

Program EVALPLOT
(Version 2021-1)

by

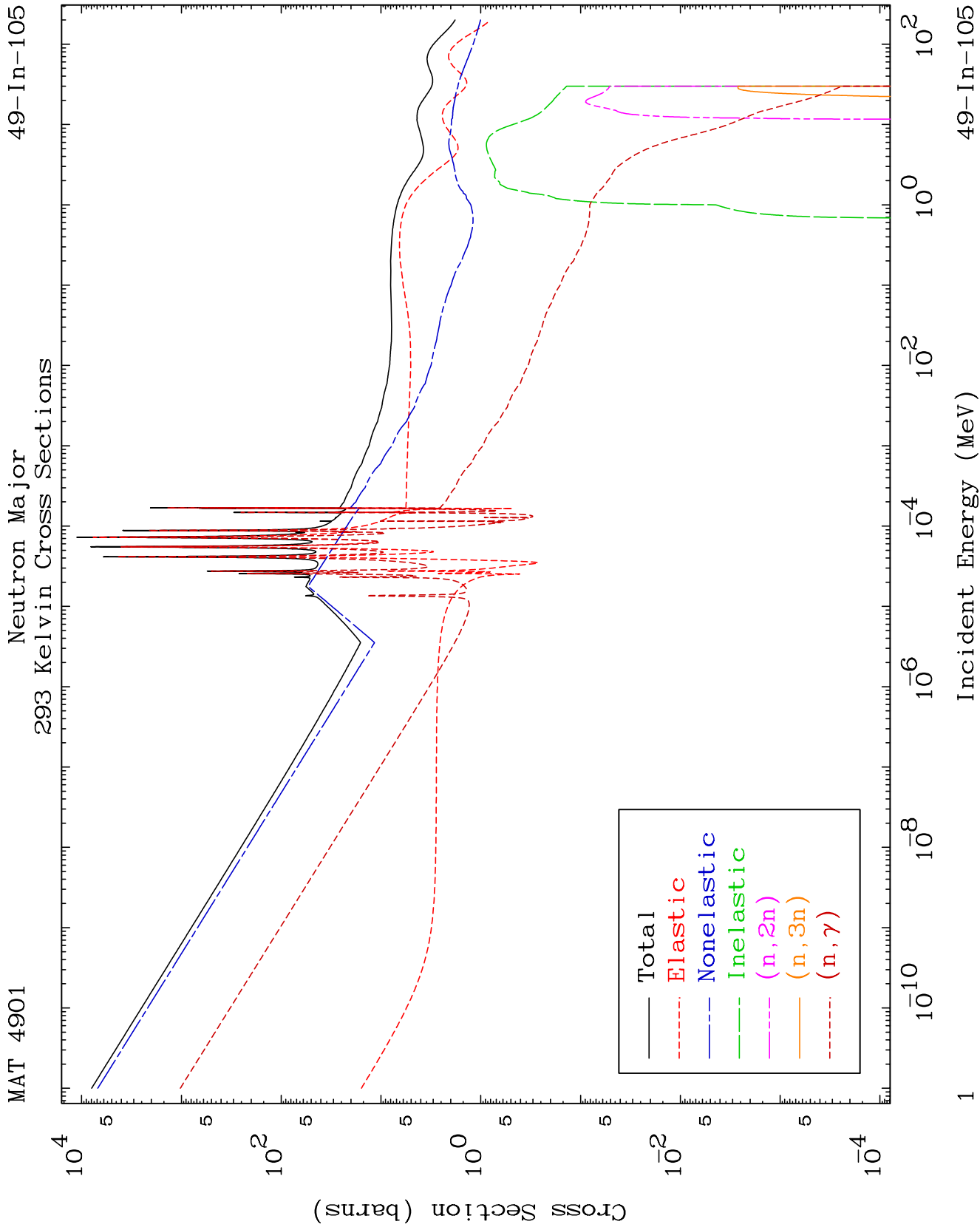
Dermott E. Cullen
(Present Contact Information)

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Livermore, CA 94550
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Press Mouse Button to Start

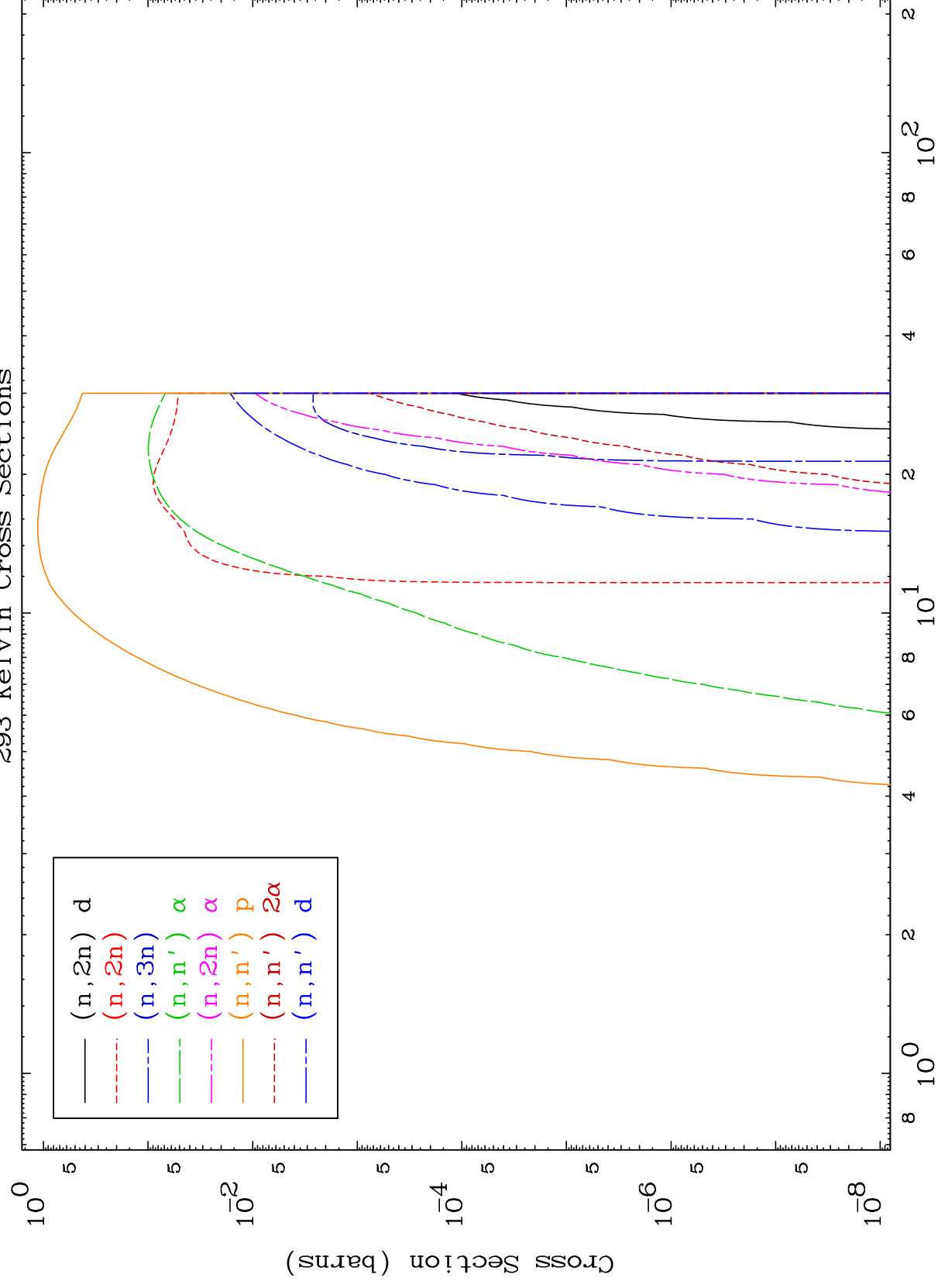


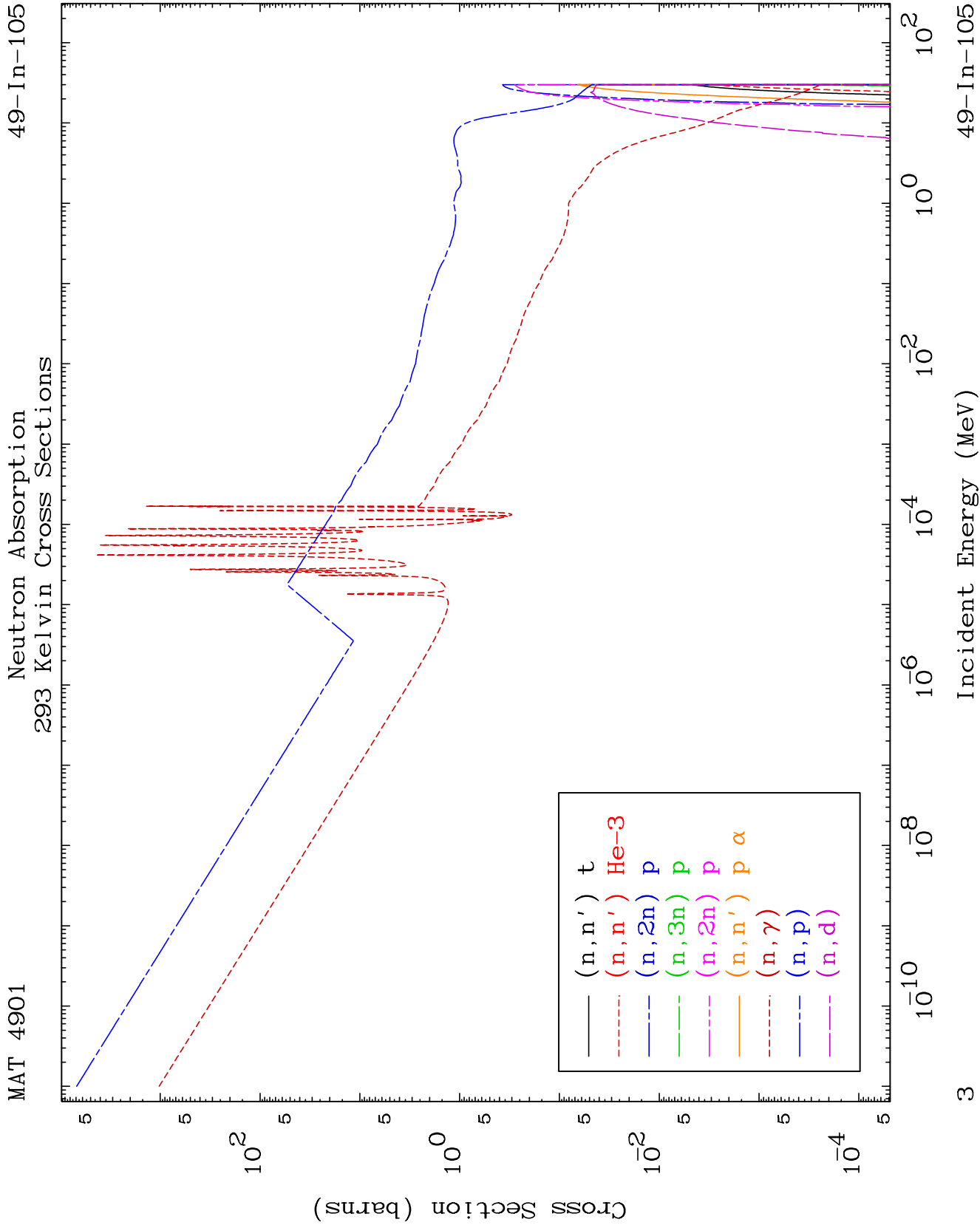
MAT 4901

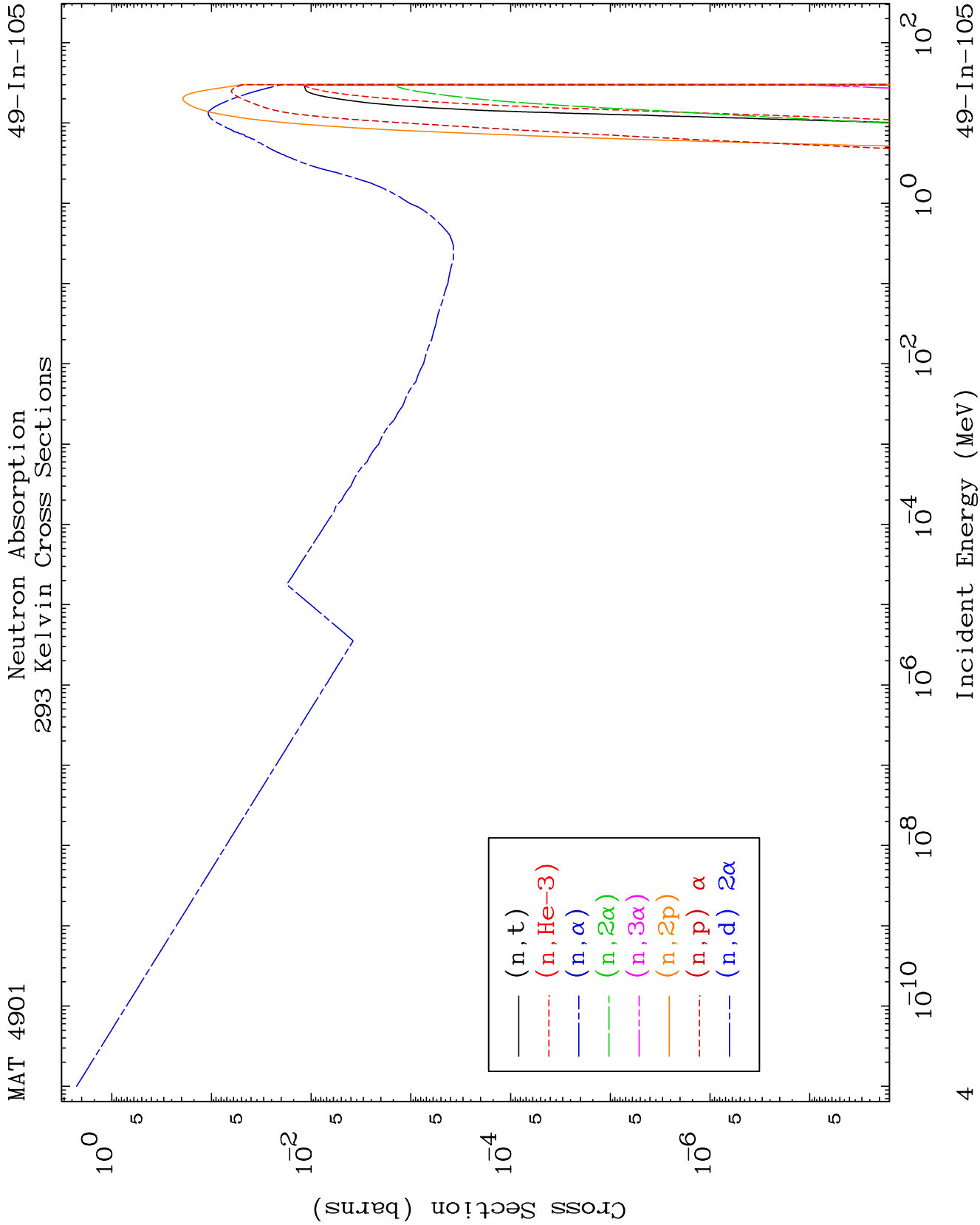
Neutron Absorption

Neutron Absorption 293 Kelvin Cross Sections

49-In-105



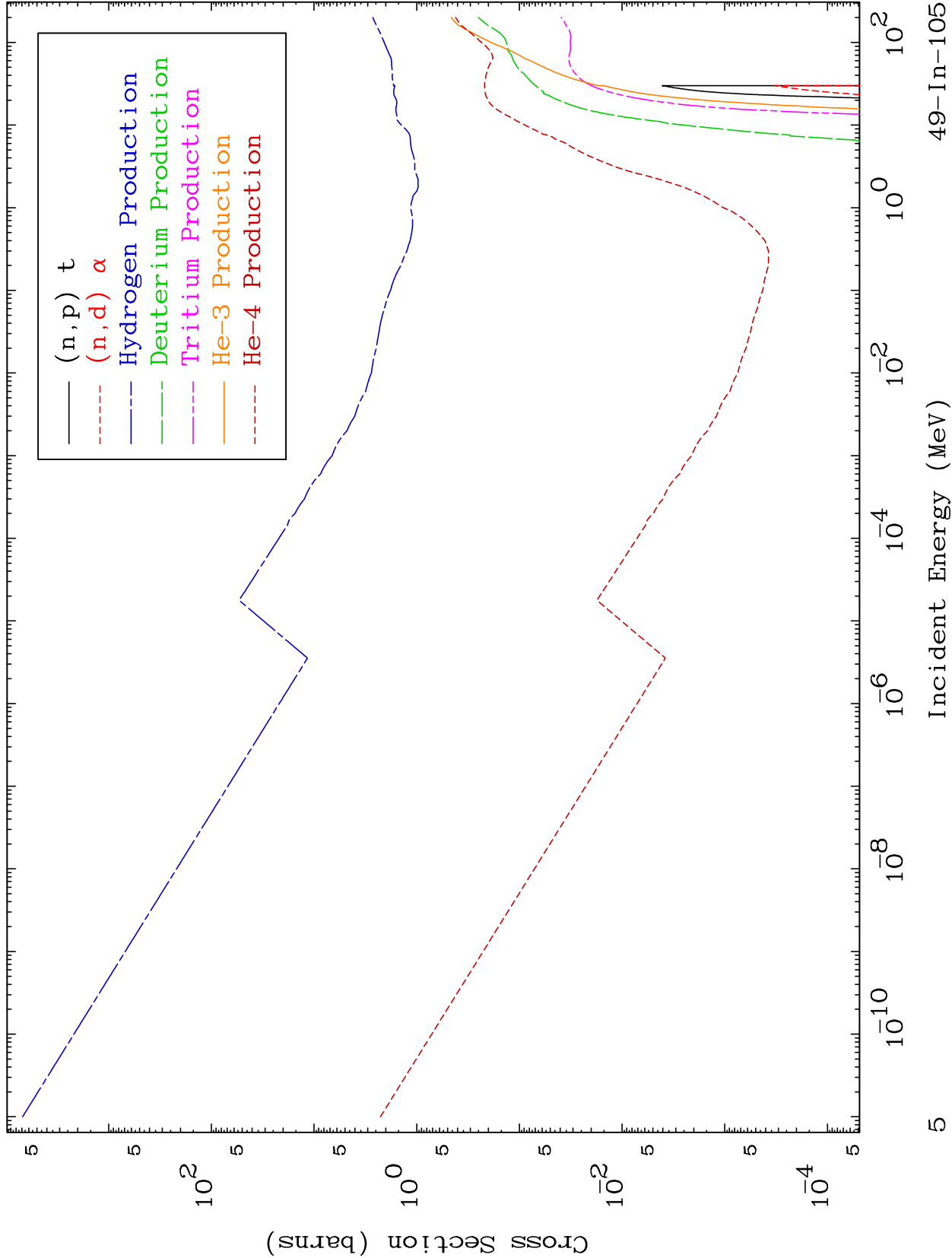




MAT 4901

Neutron Absorption
293 Kelvin Cross Sections

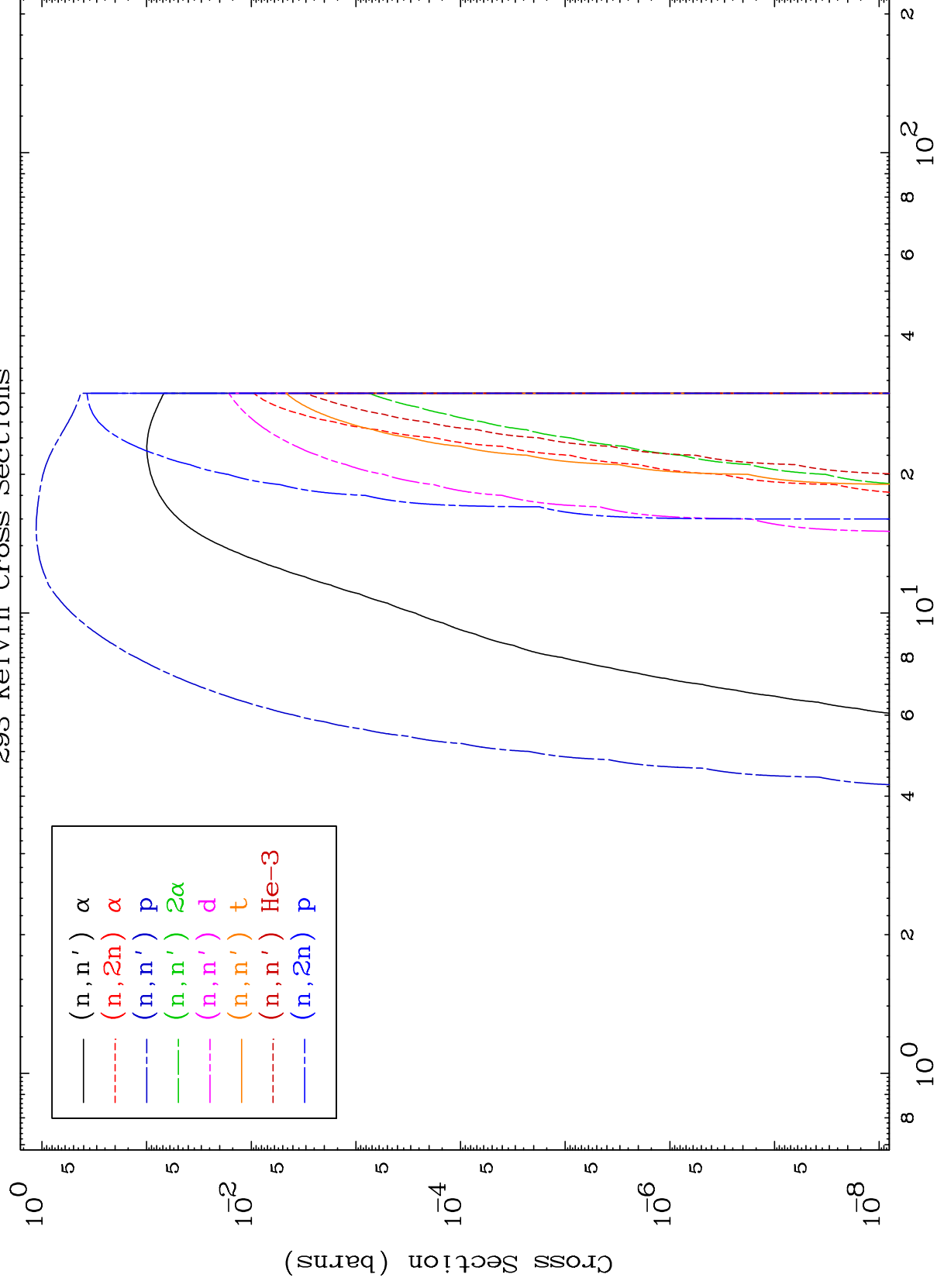
49-In-105



MAT 4901

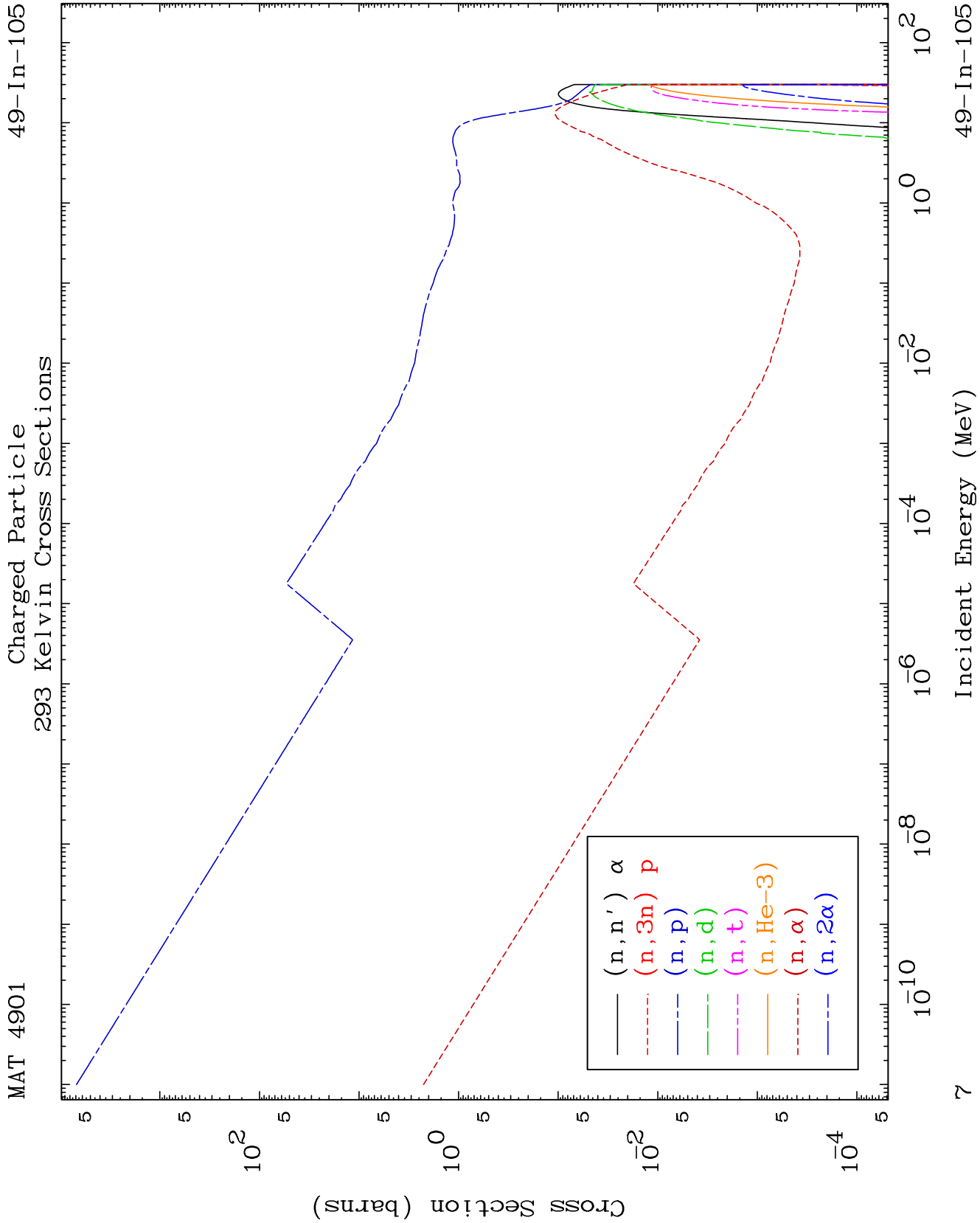
49-In-105

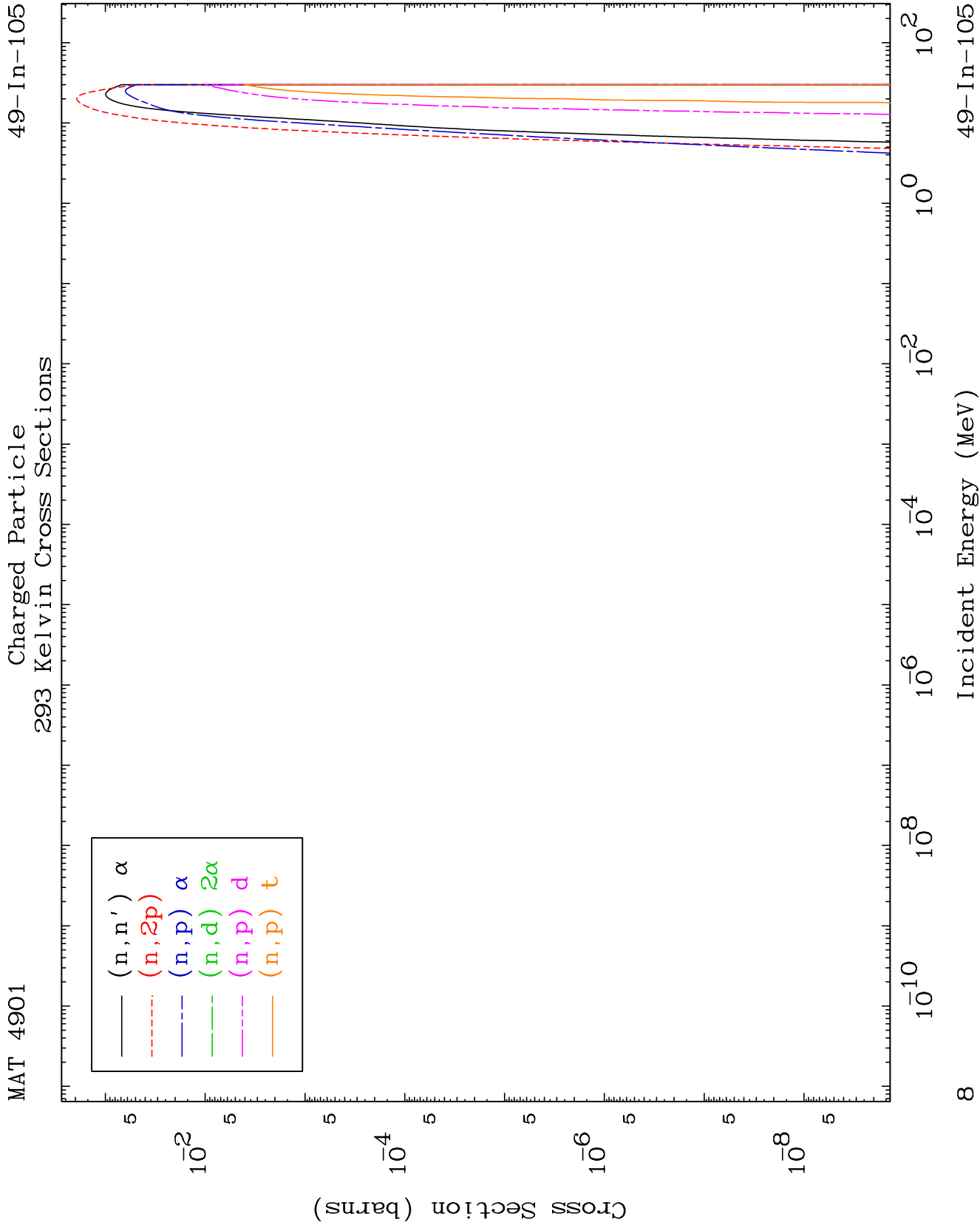
Charged Particle 293 Kelvin Cross Sections

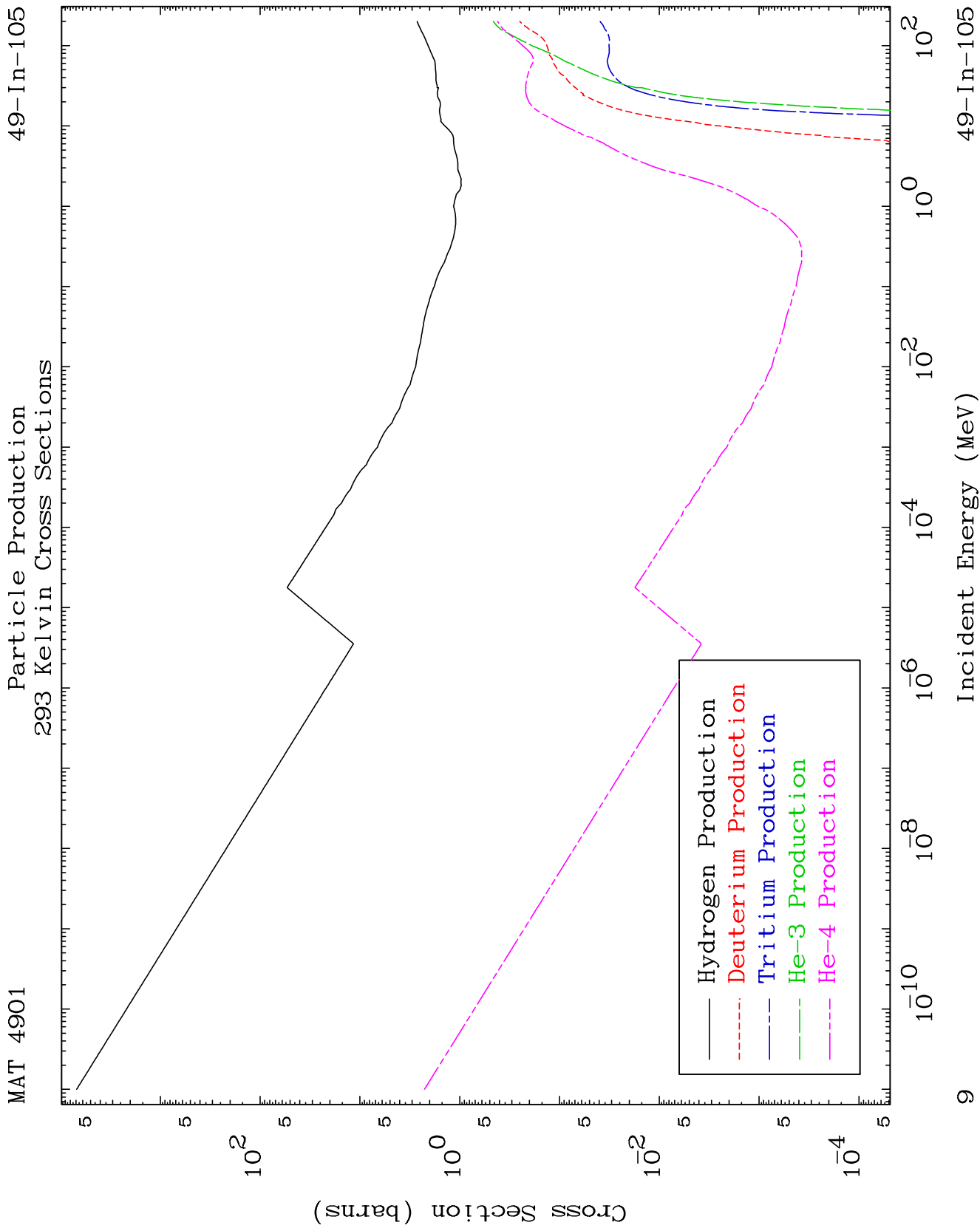


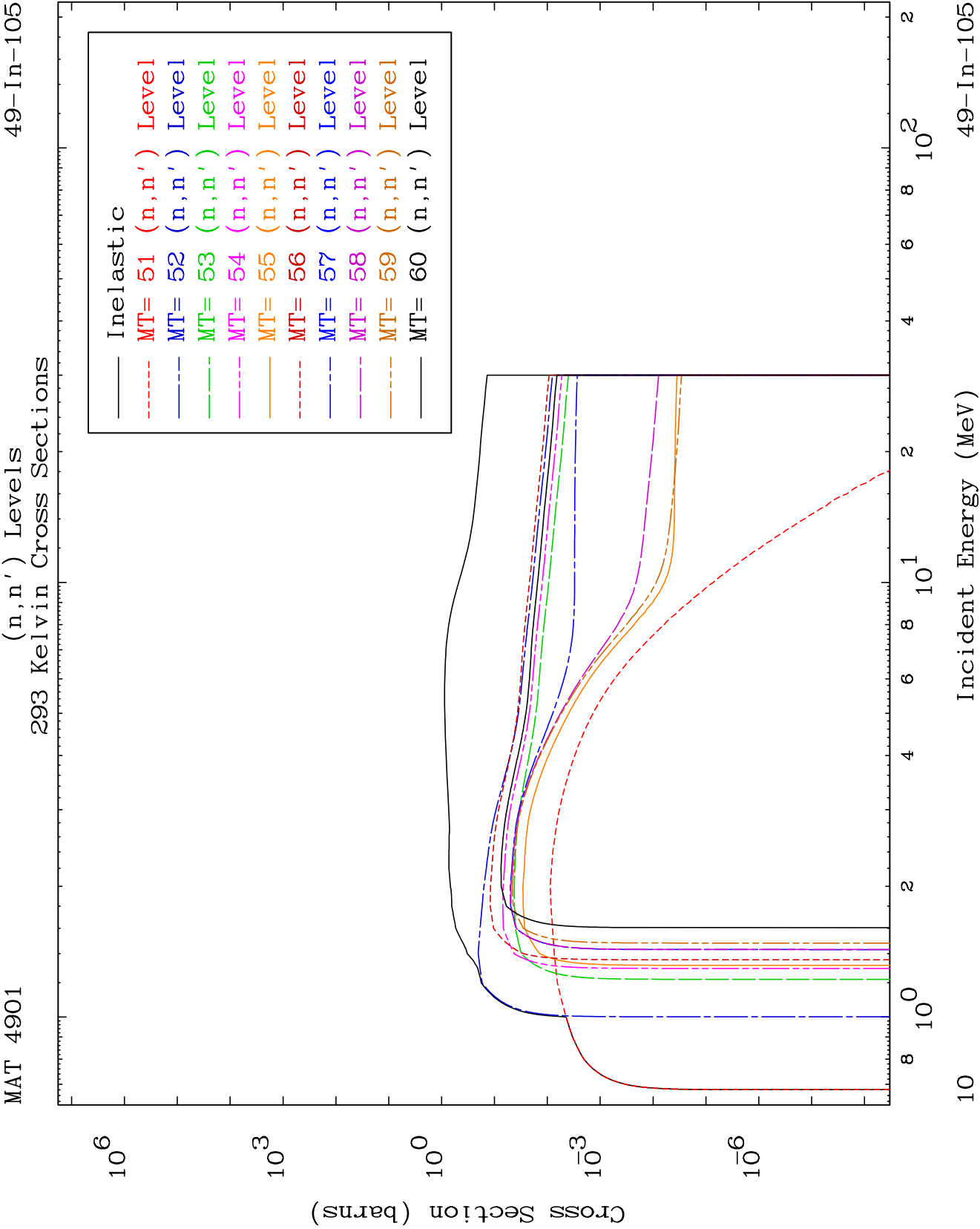
49-In-105

Incident Energy (MeV)







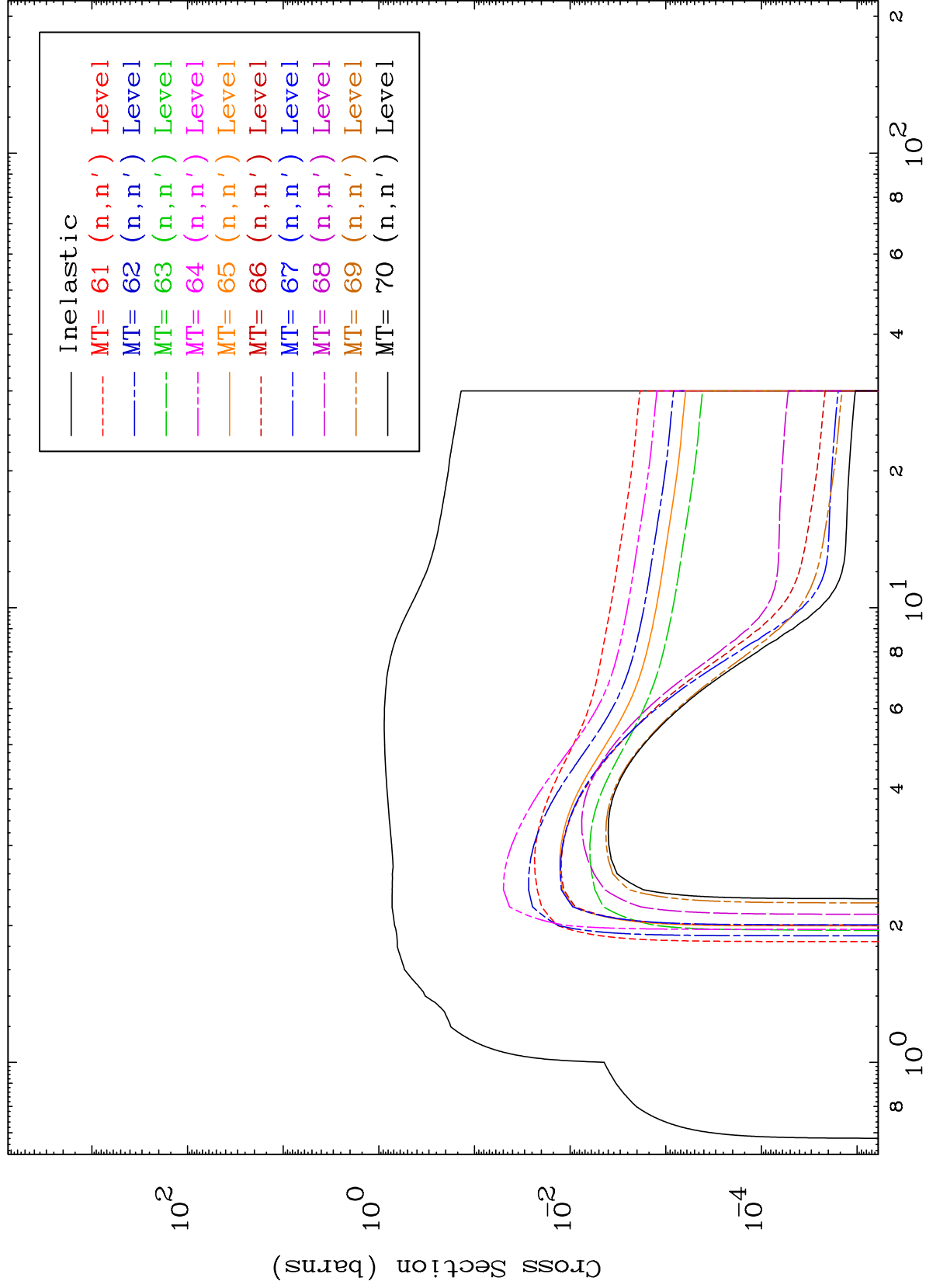


MAT 4901

(n,n') Levels

49-In-105

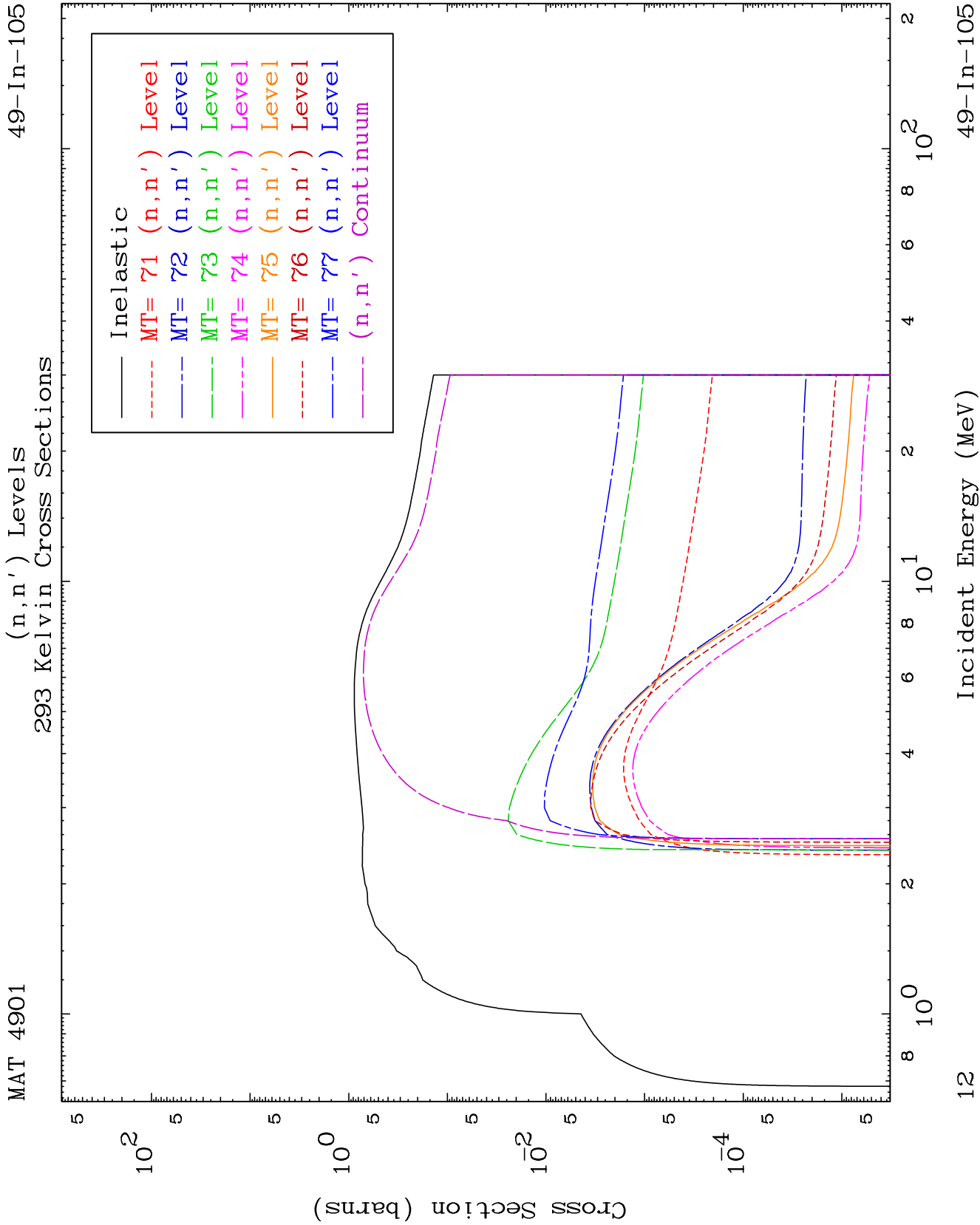
293 Kelvin Cross Sections

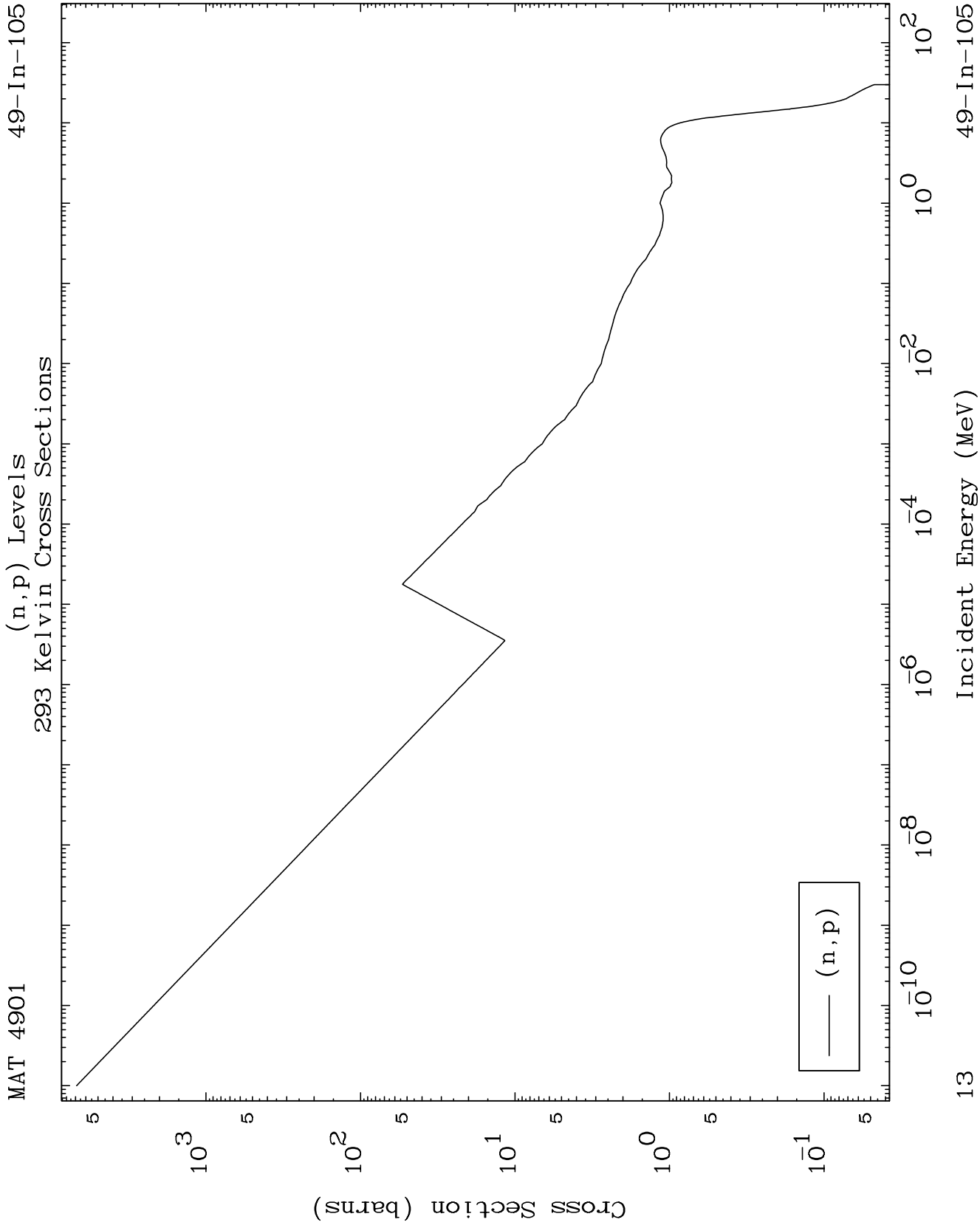


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Incident Energy (MeV)

