

Program EVALPLOT  
(Version 2021-1)

by

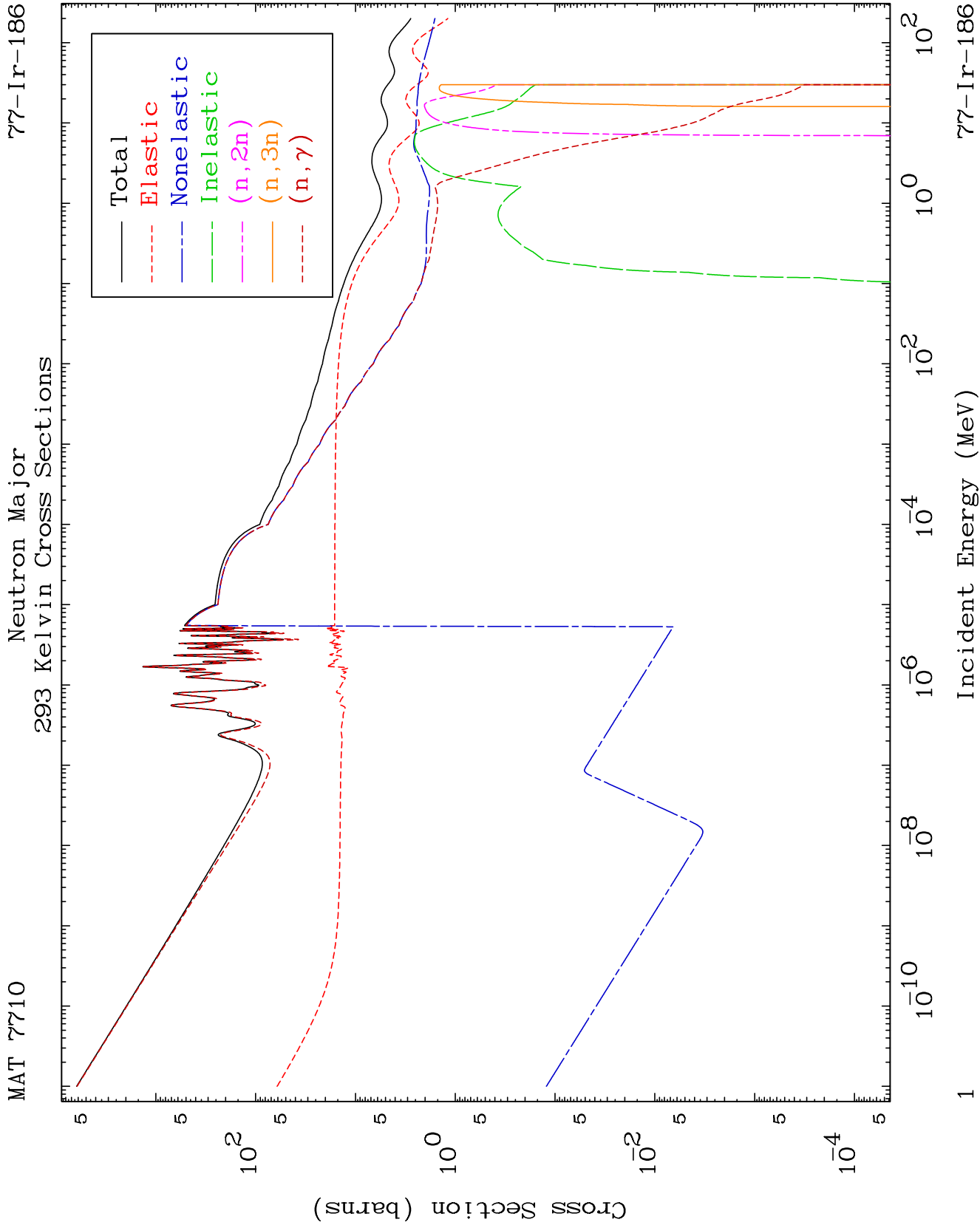
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Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

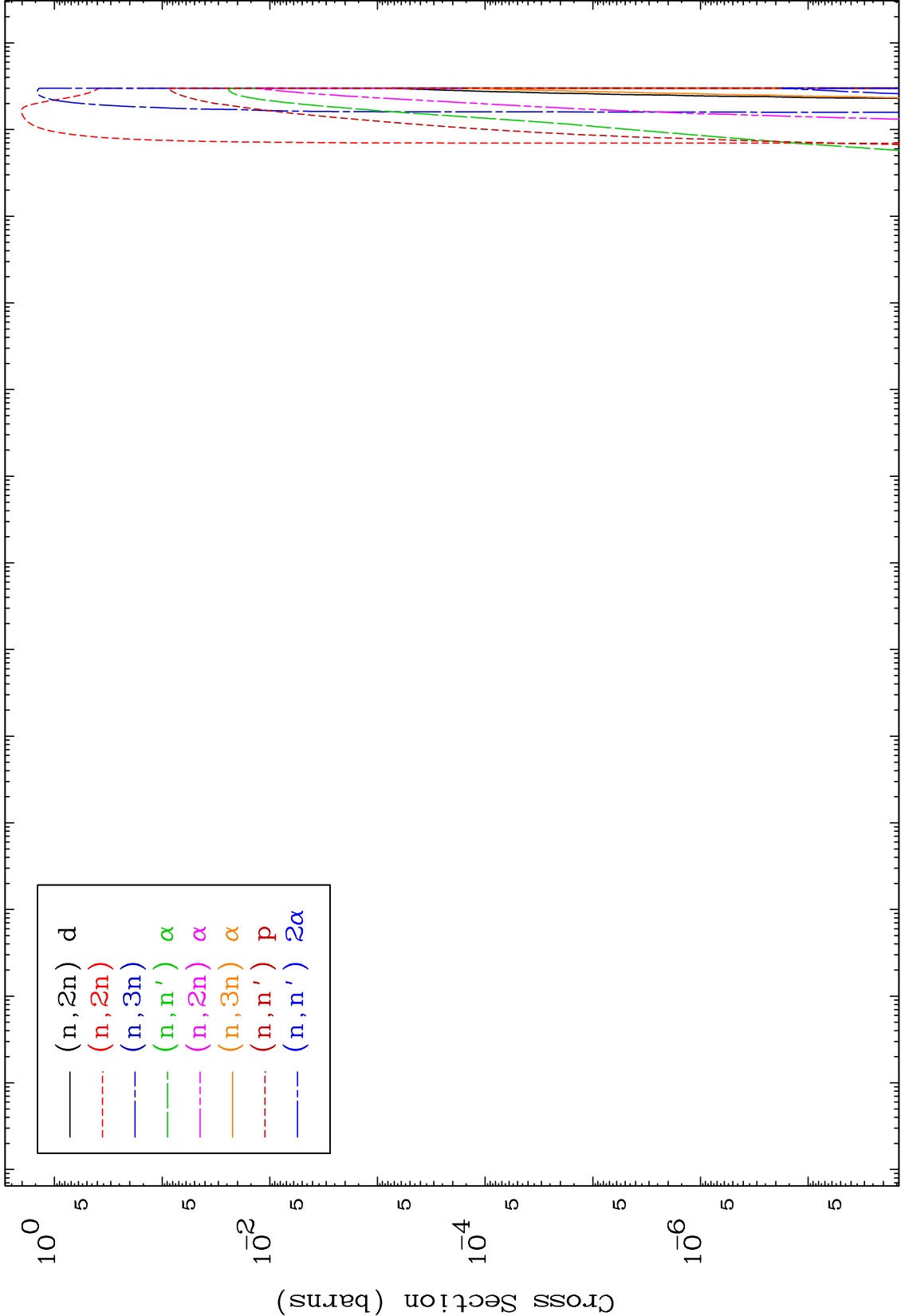
Press Mouse Button to Start

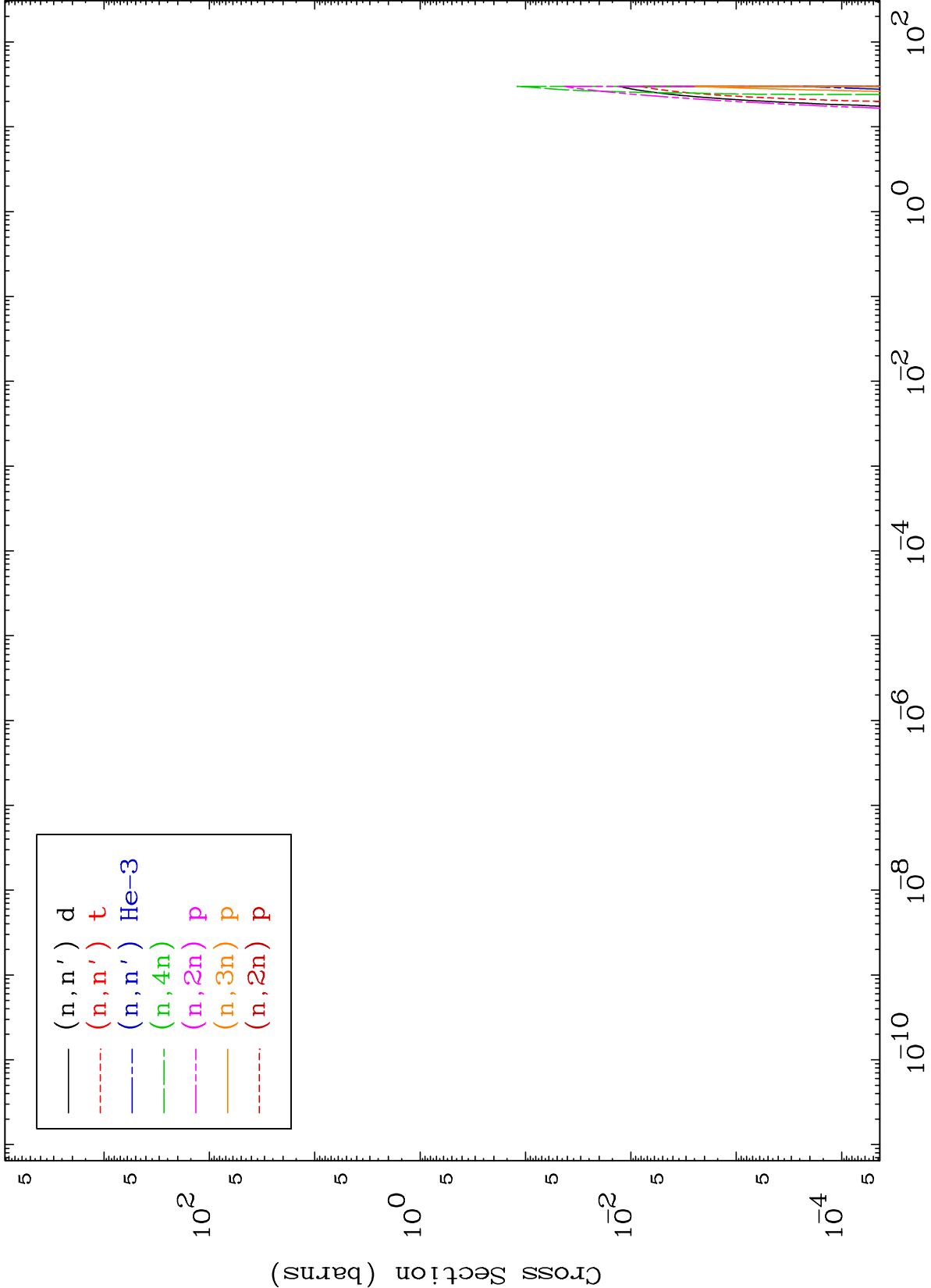


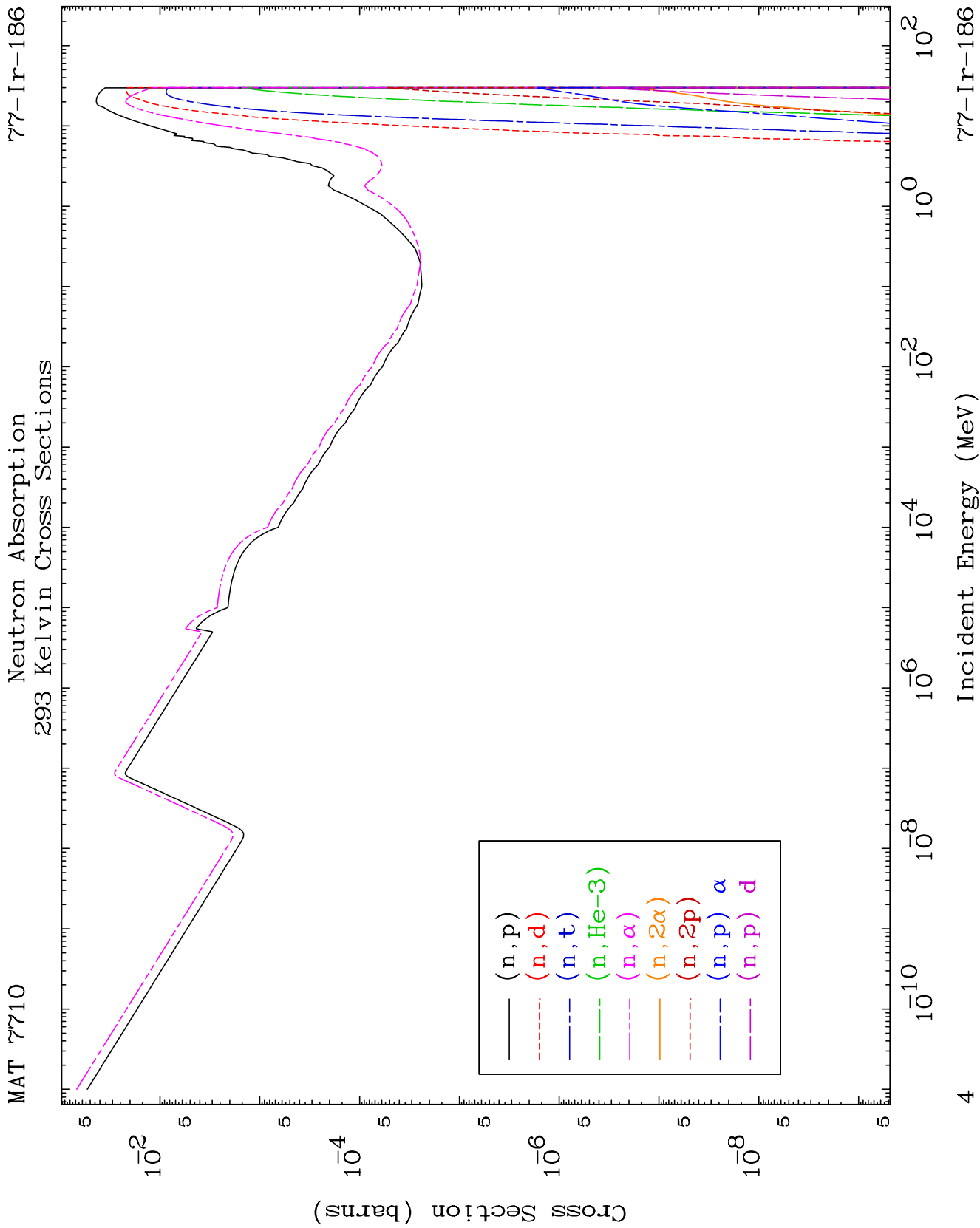
MAT 7710

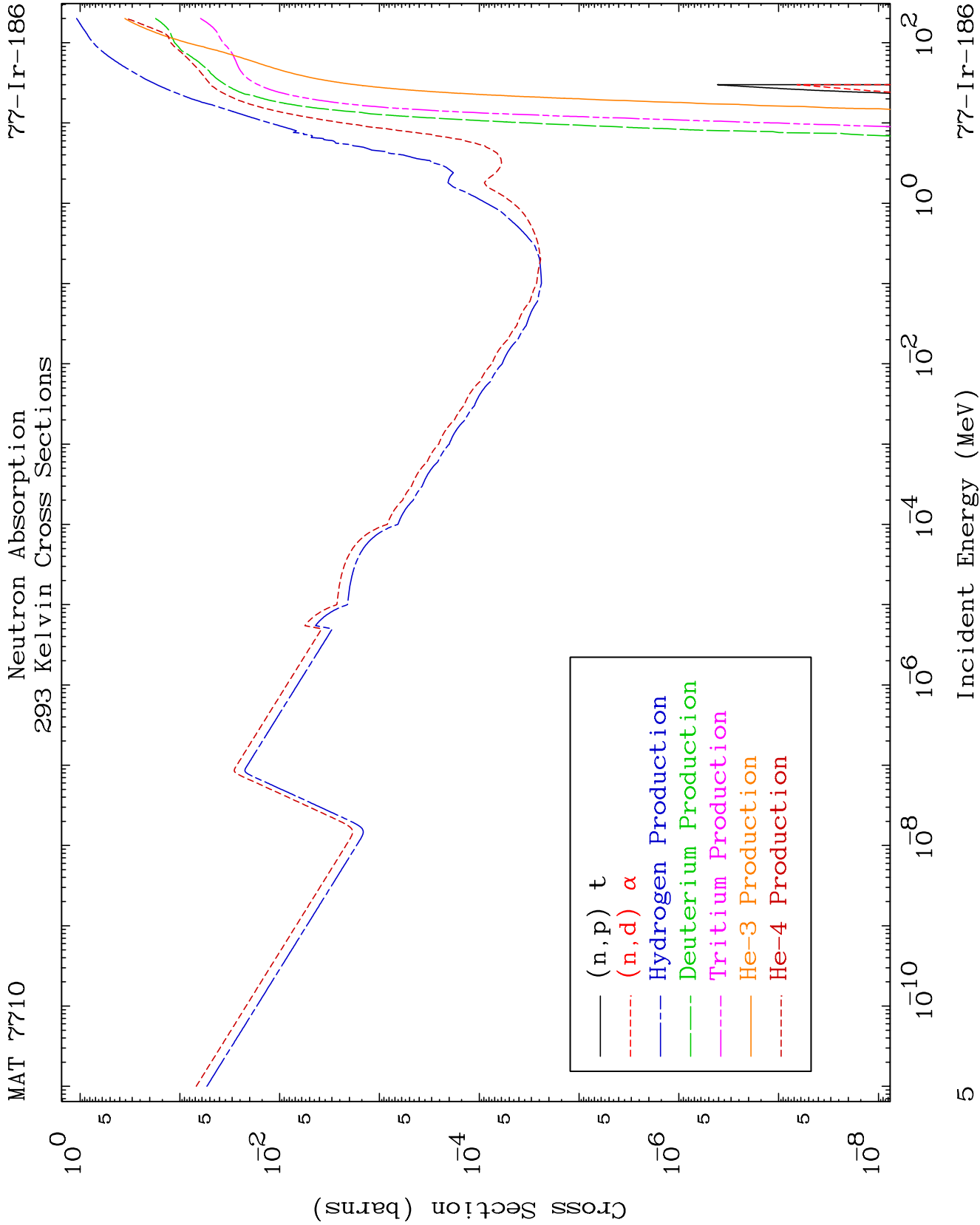
Neutron Absorption  
293 Kelvin Cross Sections

