

Program EVALPLOT
(Version 2021-1)

by

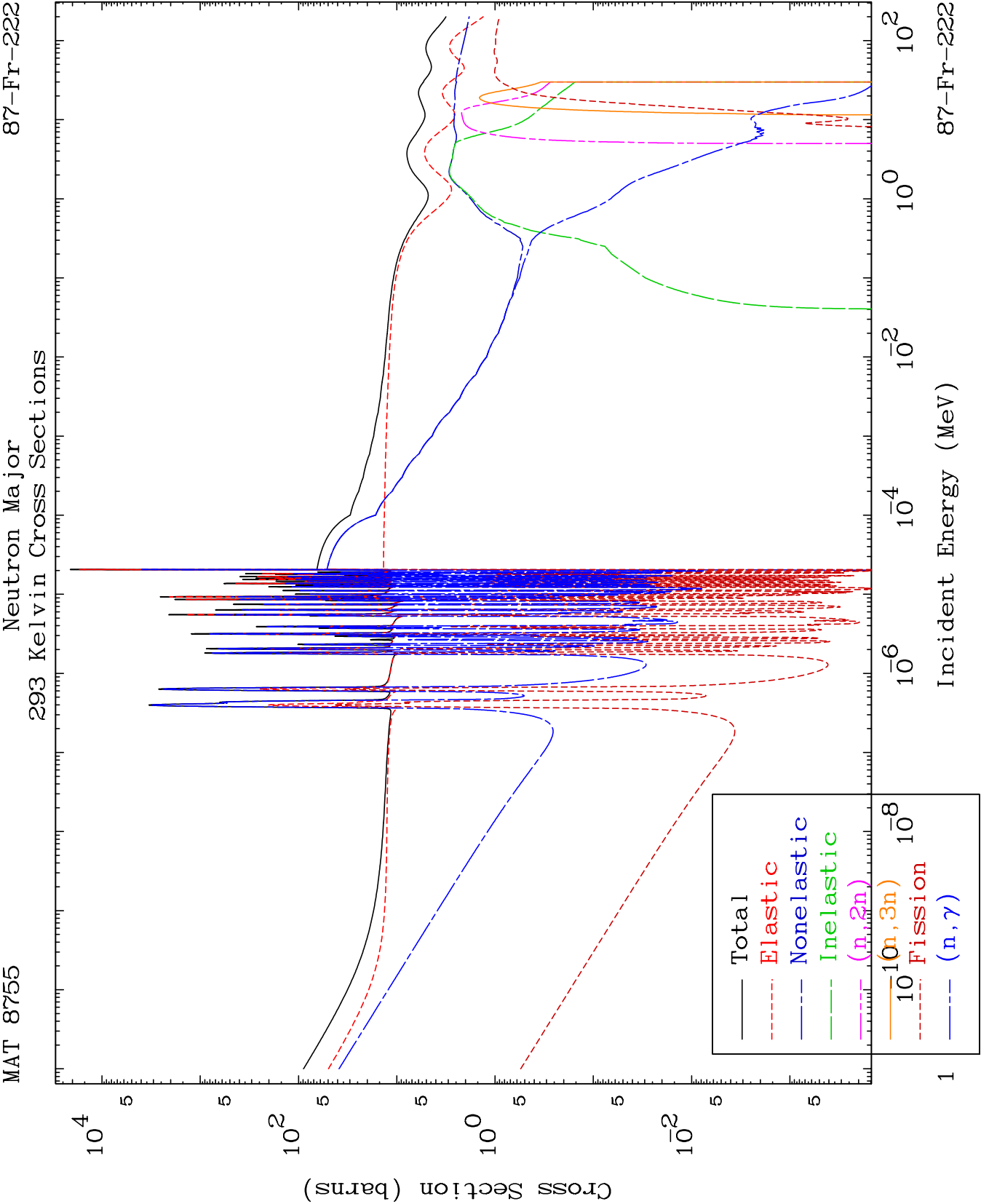
Dermott E. Cullen
(Present Contact Information)

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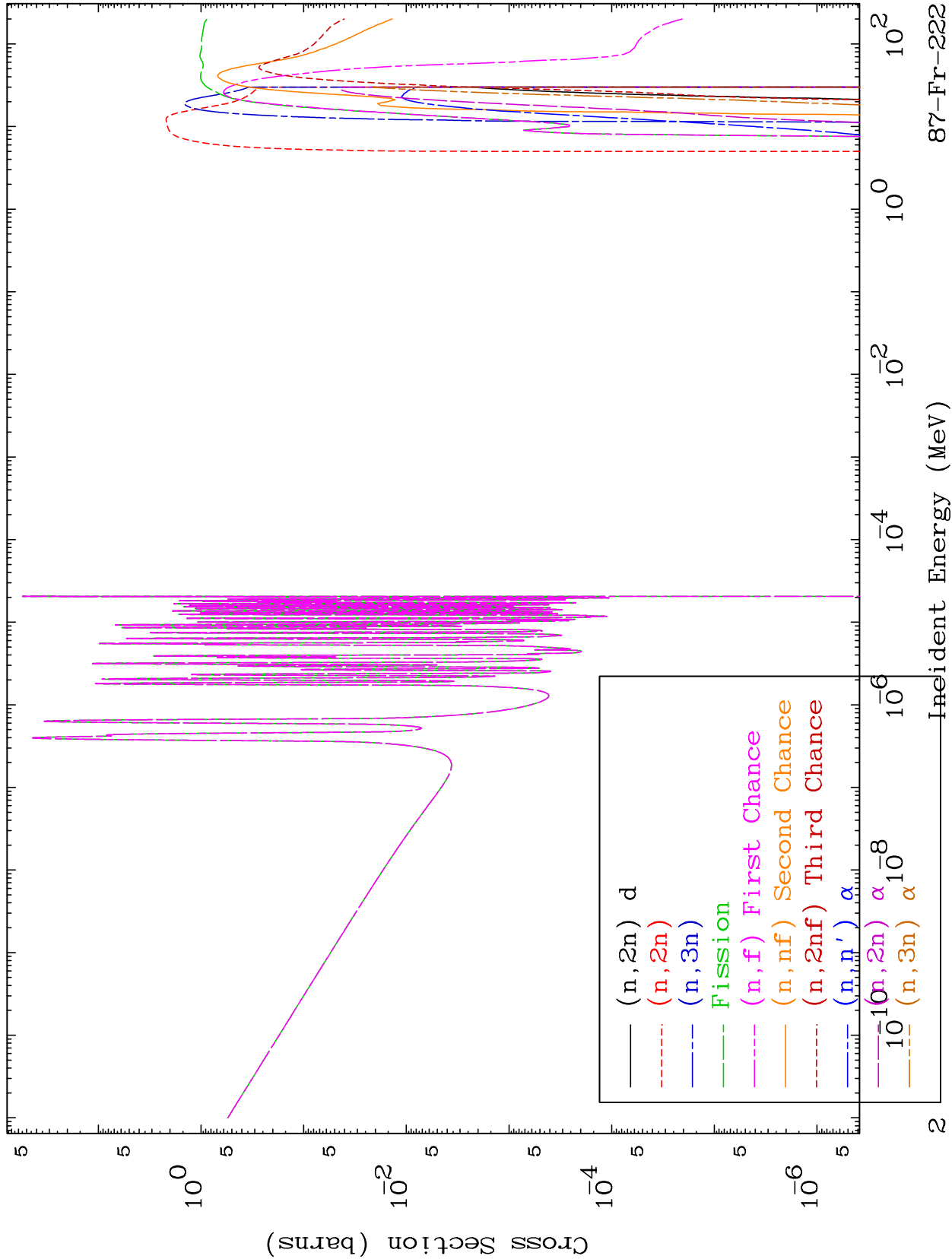
Press Mouse Button to Start