49-In-105

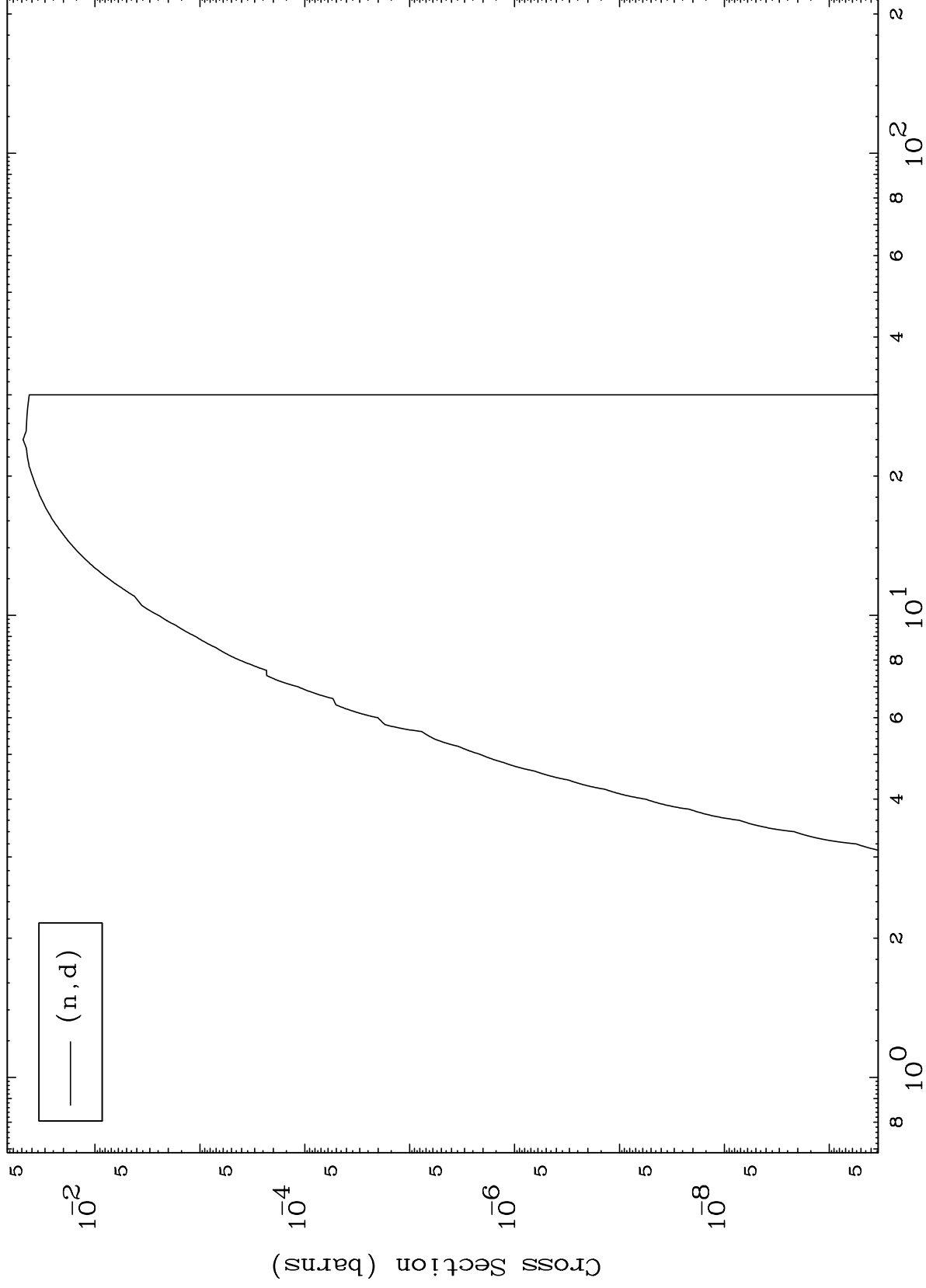




MAT 4901

49-In-105

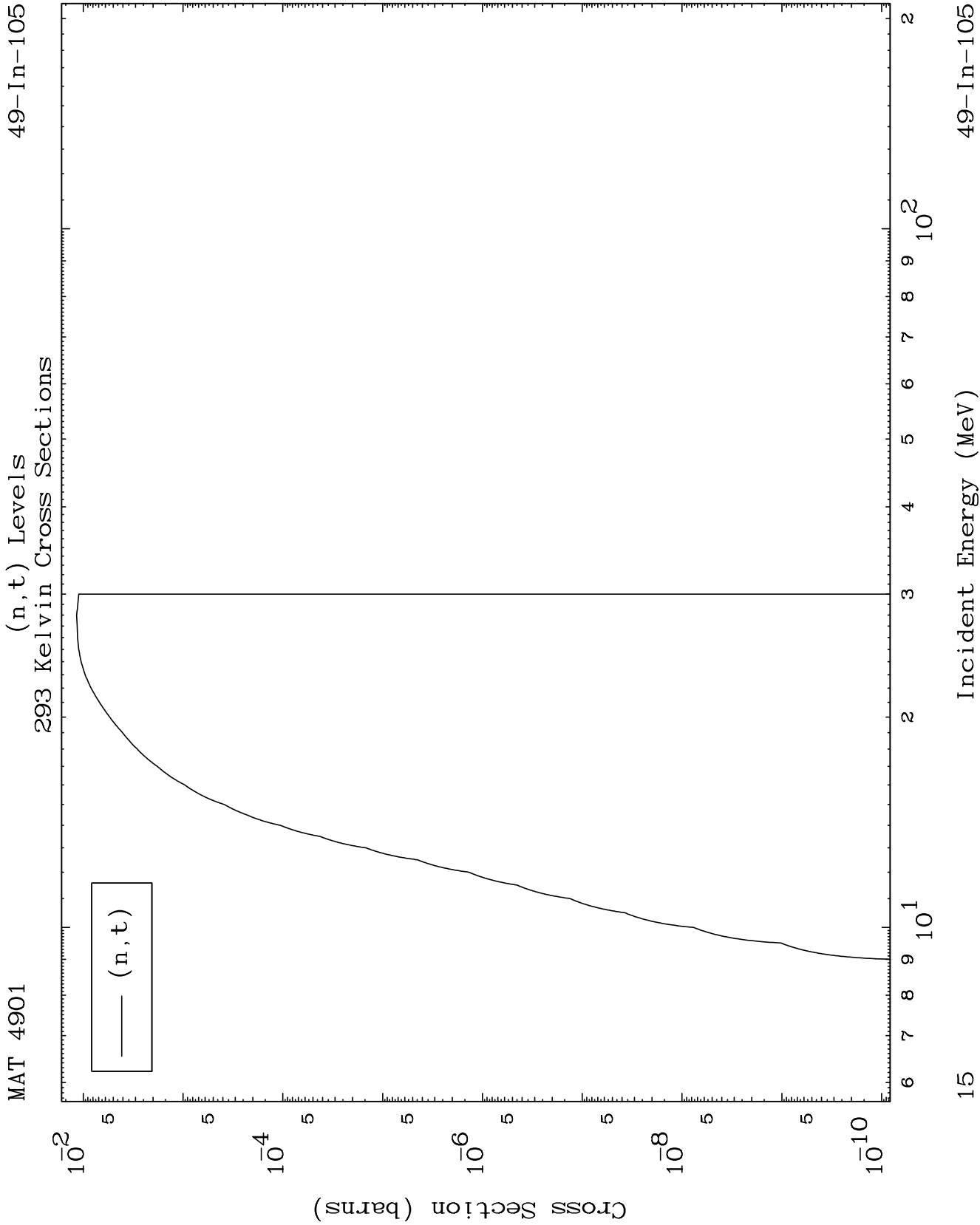
(n,d) Levels
293 Kelvin Cross Sections

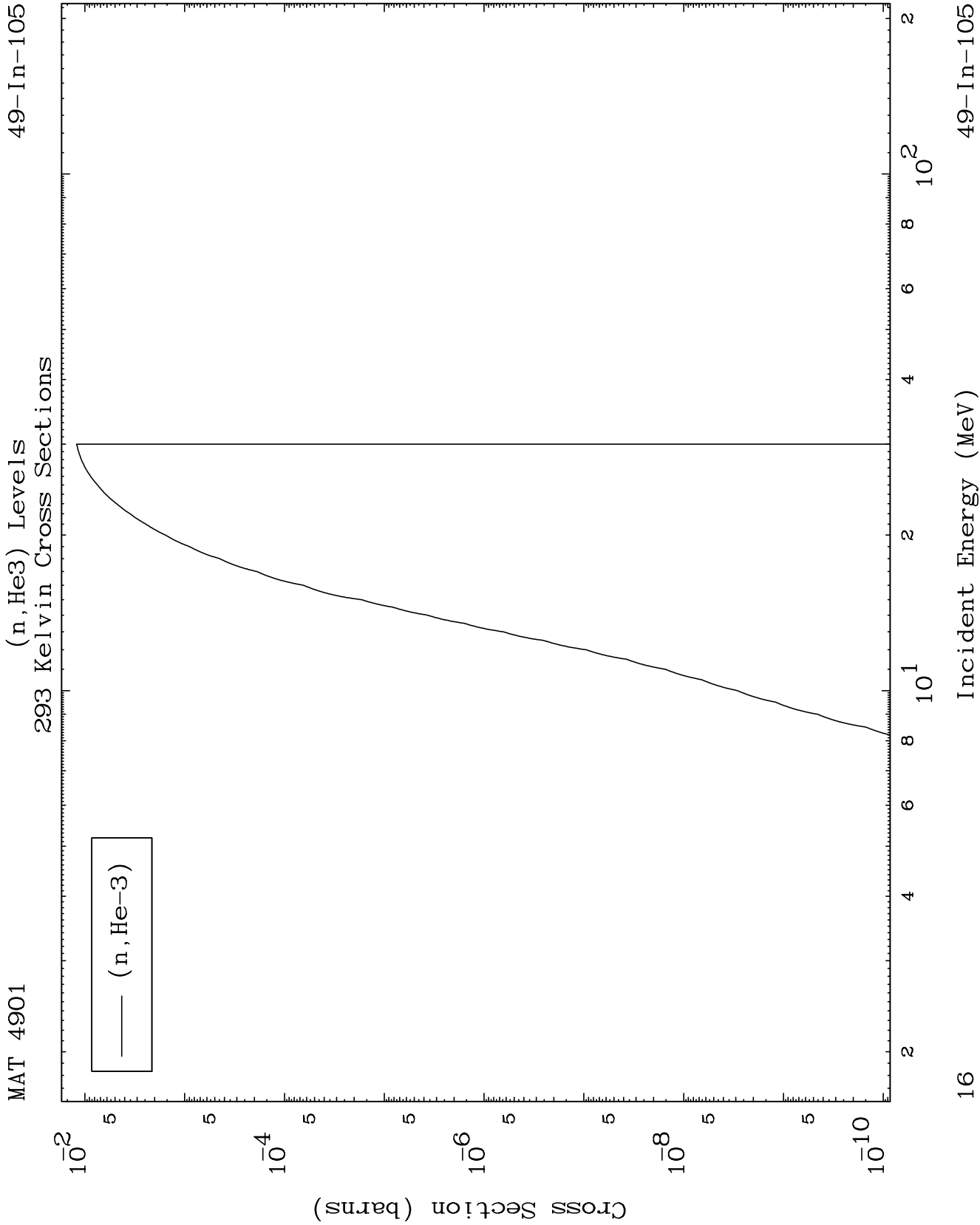


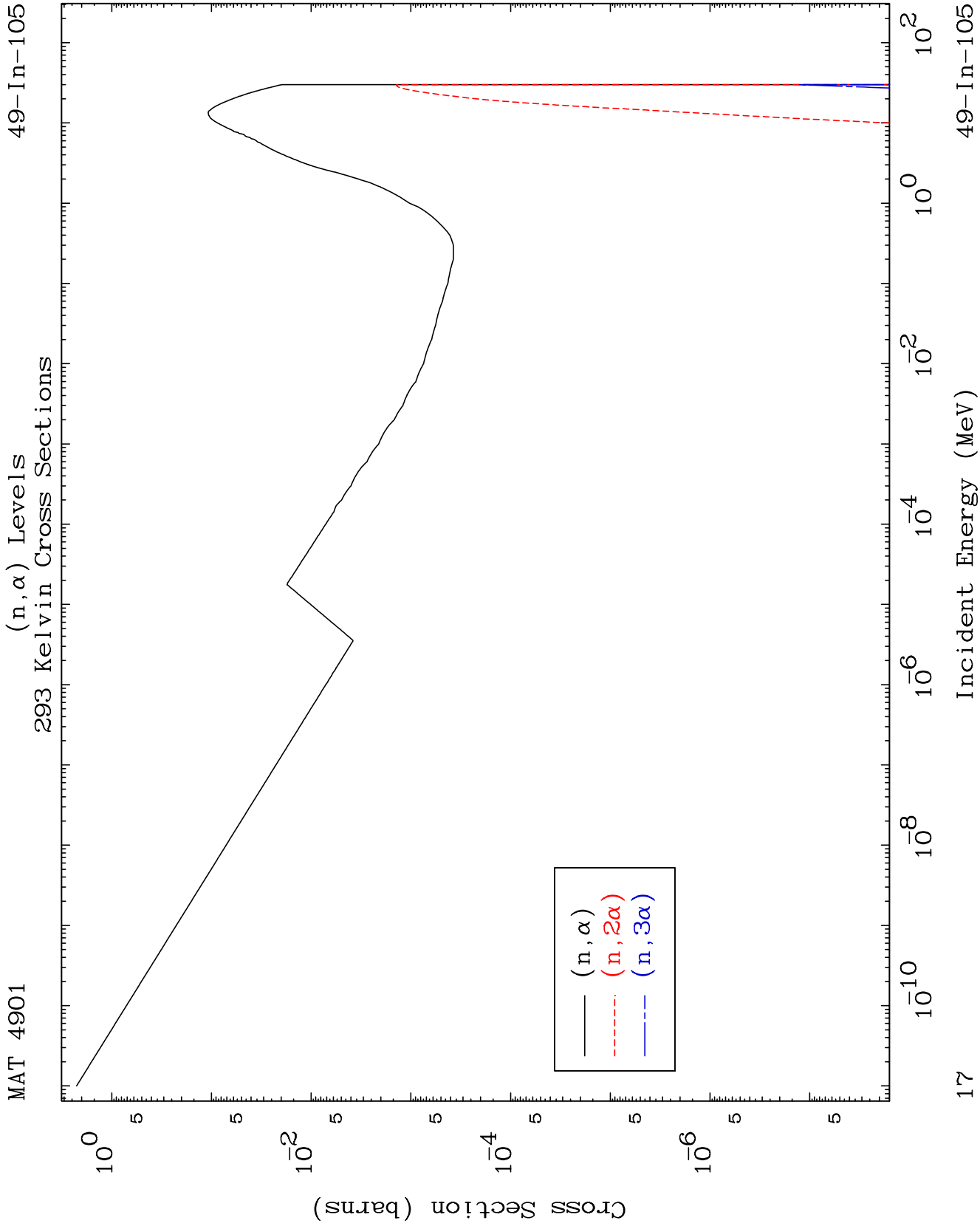
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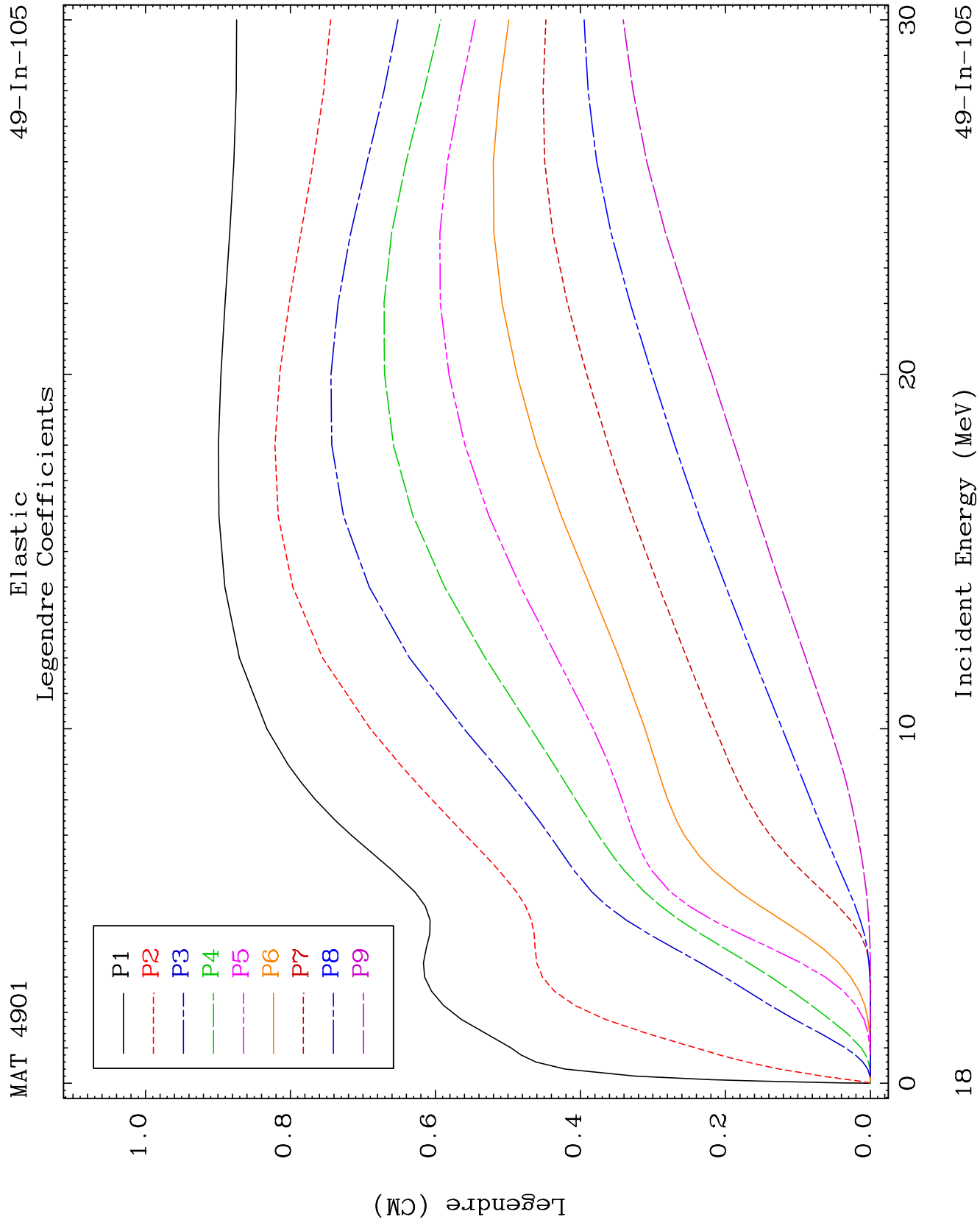
Incident Energy (MeV)

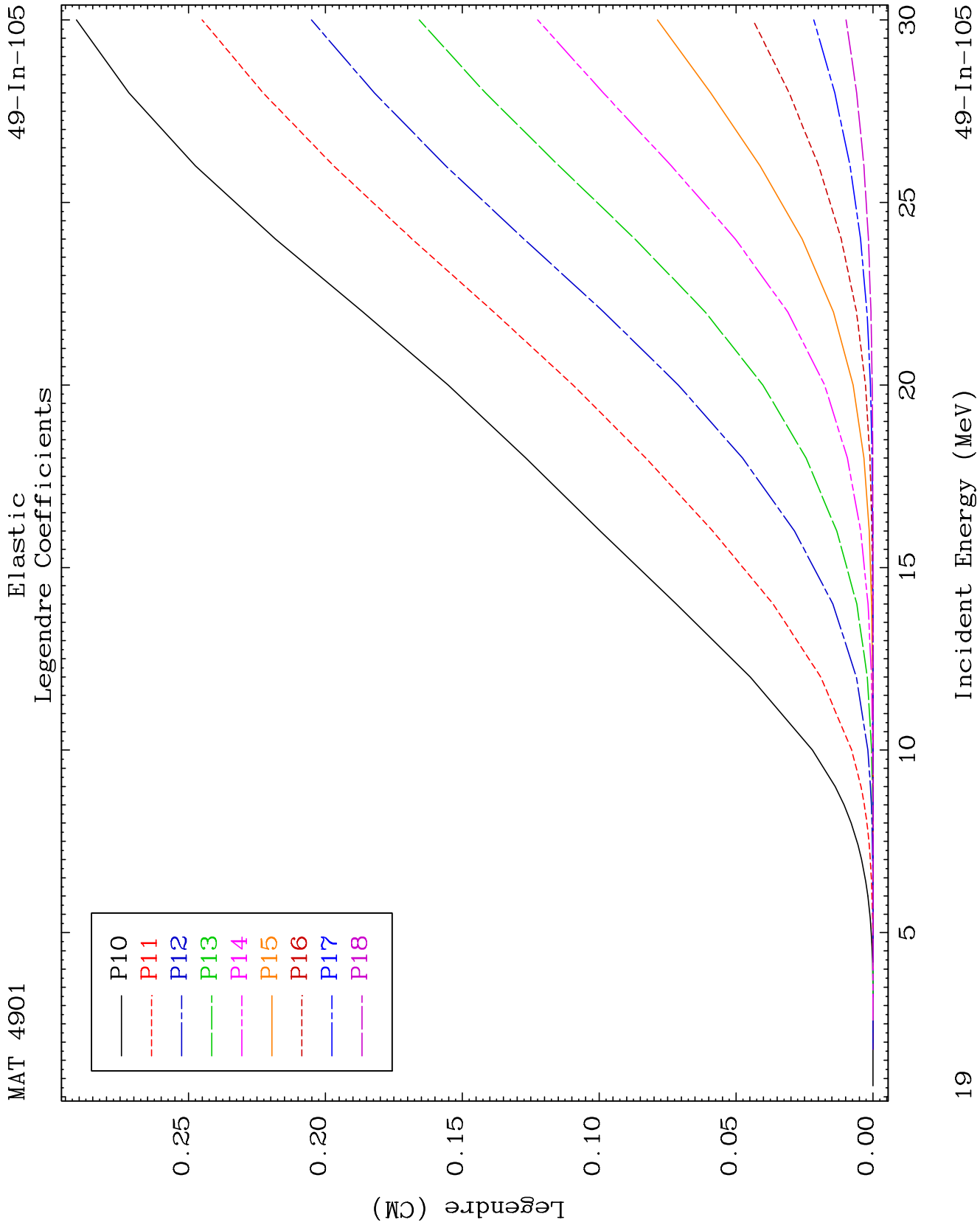
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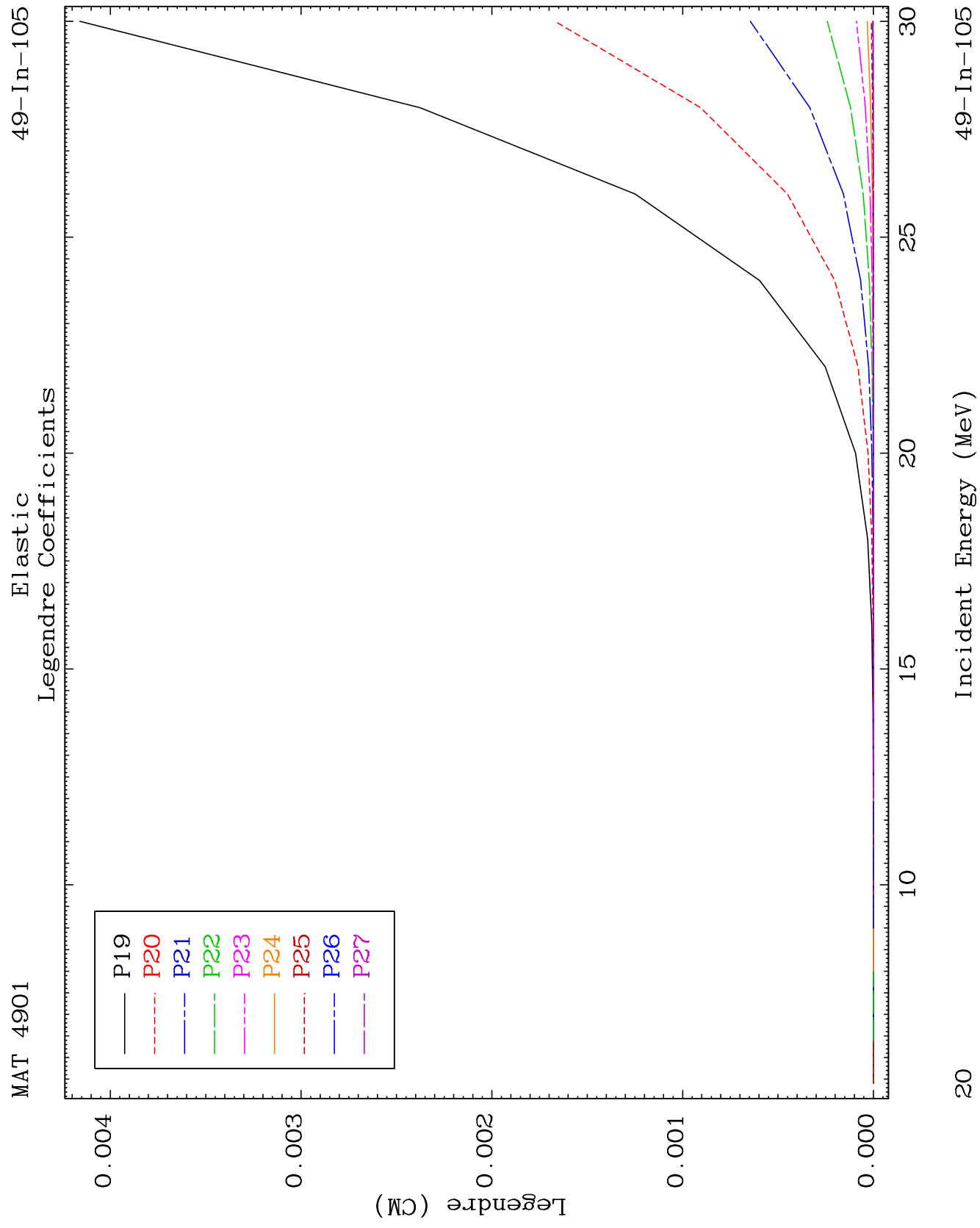


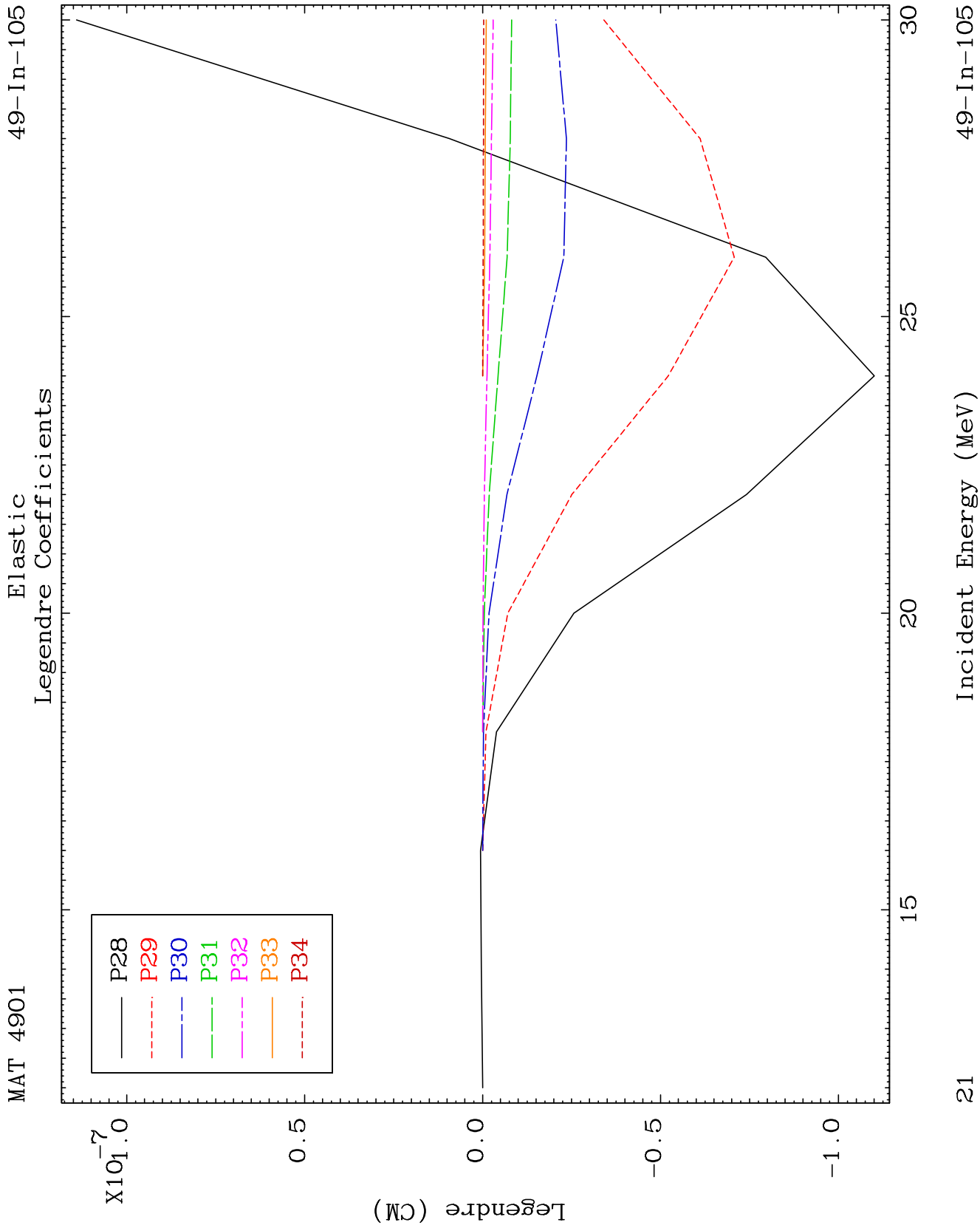


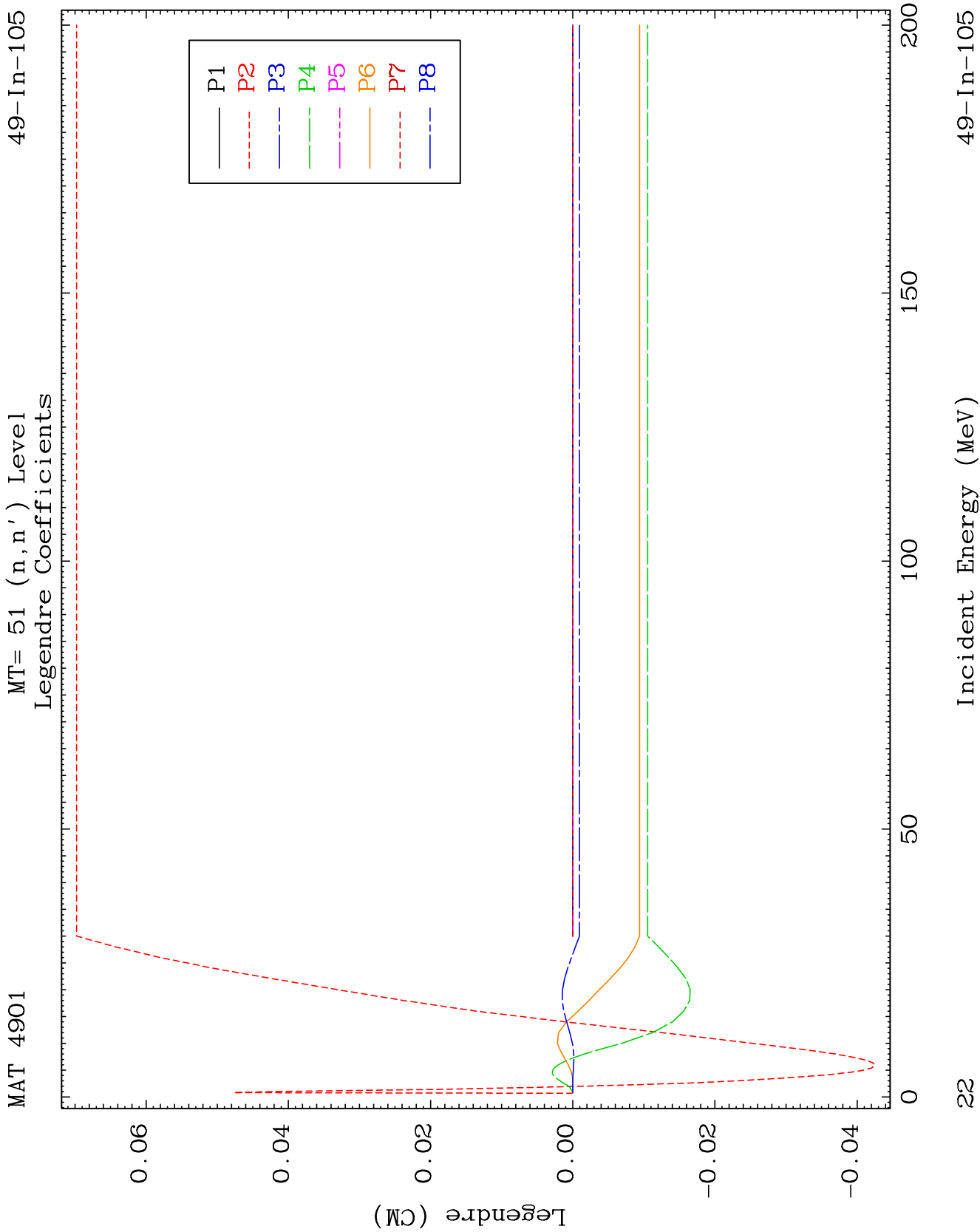


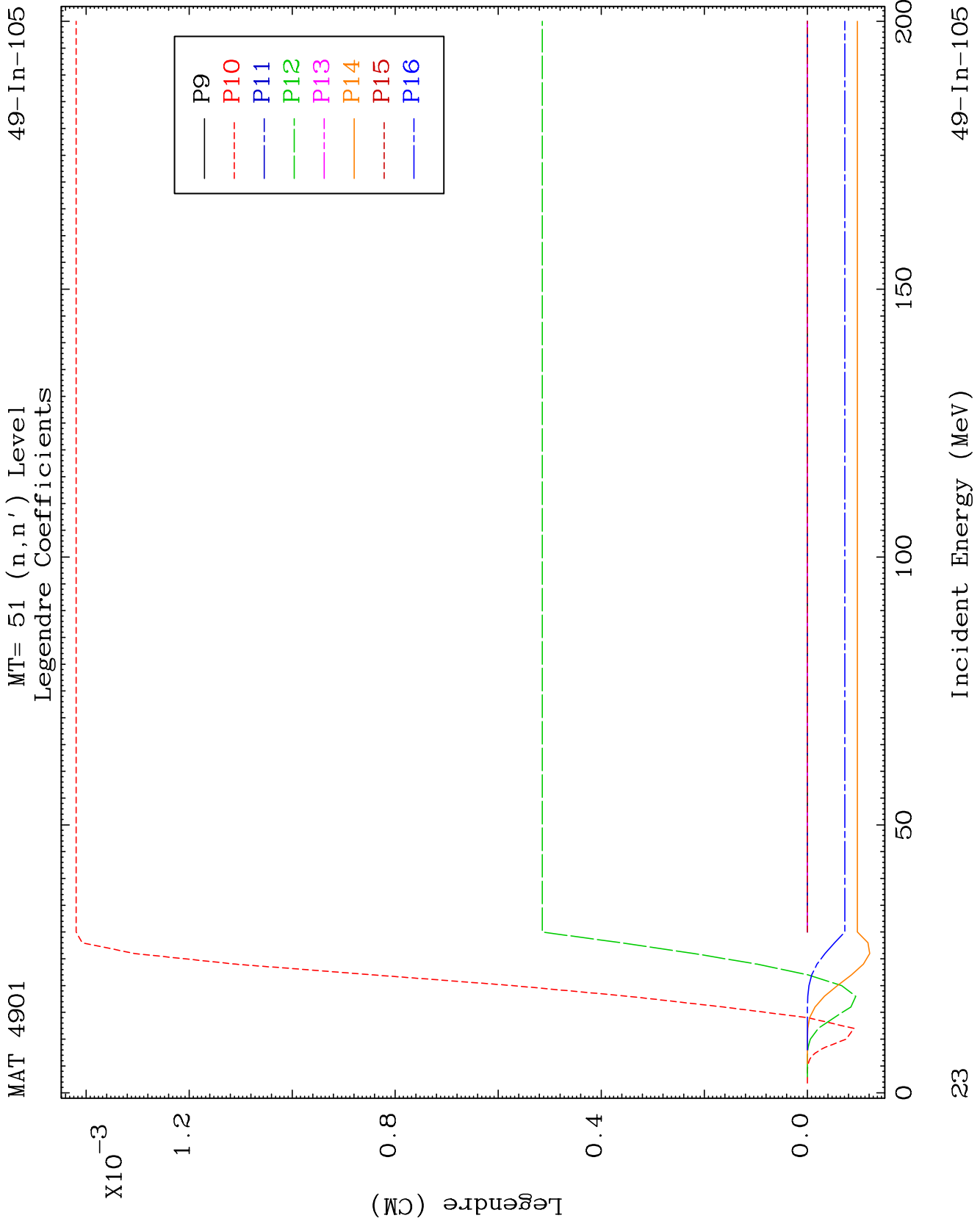


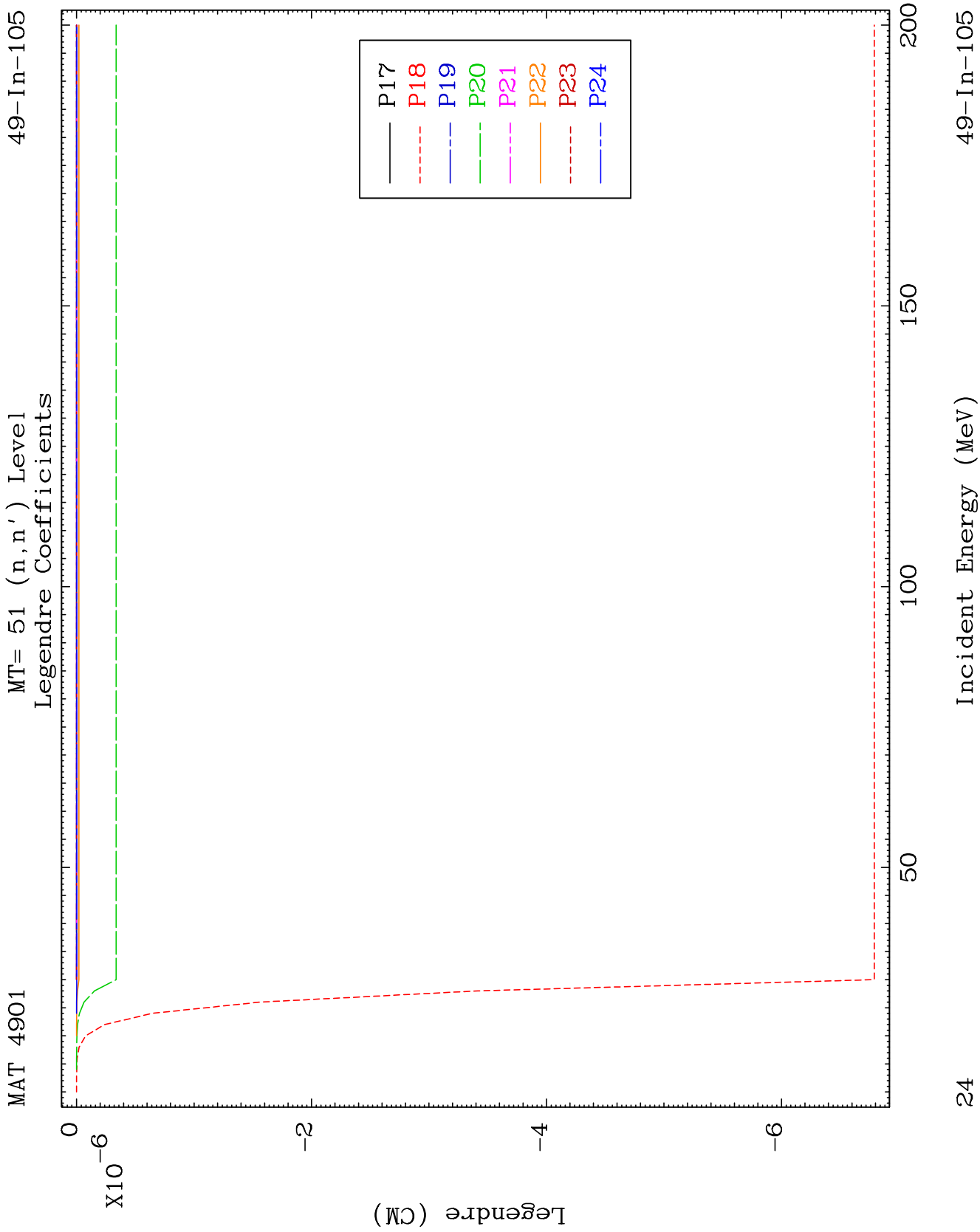


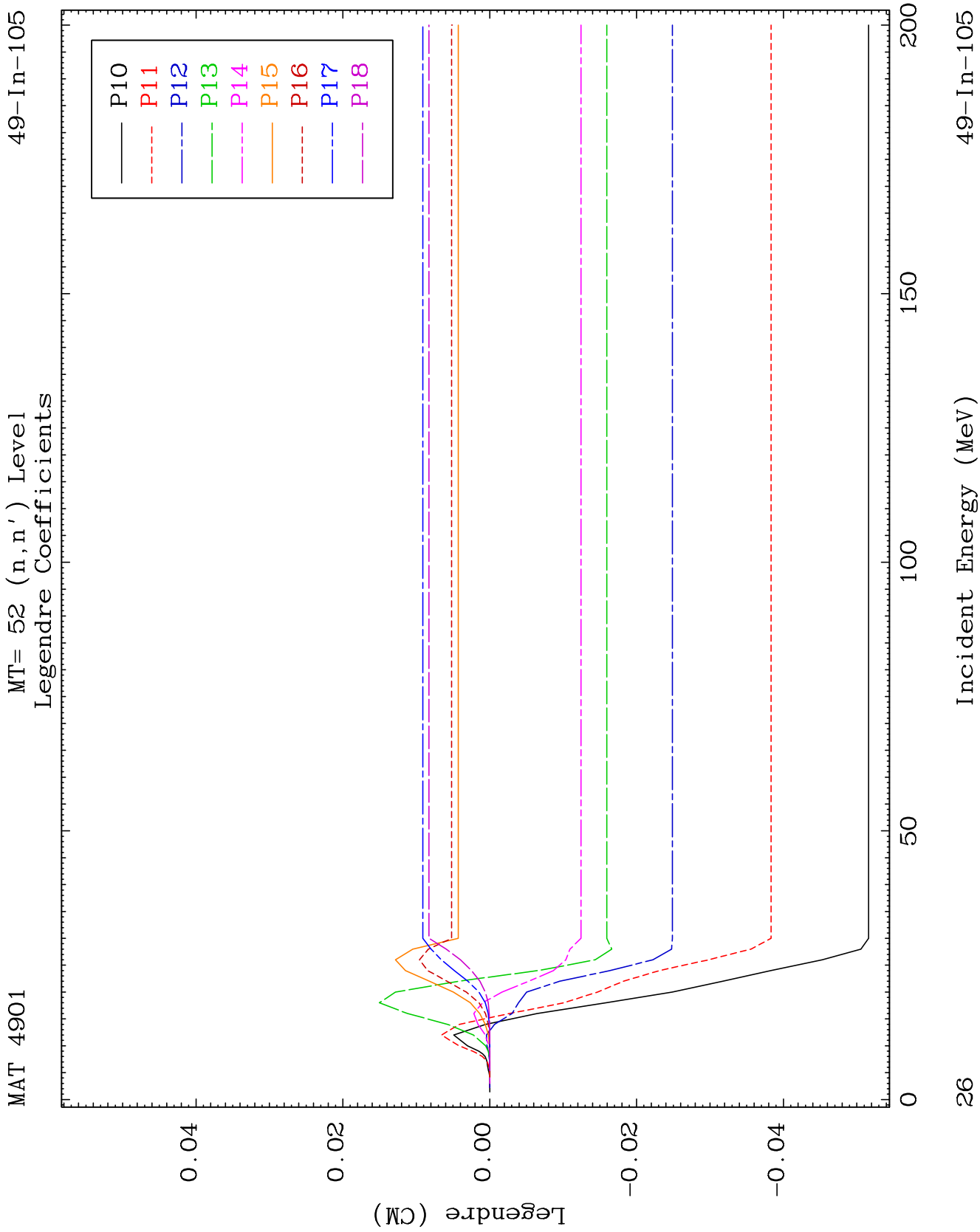


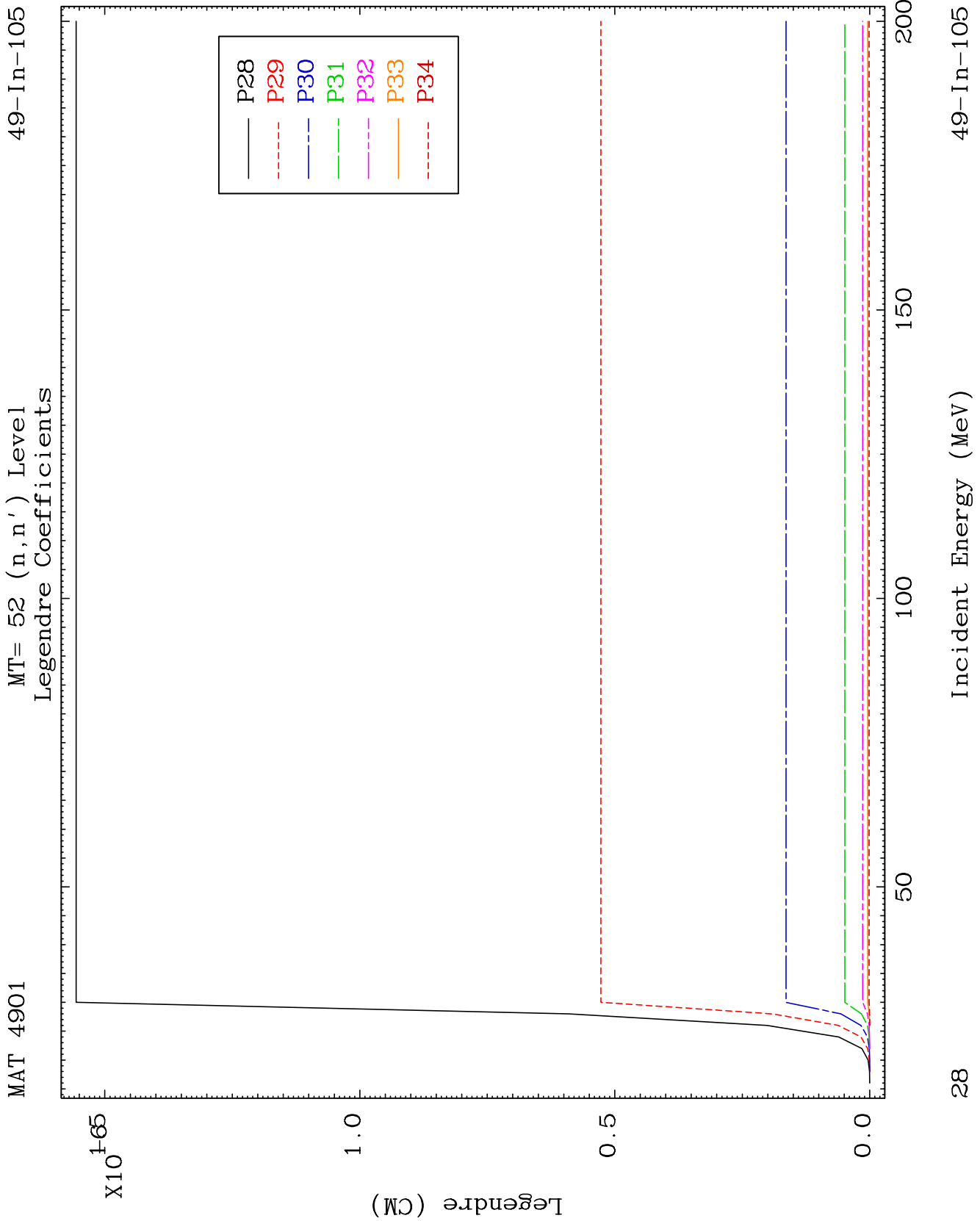


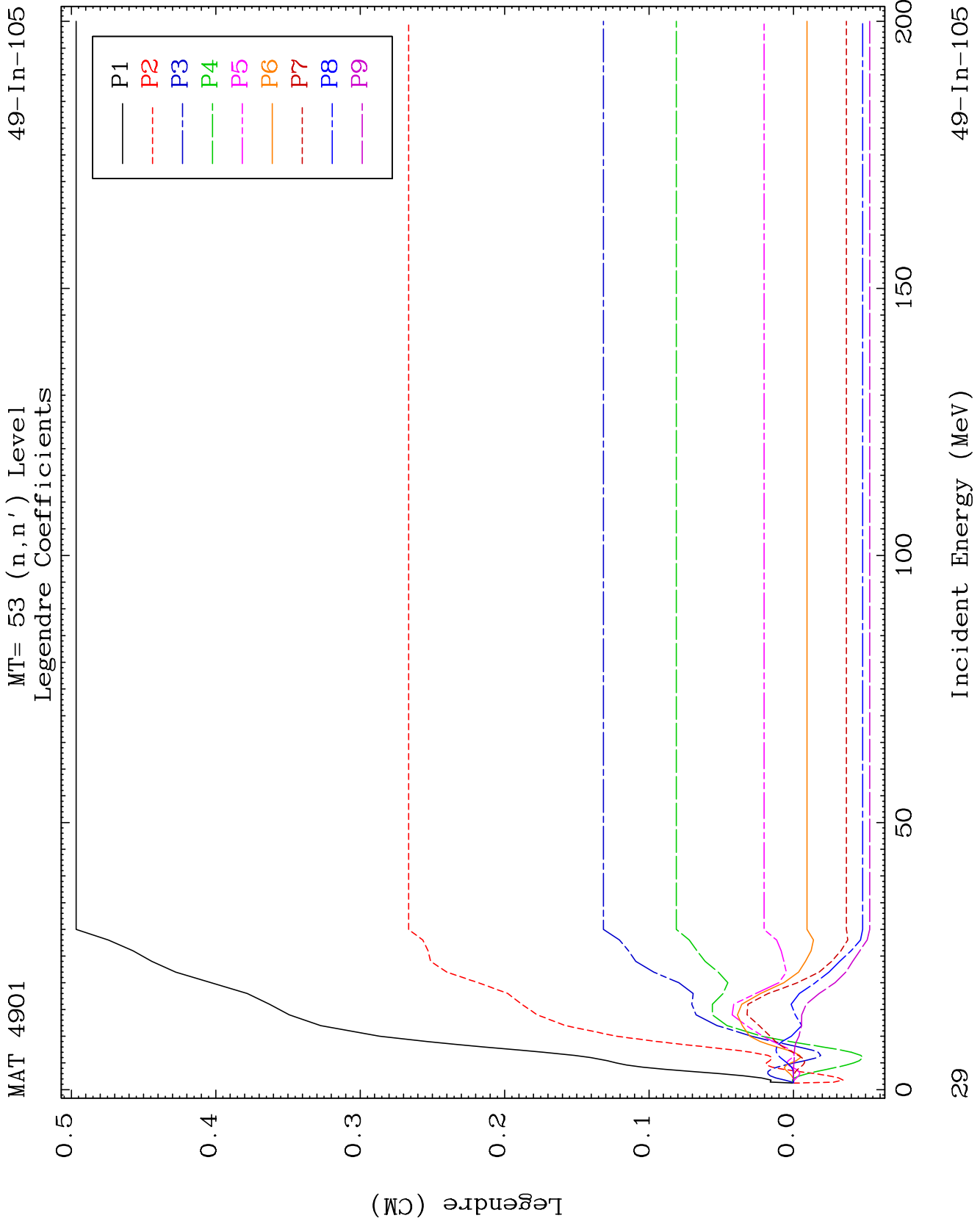








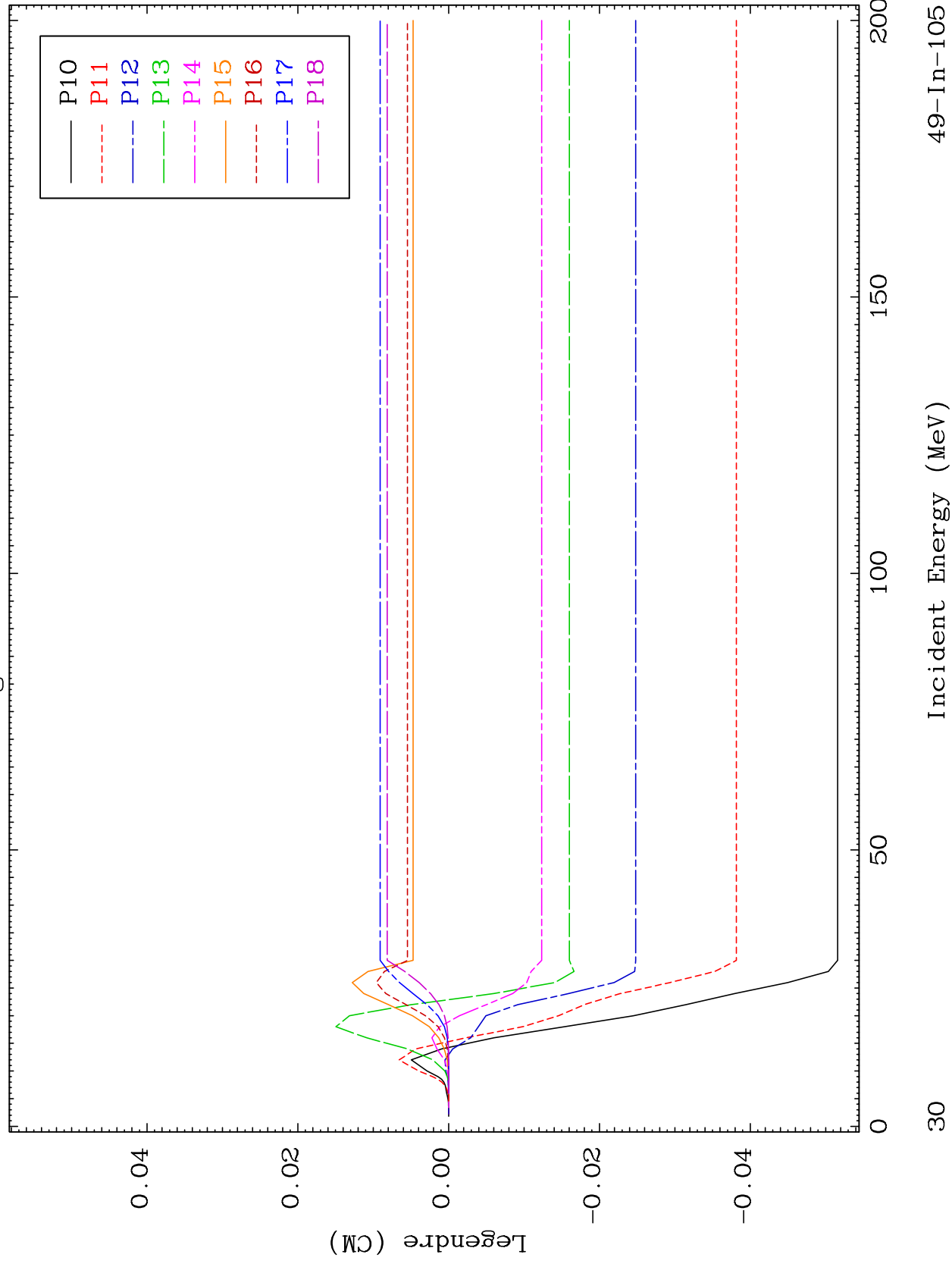


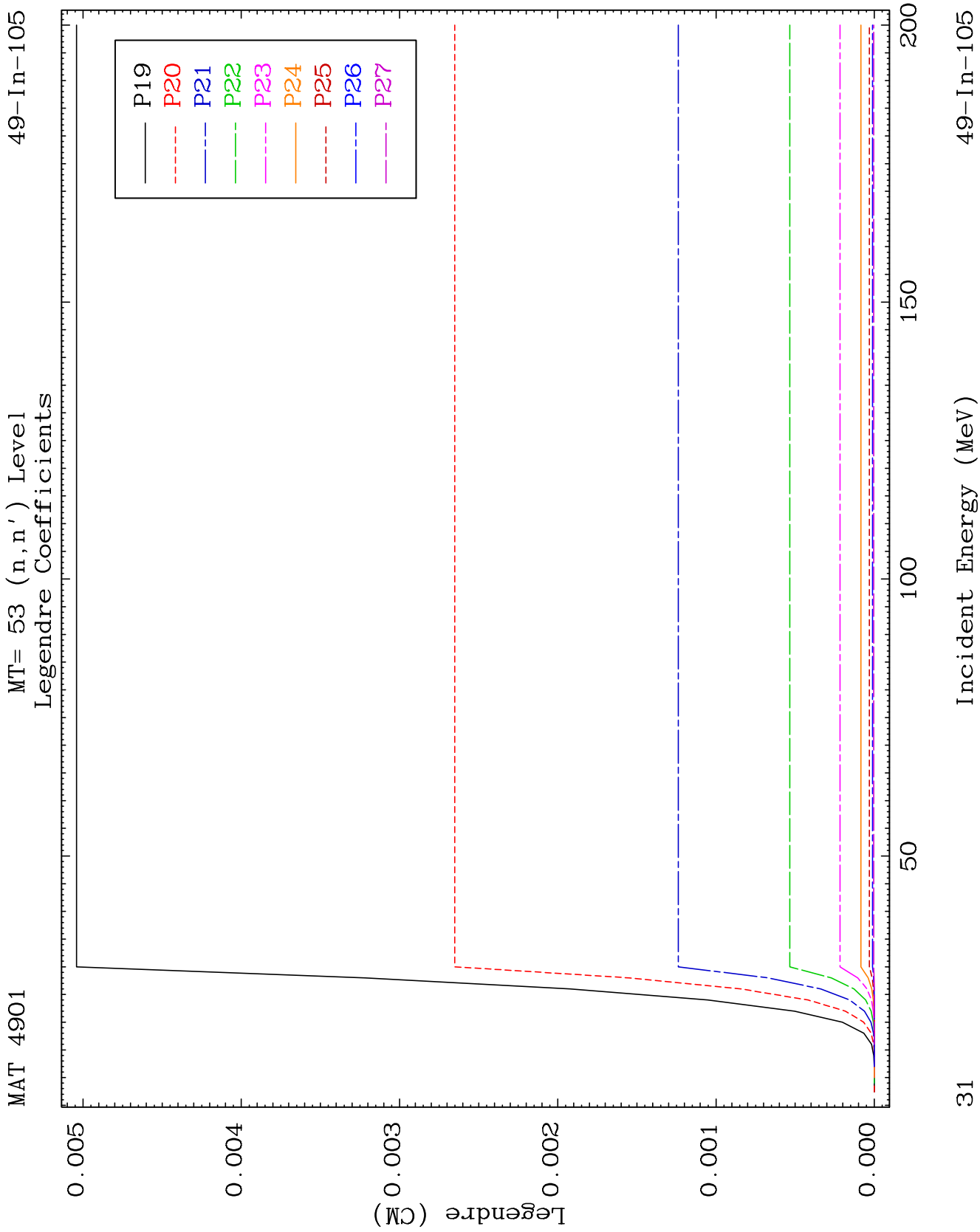


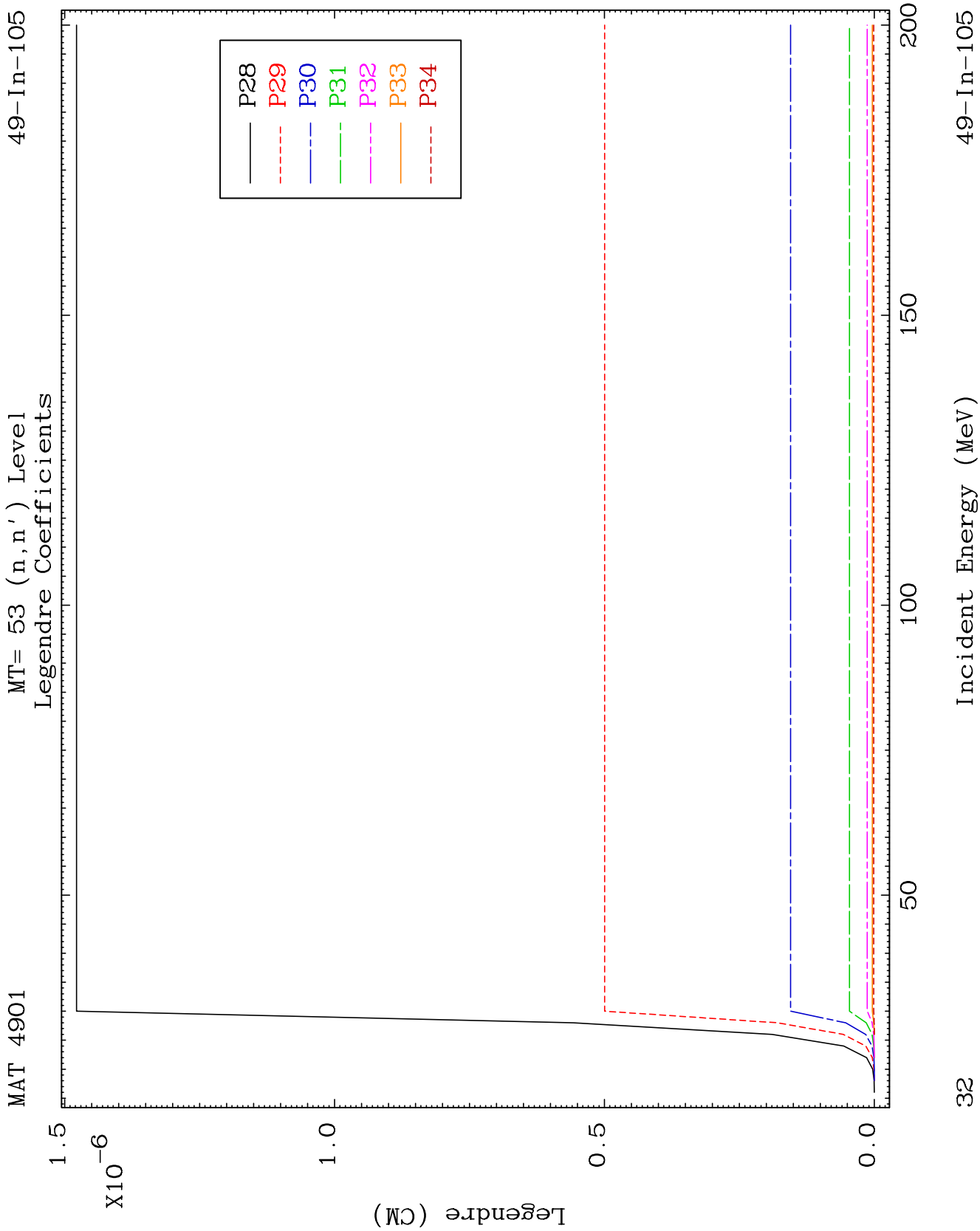
MAT 4901

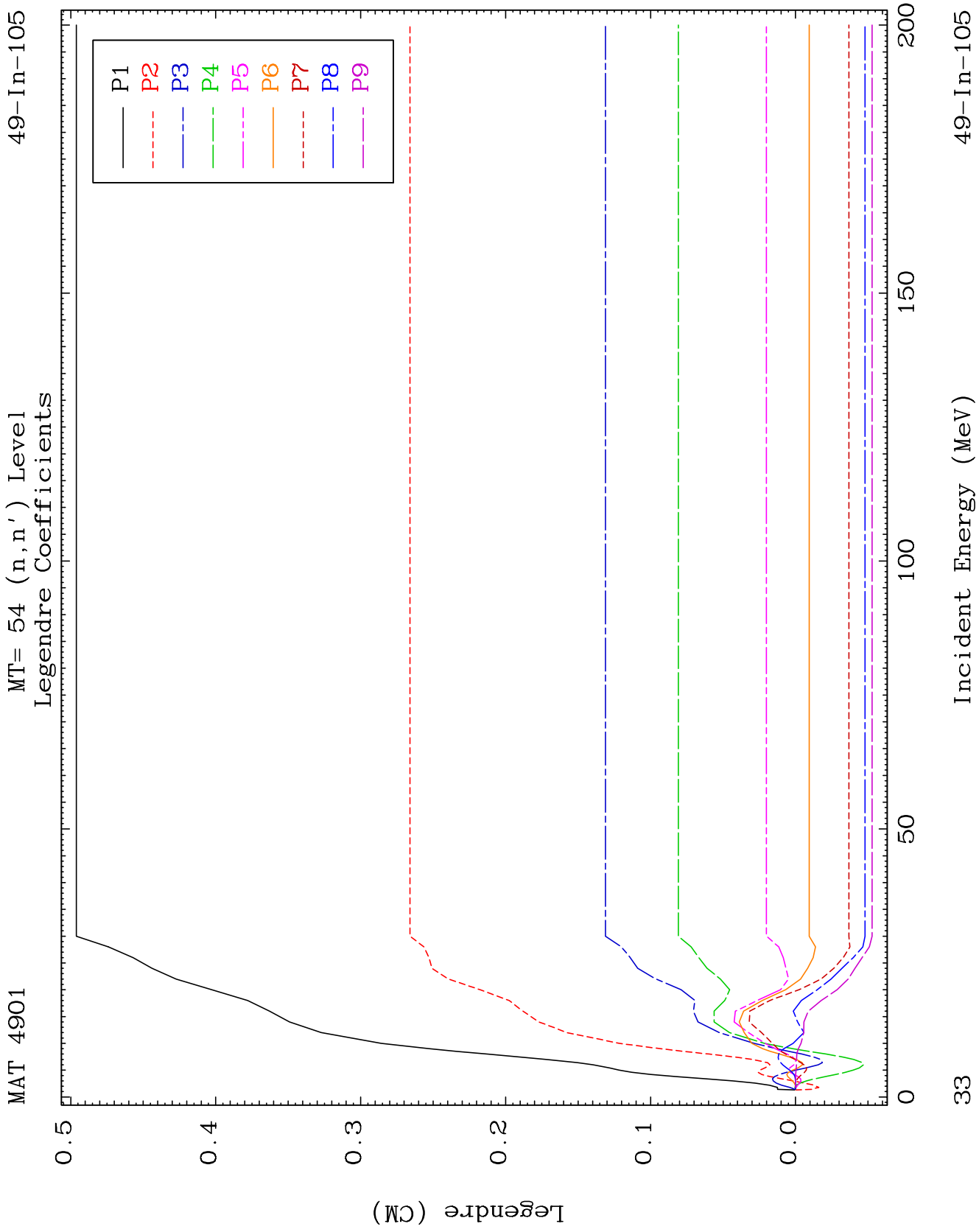
MT= 53 (n, n') Level
Legendre Coefficients

