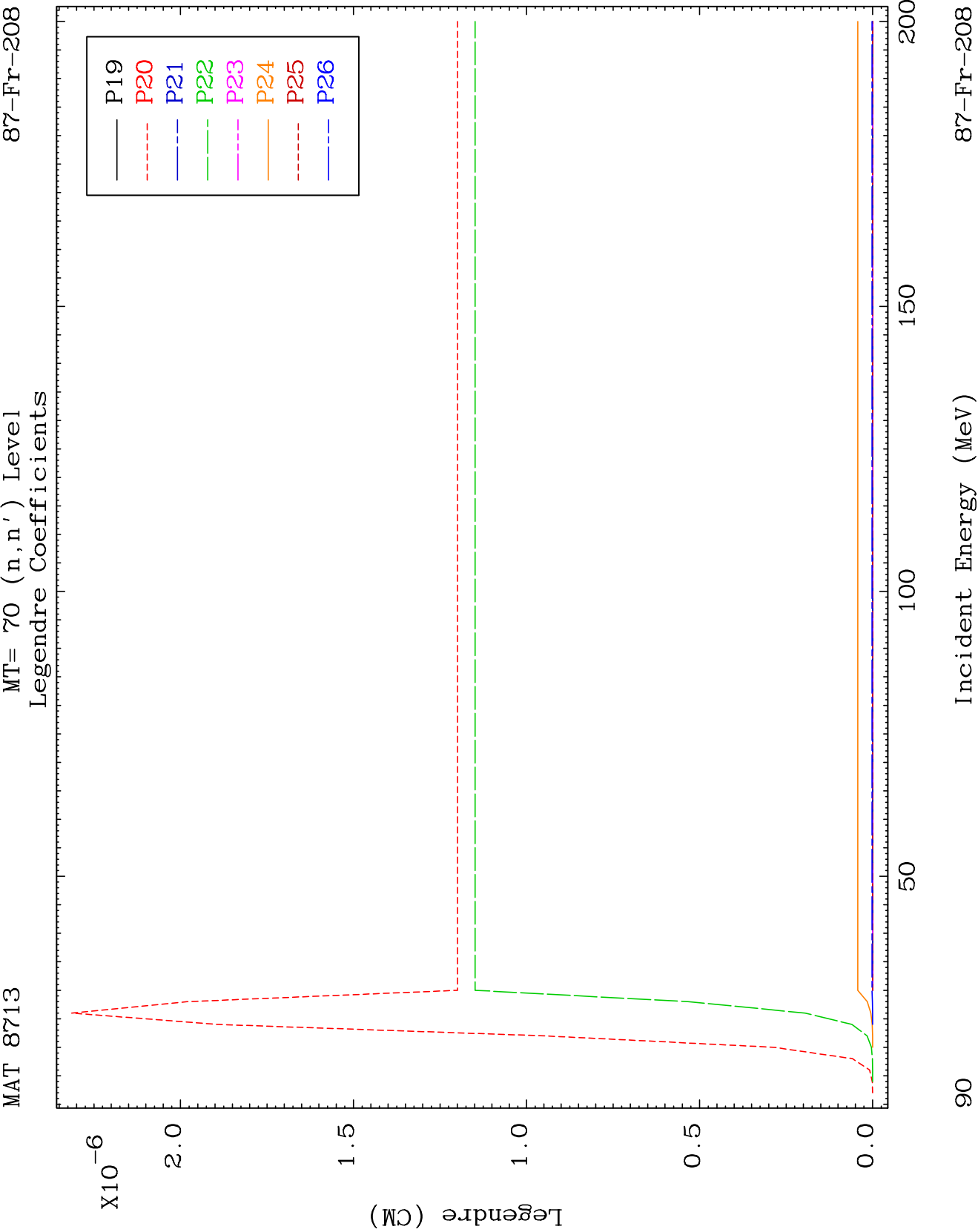








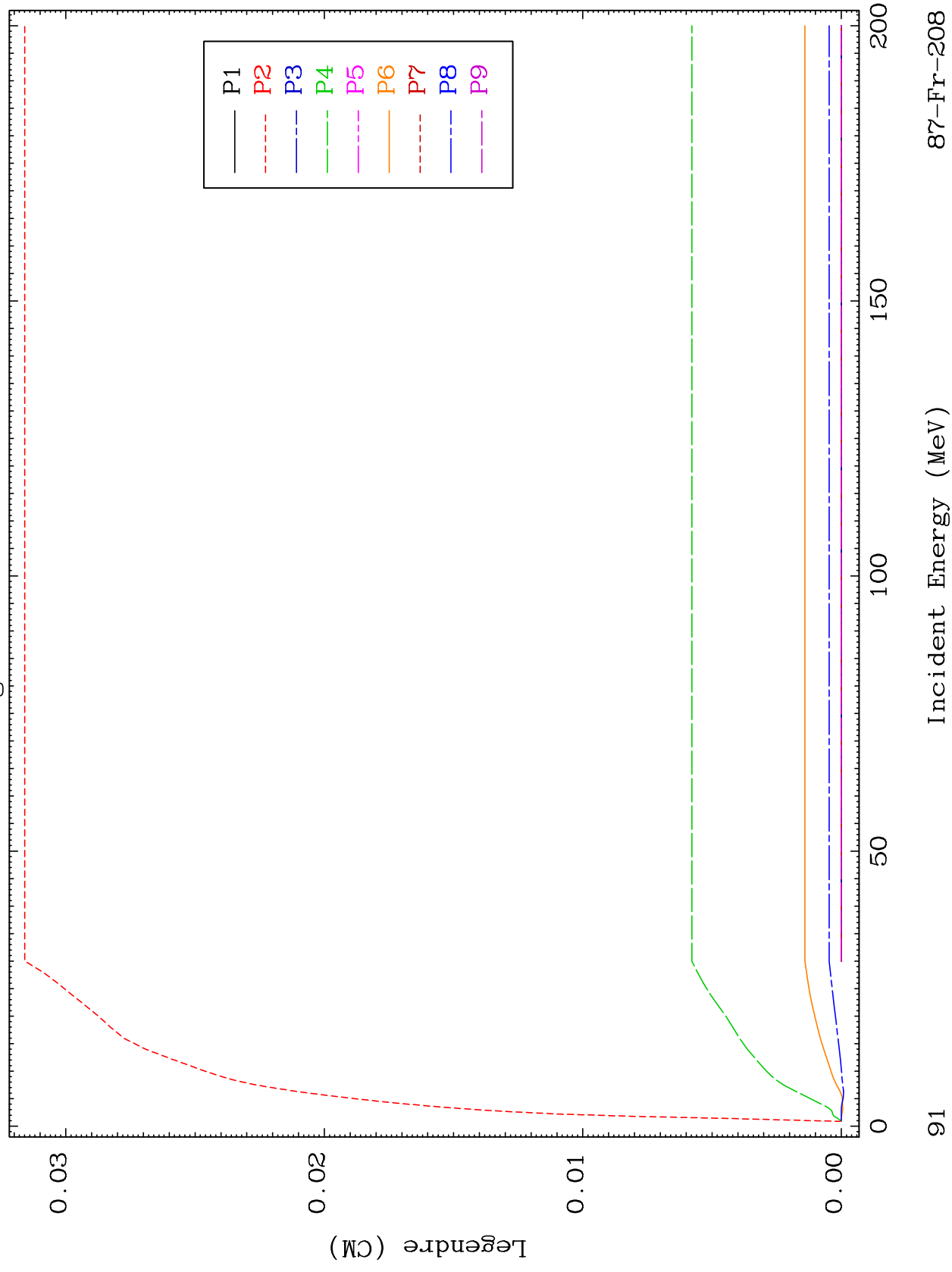


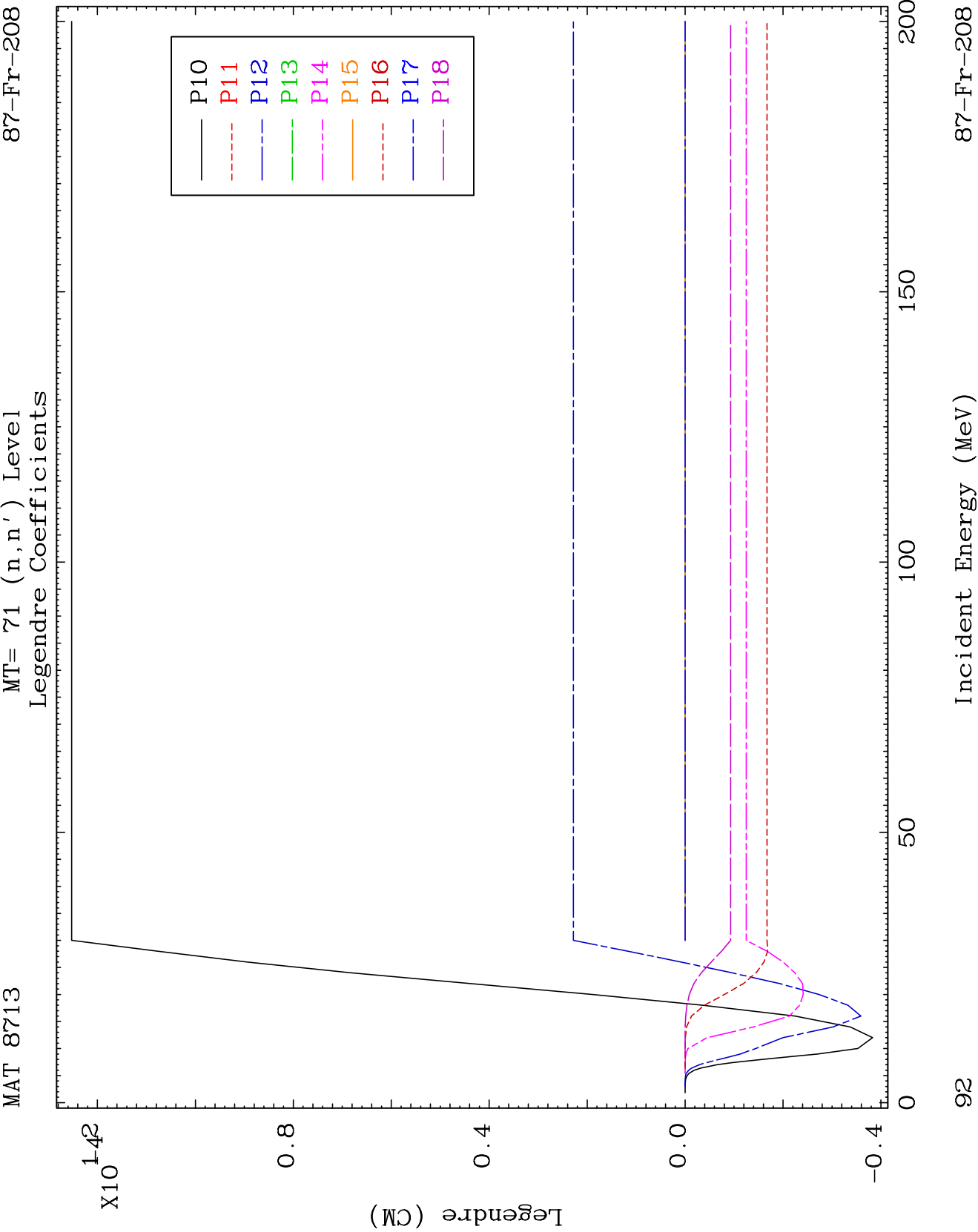


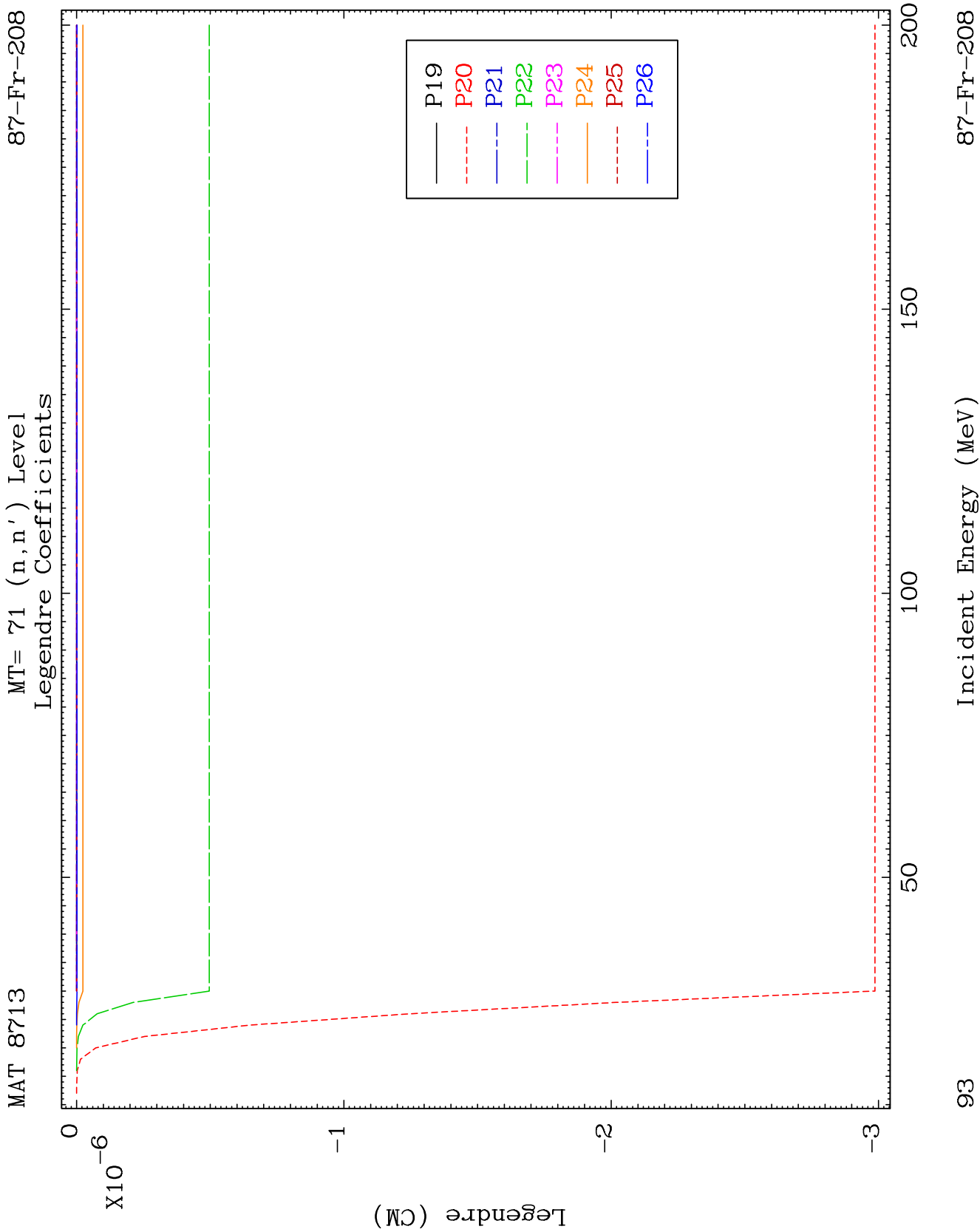
MAT 8713

MT= 71 (n, n') Level
Legendre Coefficients

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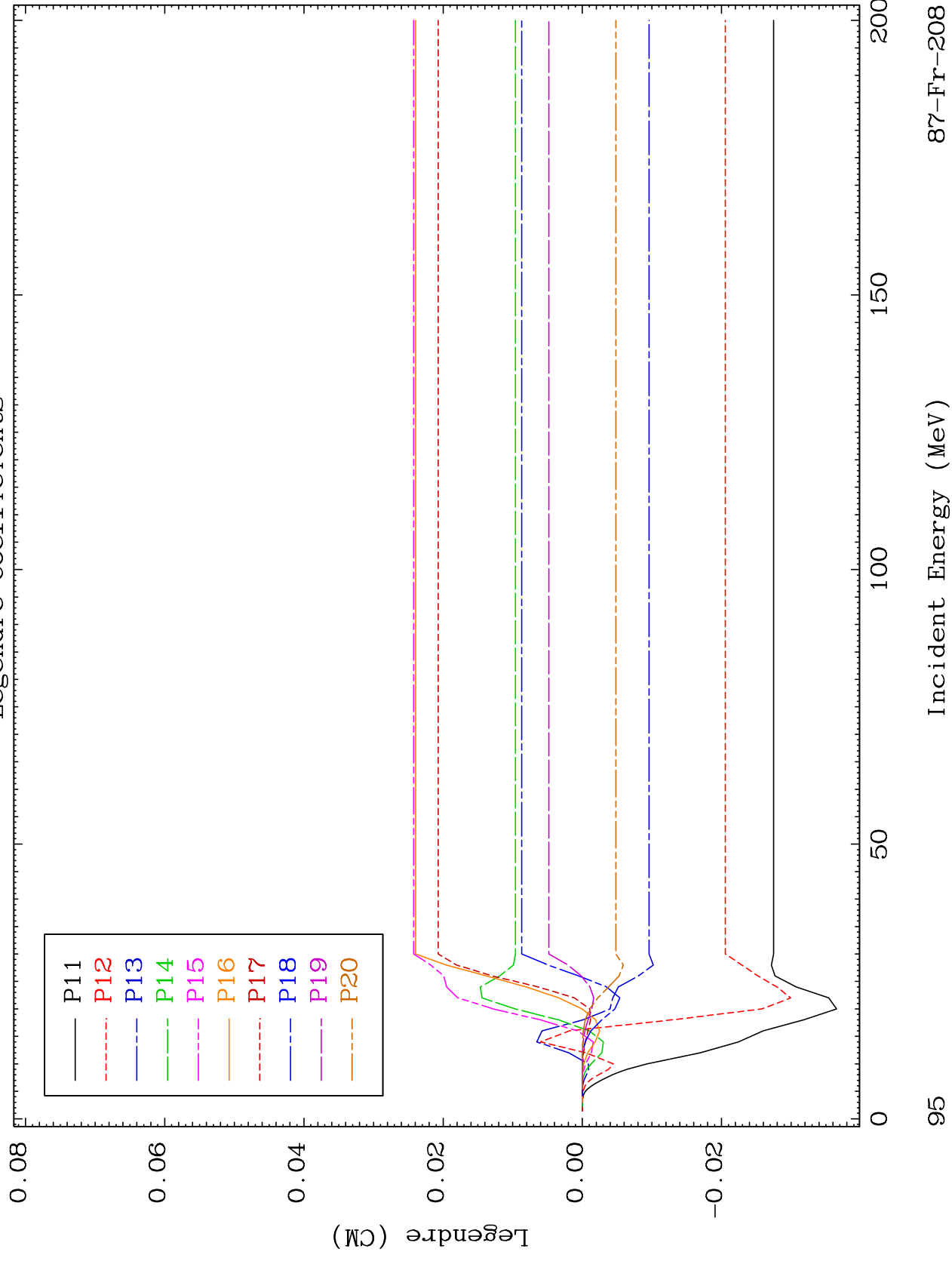