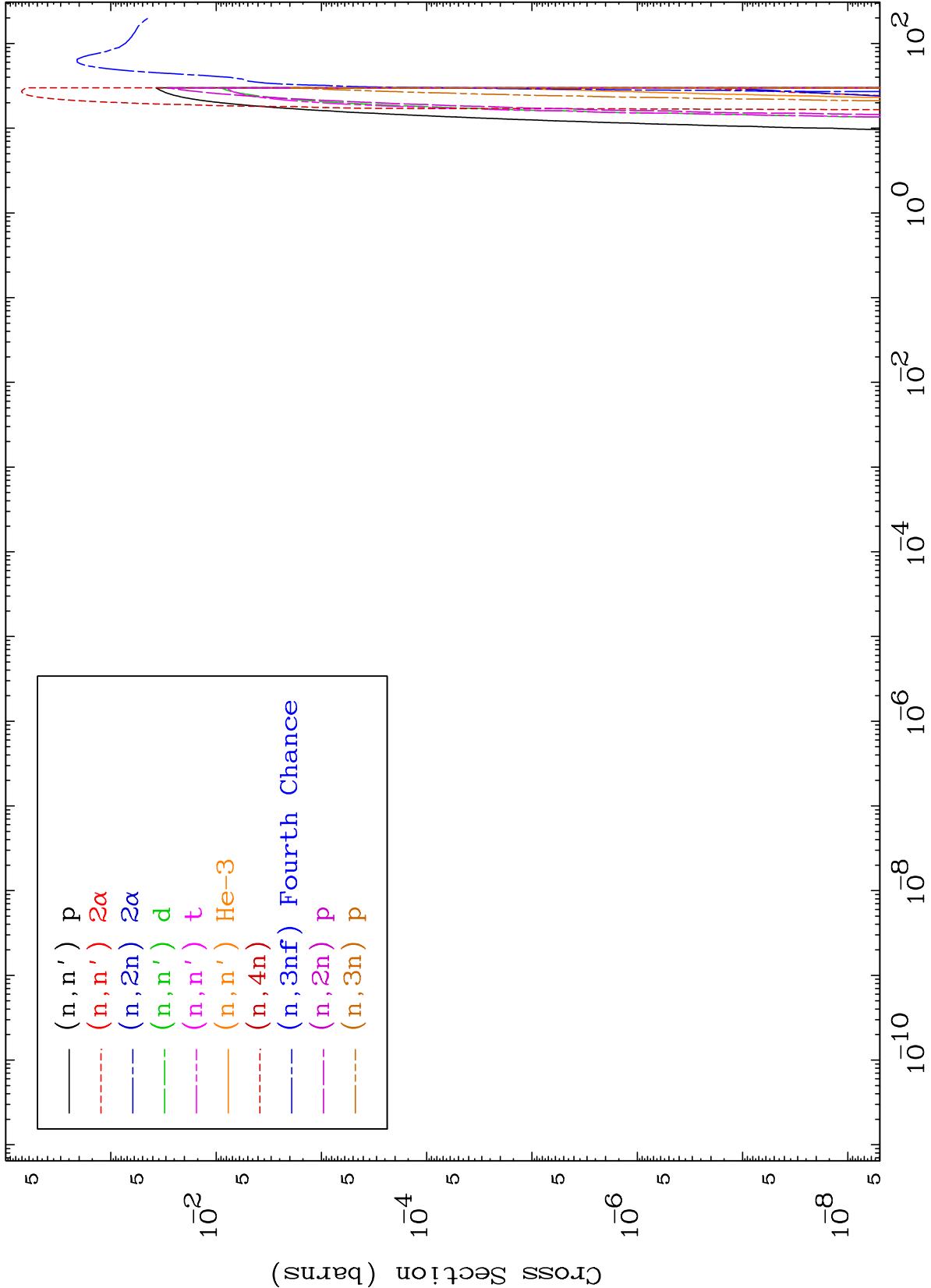


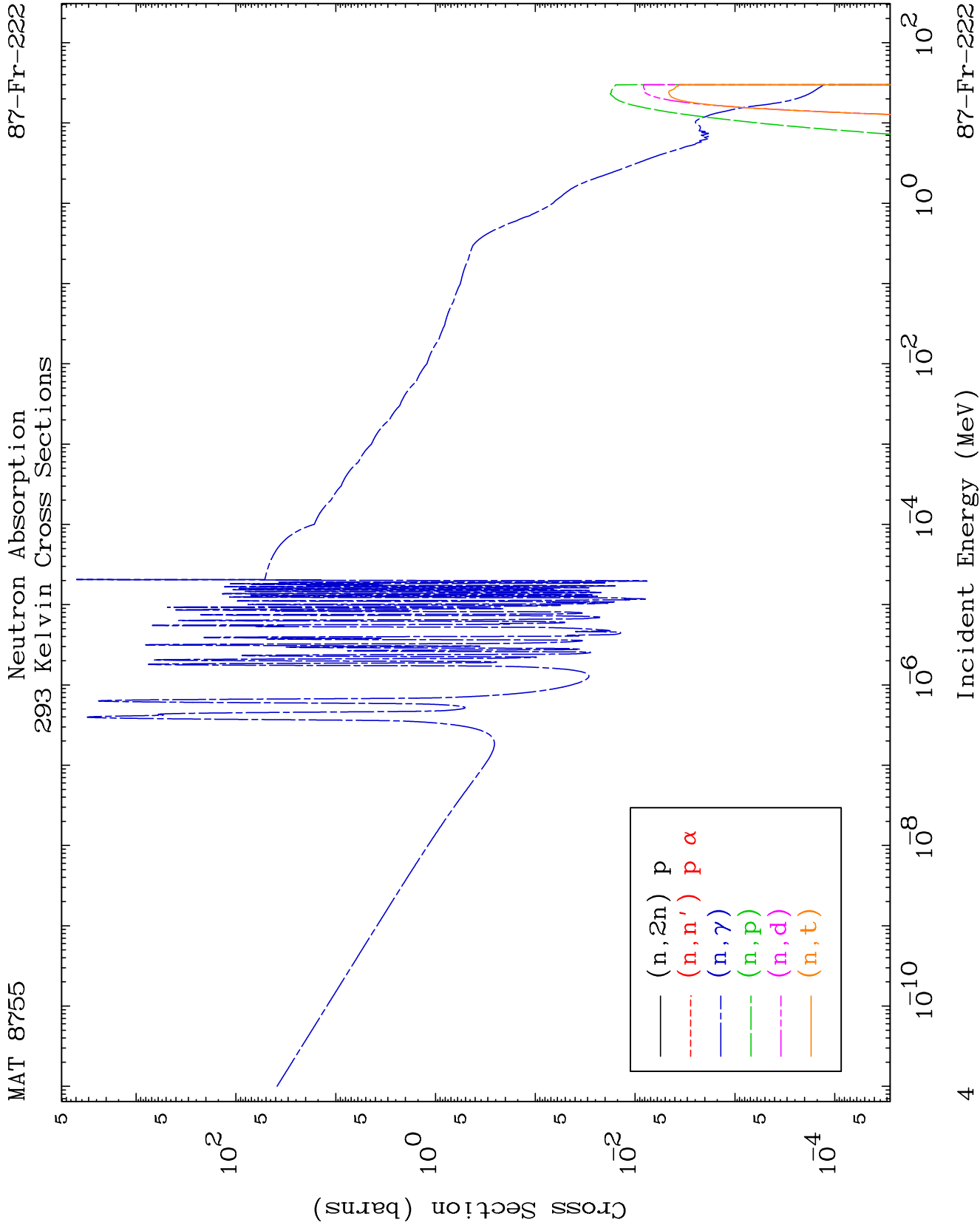
MAT 8755

Neutron Absorption
293 Kelvin Cross Sections

87-Fr-222



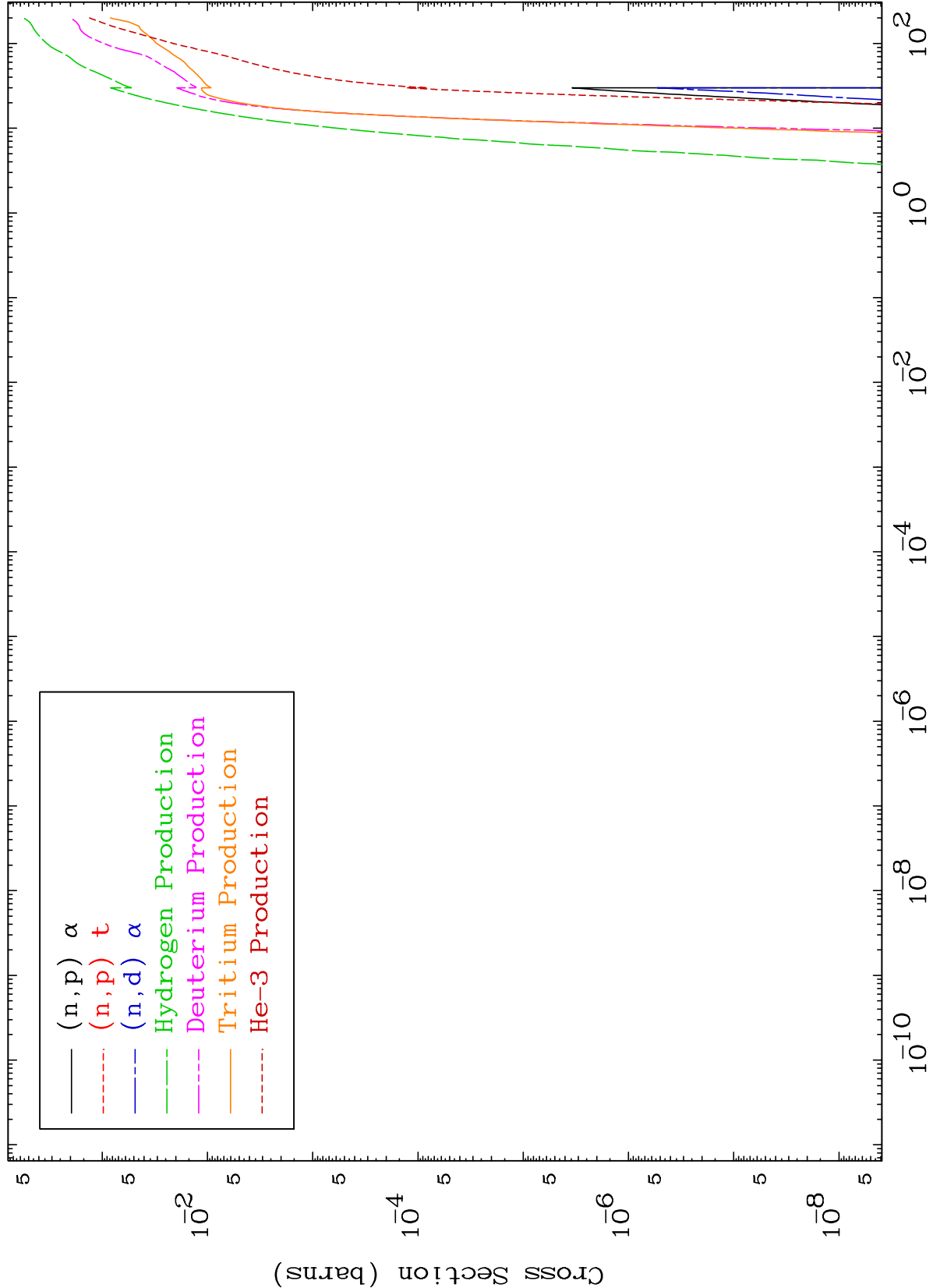




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Neutron Absorption
293 Kelvin Cross Sections

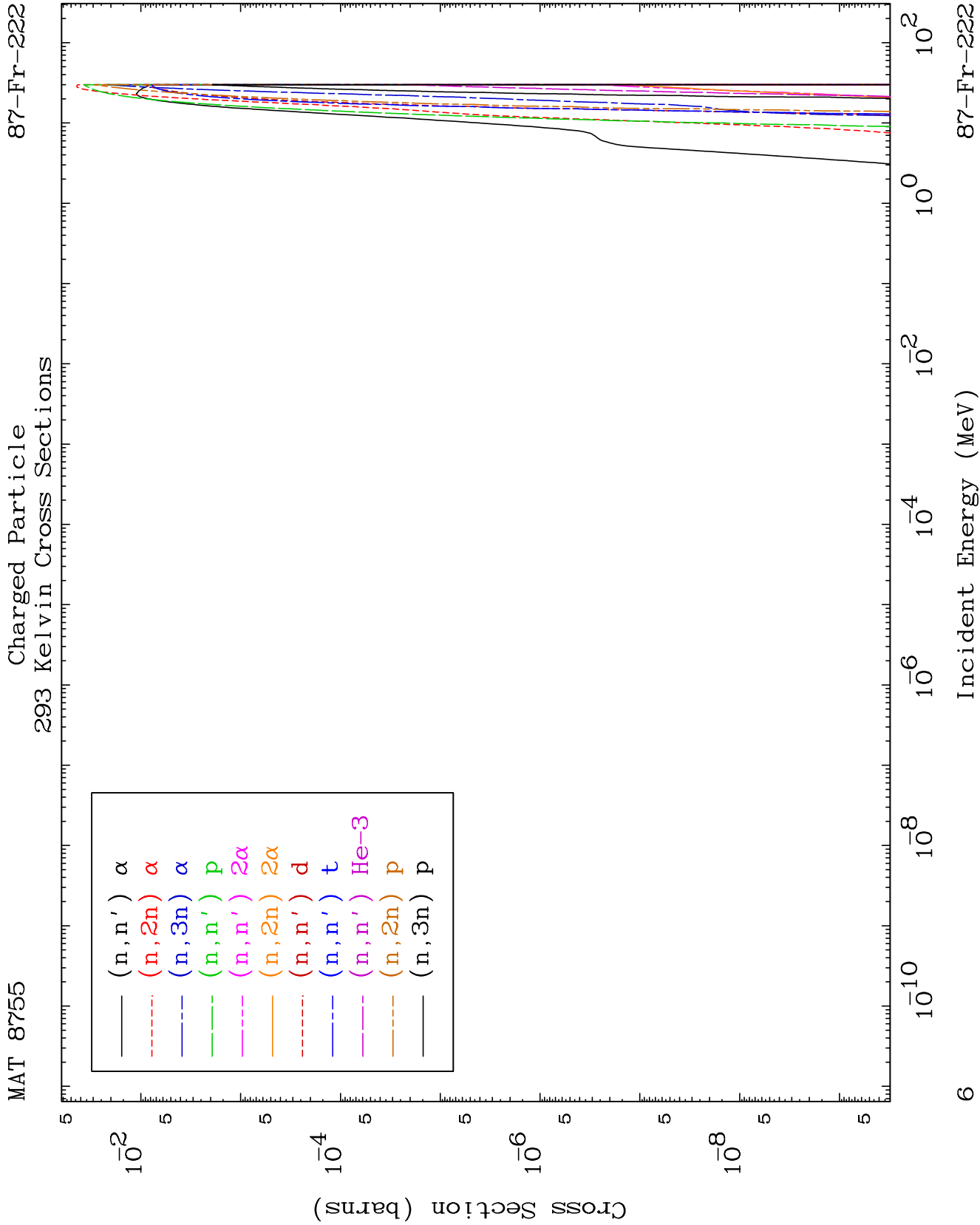
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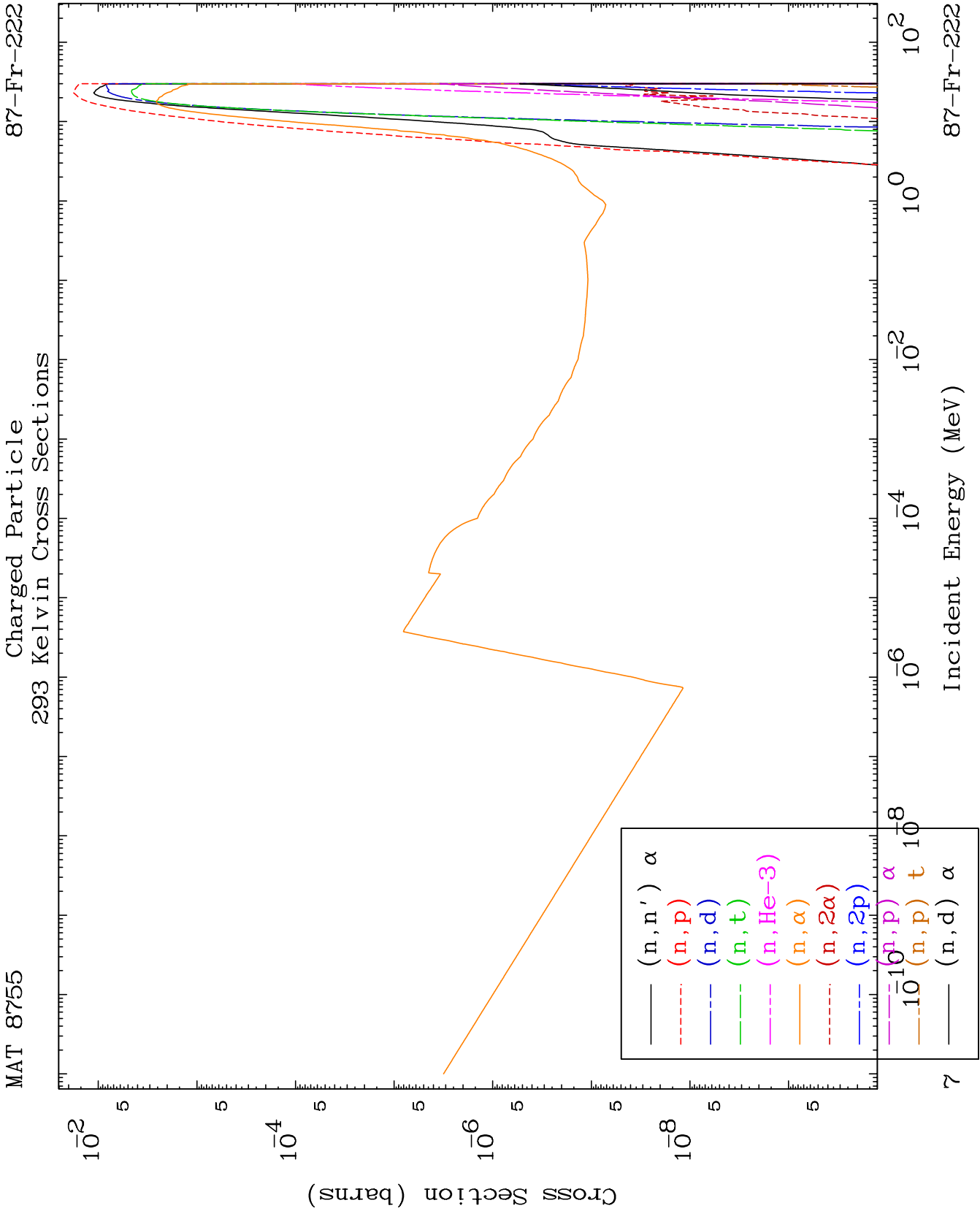