77-Ir-186





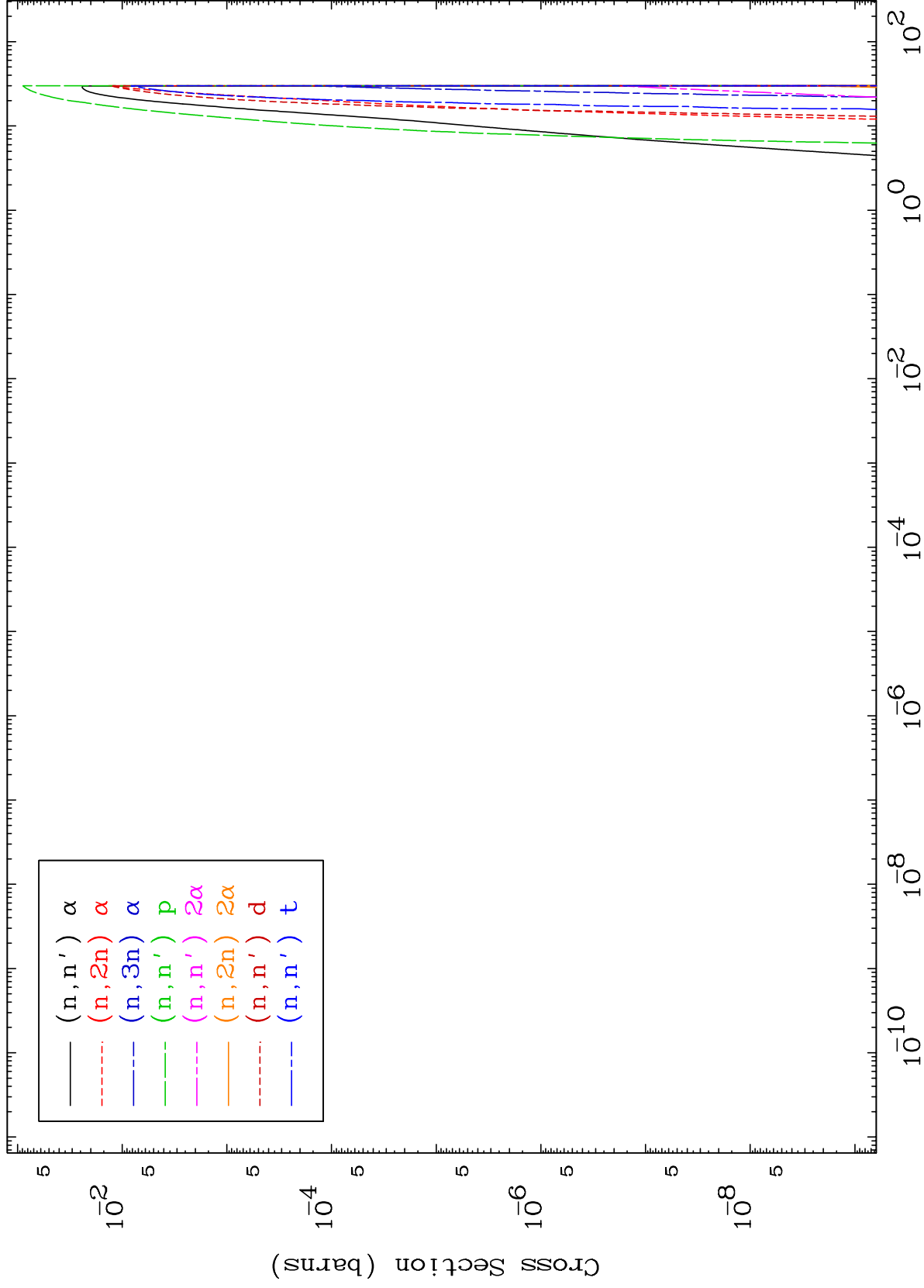


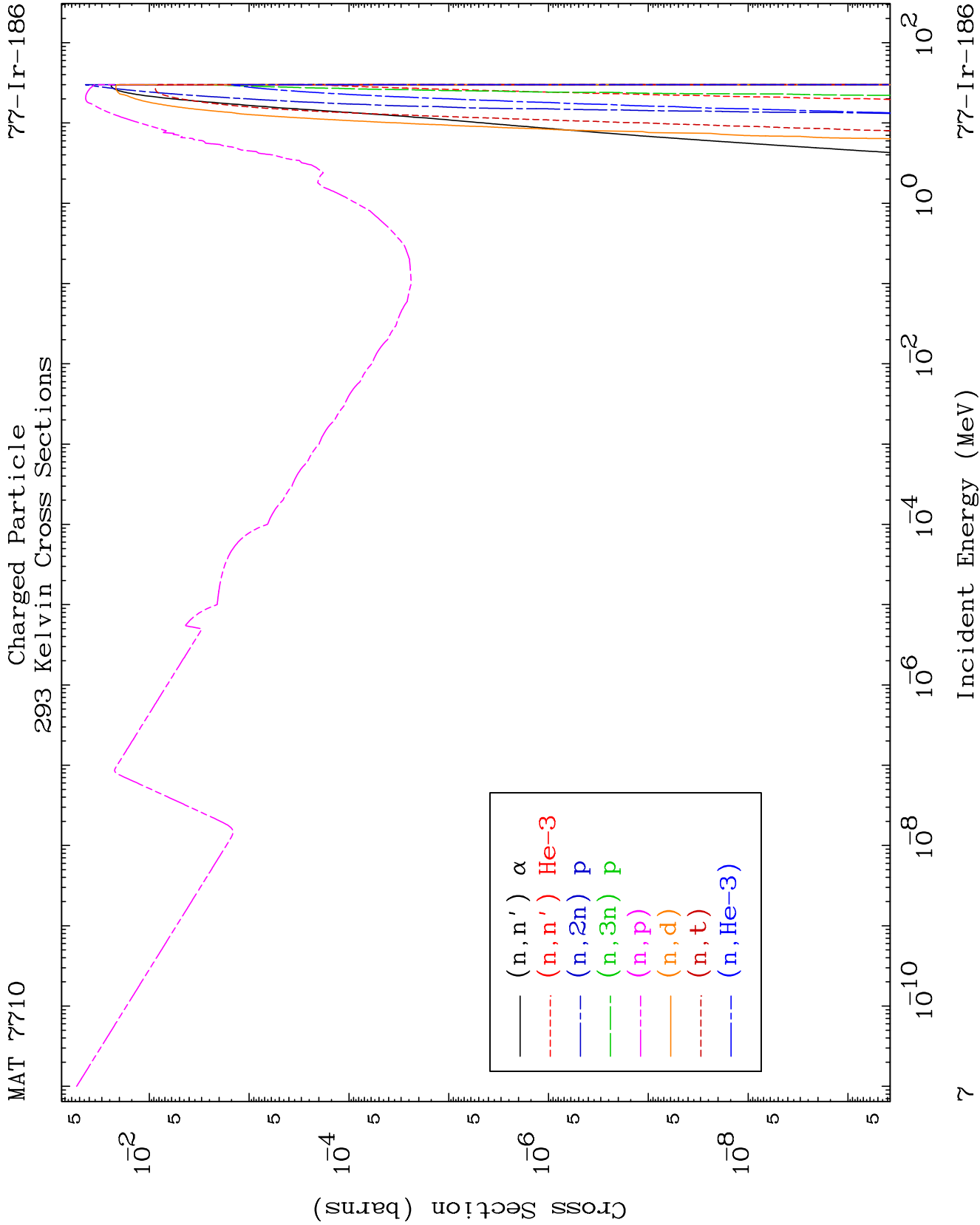


MAT 7710

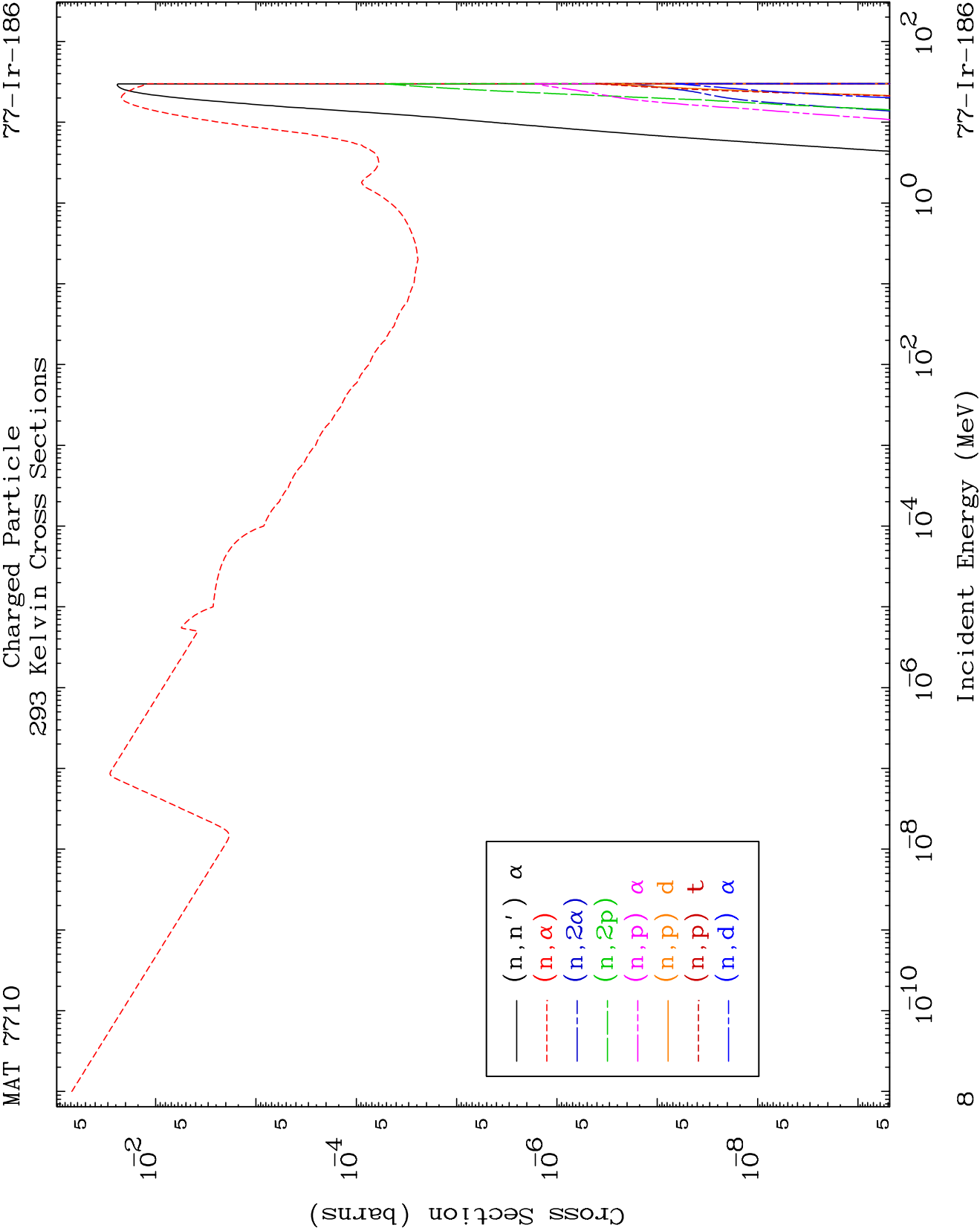
Charged Particle  
293 Kelvin Cross Sections

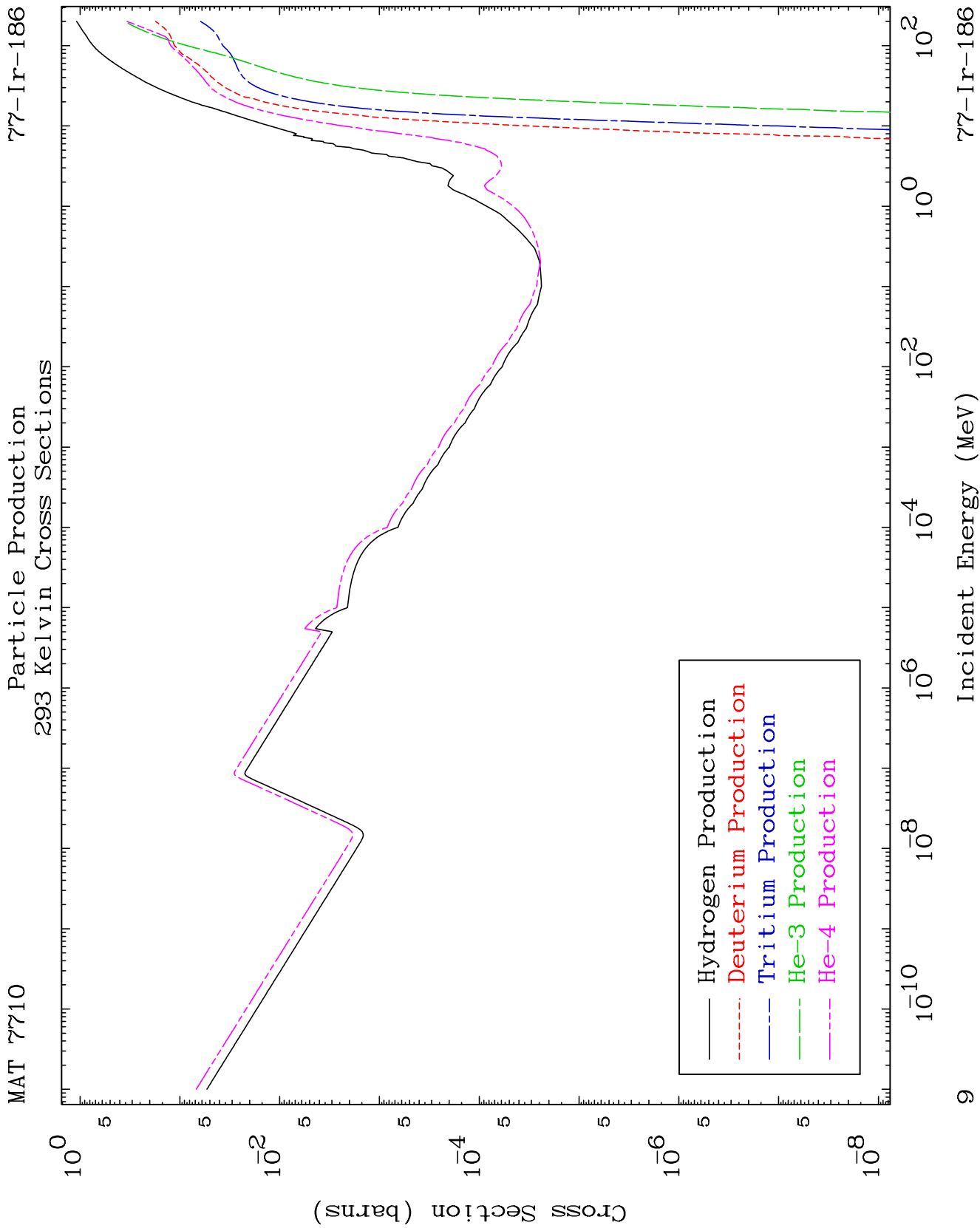
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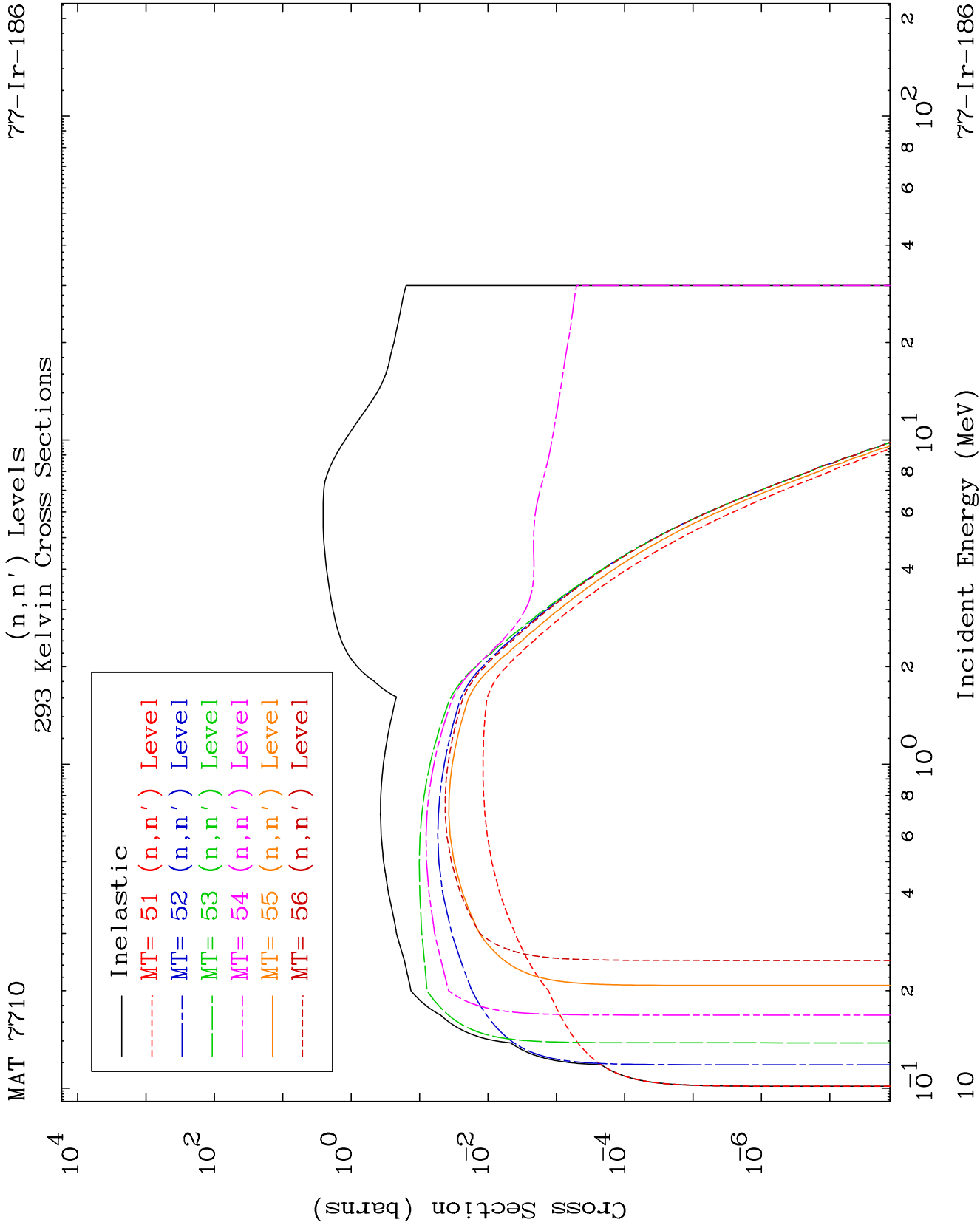