MAT 8713

MT= 72 (n, n') Level
Legendre Coefficients

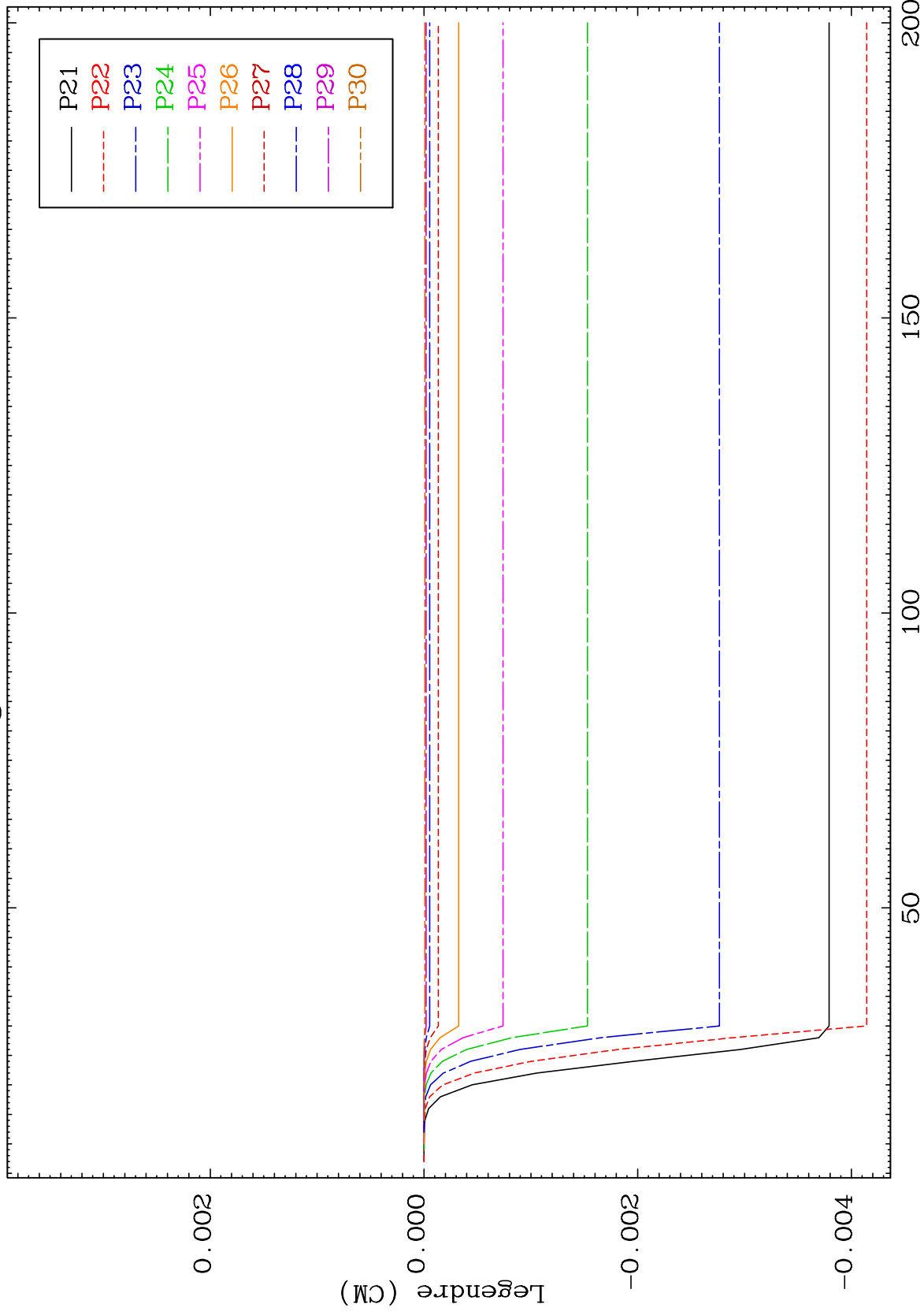
87-Fr-208



MAT 8713

MT= 72 (n,n') Level
Legendre Coefficients

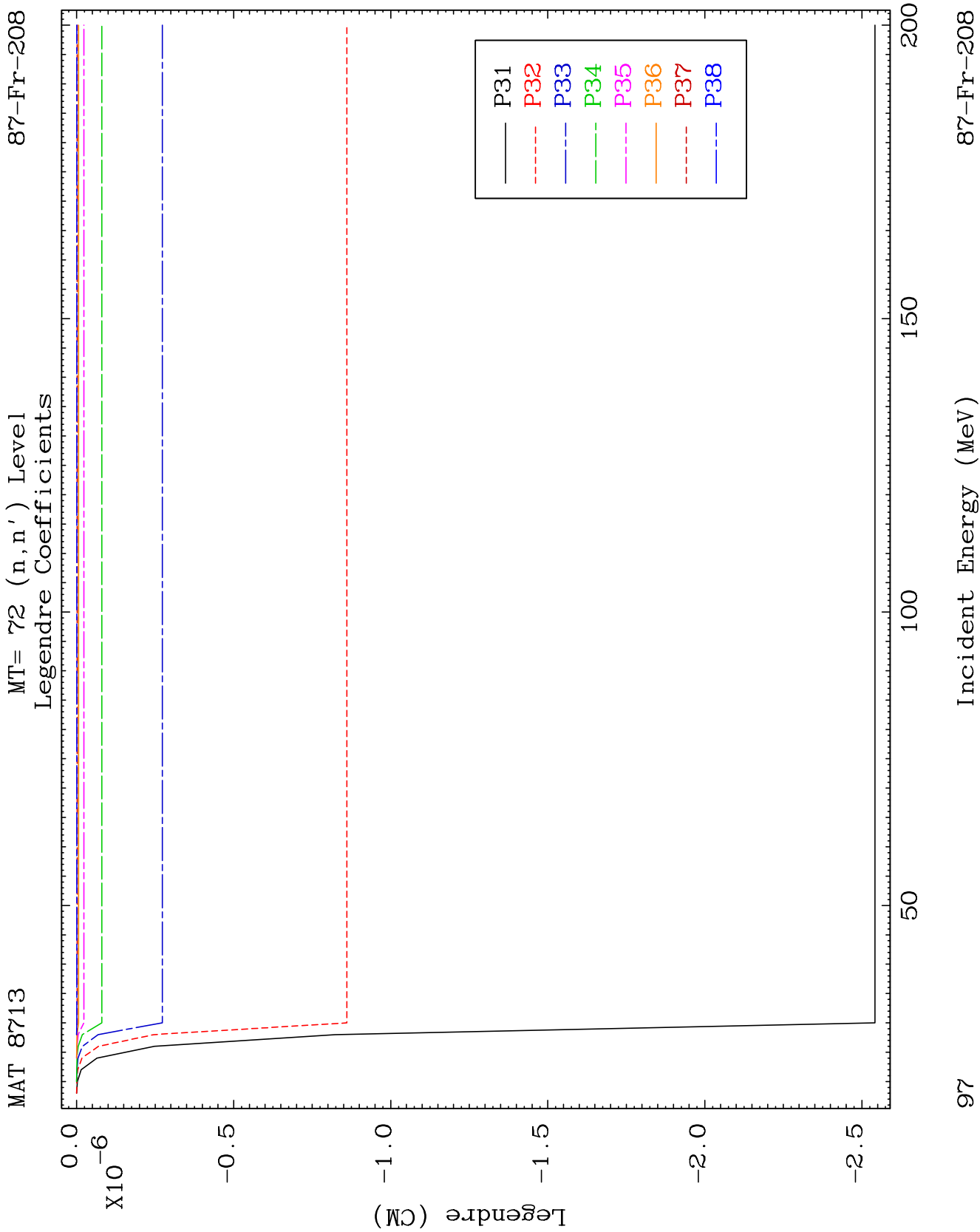
87-Fr-208

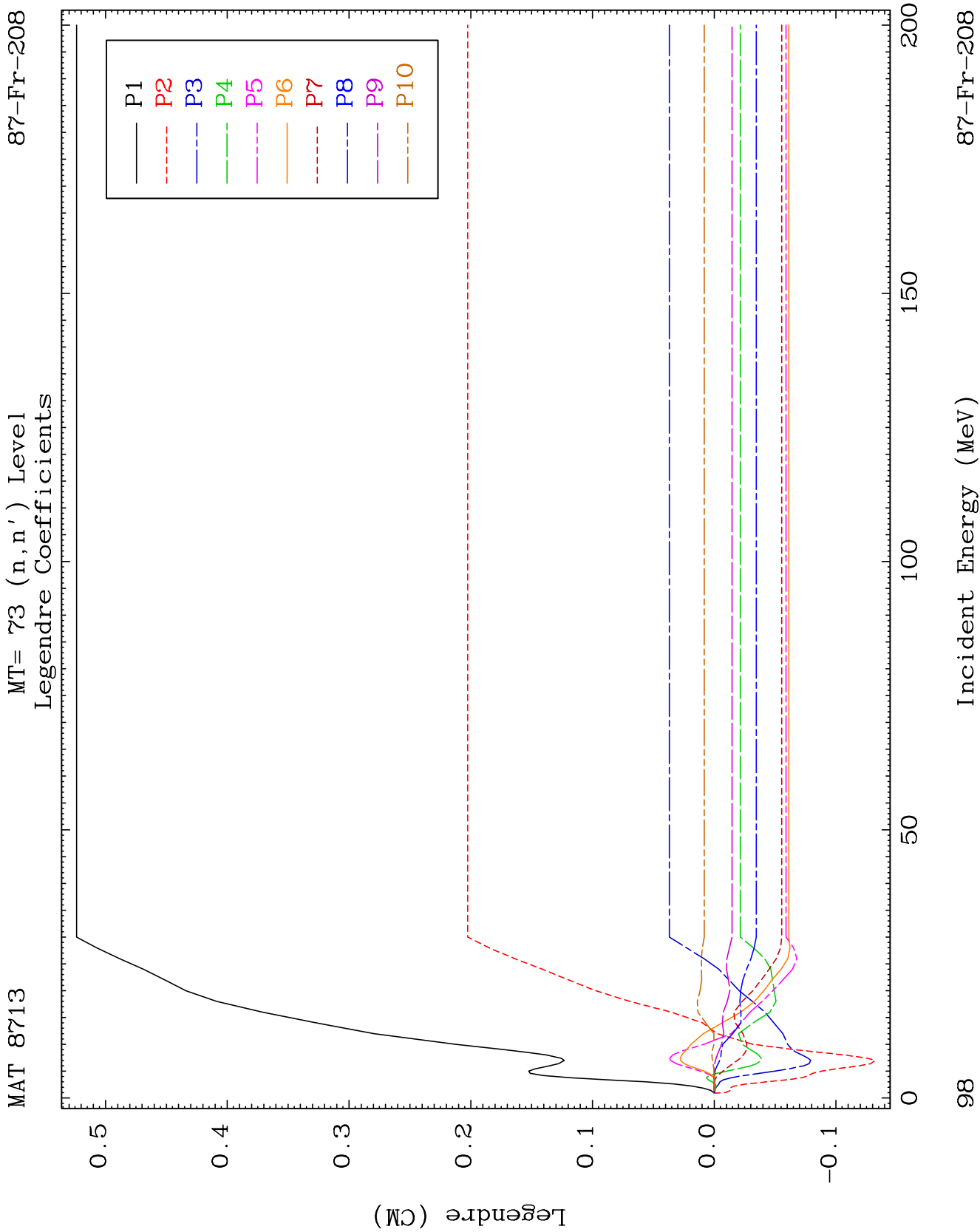


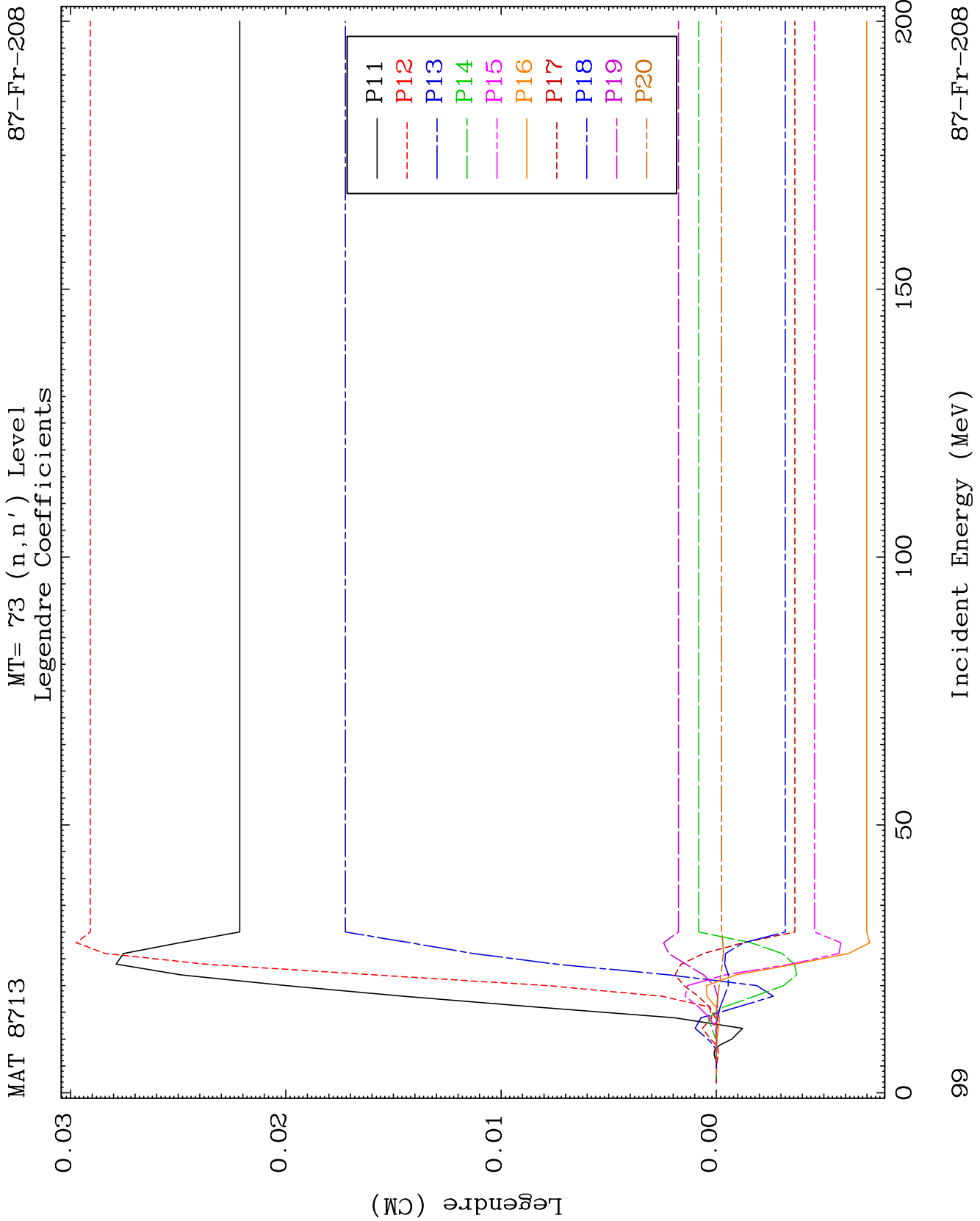
96

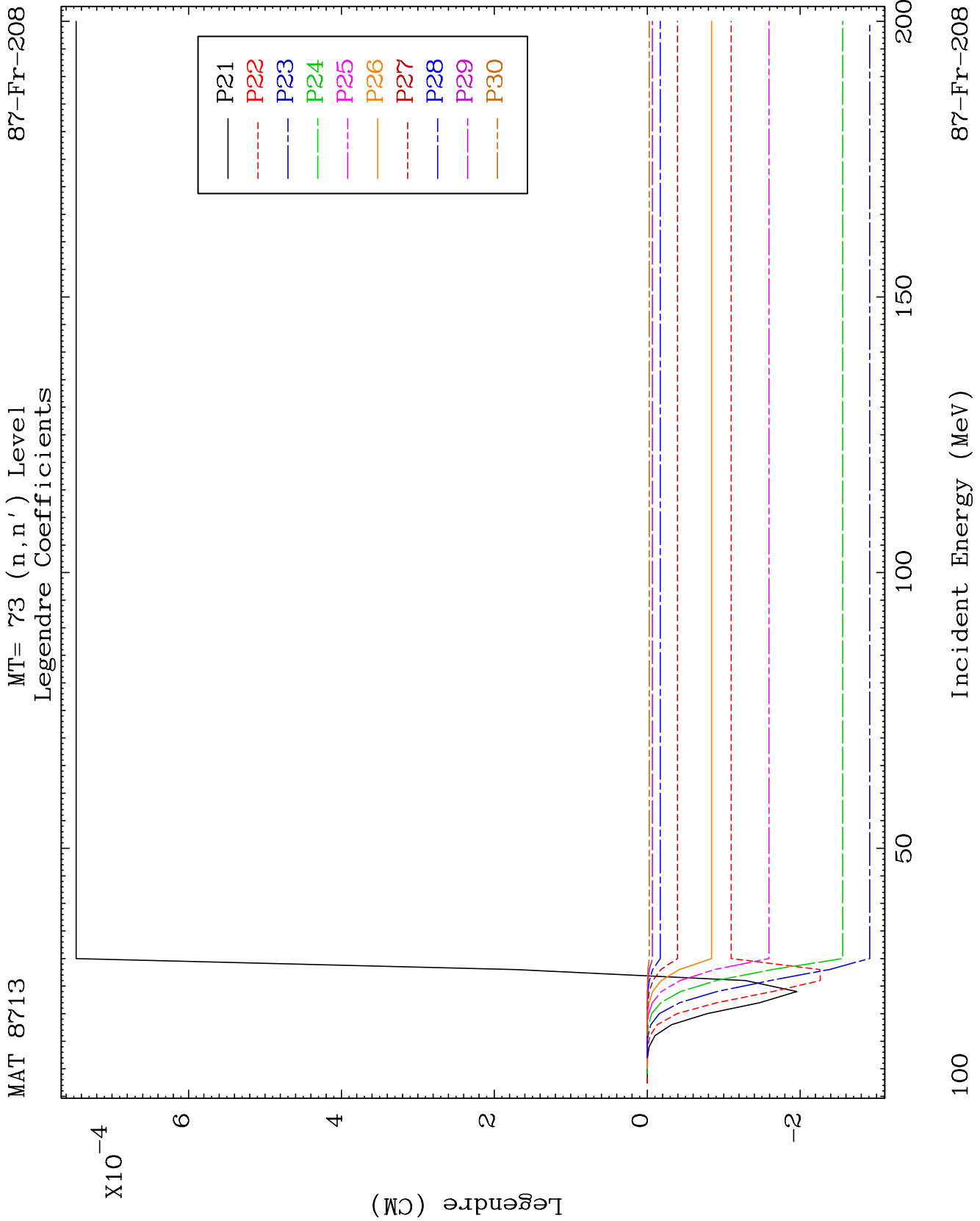
Incident Energy (MeV)