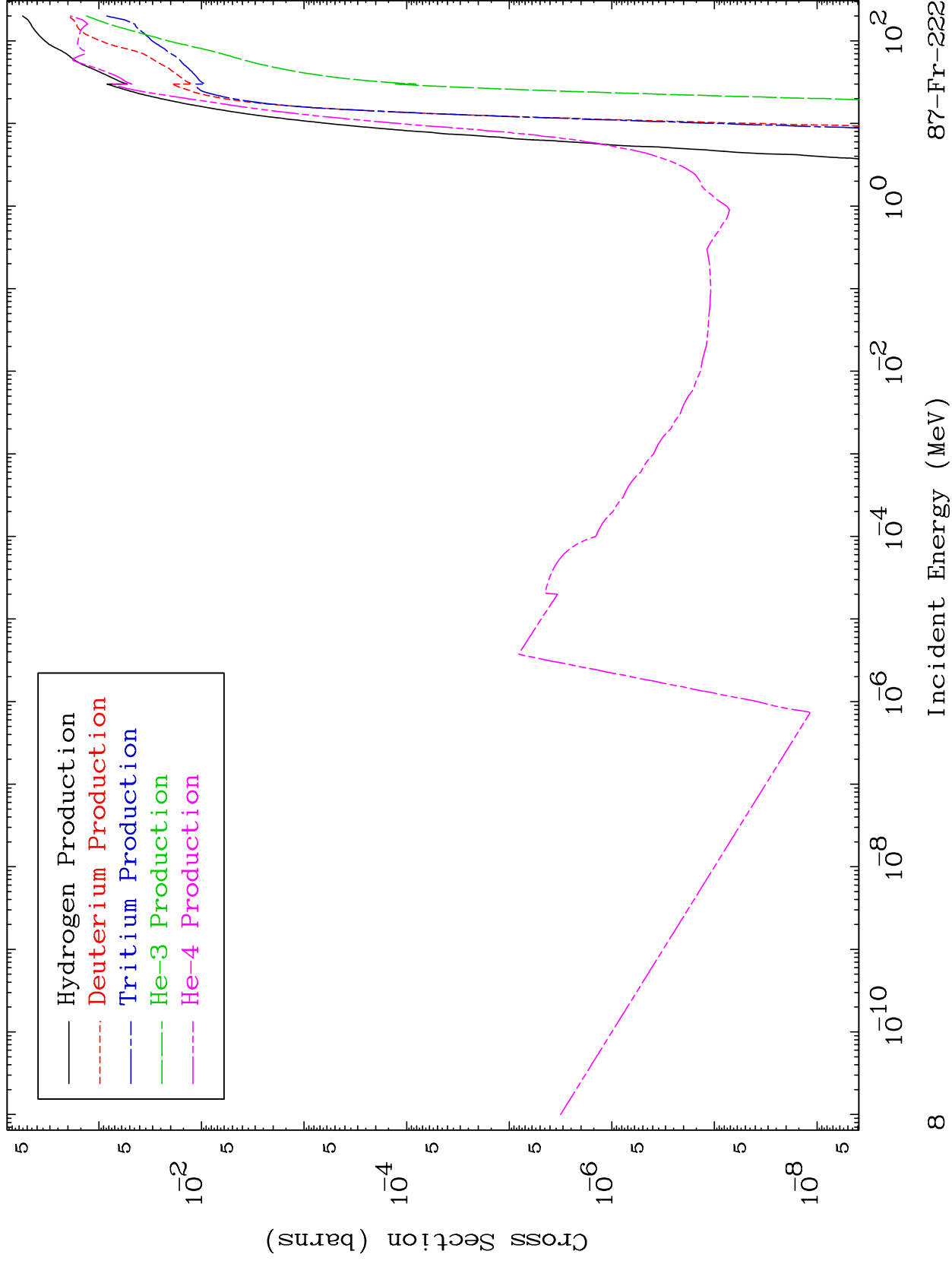
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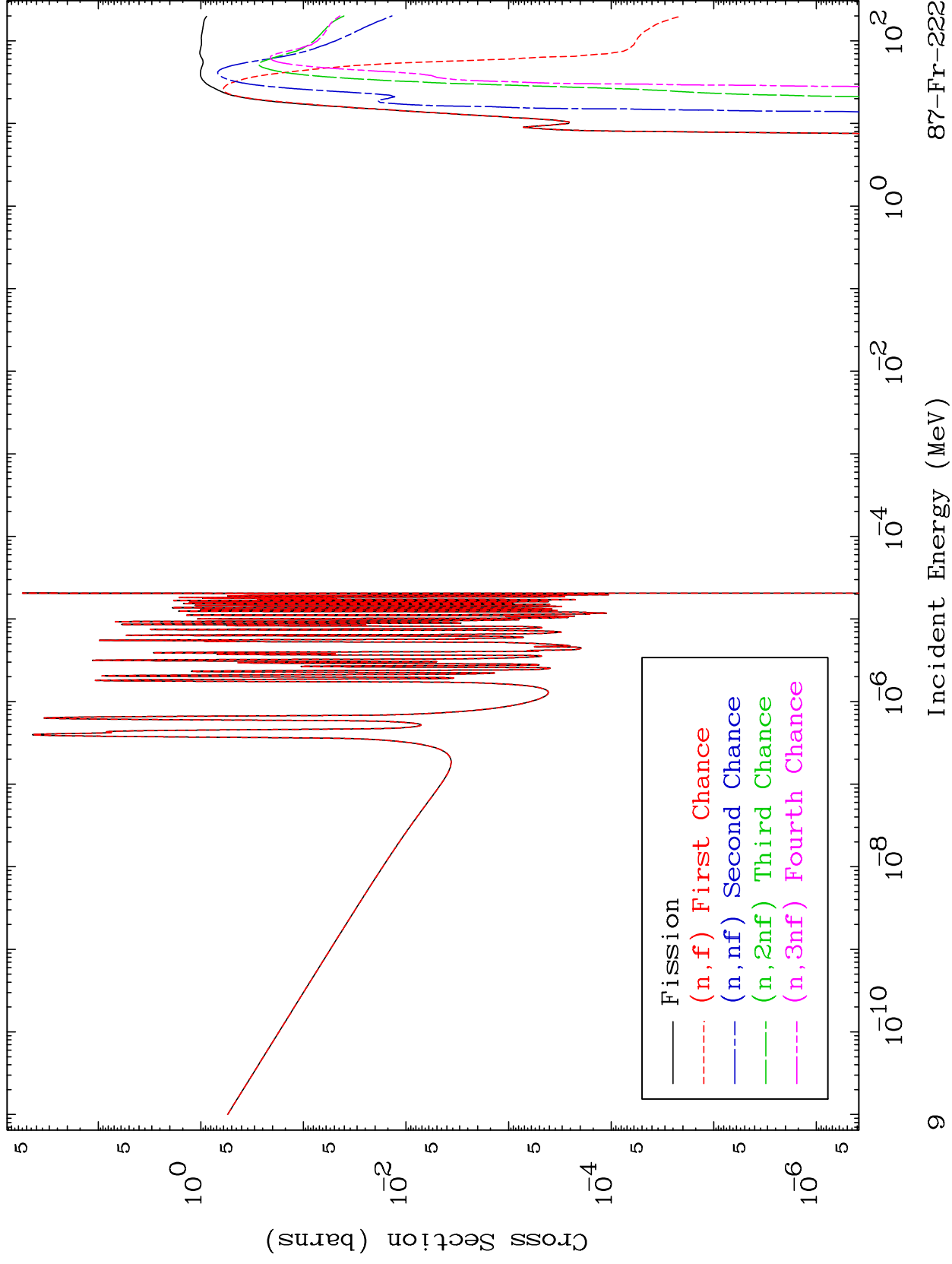
Incident Energy (MeV)

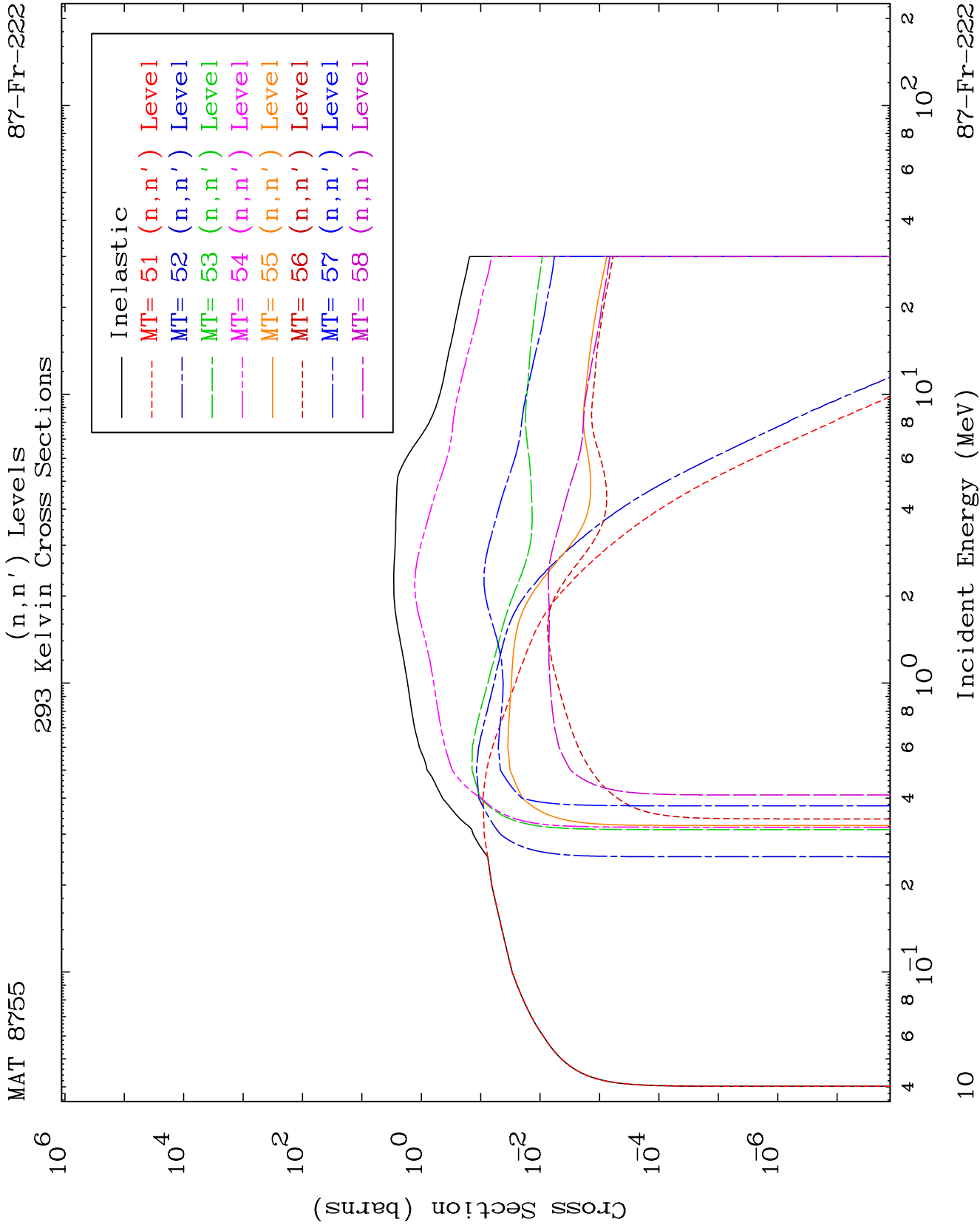
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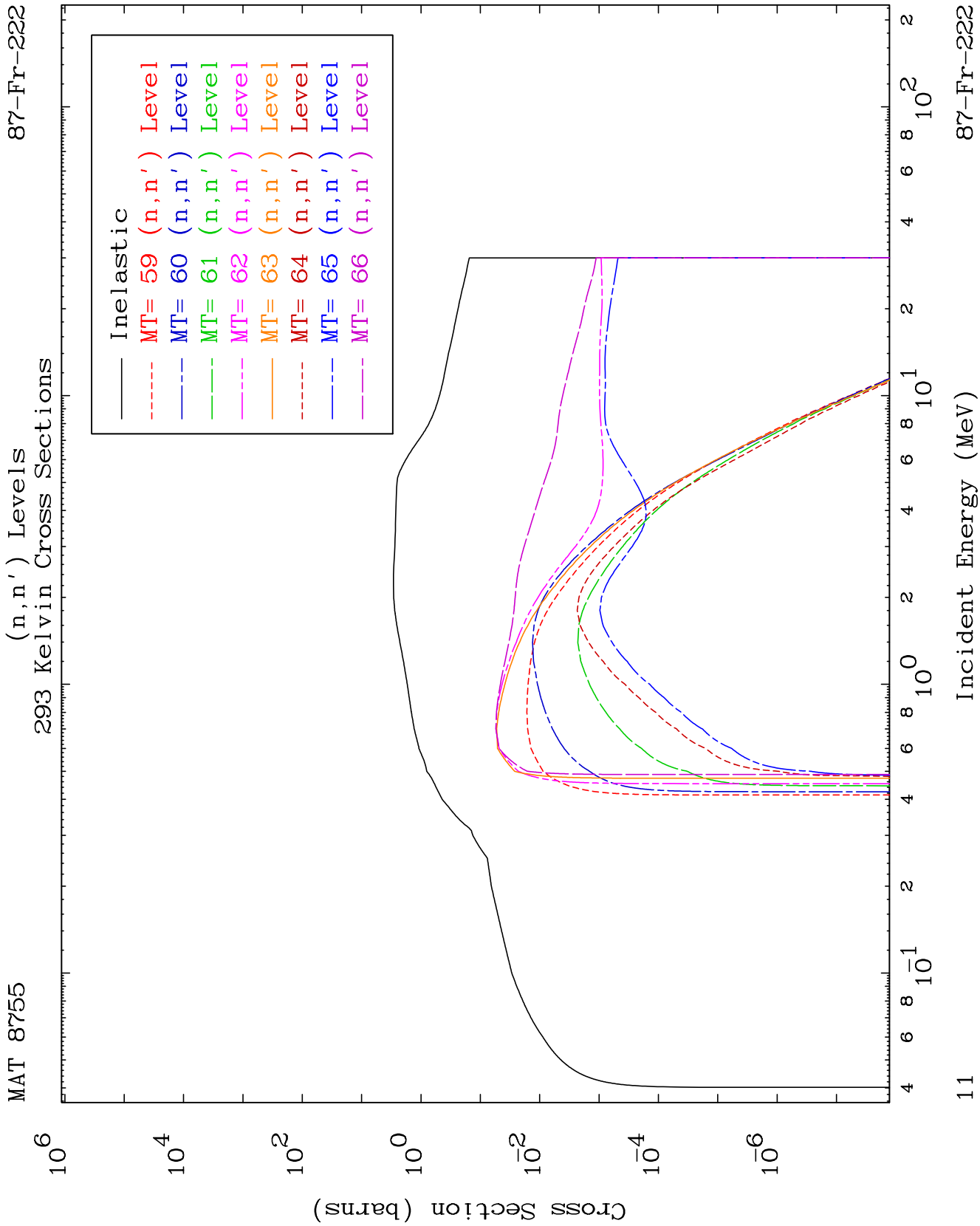