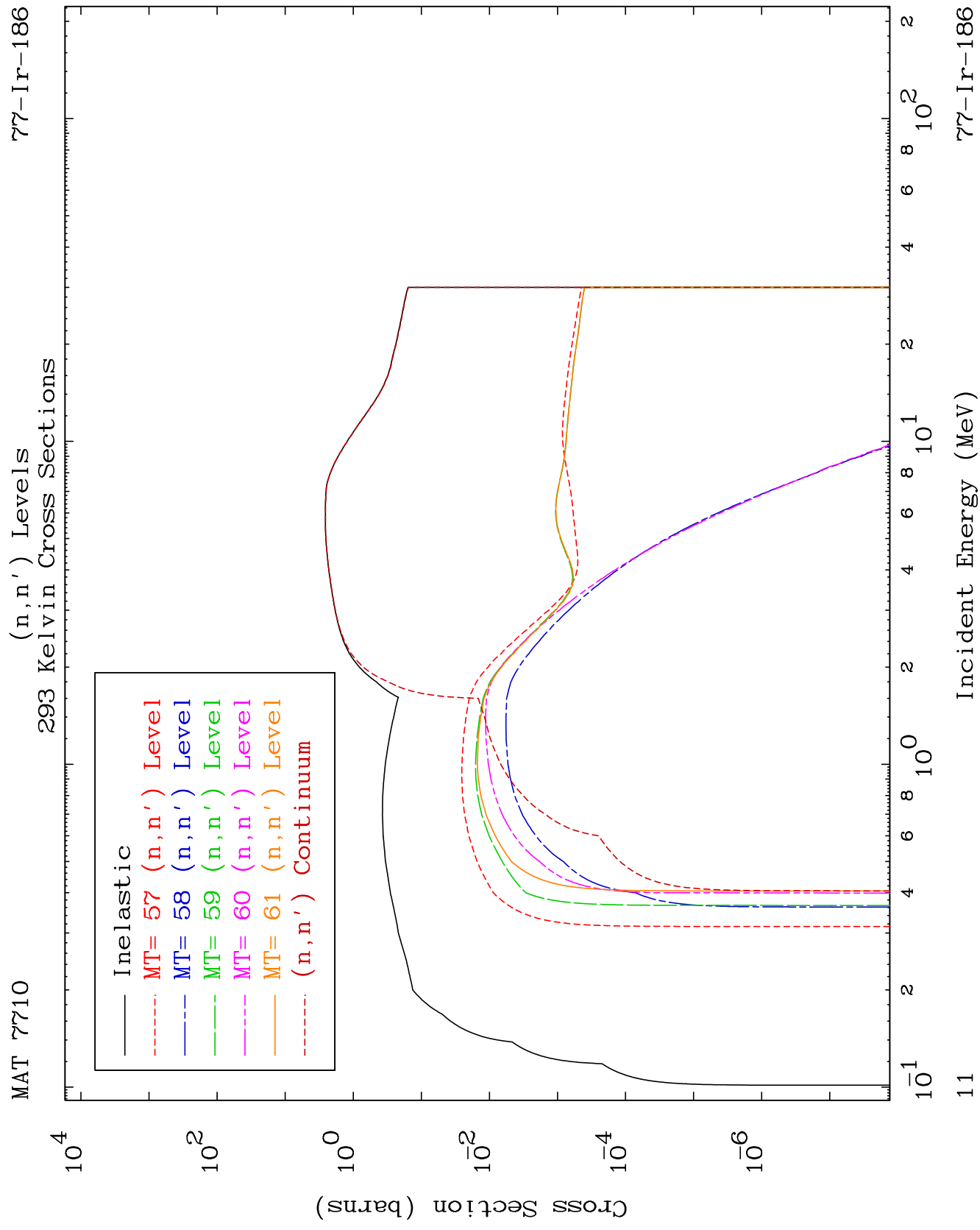


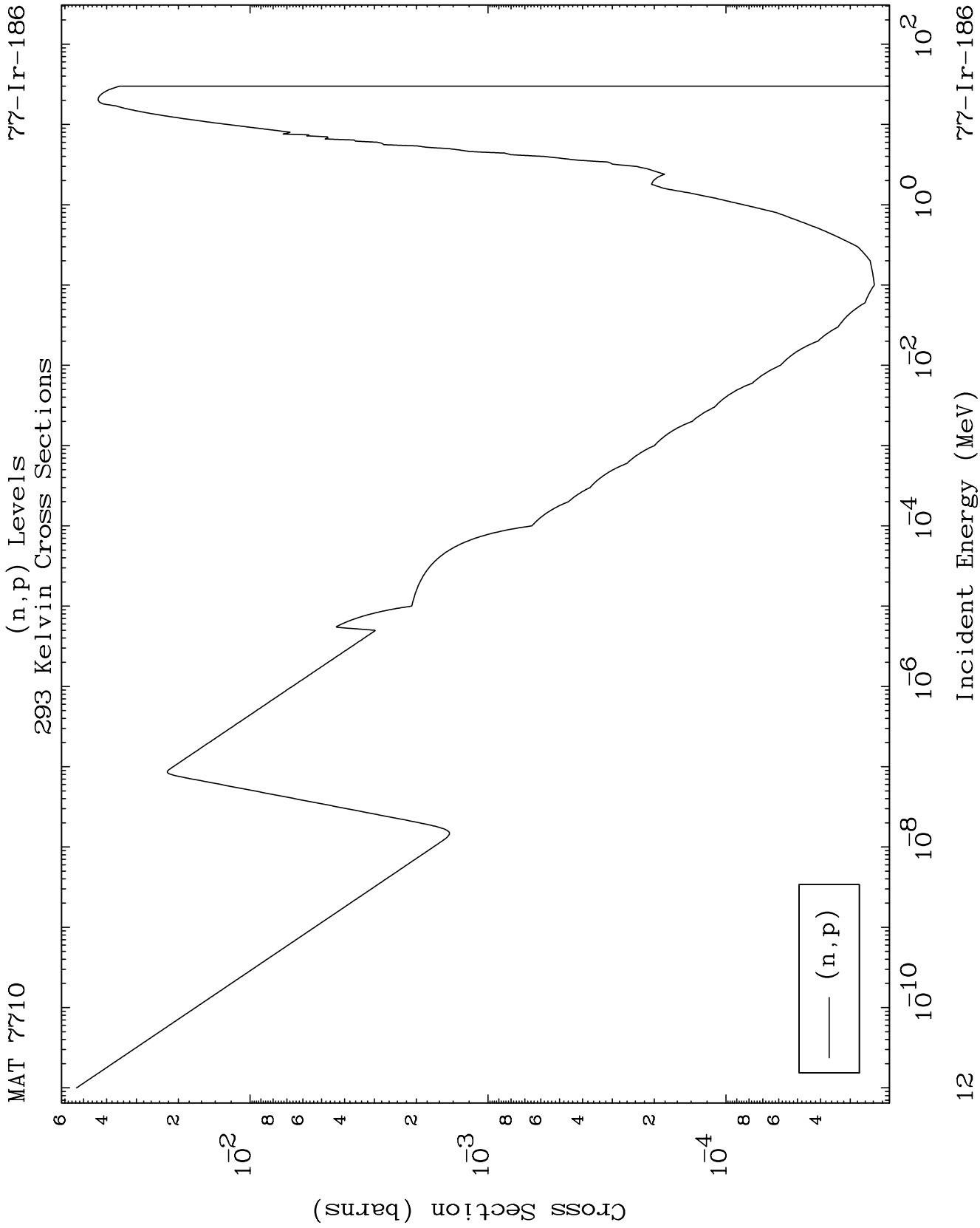








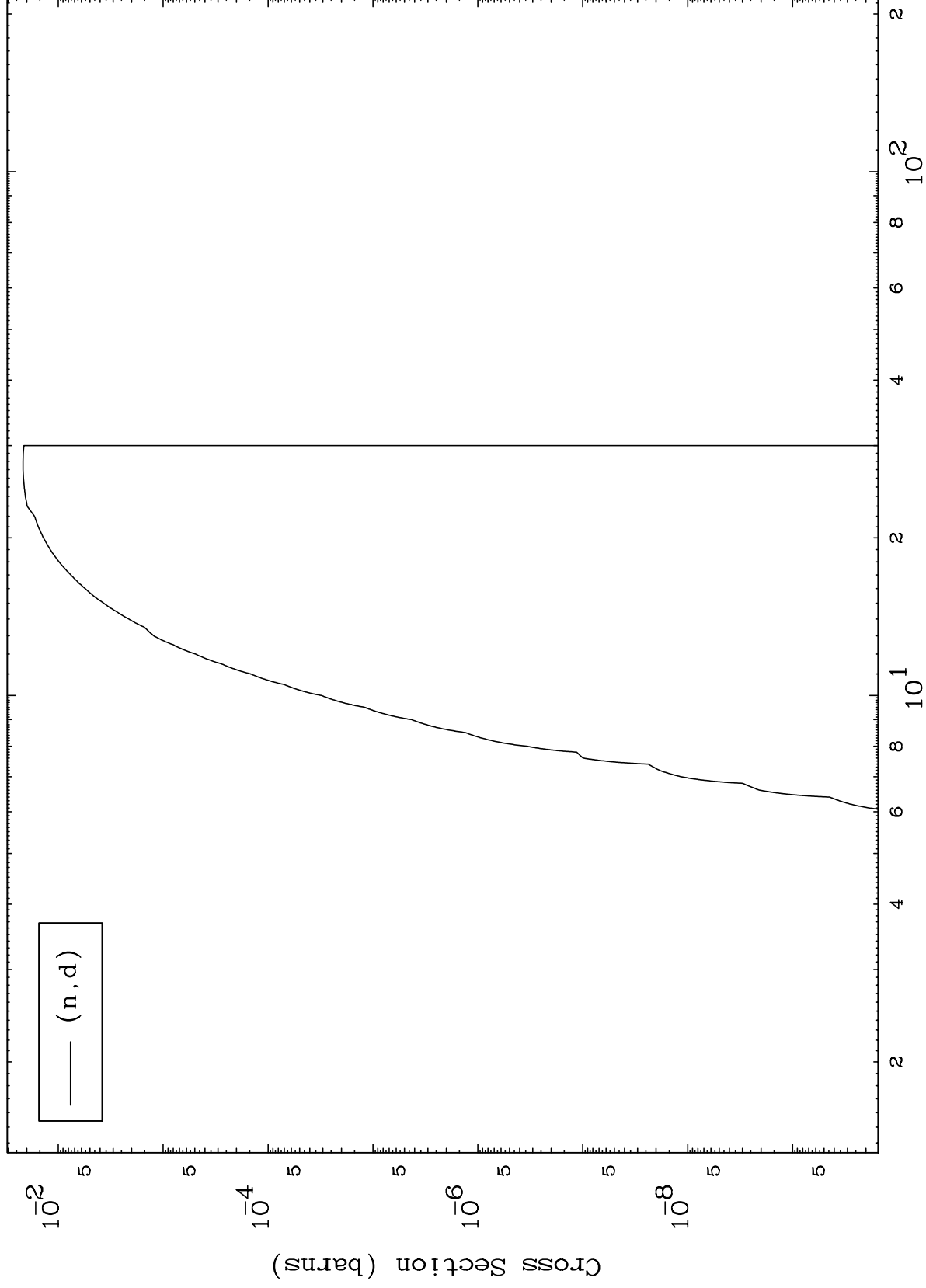




MAT 7710

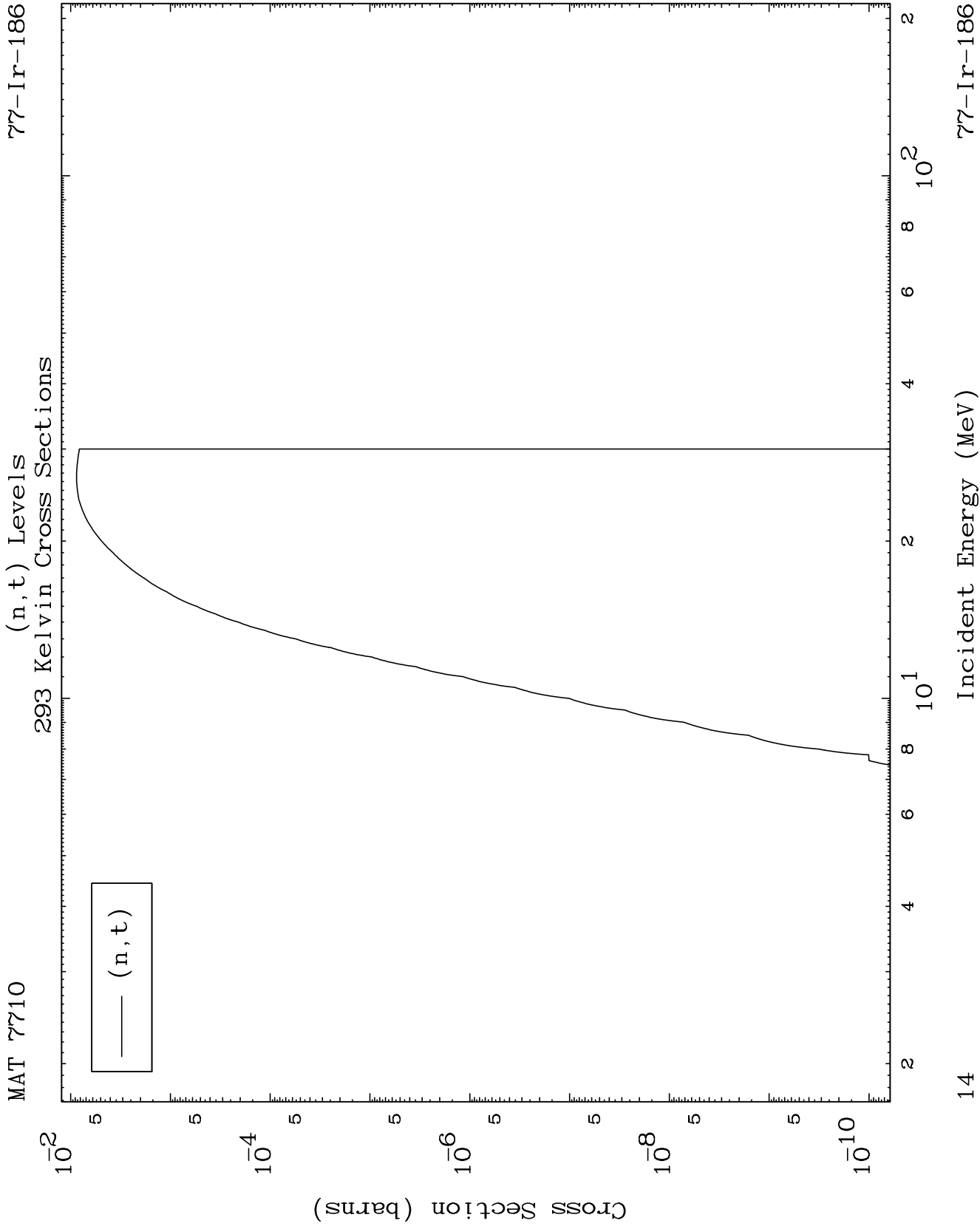
77-Ir-186

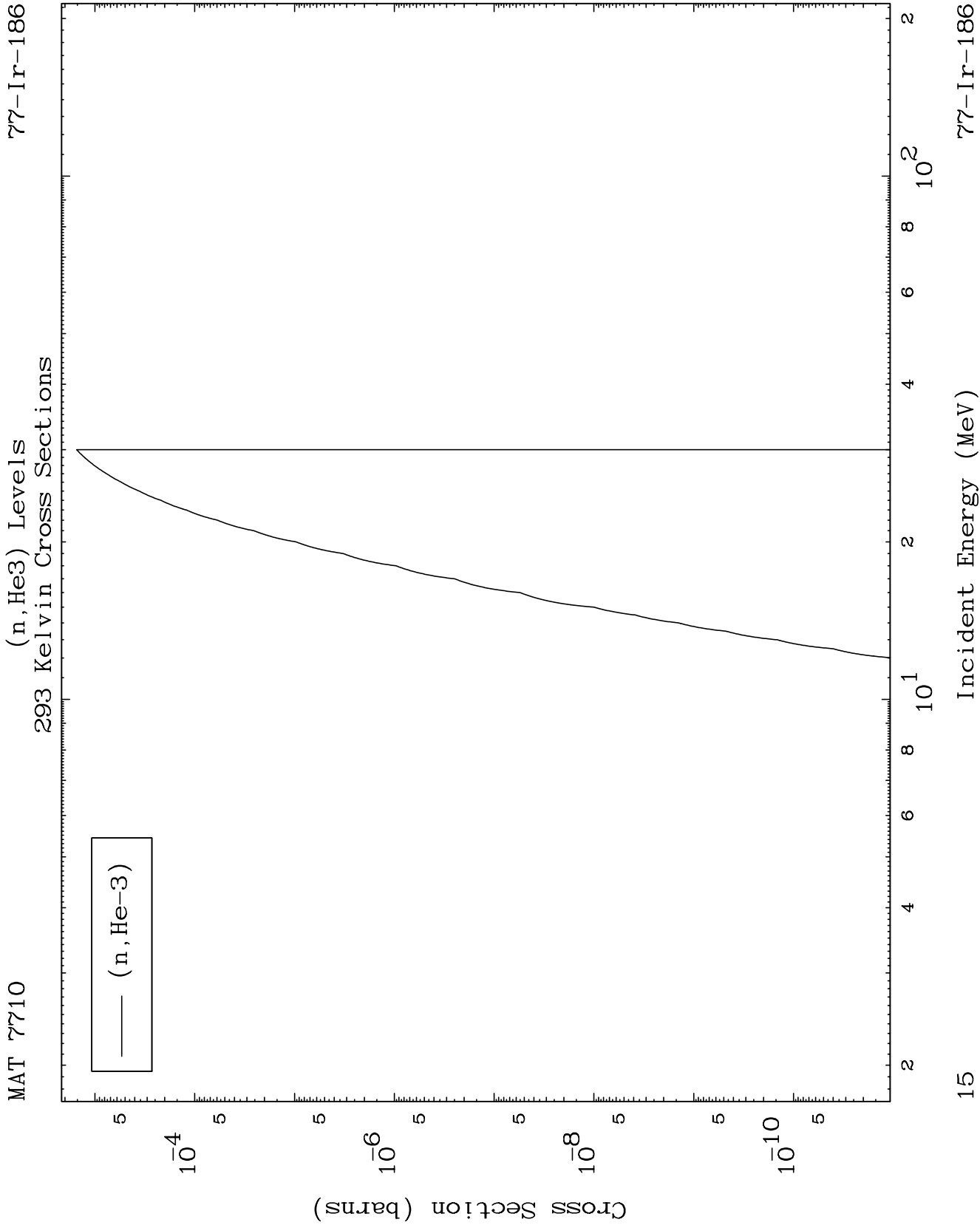
(n,d) Levels  
293 Kelvin Cross Sections



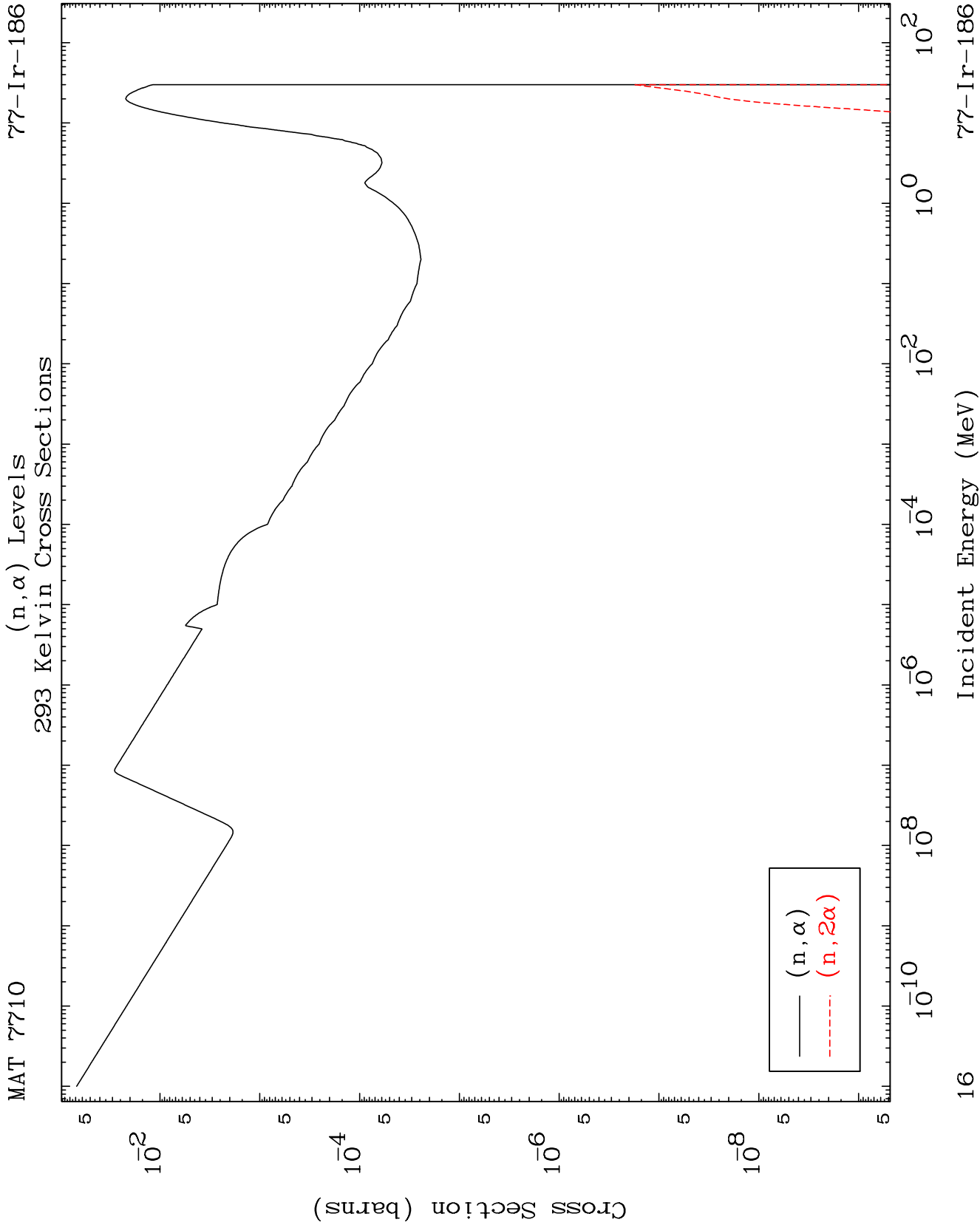
13

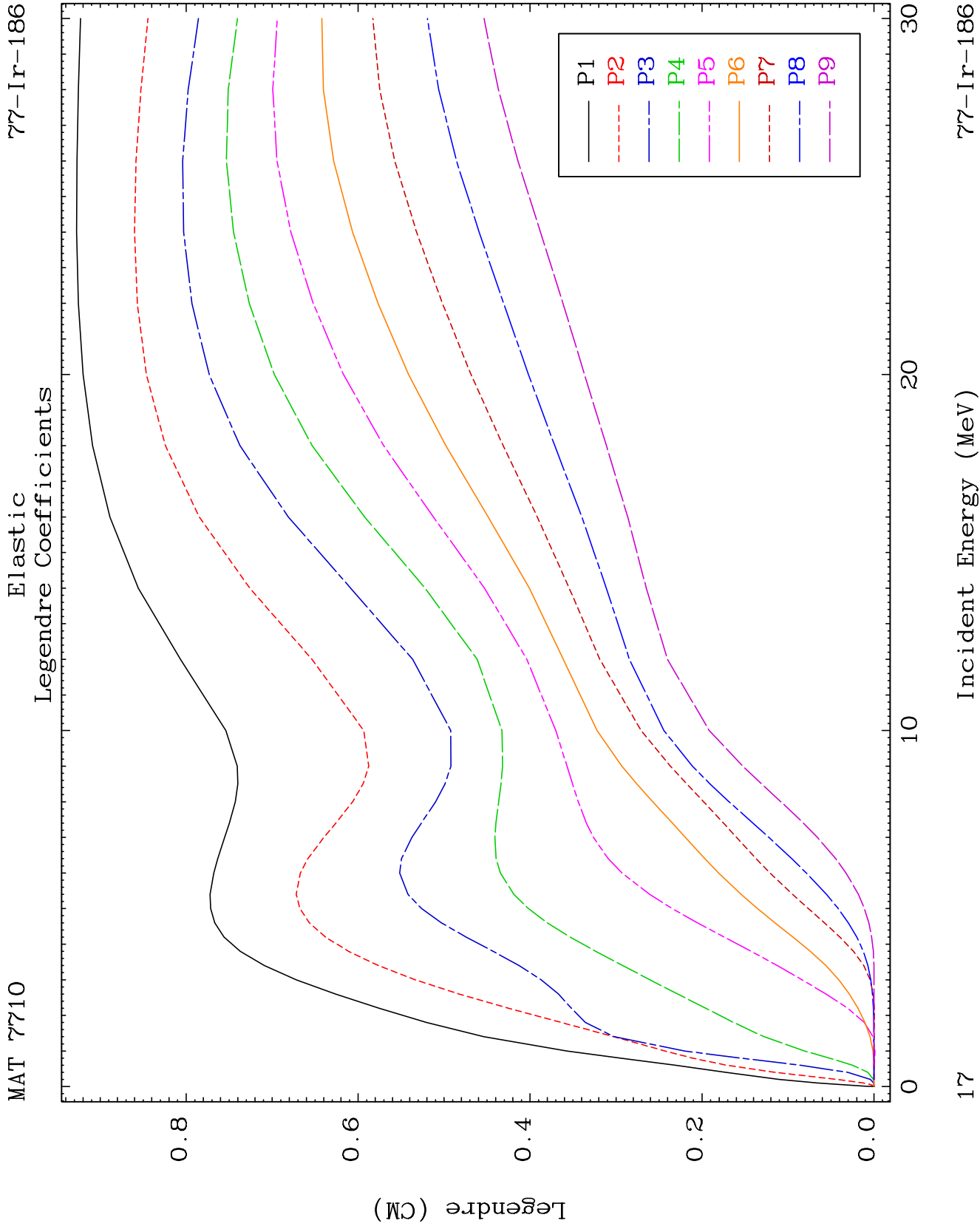
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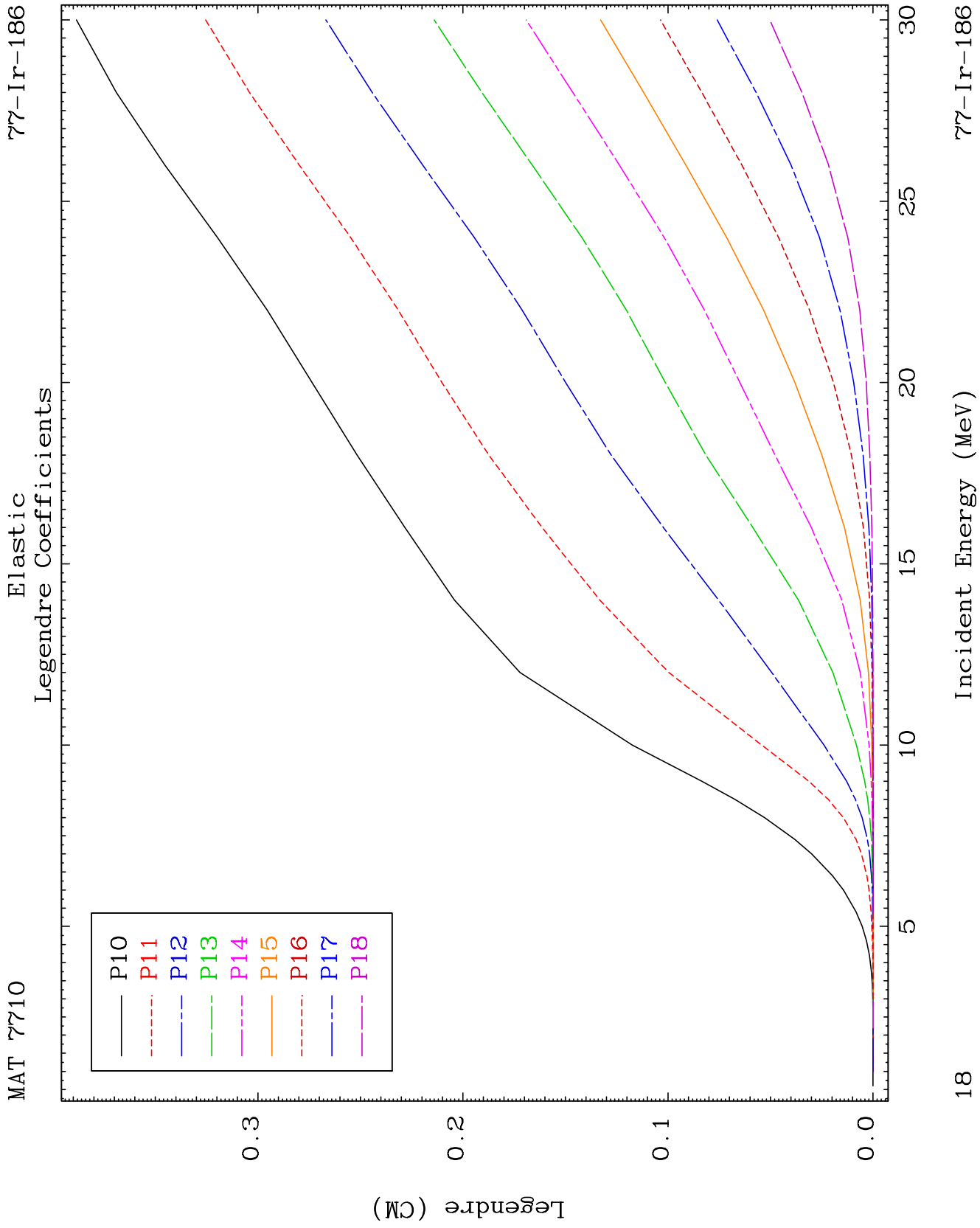