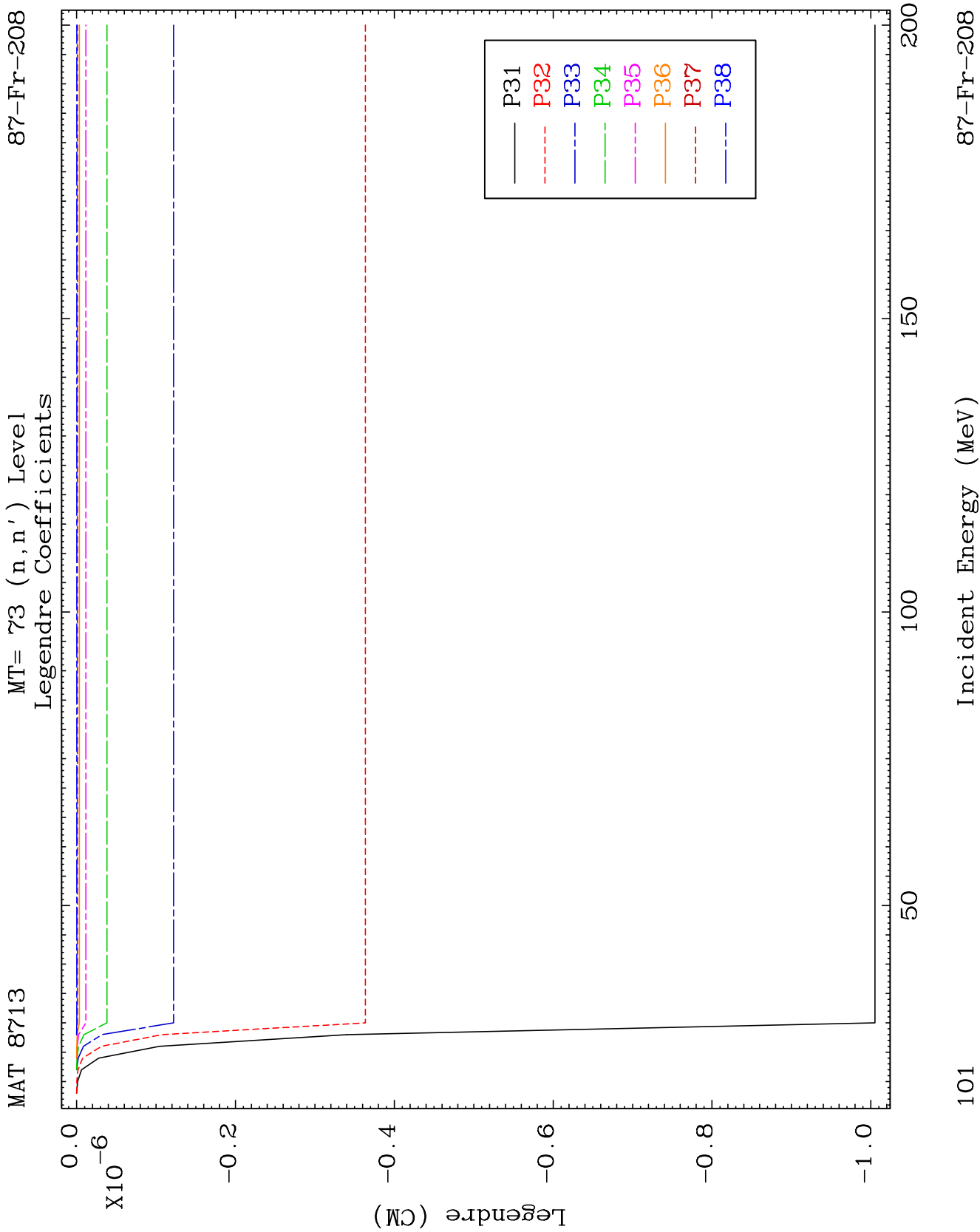
87-Fr-208







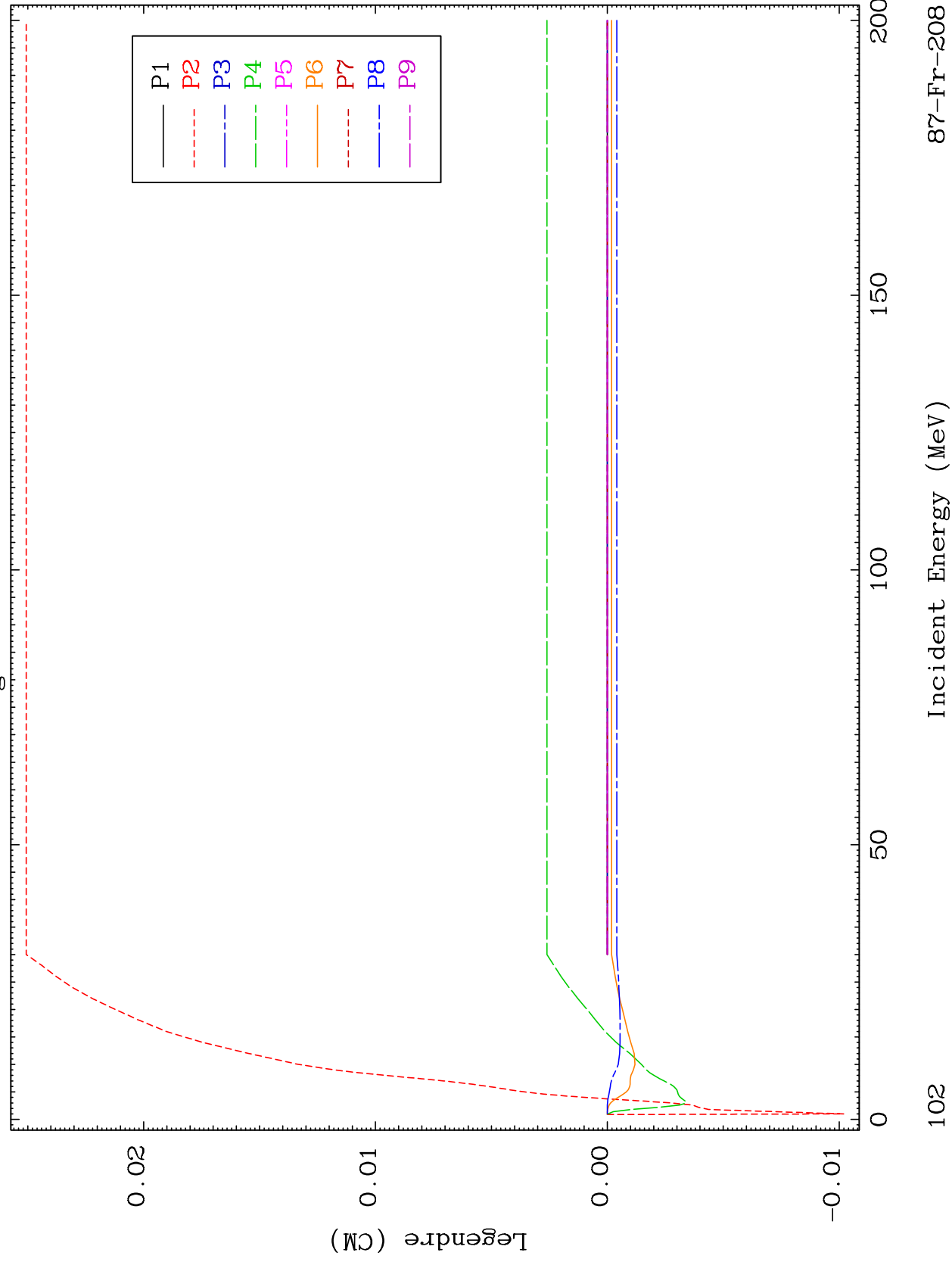


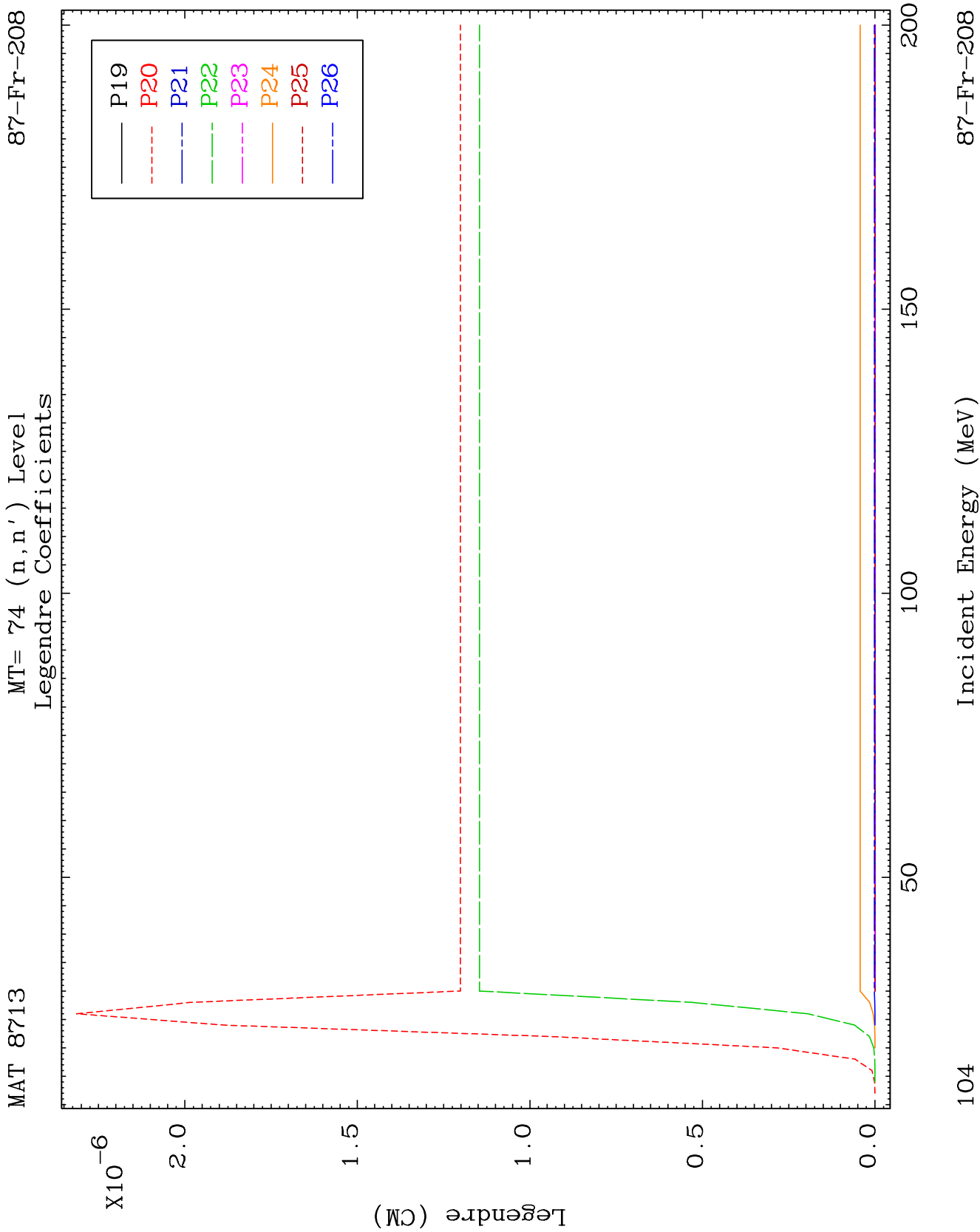


MAT 8713

MT= 74 (n, n') Level
Legendre Coefficients

87-Fr-208

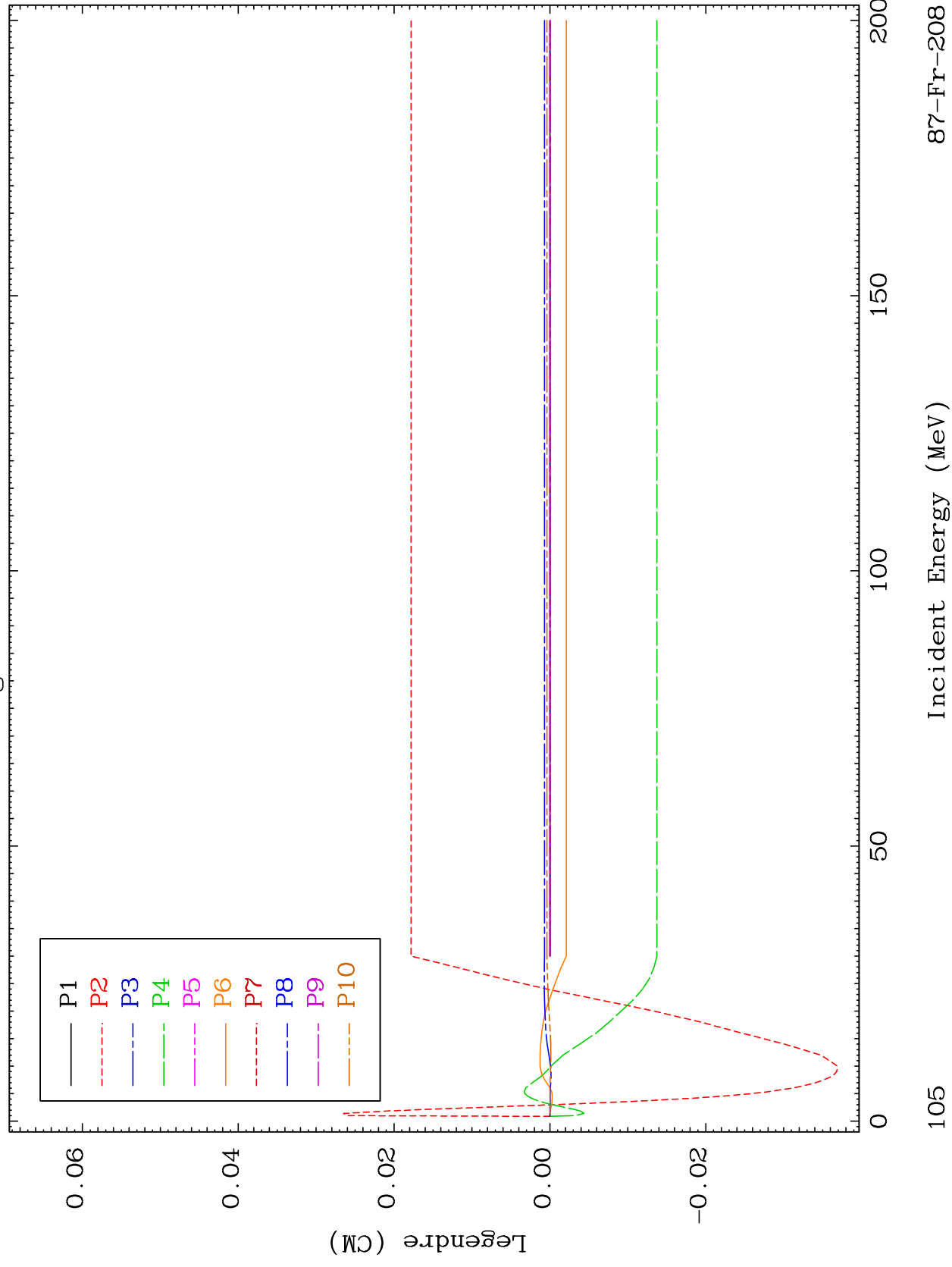


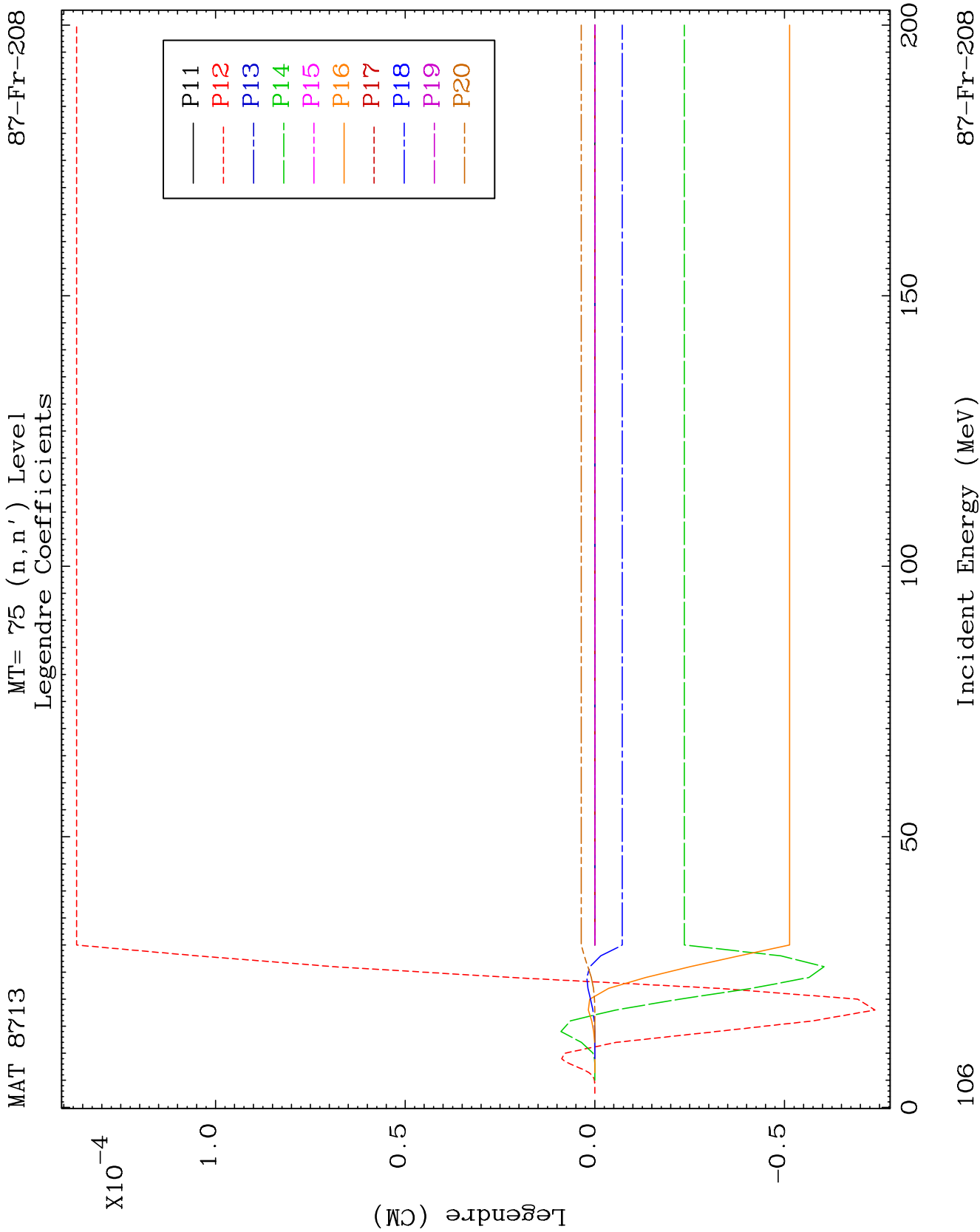