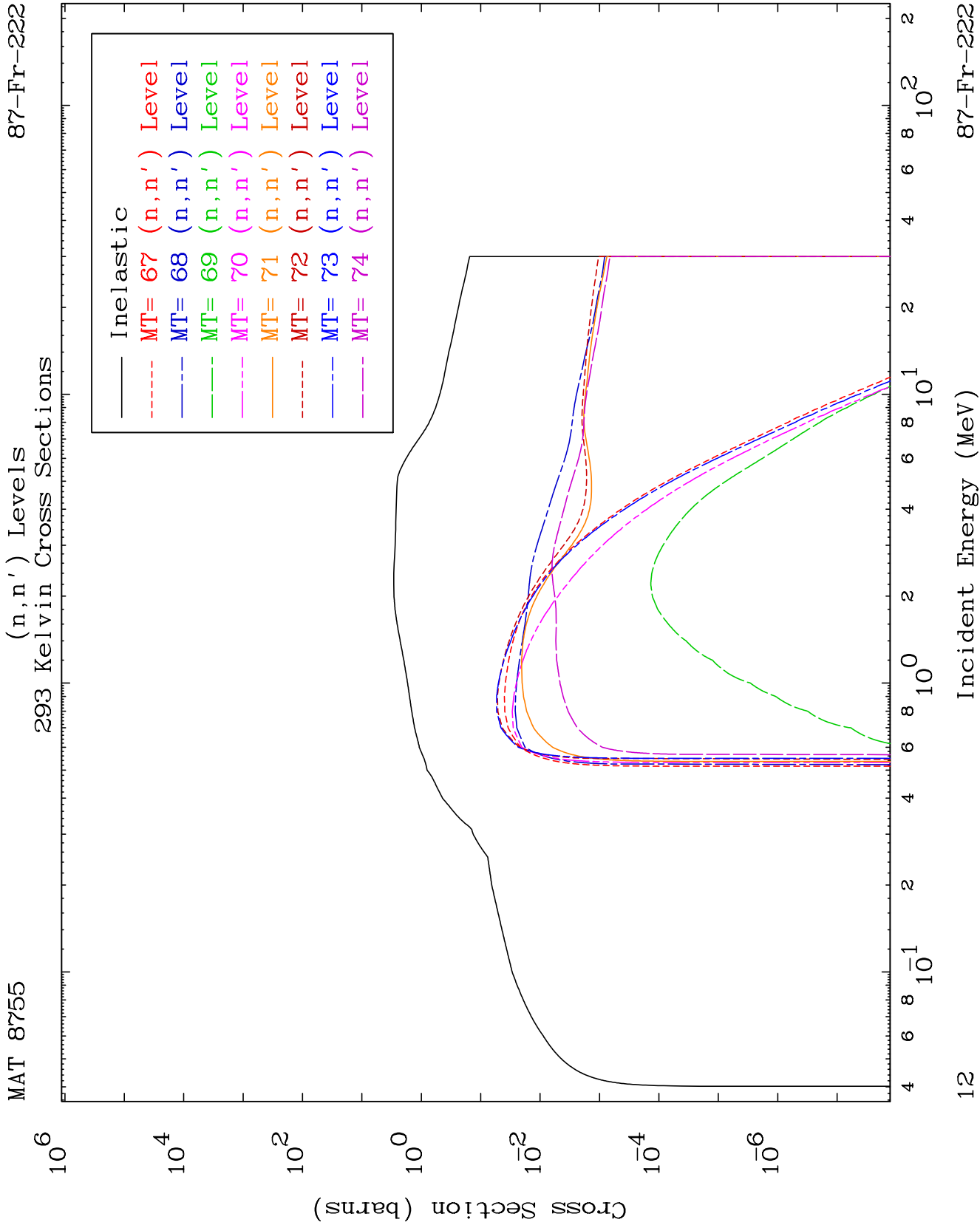


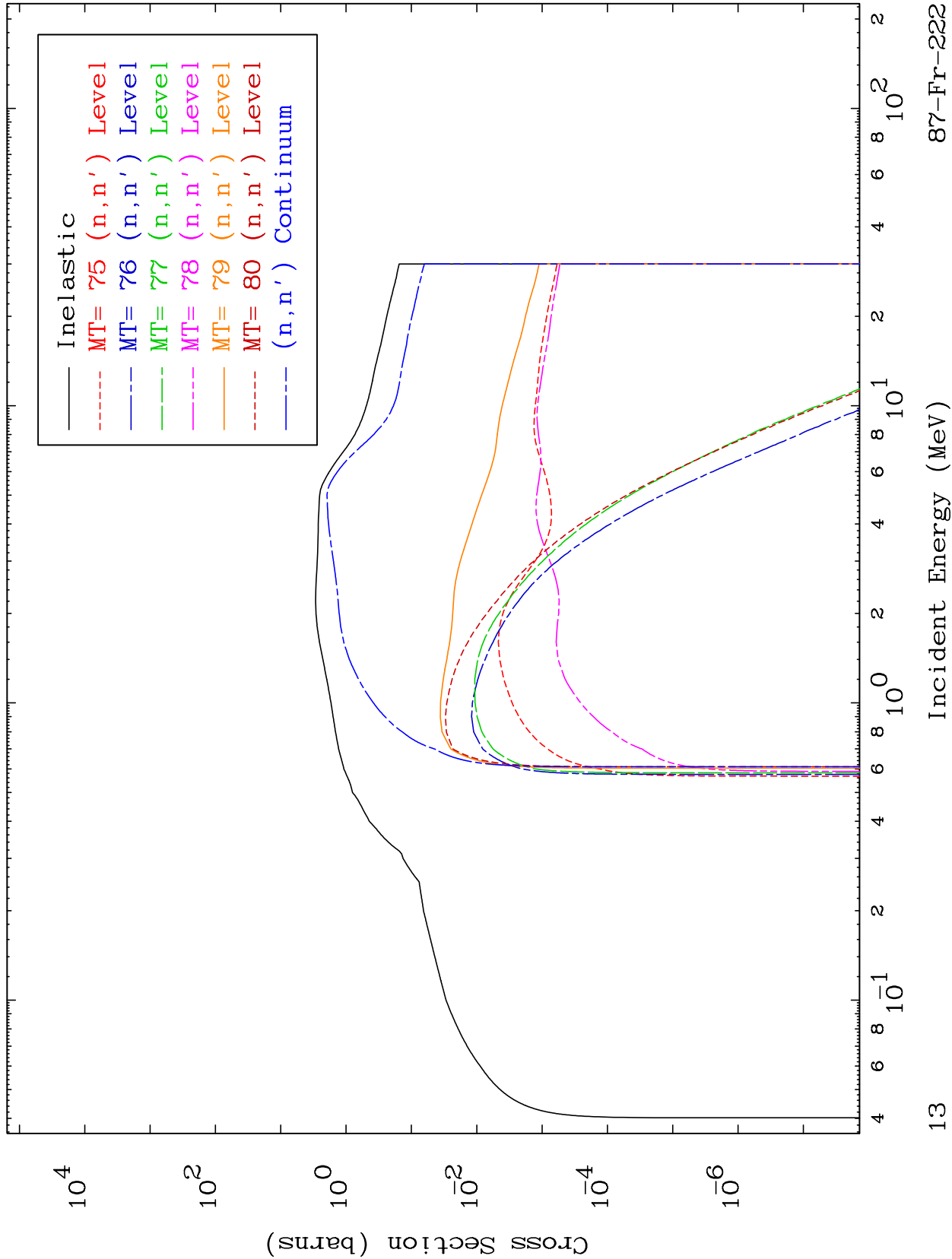








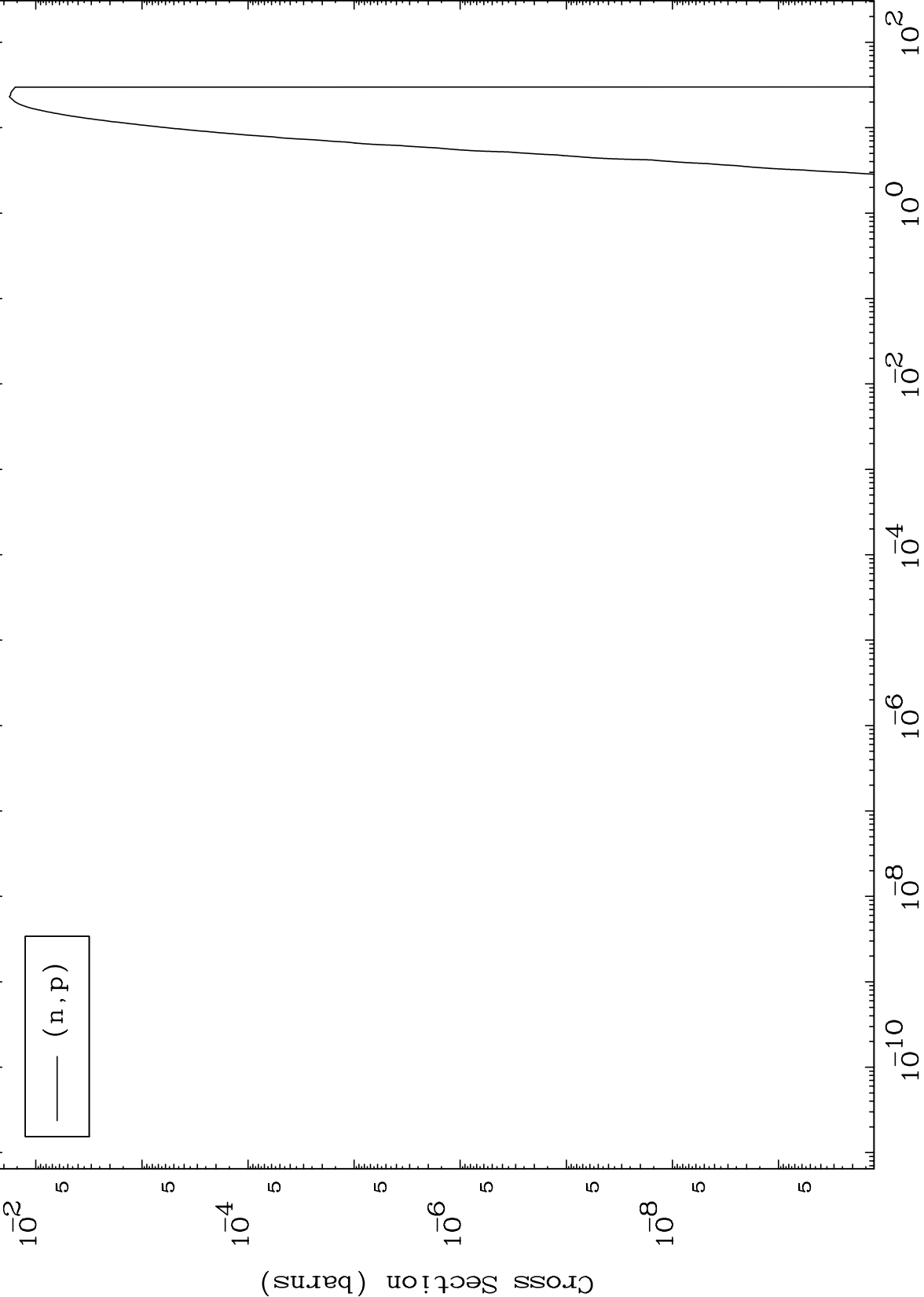




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(n,p) Levels
293 Kelvin Cross Sections