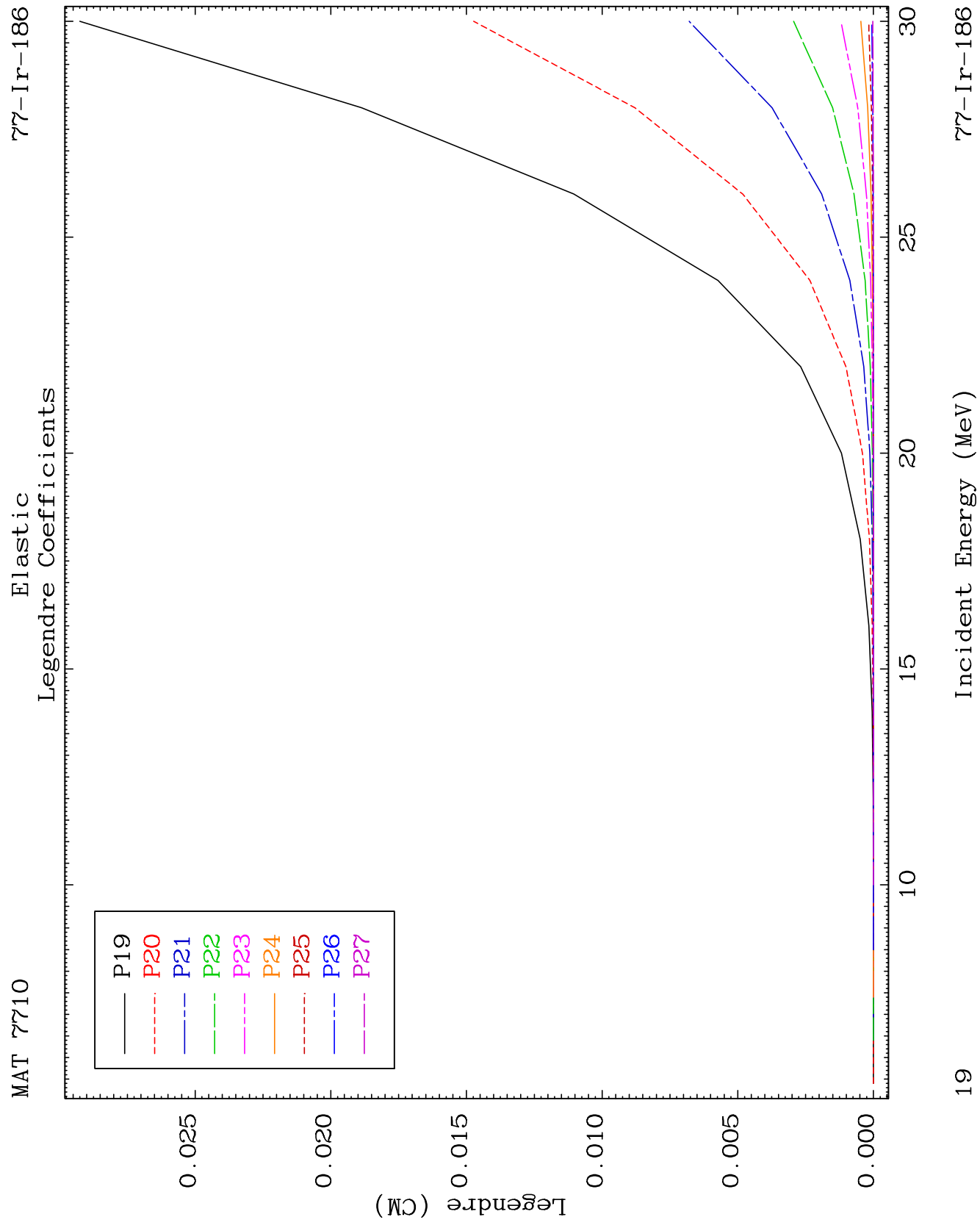


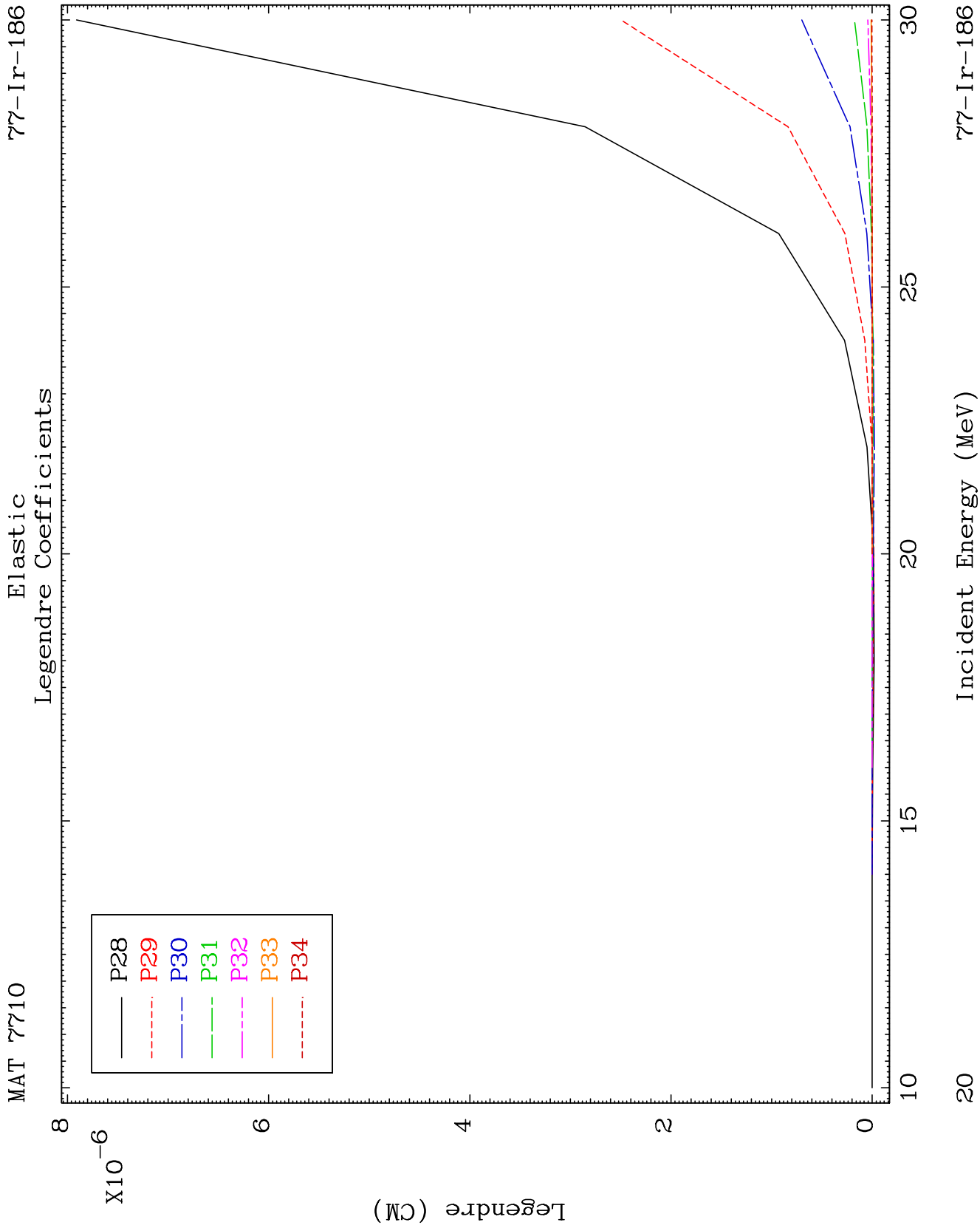








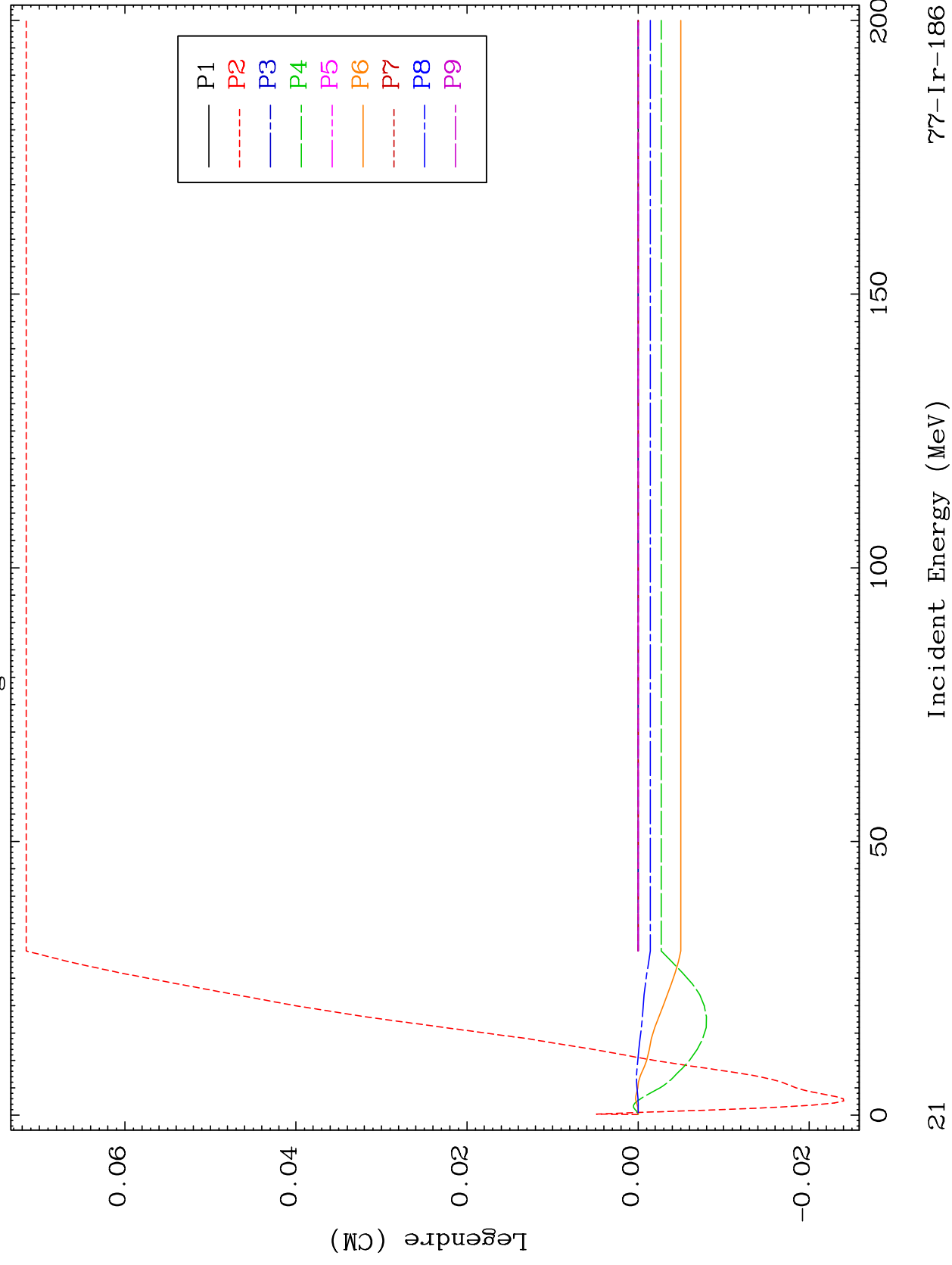


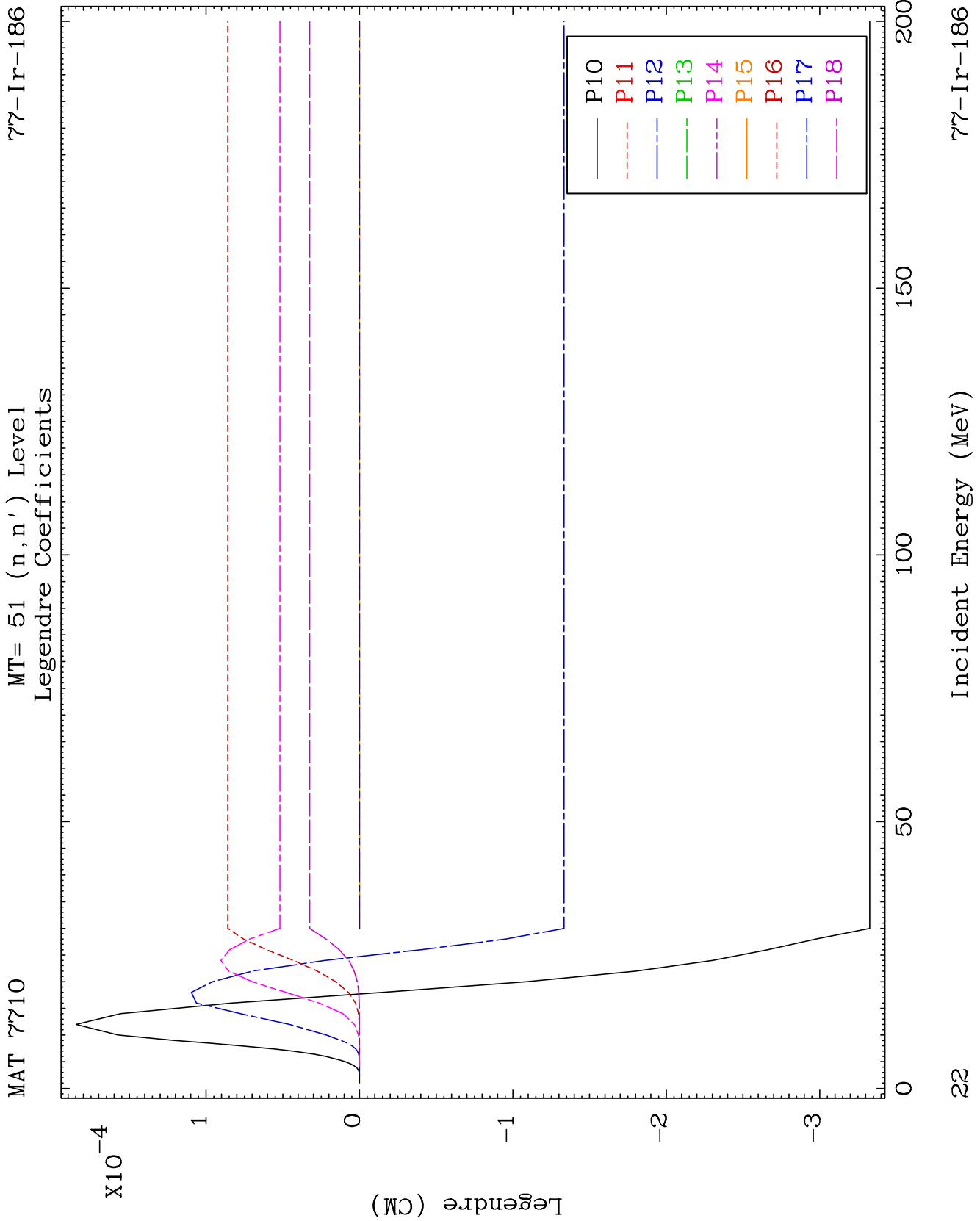


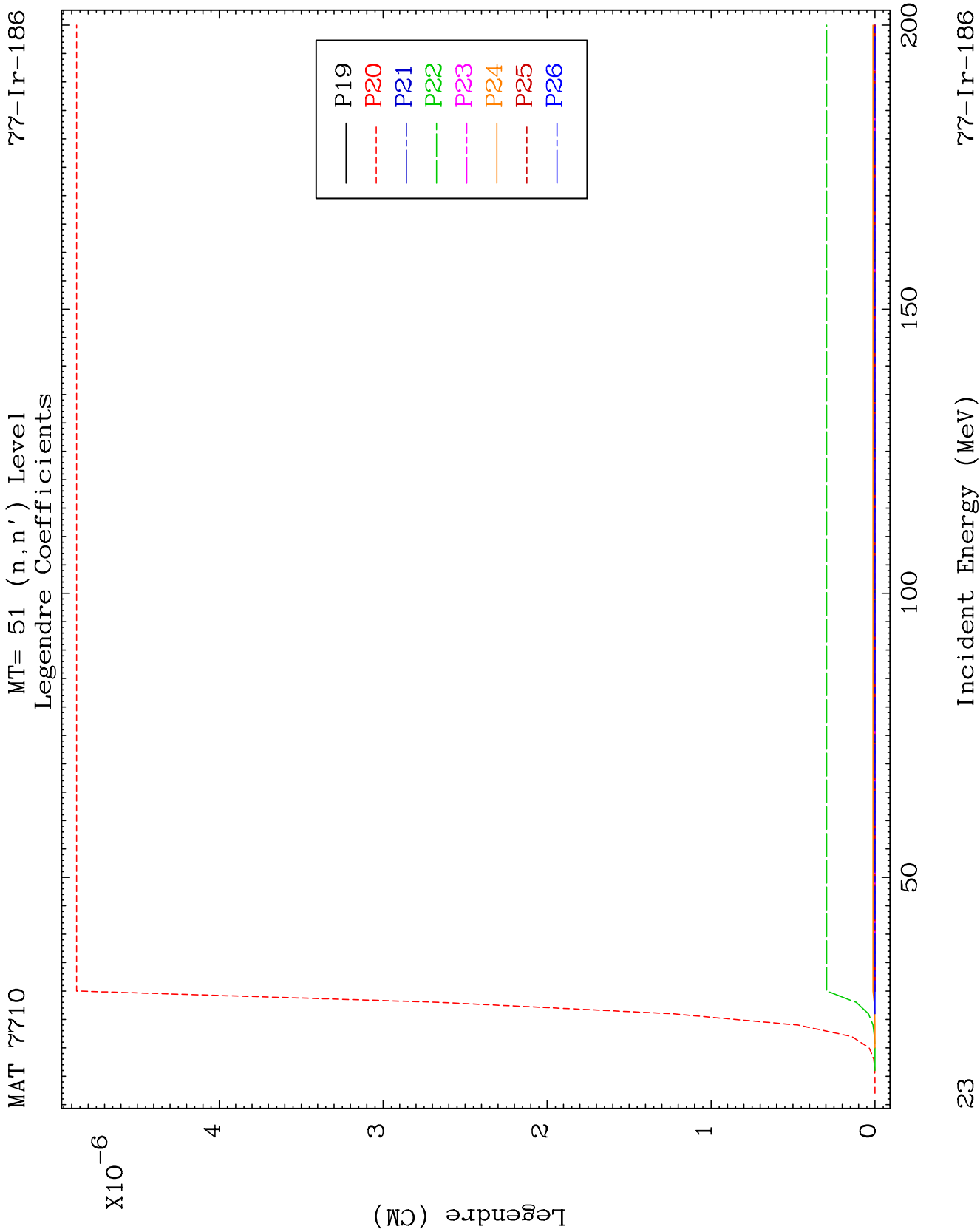
MAT 7710

MT= 51 (n, n') Level  
Legendre Coefficients

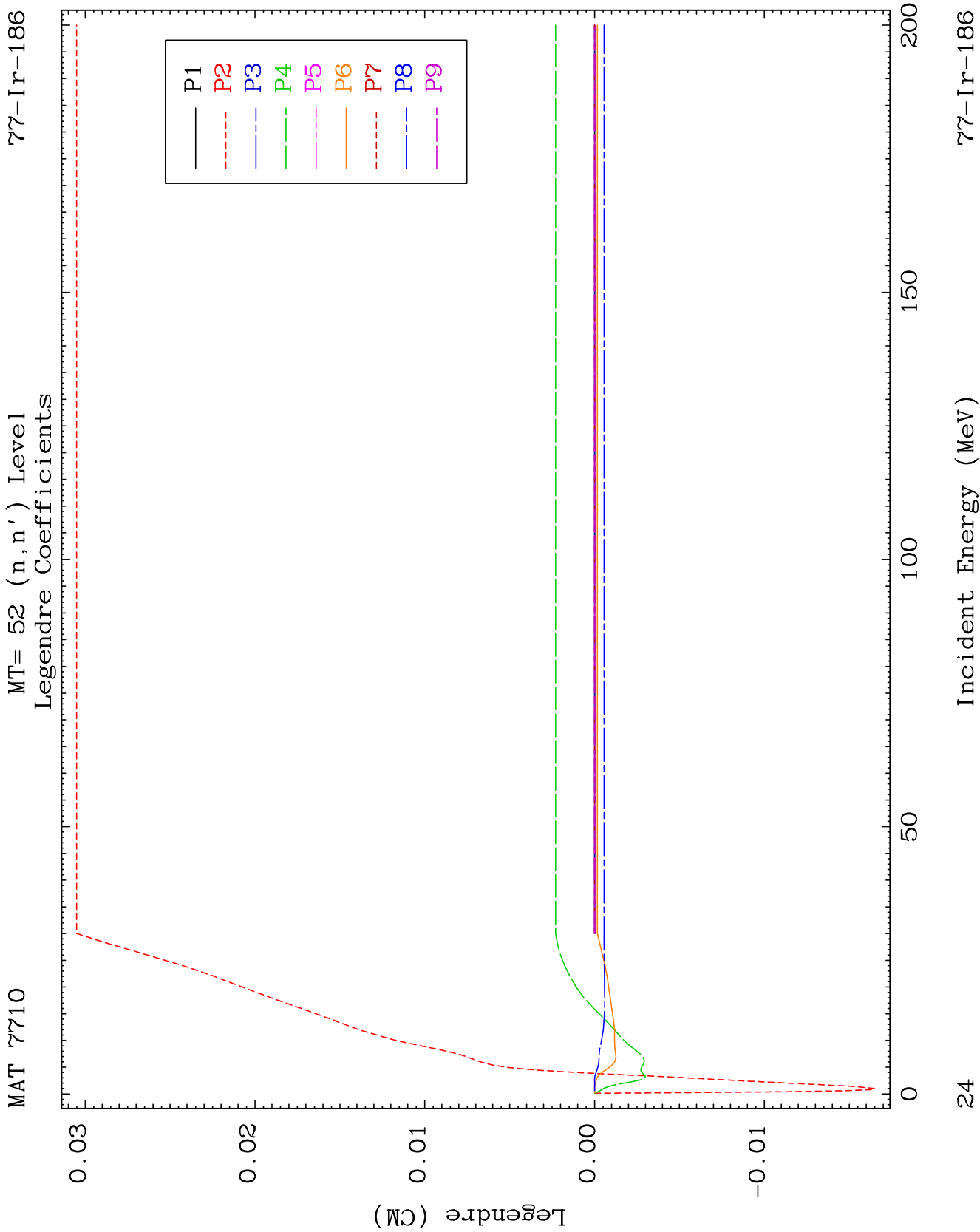
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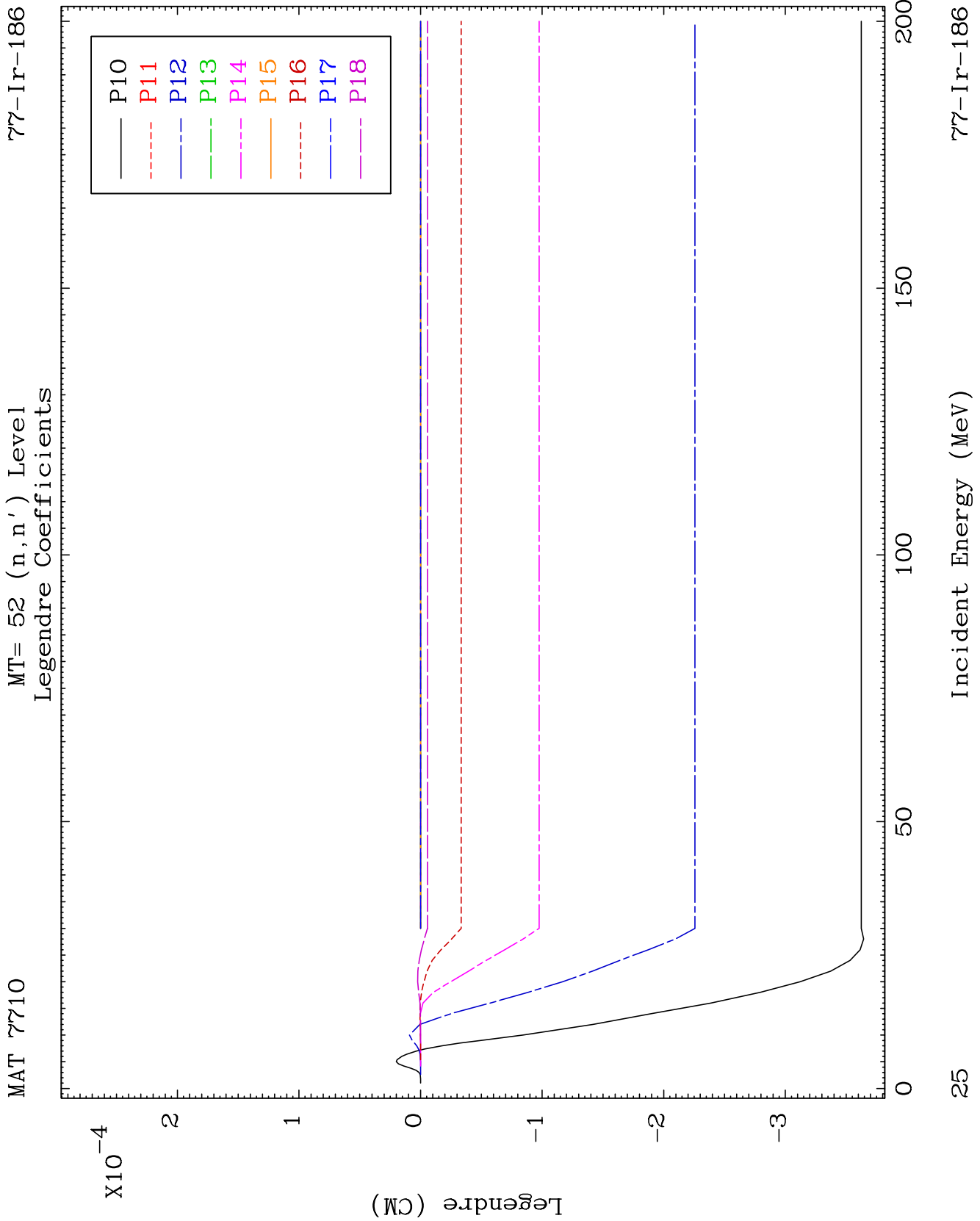