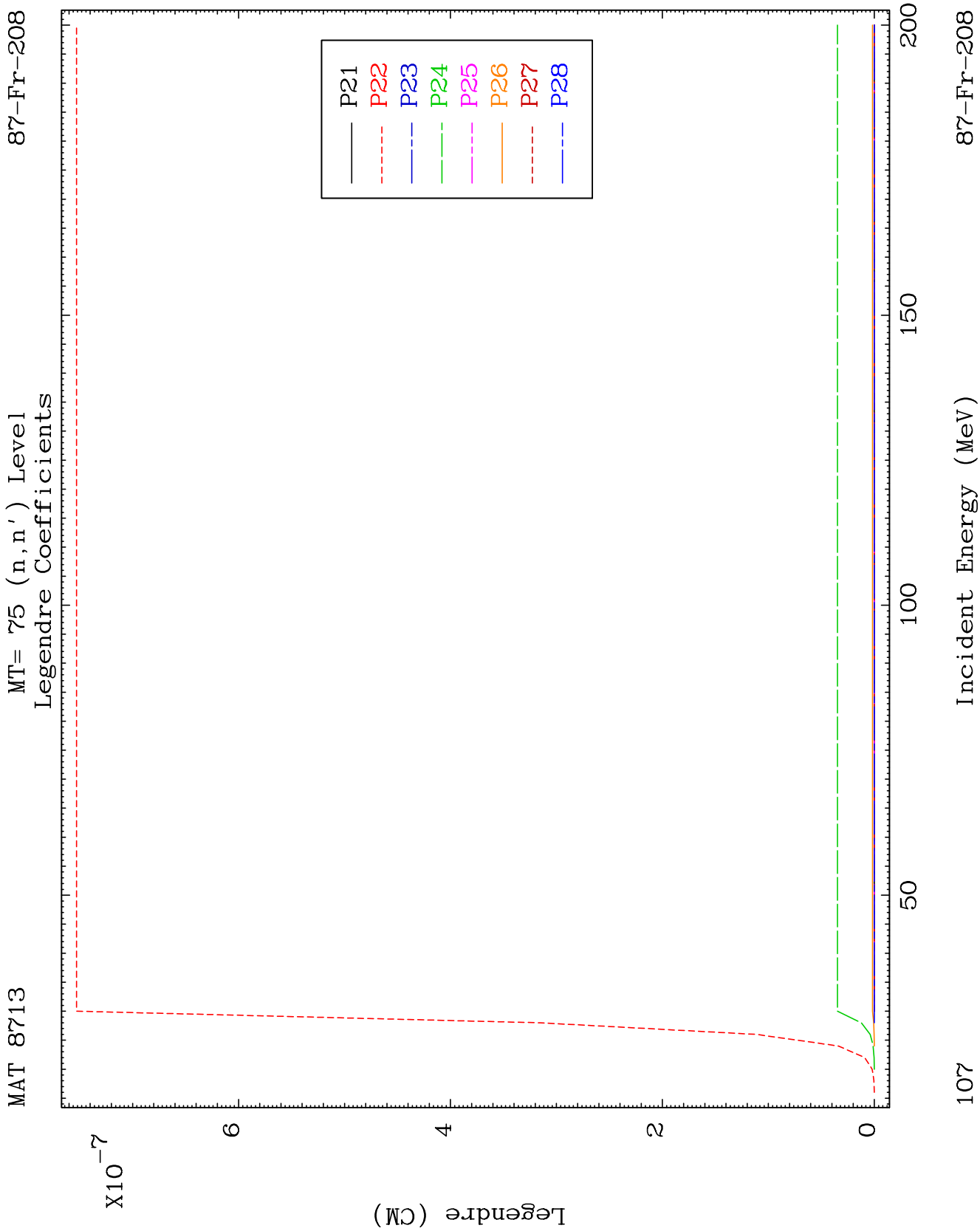
MAT 8713

MT= 75 (n, n') Level
Legendre Coefficients

87-Fr-208



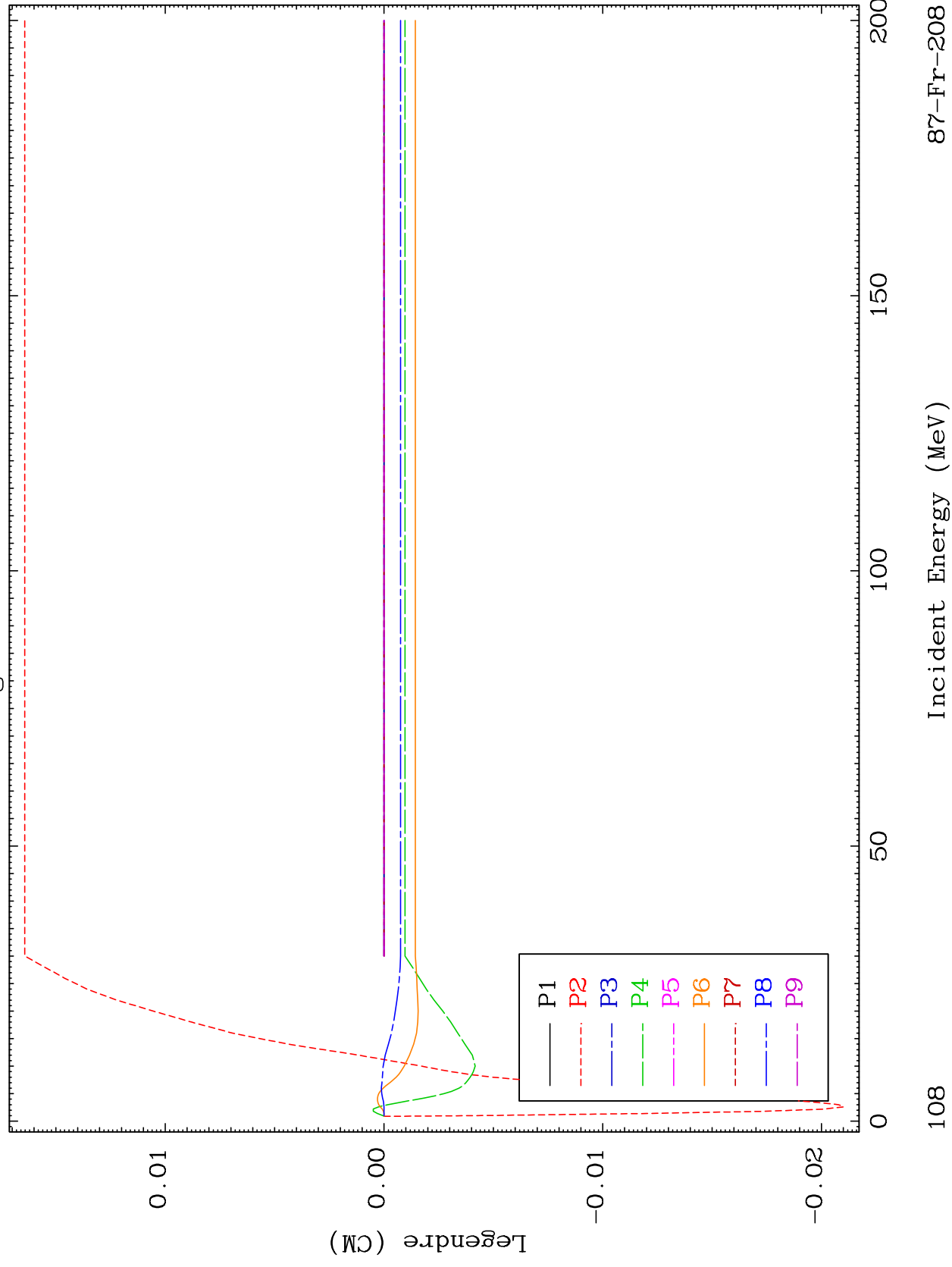


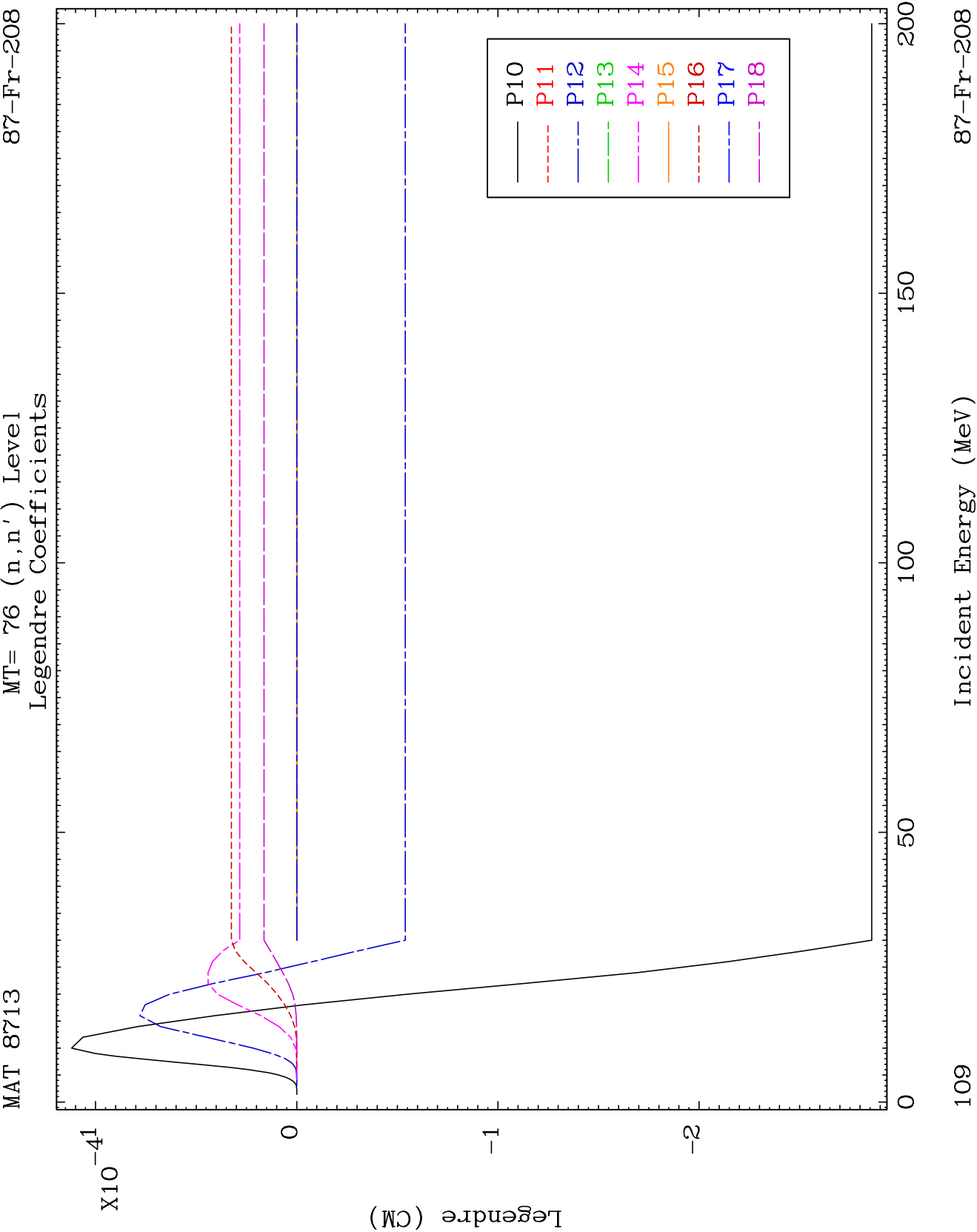


MAT 8713

MT= 76 (n, n') Level
Legendre Coefficients

87-Fr-208

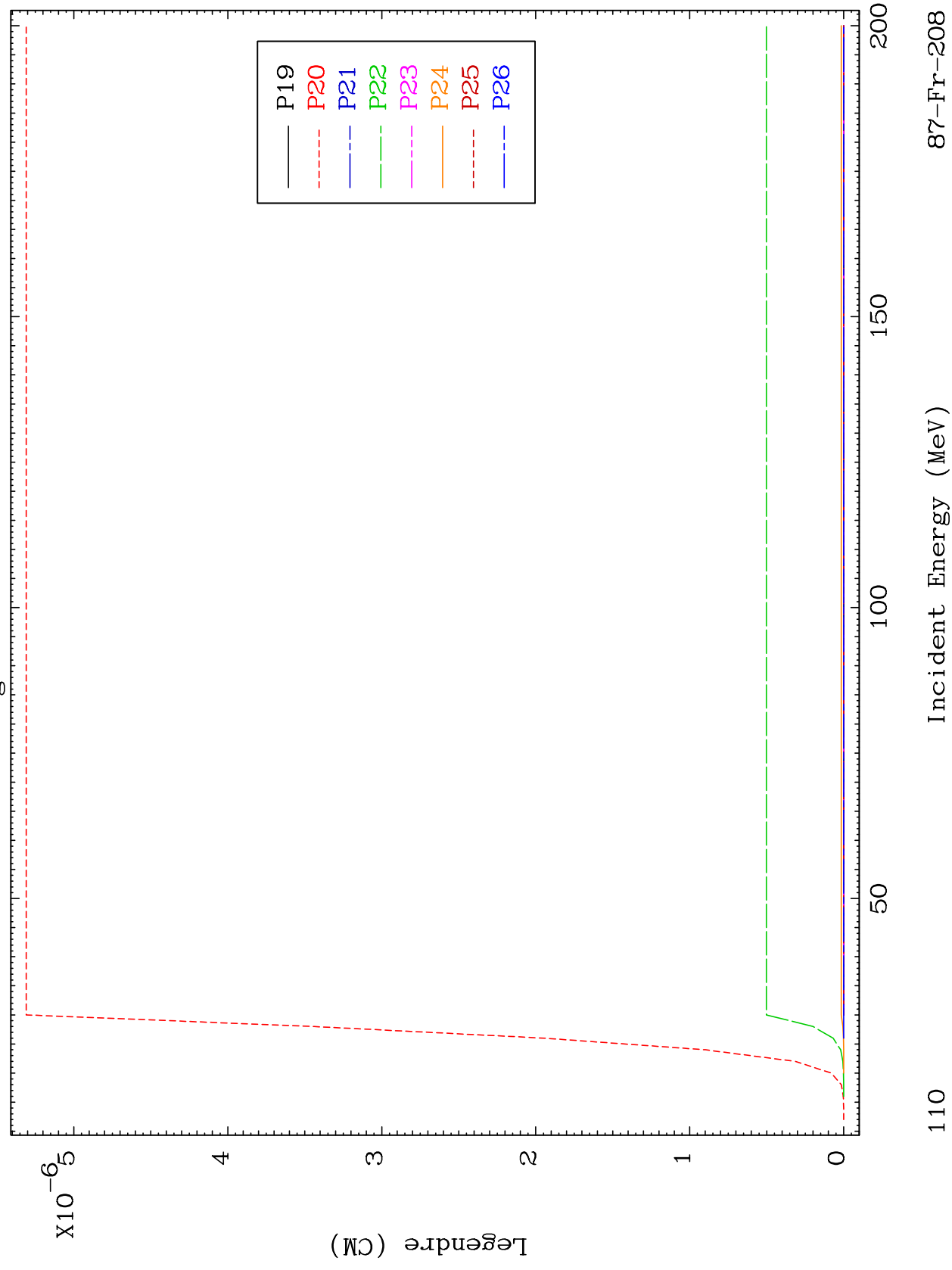




MAT 8713