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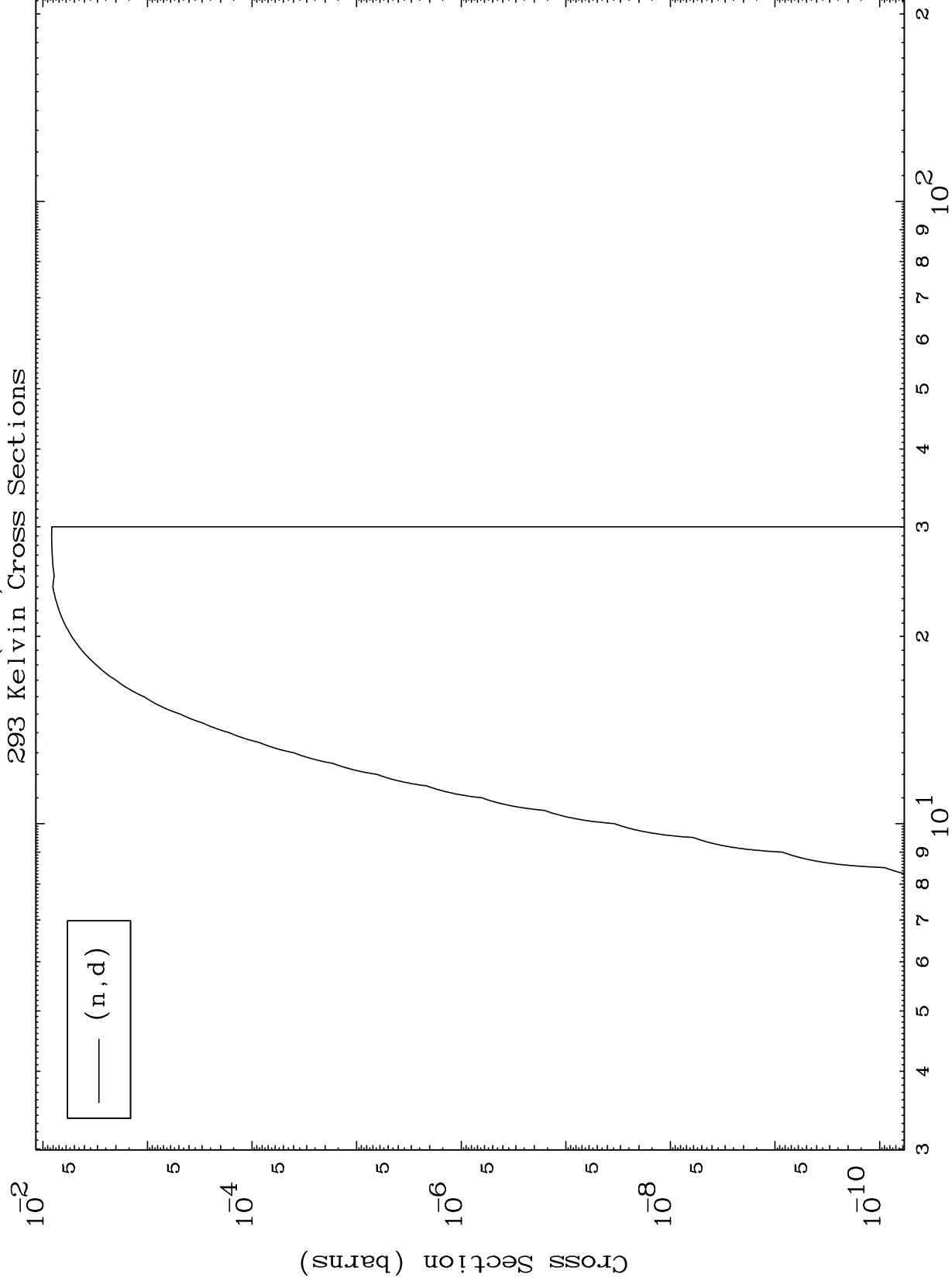


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(n,d) Levels

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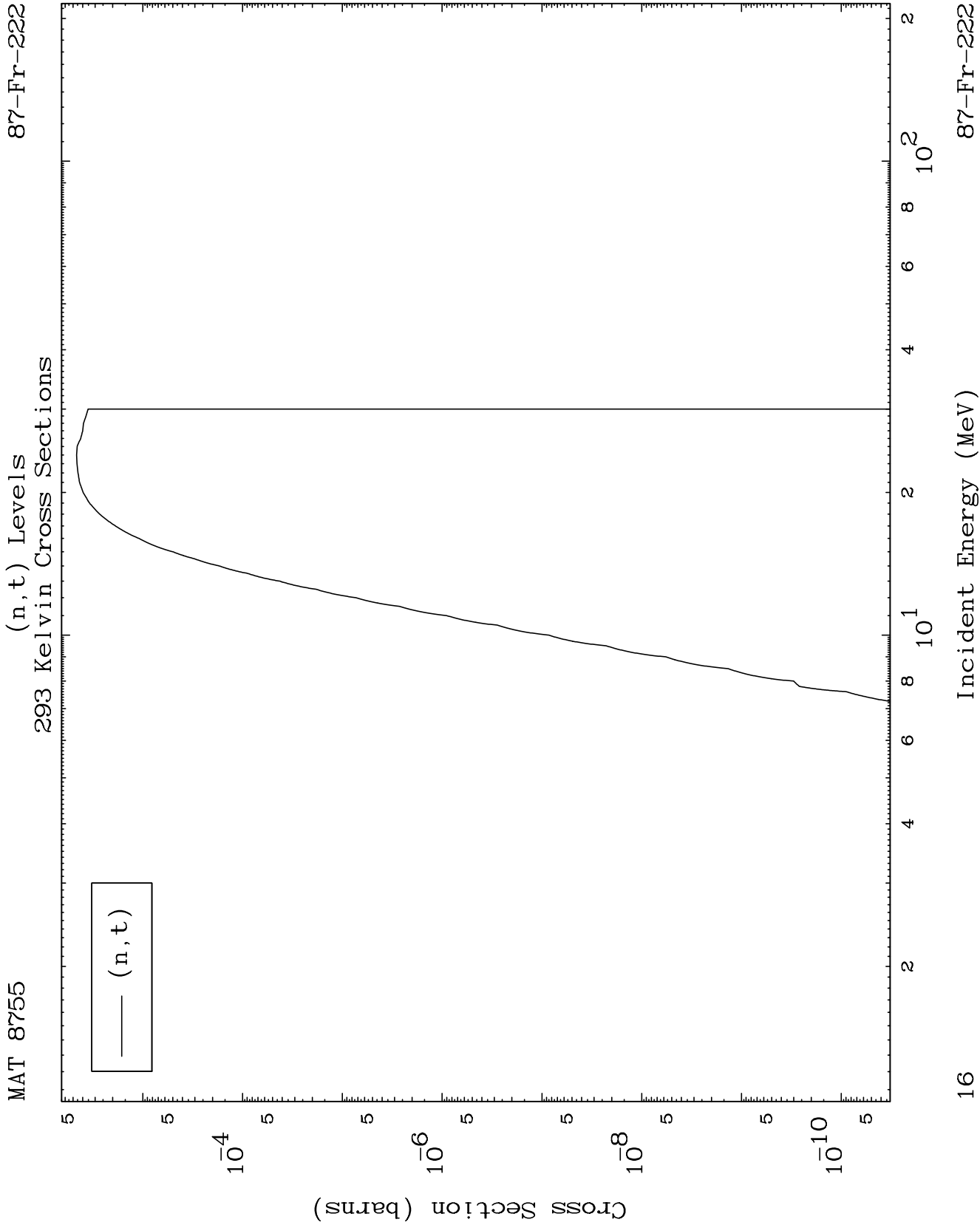
293 Kelvin Cross Sections



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Incident Energy (MeV)

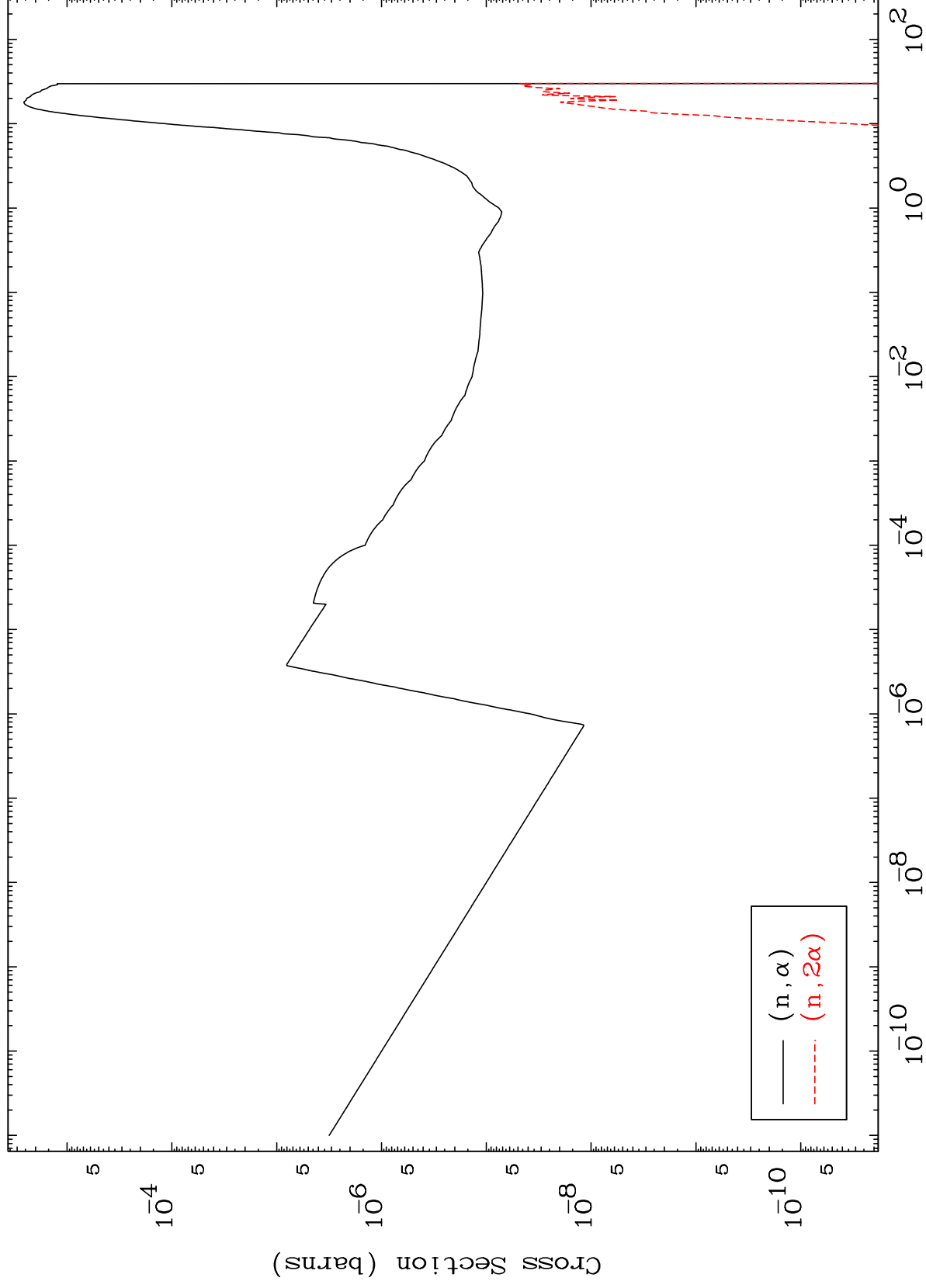
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(n, α) Levels
293 Kelvin Cross Sections

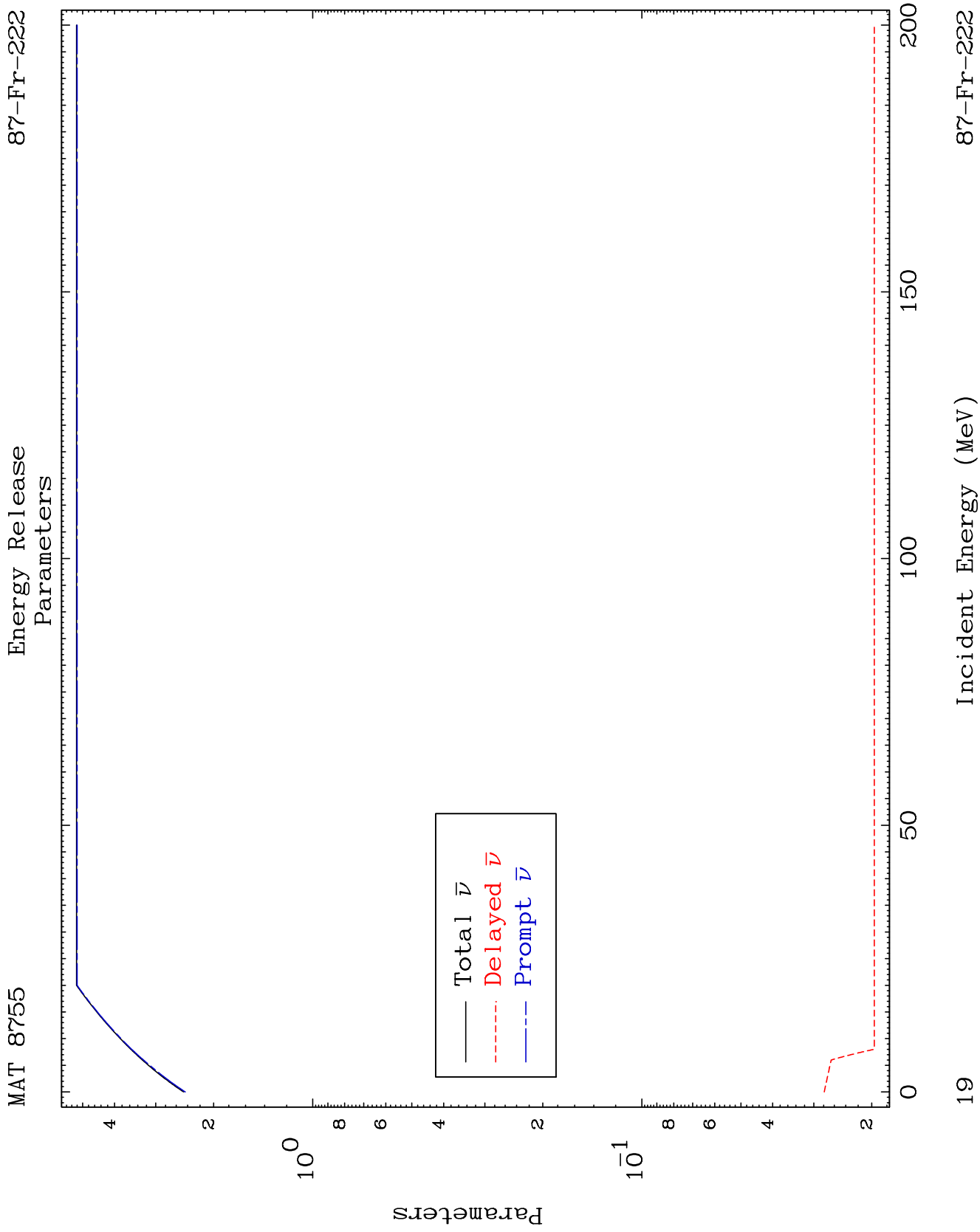
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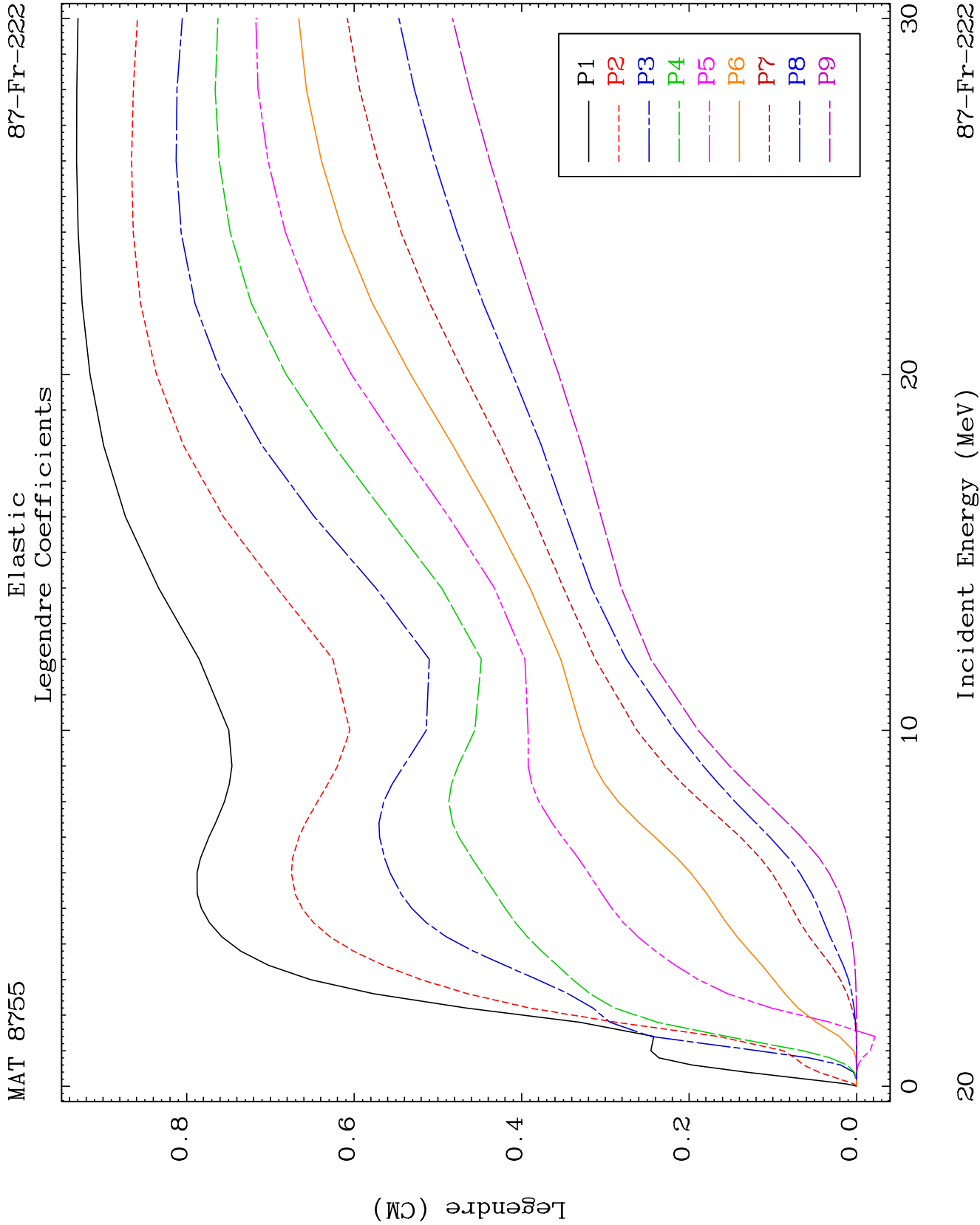