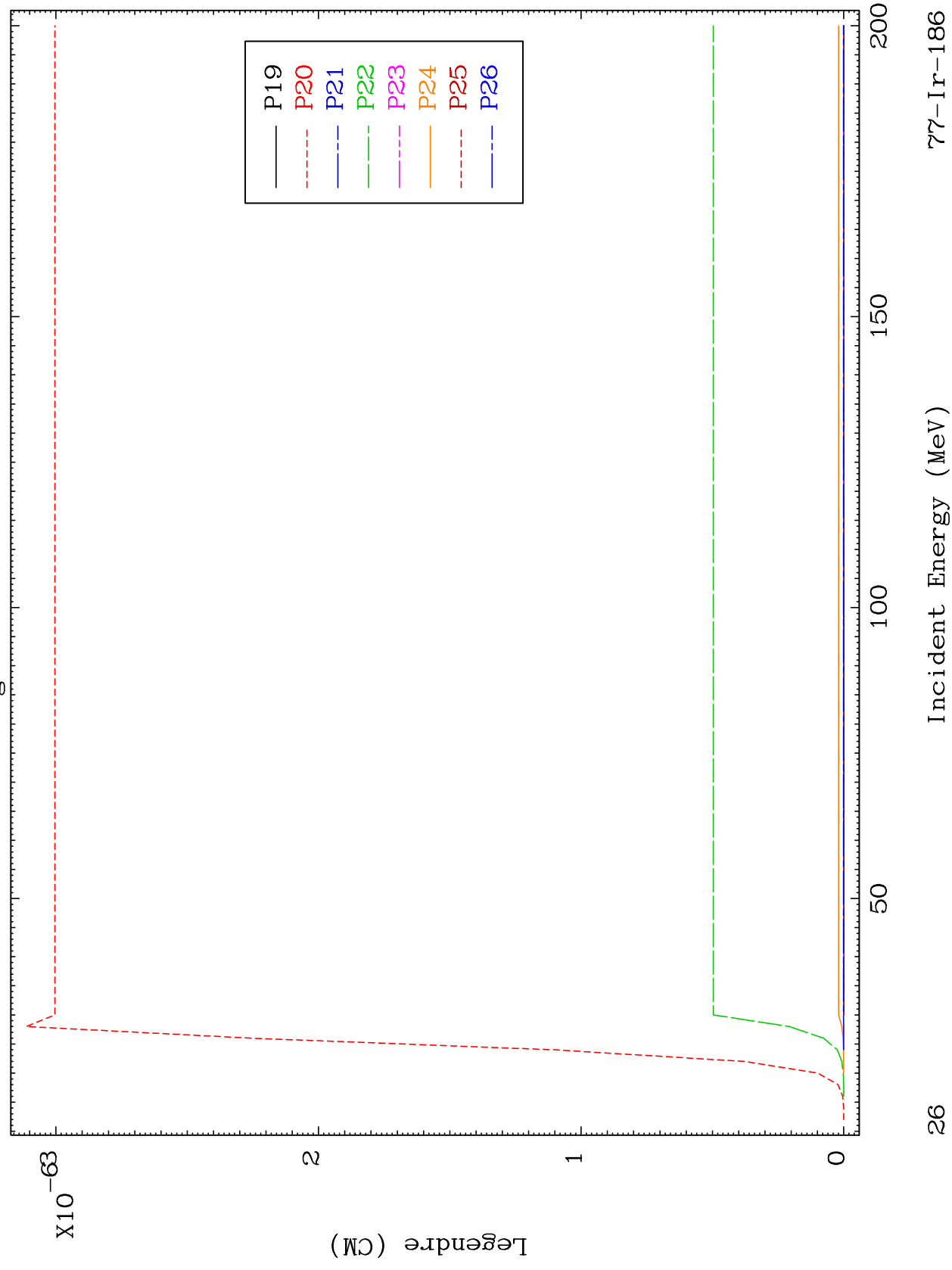


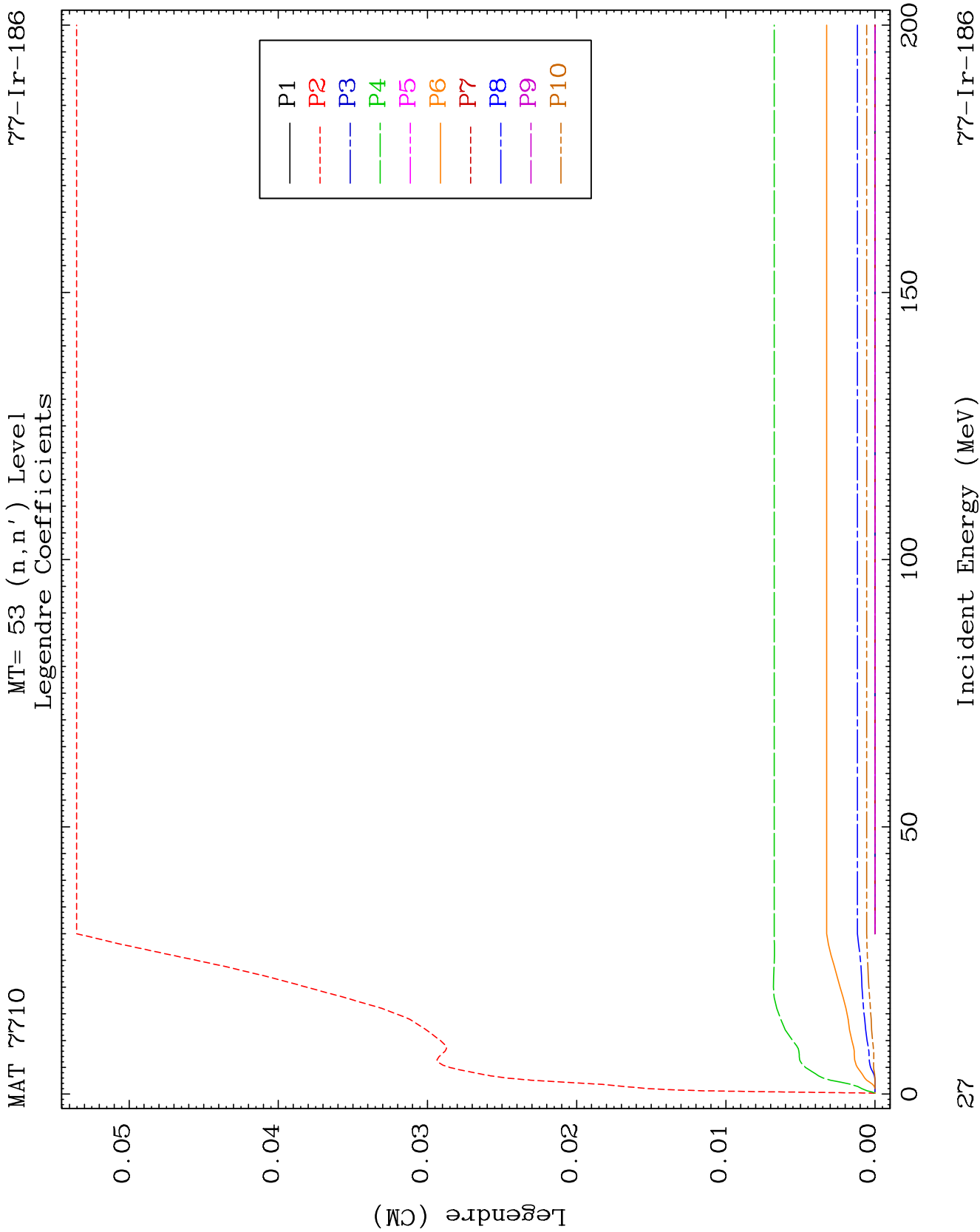


MAT 7710

MT= 52 (n, n') Level  
Legendre Coefficients

77-Ir-186

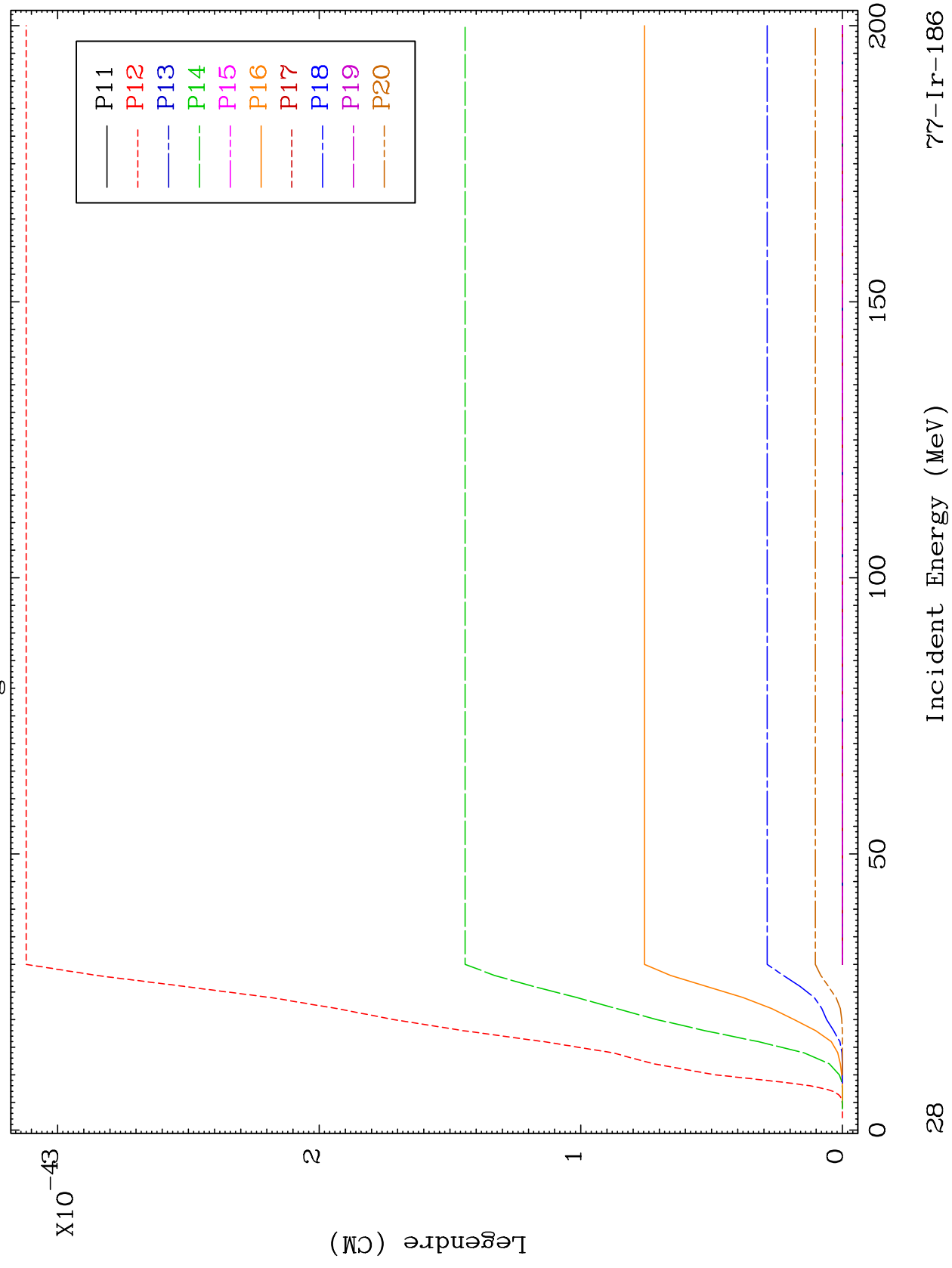


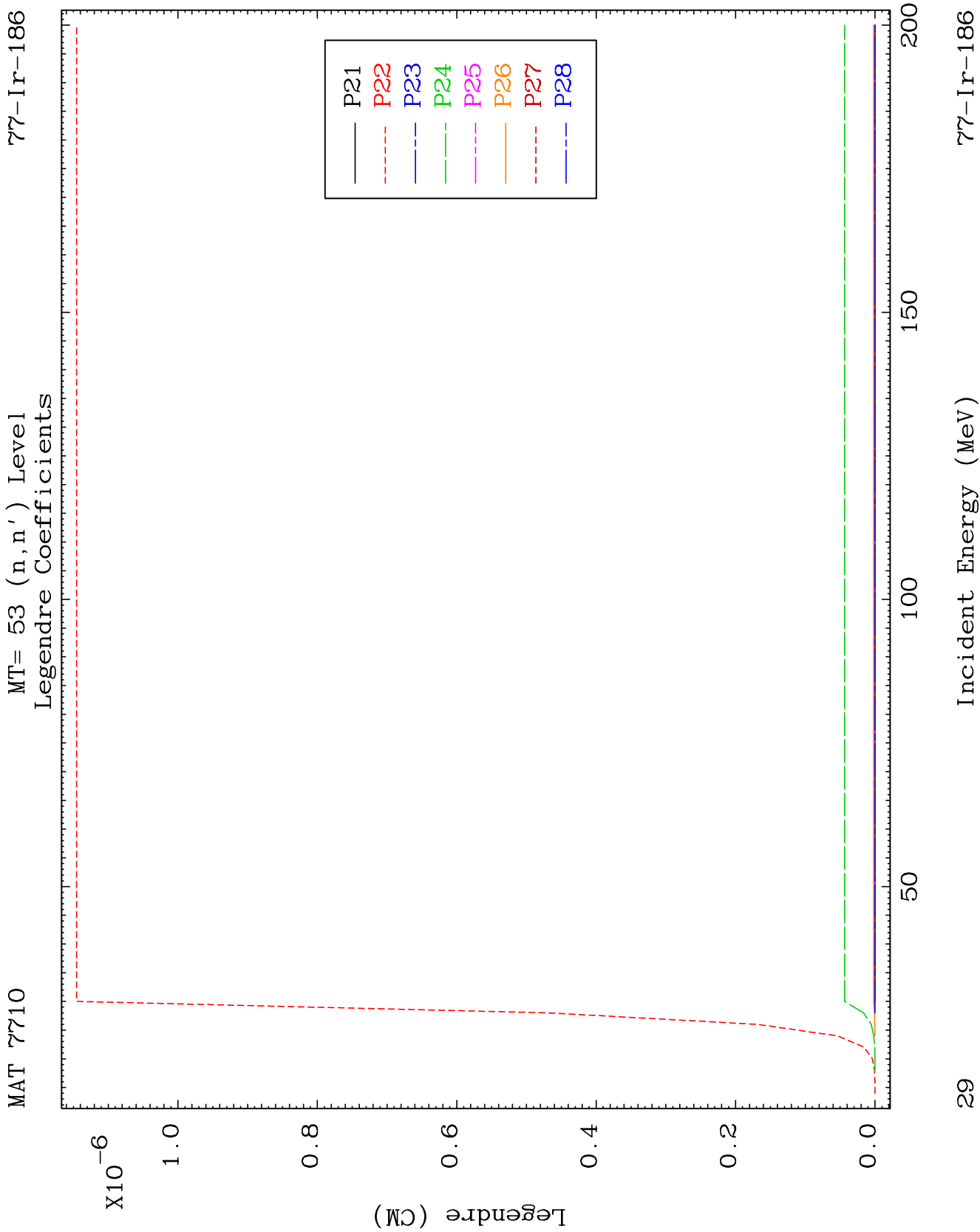


MAT 7710

MT= 53 (n, n') Level  
Legendre Coefficients

77-Ir-186

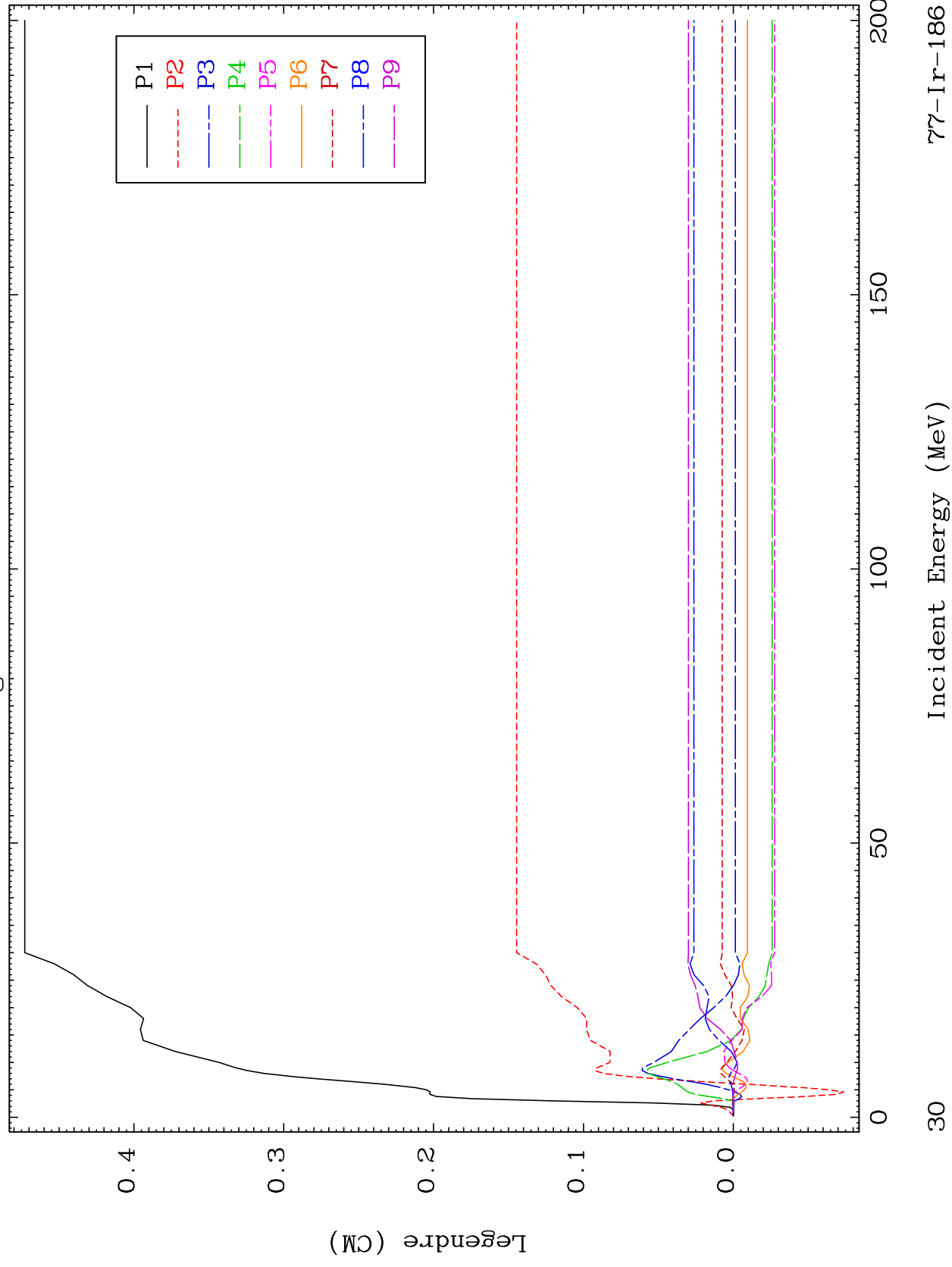


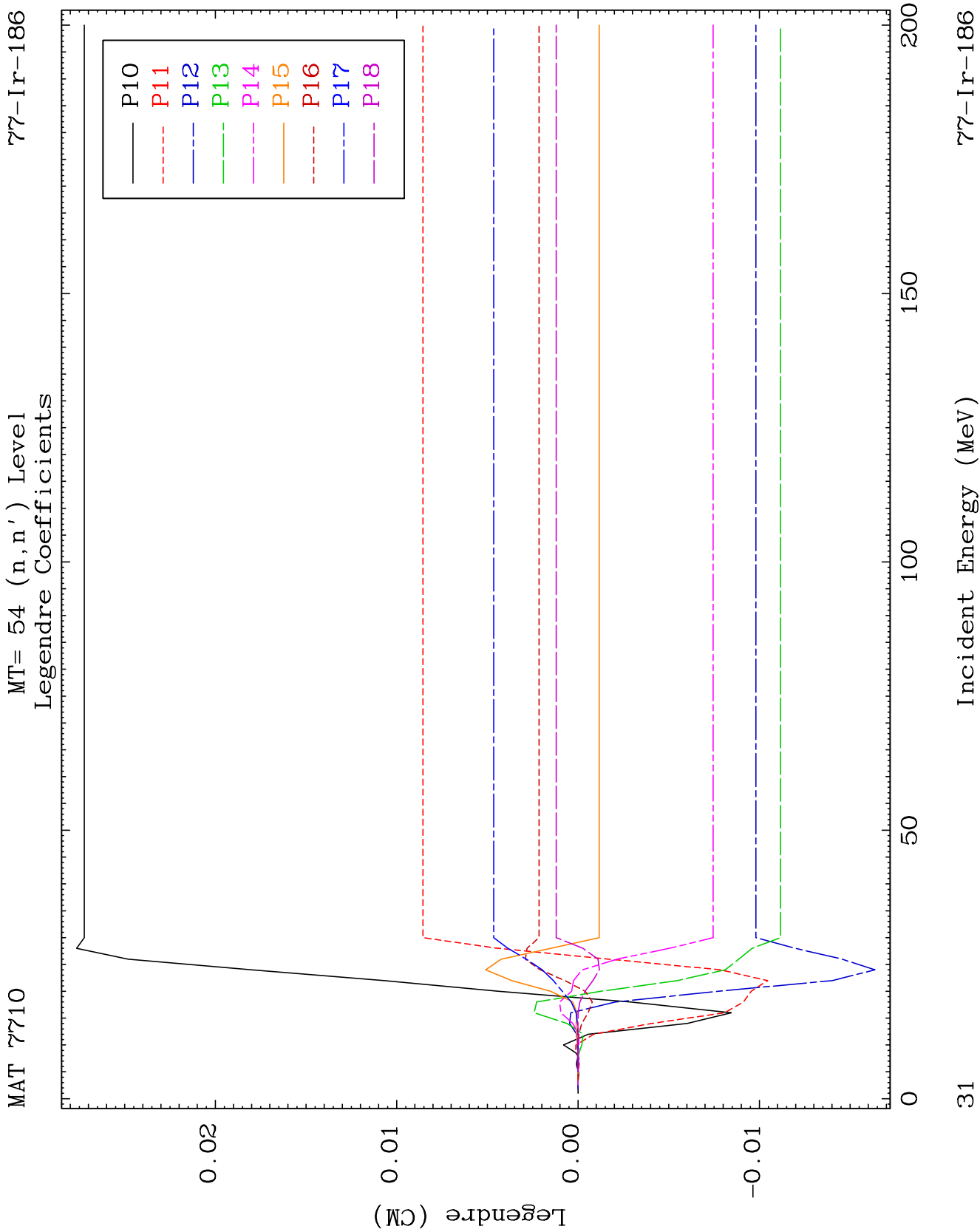


MAT 7710

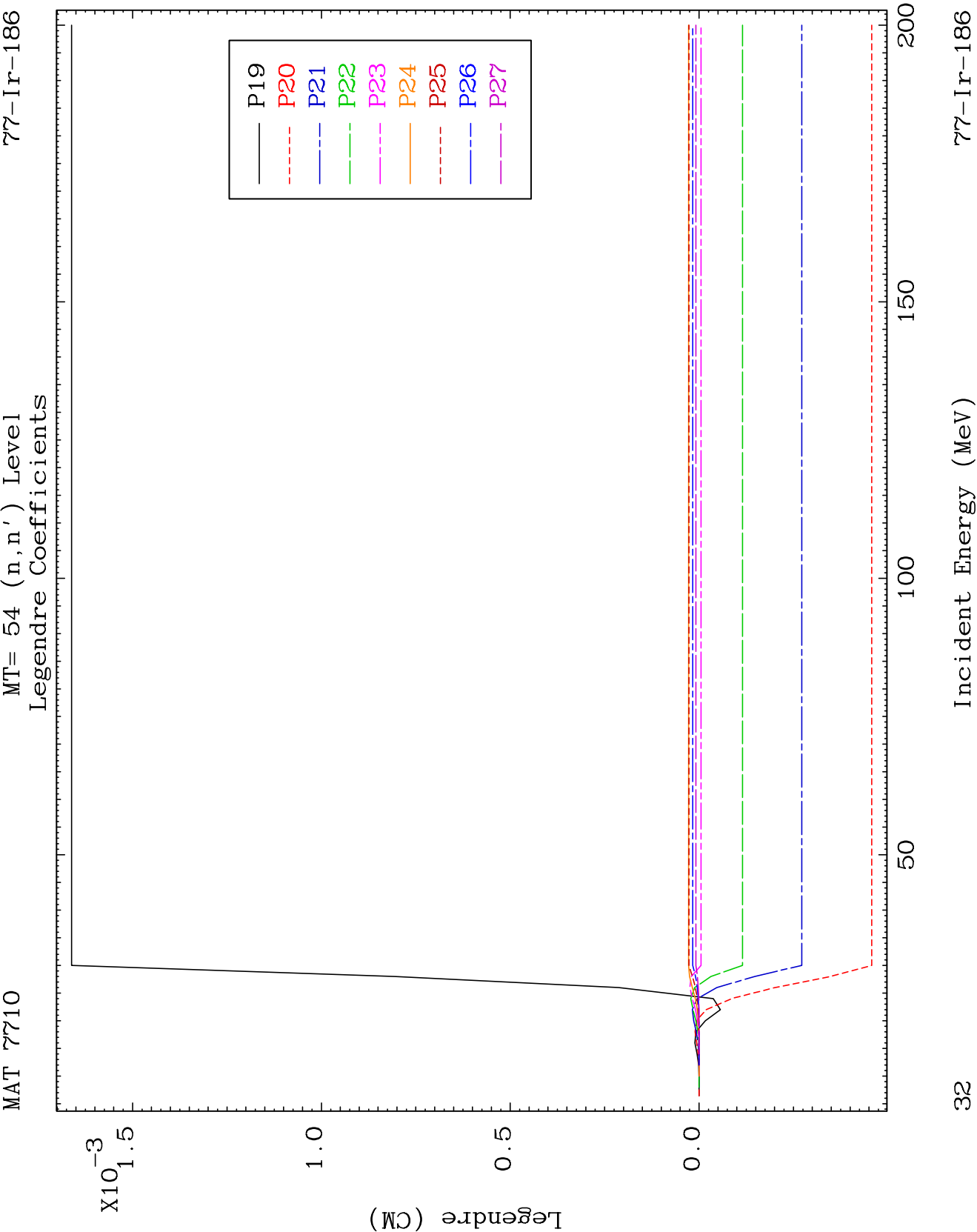
MT= 54 (n, n') Level  
Legendre Coefficients

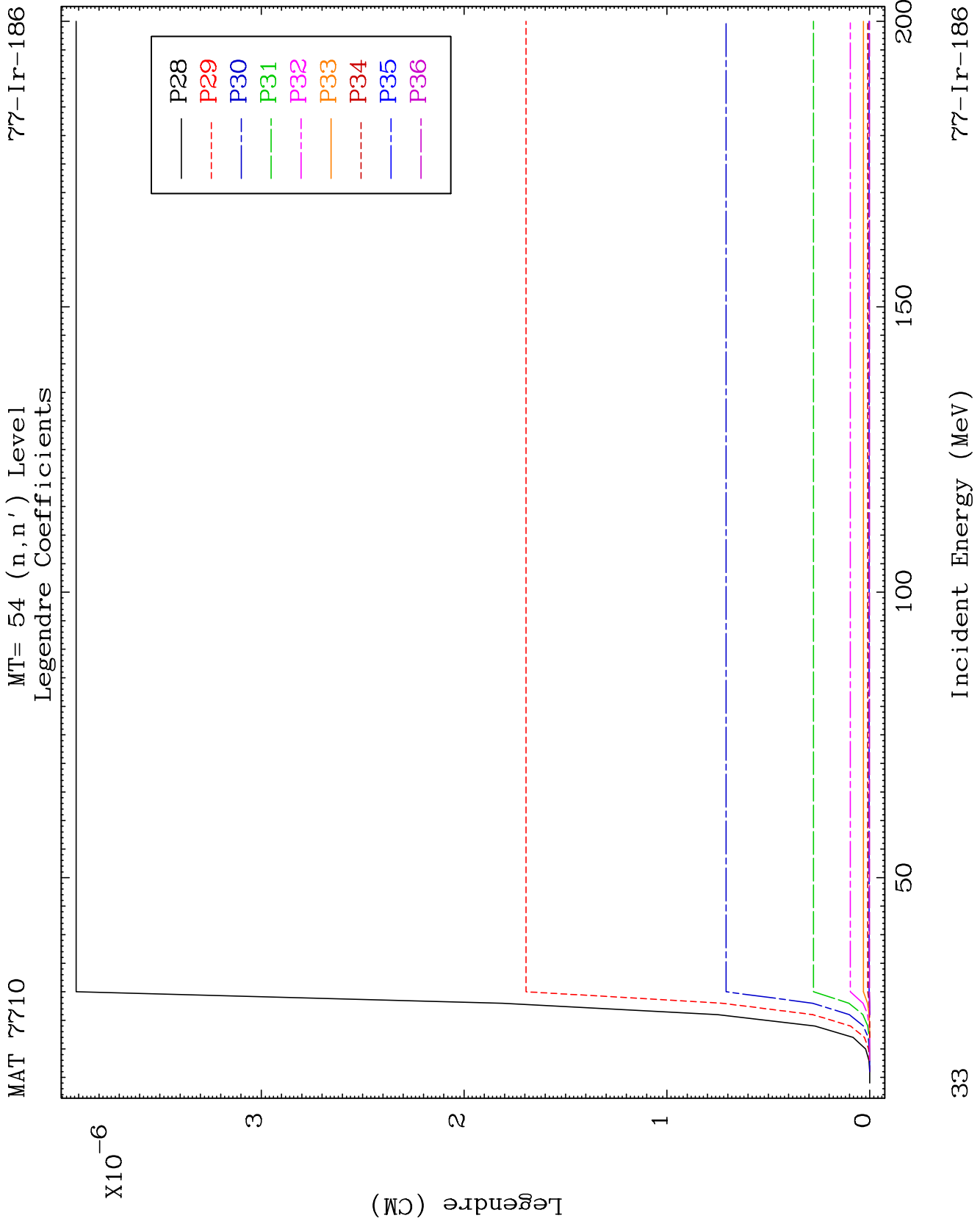
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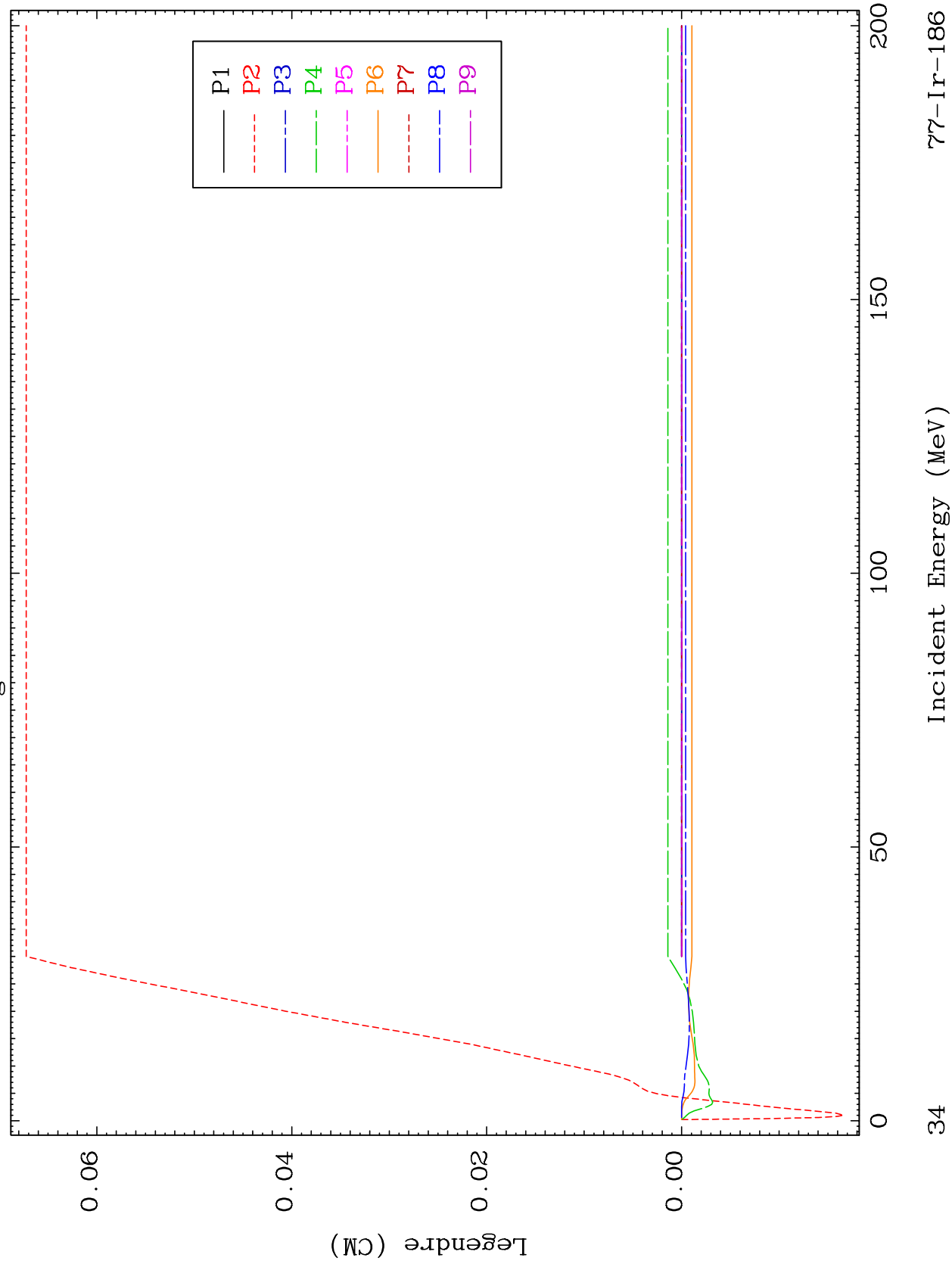