MT= 76 (n, n') Level
Legendre Coefficients

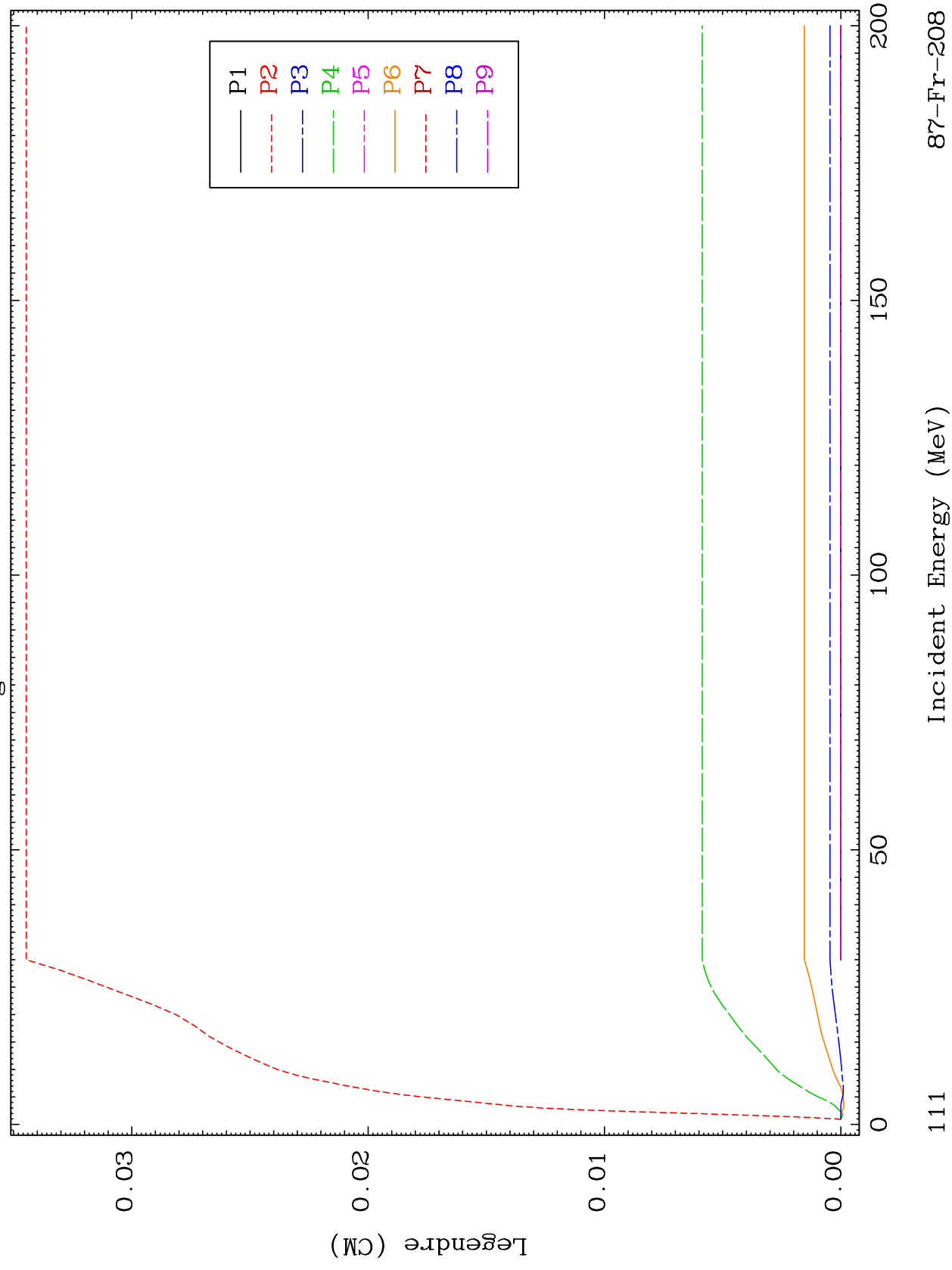
87-Fr-208



MAT 8713

MT= 77 (n, n') Level
Legendre Coefficients

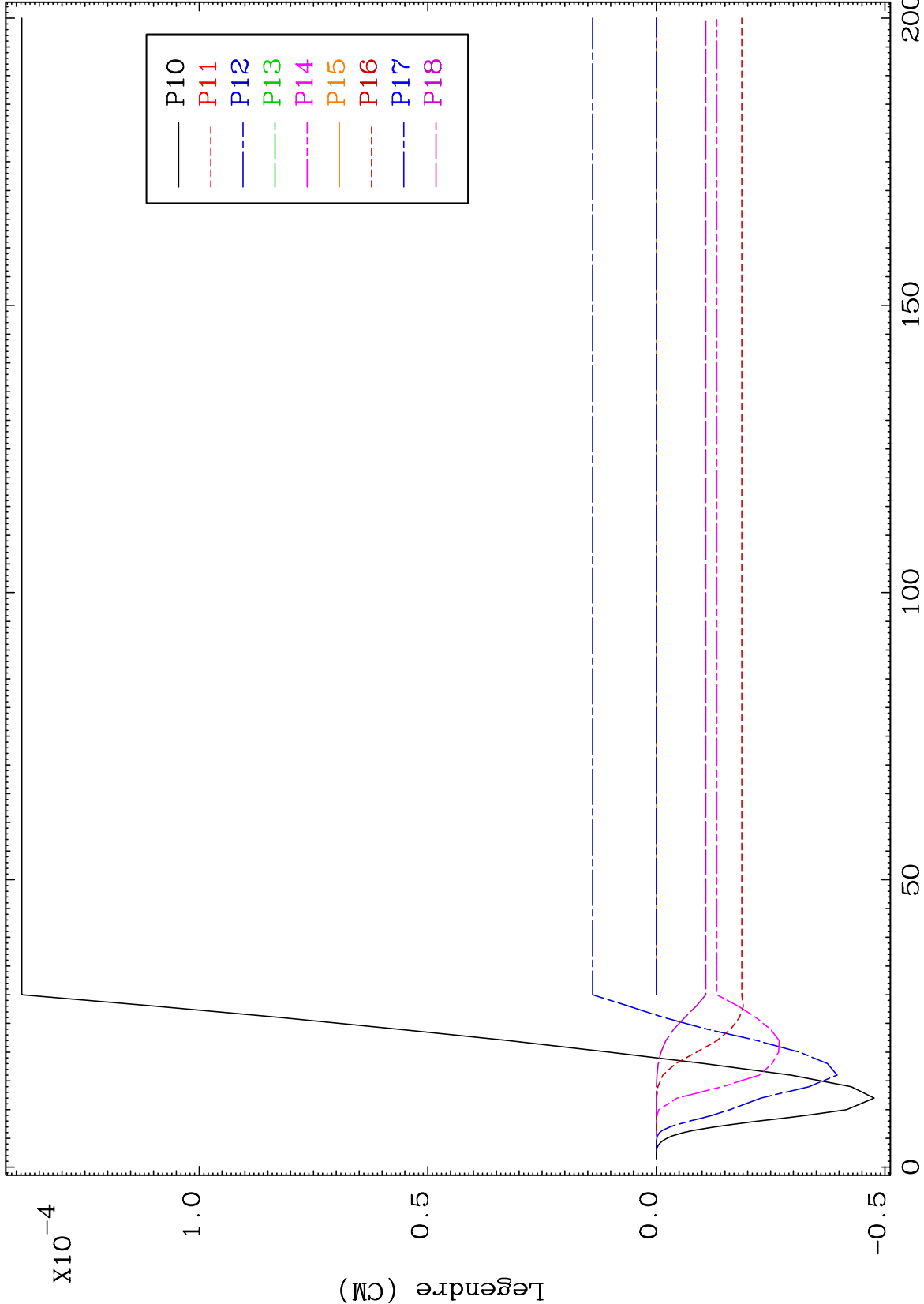
87-Fr-208



MAT 8713

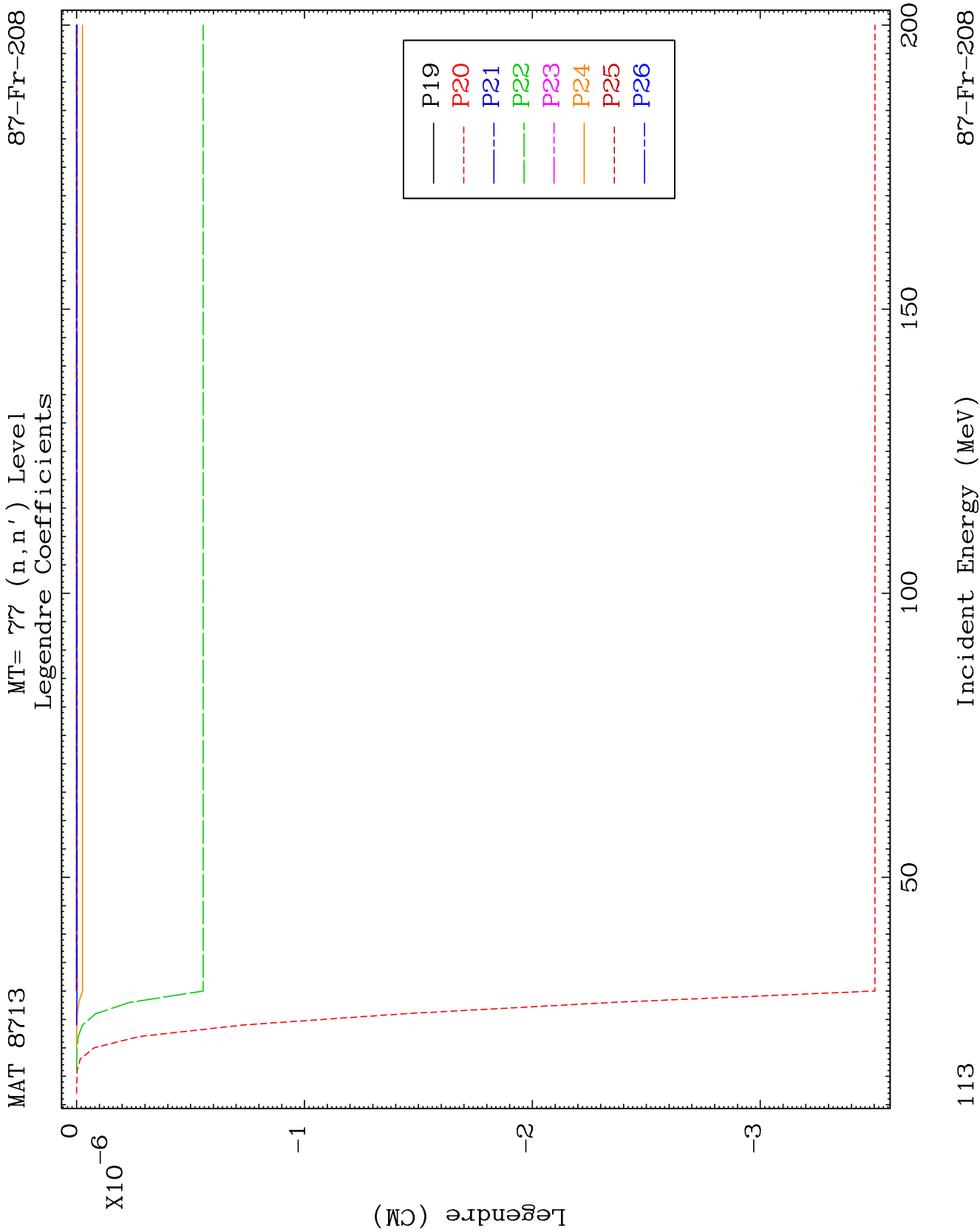
MT= 77 (n,n') Level
Legendre Coefficients