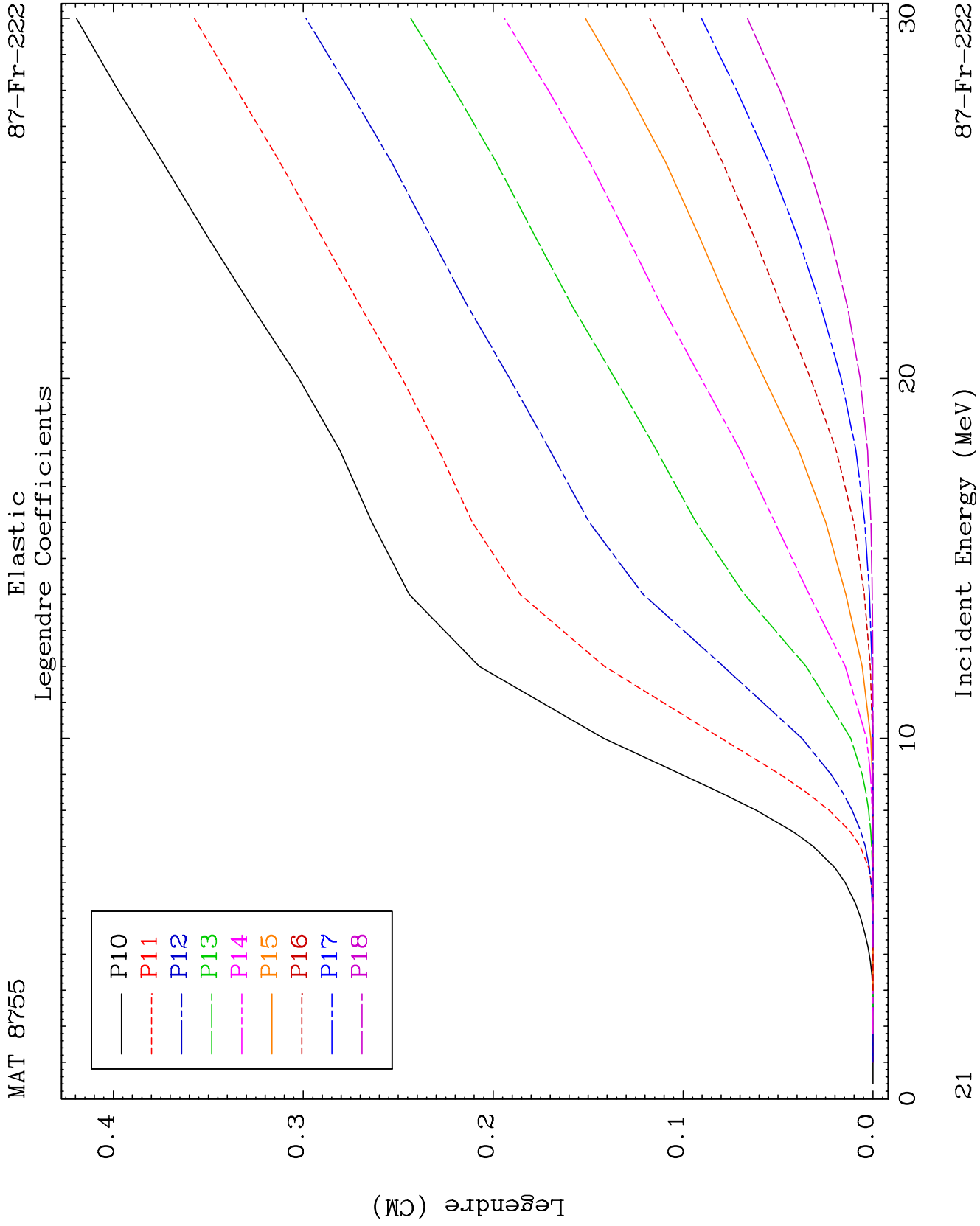
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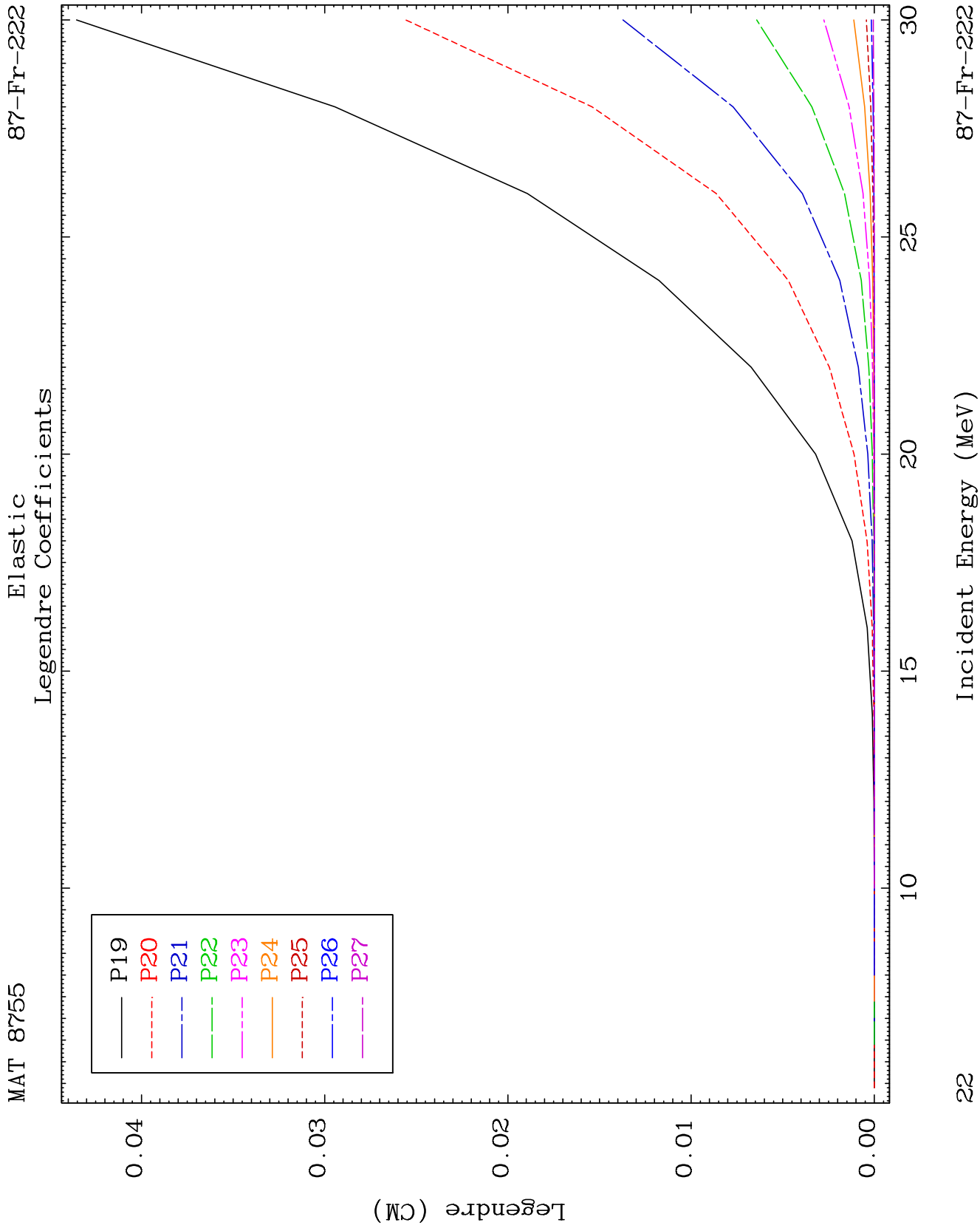
Incident Energy (MeV)

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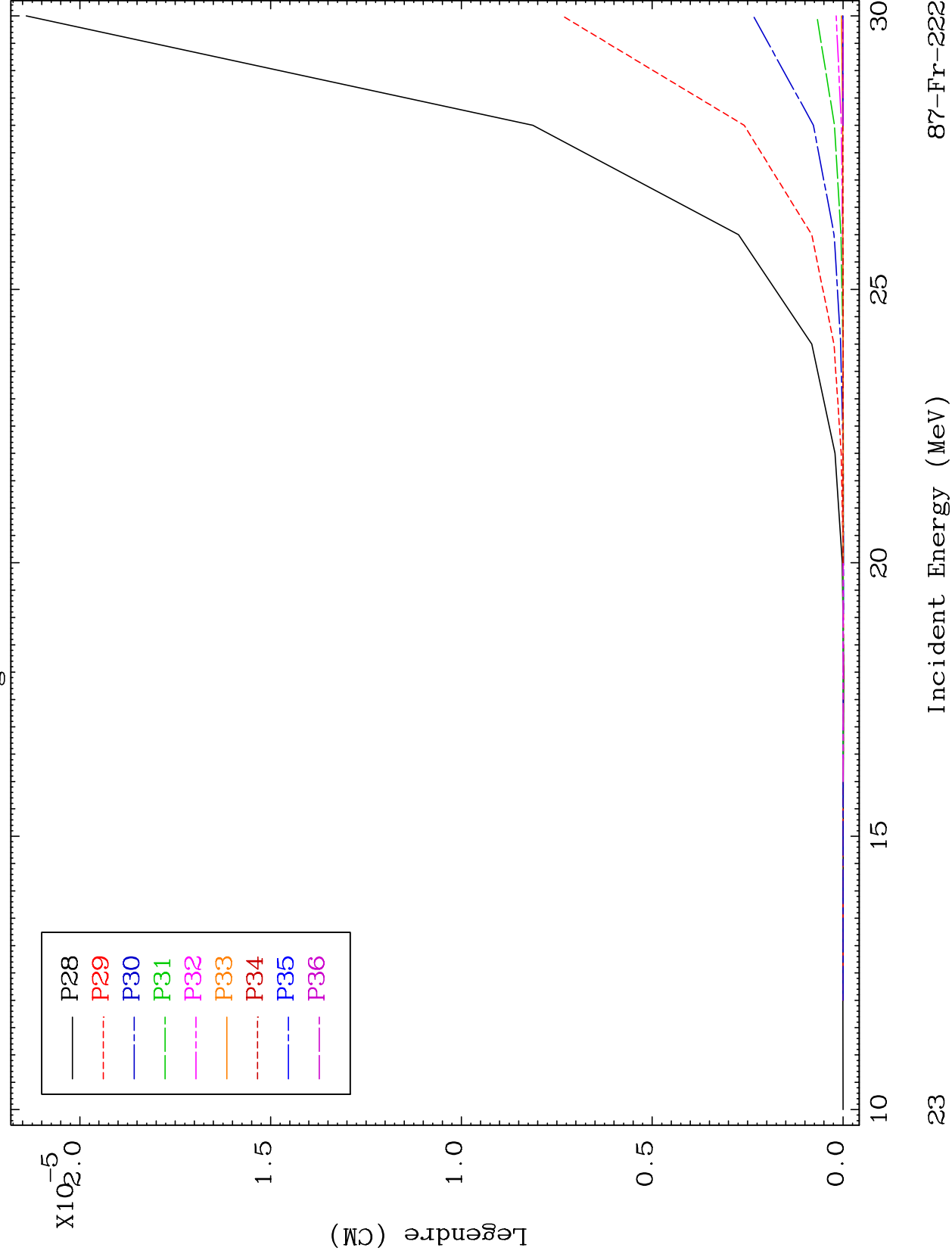