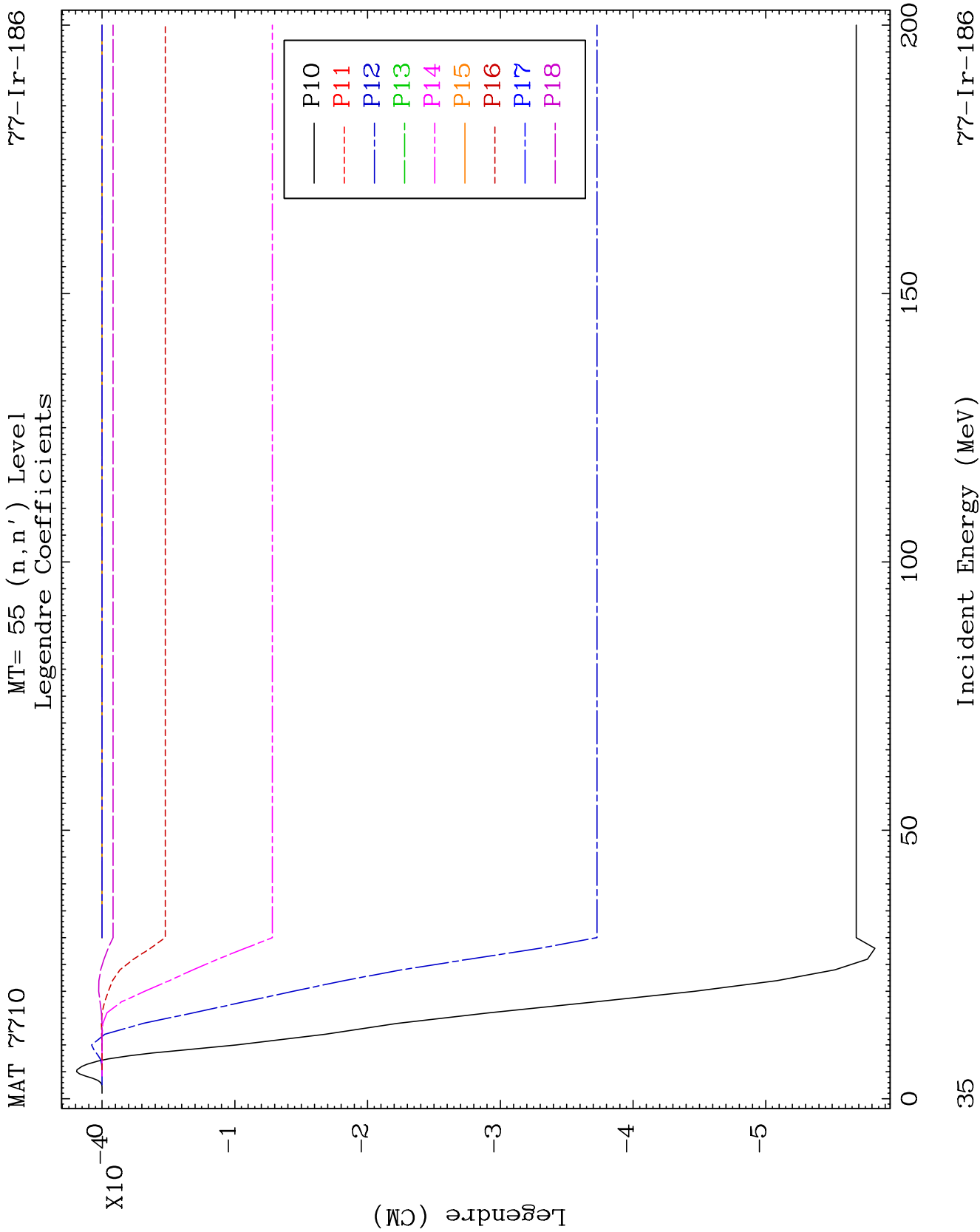


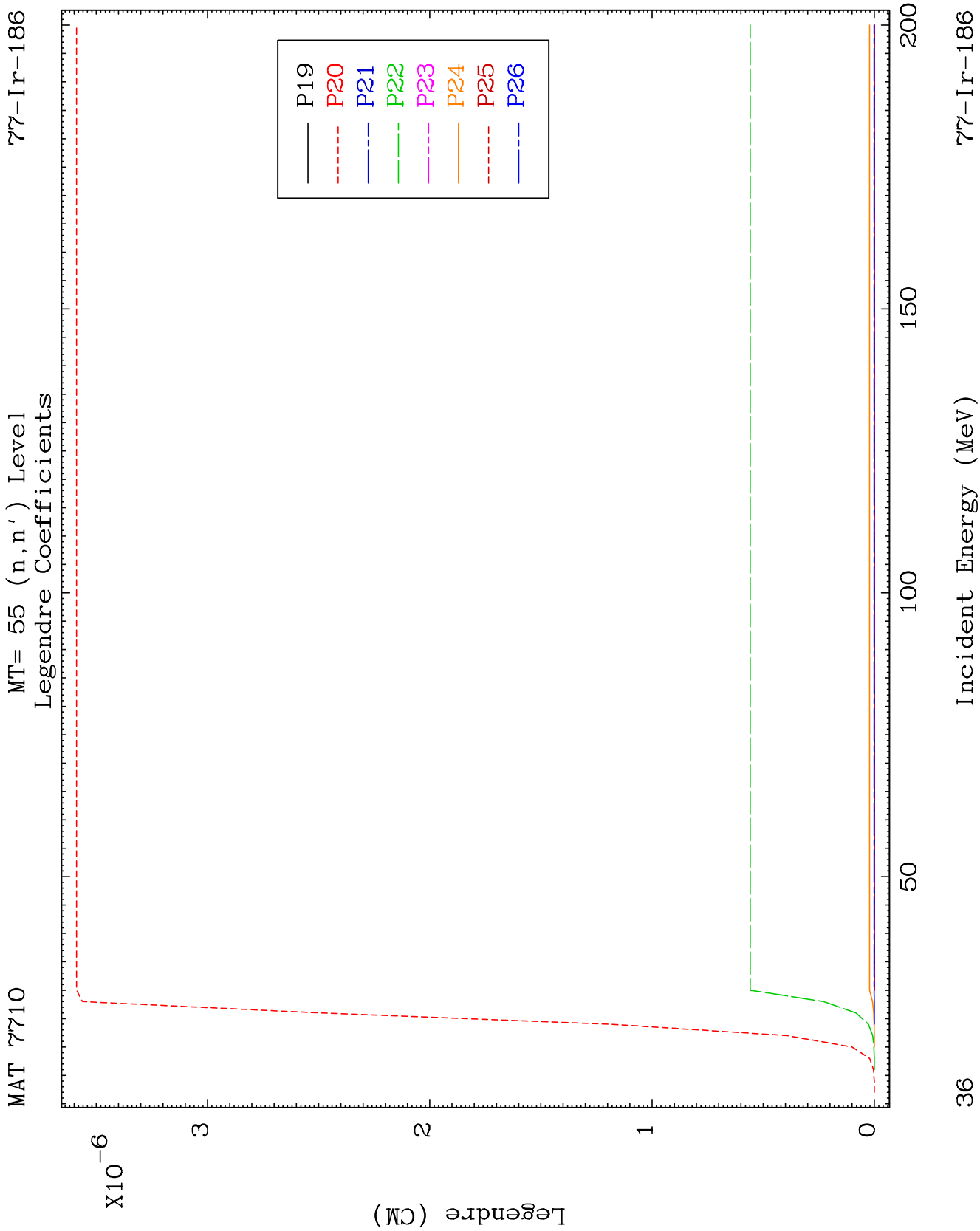
MAT 2710

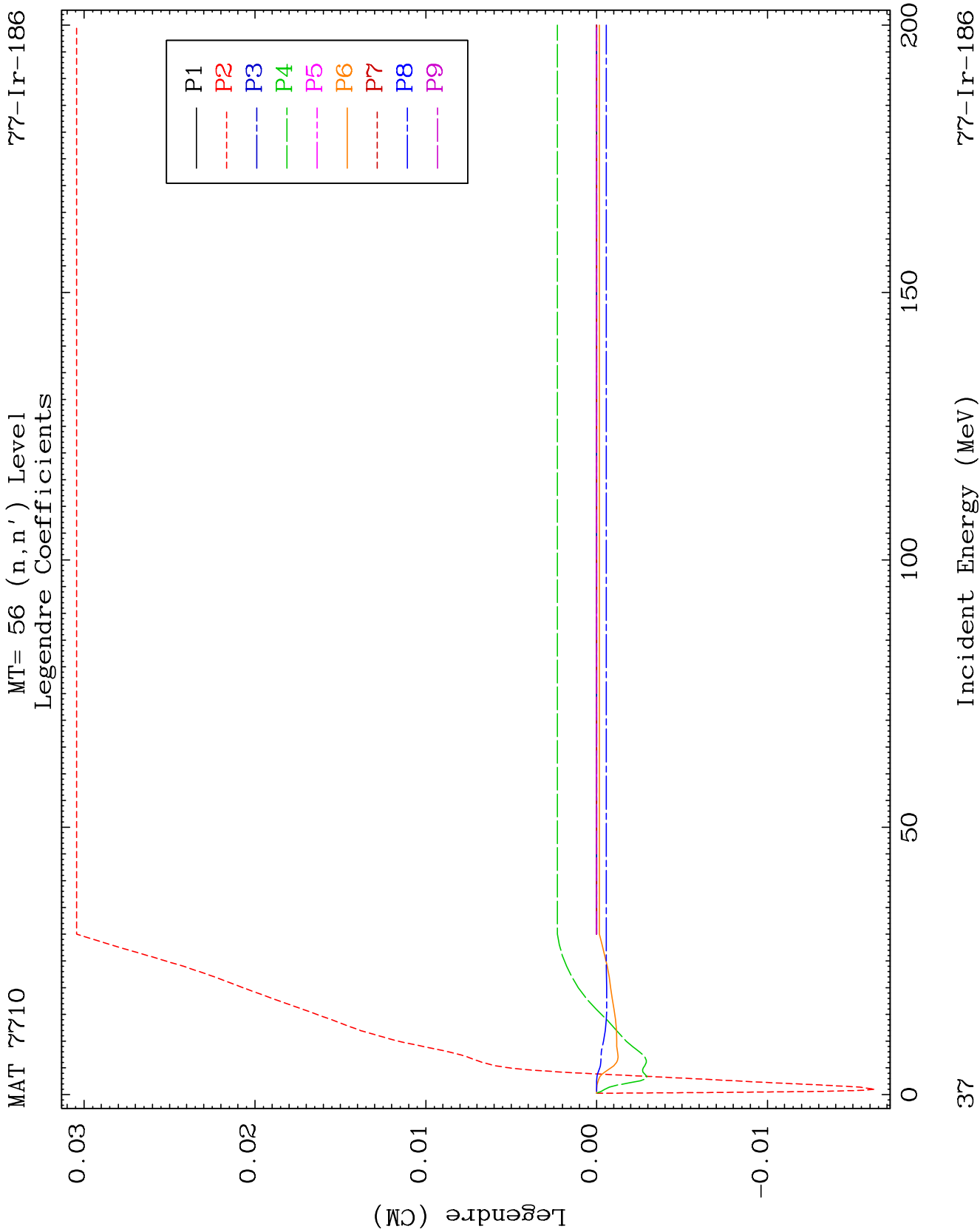
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Legendre Coefficients