87-Fr-208

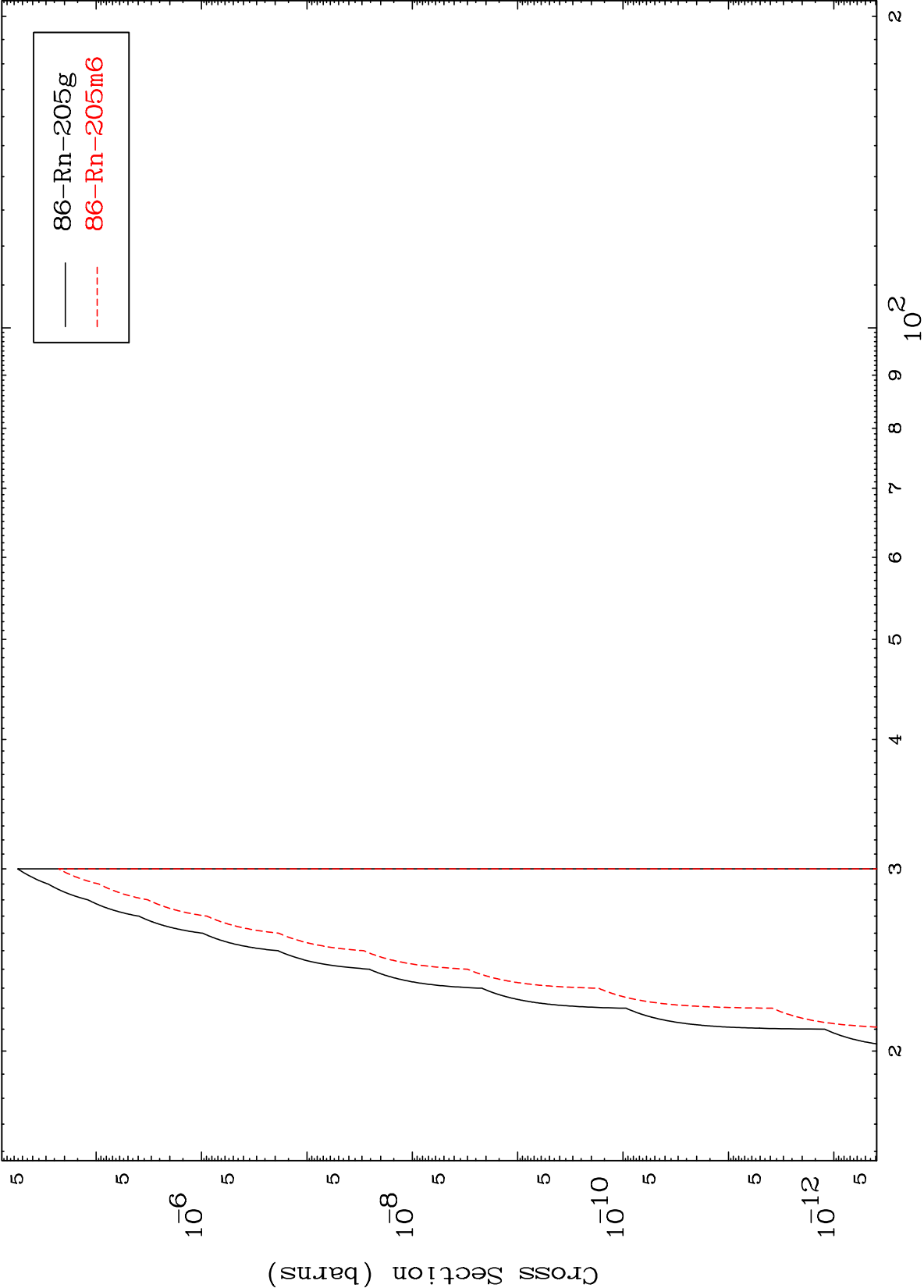


112

87-Fr-208



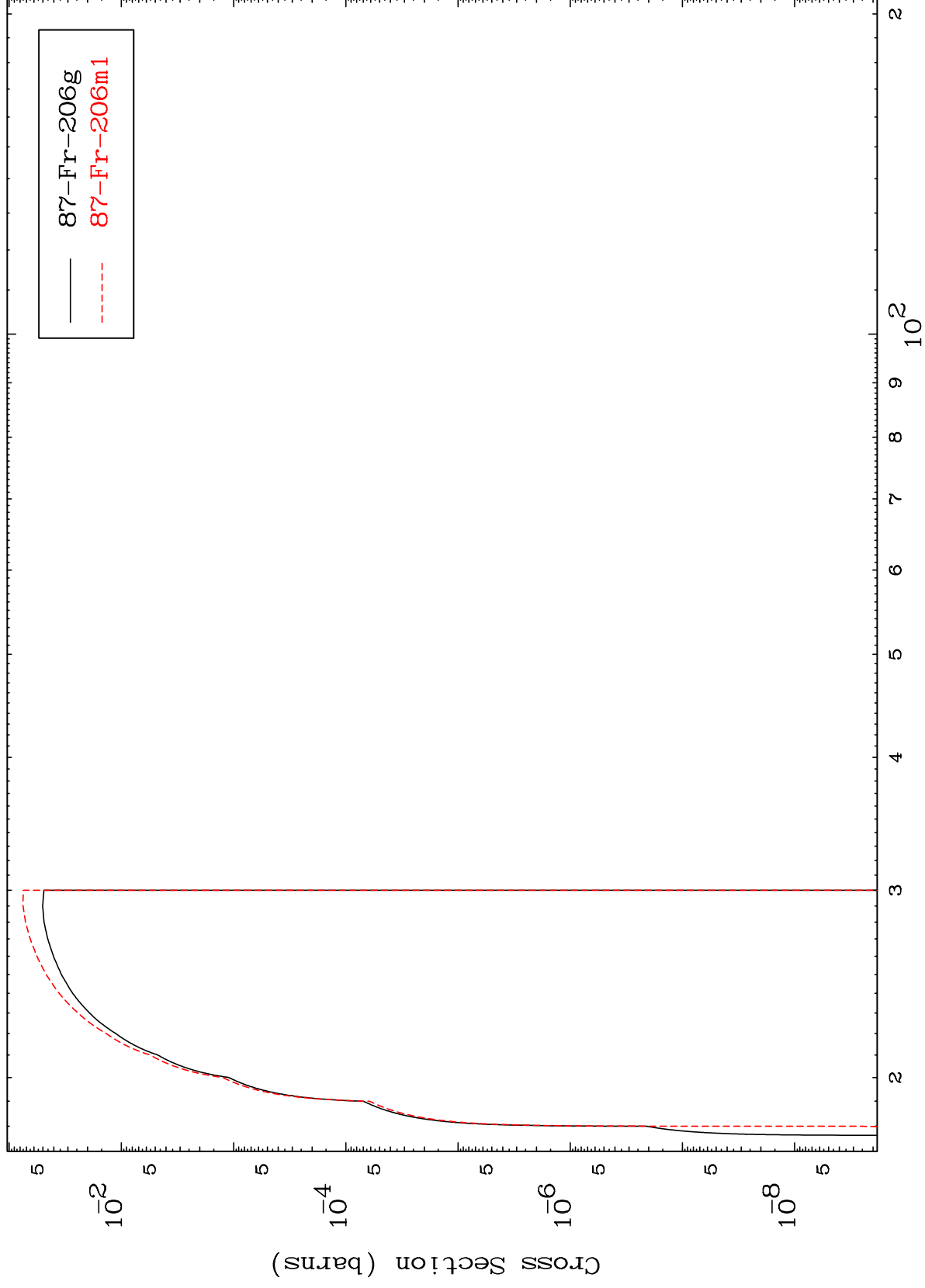
Radionuclide Production Cross Section



MAT 8713

87-Fr-208

(n,3n)
Radionuclide Production Cross Section



115

Incident Energy (MeV)