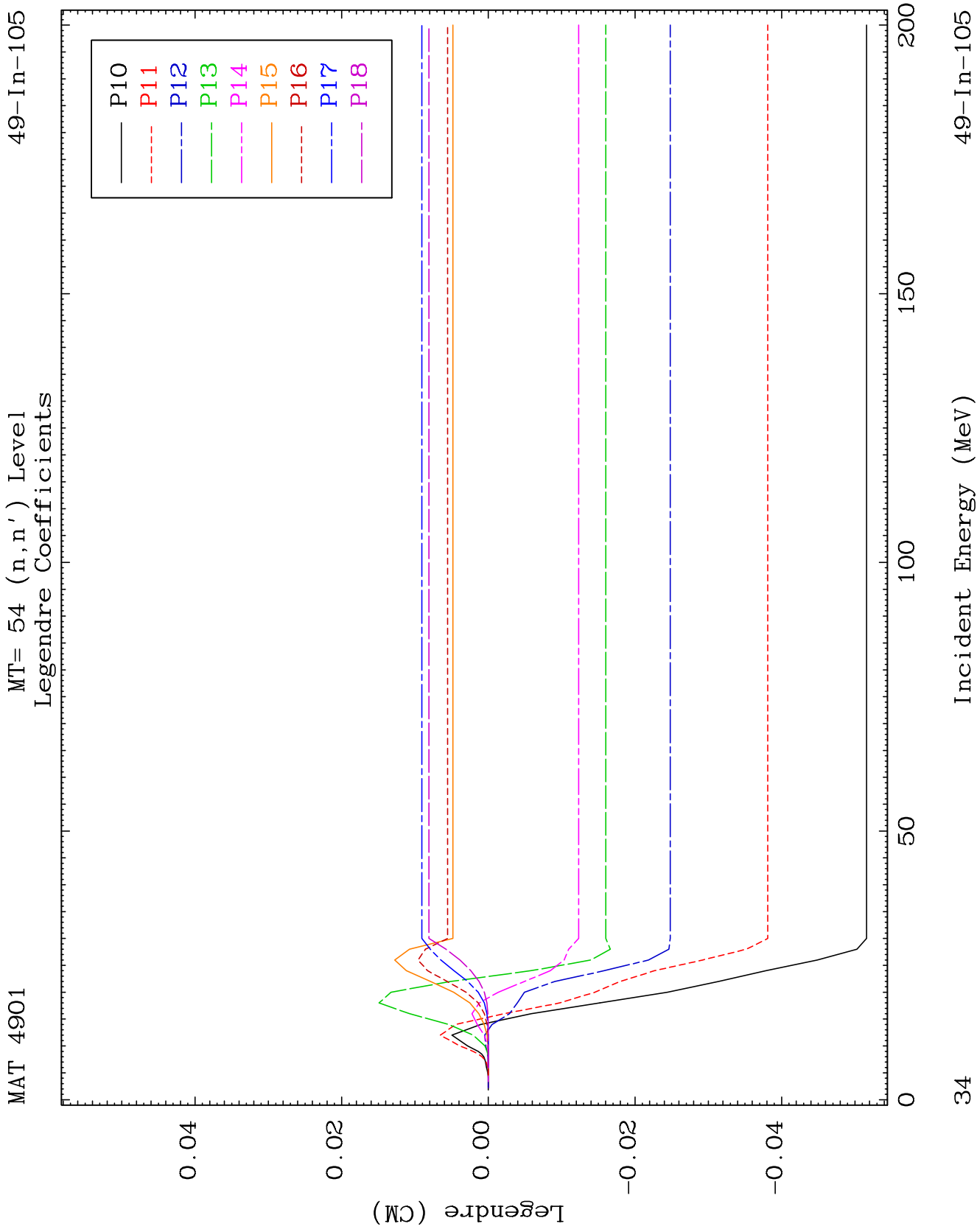
49-In-105

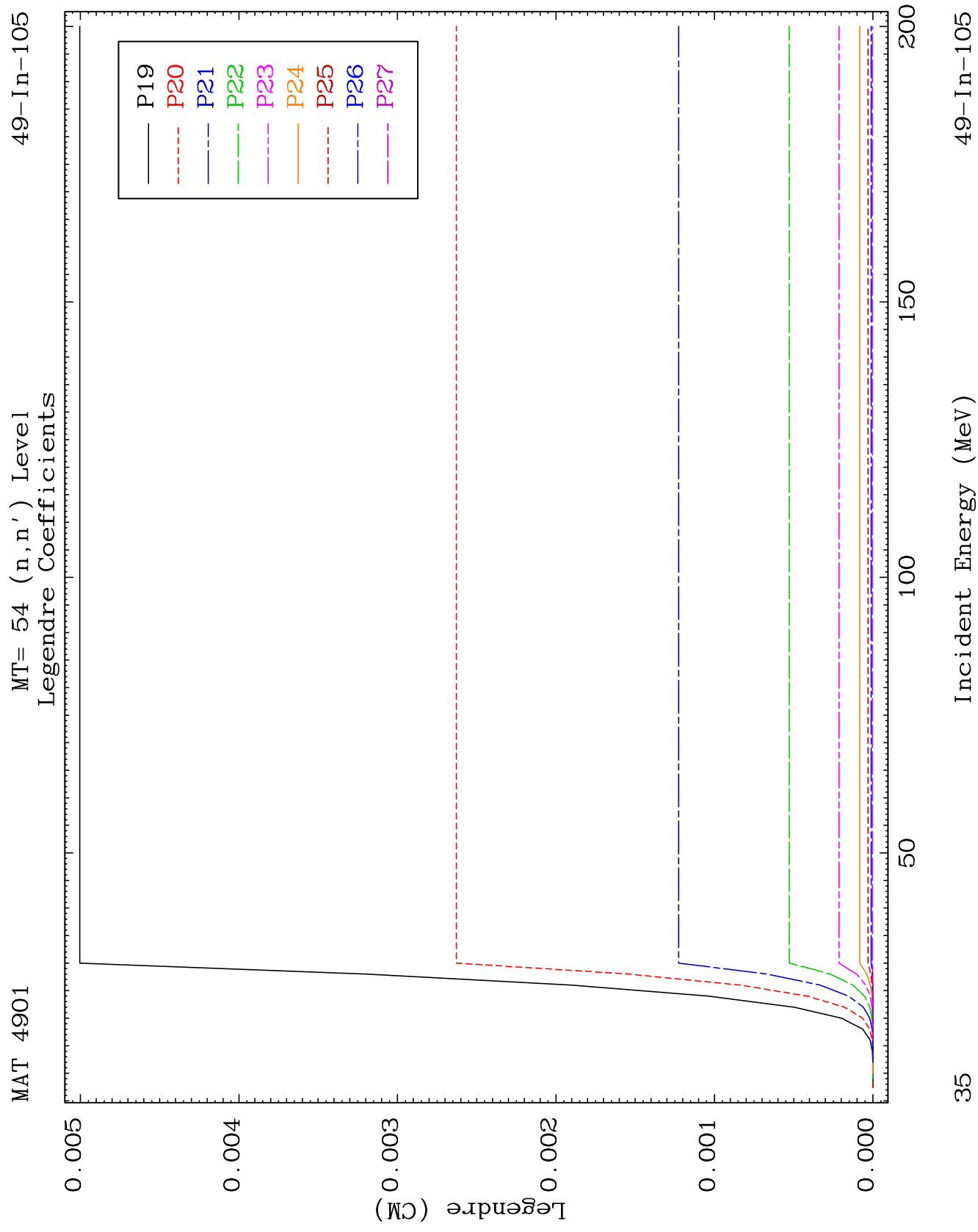


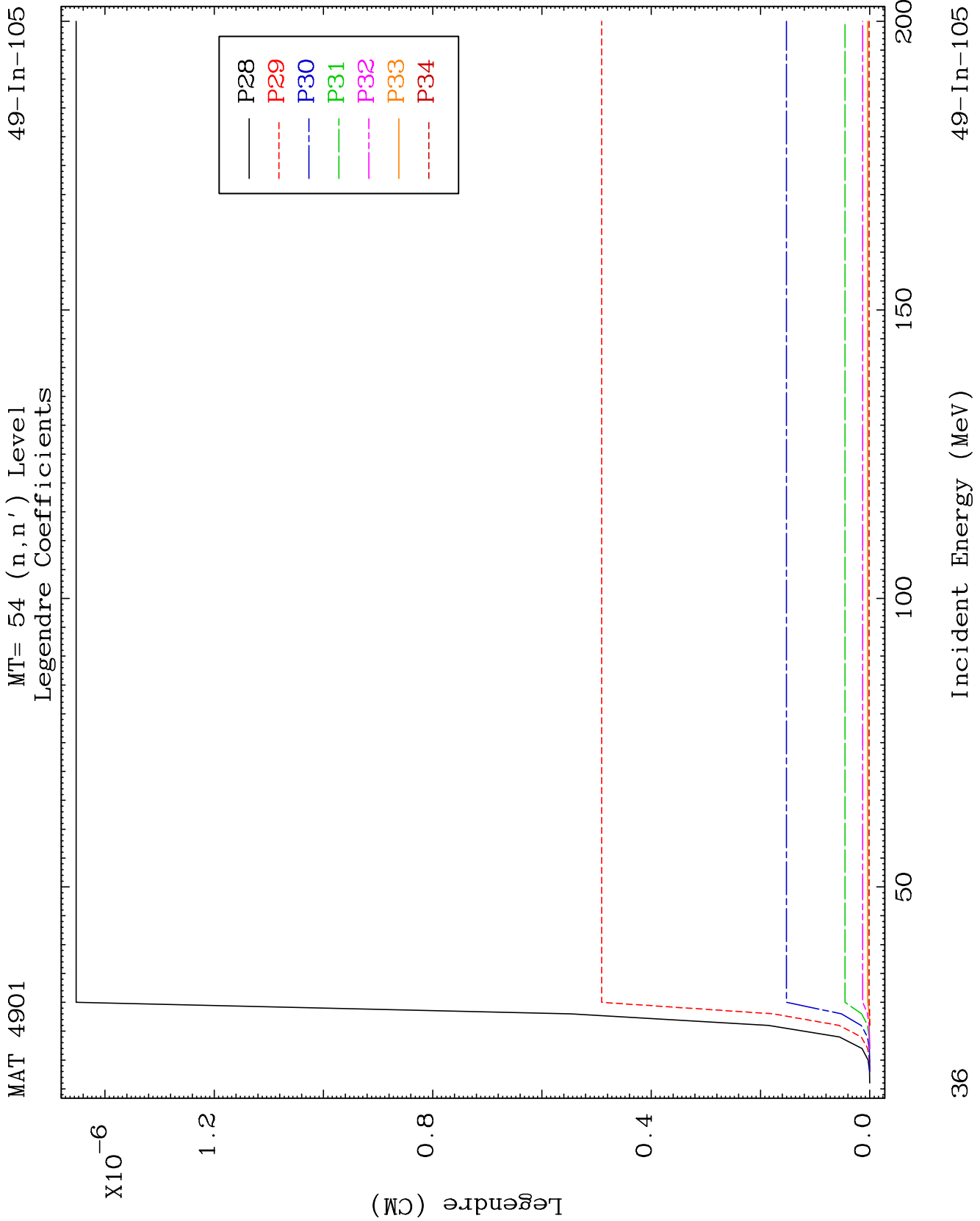








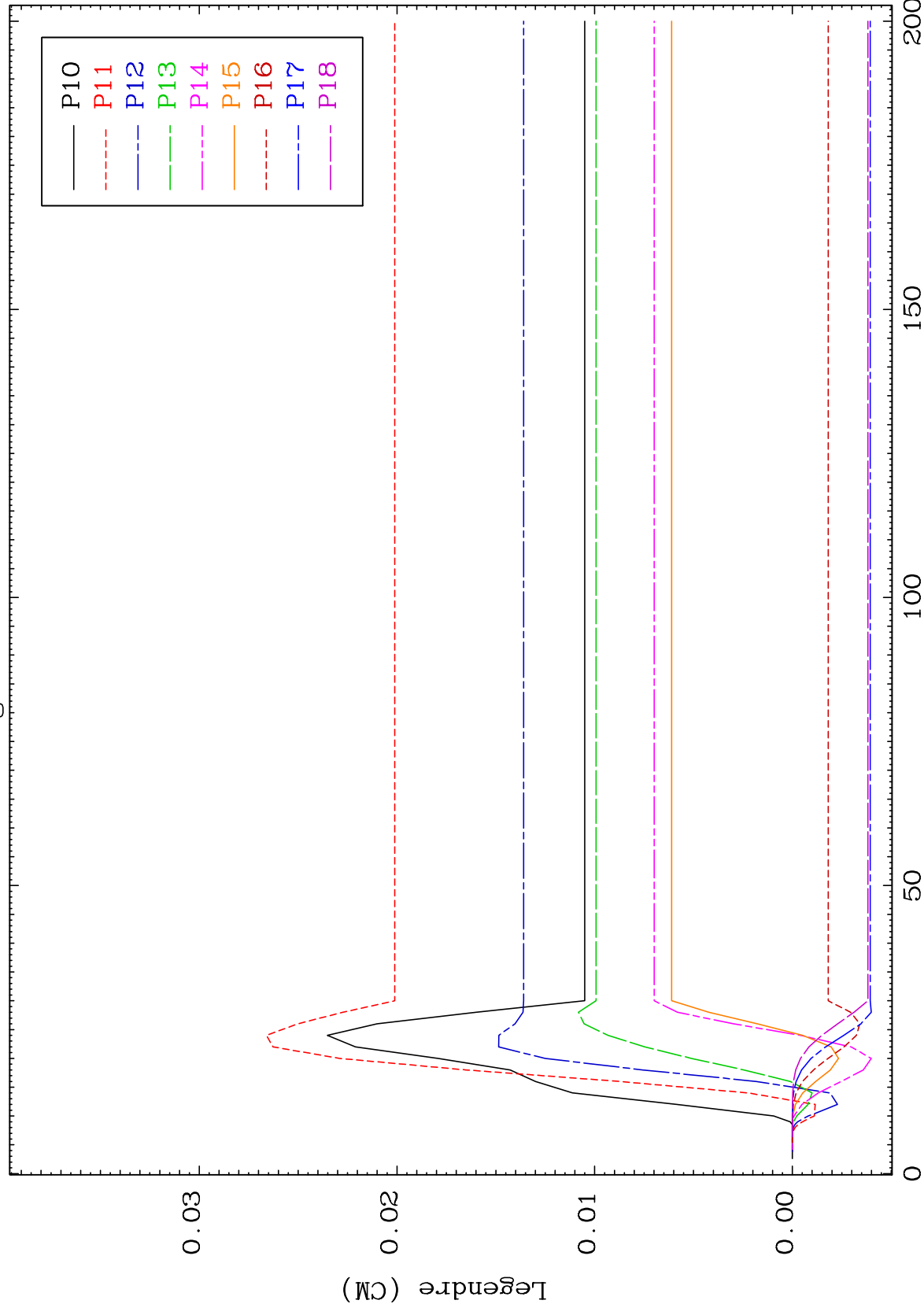




MAT 4901

MT= 55 (n, n') Level
Legendre Coefficients

49-In-105



33

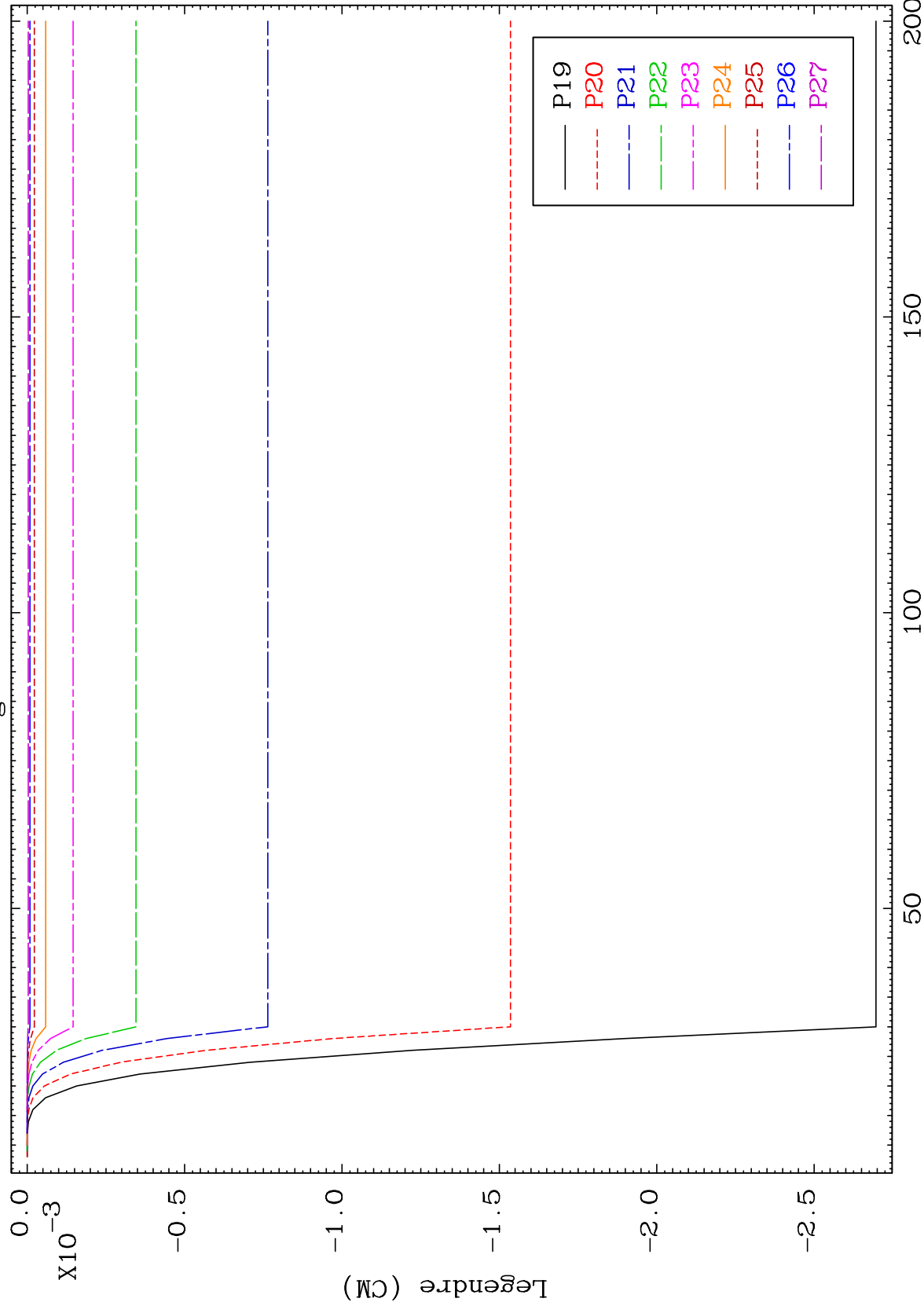
Incident Energy (MeV)

49-In-105

MAT 4901

MT= 55 (n, n') Level
Legendre Coefficients

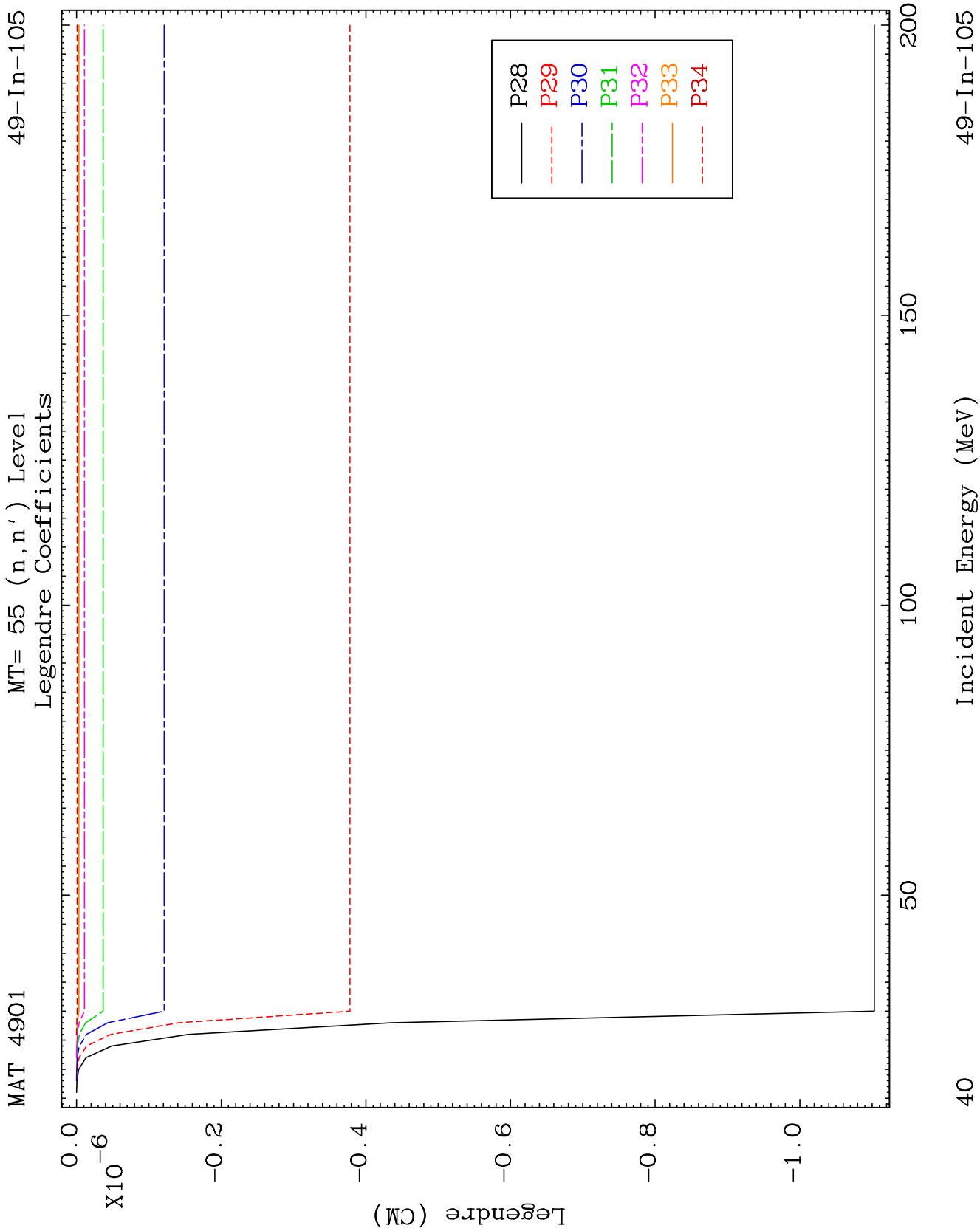
49-In-105

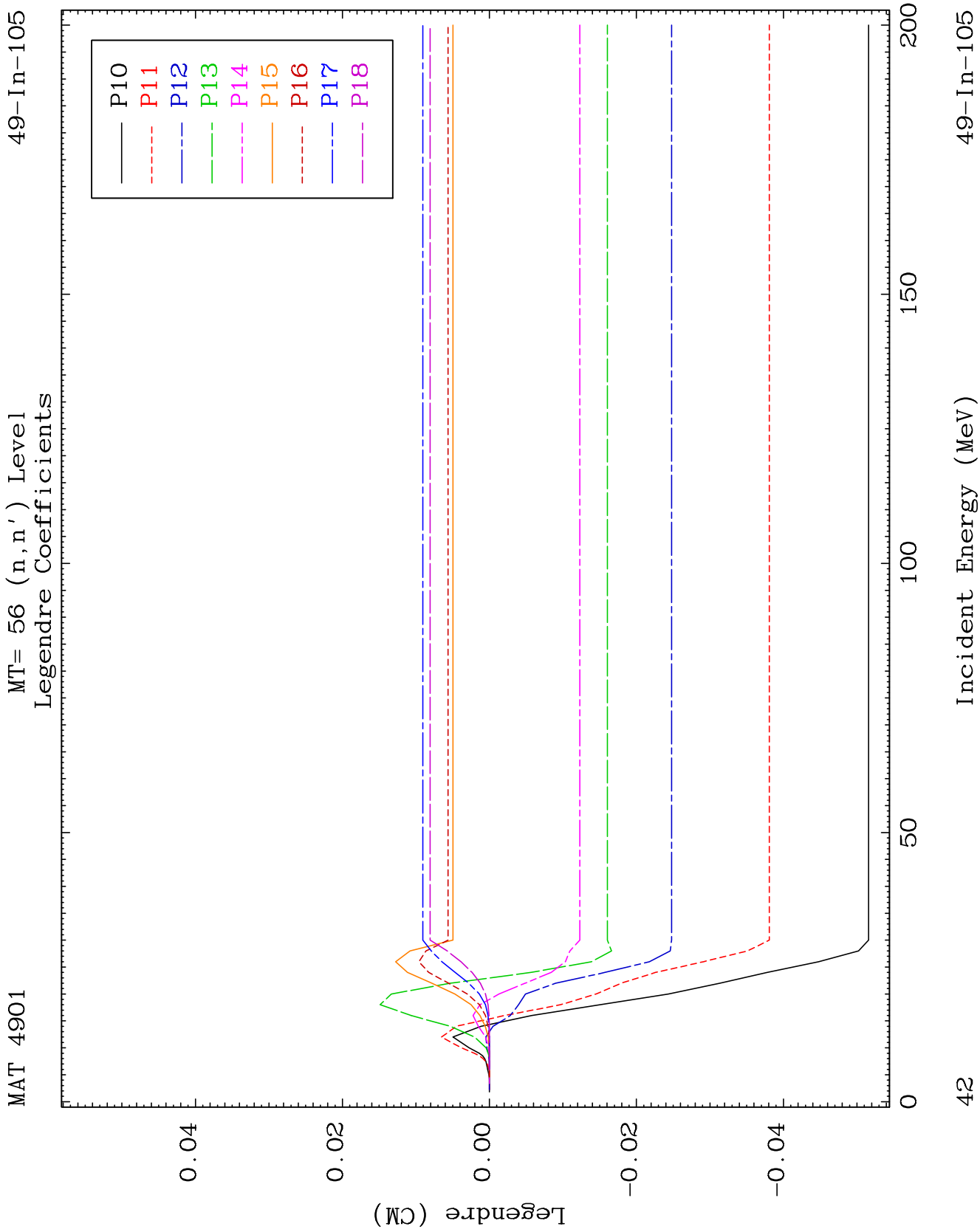


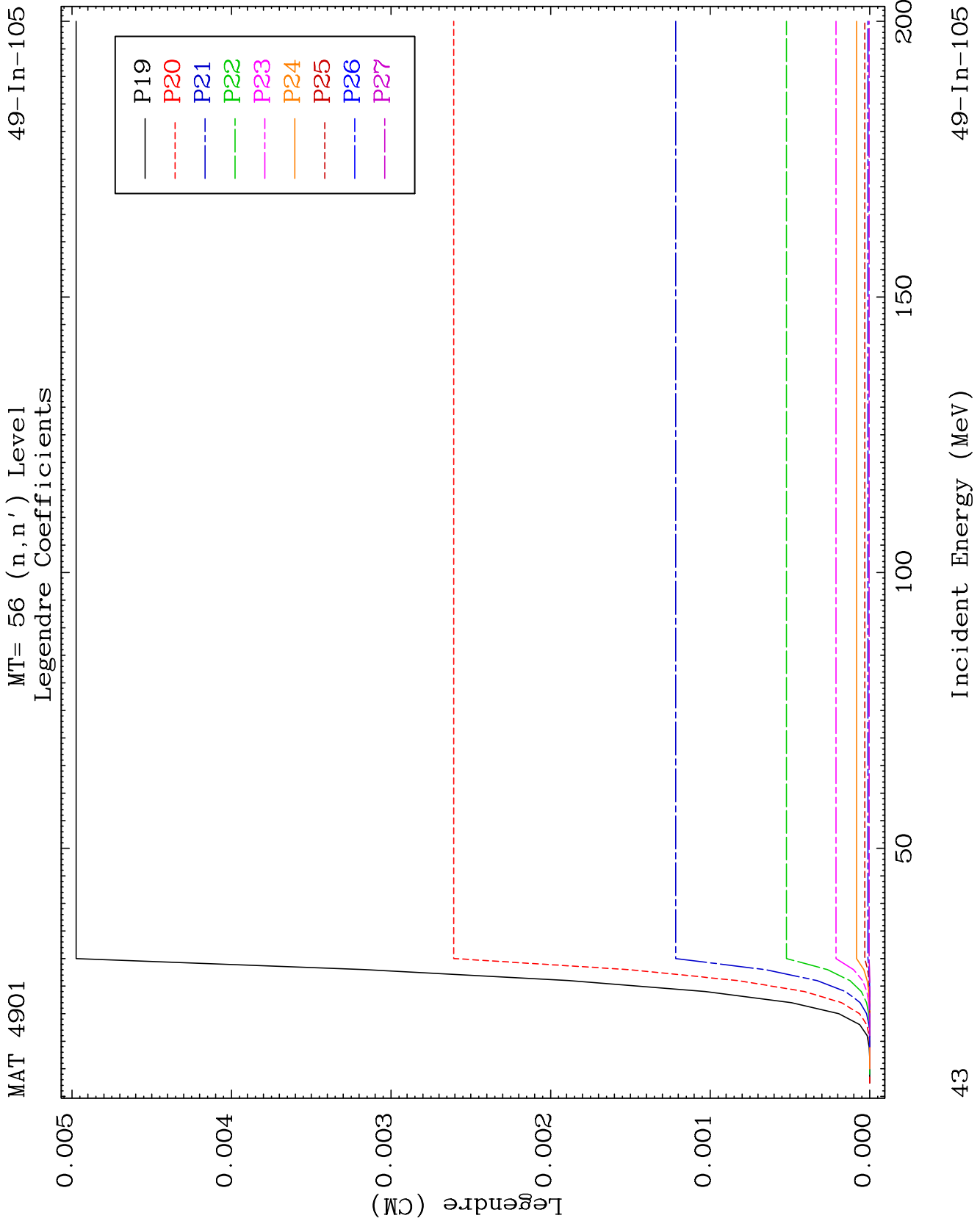
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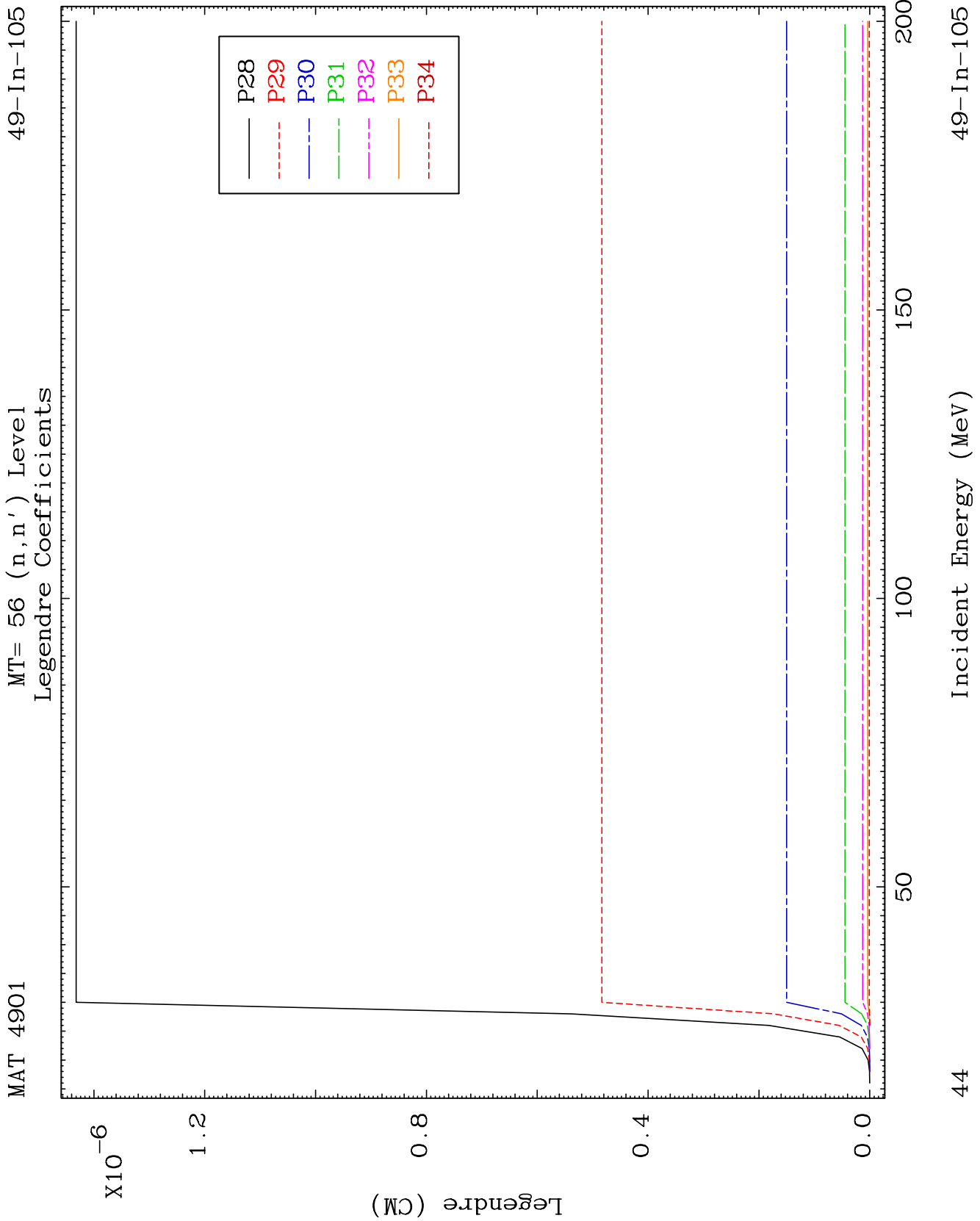
Incident Energy (MeV)

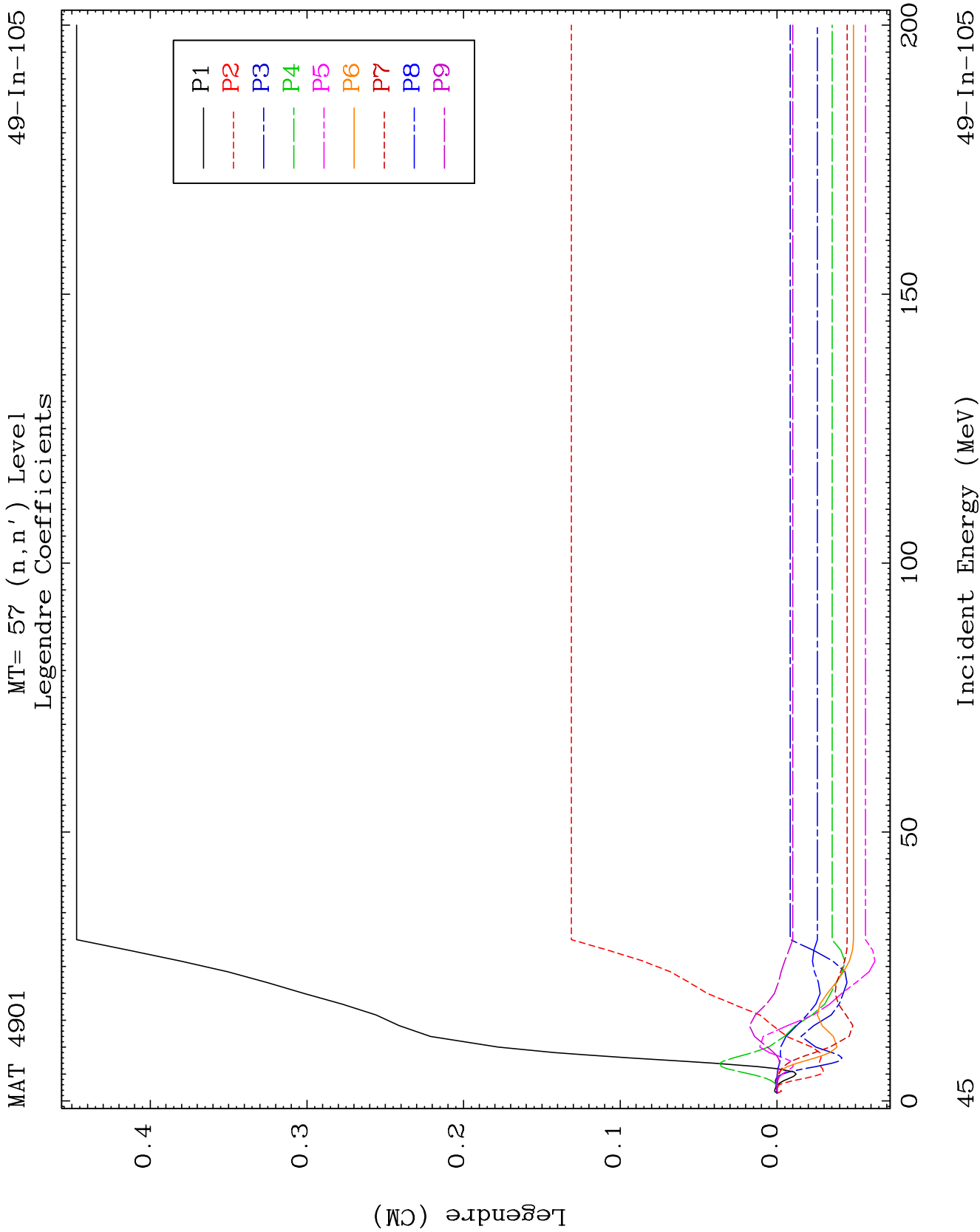
49-In-105







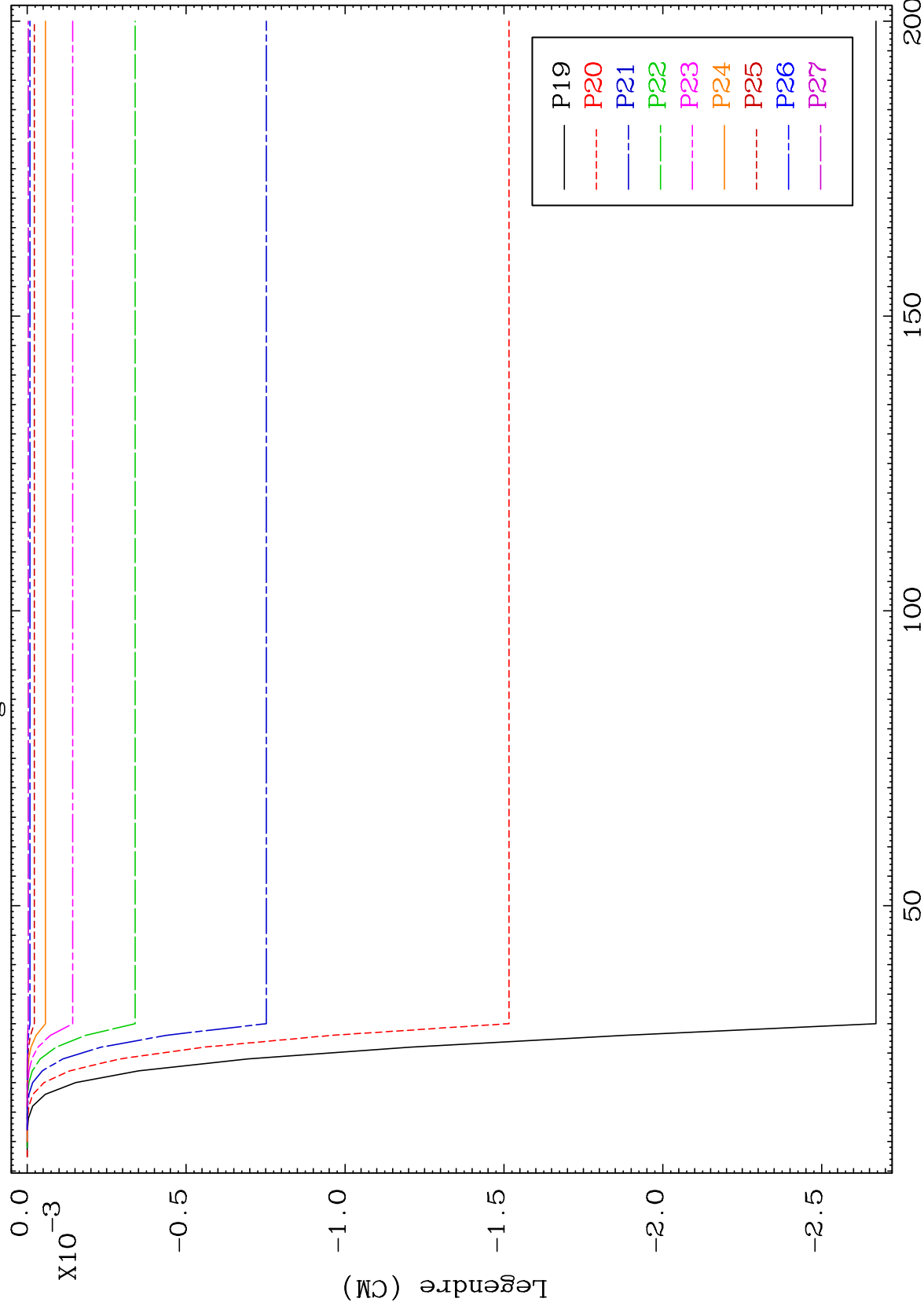




MAT 4901

MT= 57 (n, n') Level
Legendre Coefficients

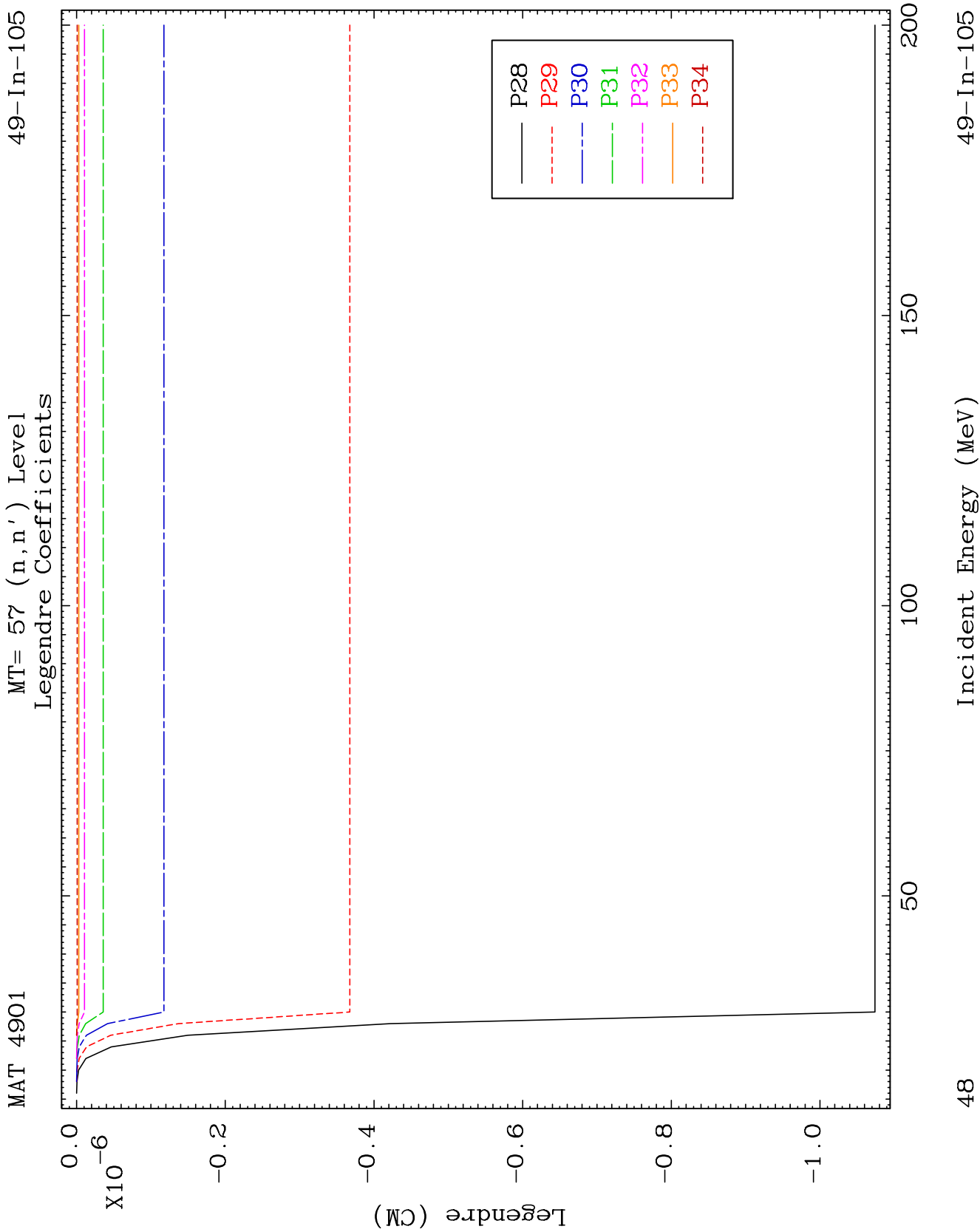
49-In-105



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Incident Energy (MeV)