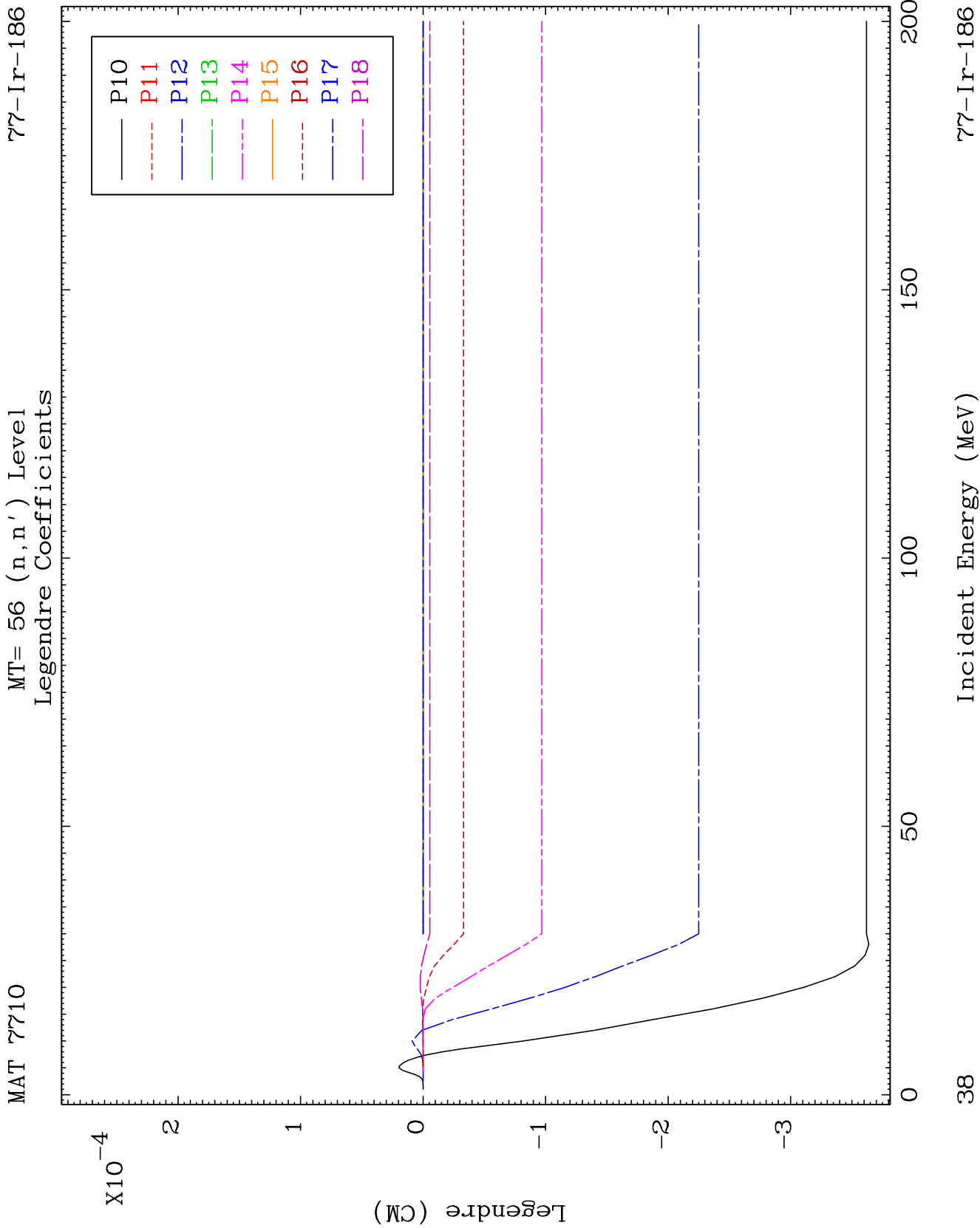
77-Ir-186

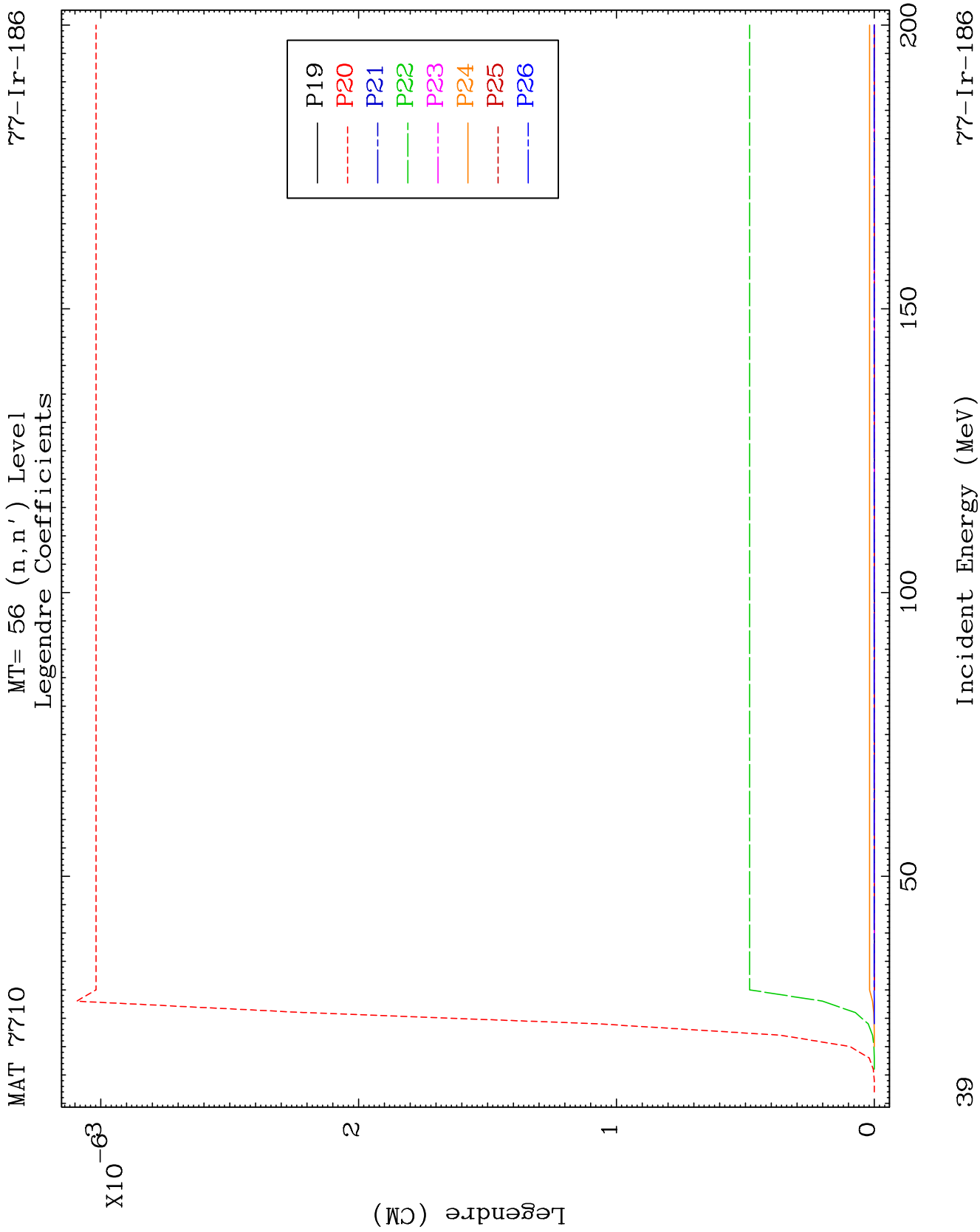










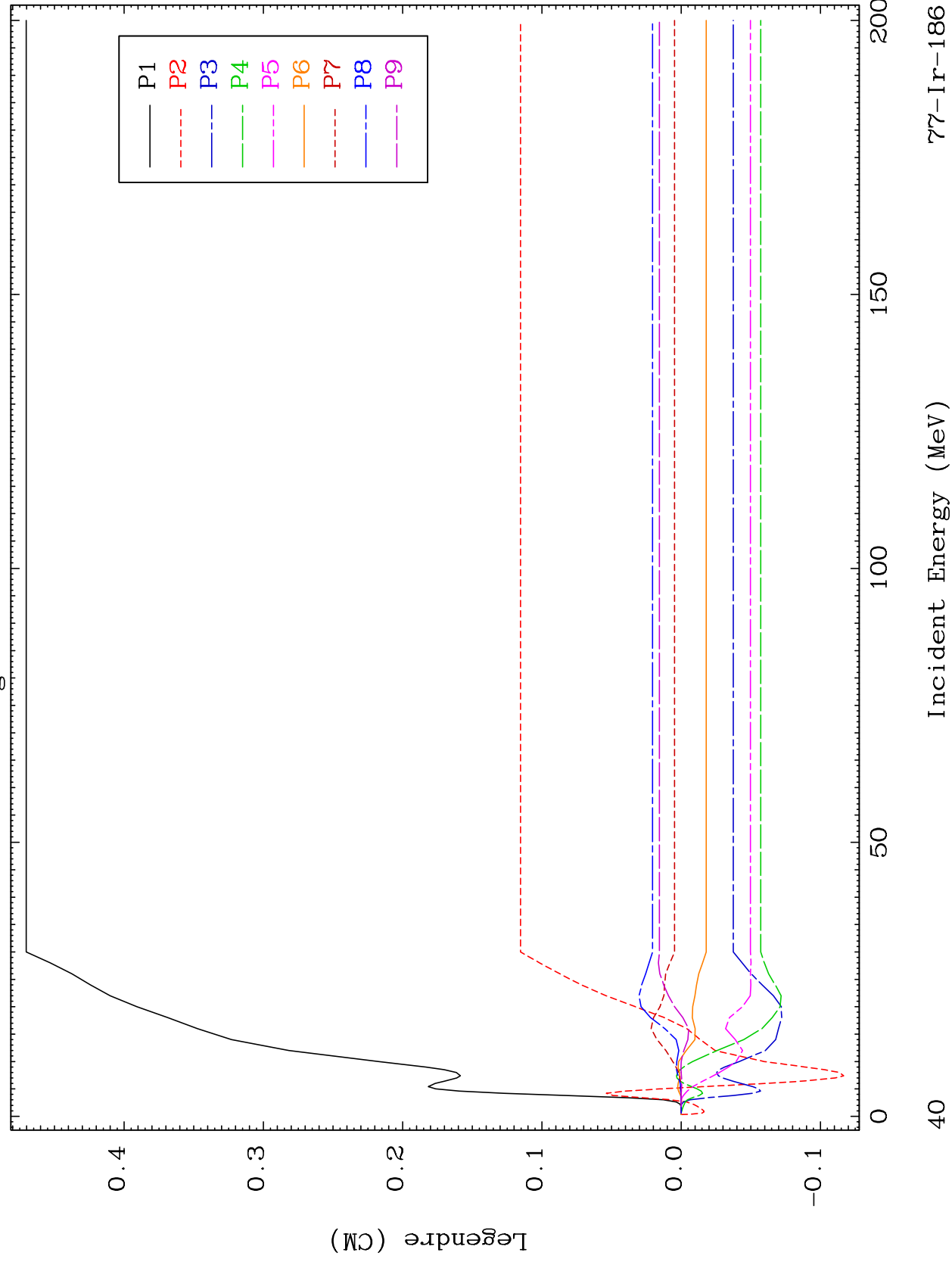


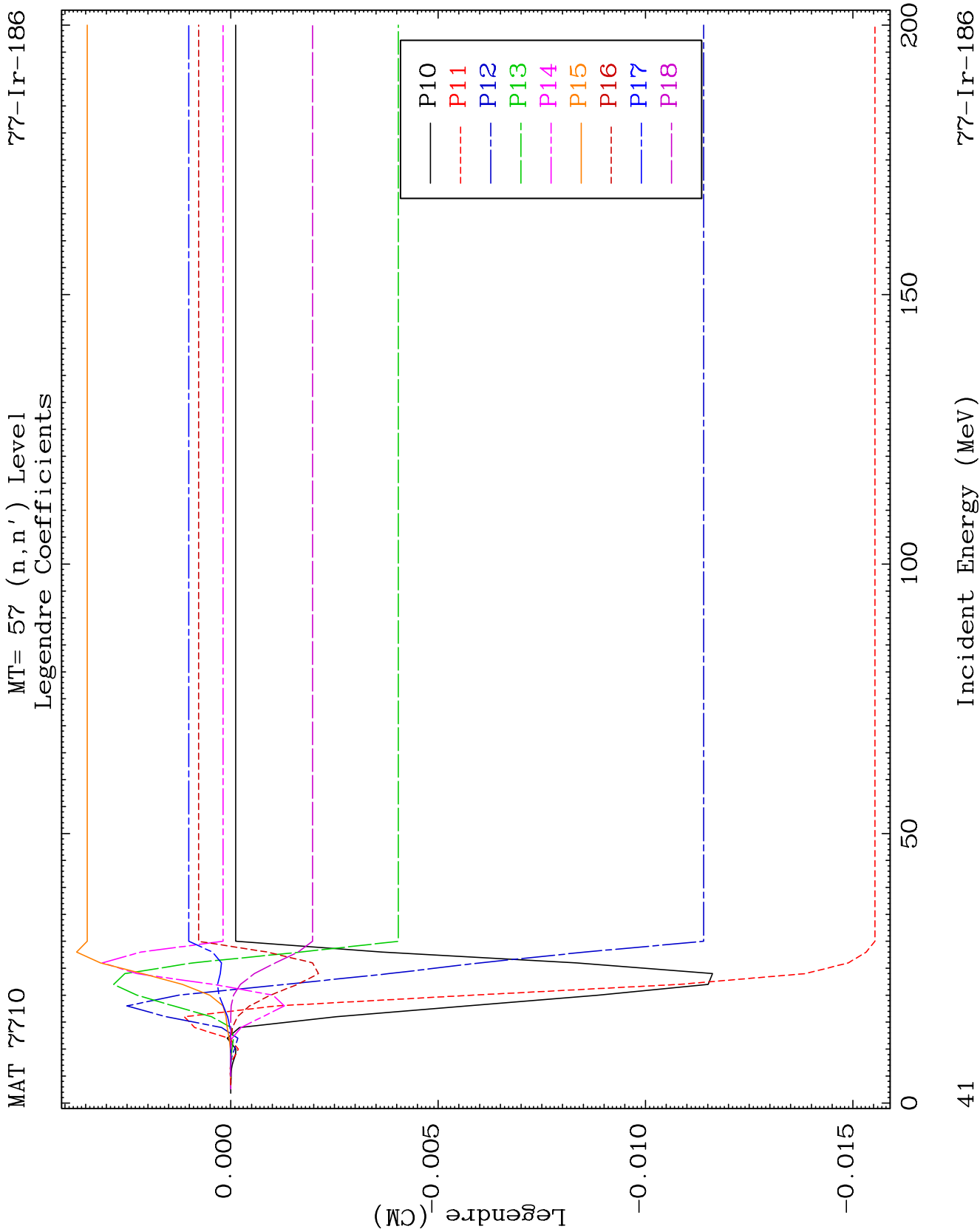


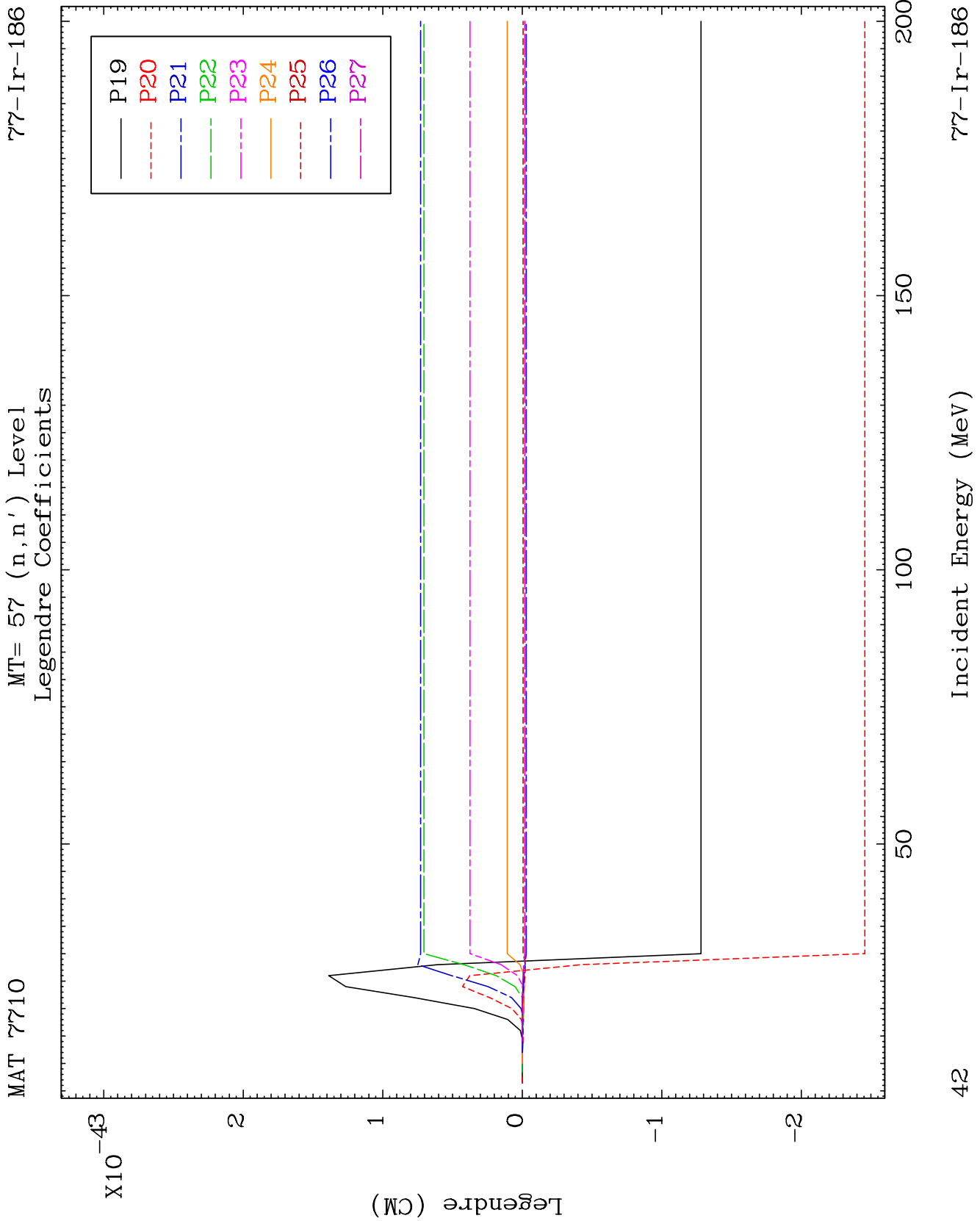
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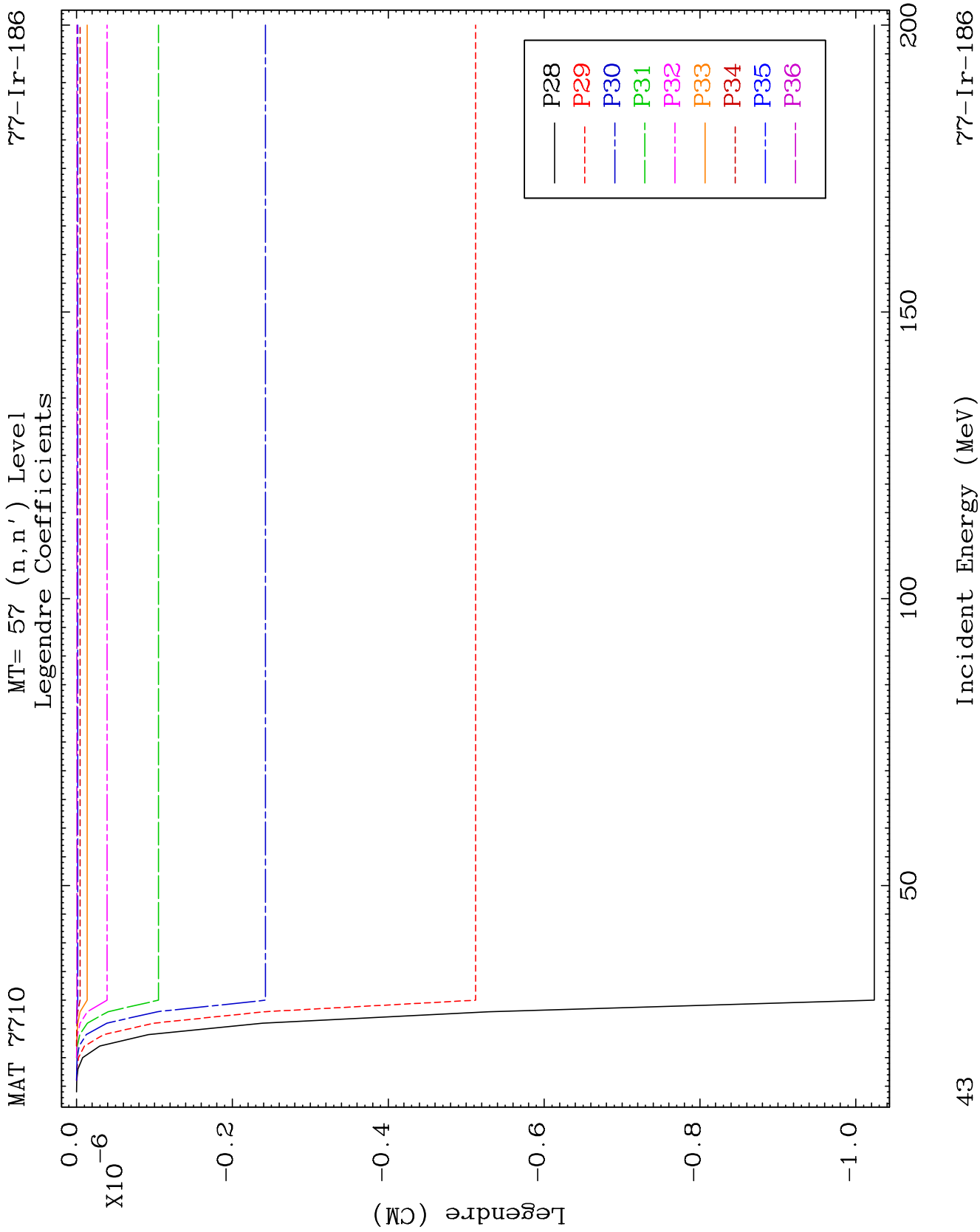
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Legendre Coefficients

77-Ir-186







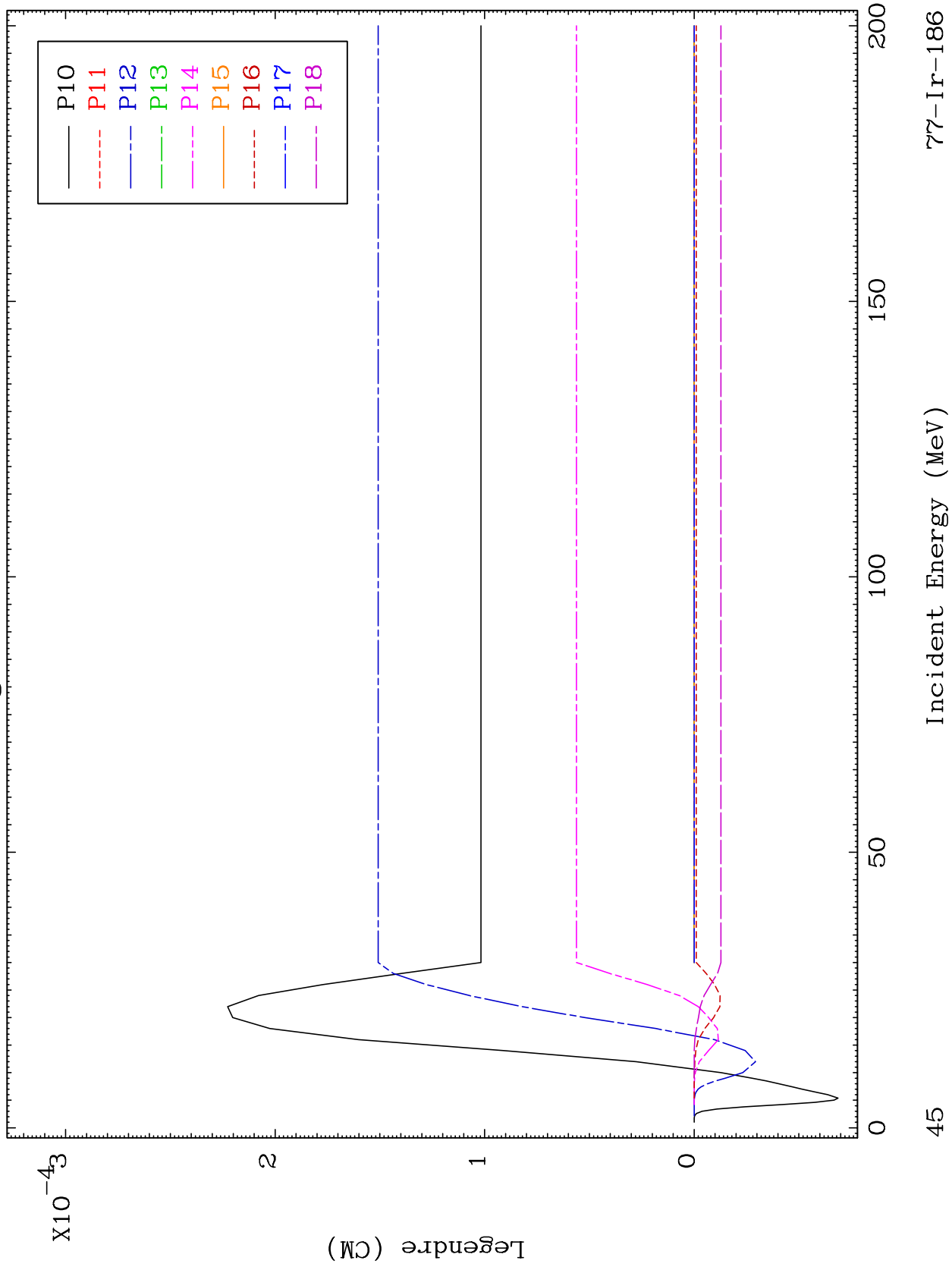


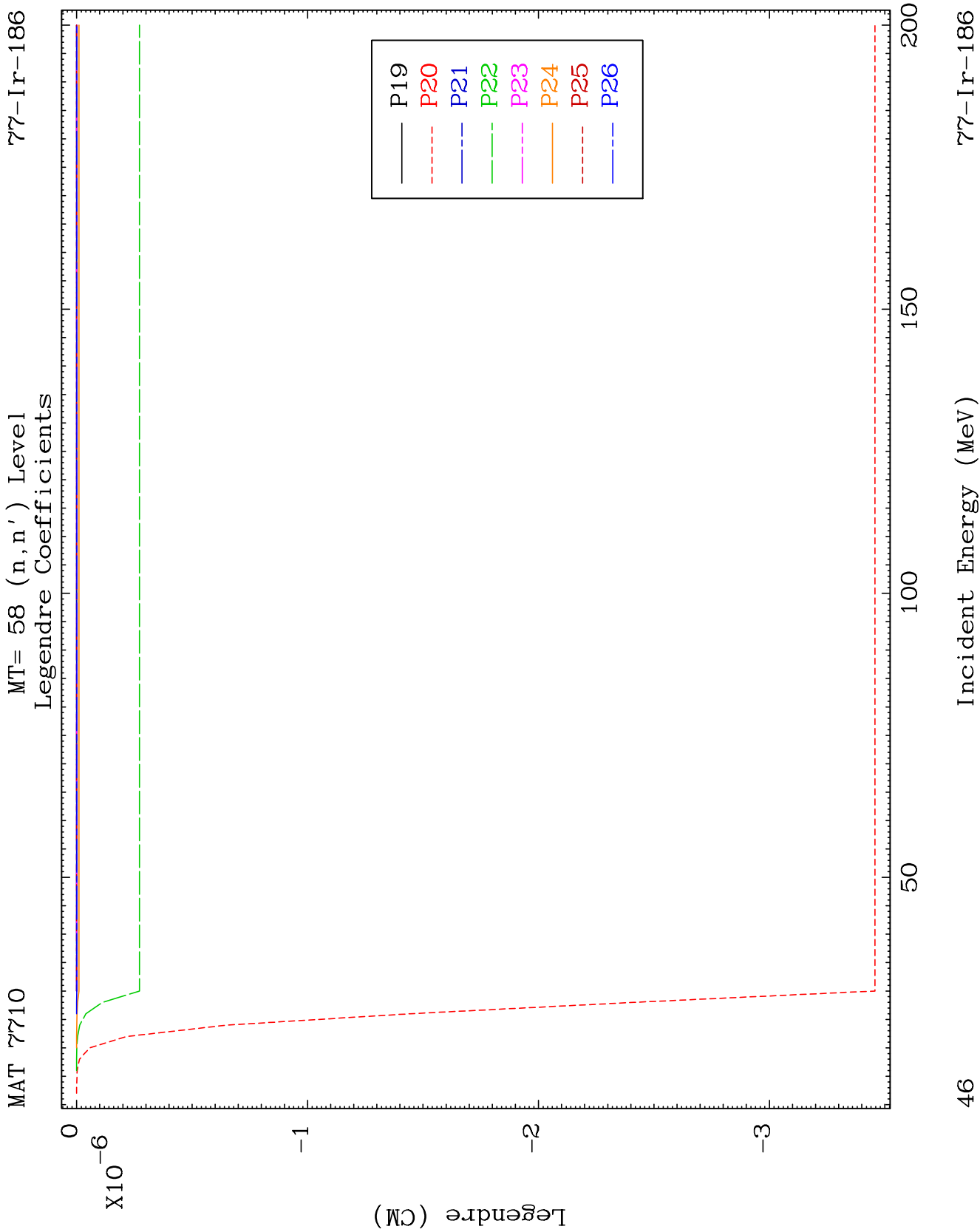


MAT 7710

MT= 58 (n, n') Level  
Legendre Coefficients

77-Ir-186

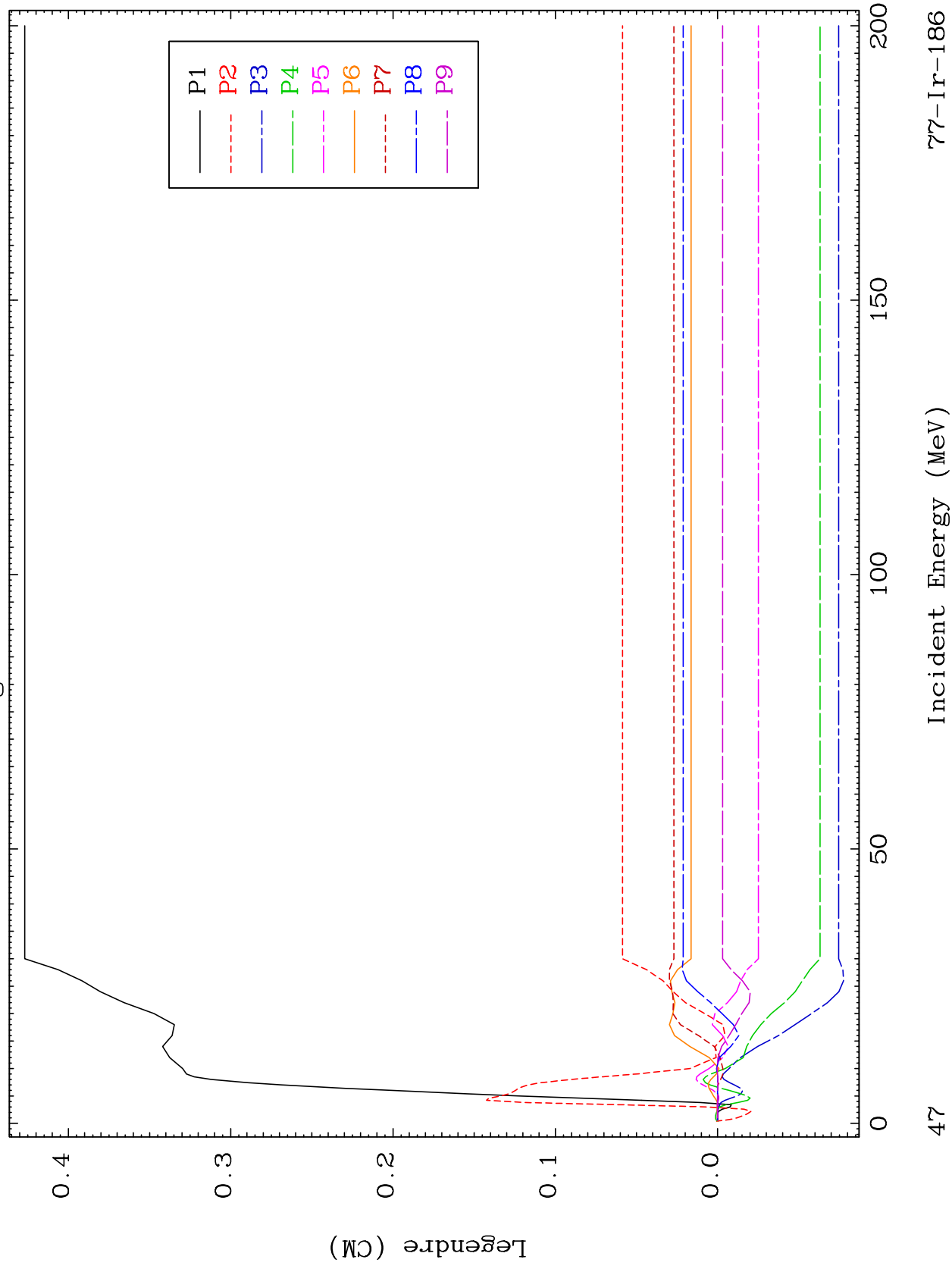




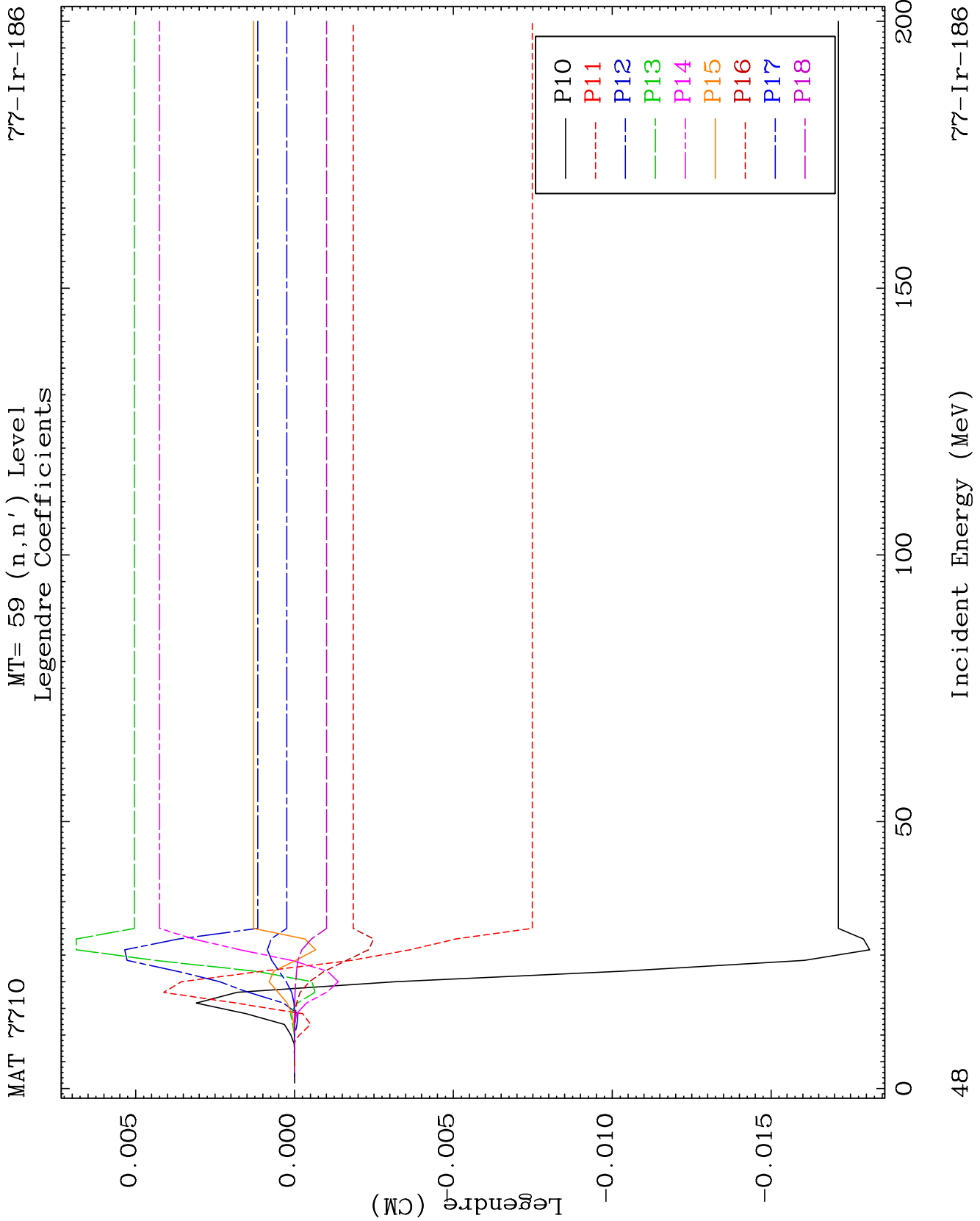
MAT 7710

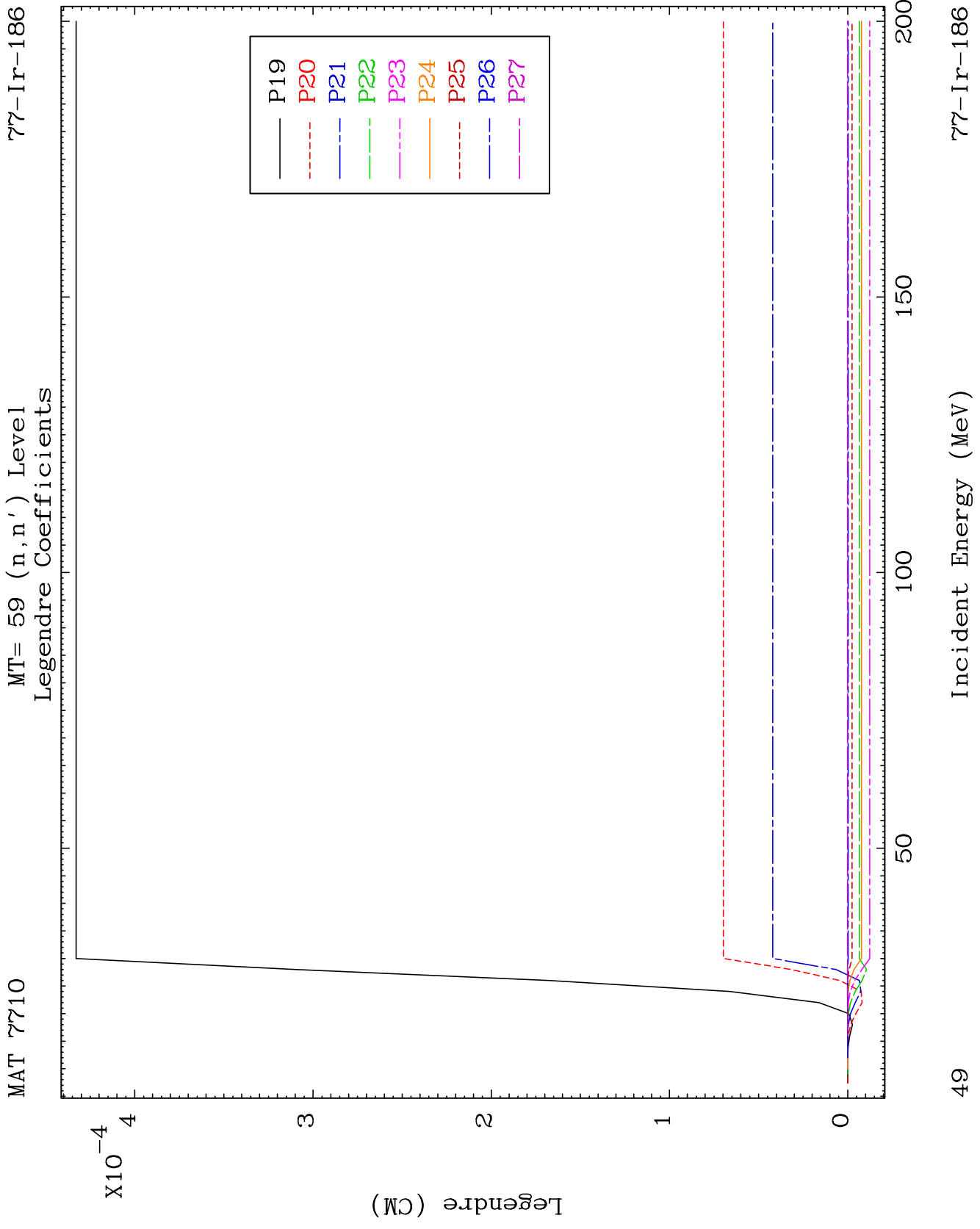
MT= 59 (n, n') Level  
Legendre Coefficients