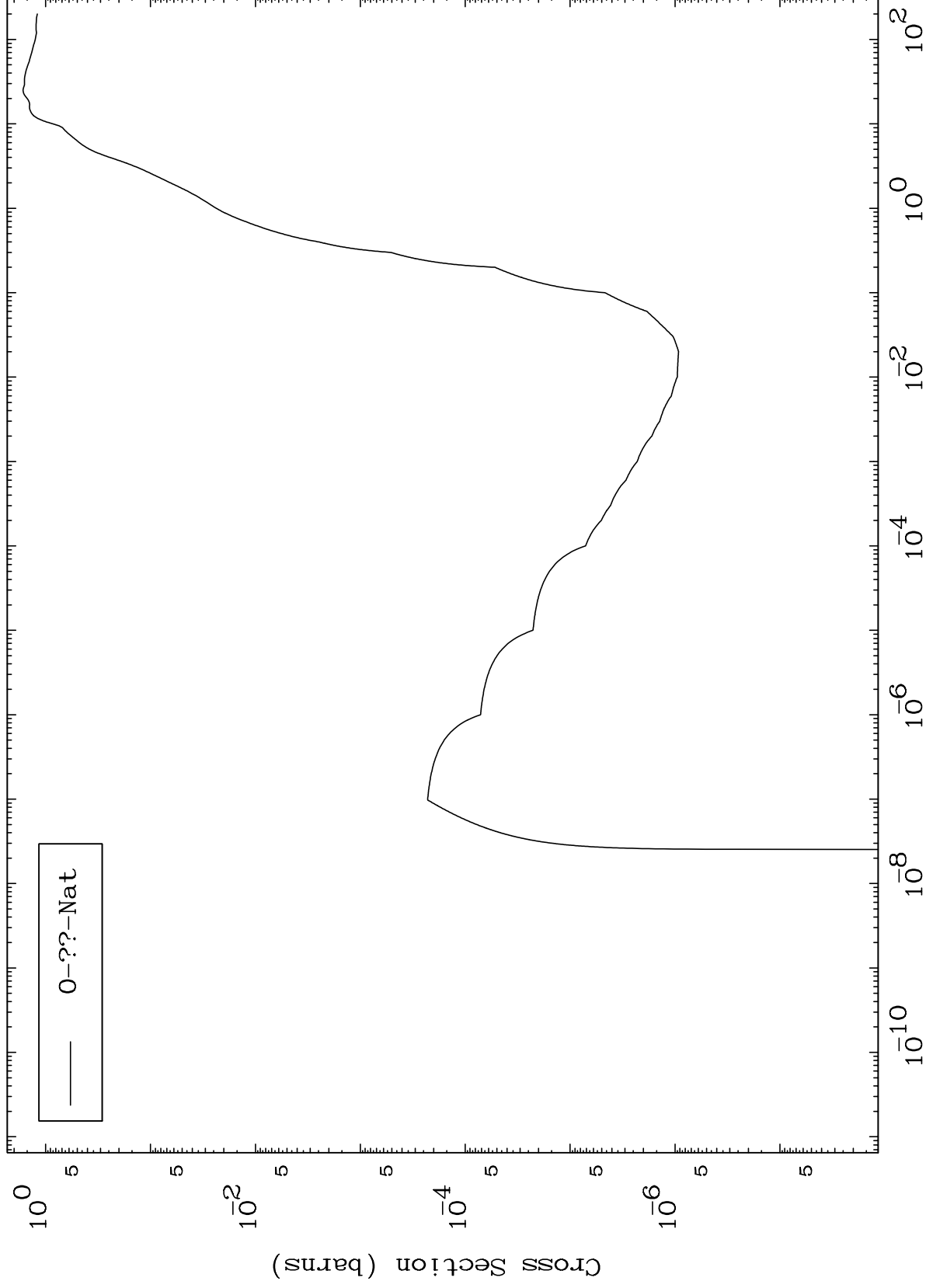
87-Fr-208

MAT 8713

Fission

87-Fr-208

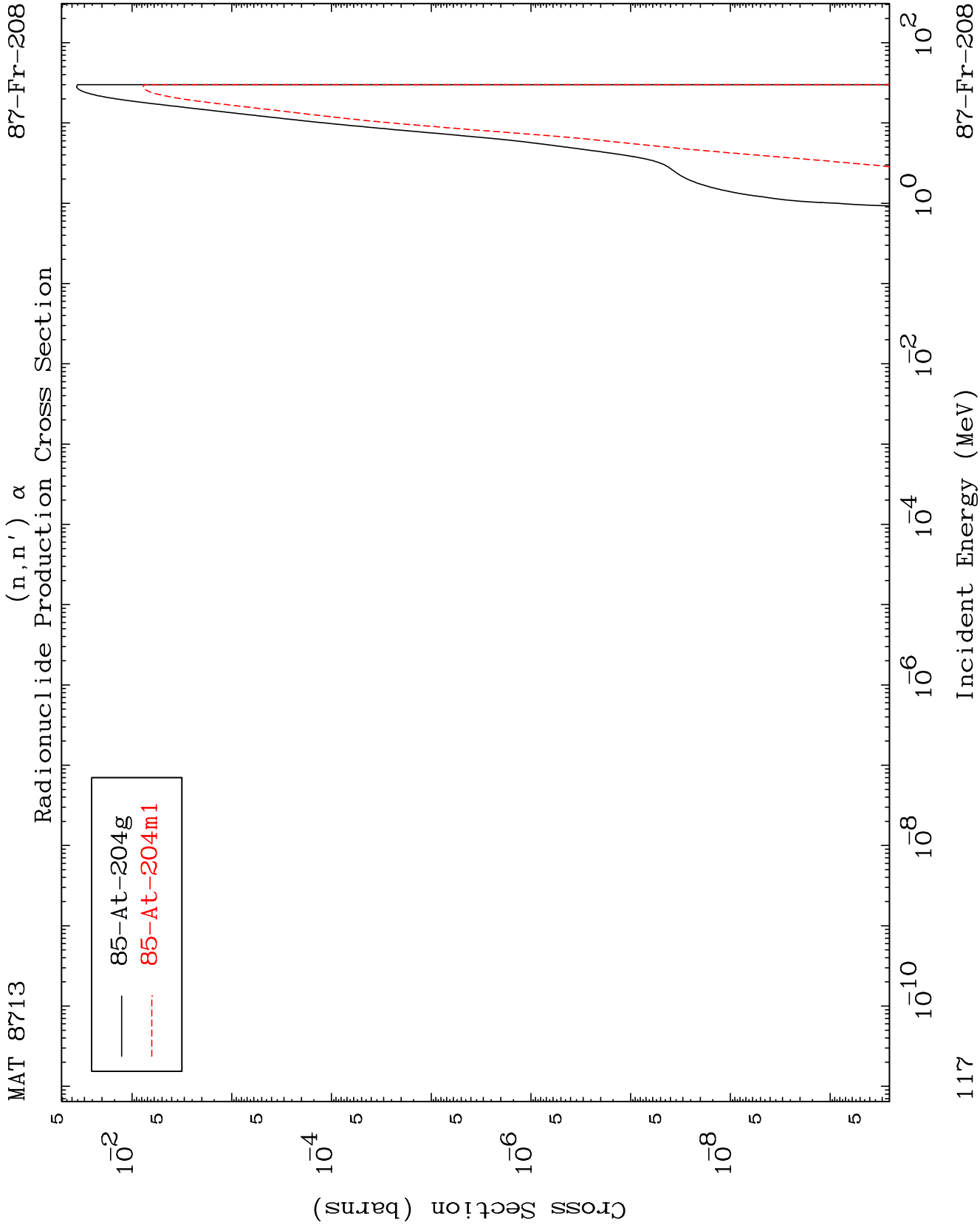
Radionuclide Production Cross Section



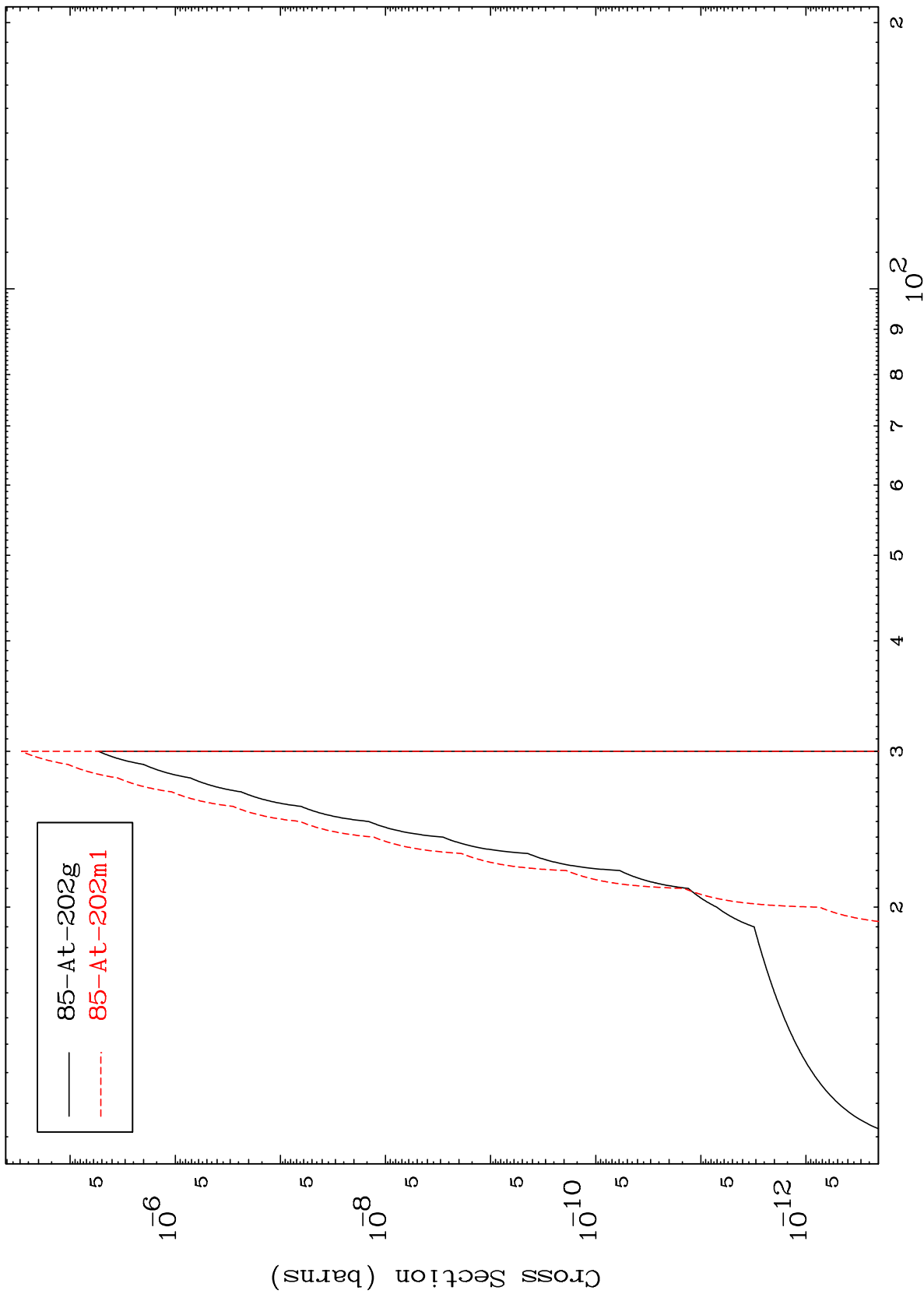
116

Incident Energy (MeV)

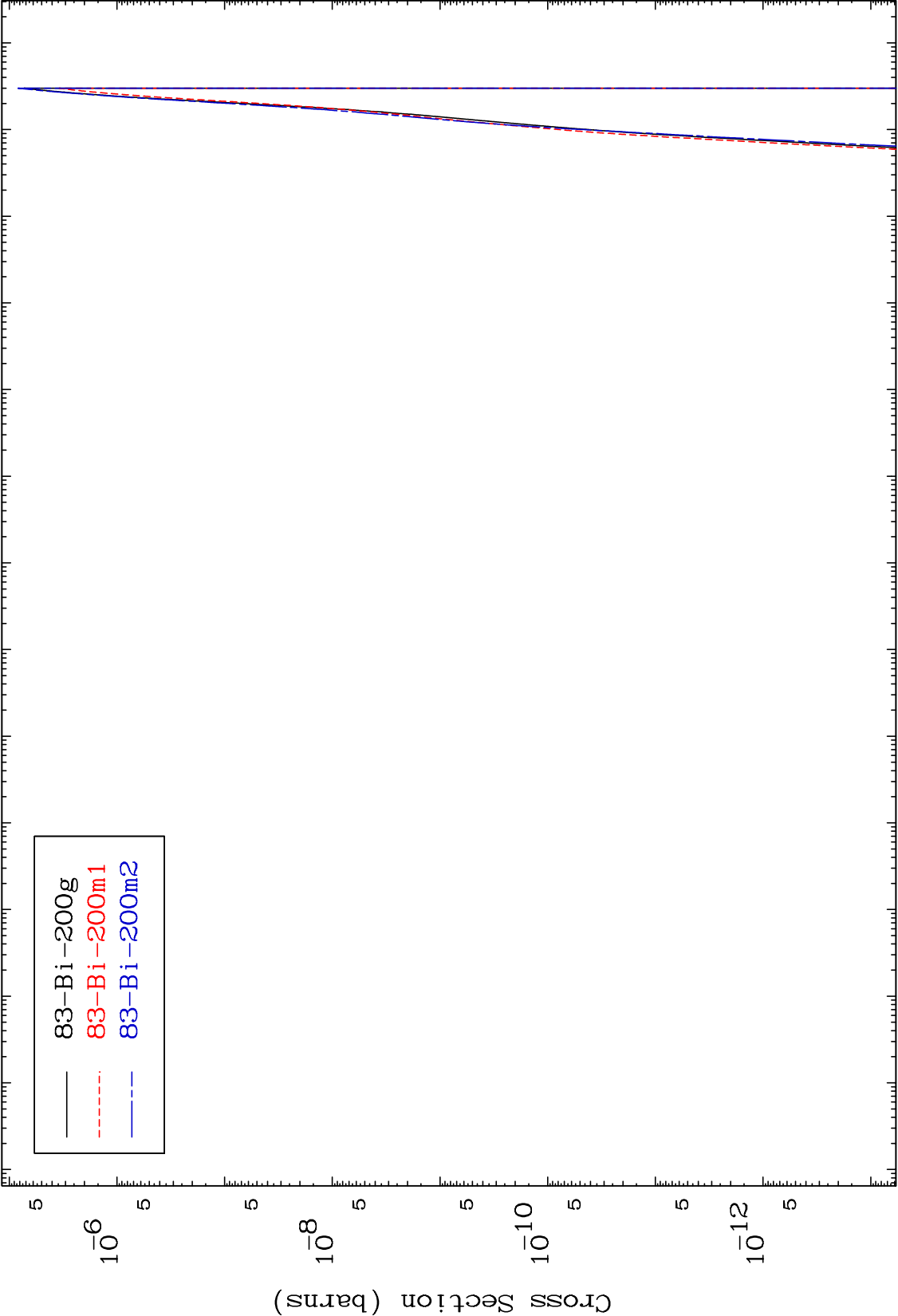
87-Fr-208



Radionuclide Production Cross Section



Radionuclide Production Cross Section

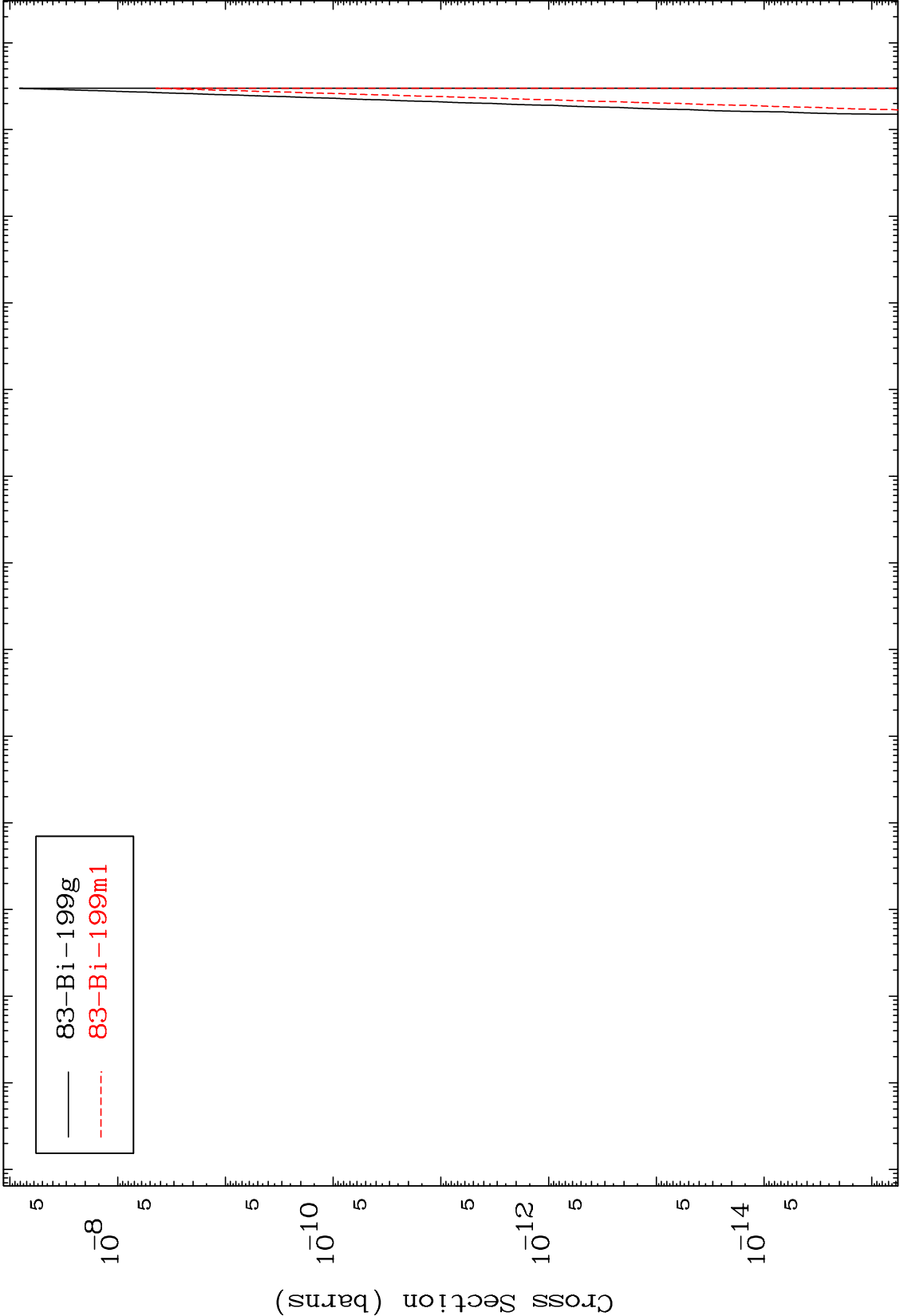
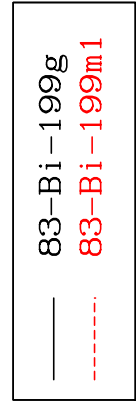


MAT 8713

(n,2n) 2 α

87-Fr-208

Radionuclide Production Cross Section



120

Incident Energy (MeV)