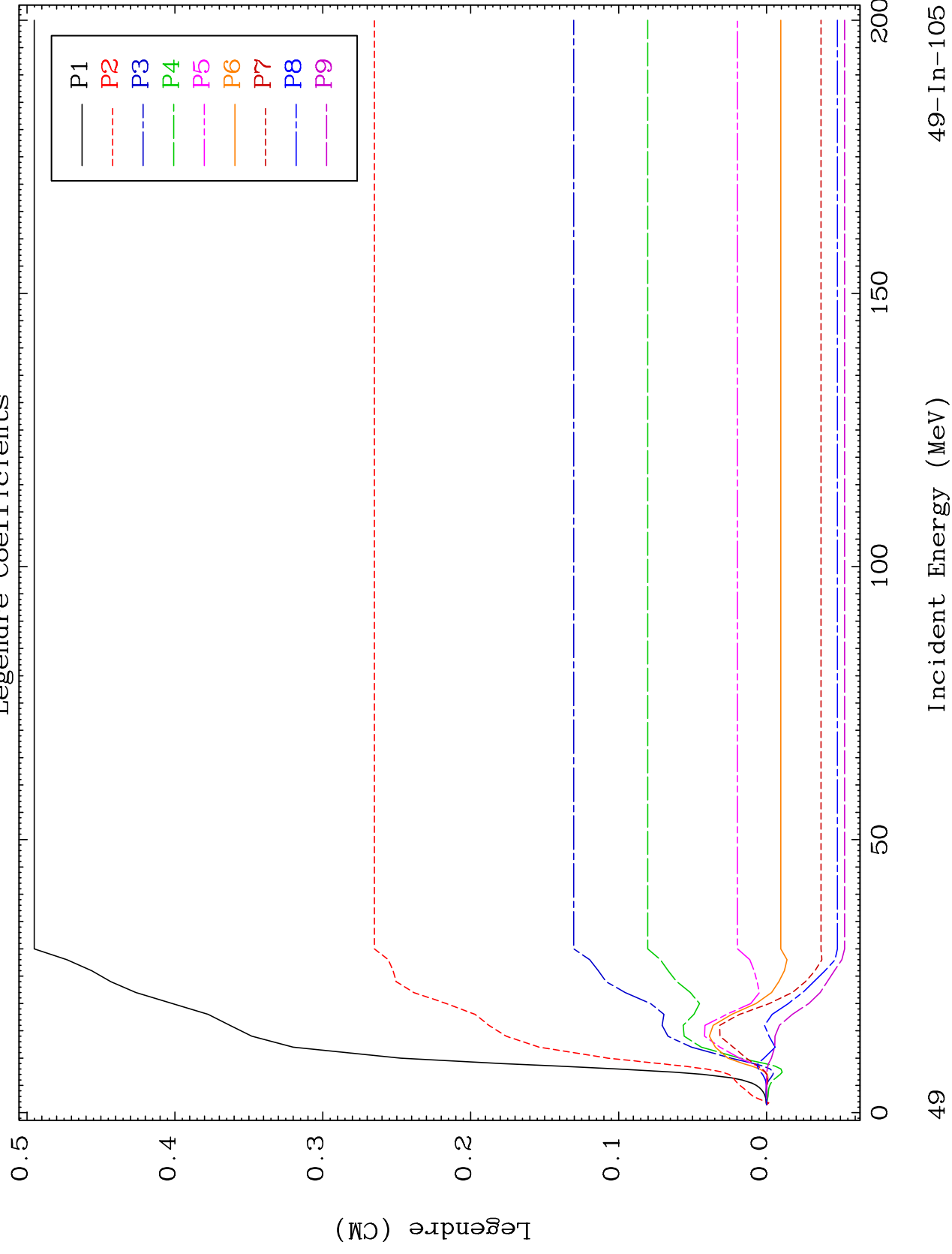
49-In-105

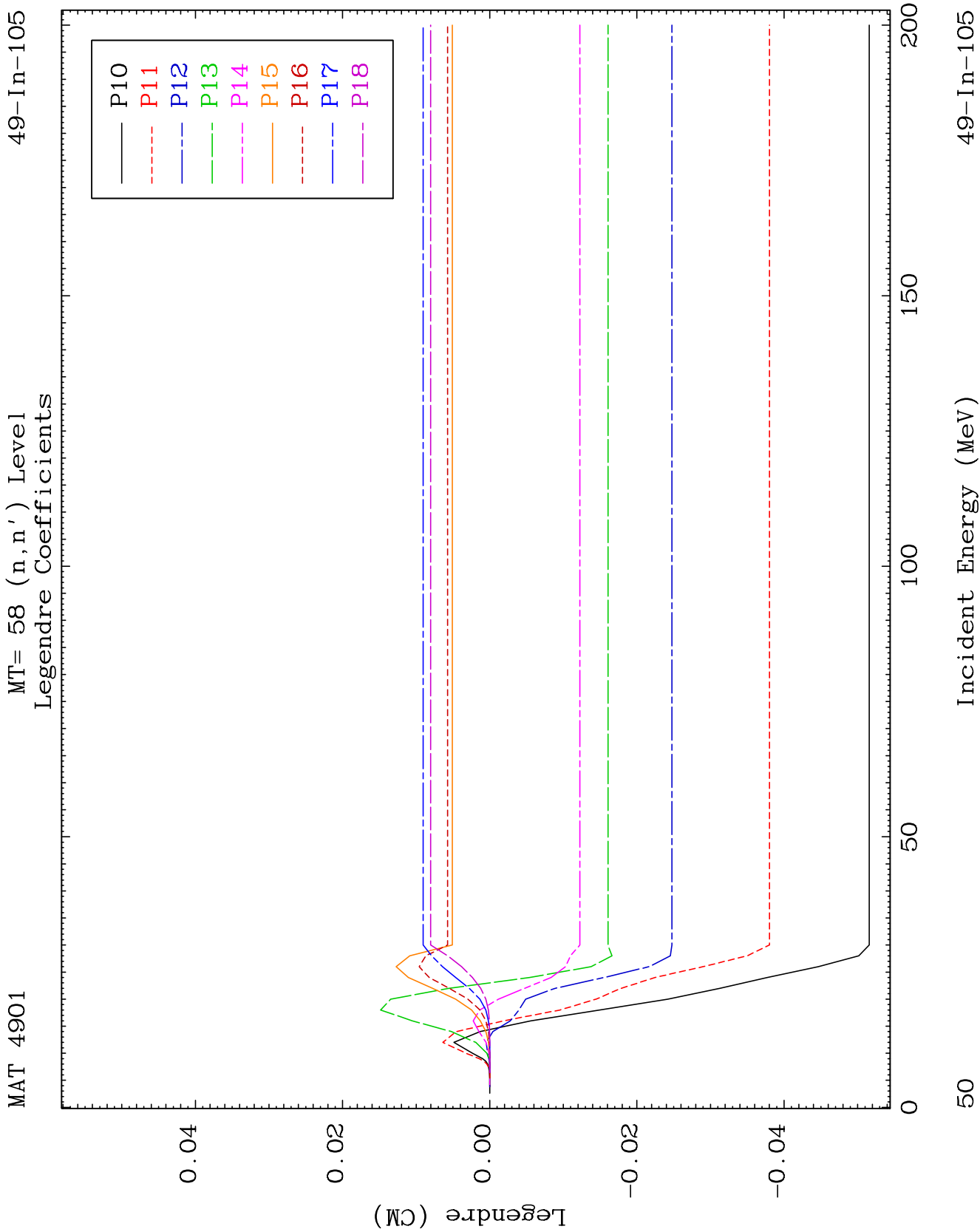


MAT 4901

MT= 58 (n, n') Level
Legendre Coefficients

49-In-105

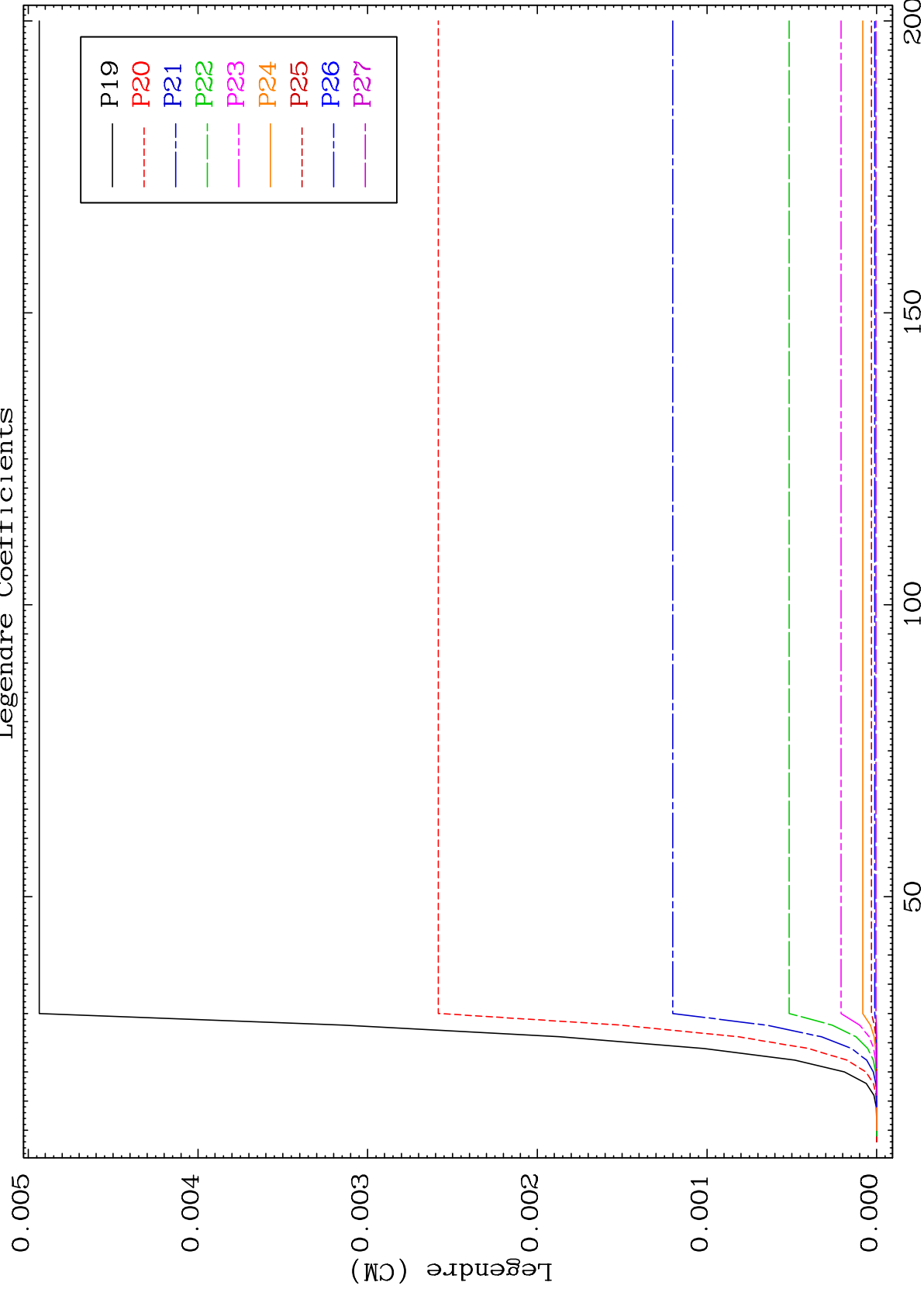




MAT 4901

MT= 58 (n, n') Level
Legendre Coefficients

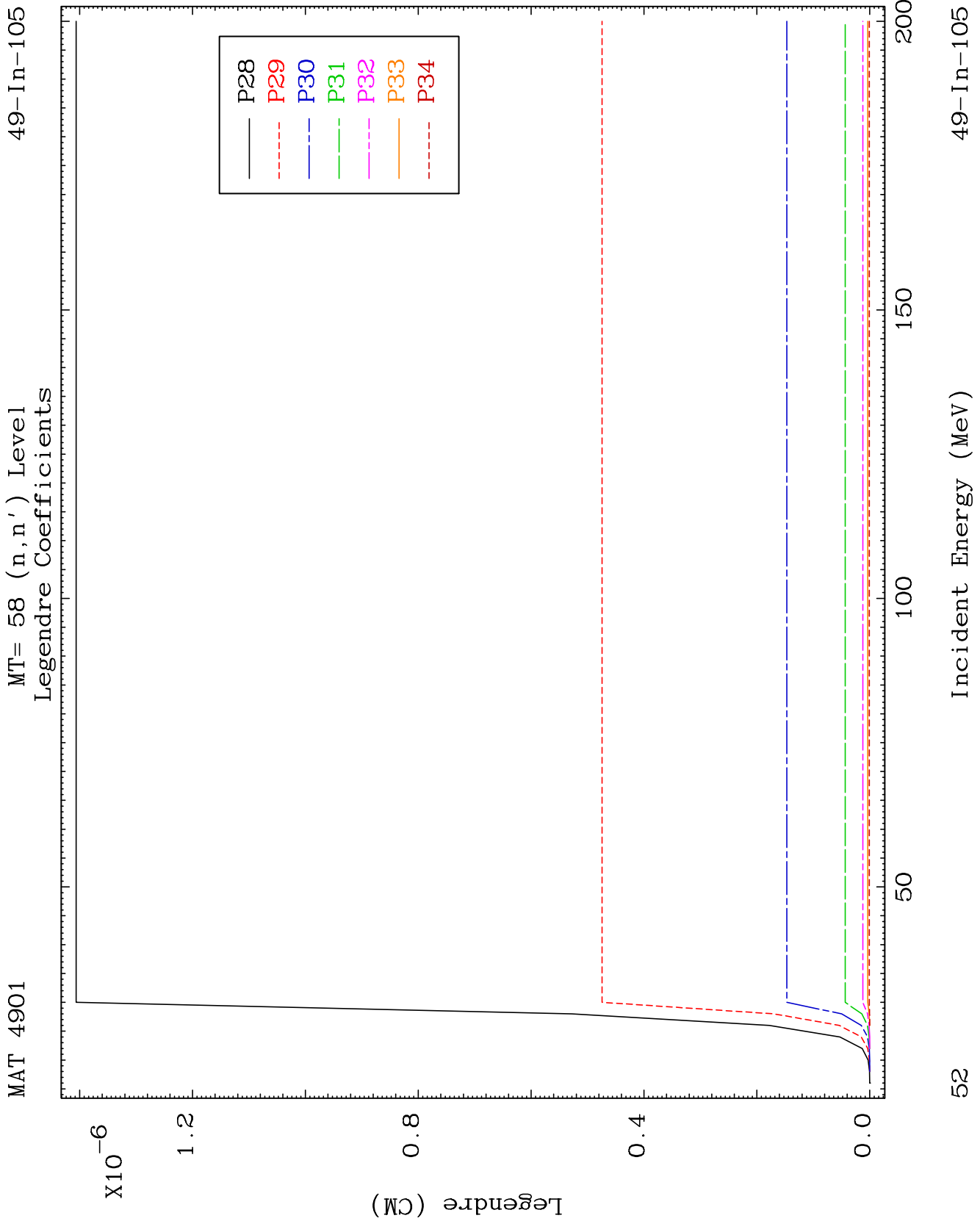
49-In-105

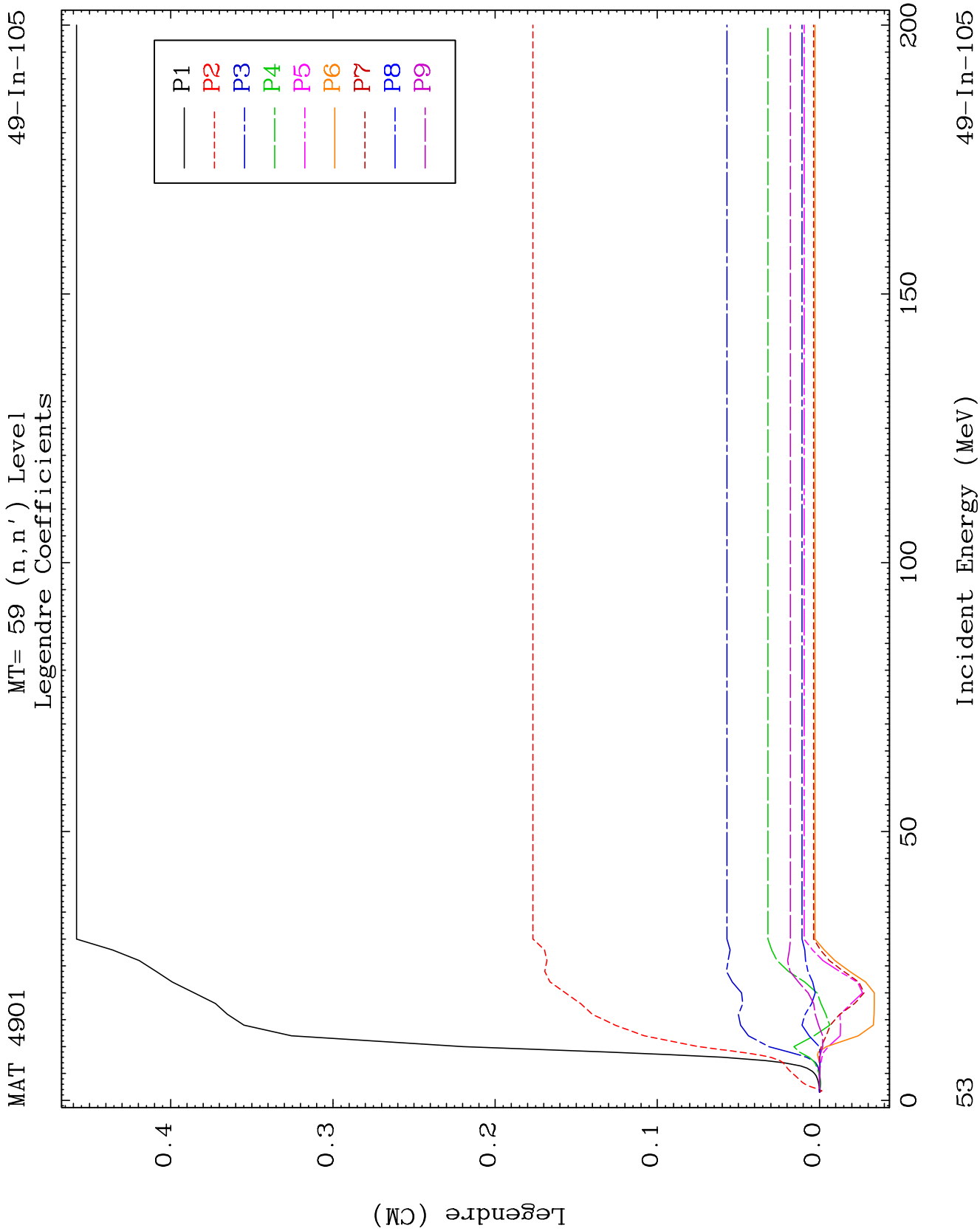


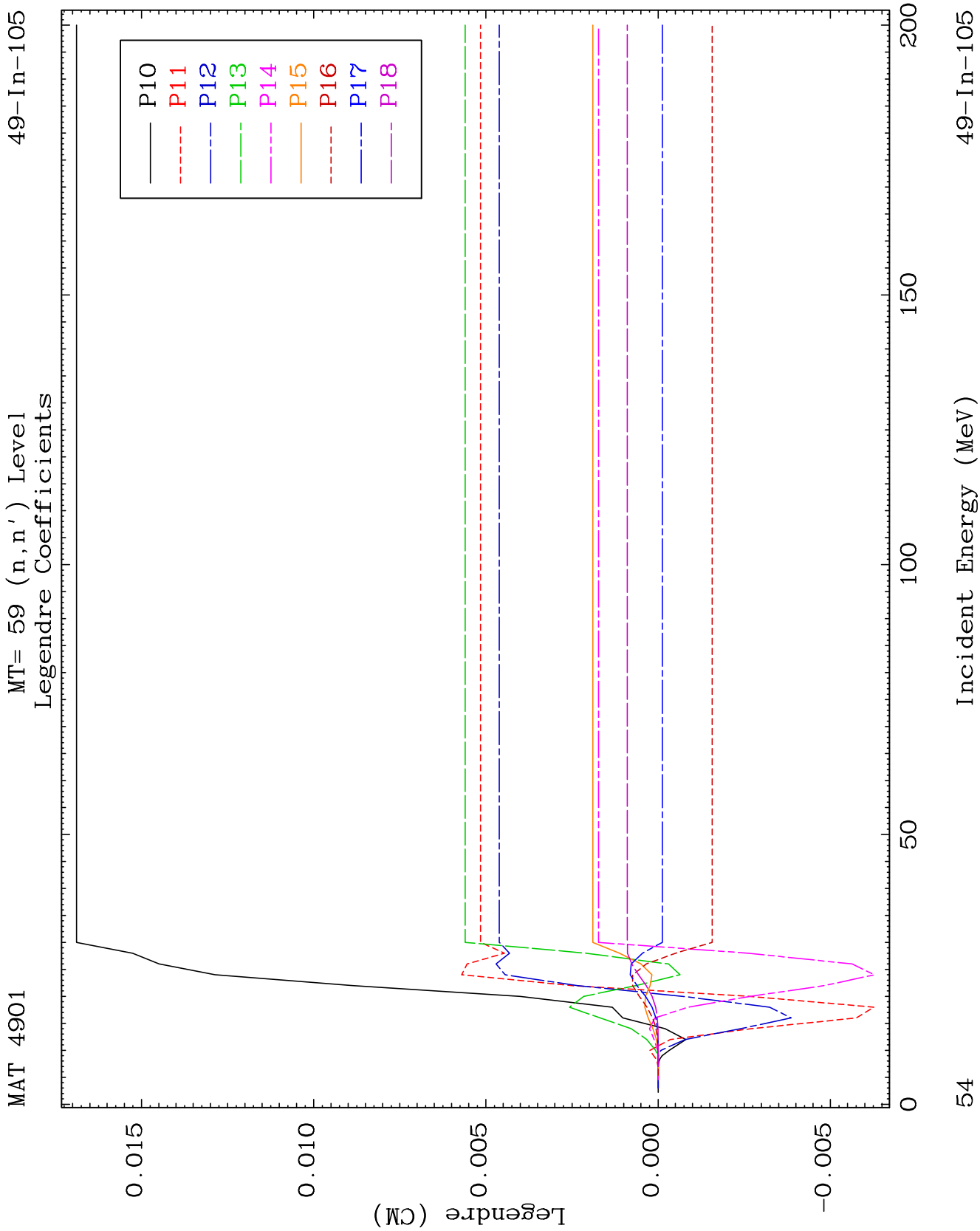
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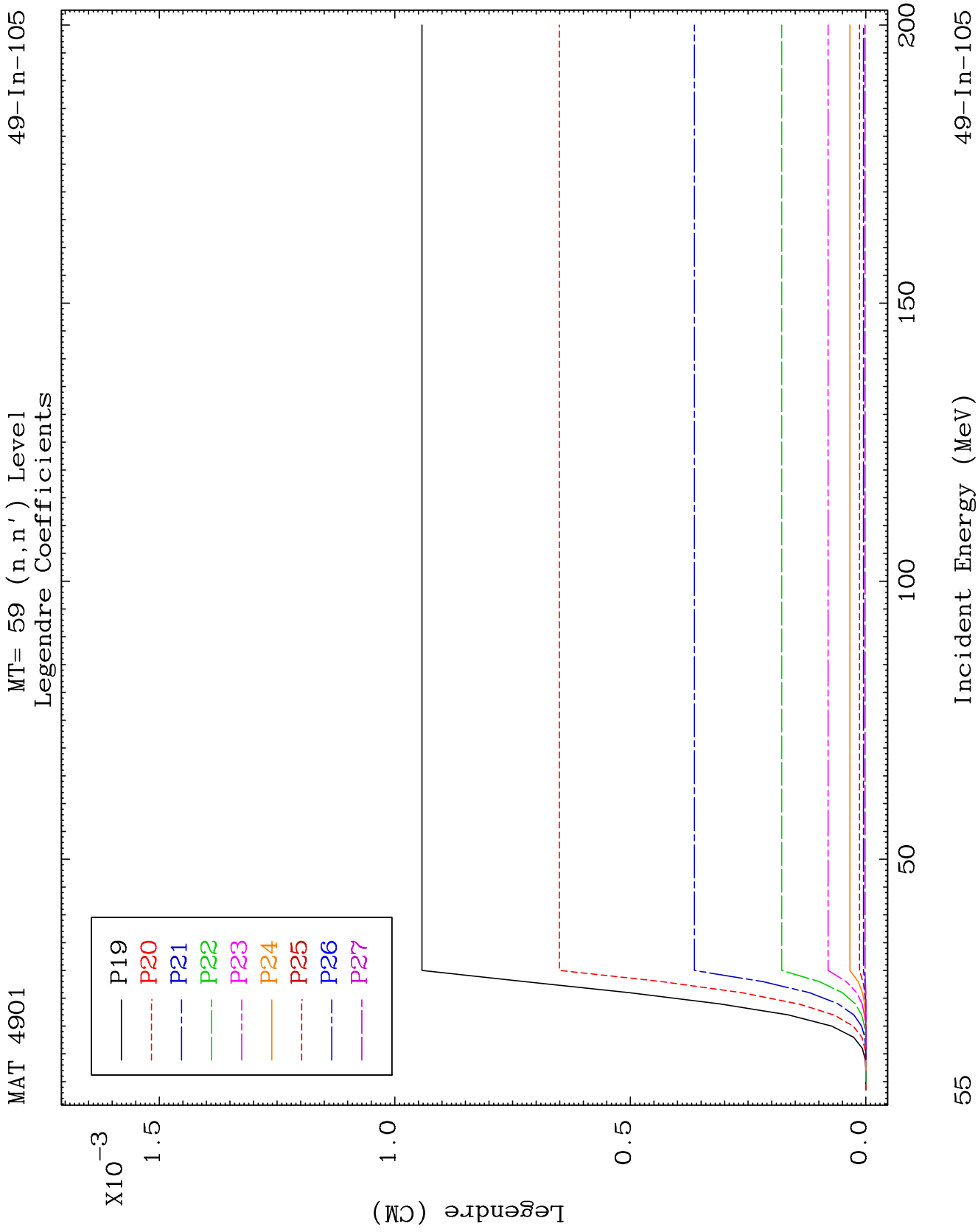
Incident Energy (MeV)

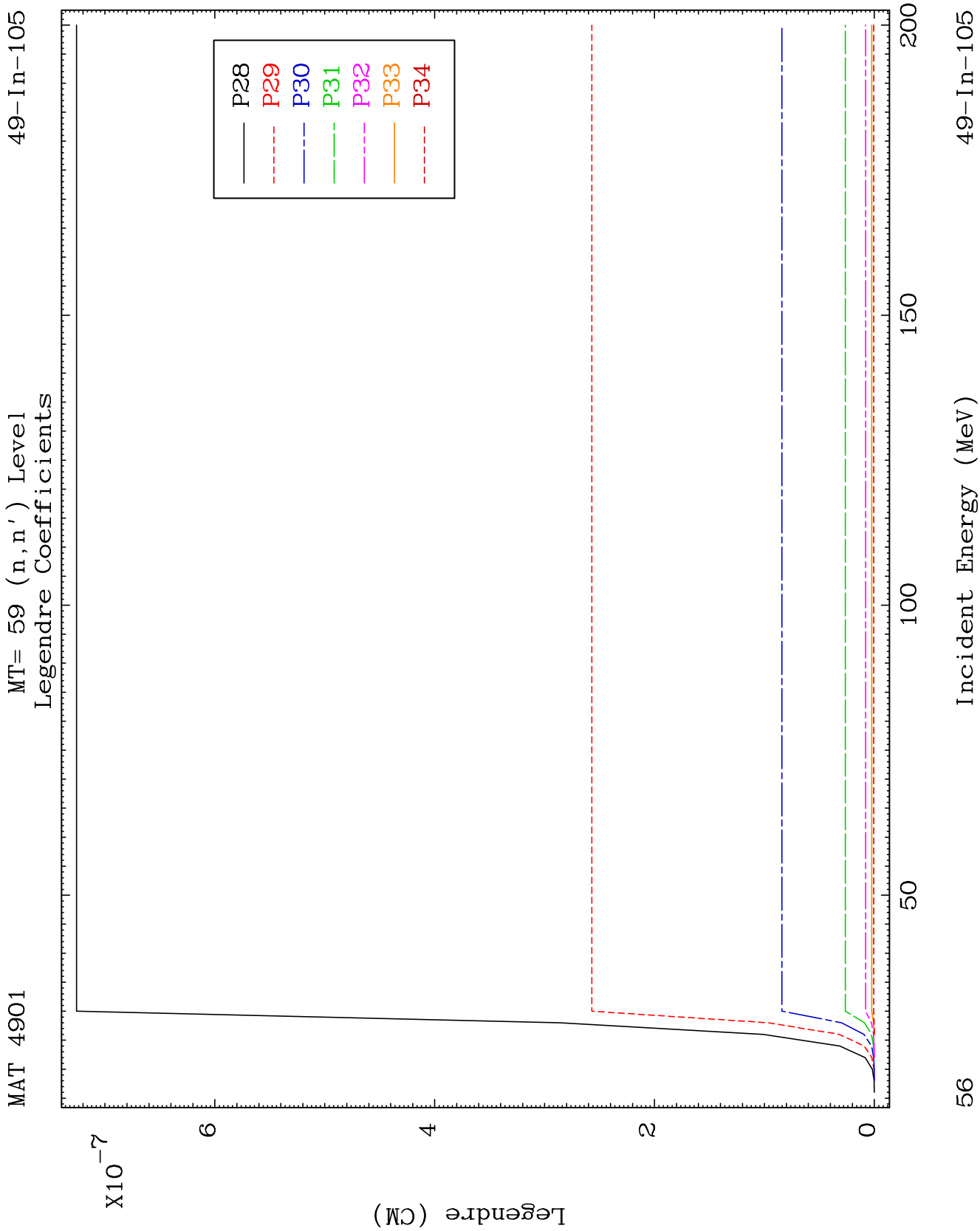
49-In-105

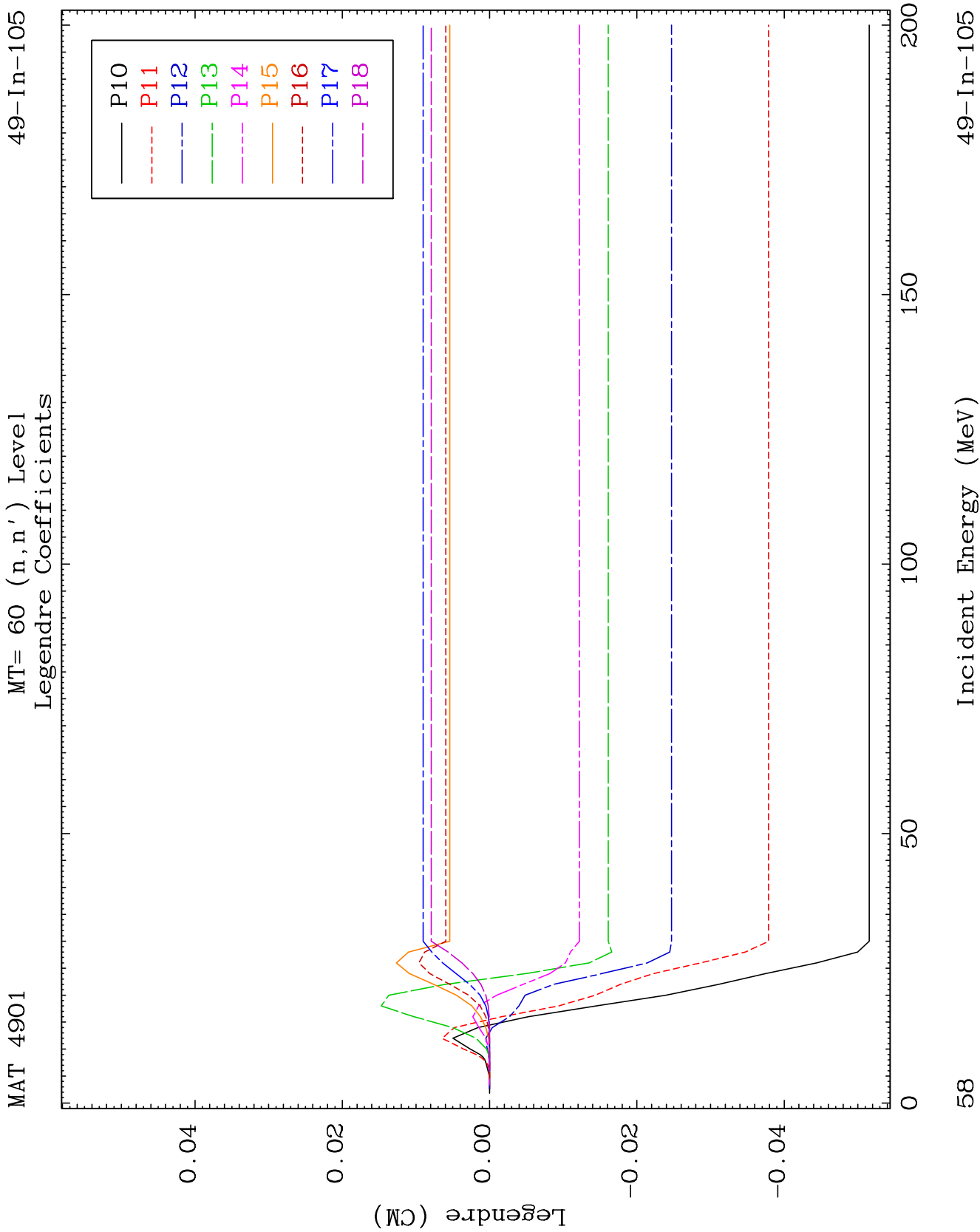


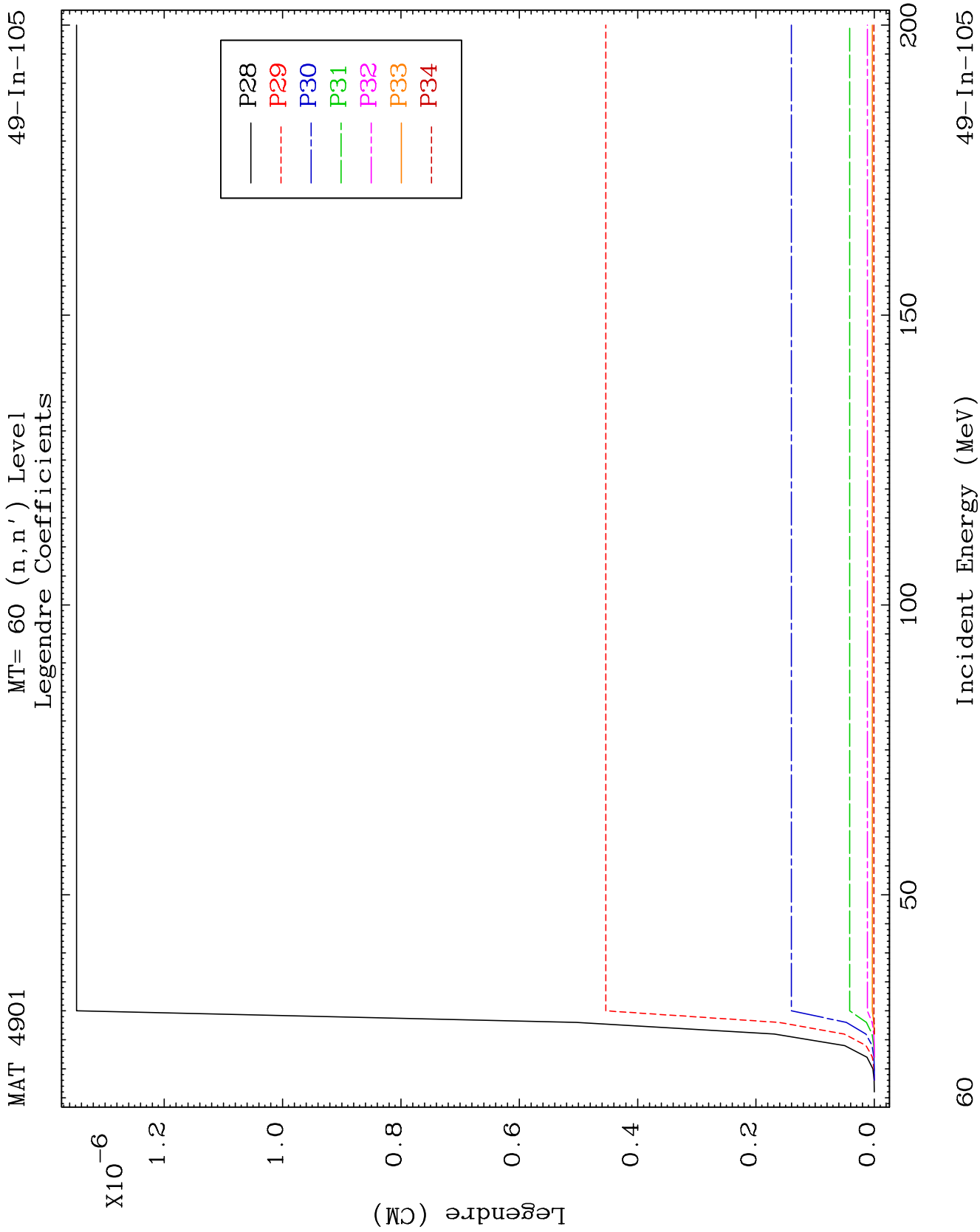








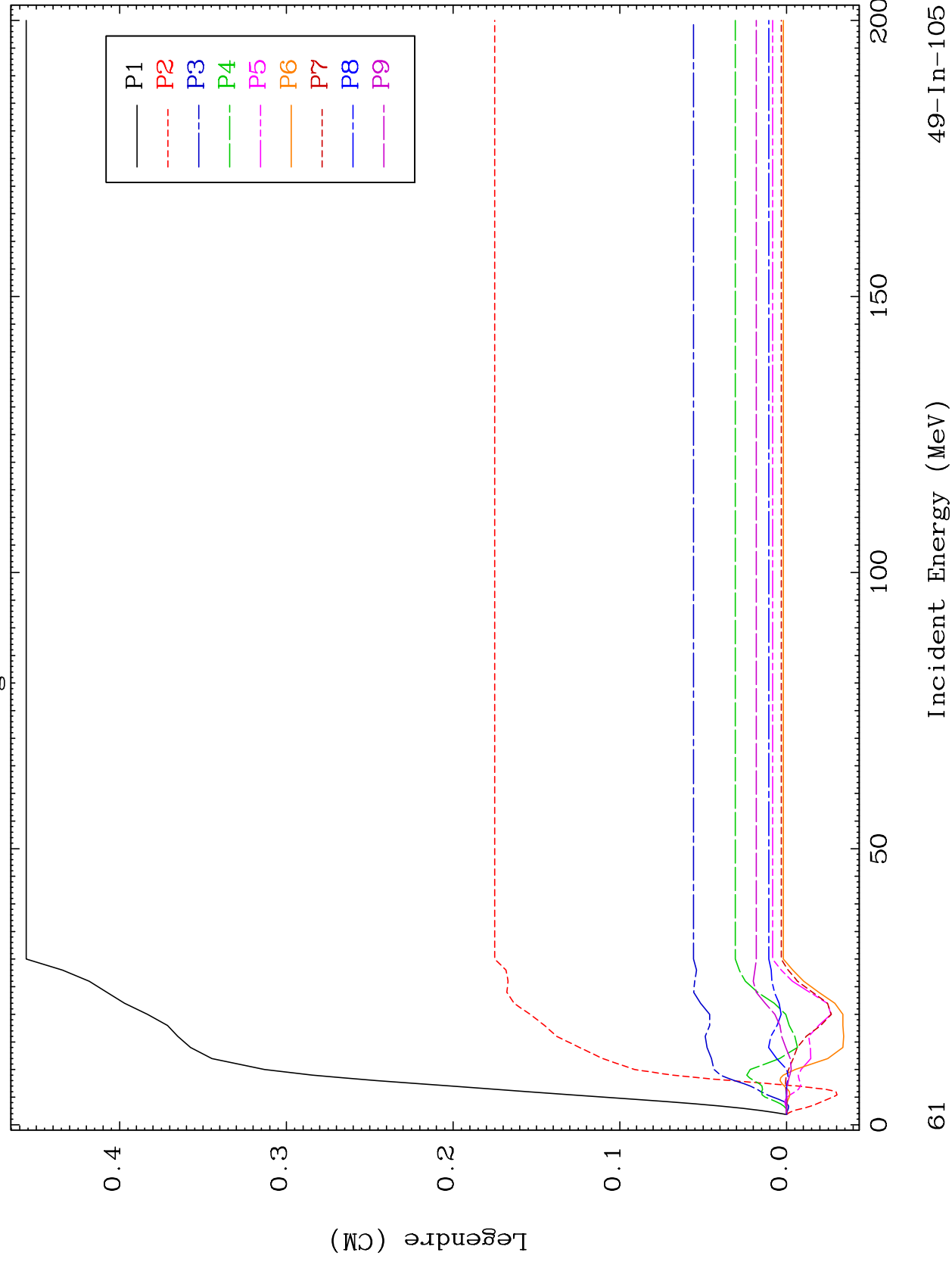


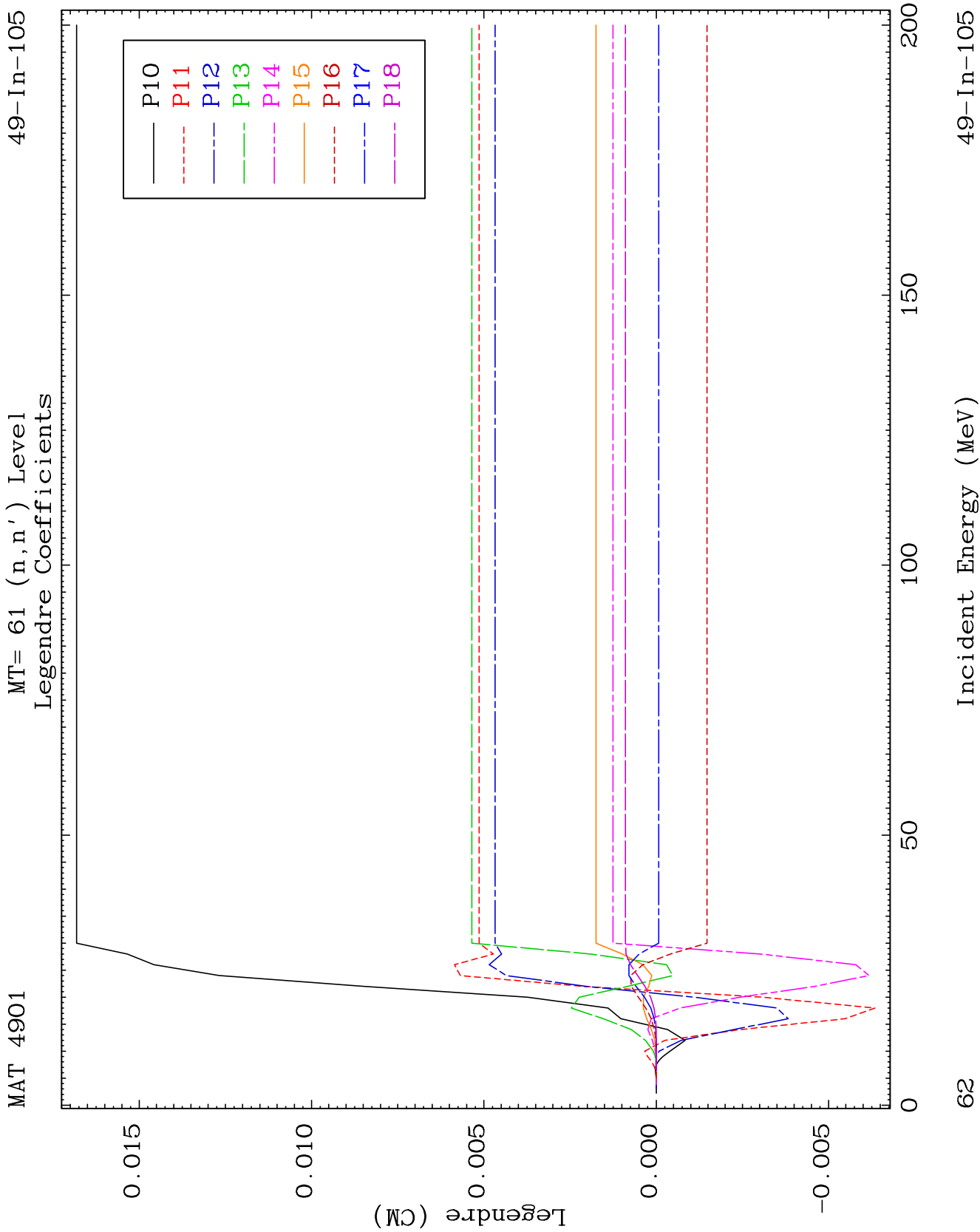


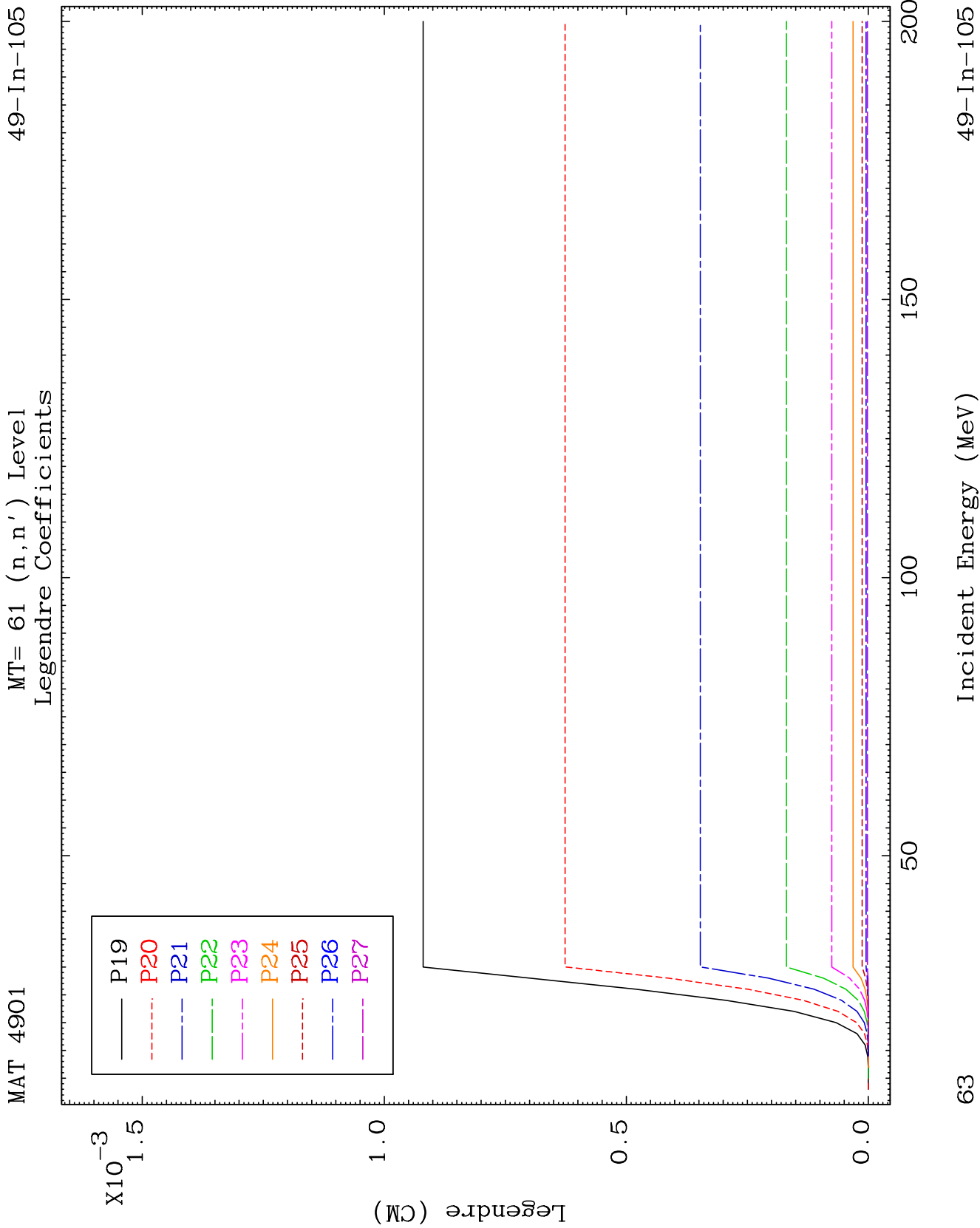
MAT 4901

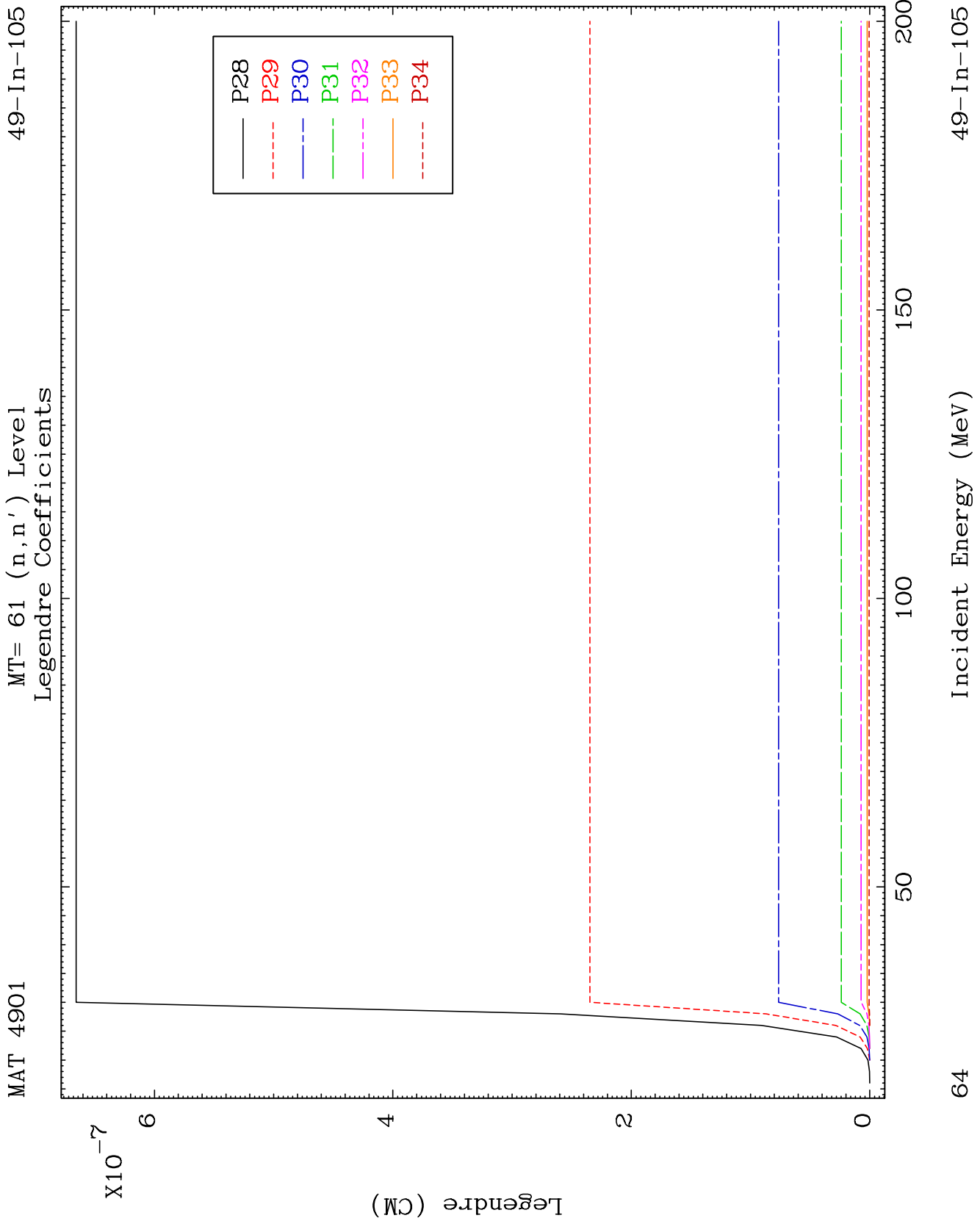
MT= 61 (n, n') Level
Legendre Coefficients

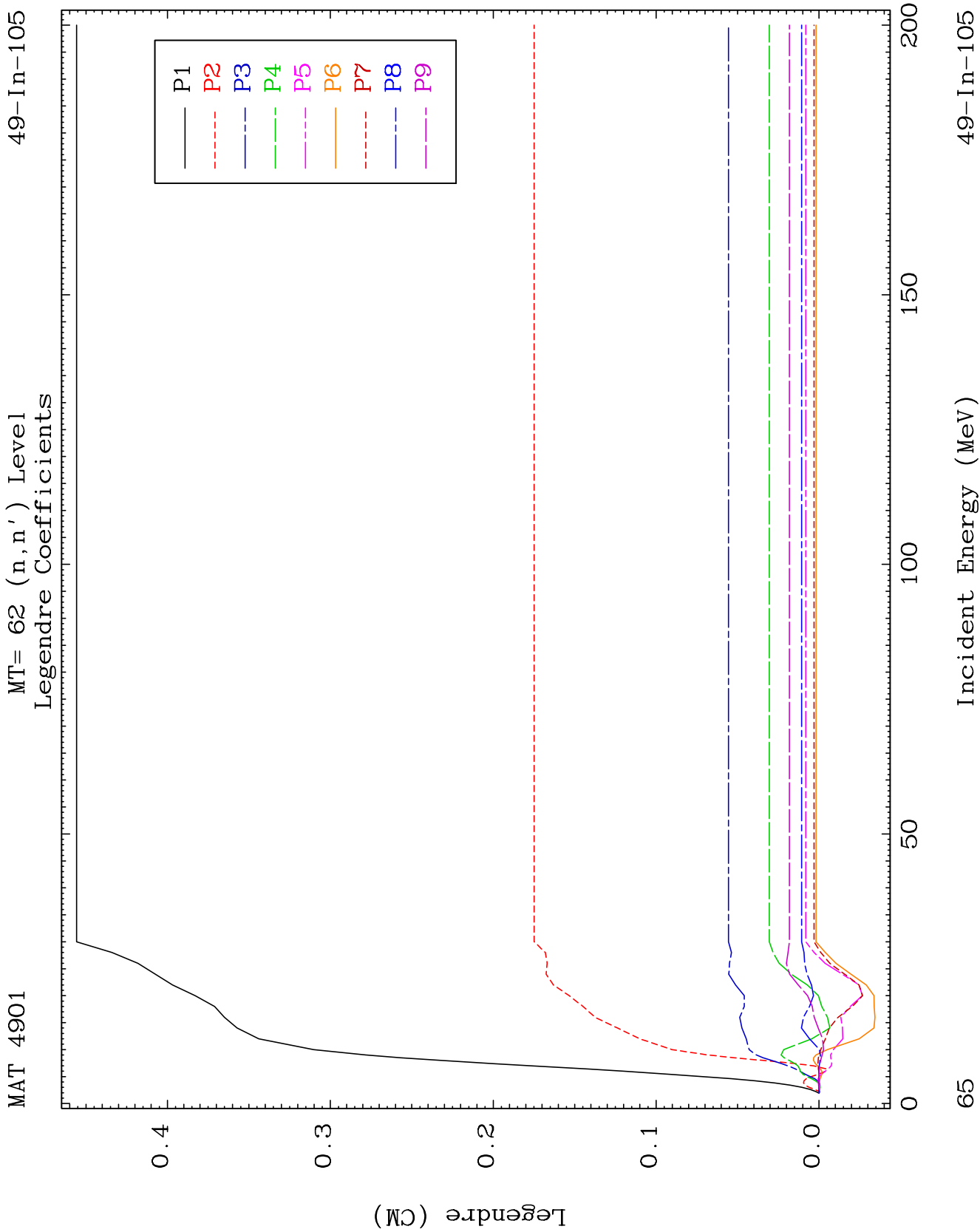
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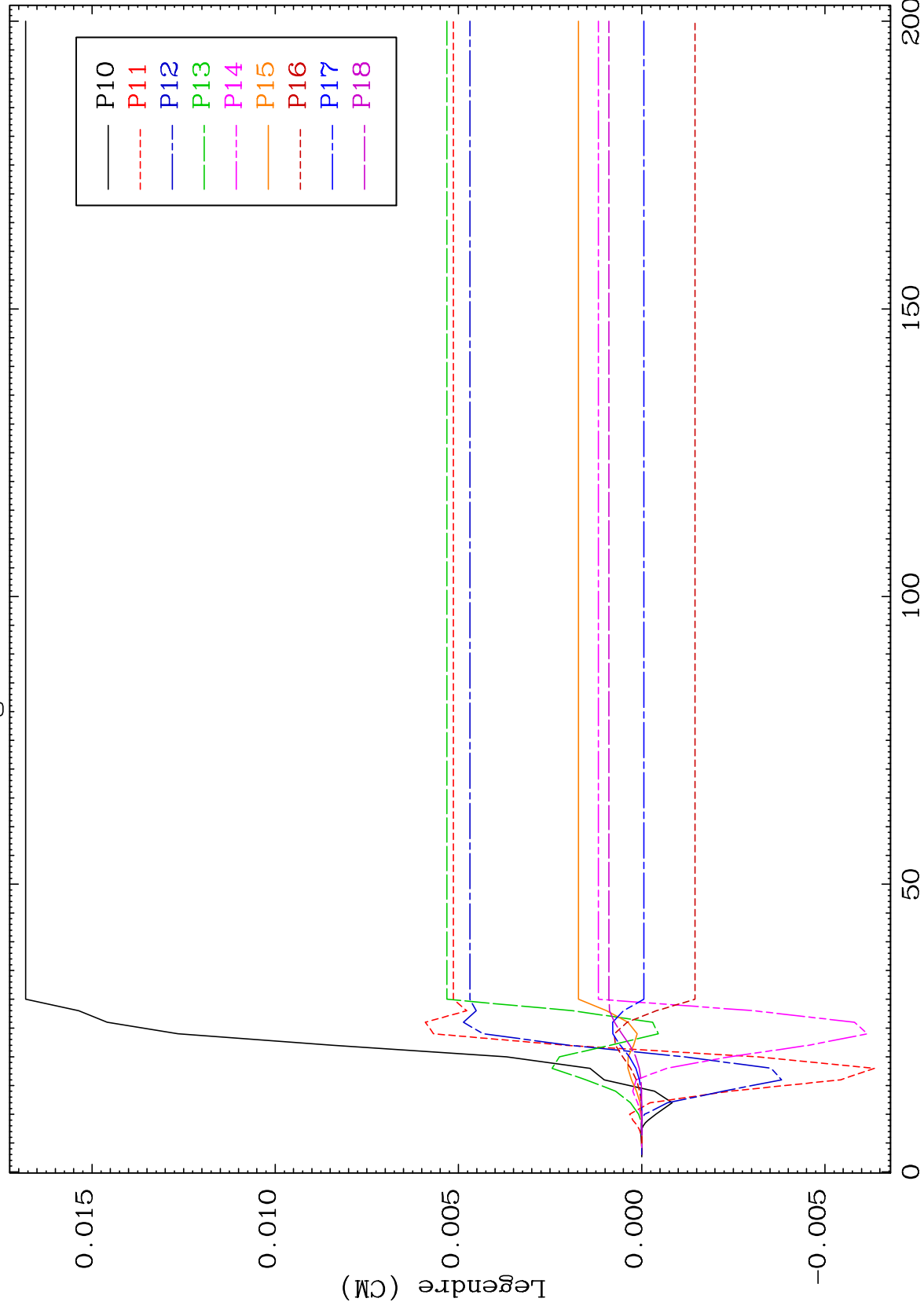




MAT 4901

MT= 62 (n, n') Level
Legendre Coefficients

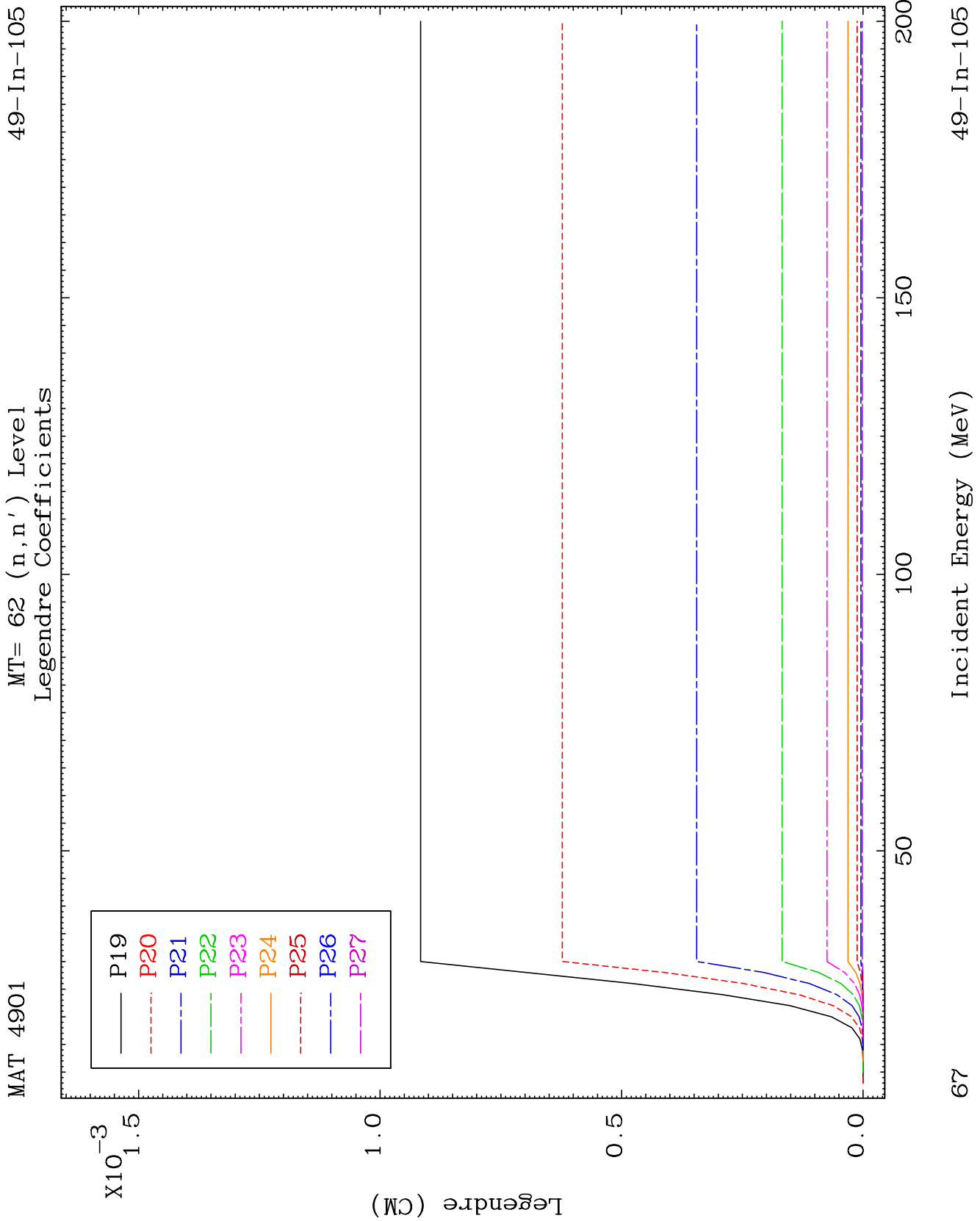
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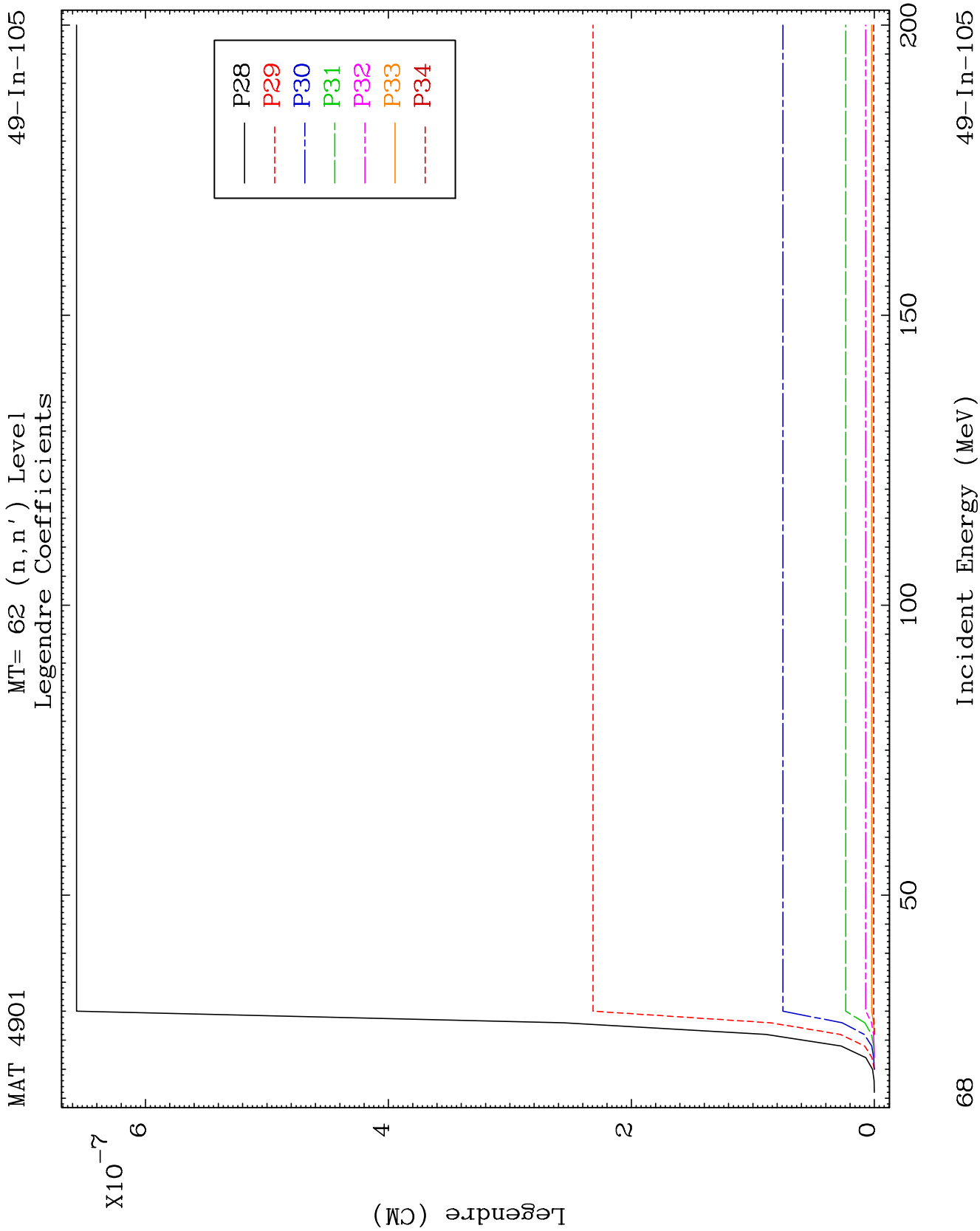


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Incident Energy (MeV)