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Elastic Legendre Coefficients

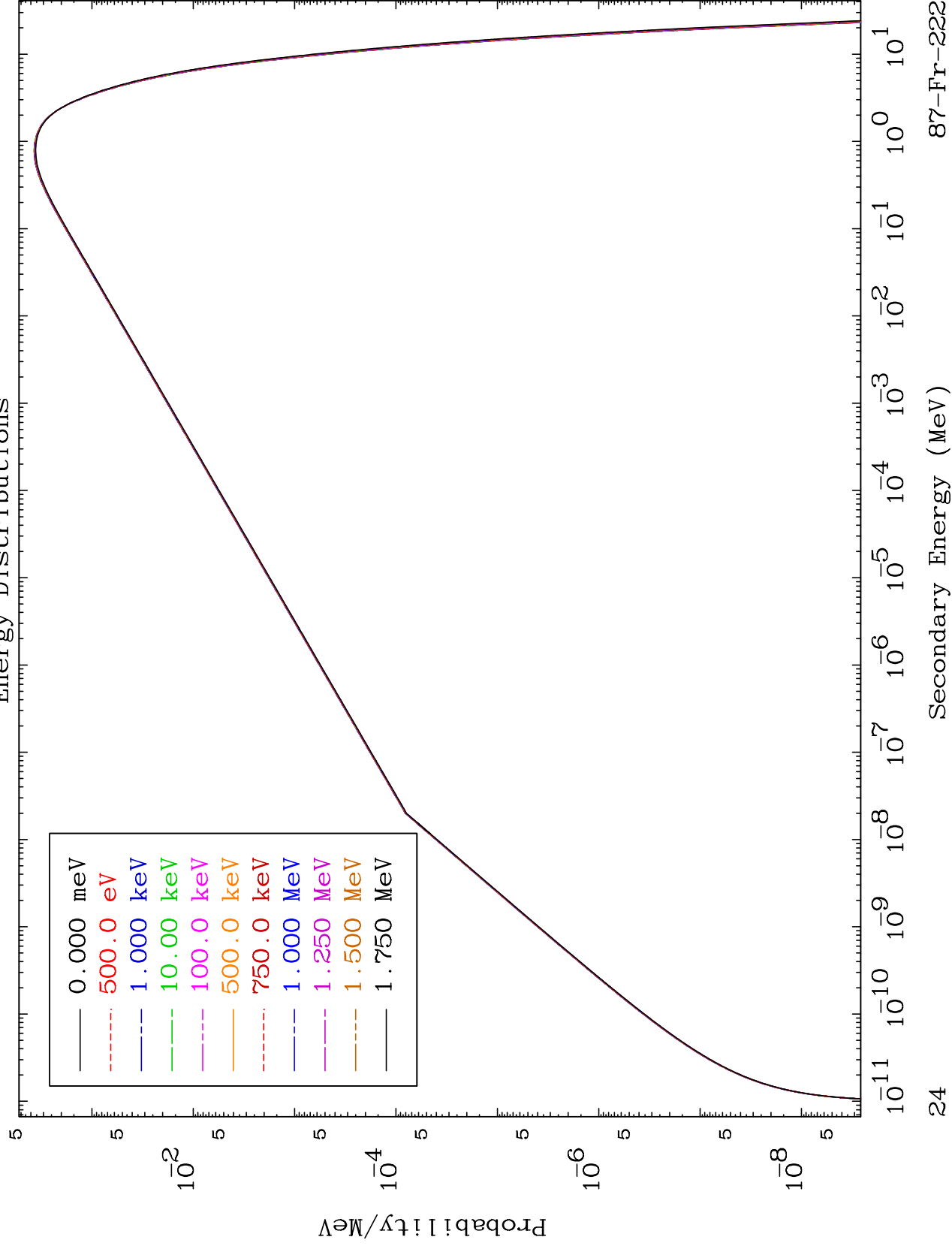
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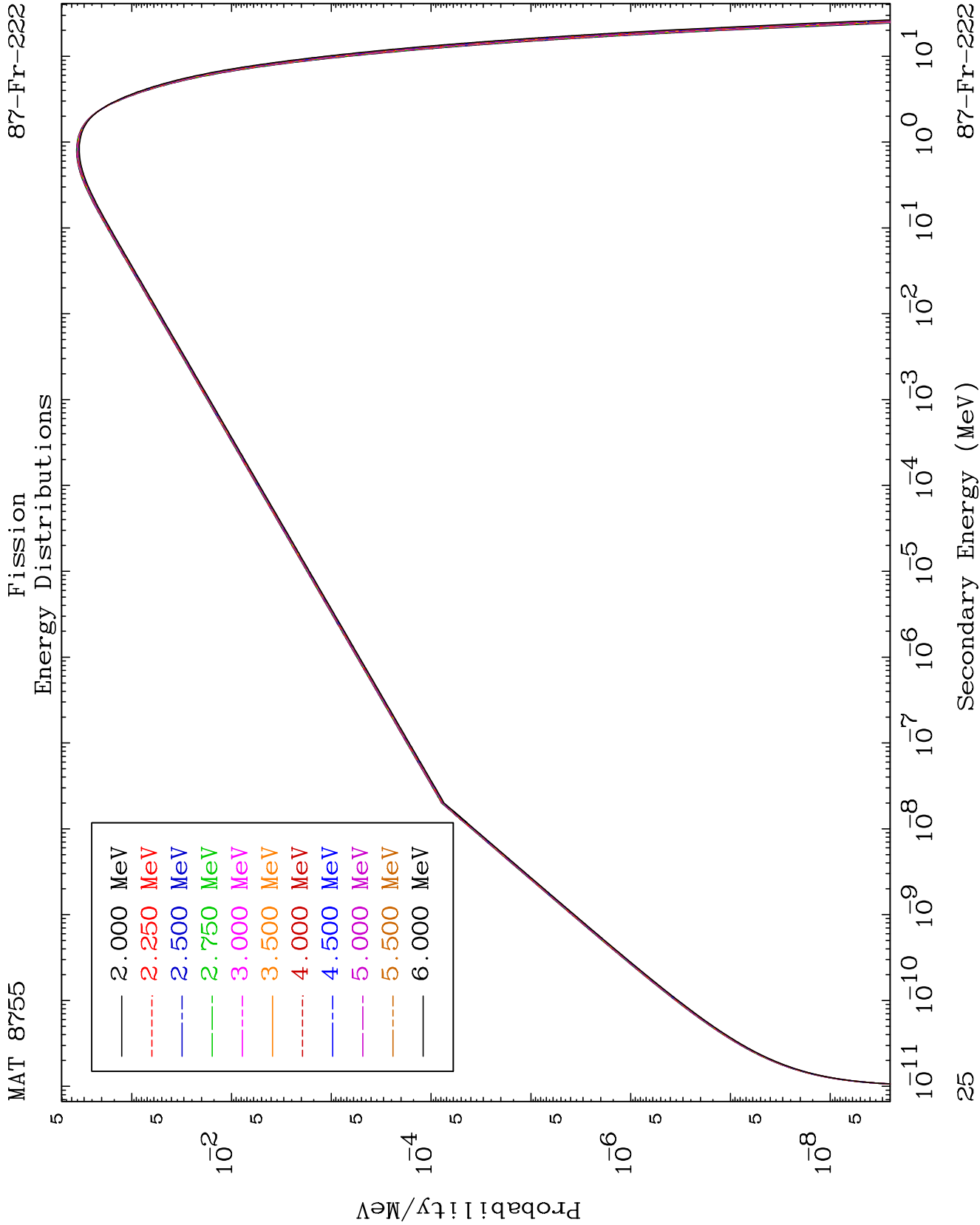


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Fission
Energy Distributions

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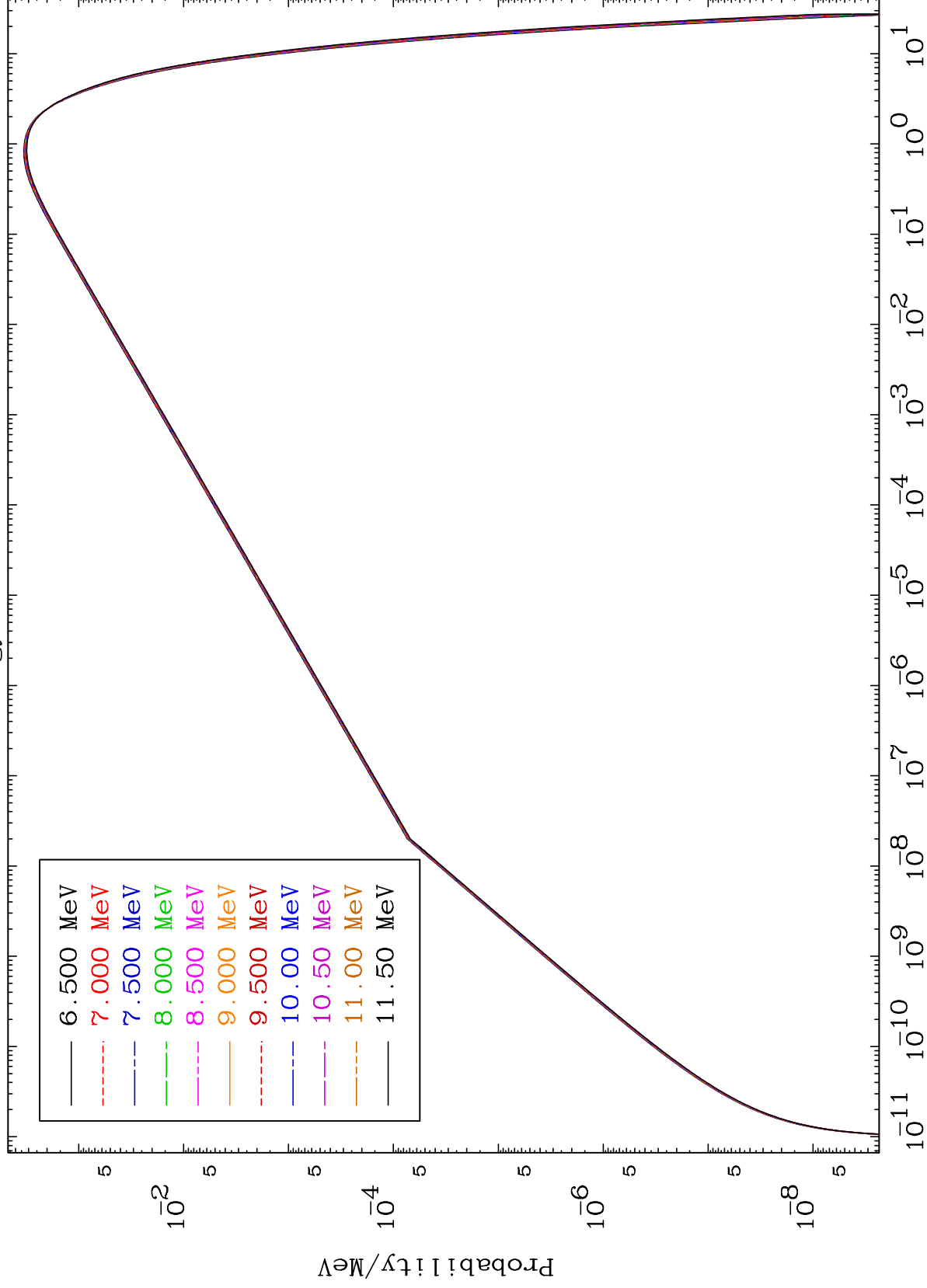




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Fission
Energy Distributions