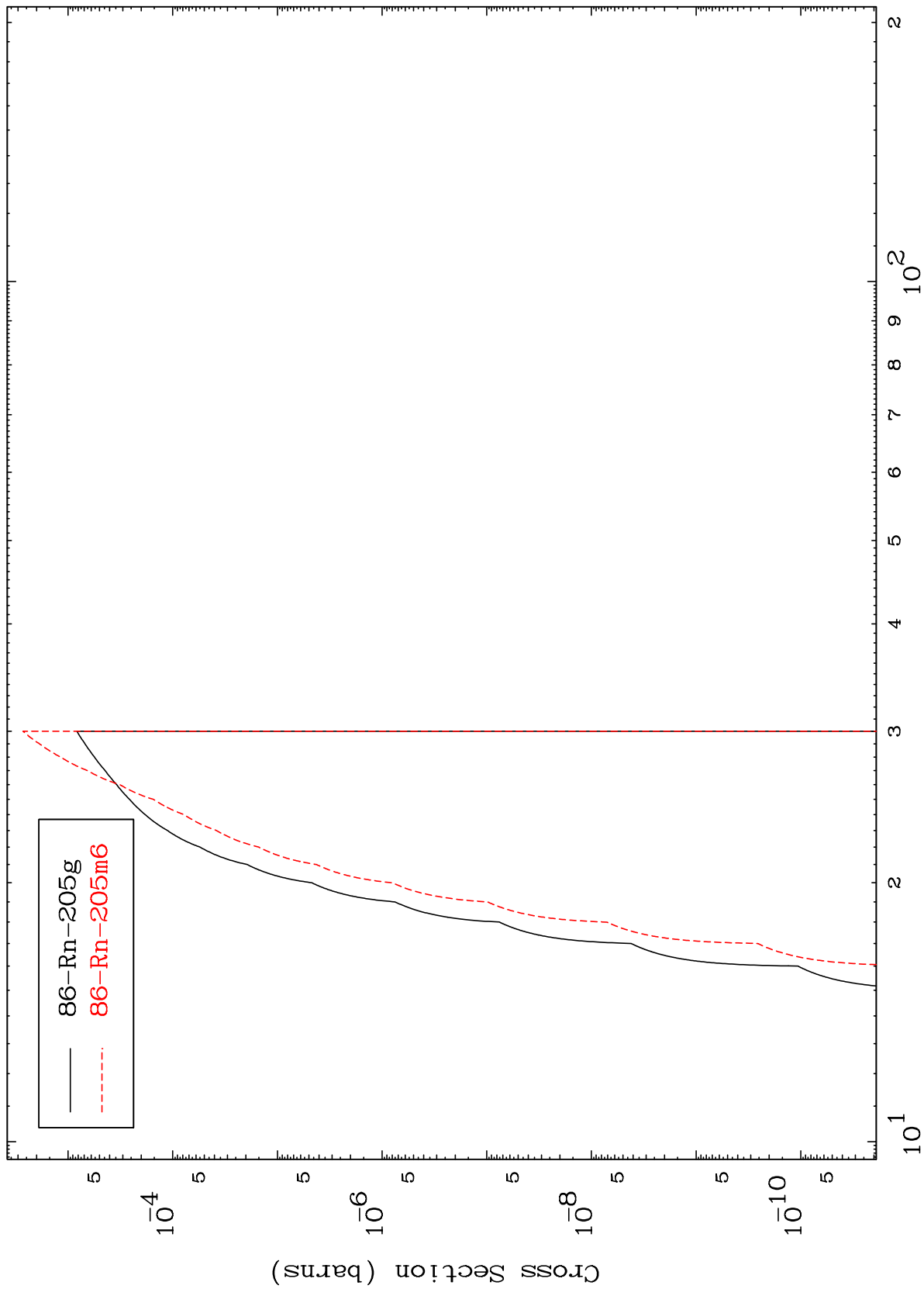
87-Fr-208

MAT 8713

(n,n') t

87-Fr-208

Radionuclide Production Cross Section



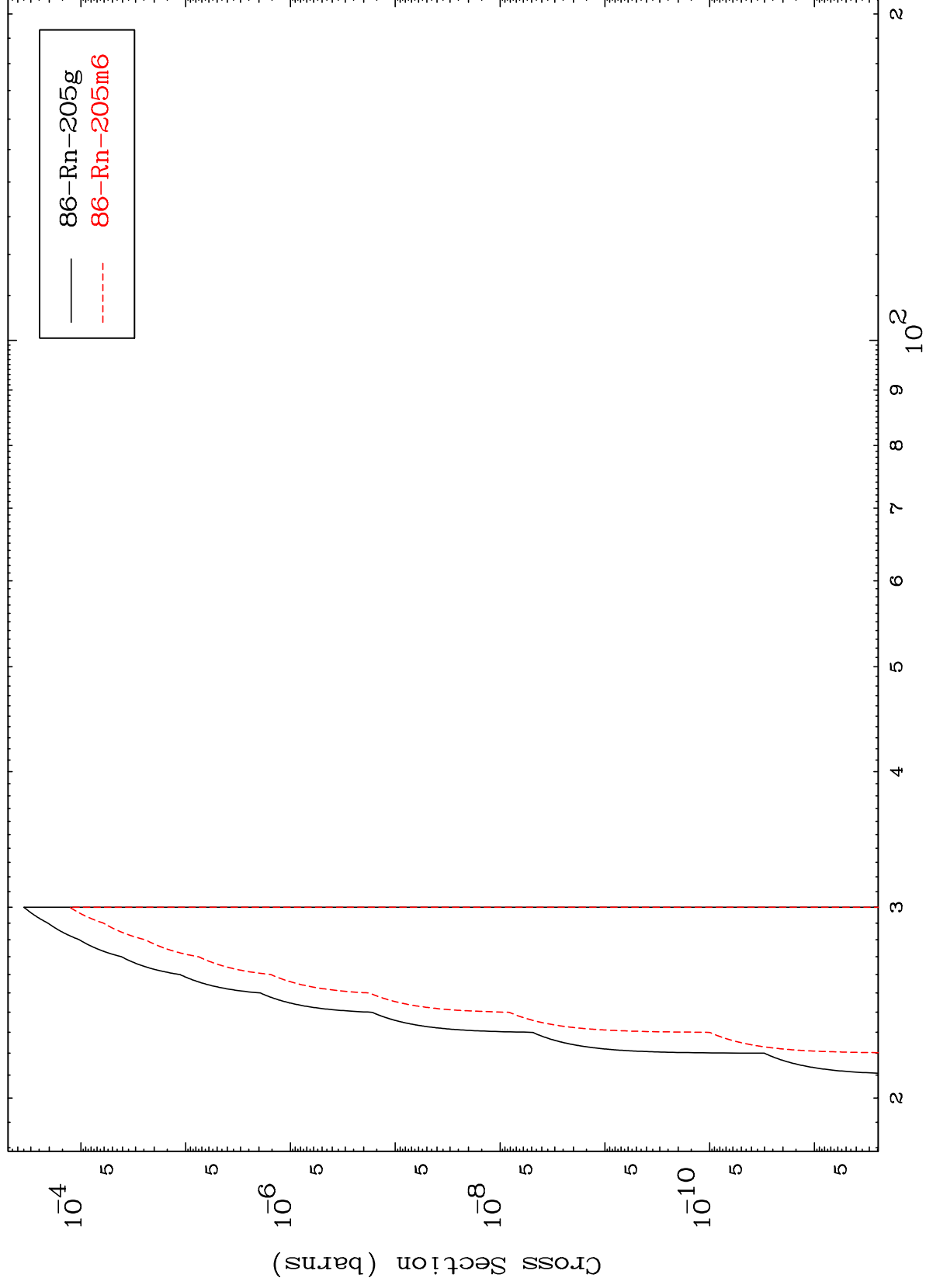
Incident Energy (MeV)

87-Fr-208

MAT 8713

87-Fr-208

(n,3n) p
Radionuclide Production Cross Section



87-Fr-208

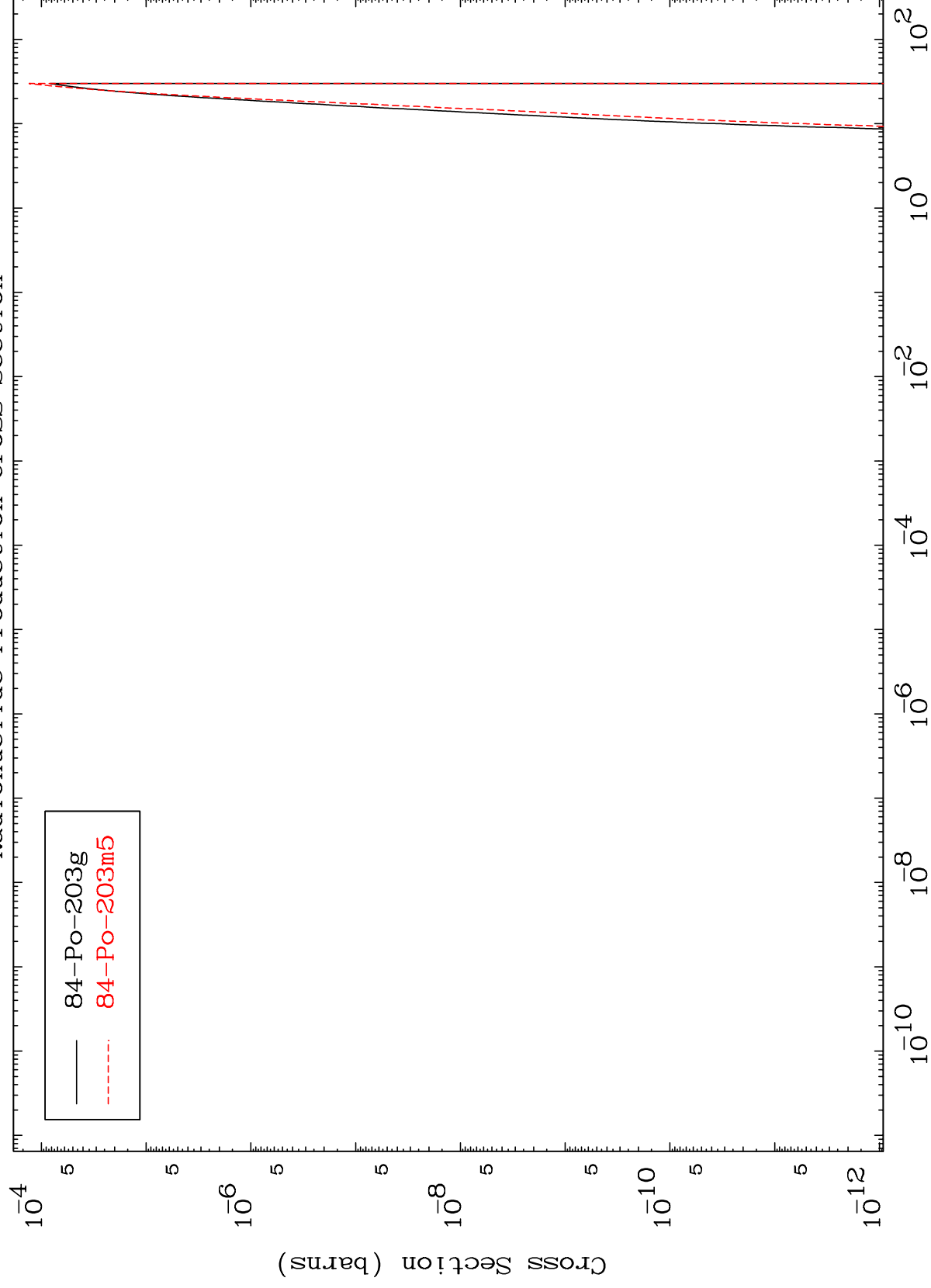
Incident Energy (MeV)

122

MAT 8713

87-Fr-208

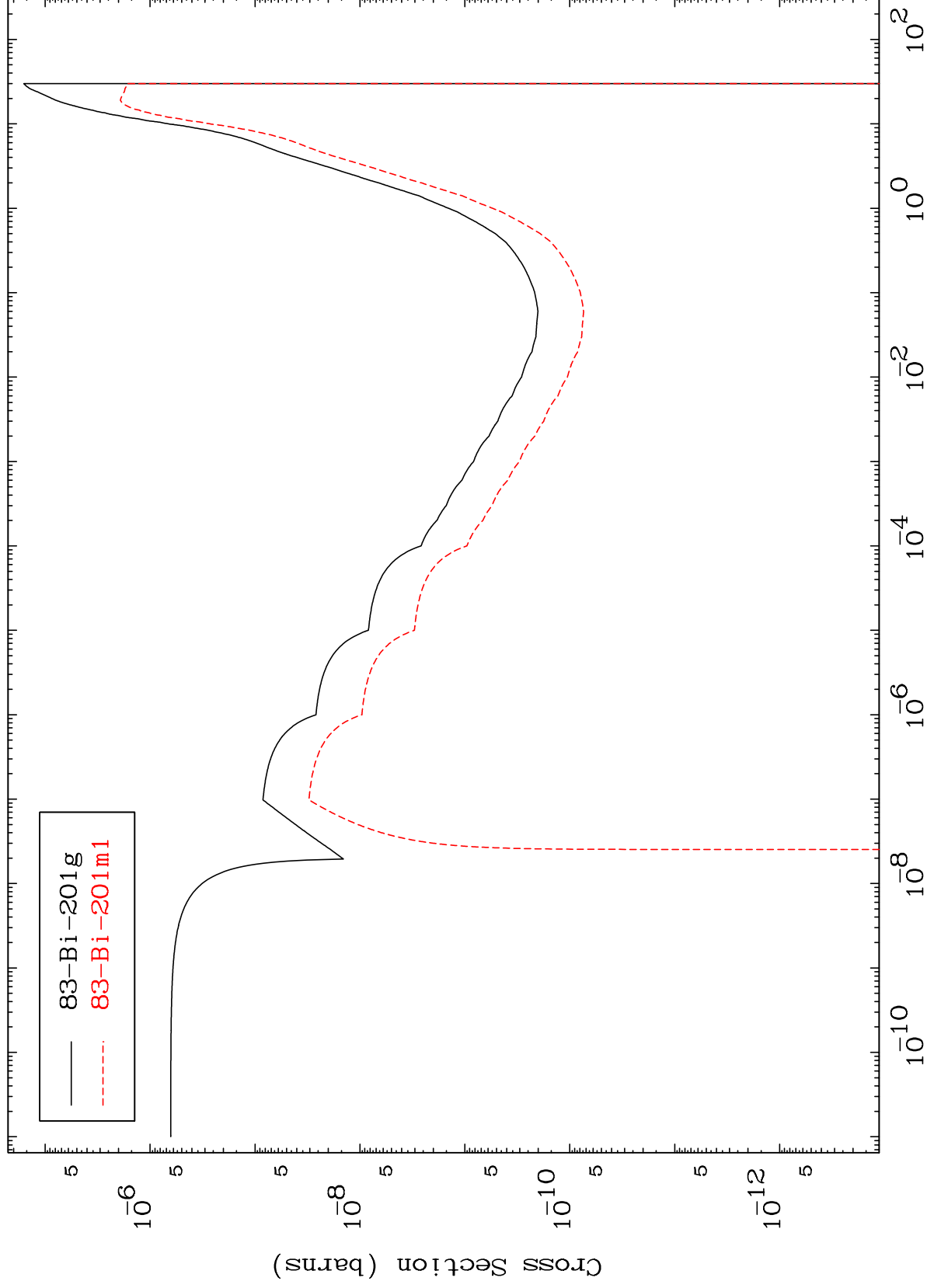
(n,n') p α
Radionuclide Production Cross Section



123

87-Fr-208

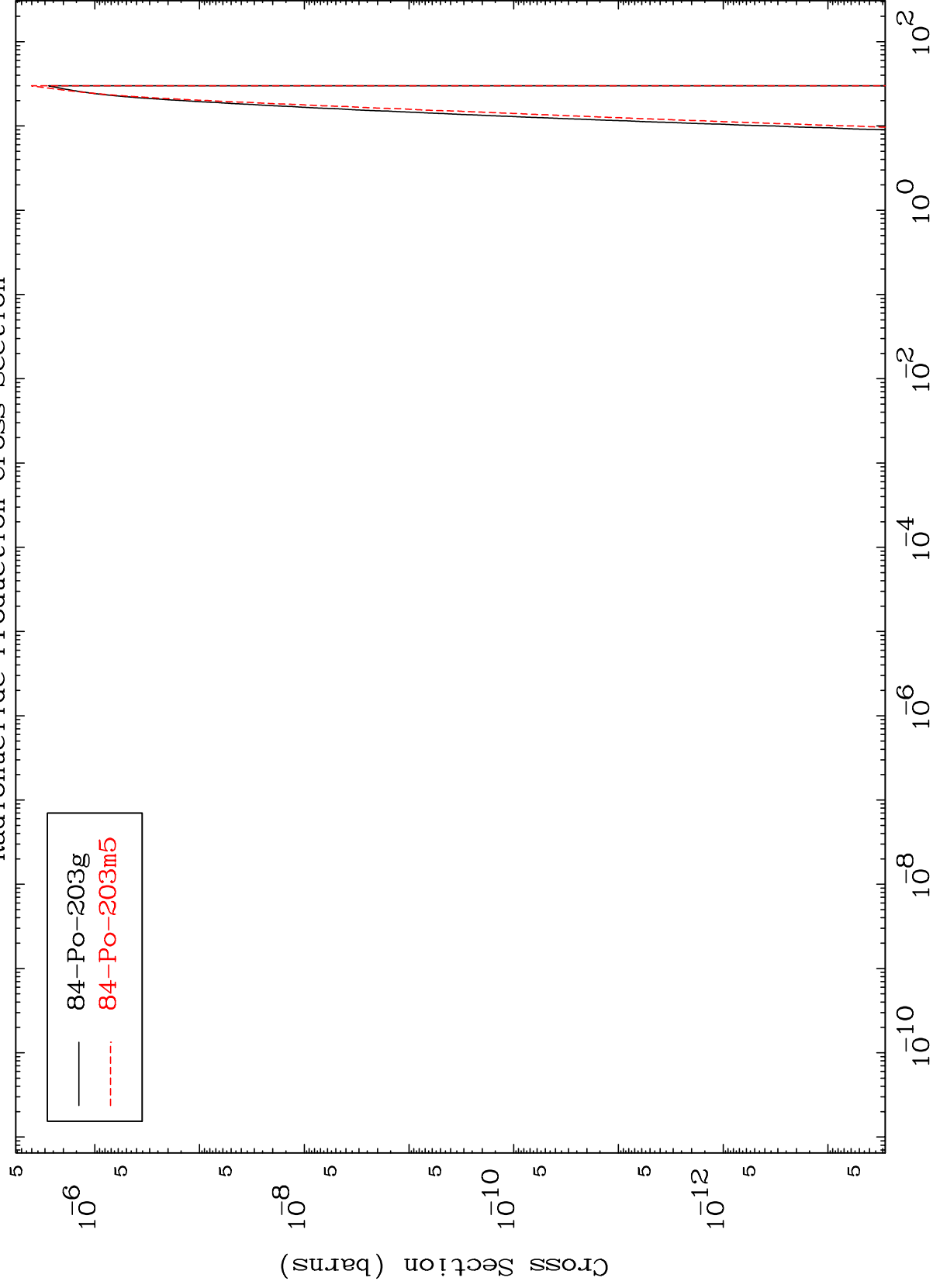
Radionuclide Production Cross Section
(n,2α)



MAT 8713

87-Fr-208

(n,d) α
Radionuclide Production Cross Section



125

Incident Energy (MeV)

87-Fr-208