49-In-105

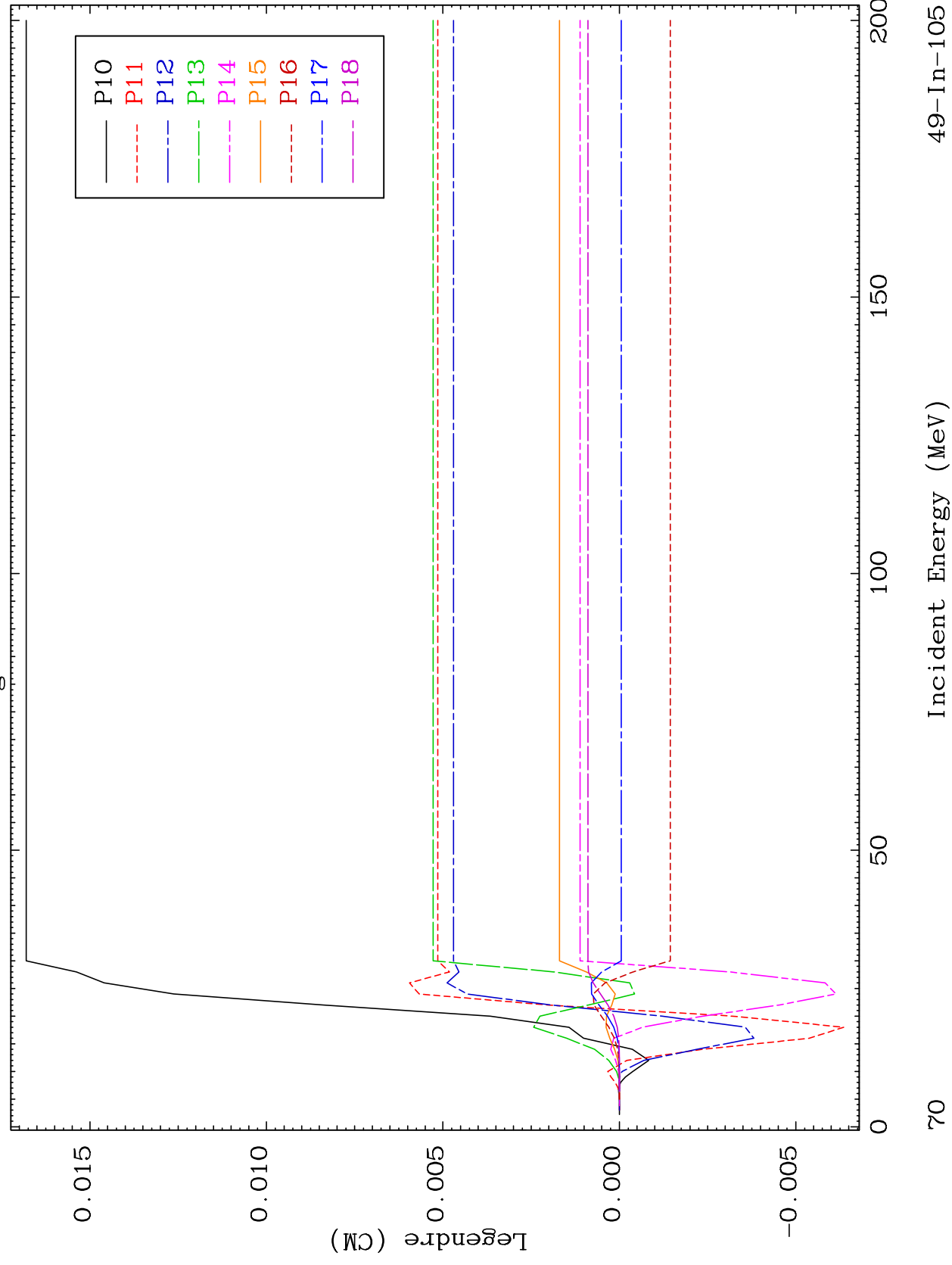


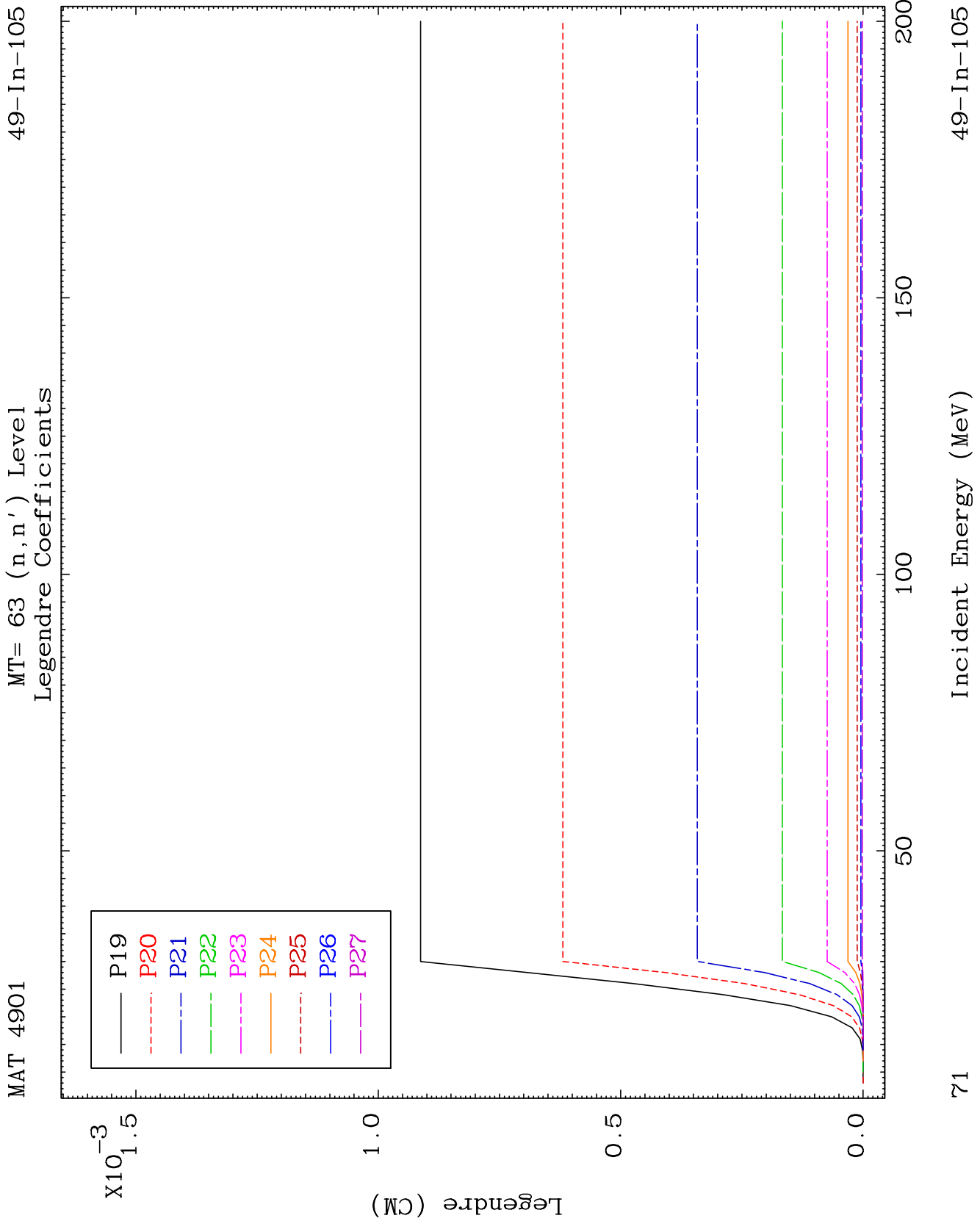


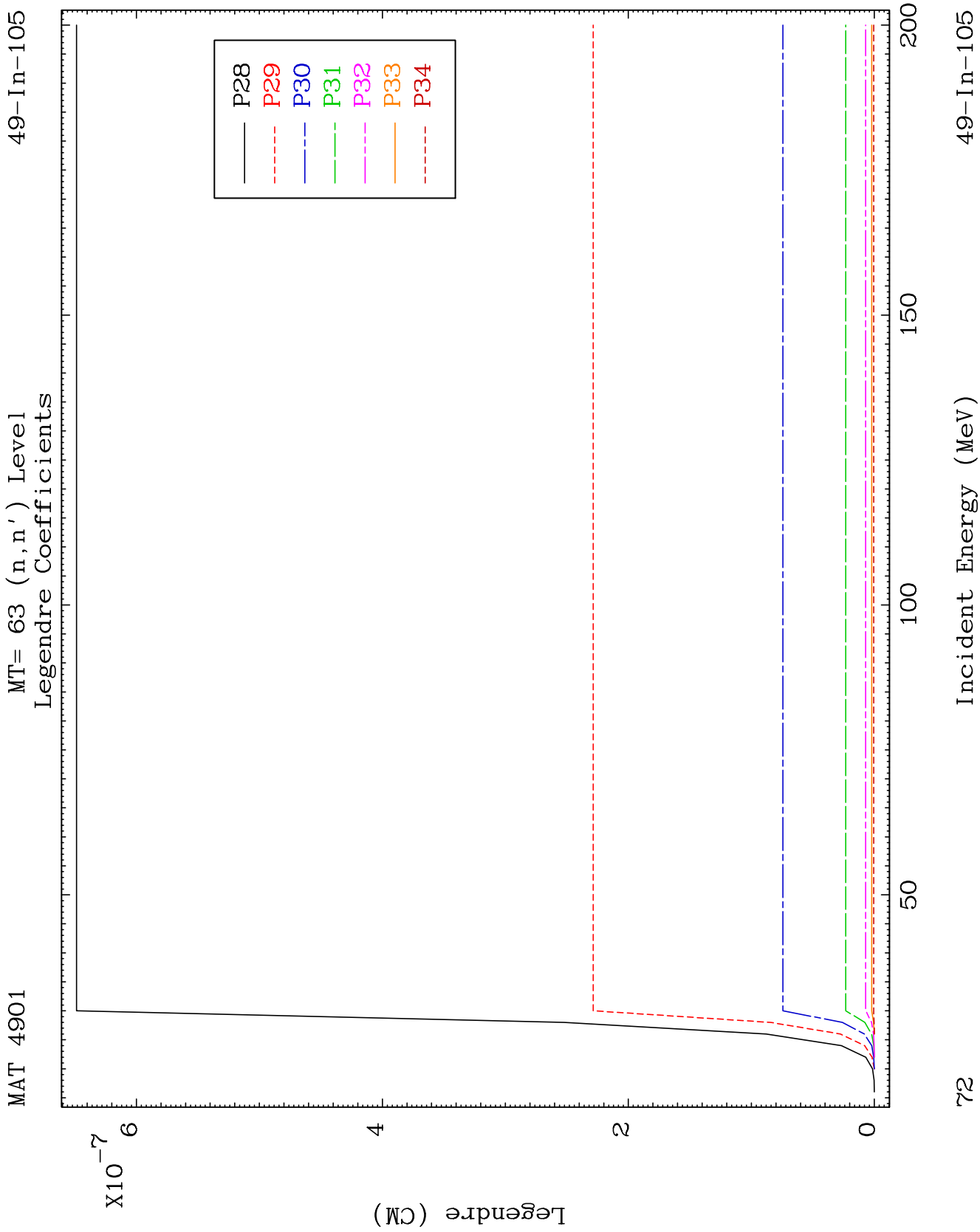
MAT 4901

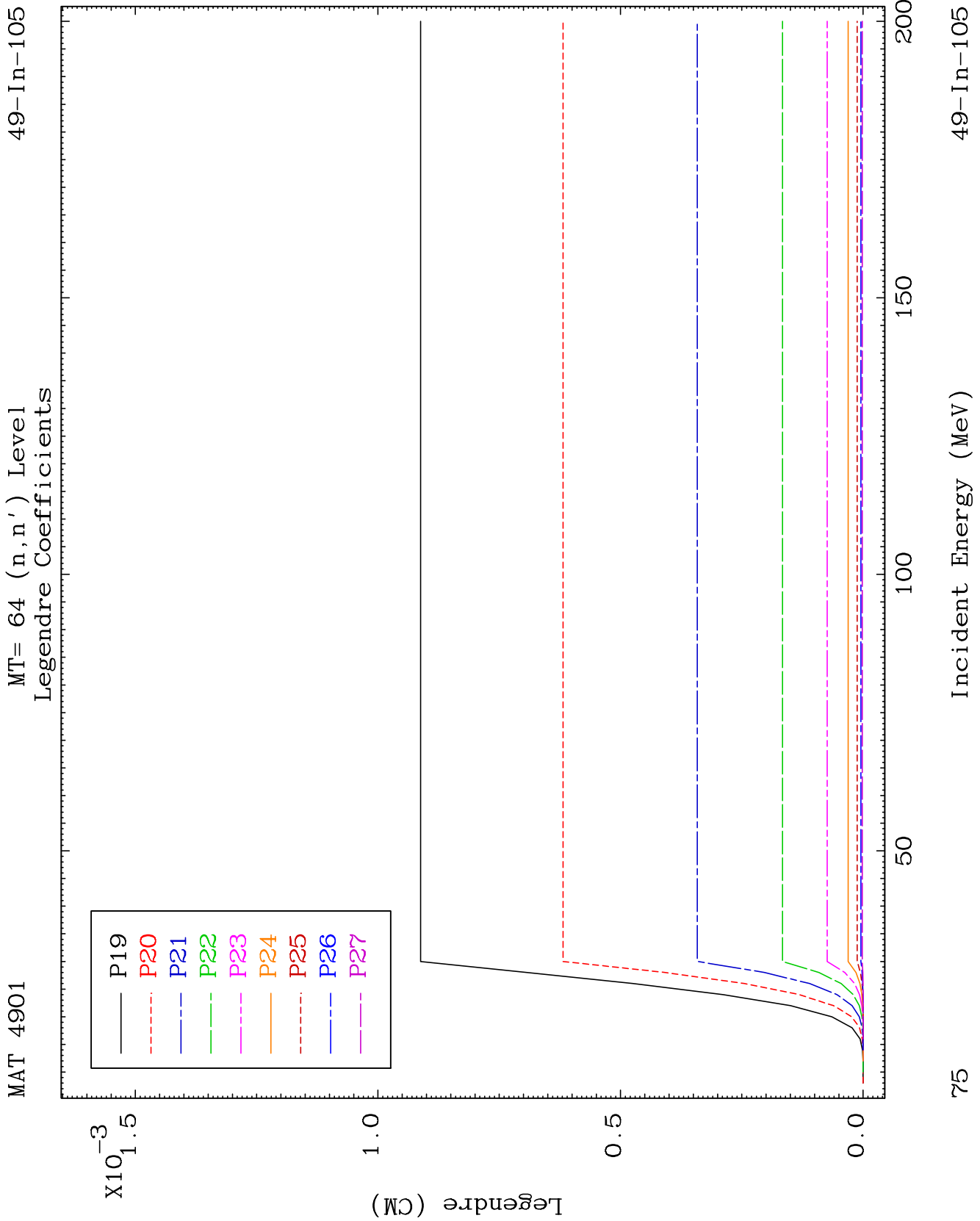
MT= 63 (n, n') Level
Legendre Coefficients

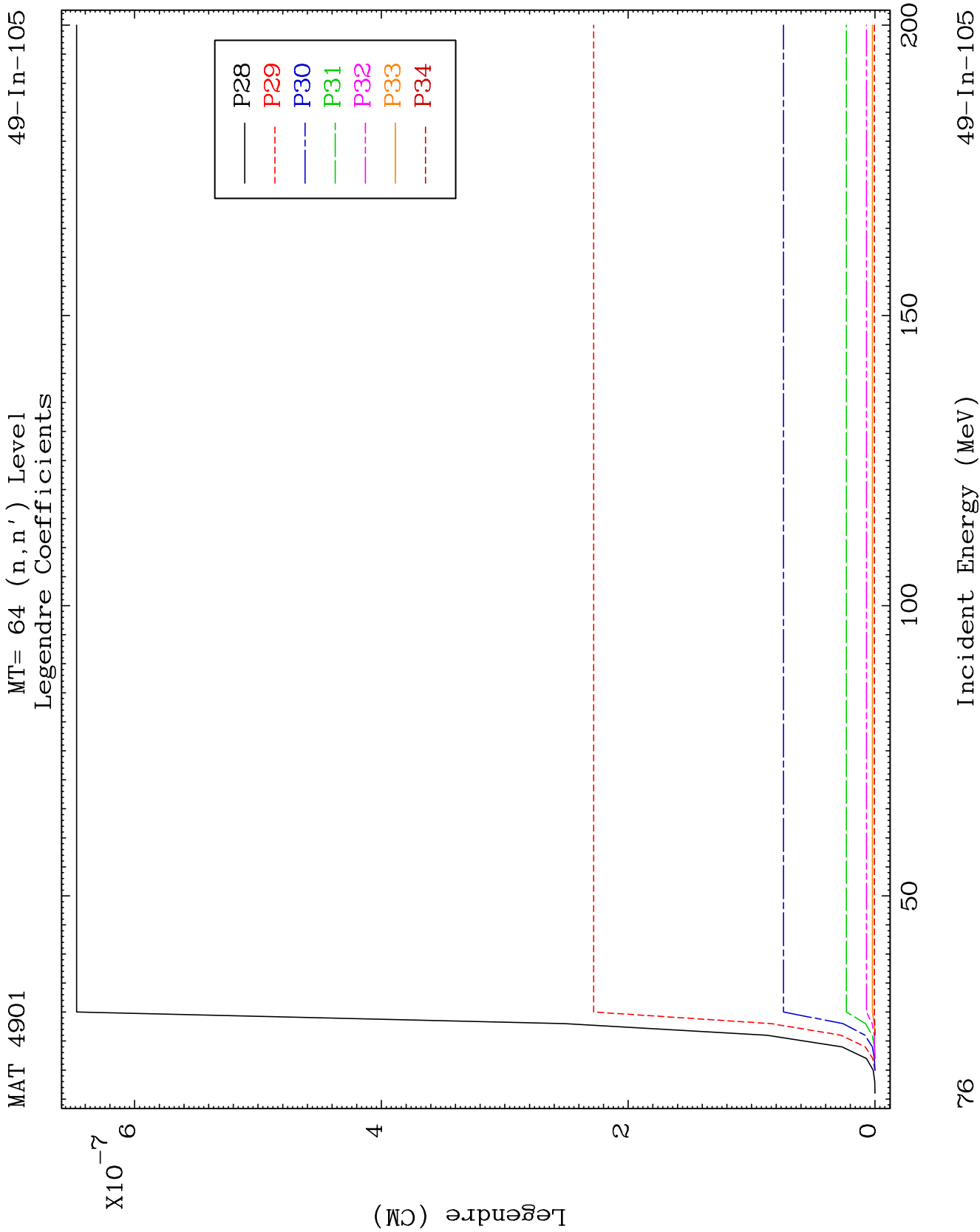
49-In-105

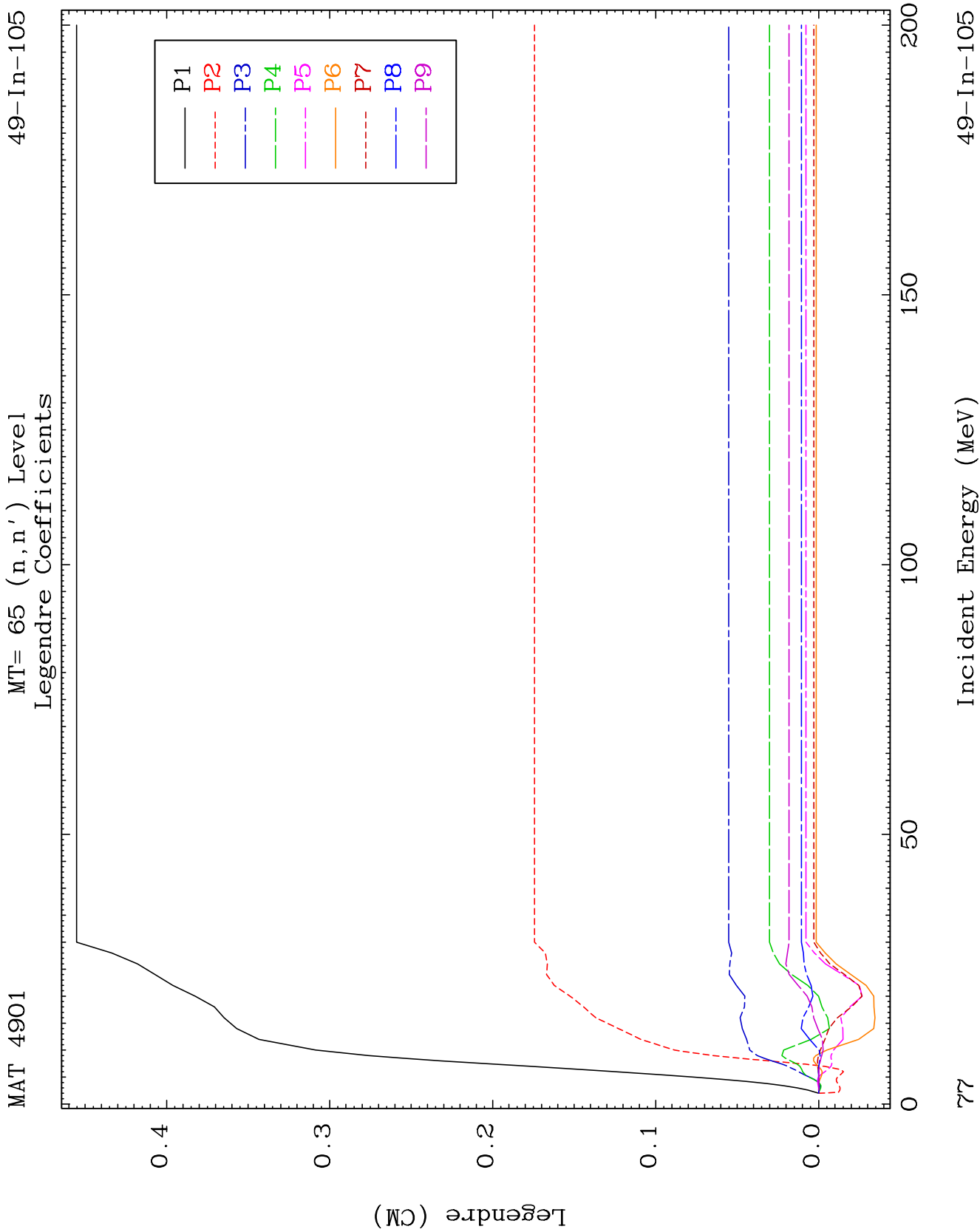


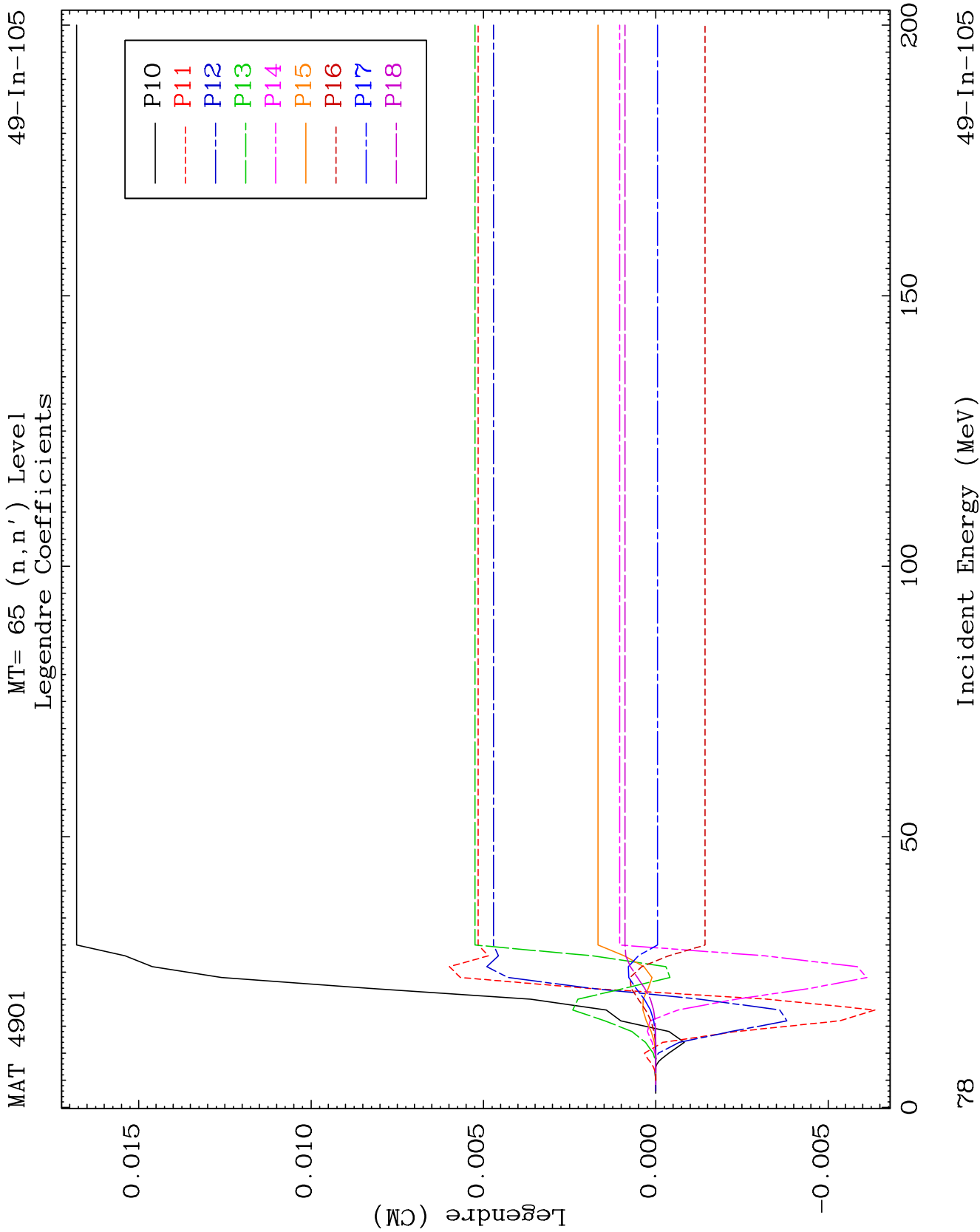


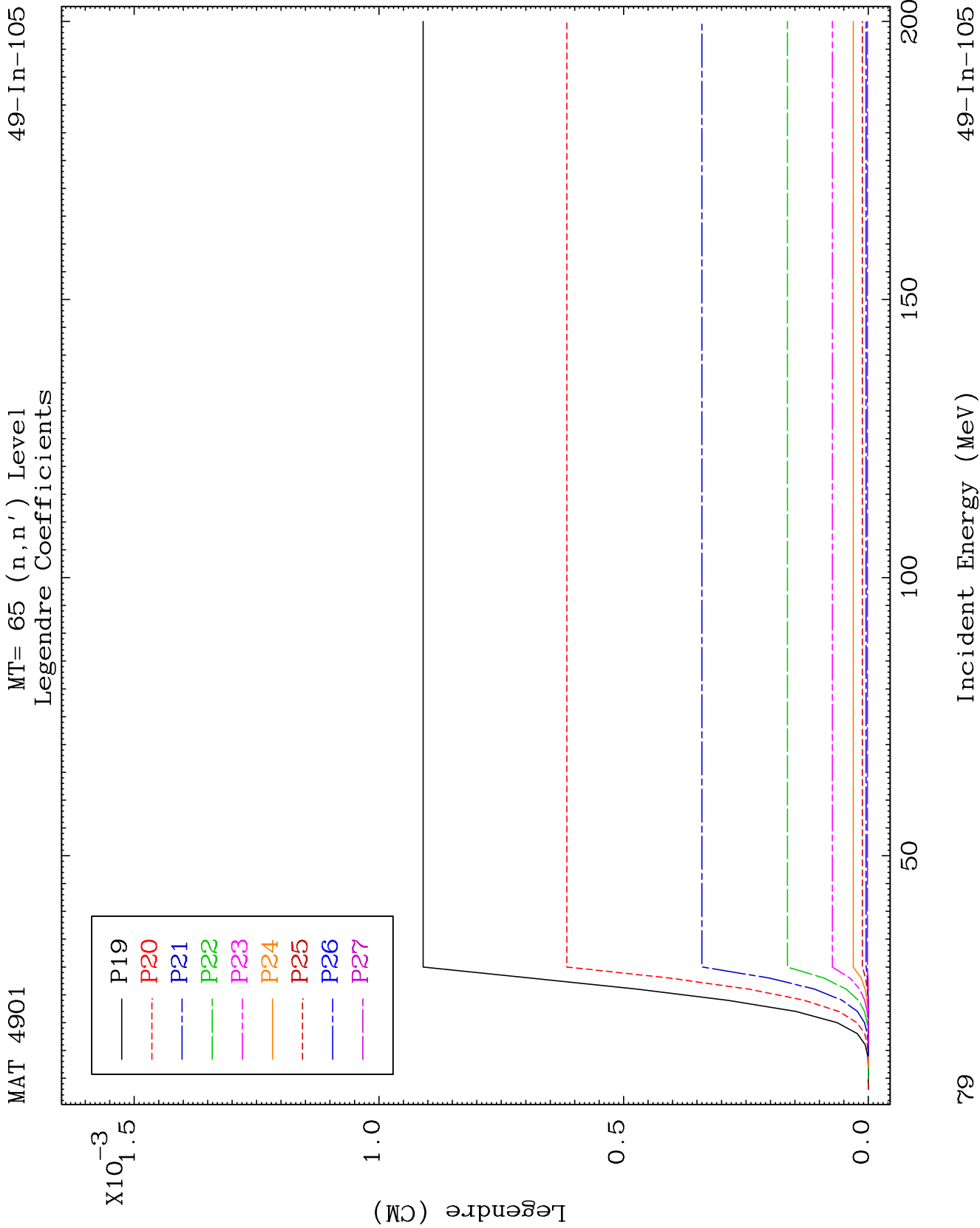


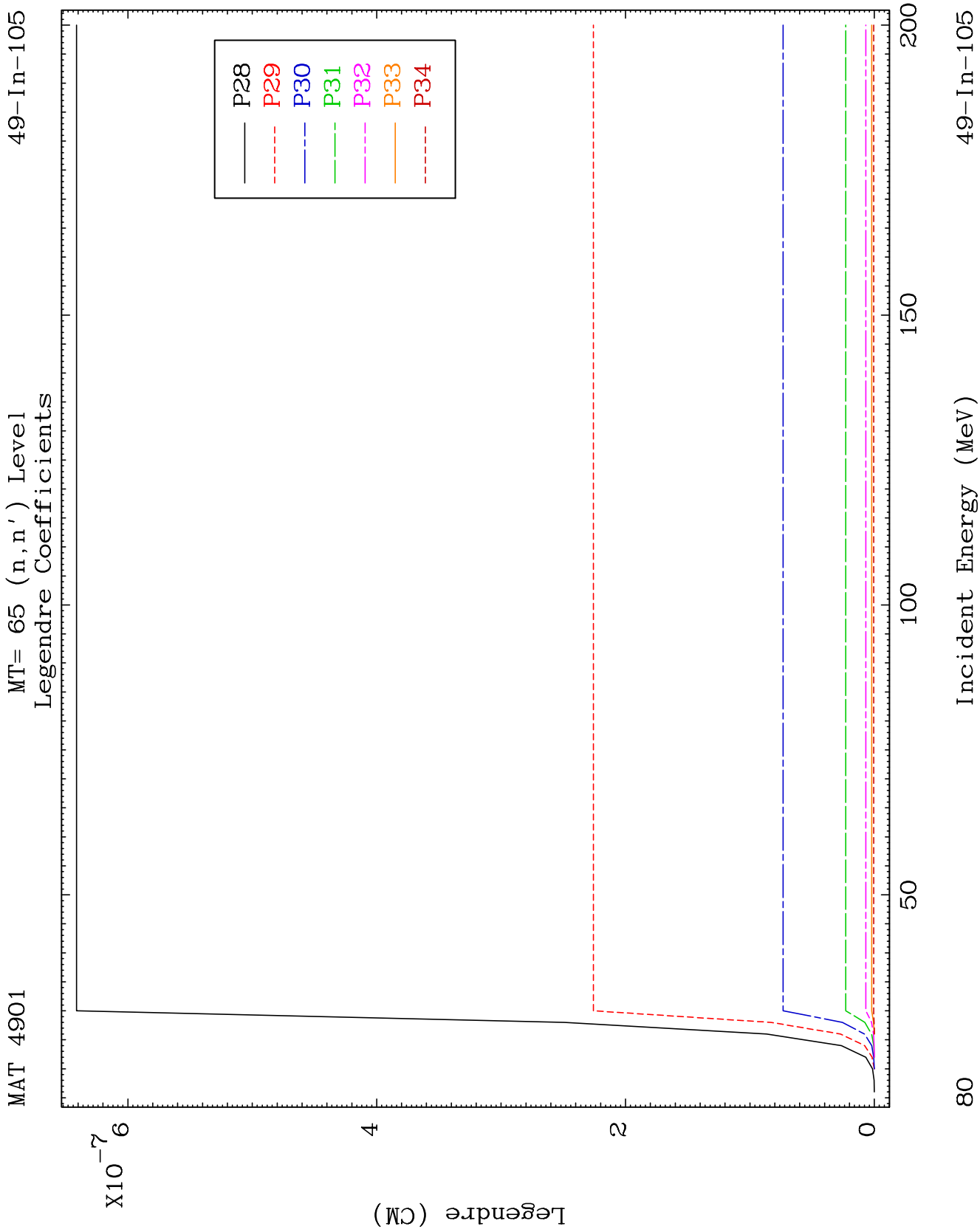


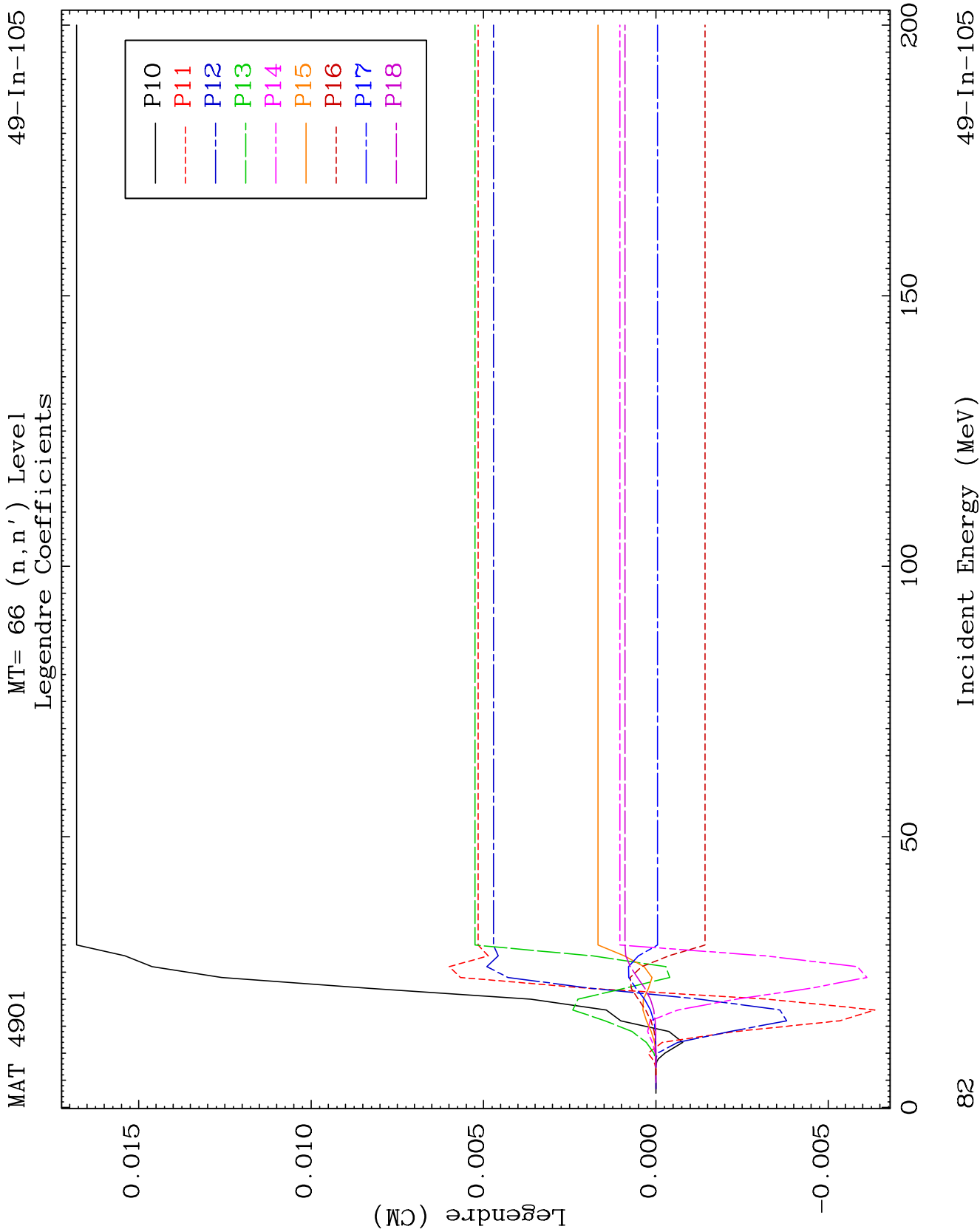








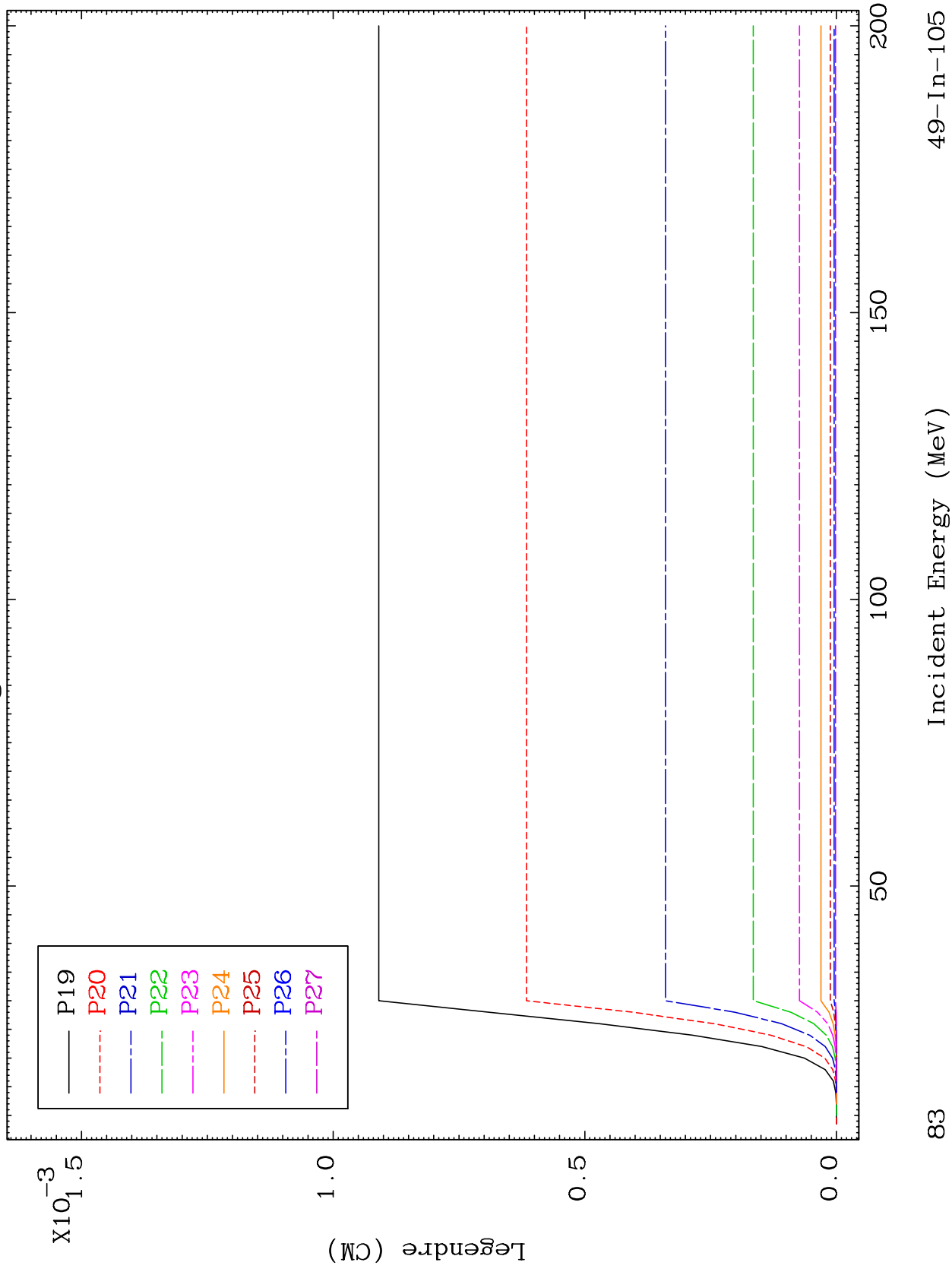


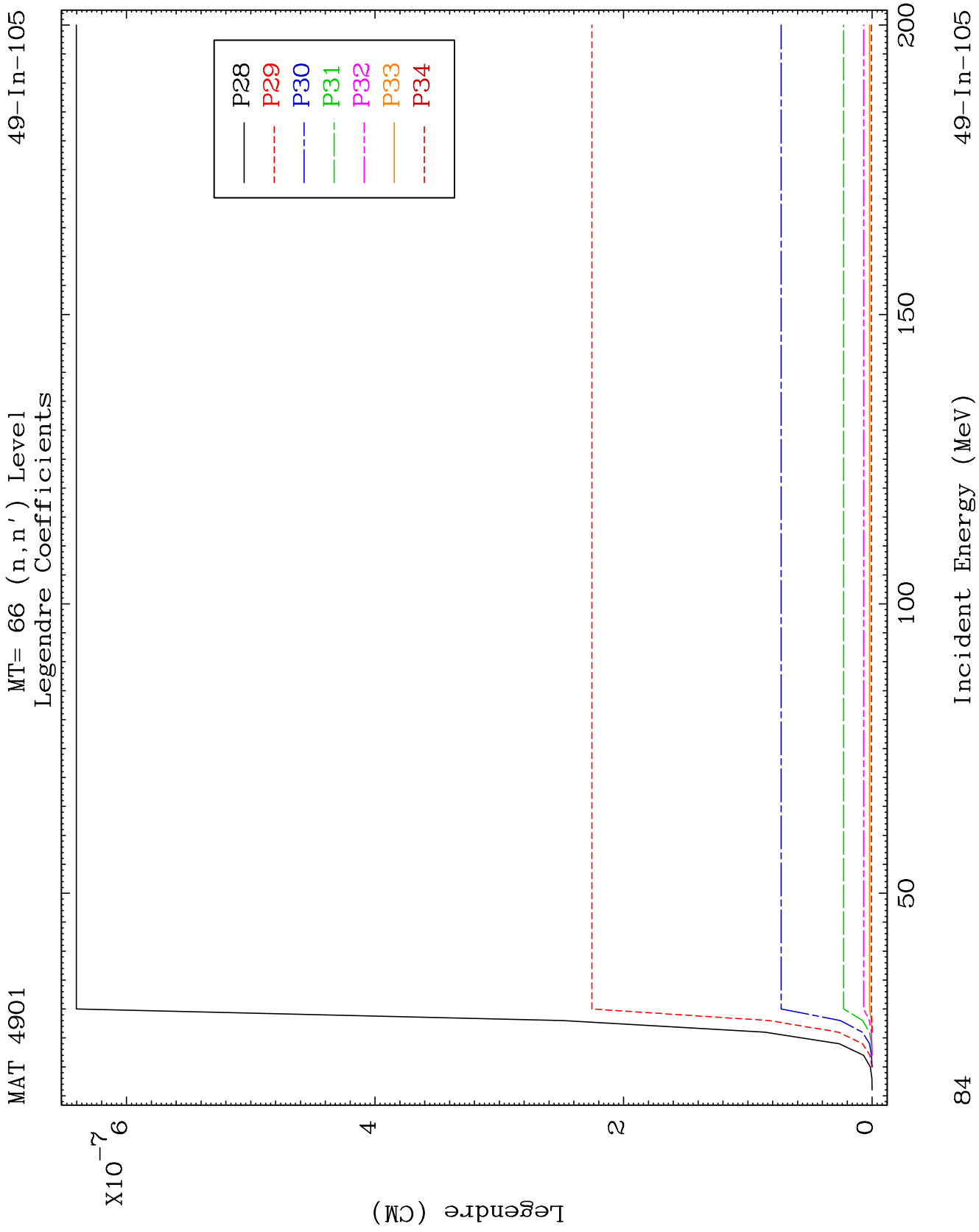


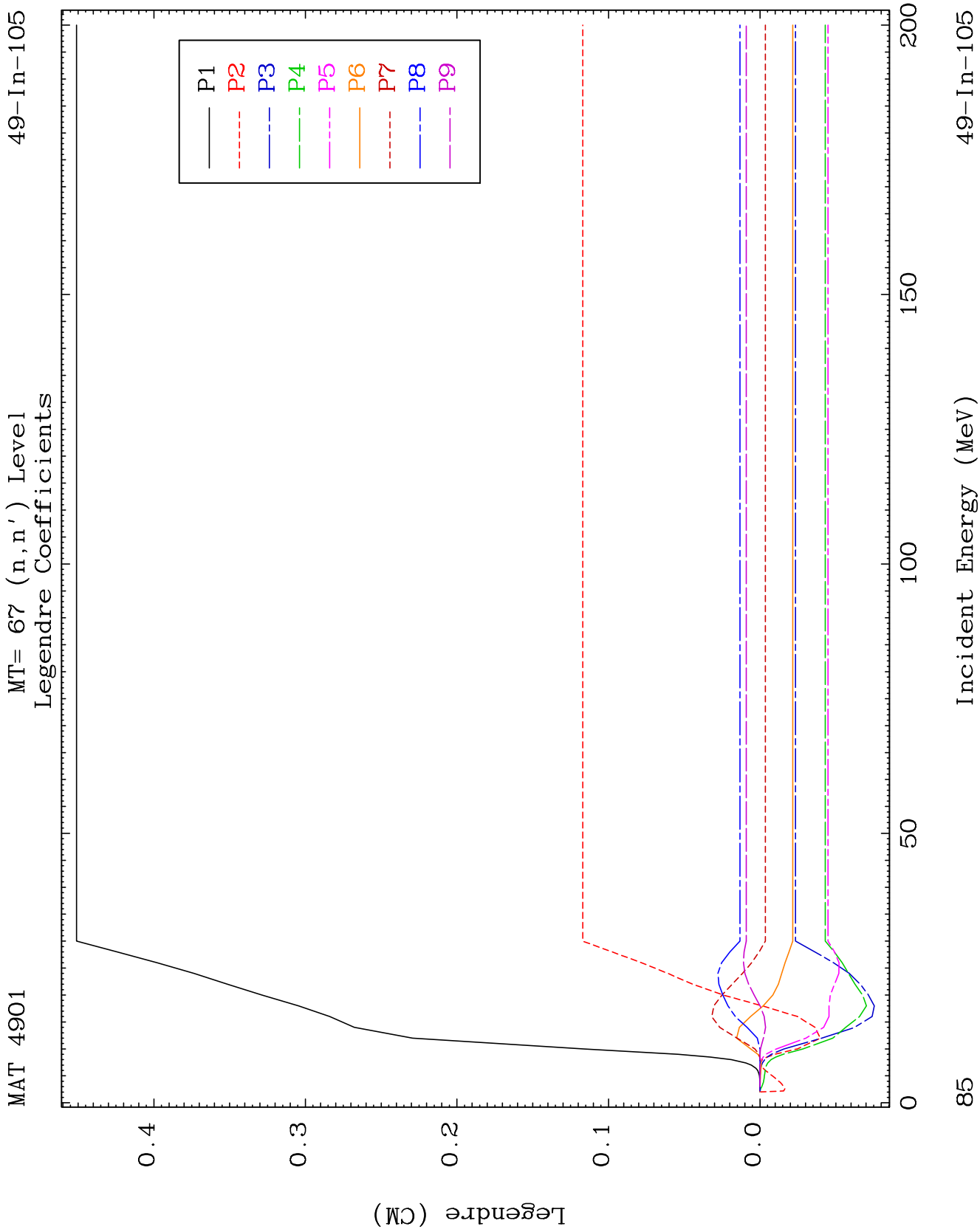
MAT 4901

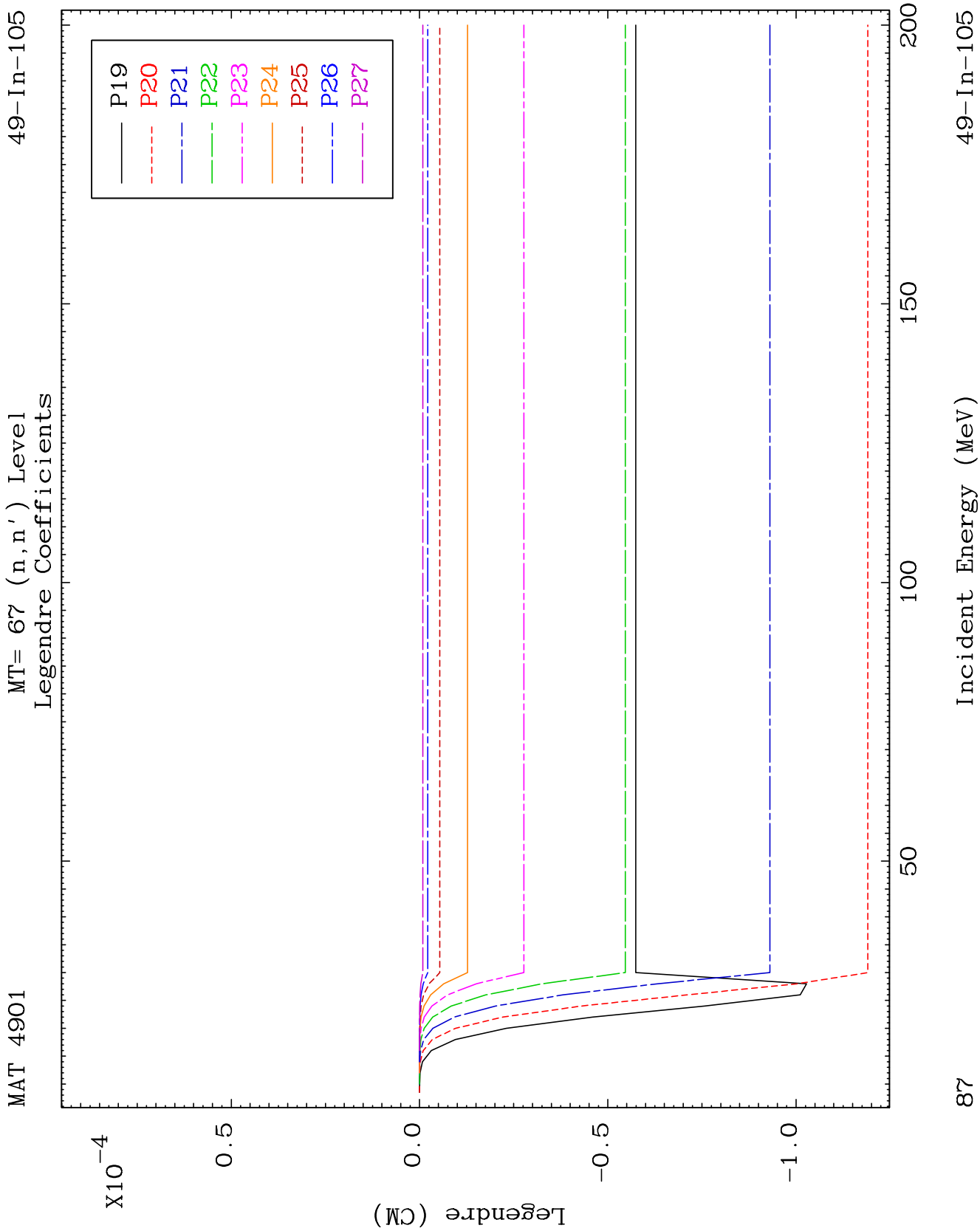
MT= 66 (n, n') Level
Legendre Coefficients