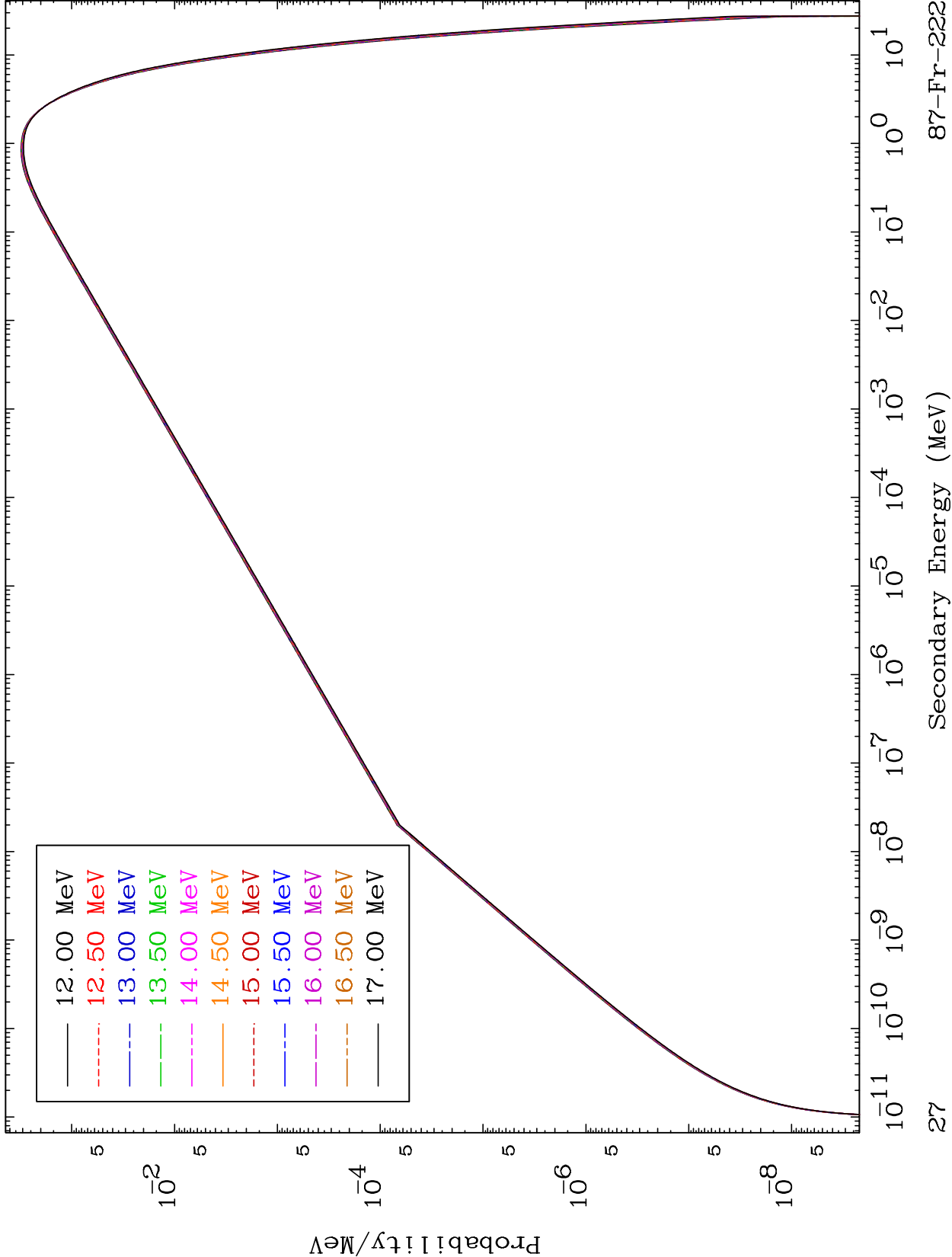
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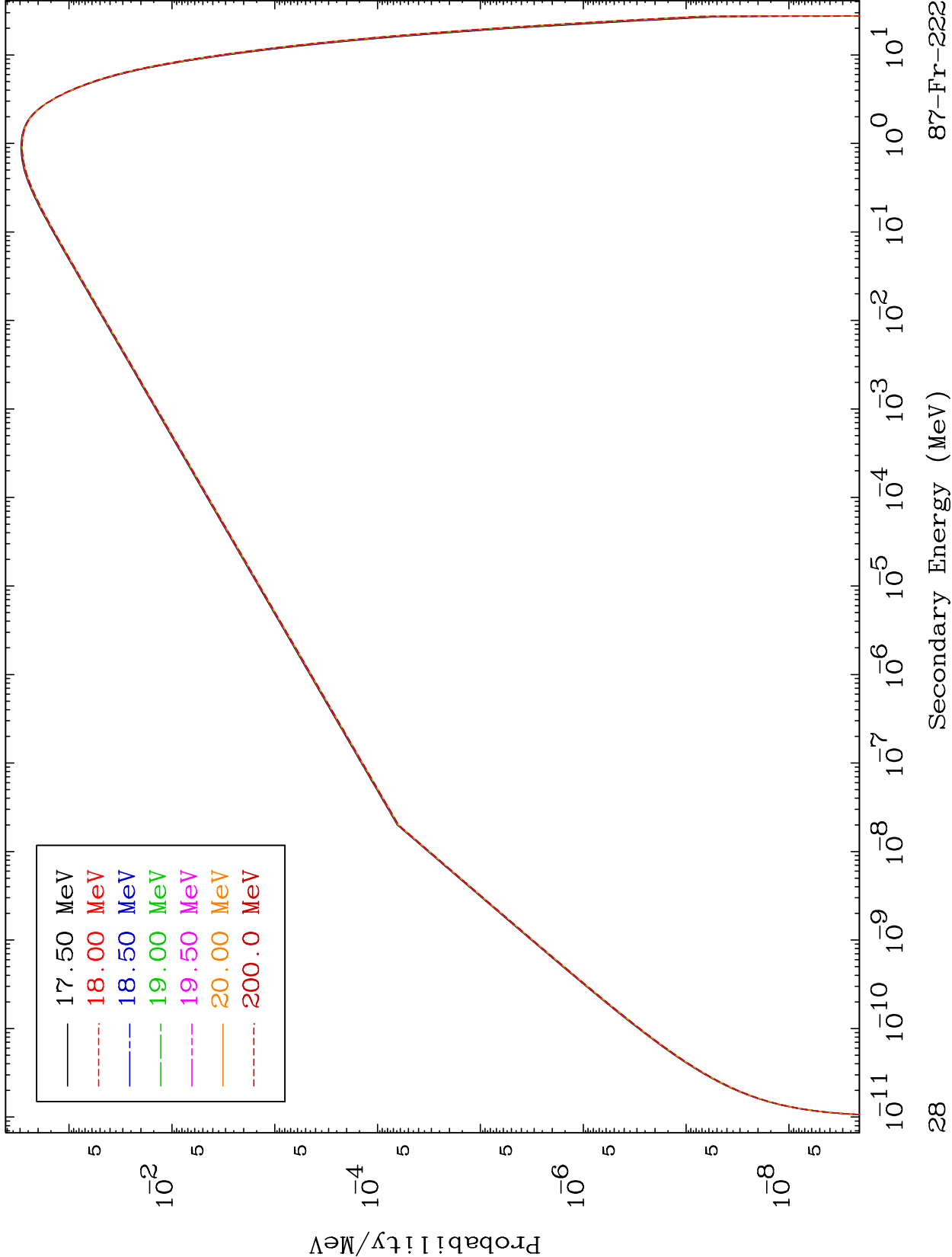


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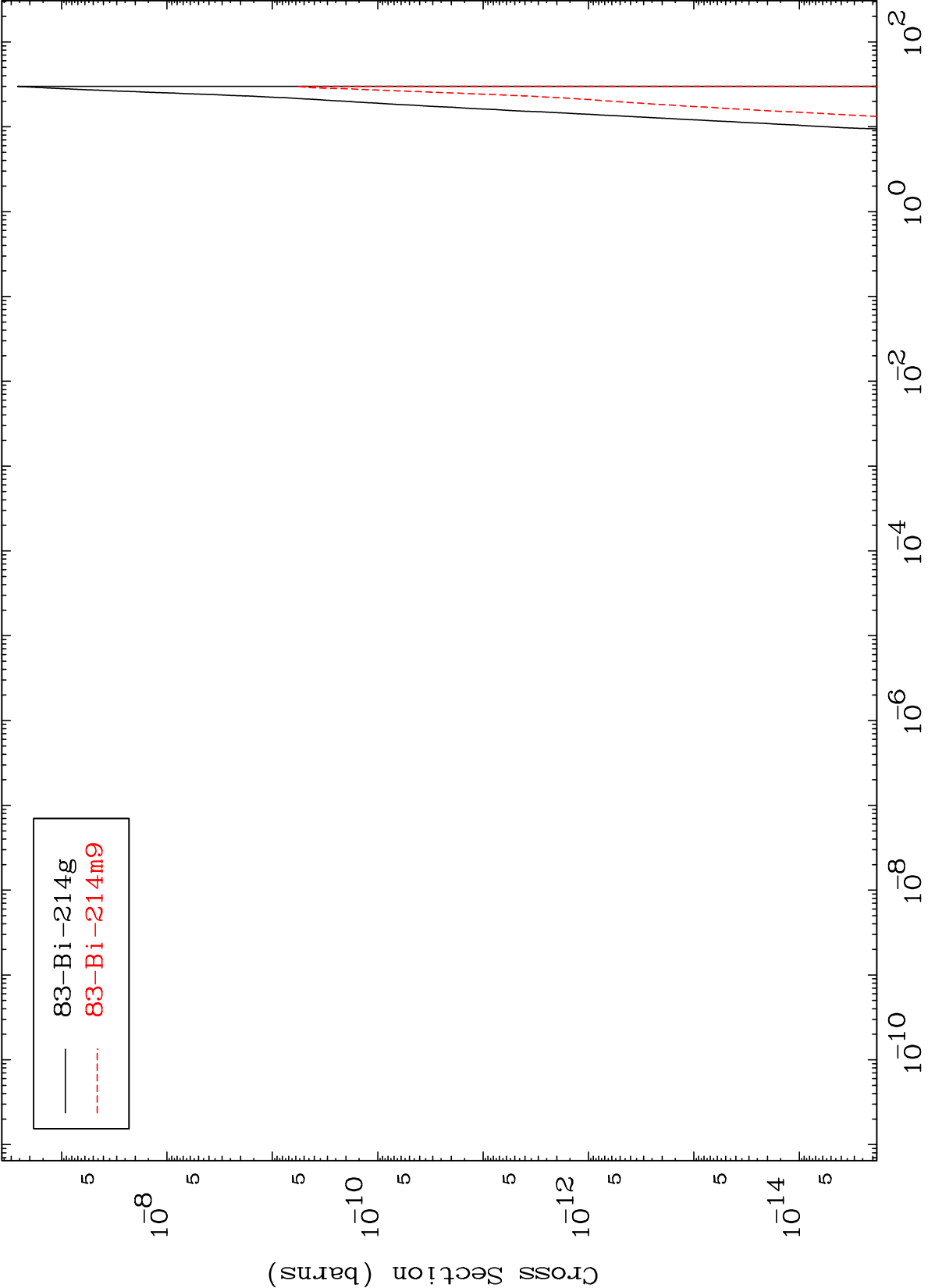
Secondary Energy (MeV)

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Radionuclide Production Cross Section

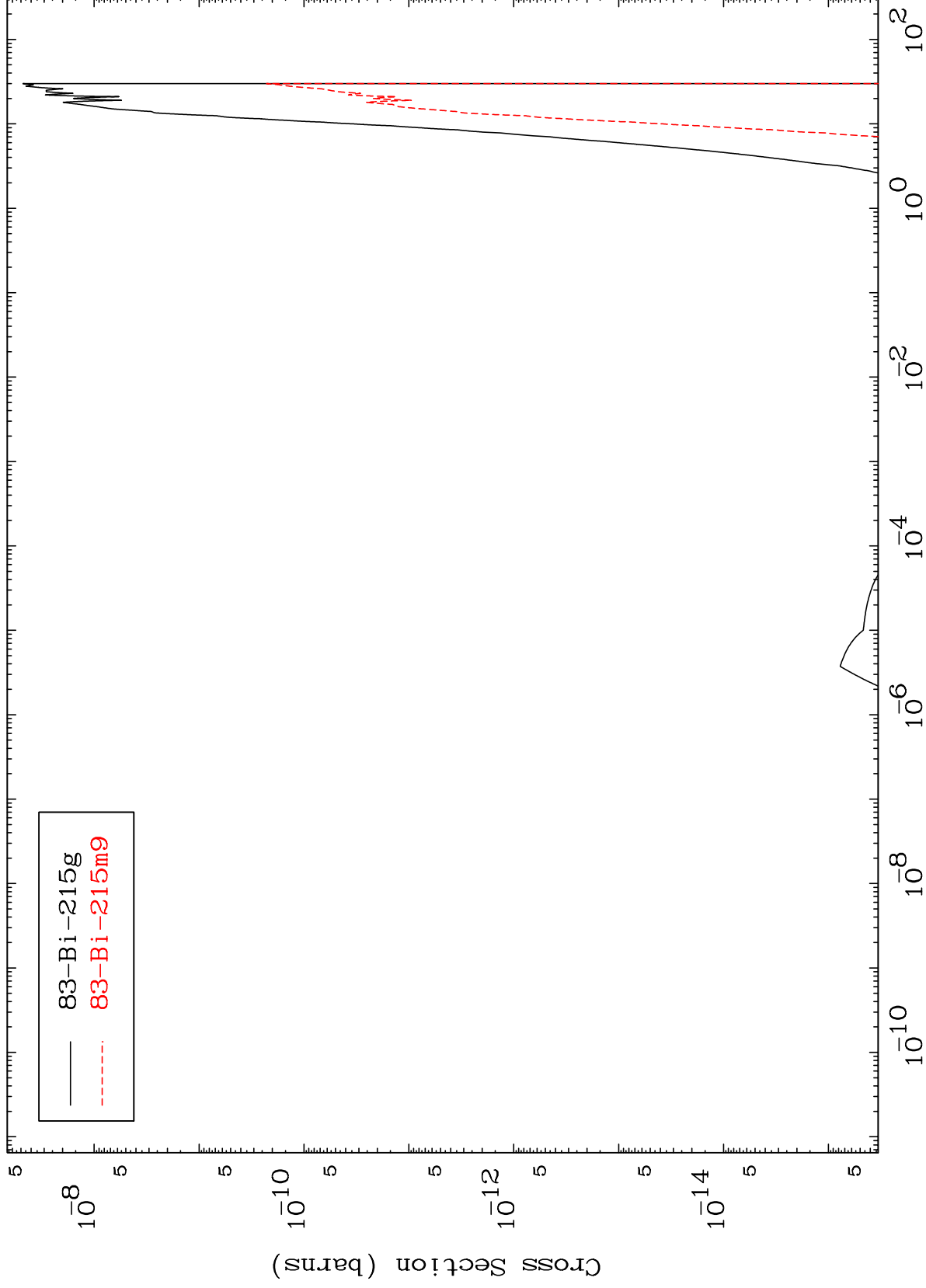


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(n,2α)

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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