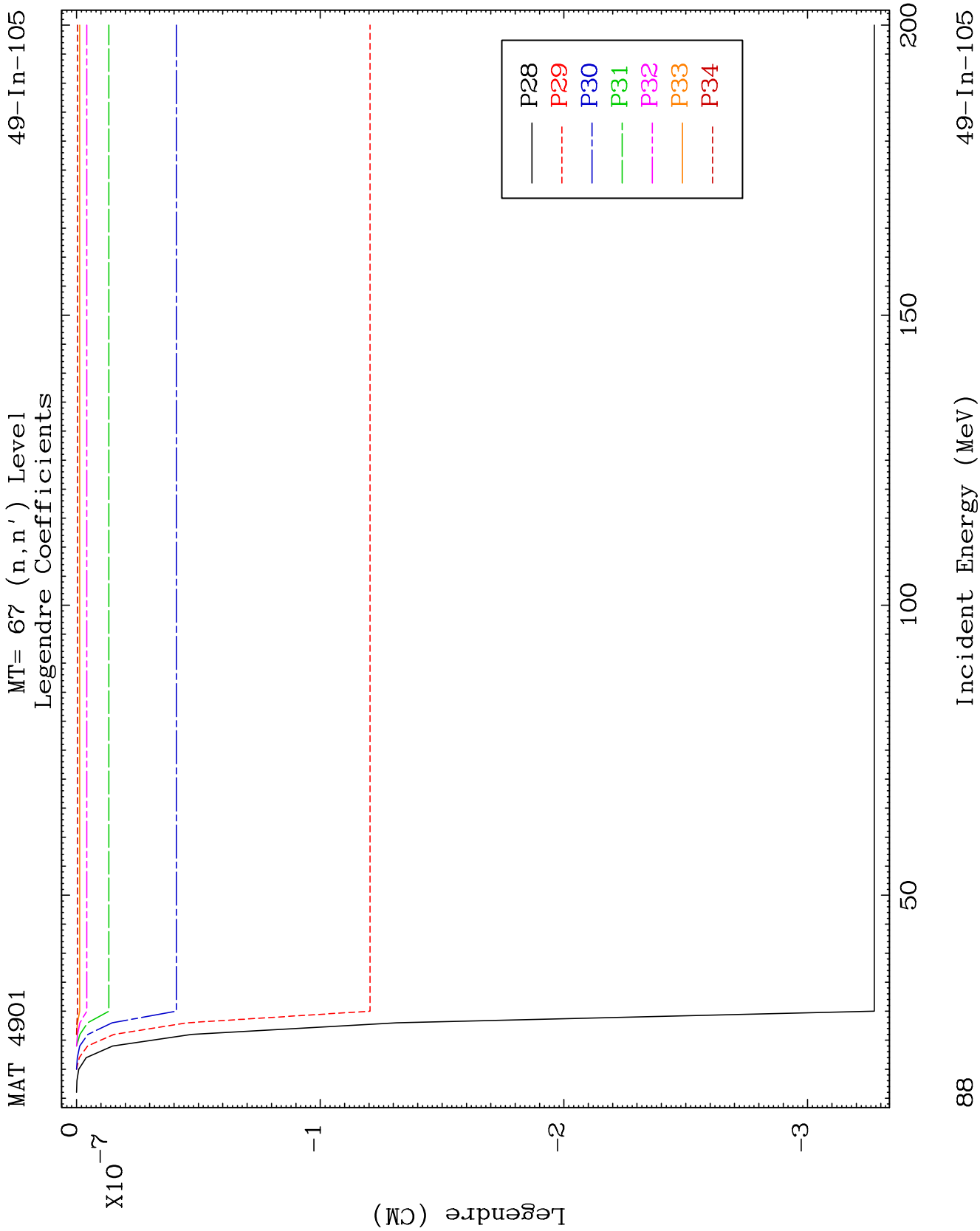
49-In-105







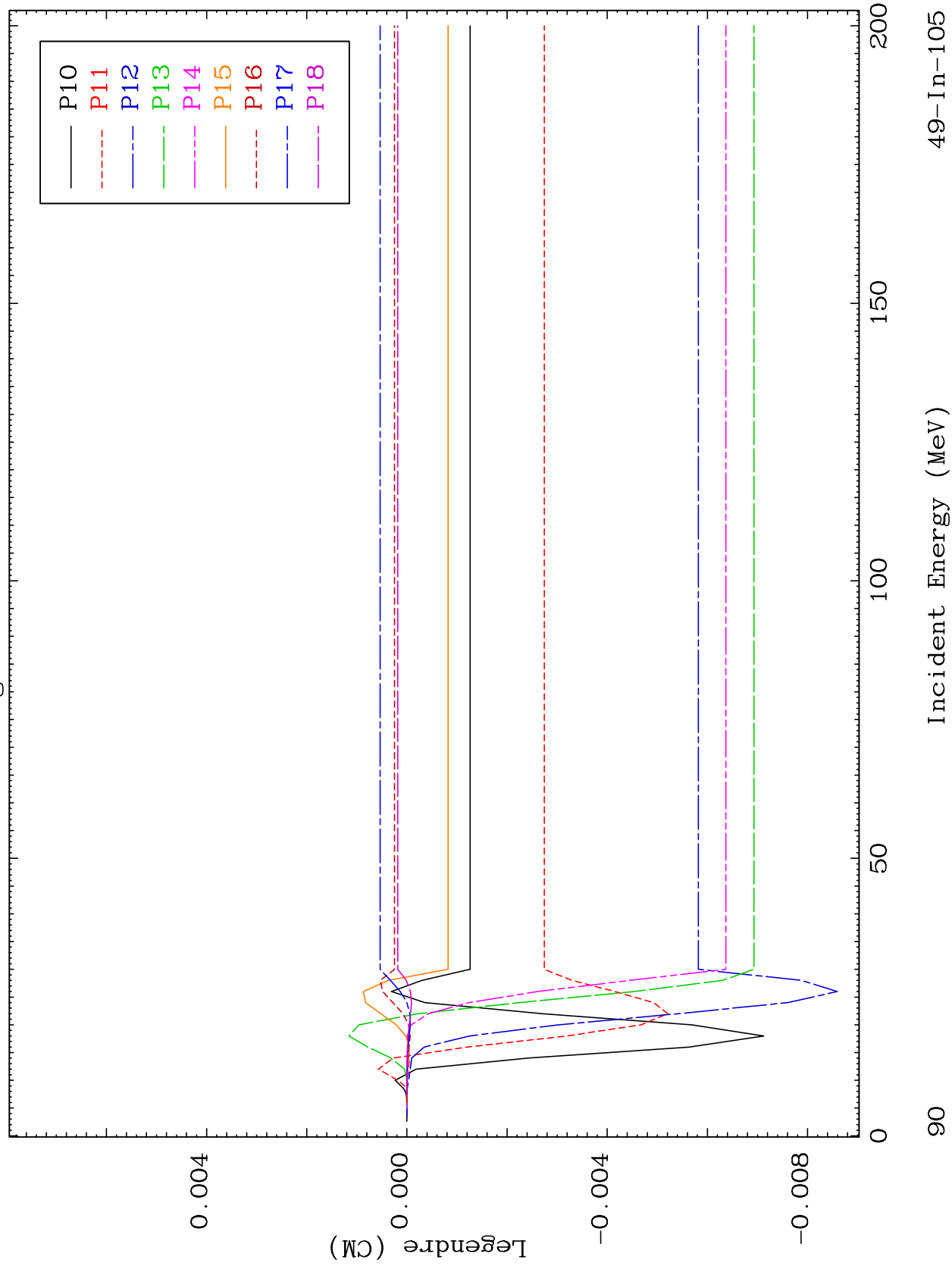


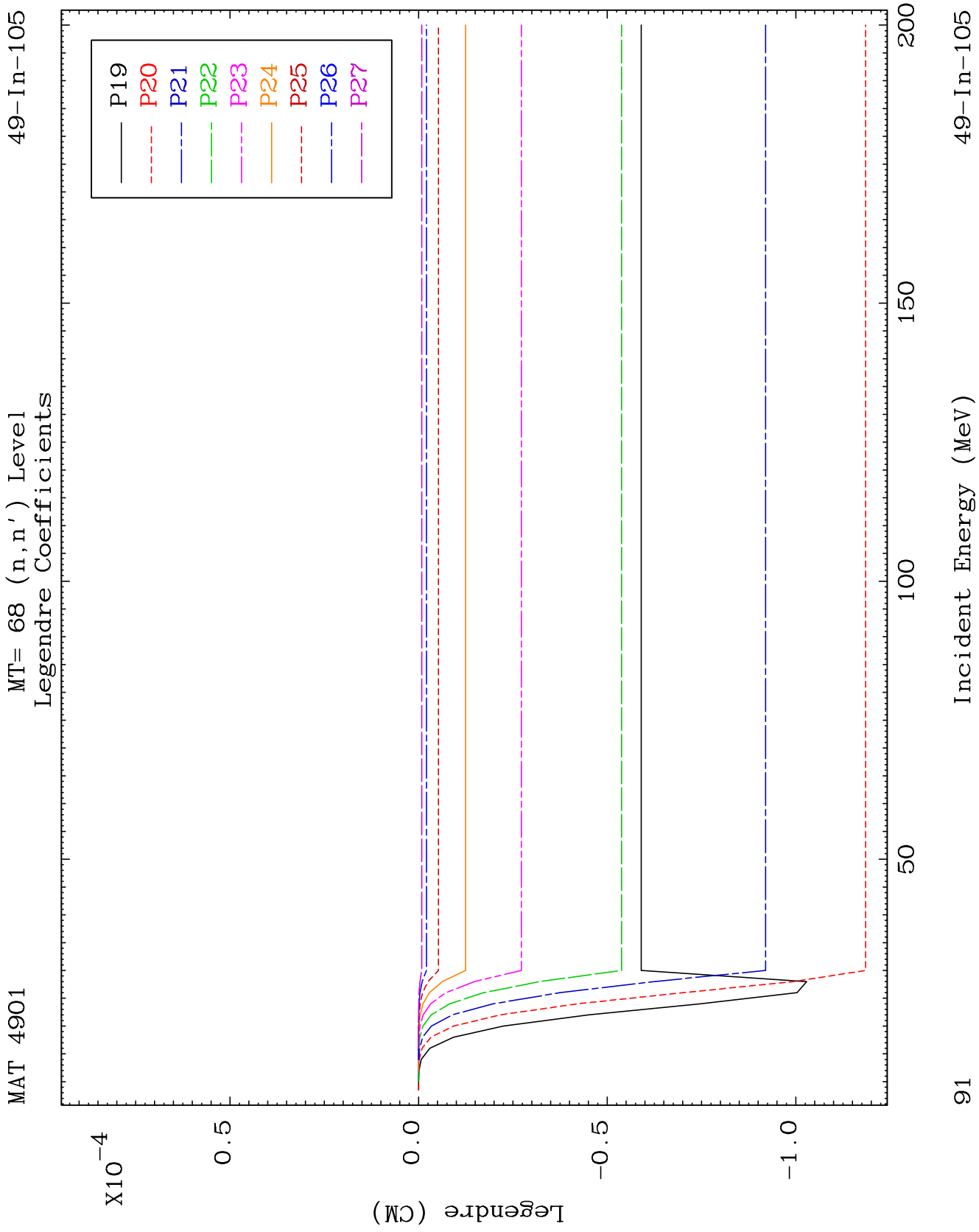


MAT 4901

MT= 68 (n, n') Level
Legendre Coefficients

49-In-105



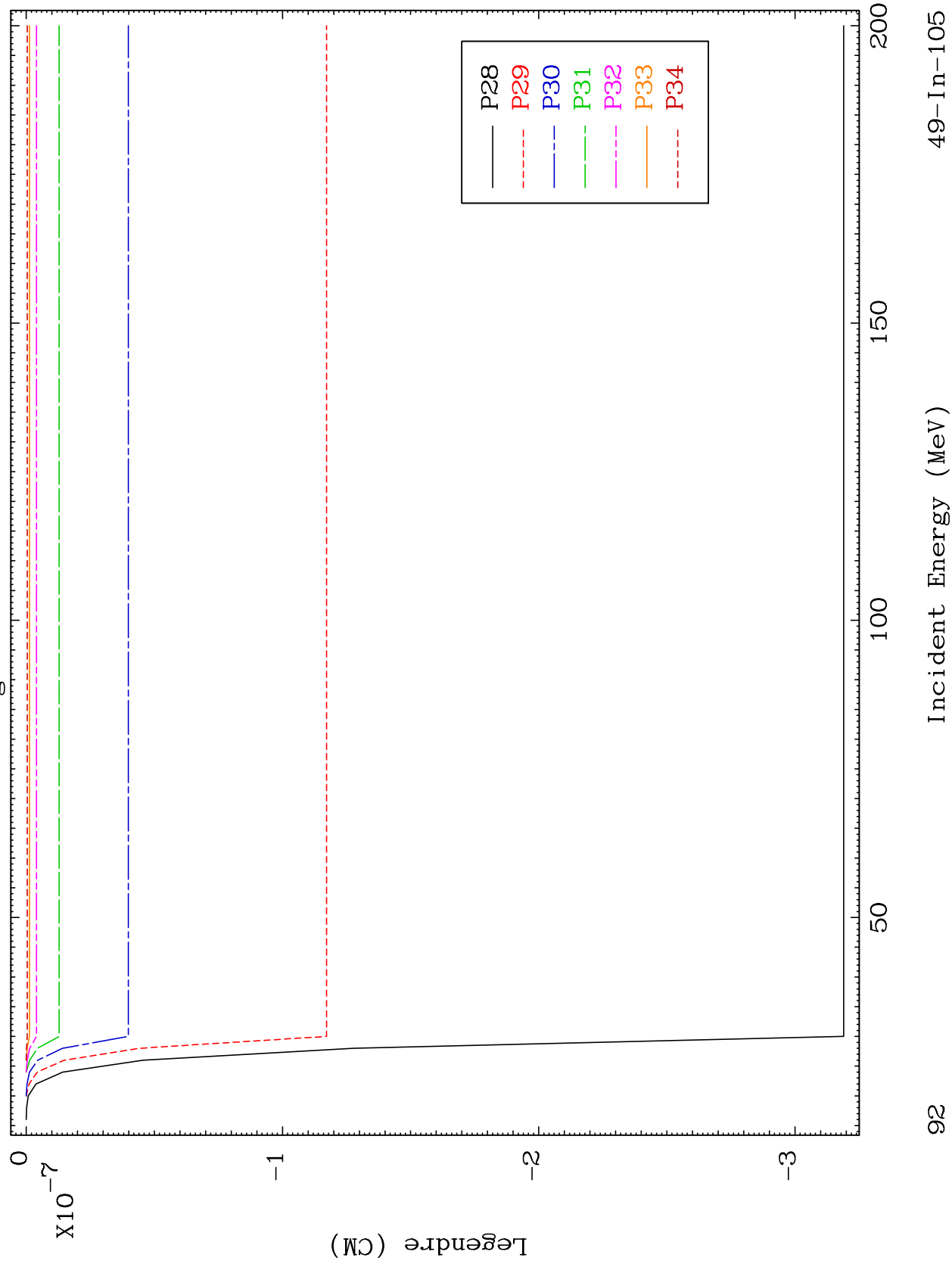


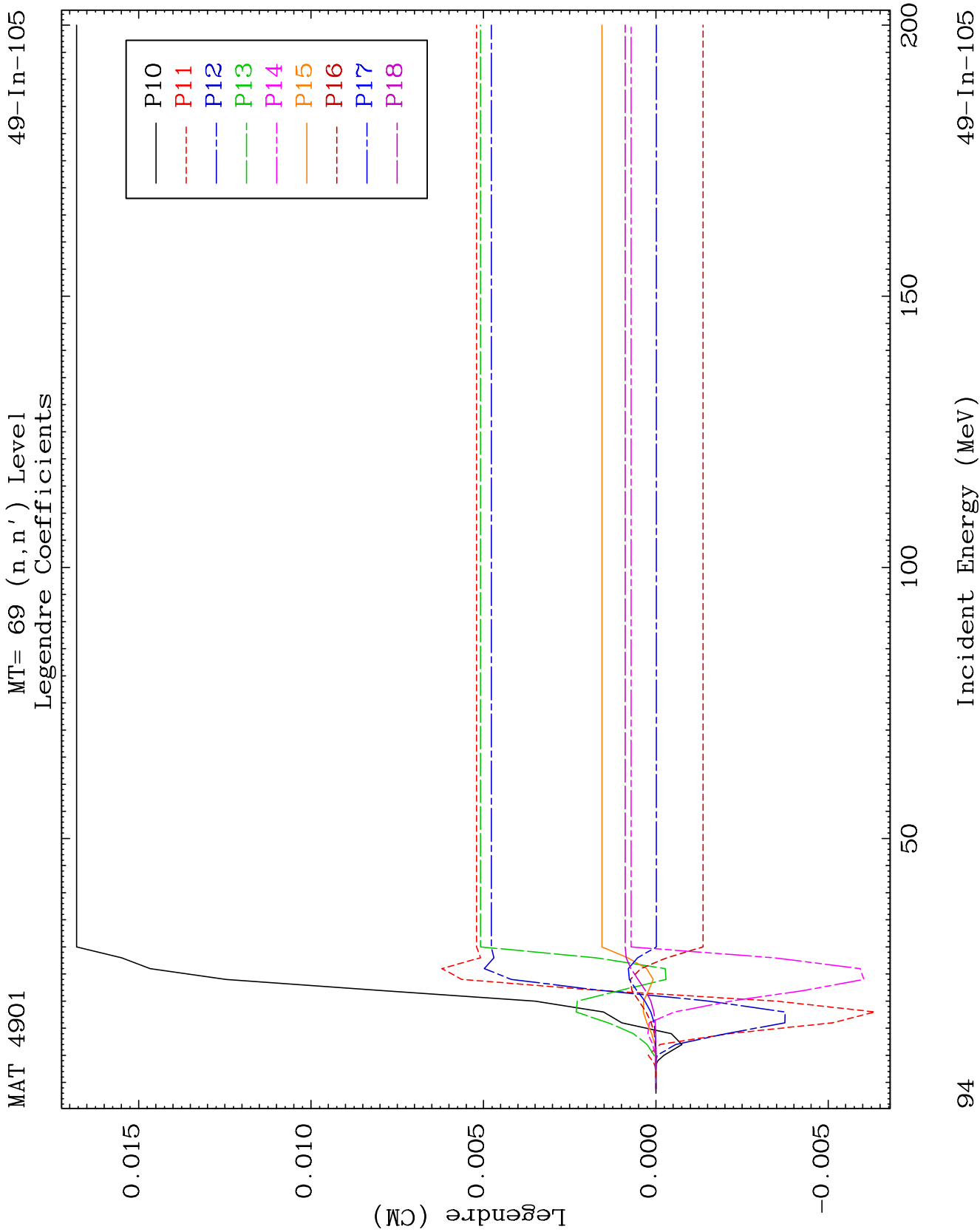
MAT 4901

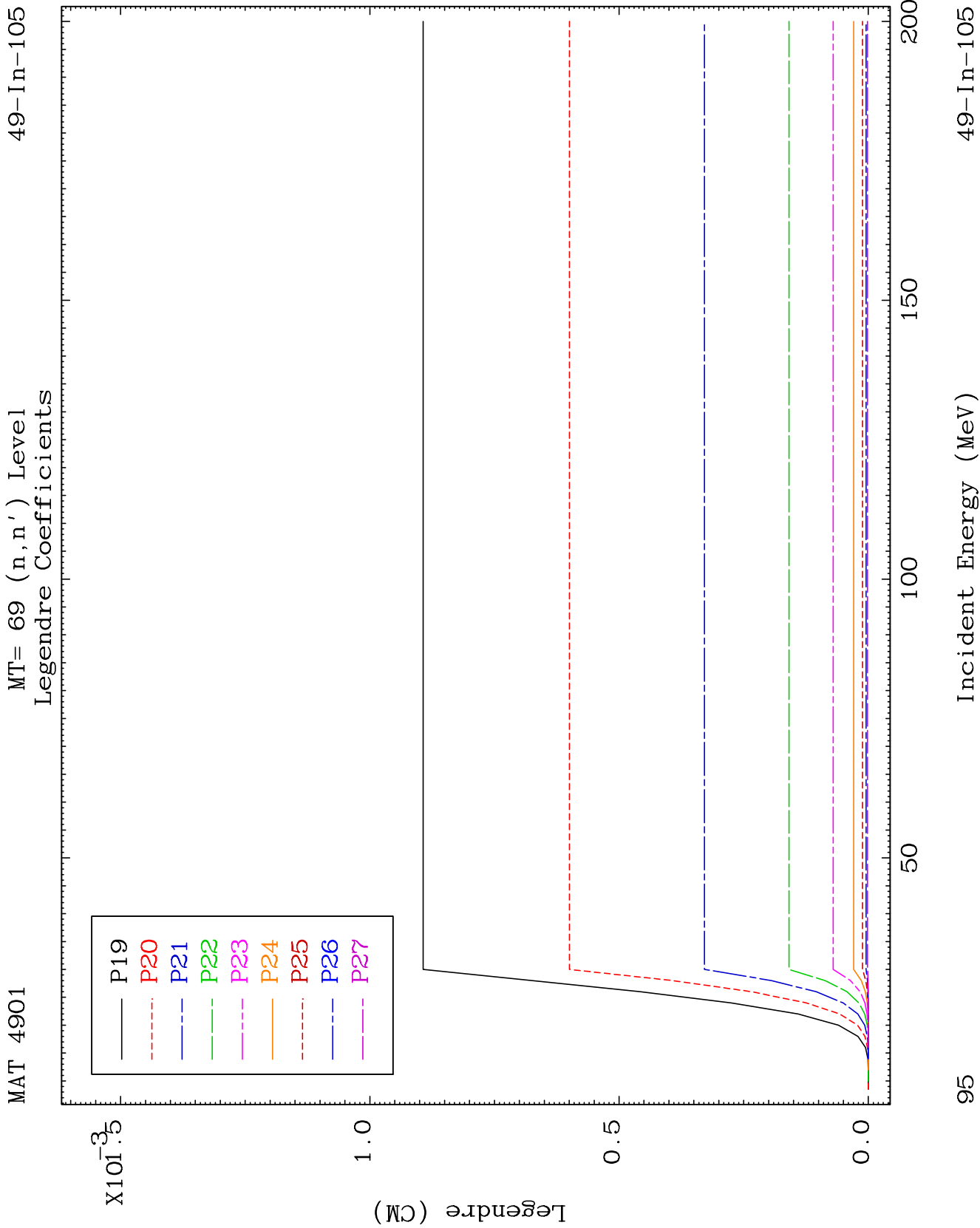
MT= 68 (n, n') Level

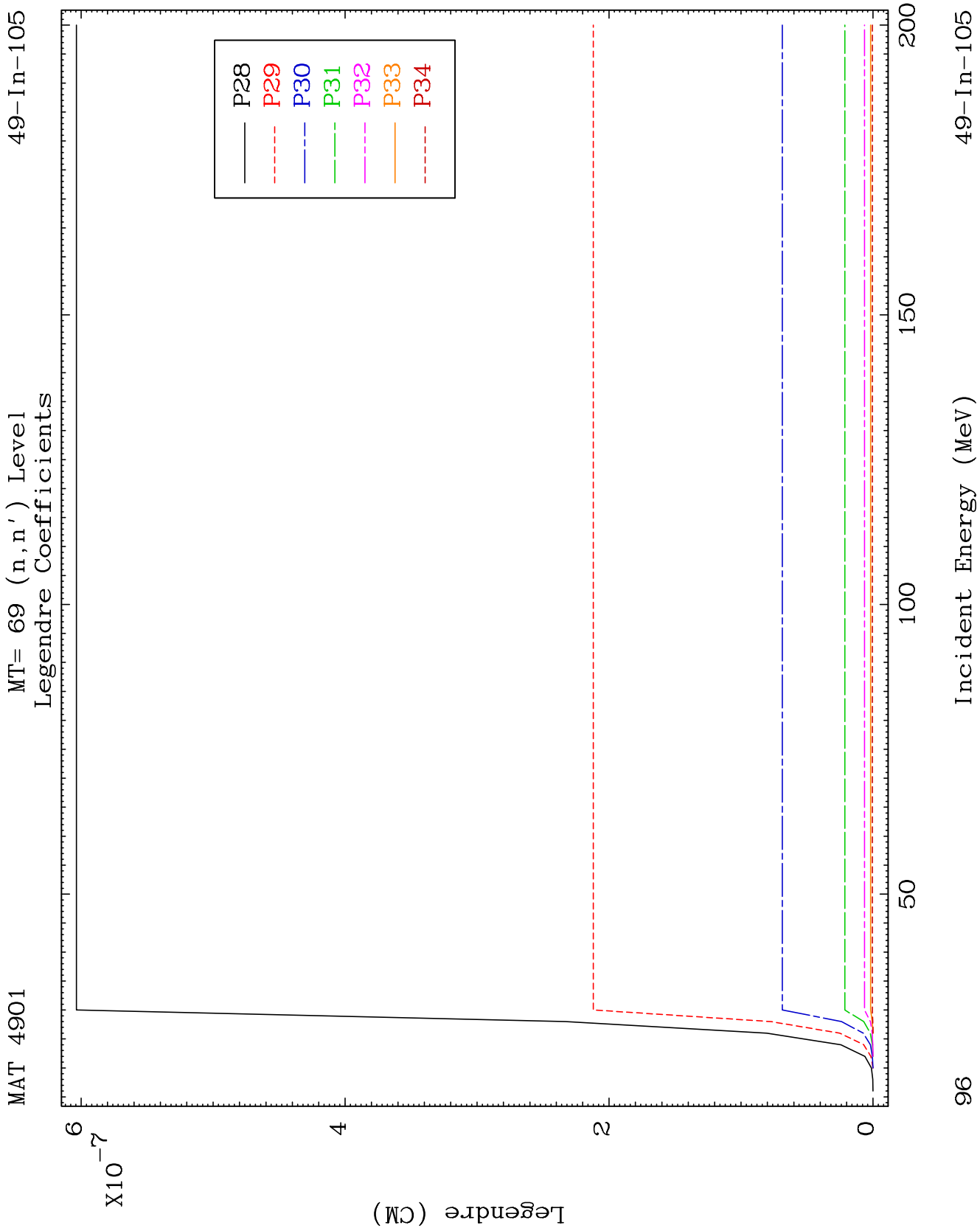
49-In-105

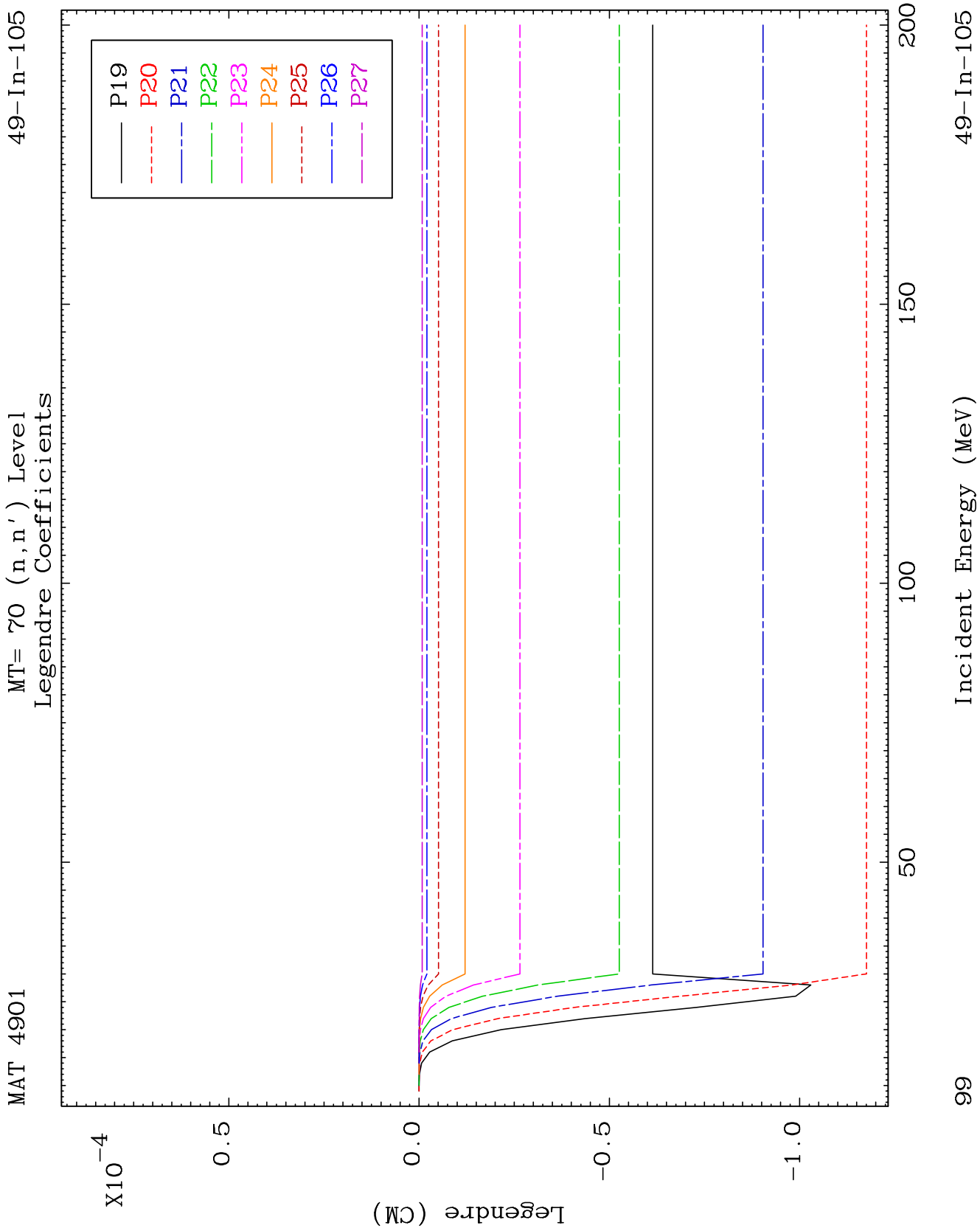
Legendre Coefficients

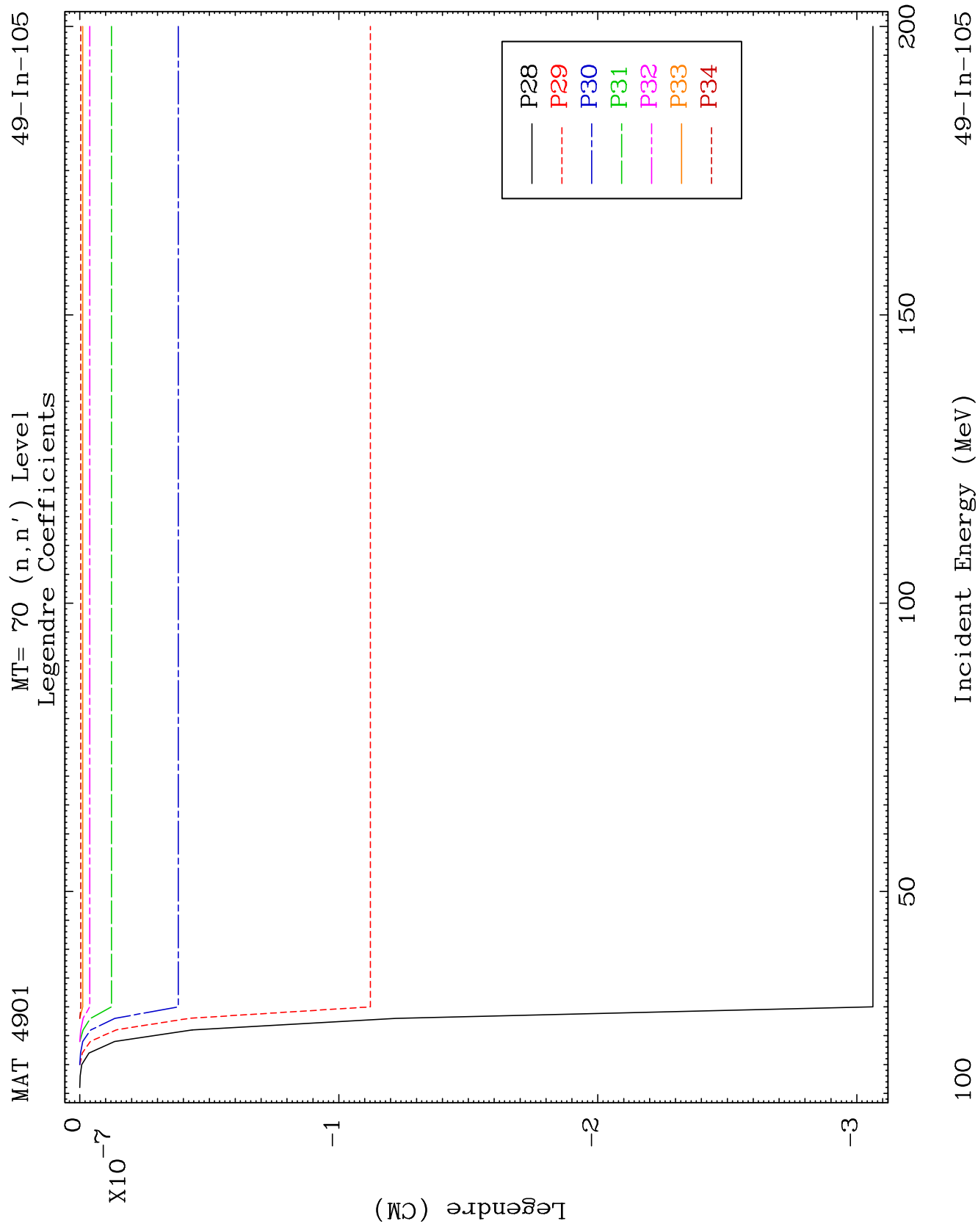


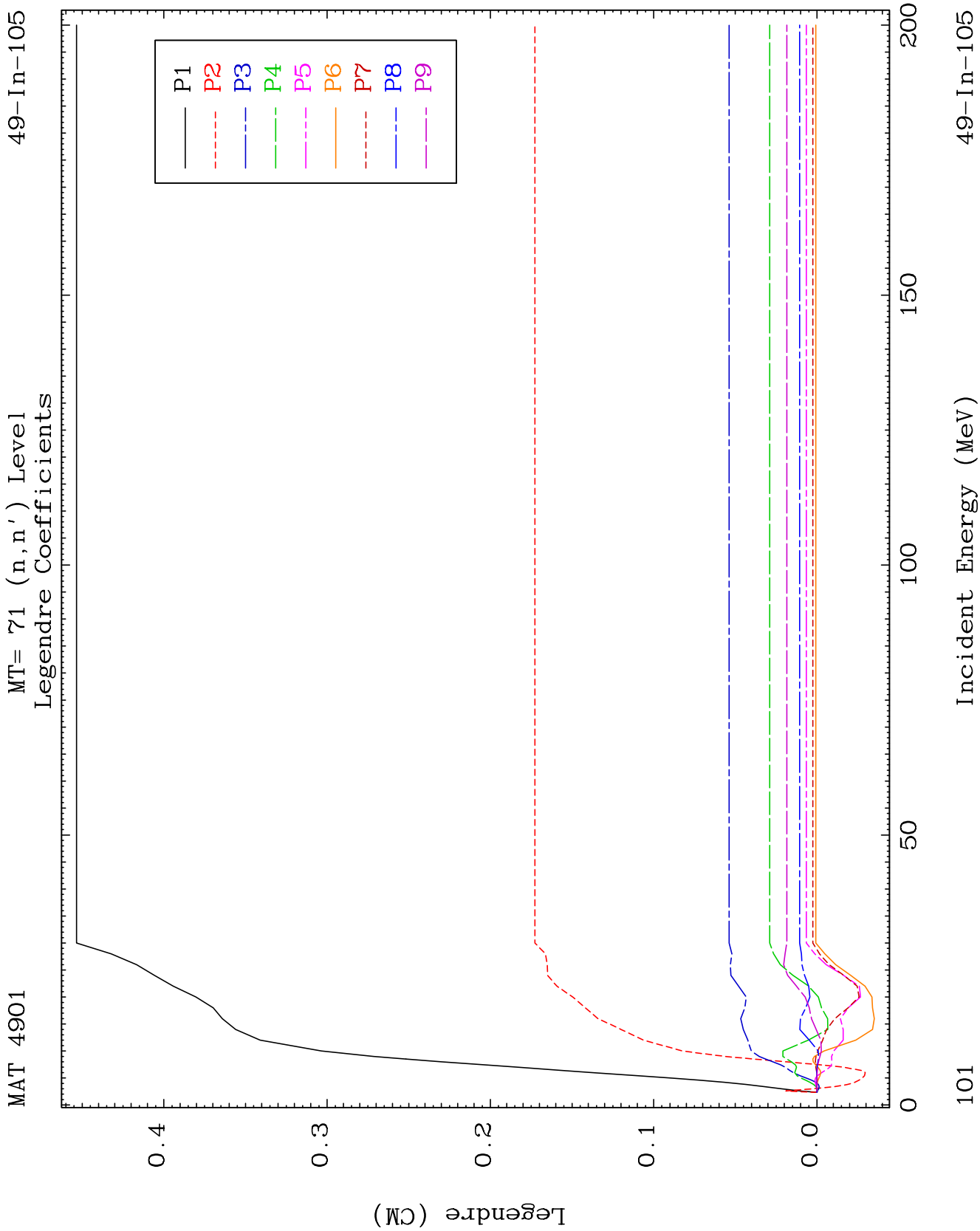


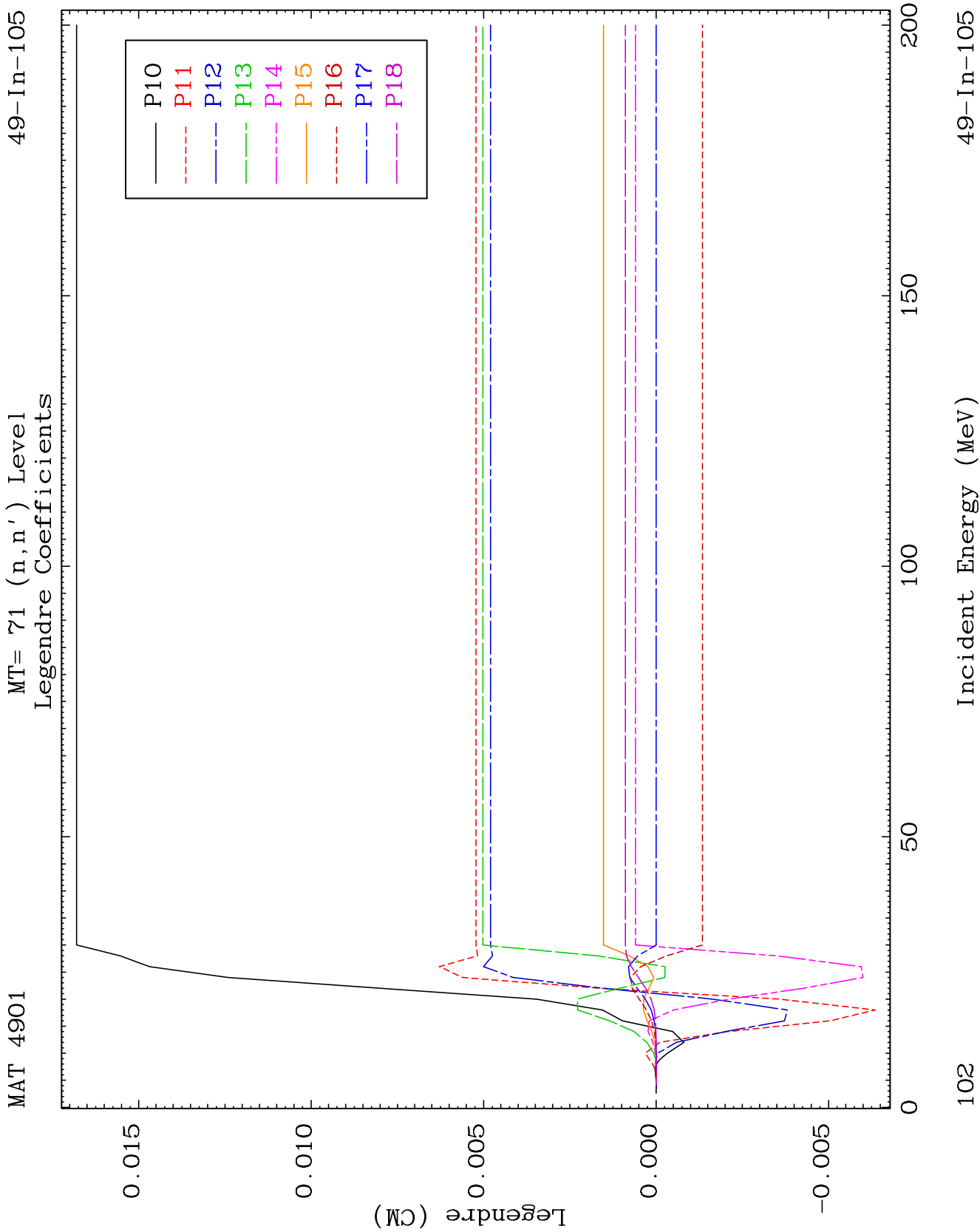


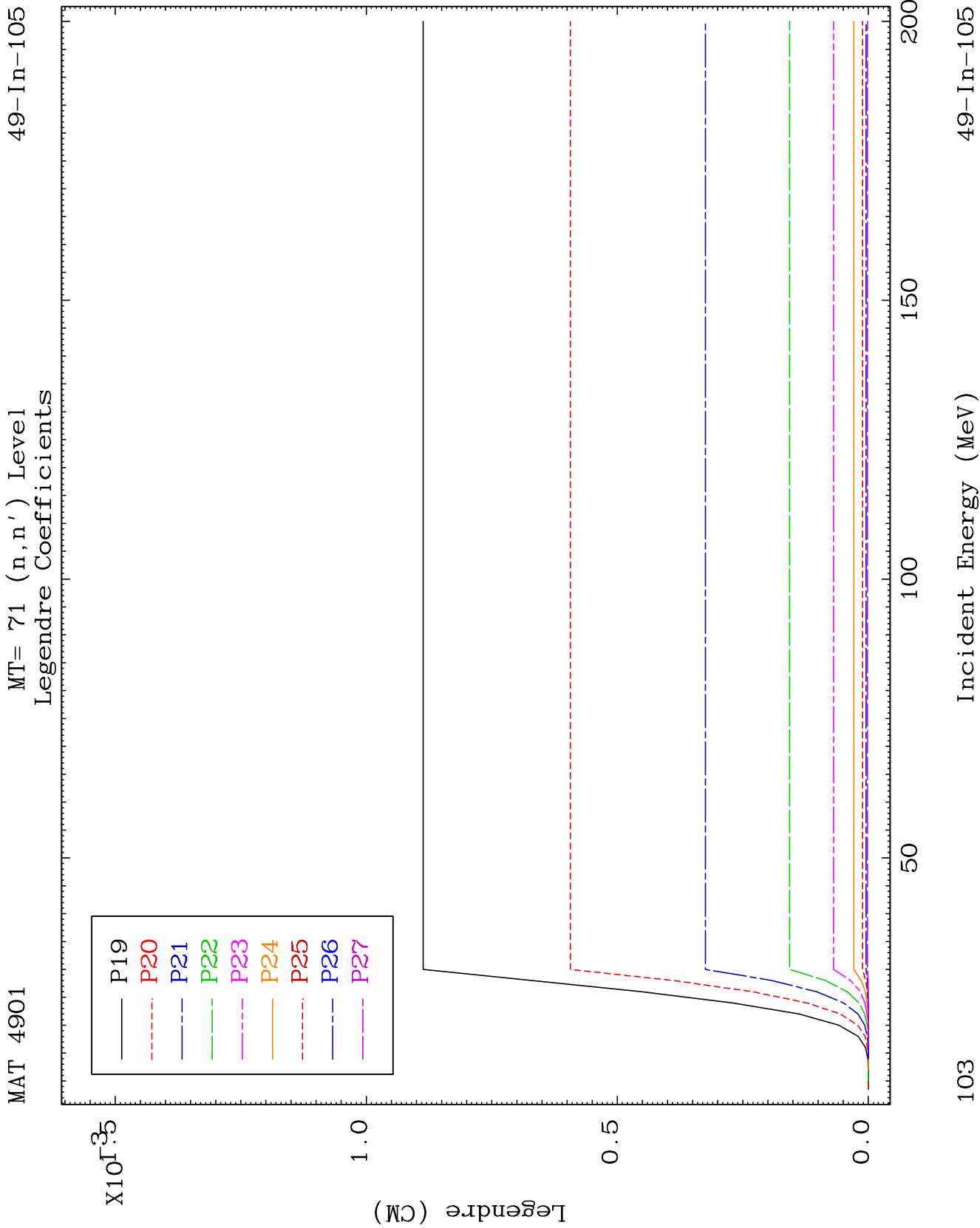


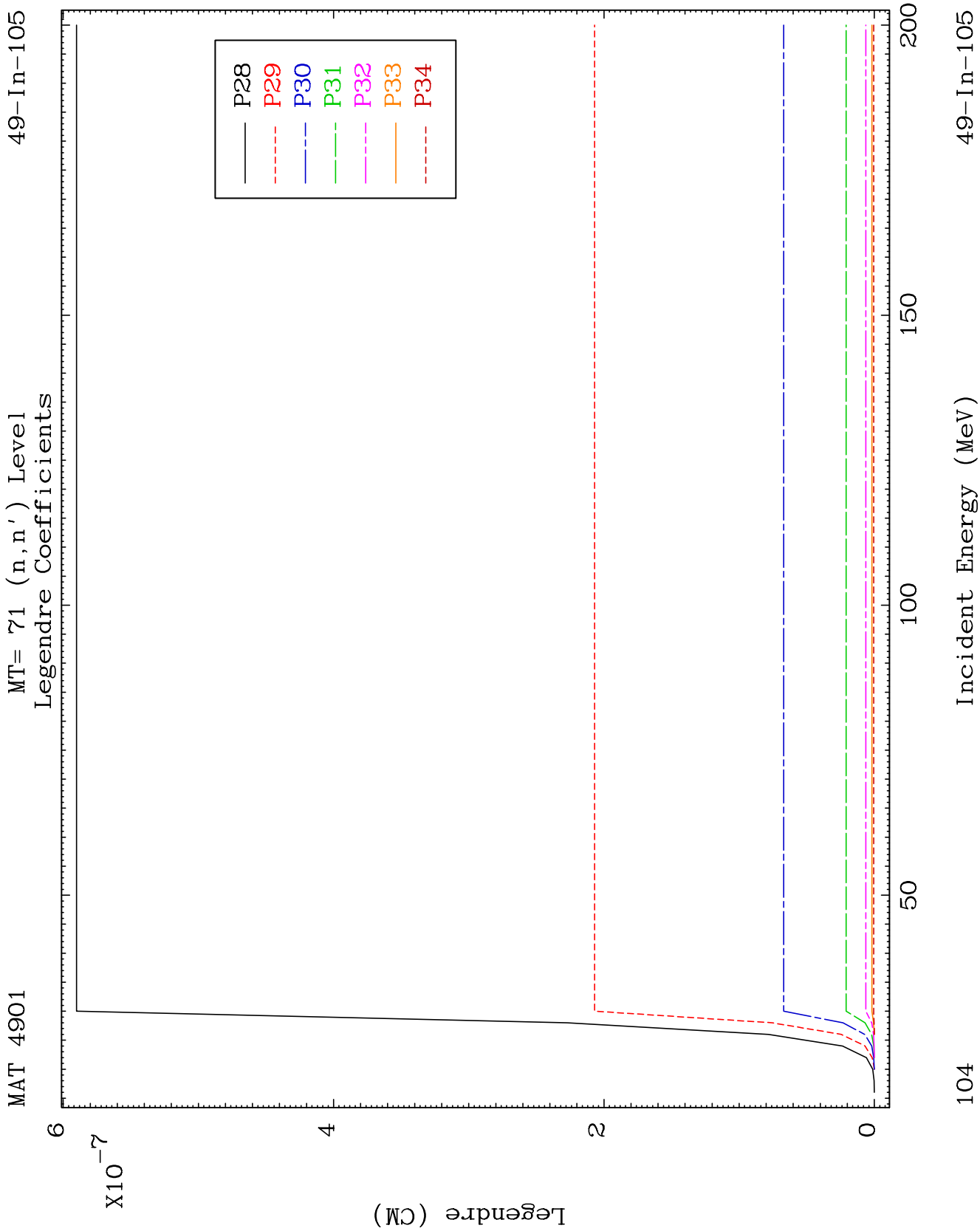








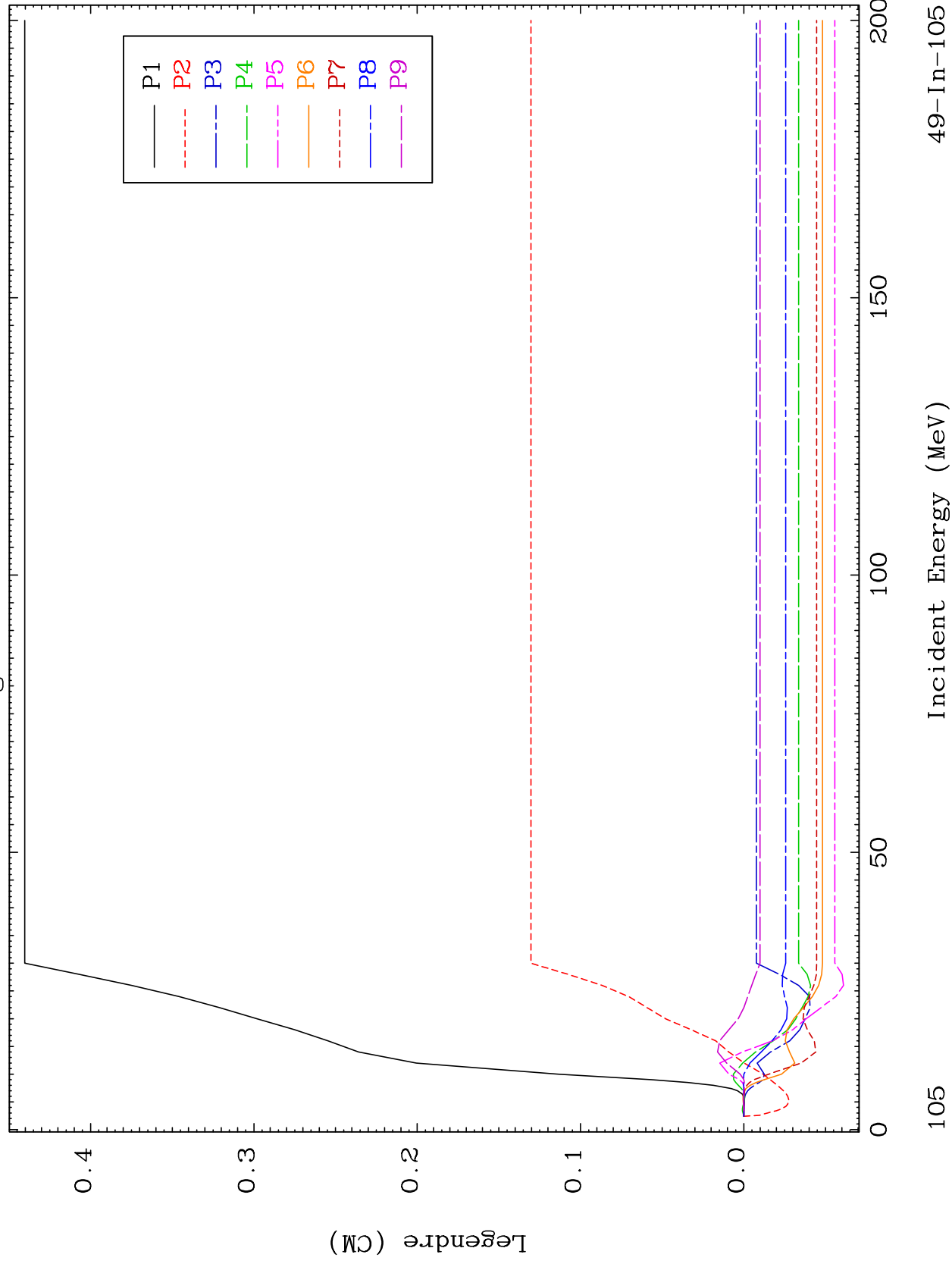


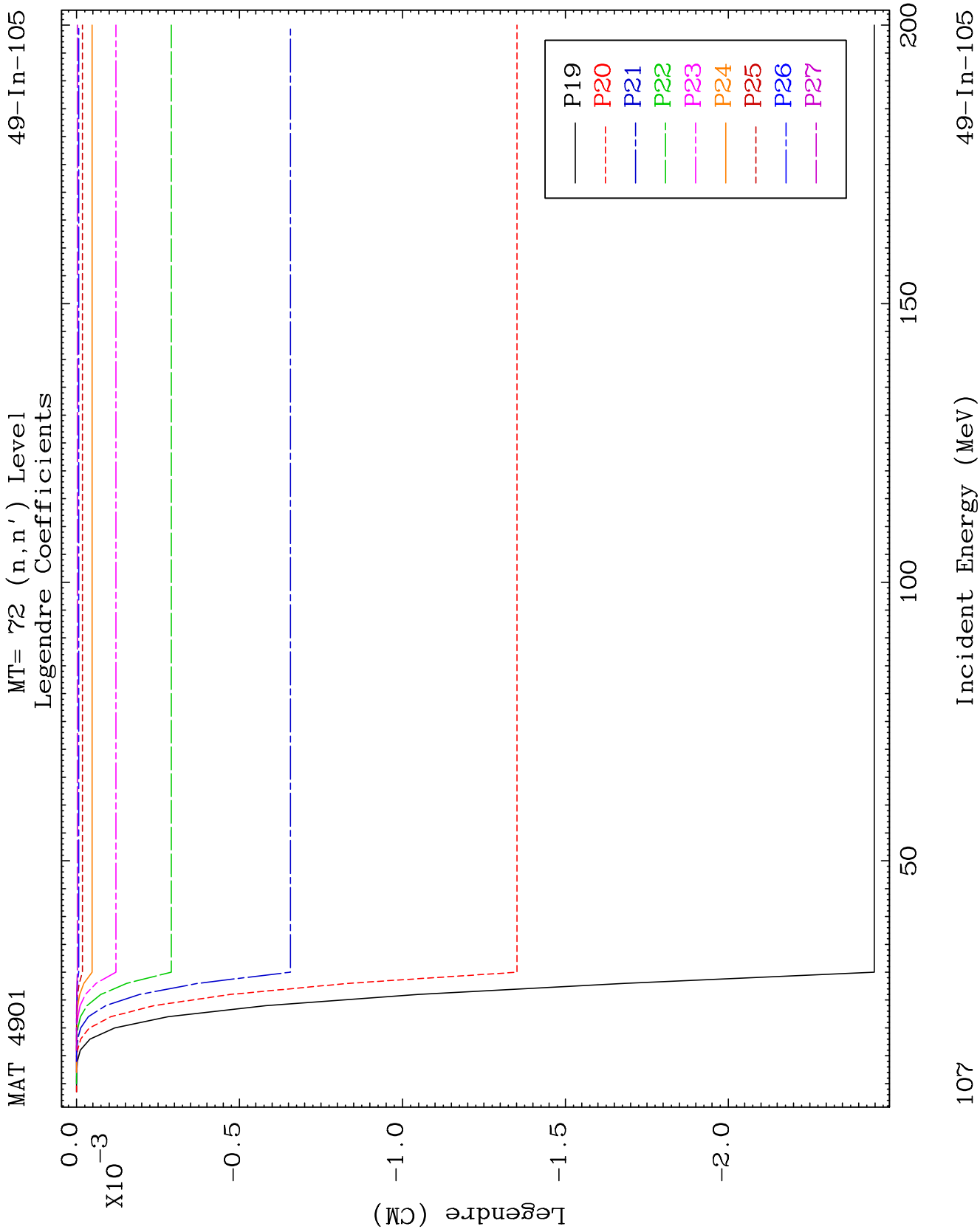


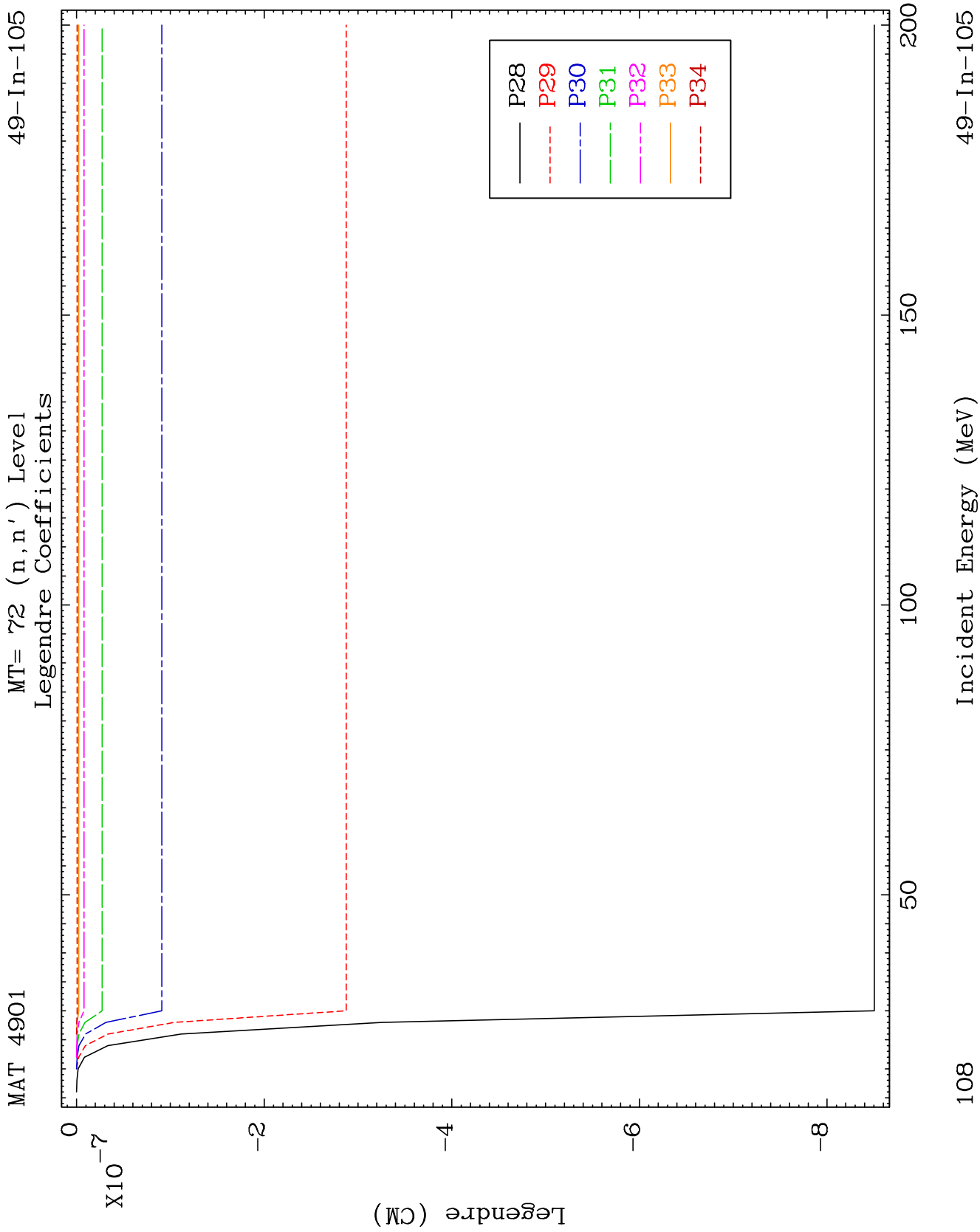
MAT 4901

MT= 72 (n, n') Level
Legendre Coefficients

49-In-105



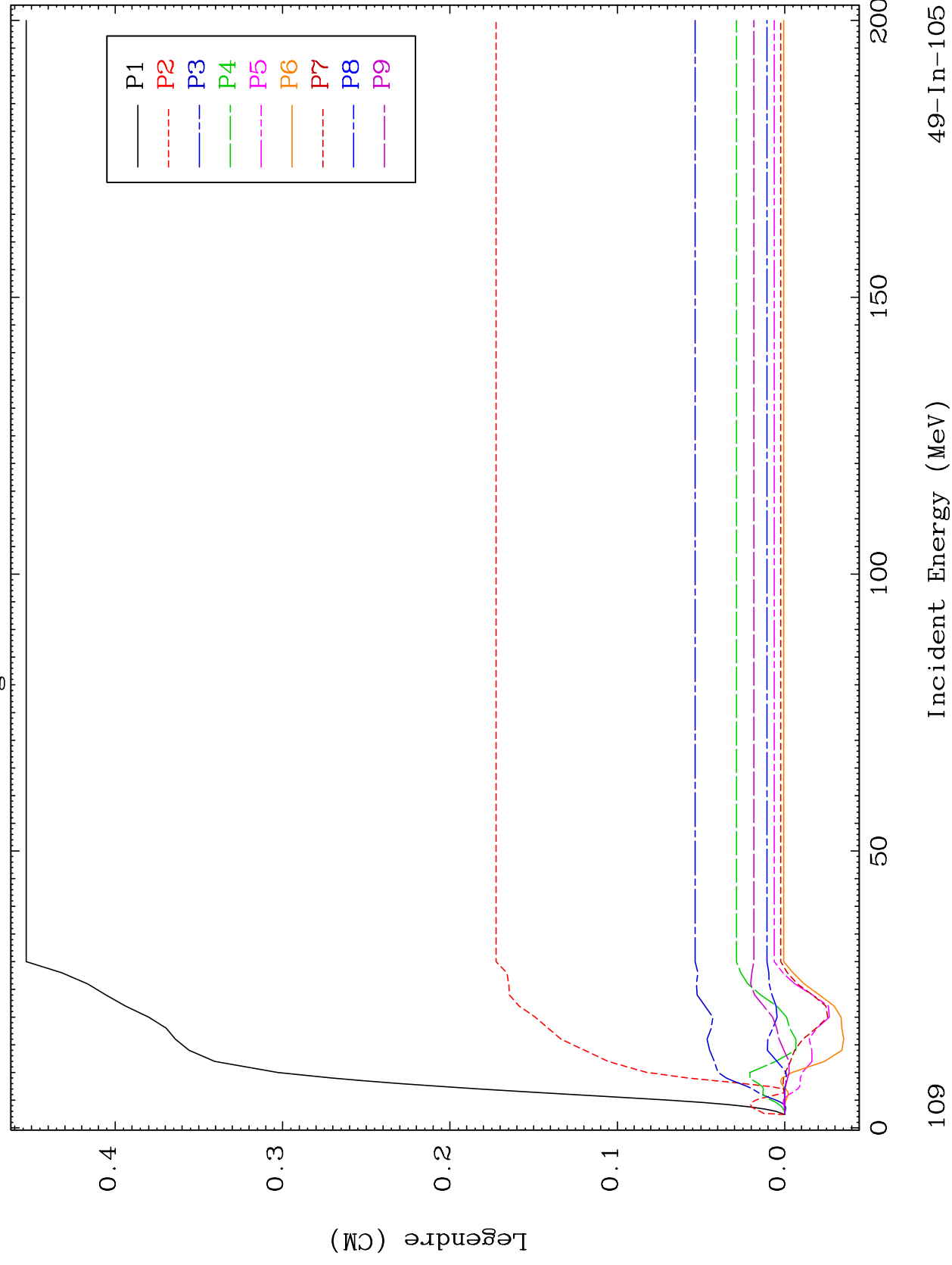


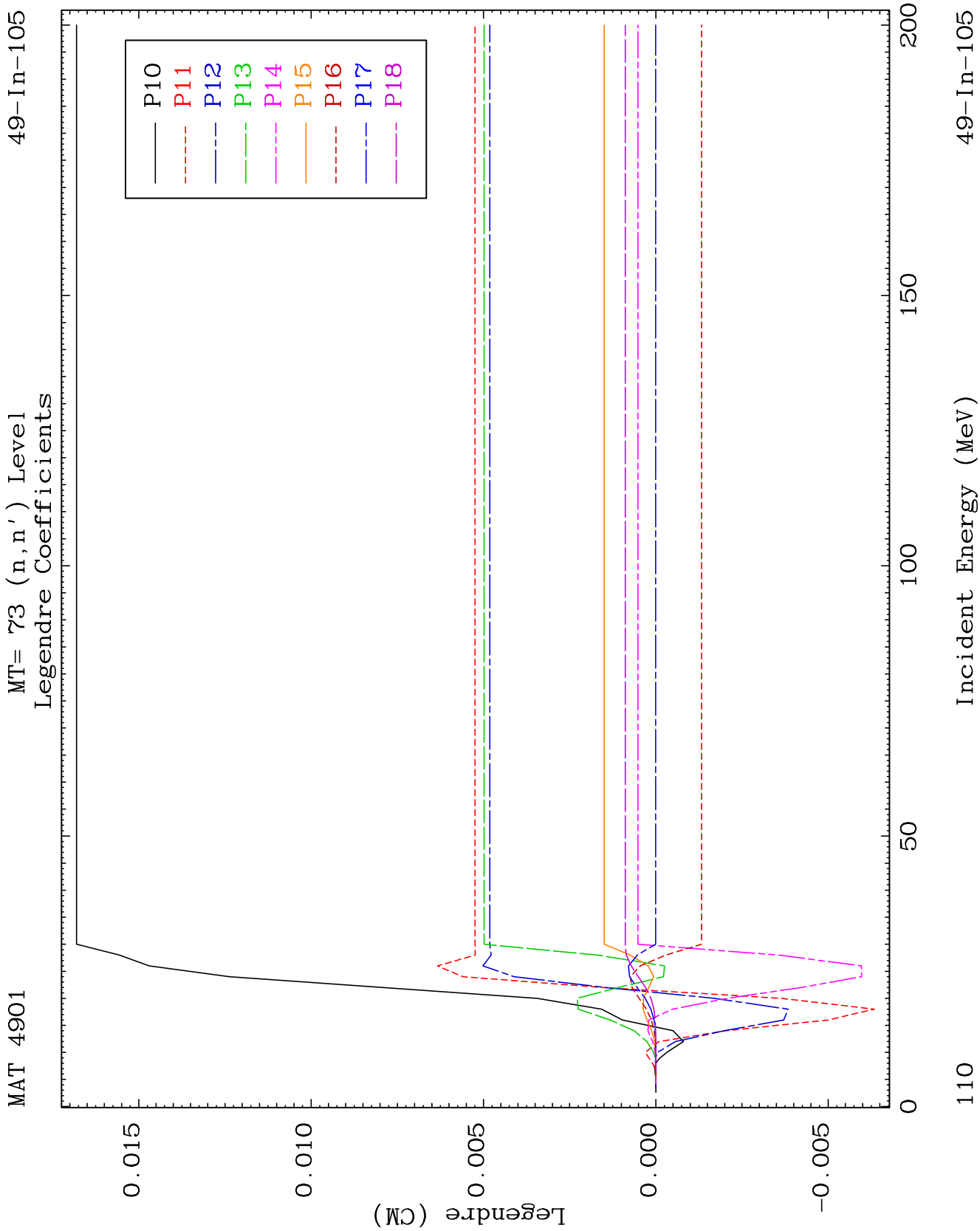


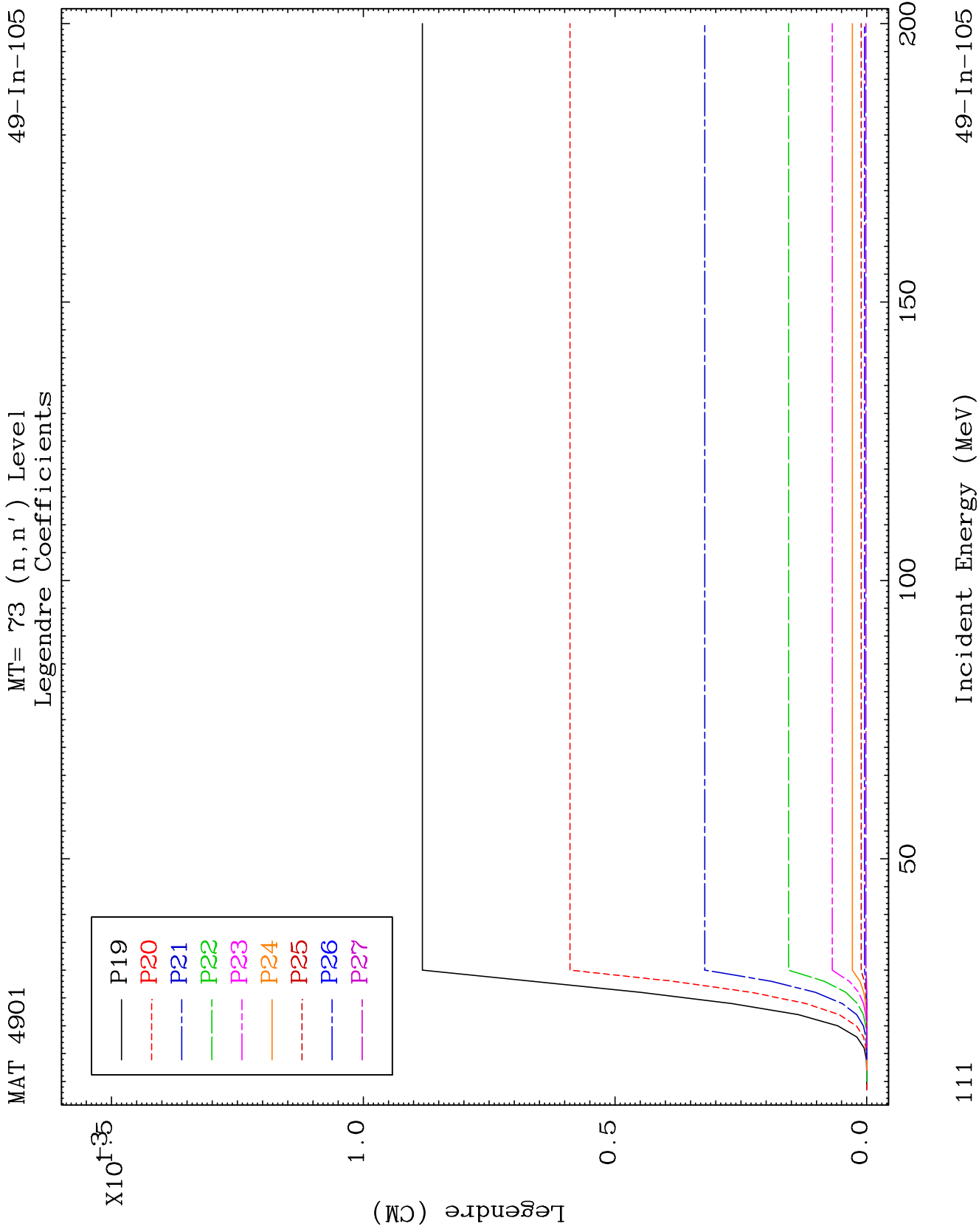
MAT 4901

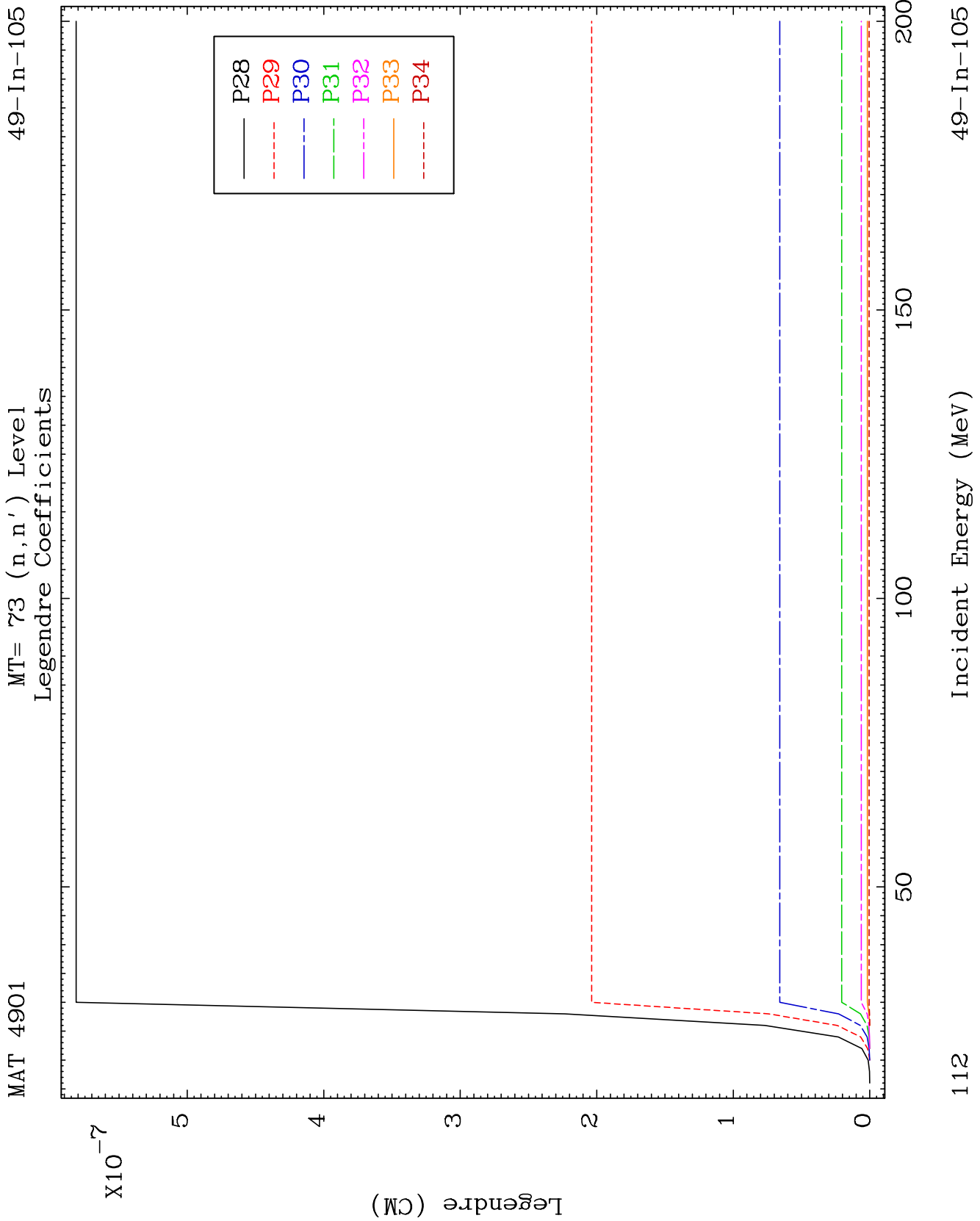
MT= 73 (n, n') Level
Legendre Coefficients

49-In-105





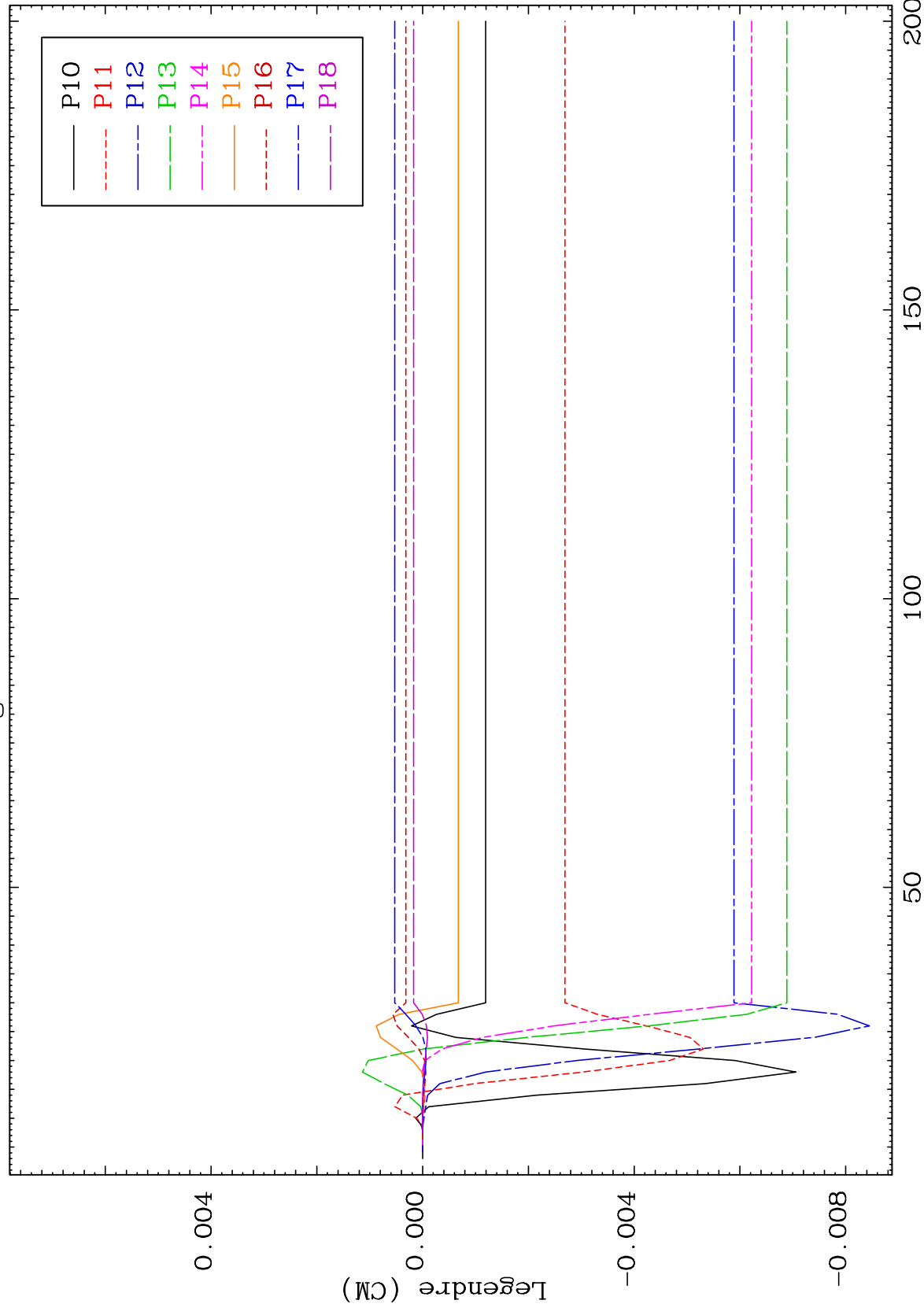




MAT 4901

MT= 74 (n, n') Level
Legendre Coefficients

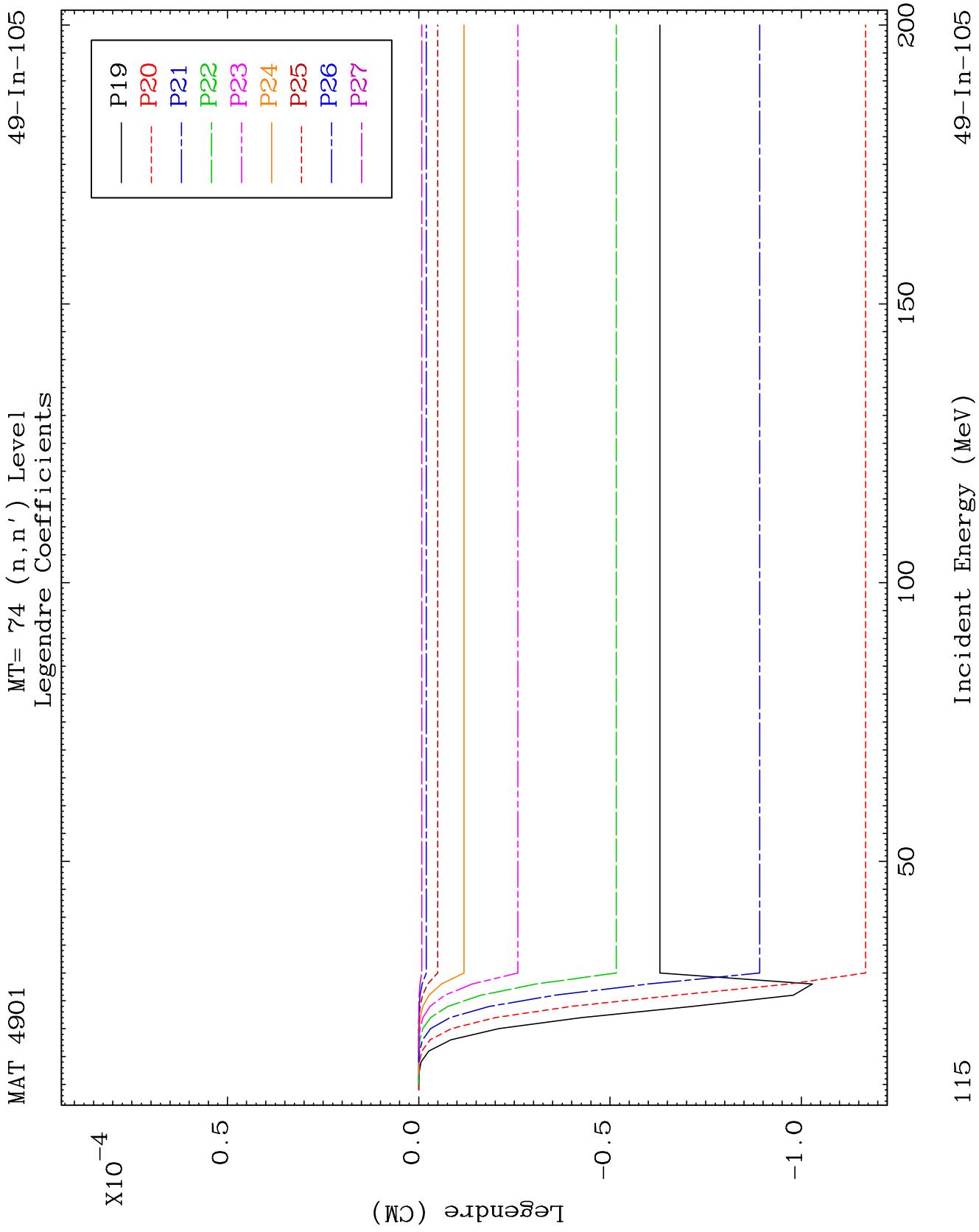
49-In-105

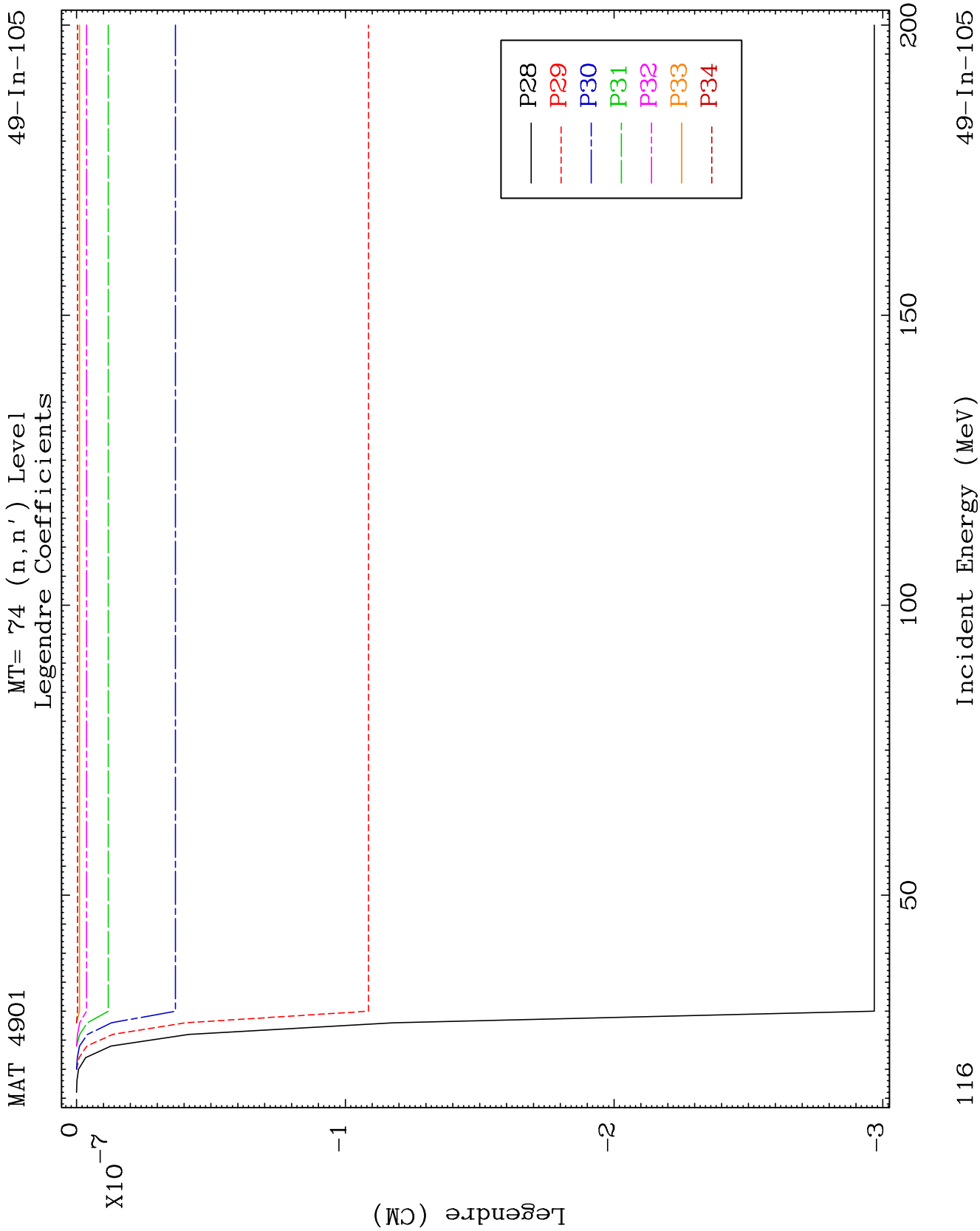


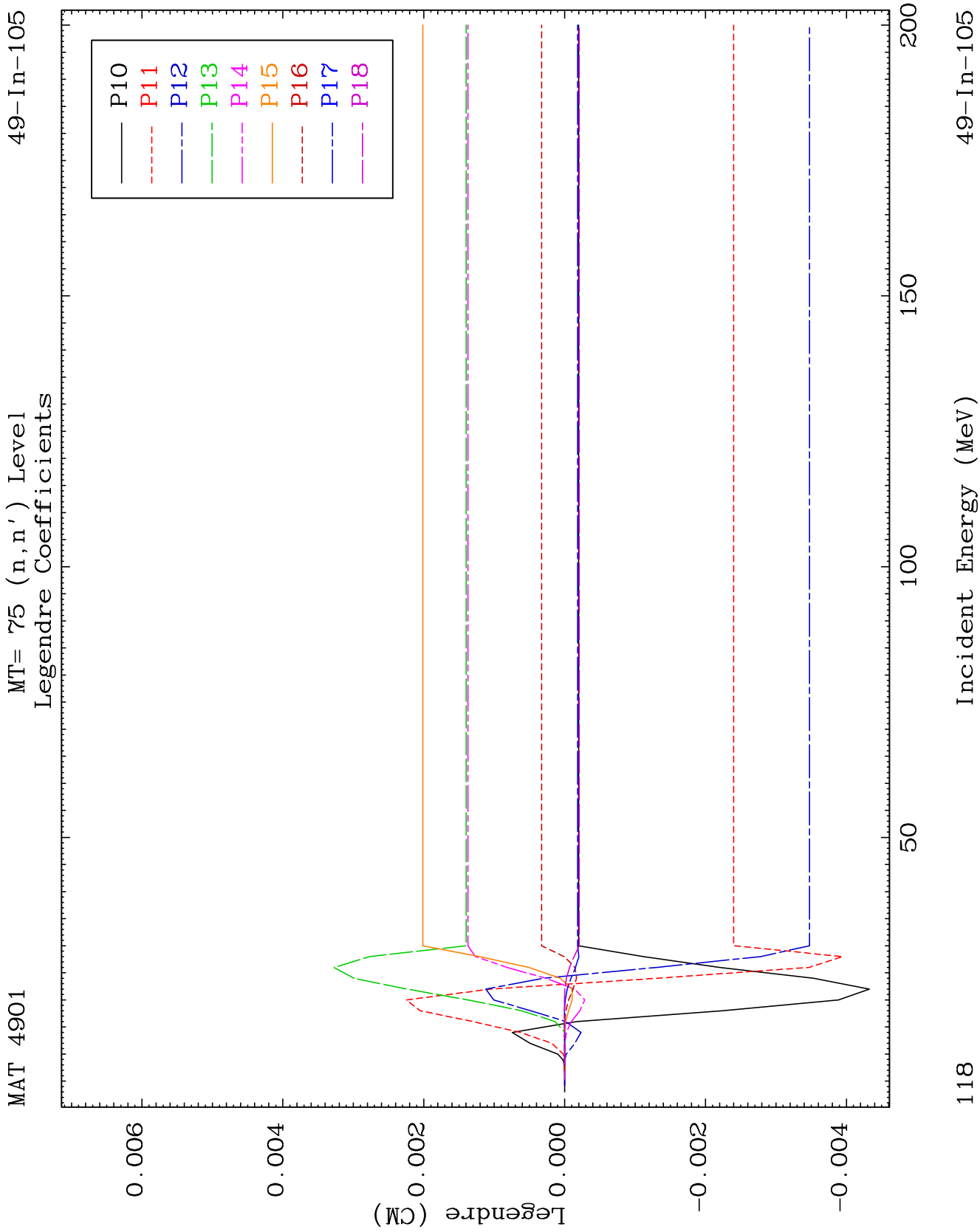
114

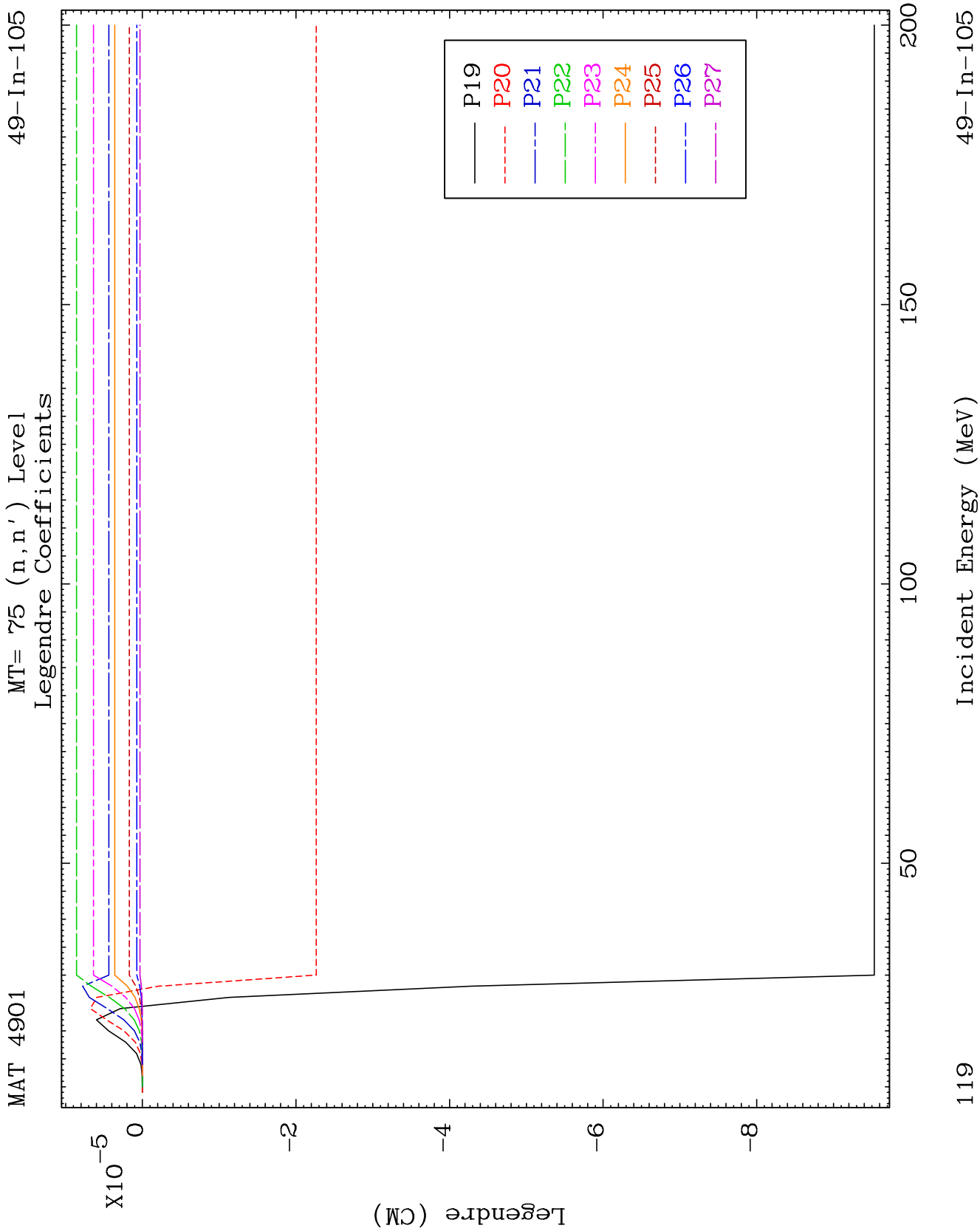
Incident Energy (MeV)

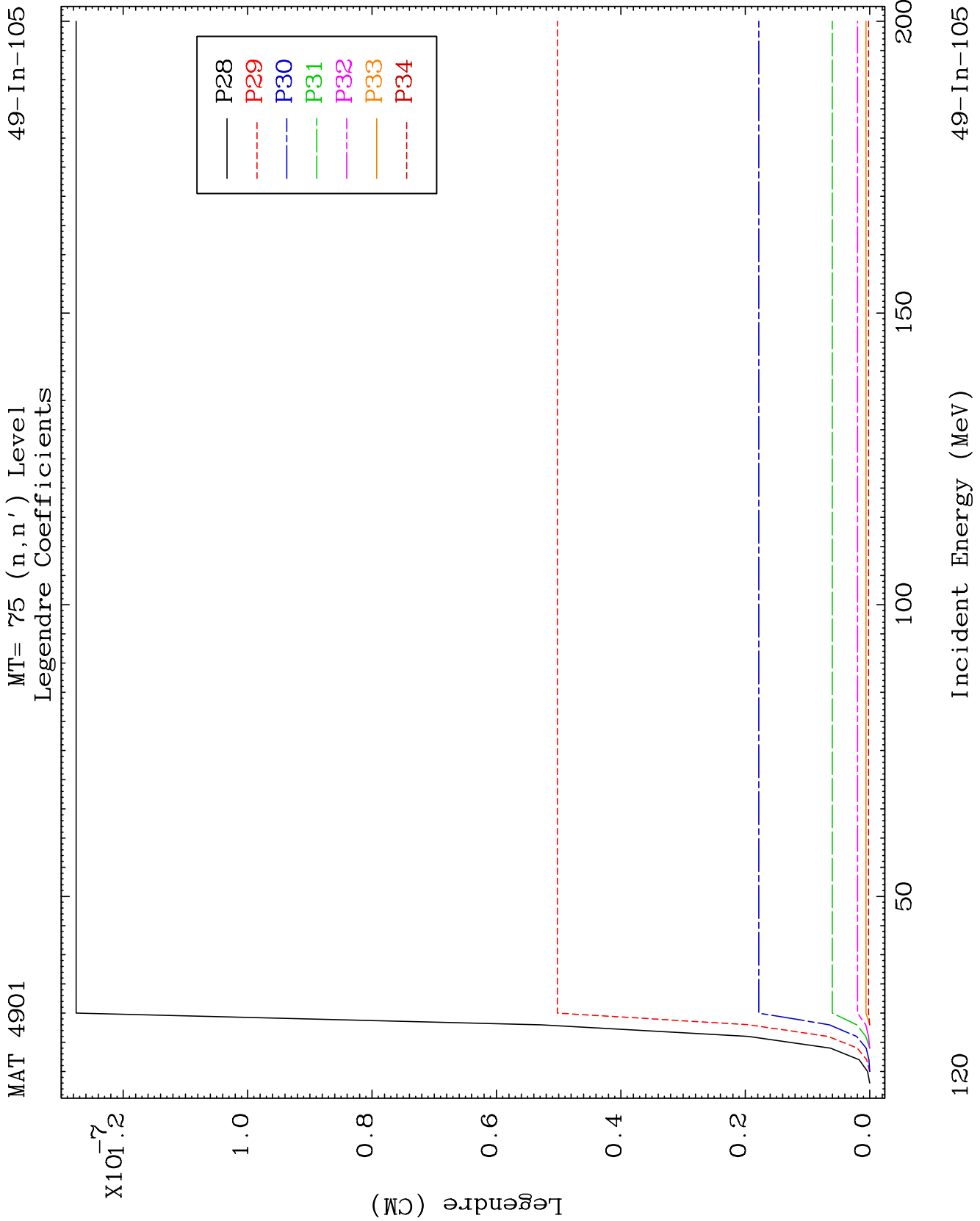
49-In-105







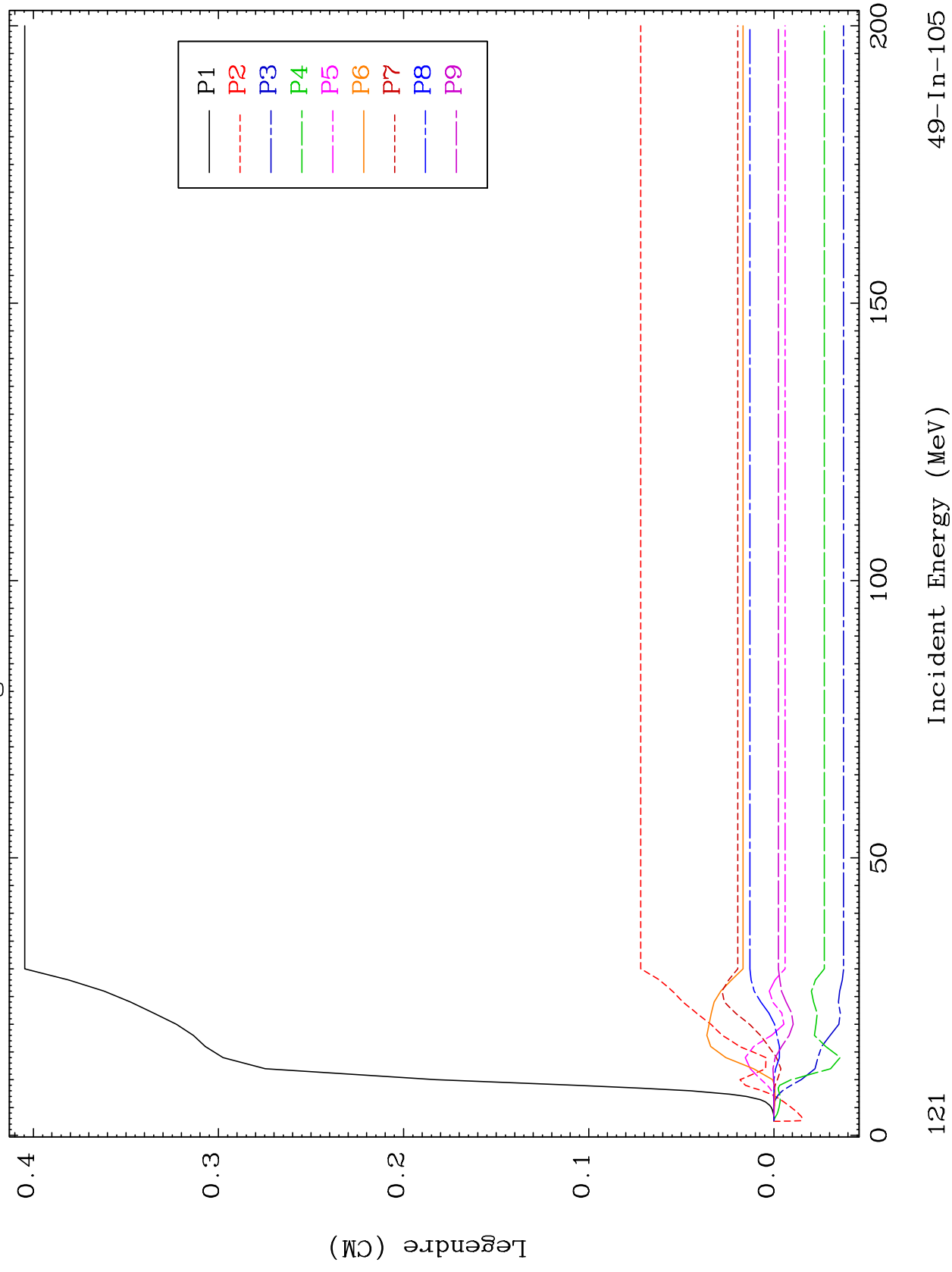




MAT 4901

MT= 76 (n, n') Level
Legendre Coefficients

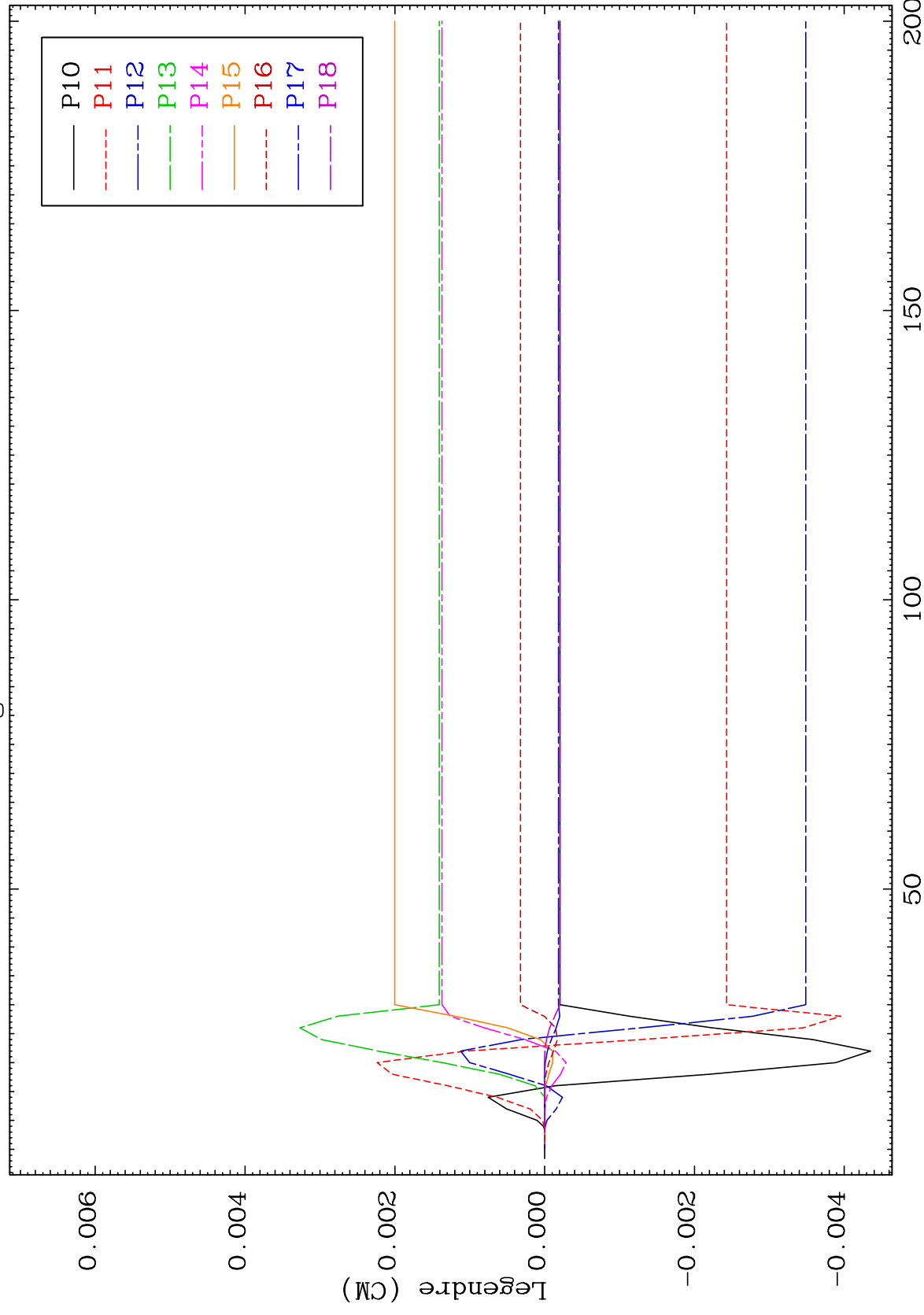
49-In-105



MAT 4901

MT= 76 (n, n') Level
Legendre Coefficients

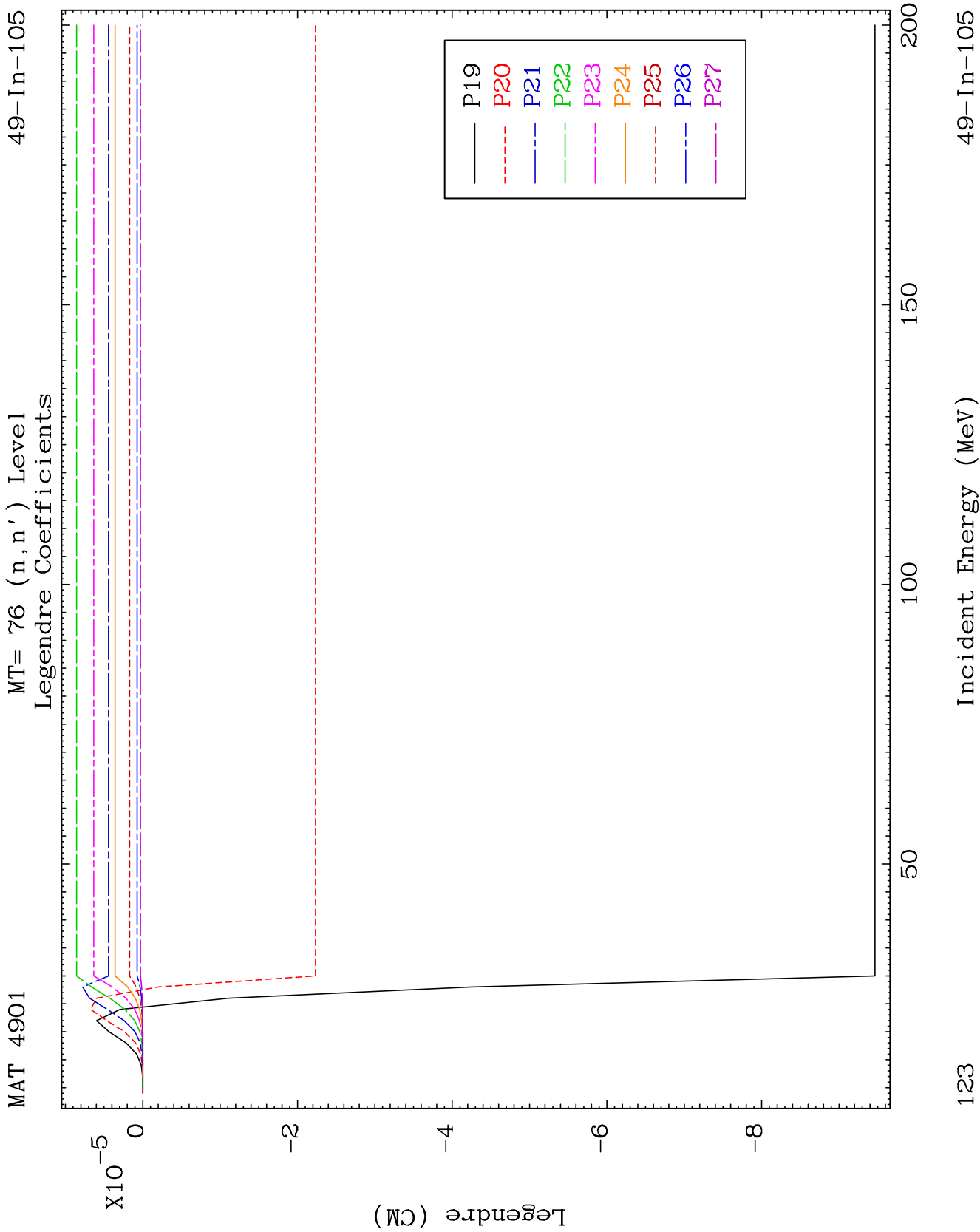
49-In-105

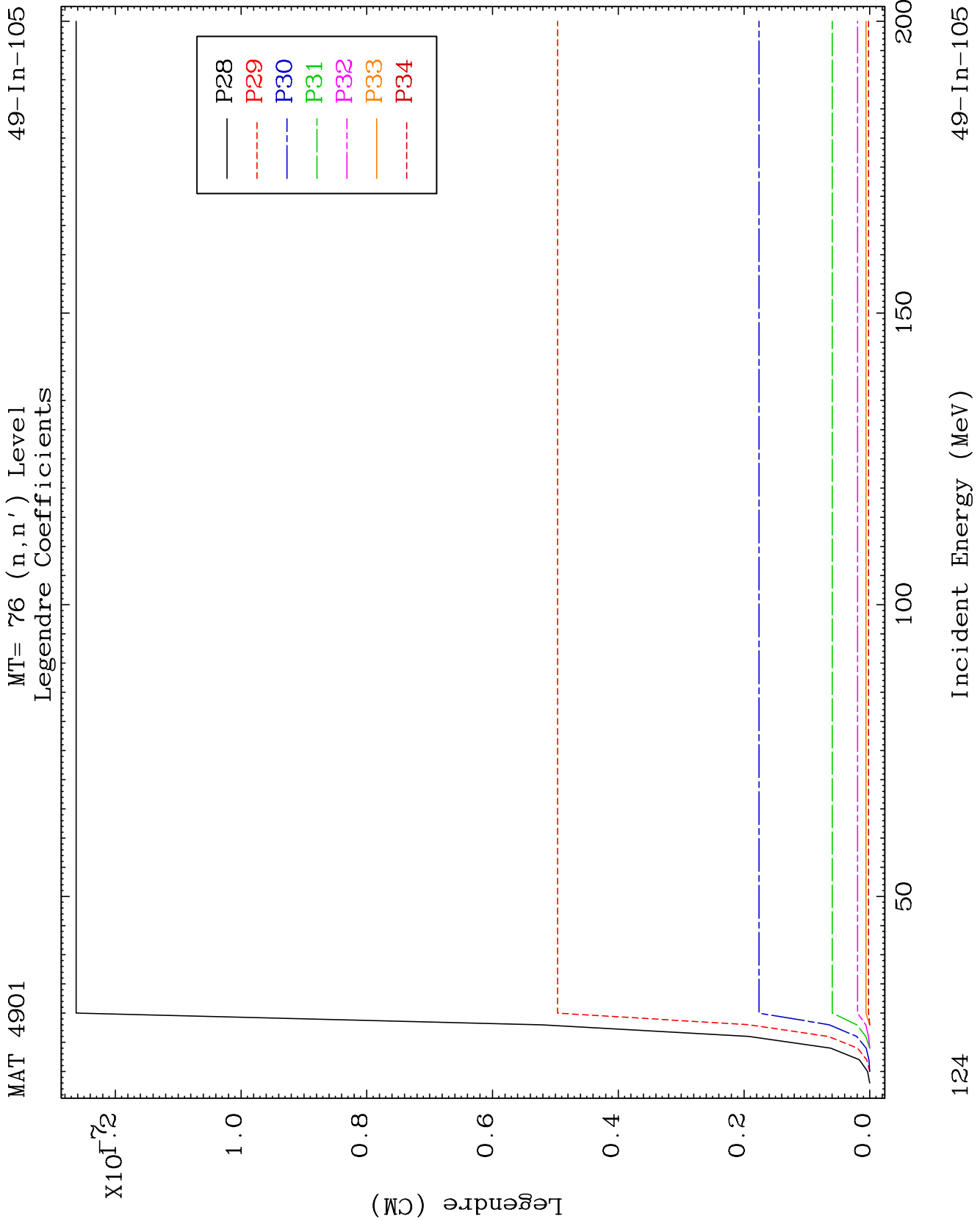


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Incident Energy (MeV)

49-In-105

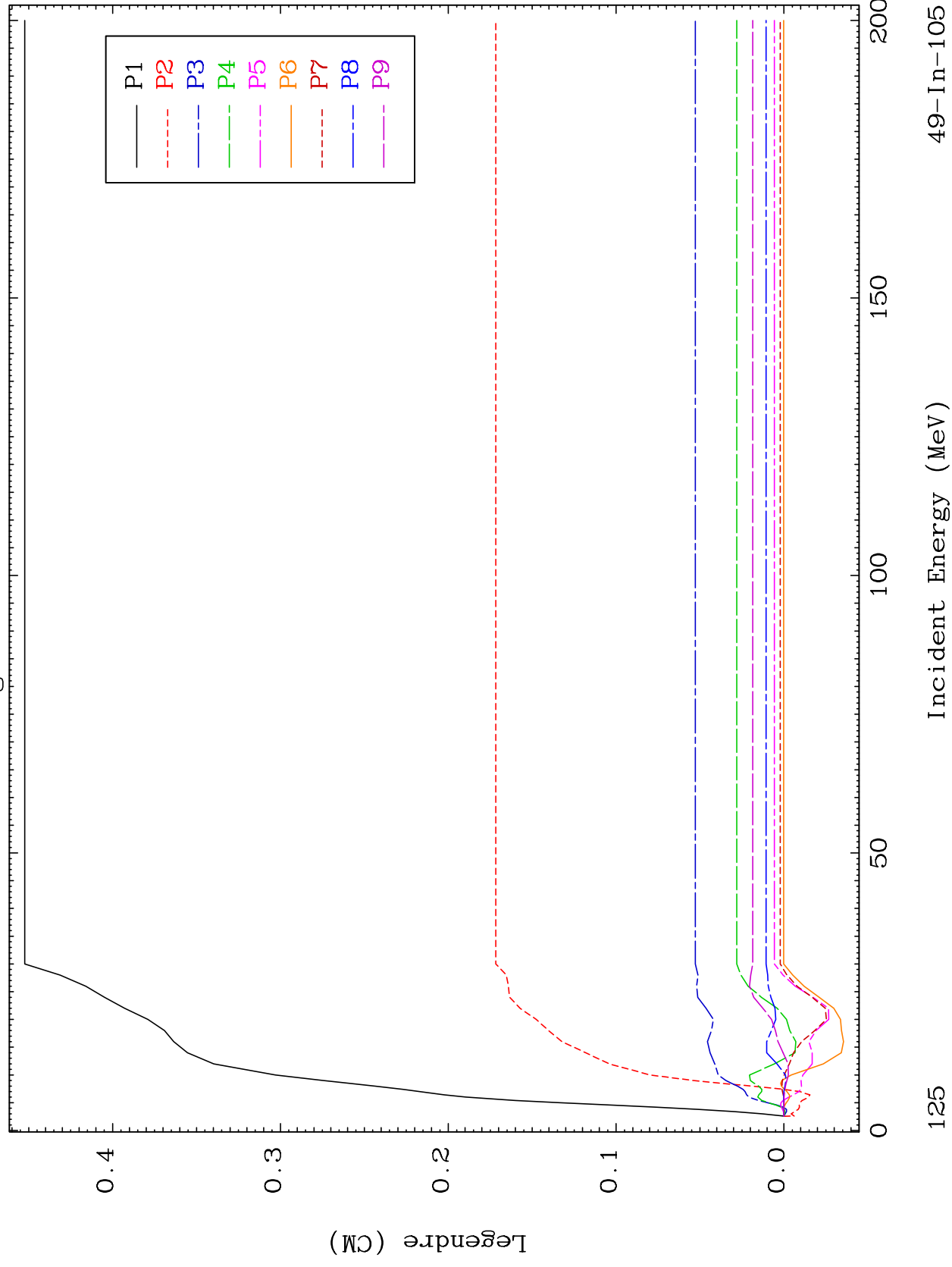


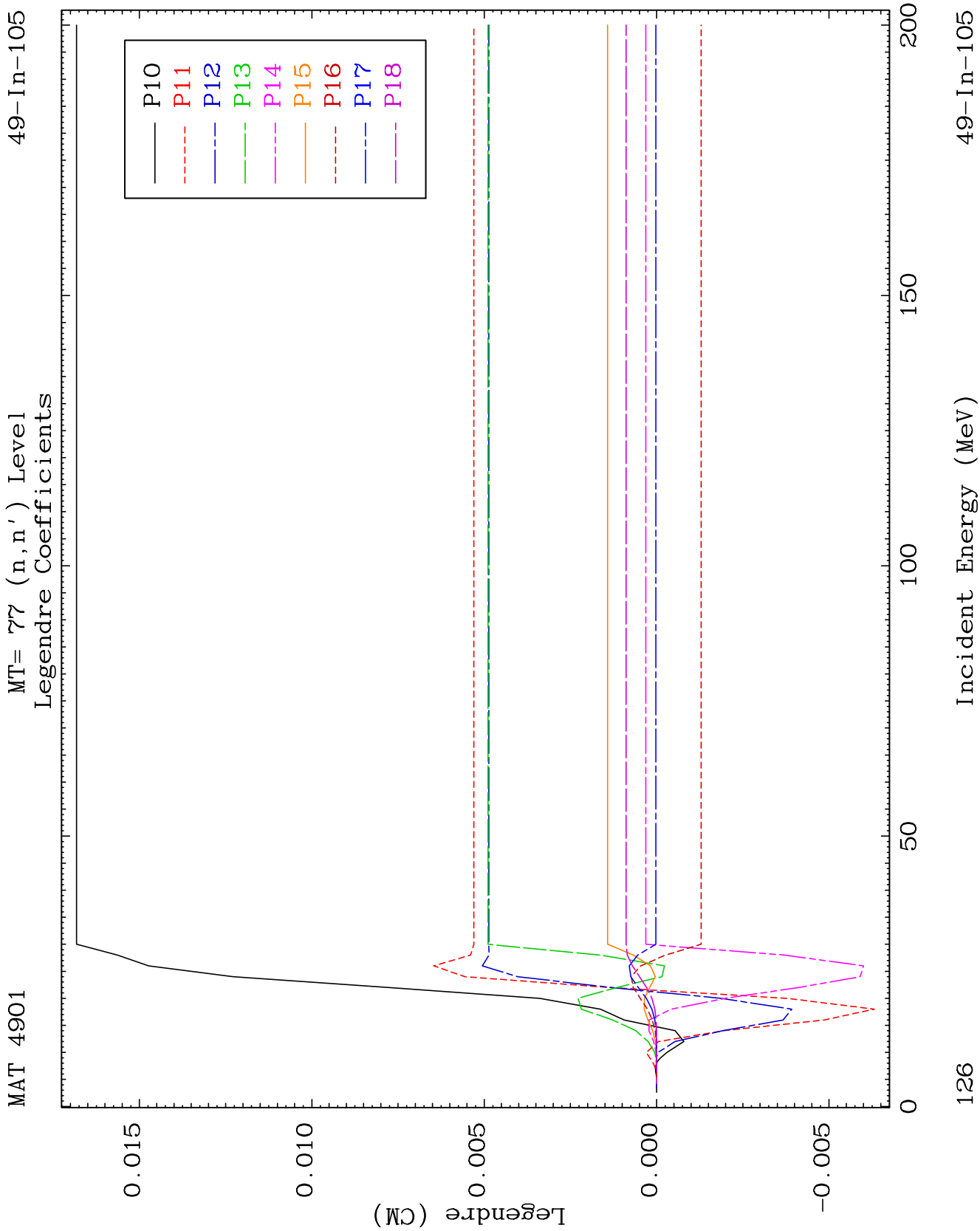


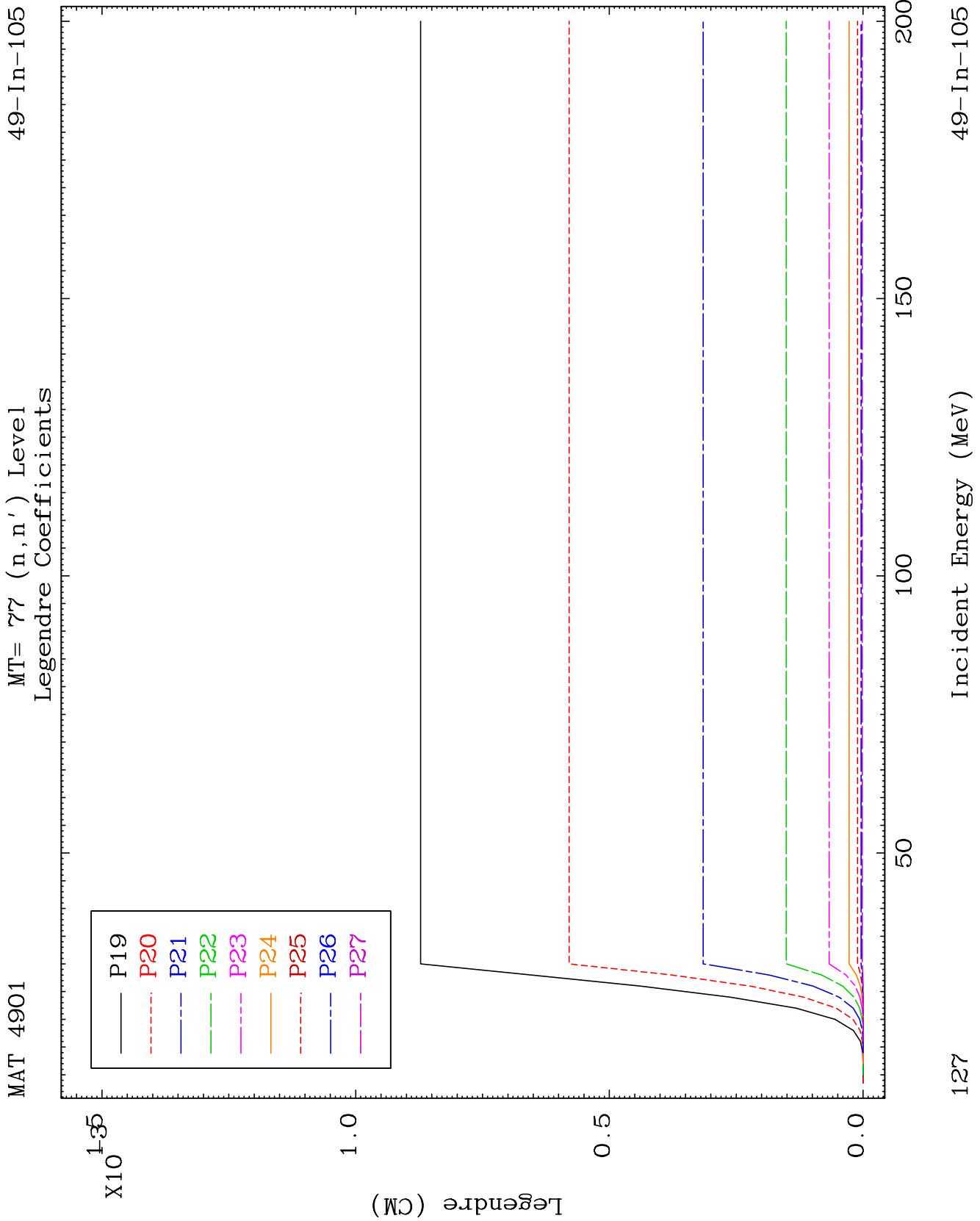
MAT 4901

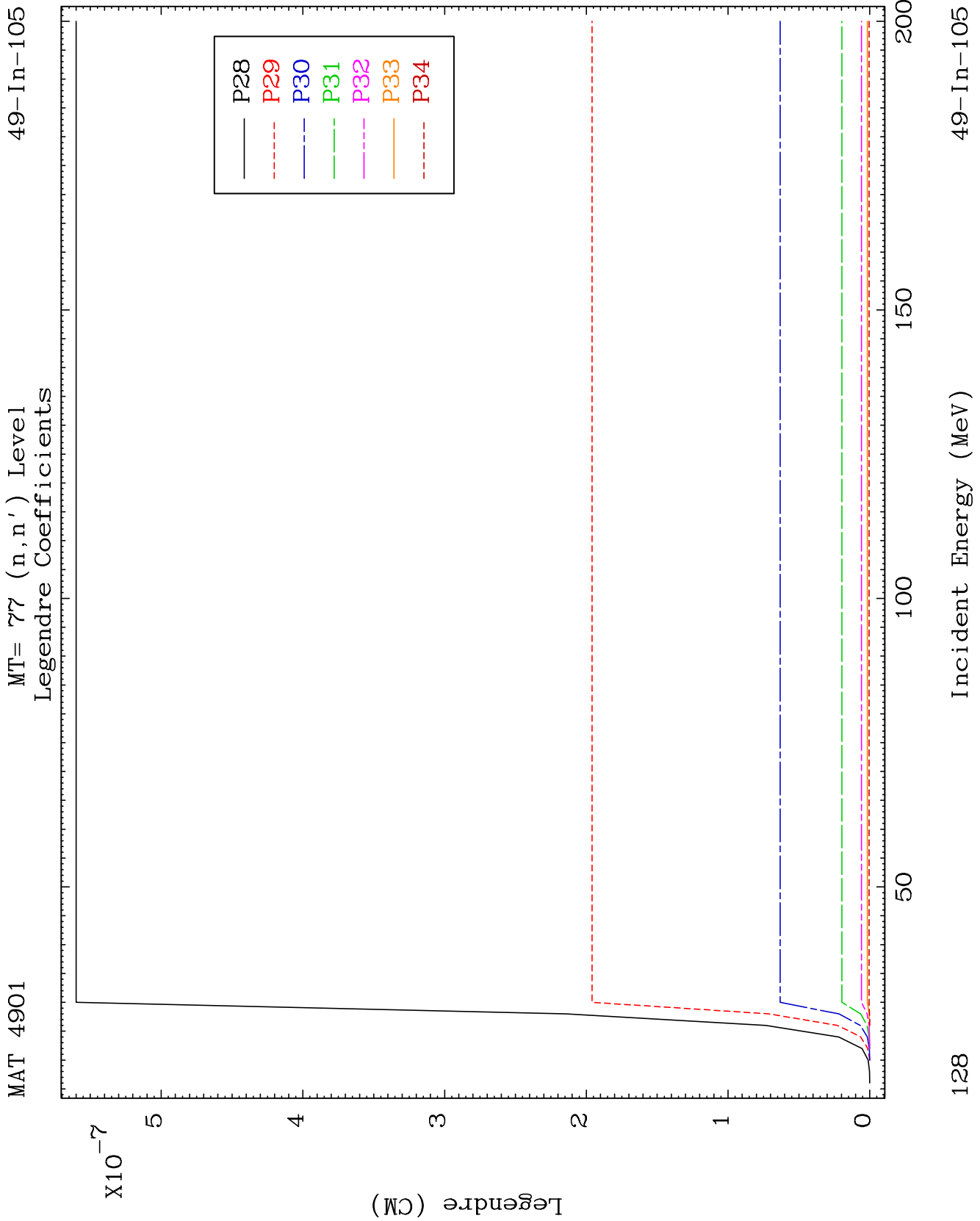
MT= 77 (n, n') Level
Legendre Coefficients

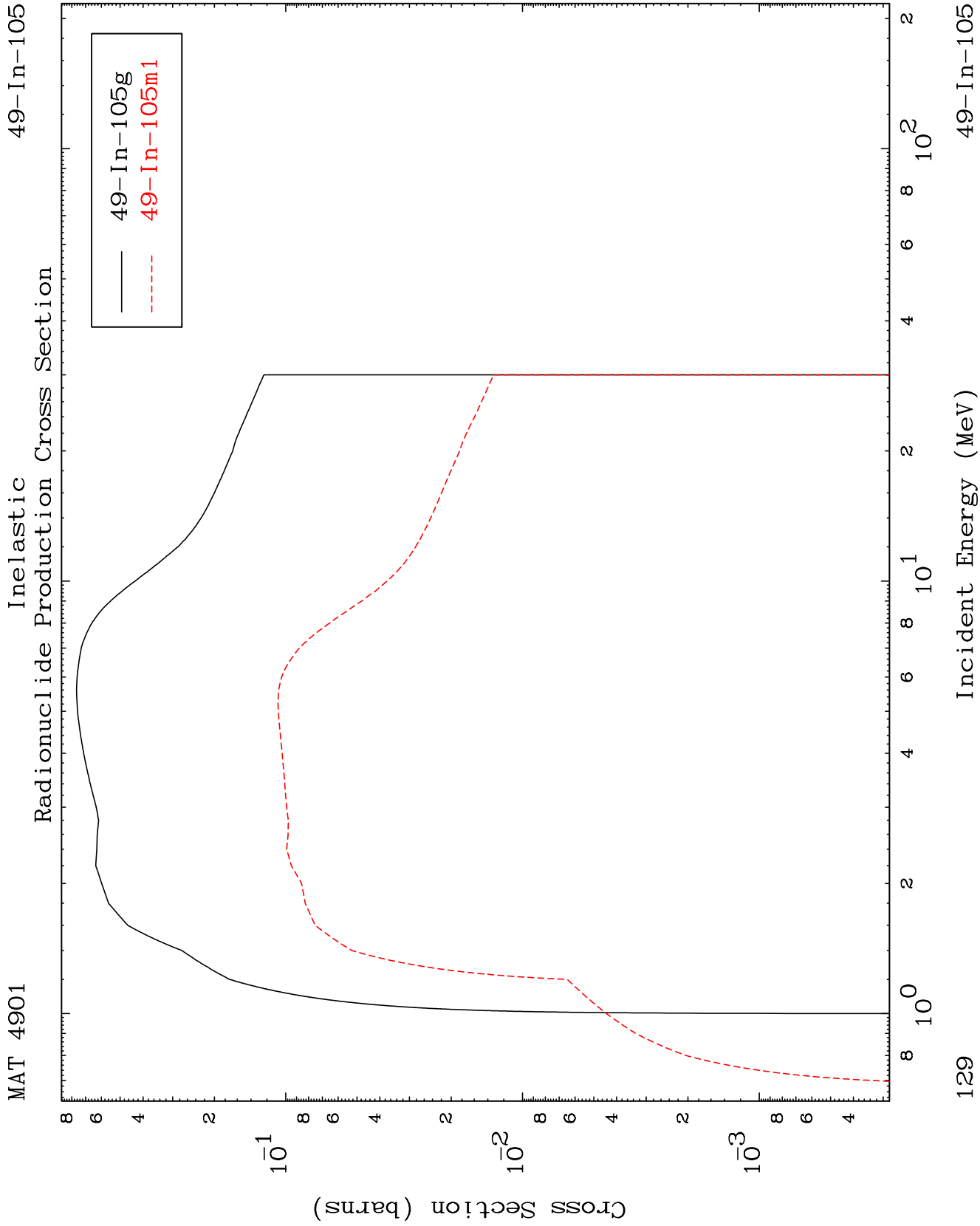
49-In-105







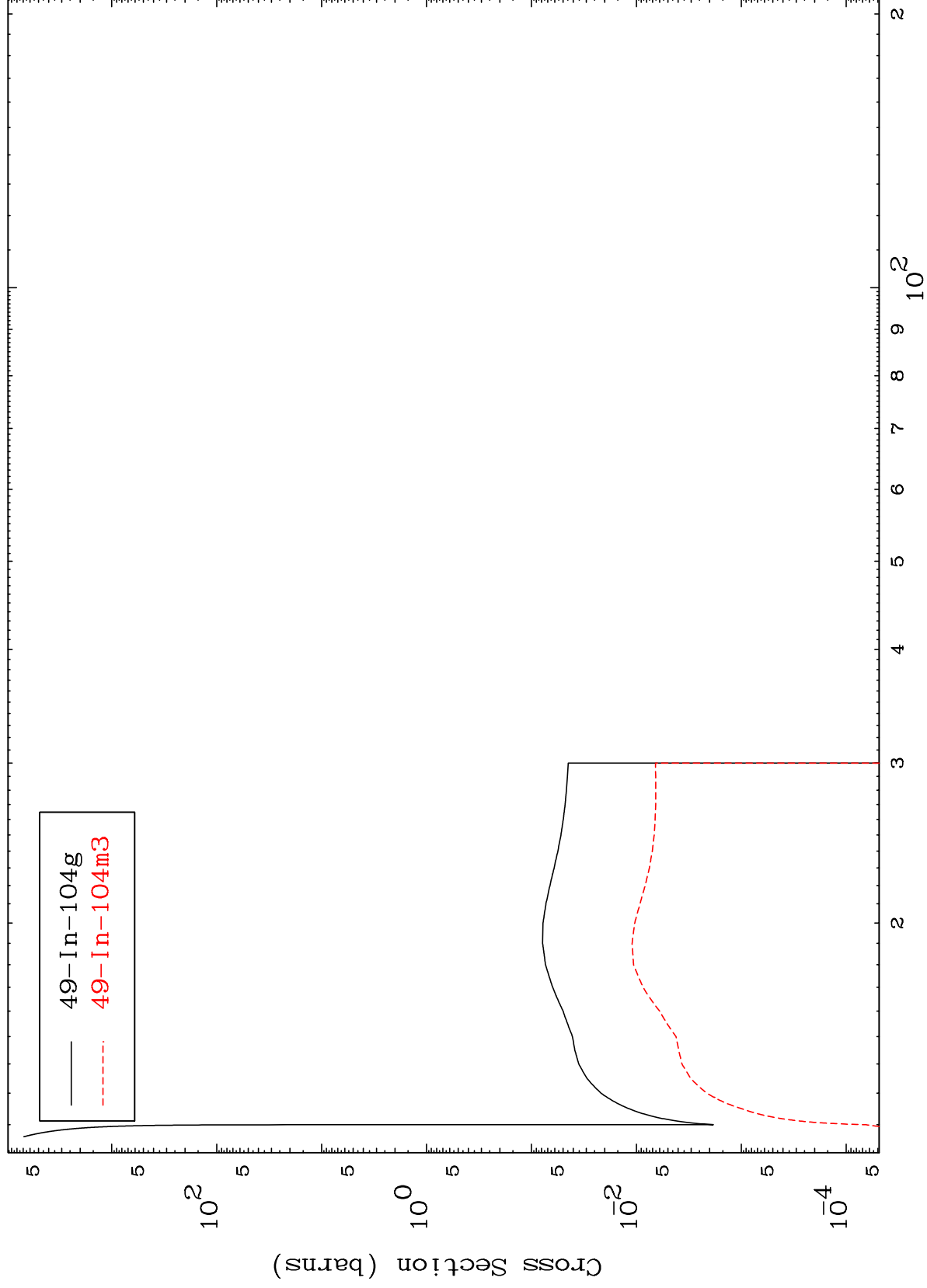




MAT 4901

49-In-105

Radionuclide Production Cross Section
(n,2n)



130

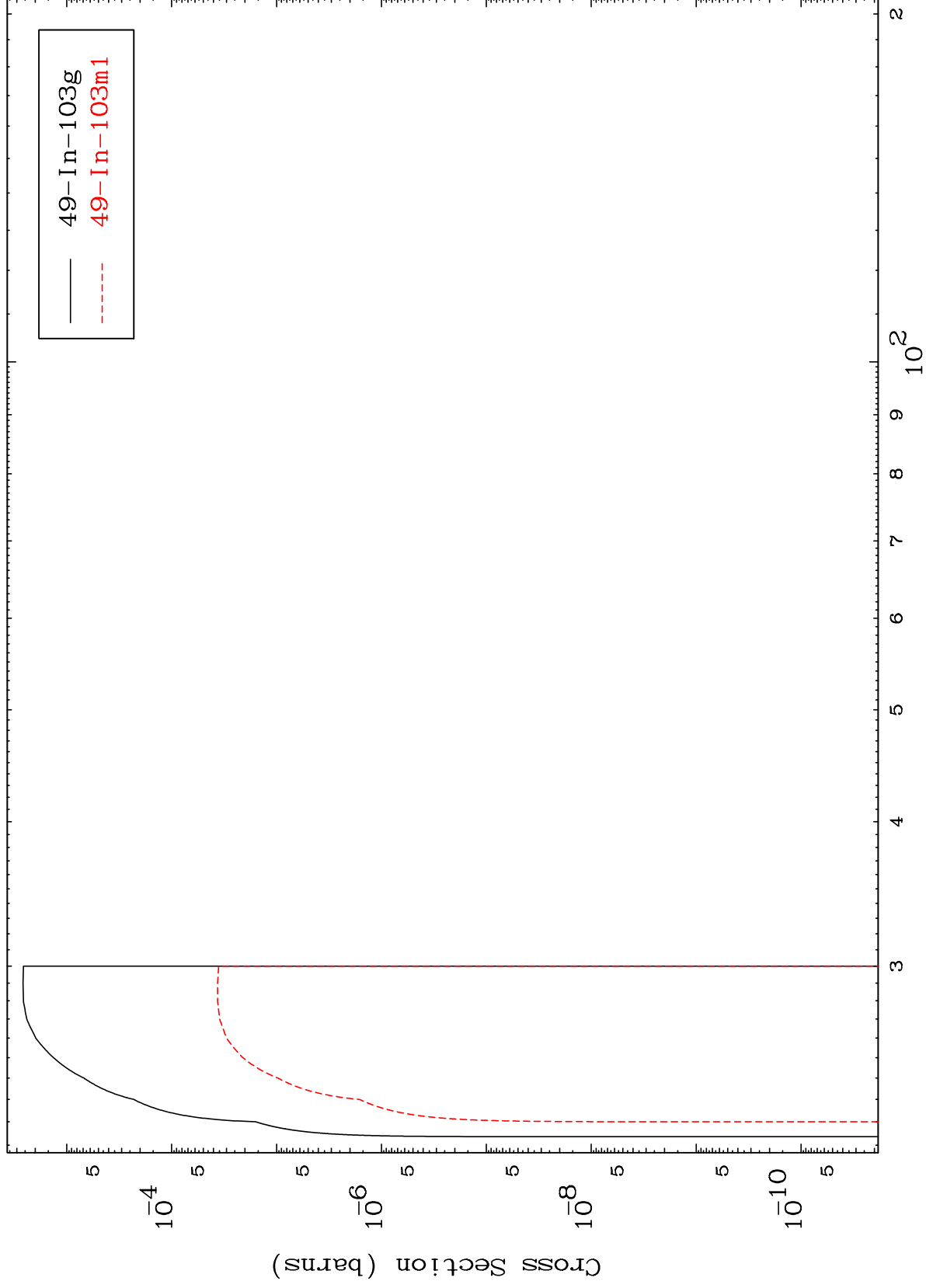
Incident Energy (MeV)

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MAT 4901

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(n,3n)
Radionuclide Production Cross Section



131

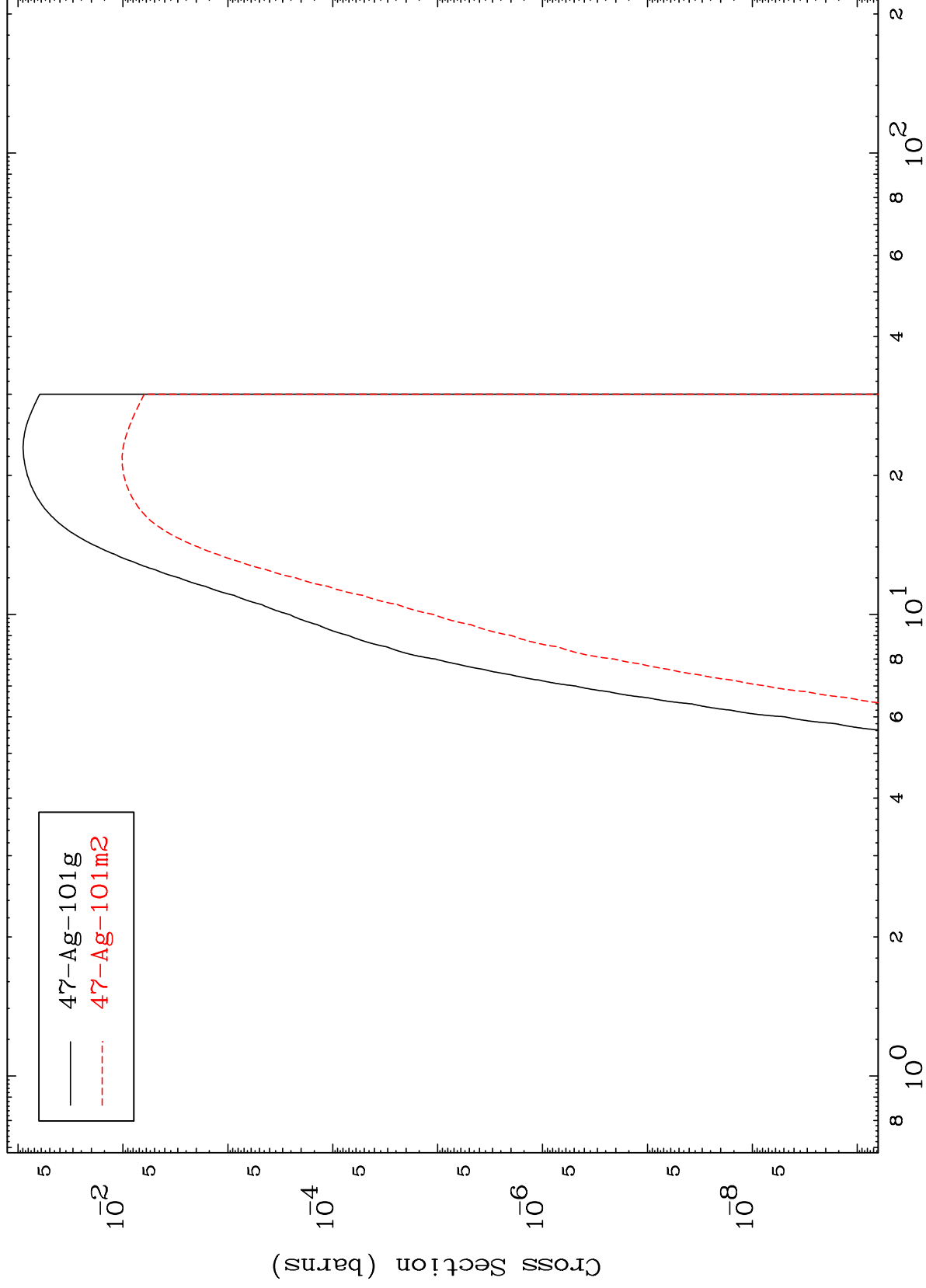
Incident Energy (MeV)

49-In-105

MAT 4901

49-In-105

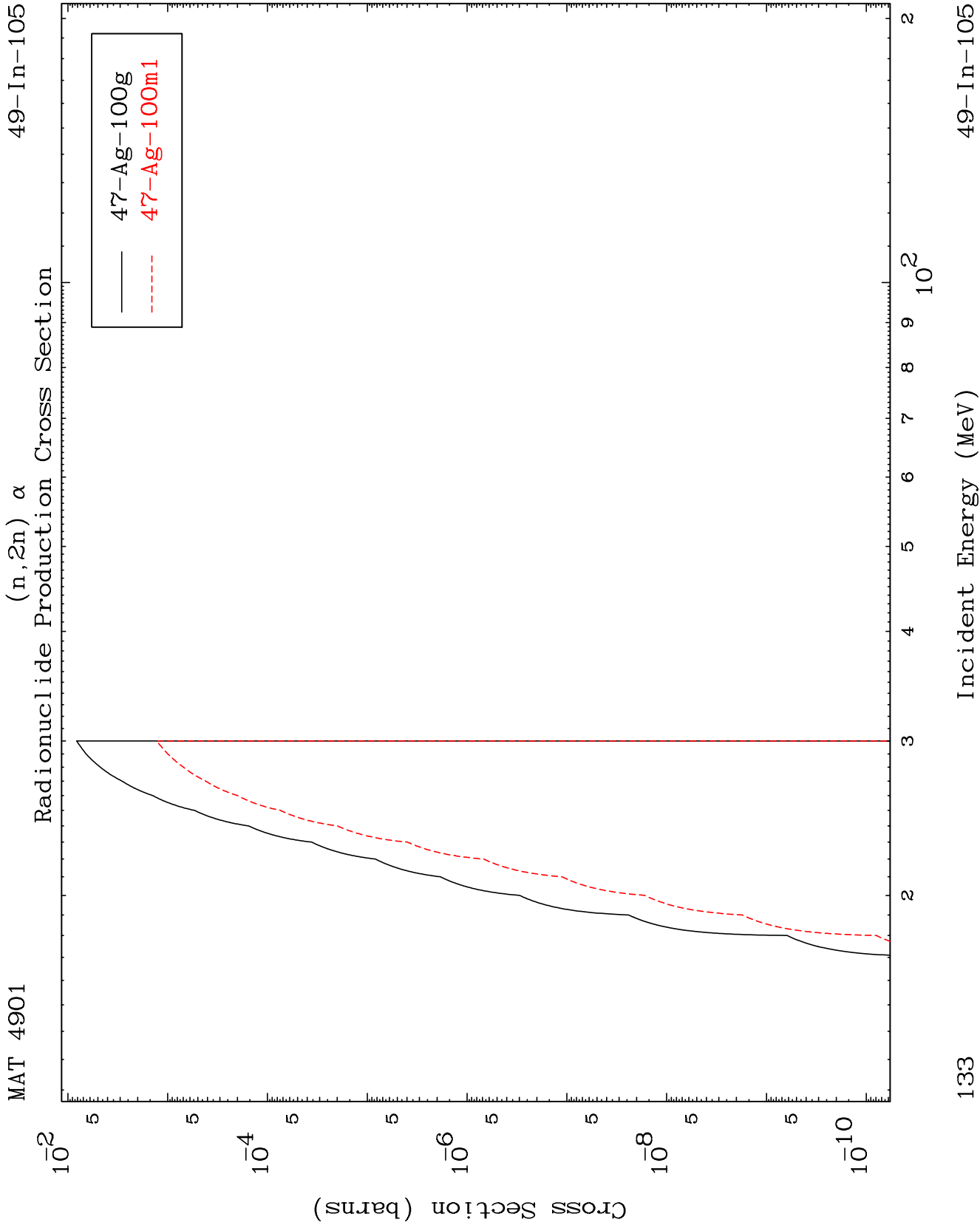
$(n,n') \alpha$
Radionuclide Production Cross Section

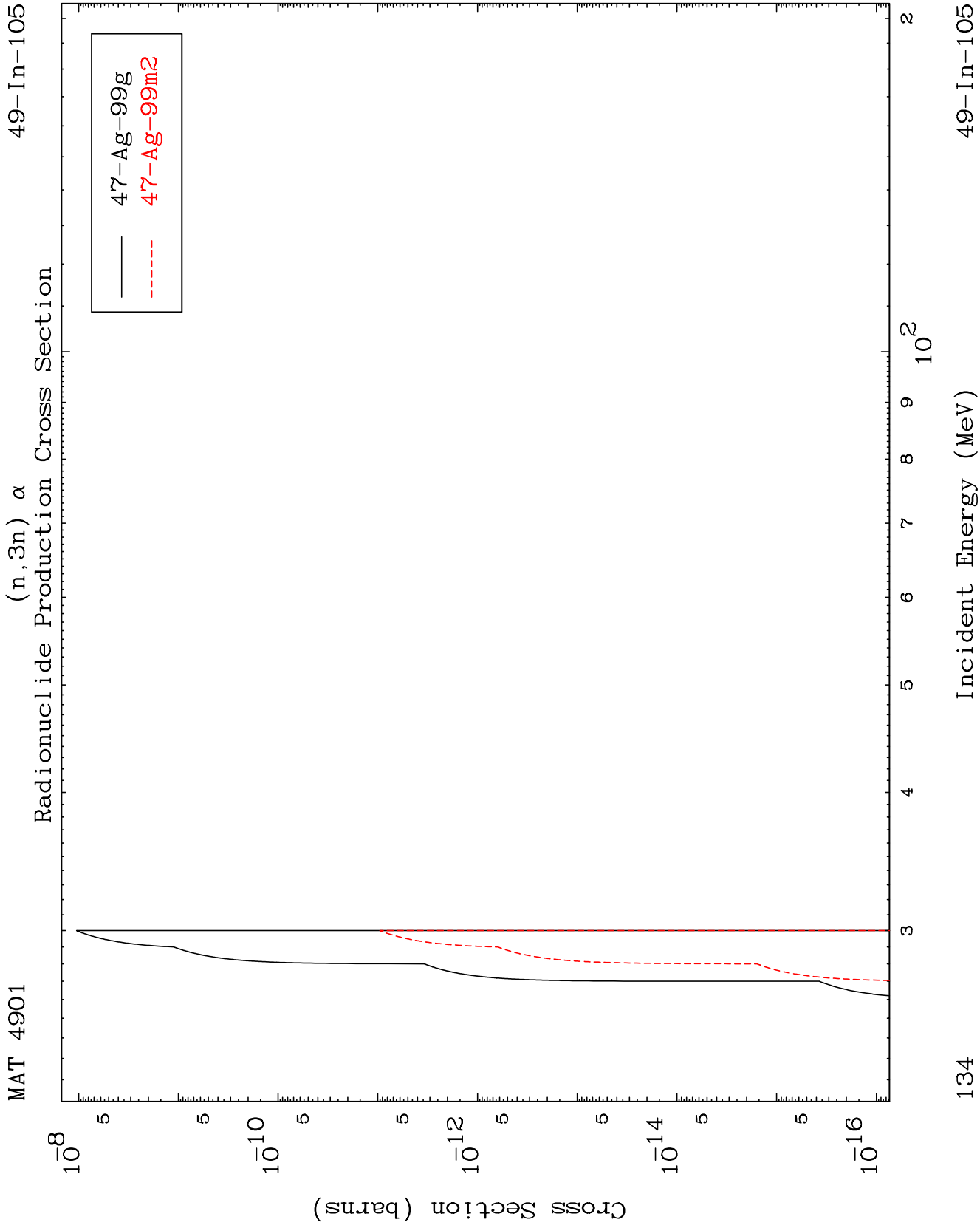


132

Incident Energy (MeV)

49-In-105

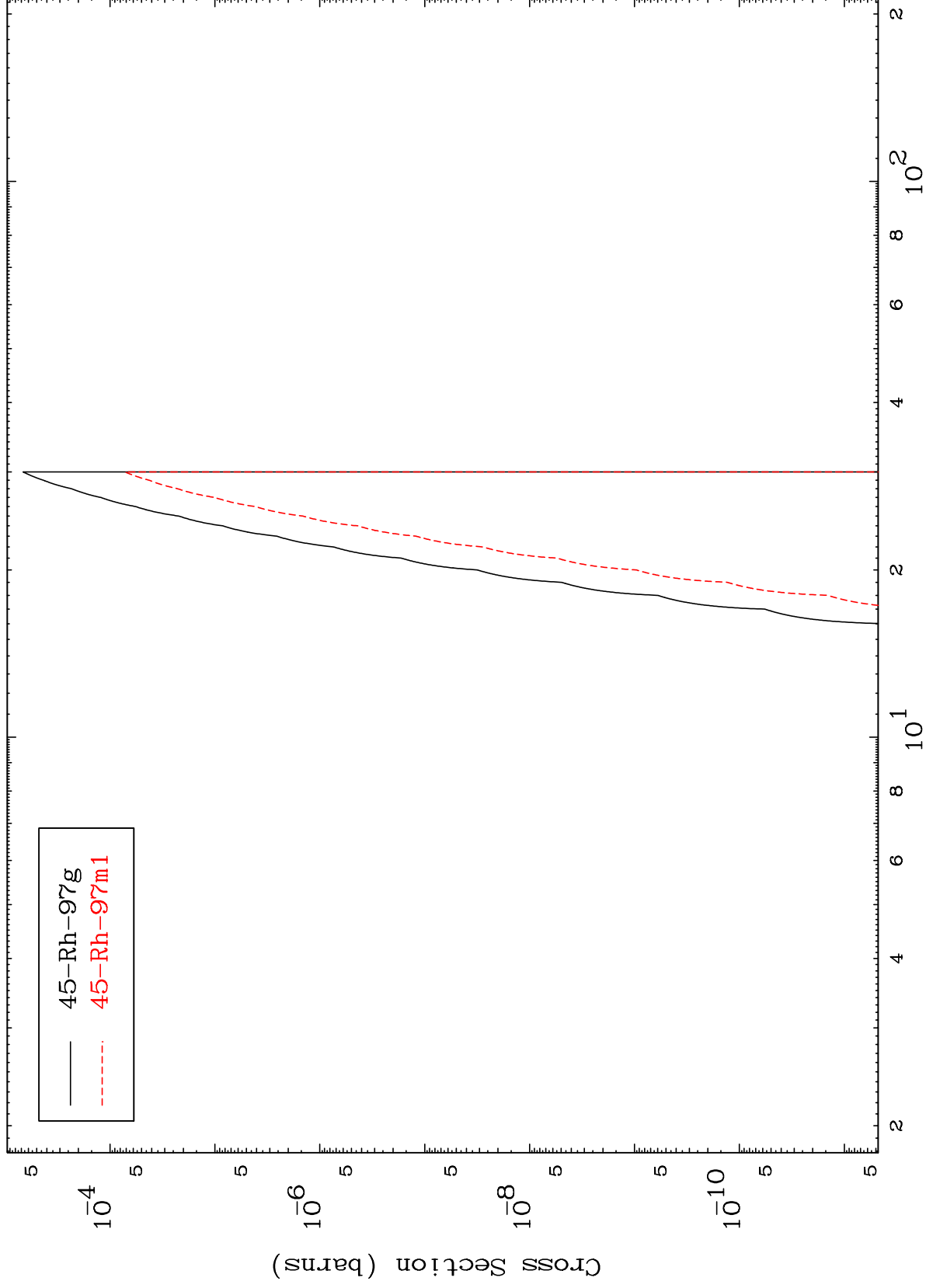




MAT 4901

49-In-105

(n,n') 2 α
Radionuclide Production Cross Section

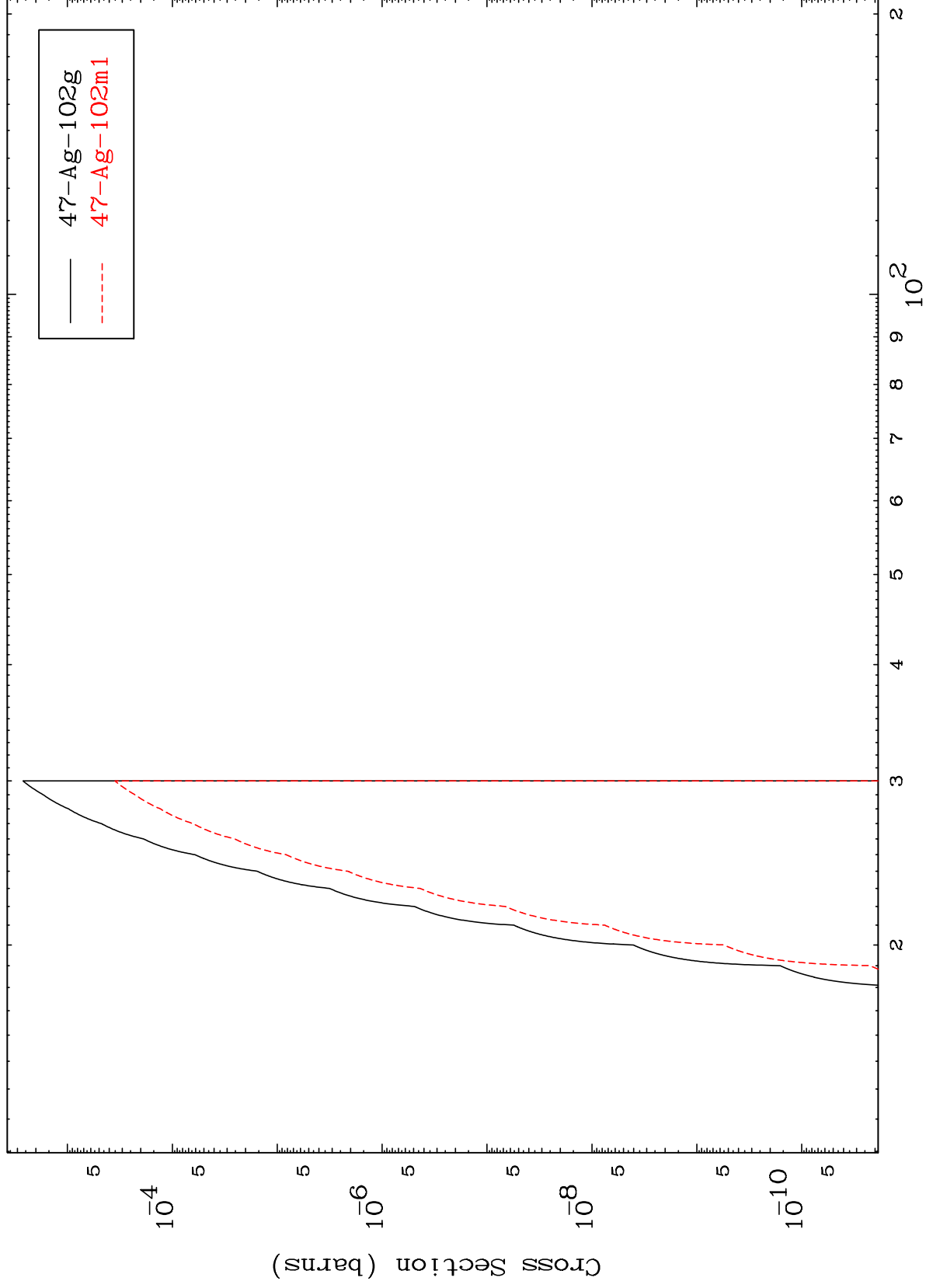


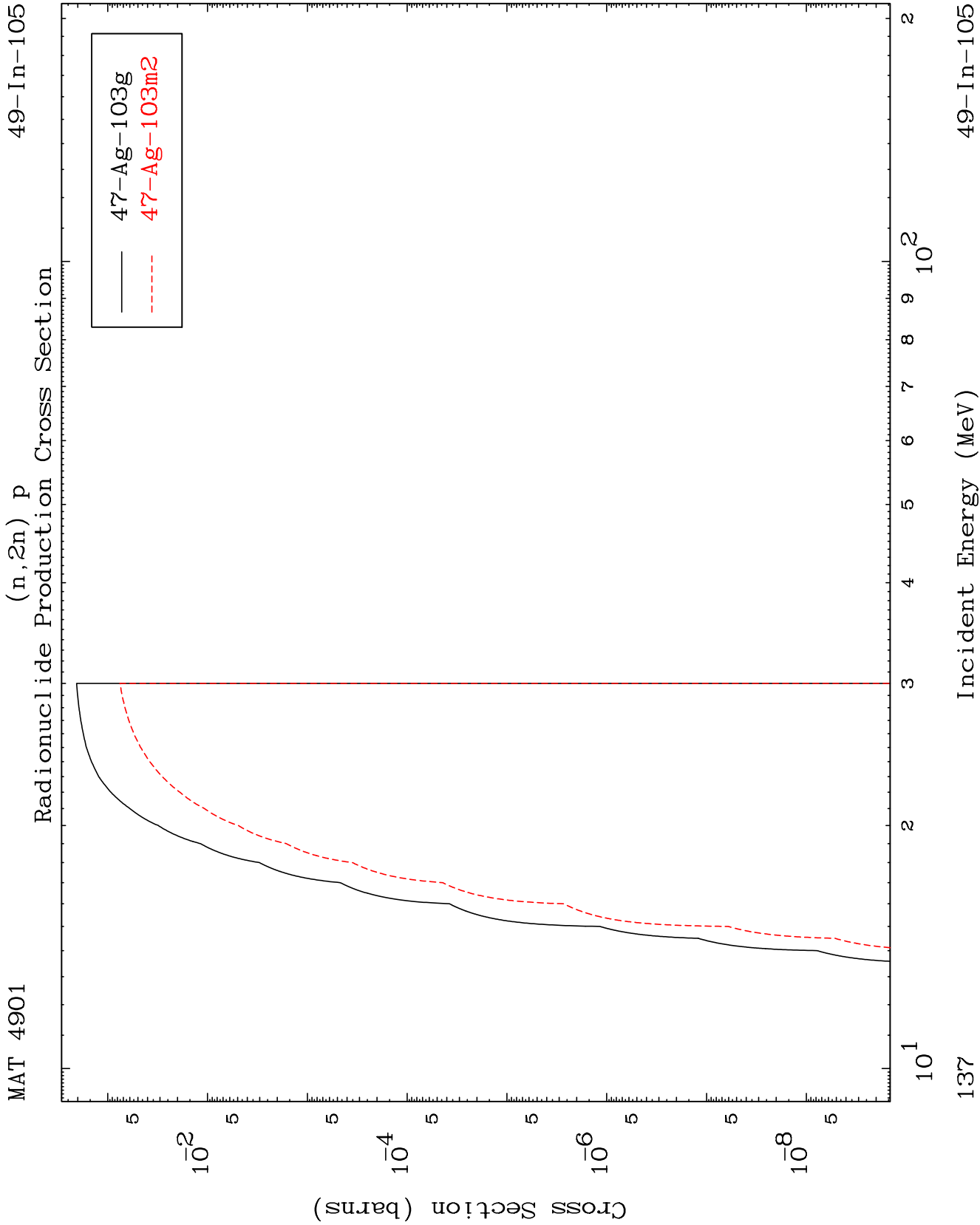
135

Incident Energy (MeV)

49-In-105

Radionuclide Production Cross Section

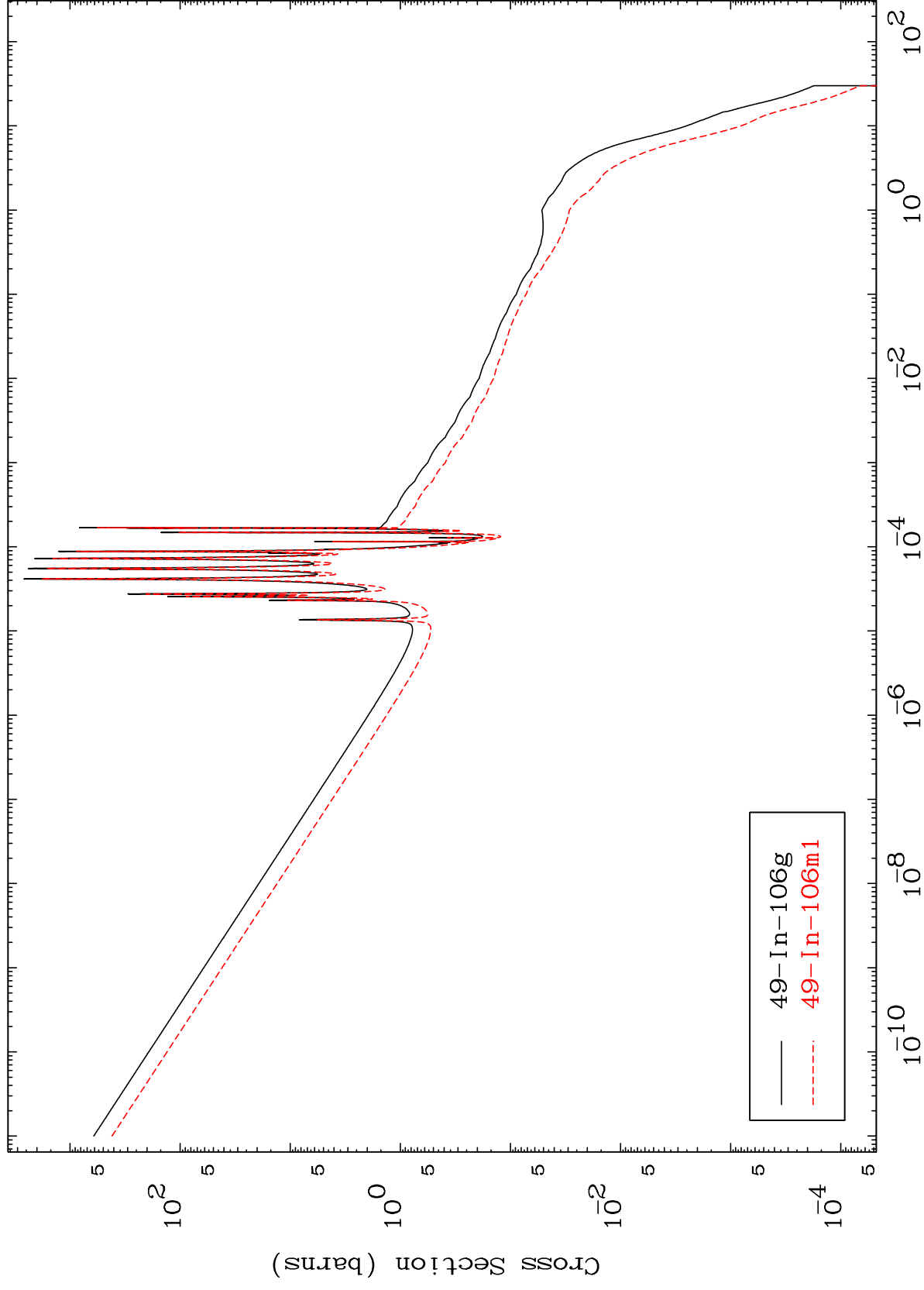




MAT 4901

49-In-105

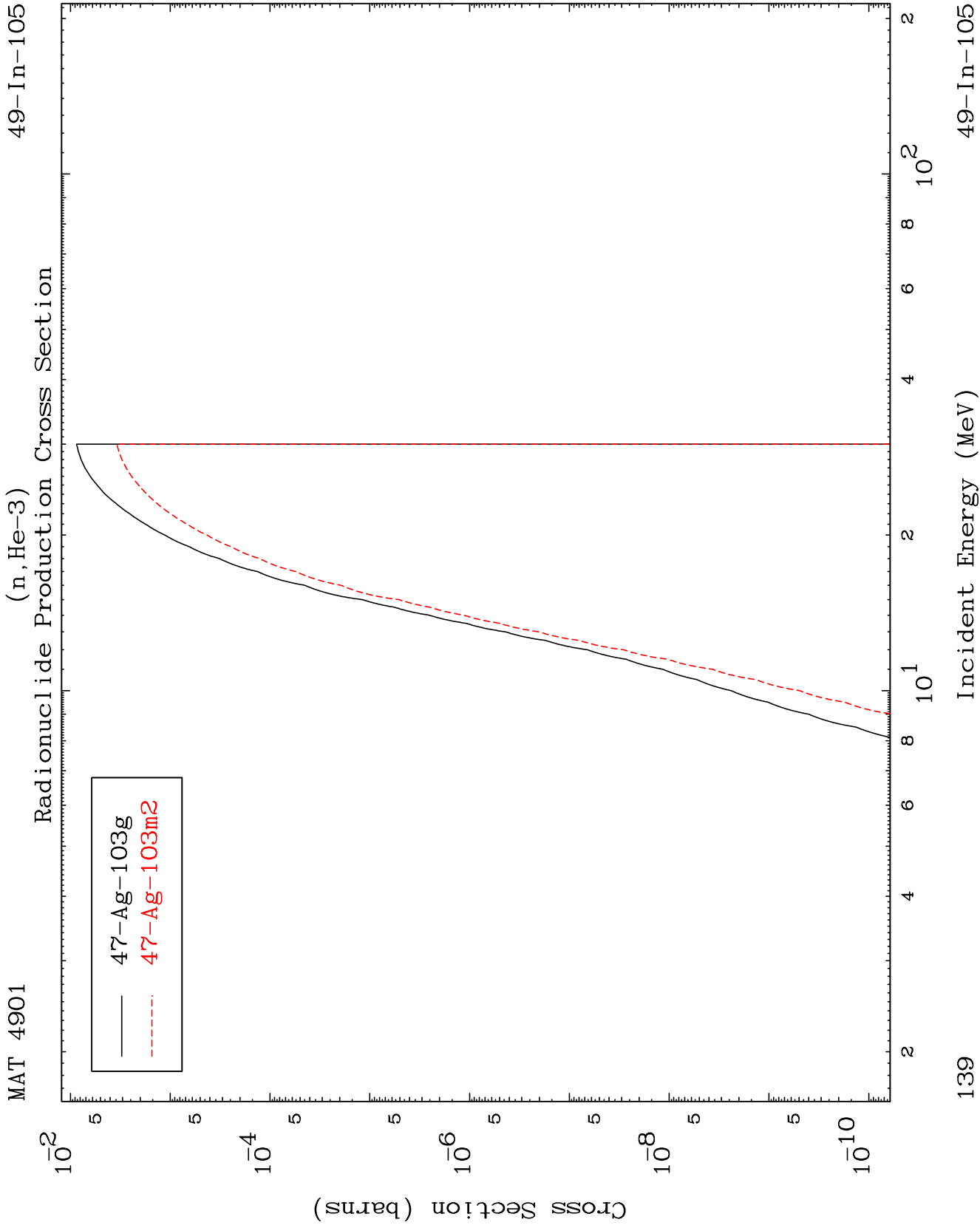
(n, γ)
Radionuclide Production Cross Section

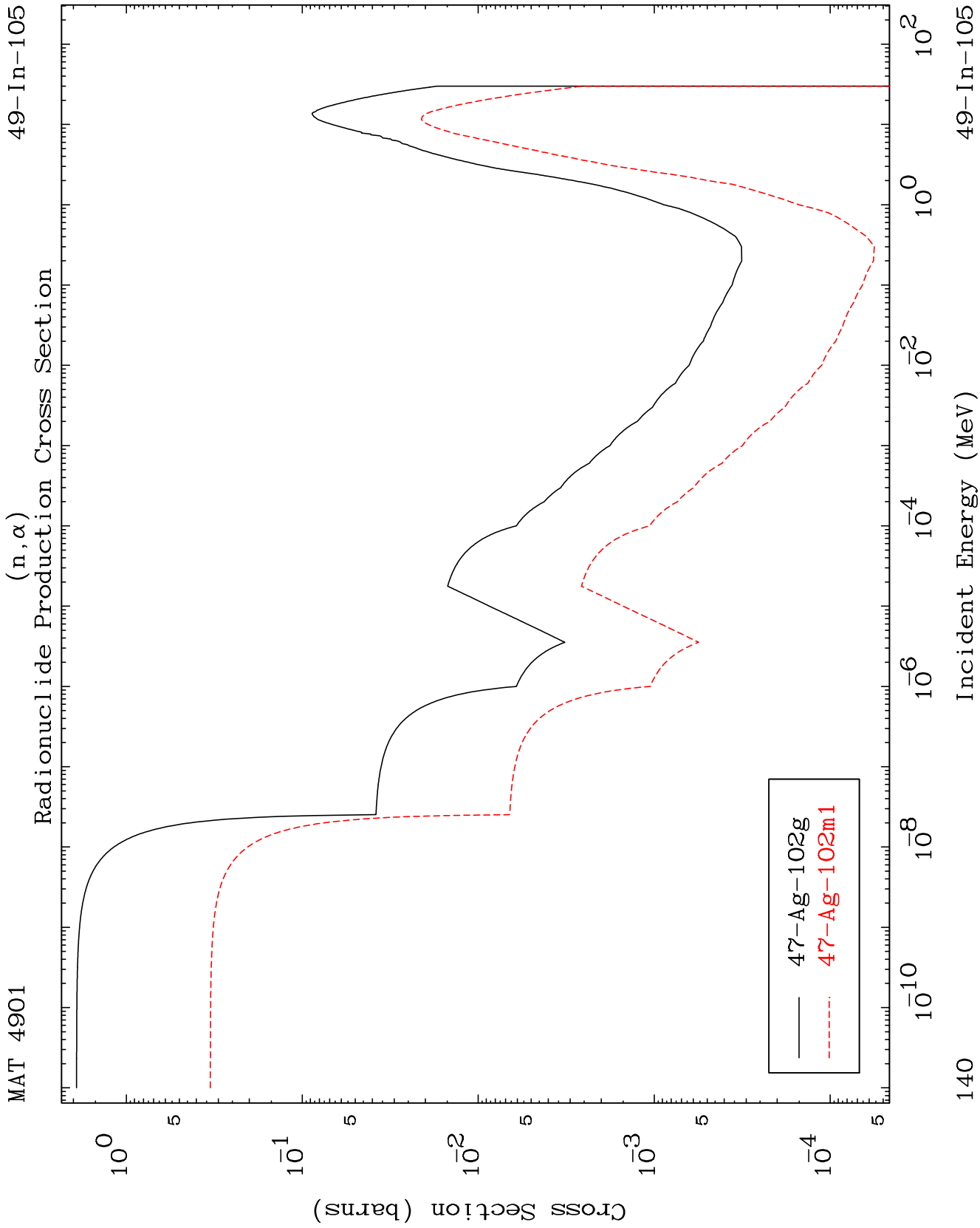


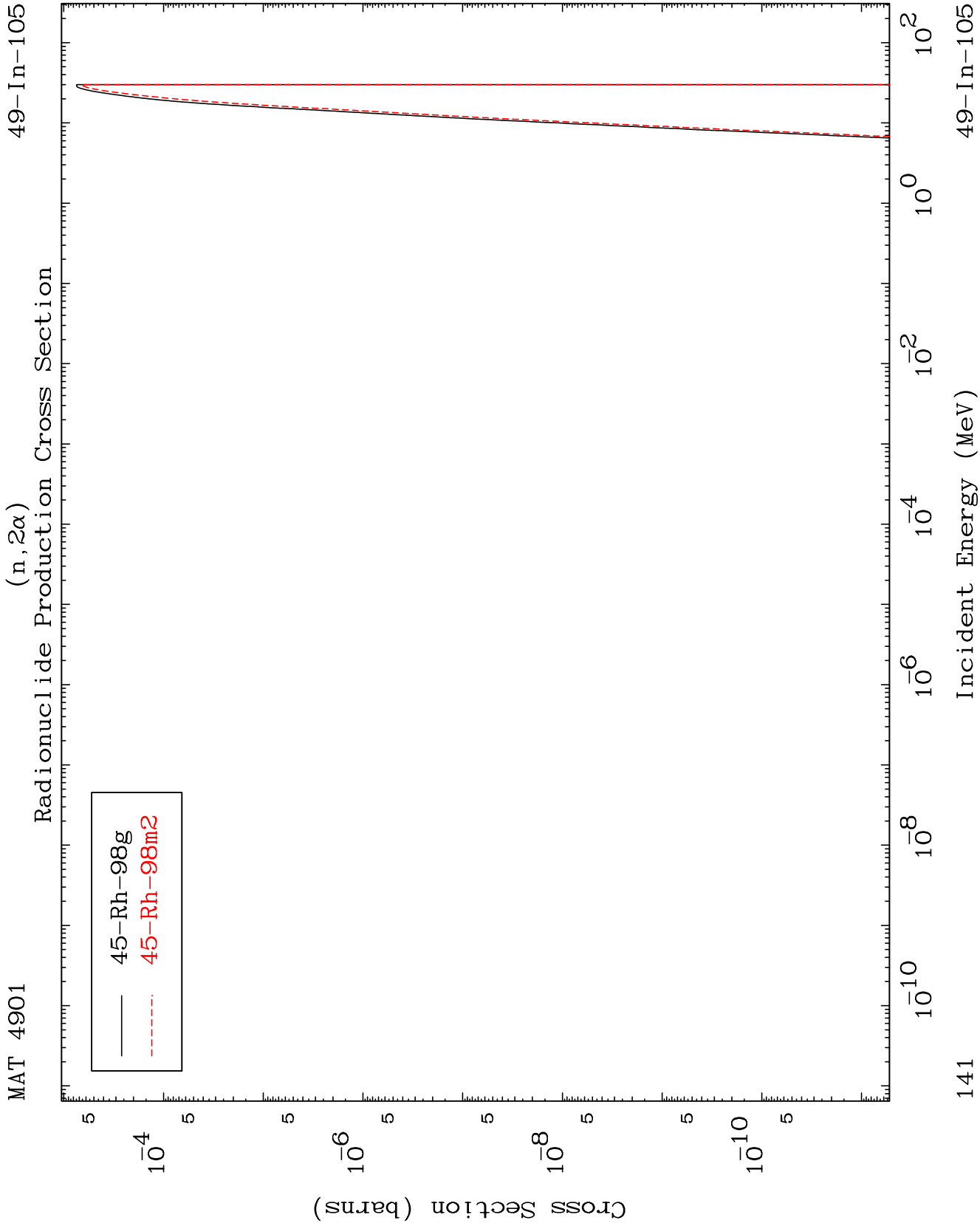
138

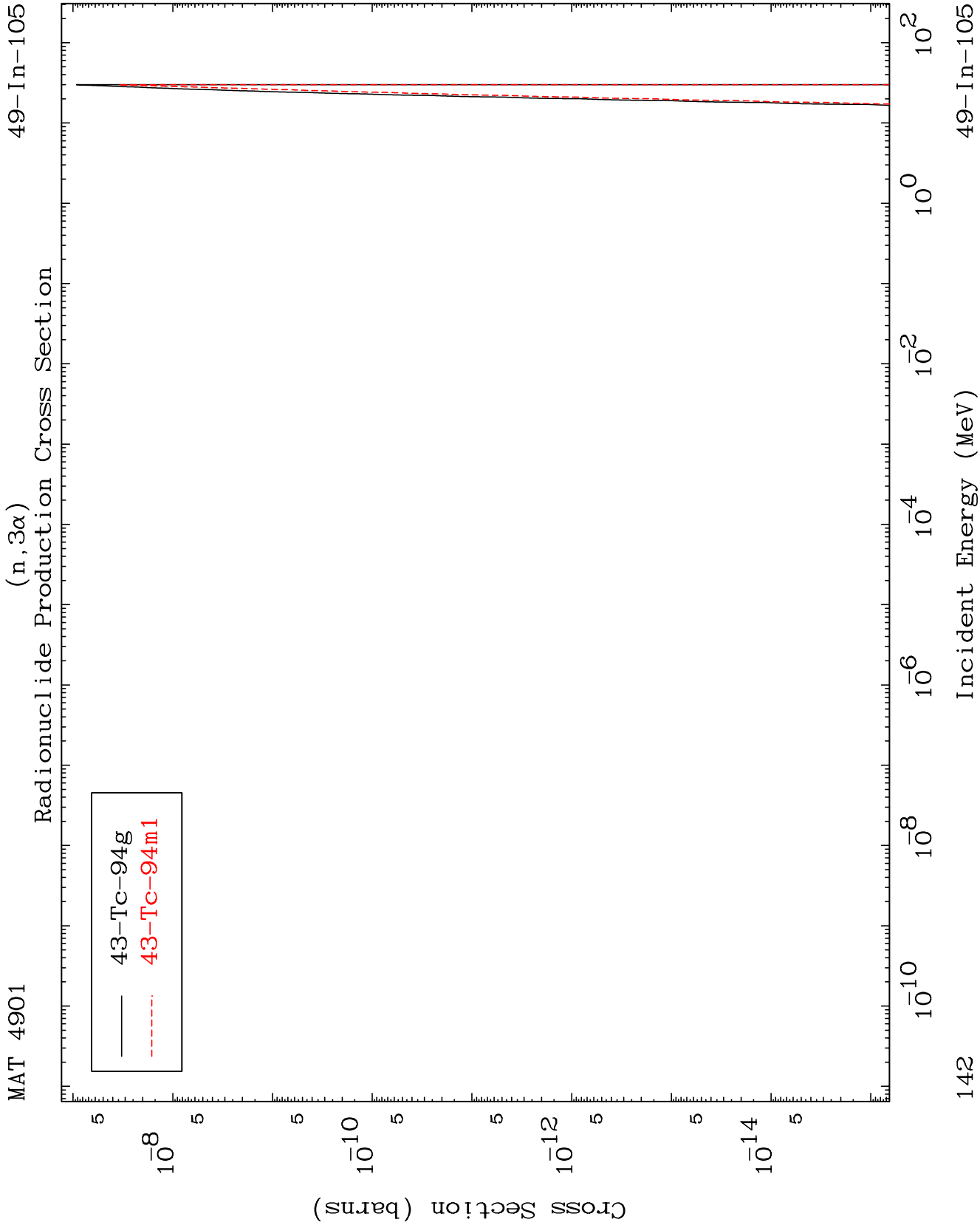
49-In-105

Incident Energy (MeV)





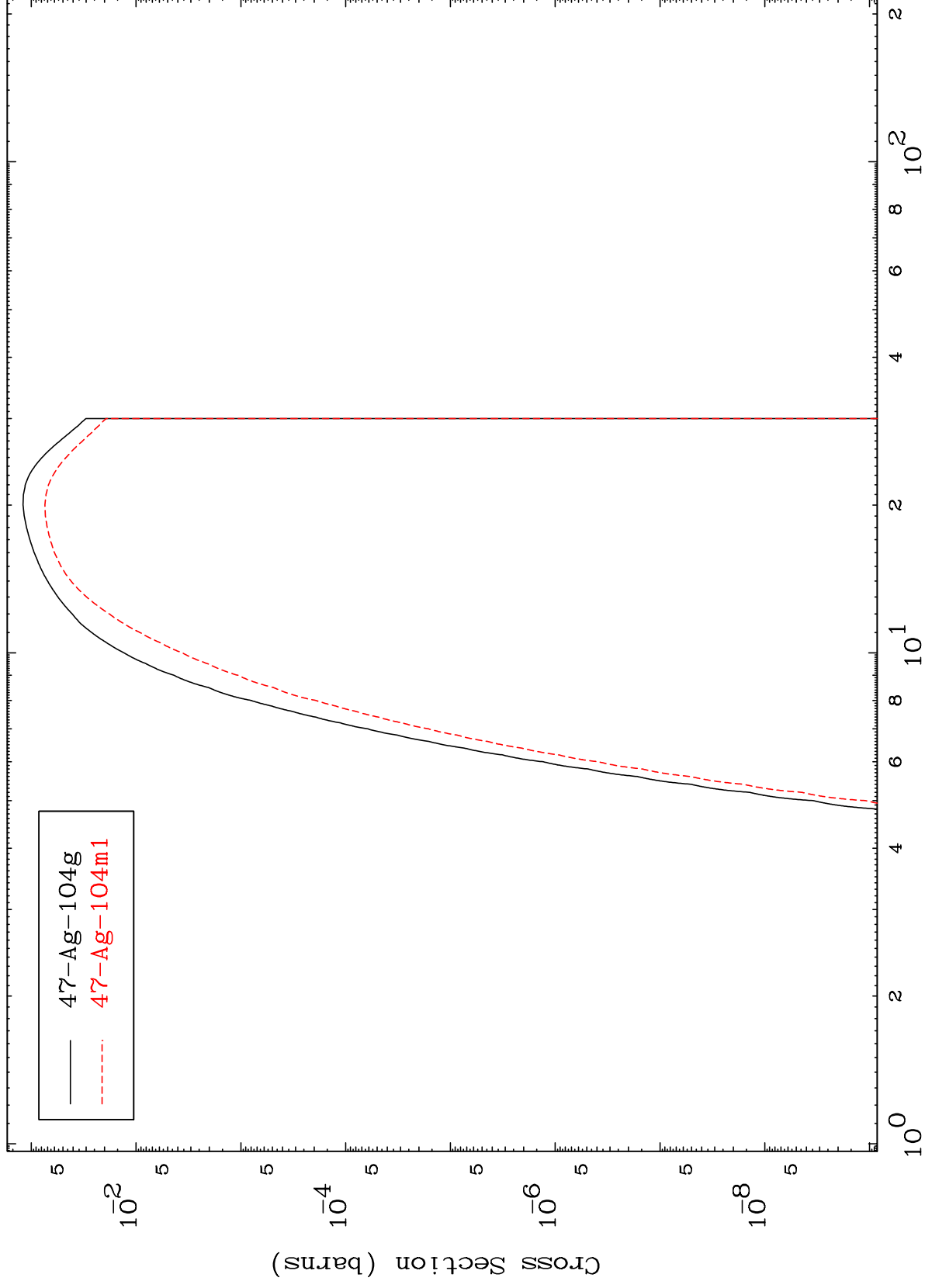




MAT 4901

49-In-105

Radionuclide Production Cross Section
(n,2p)



Incident Energy (MeV)

49-In-105

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