77-Ir-186





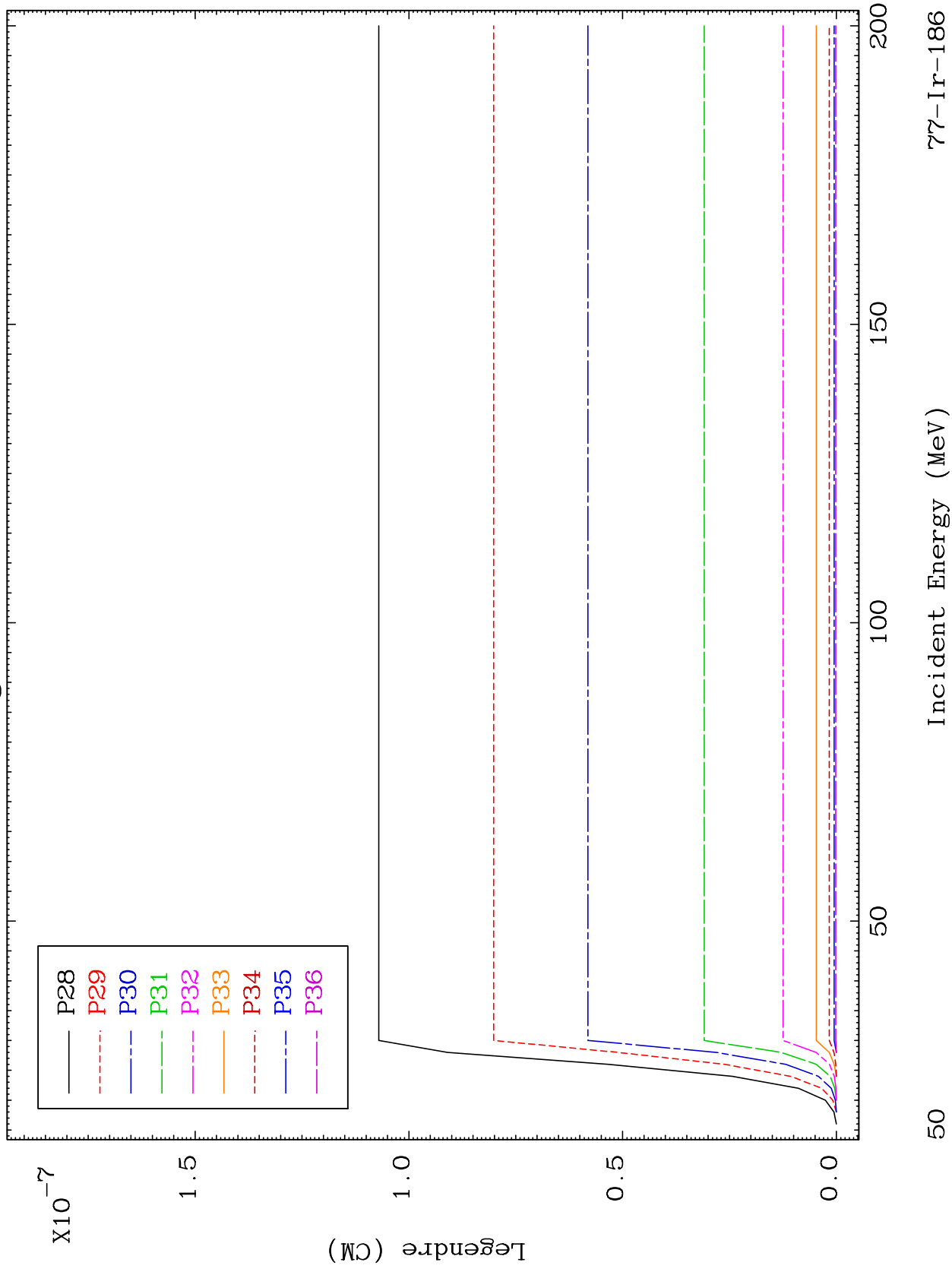


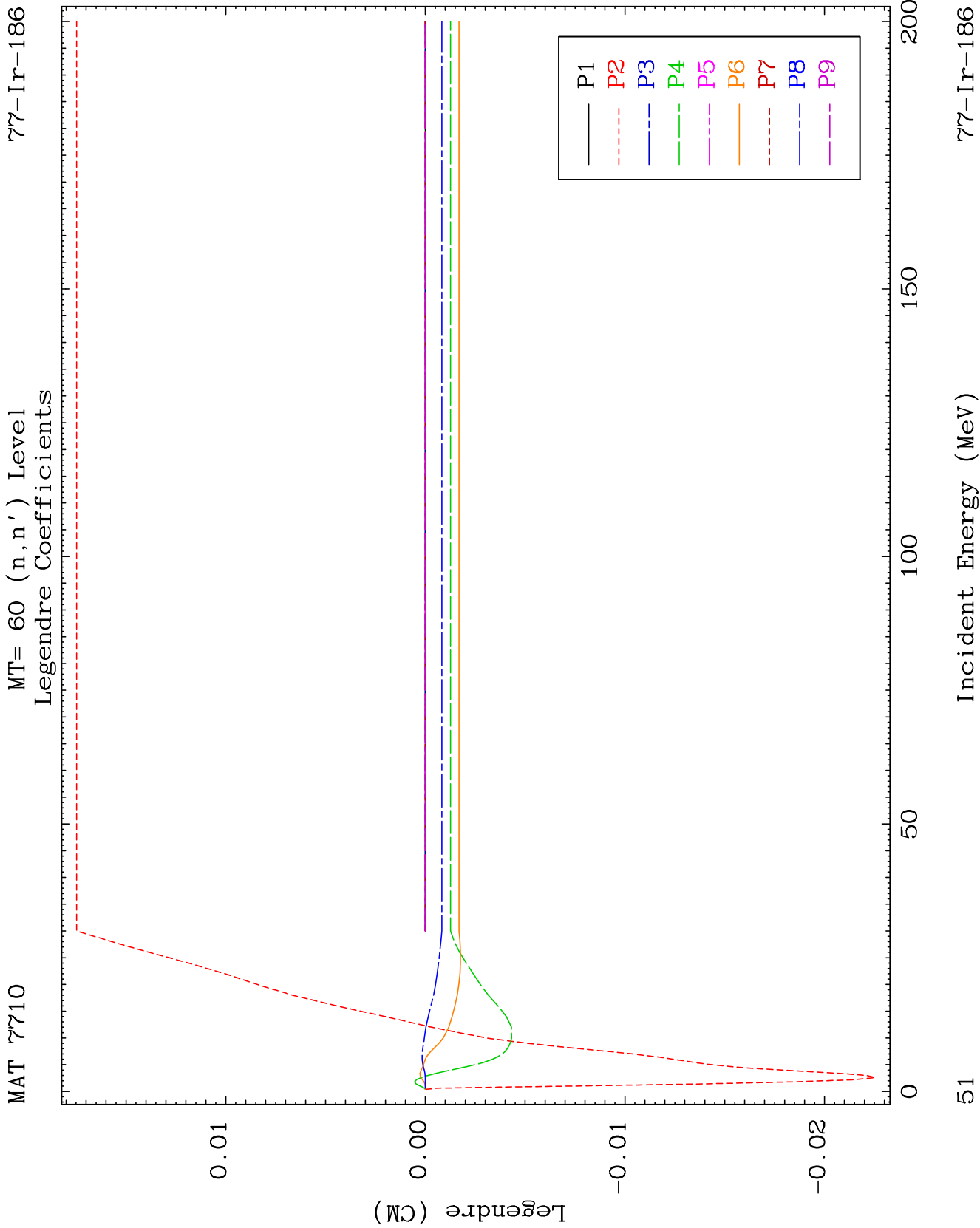


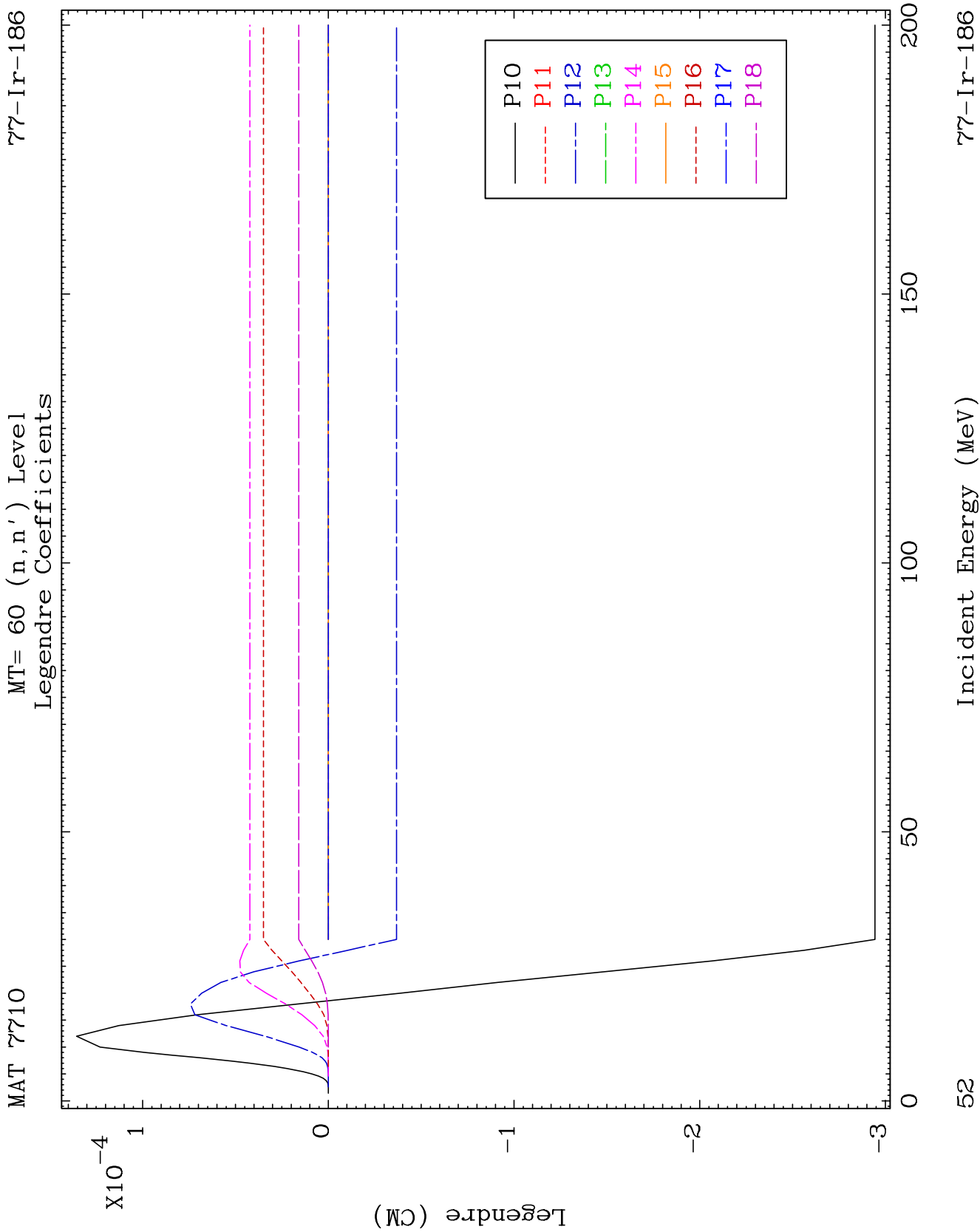
MAT 7710

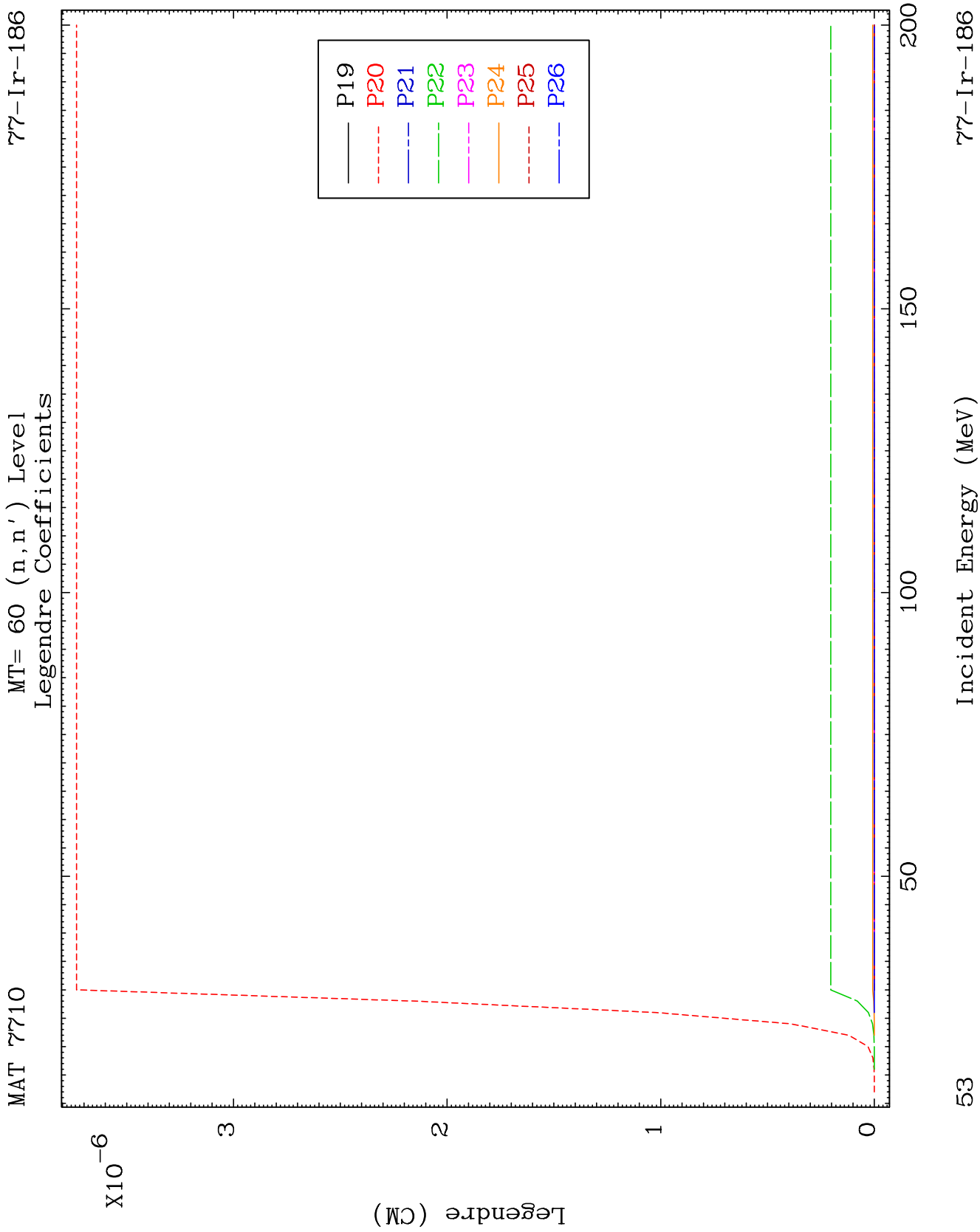
MT= 59 (n, n') Level  
Legendre Coefficients

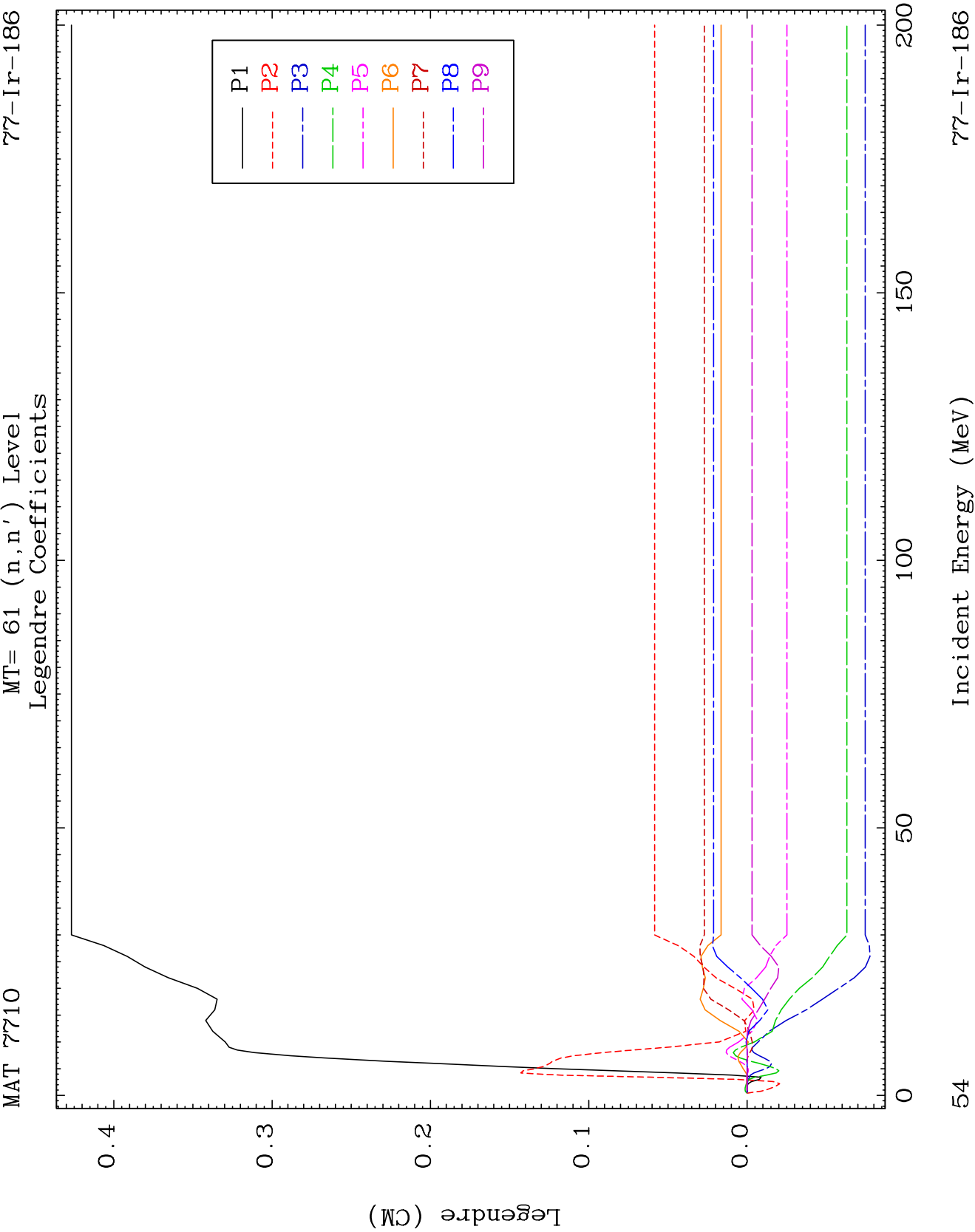
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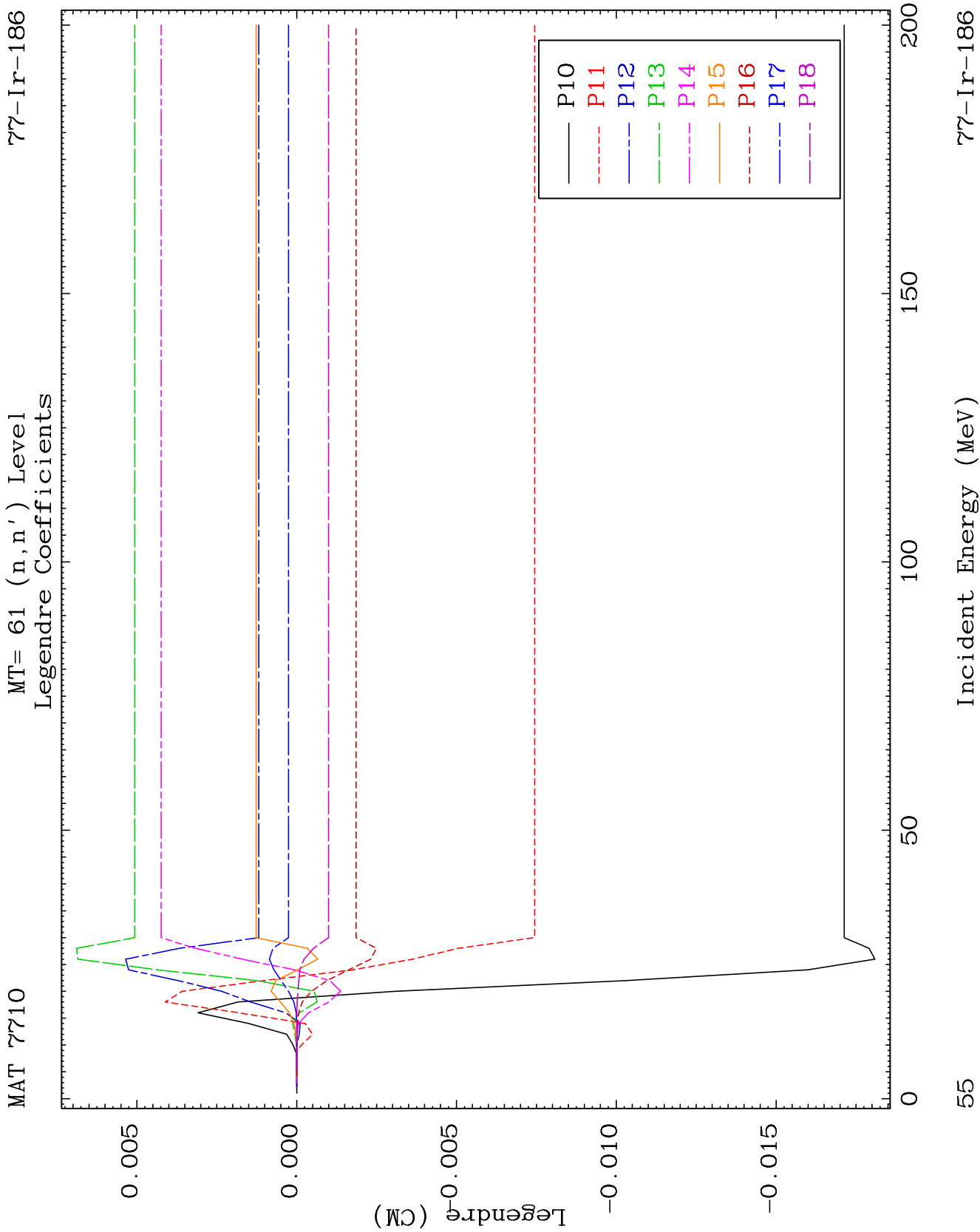




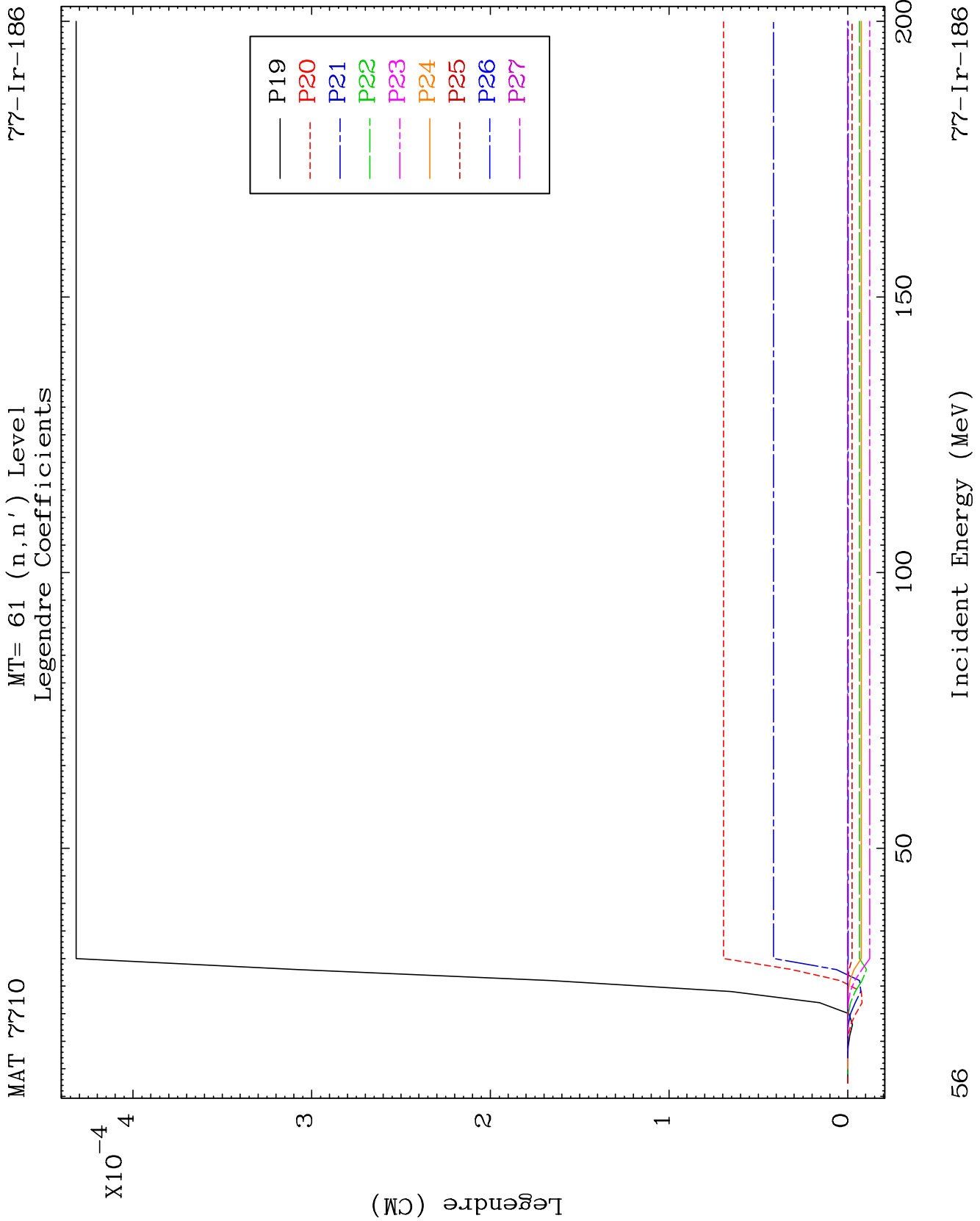








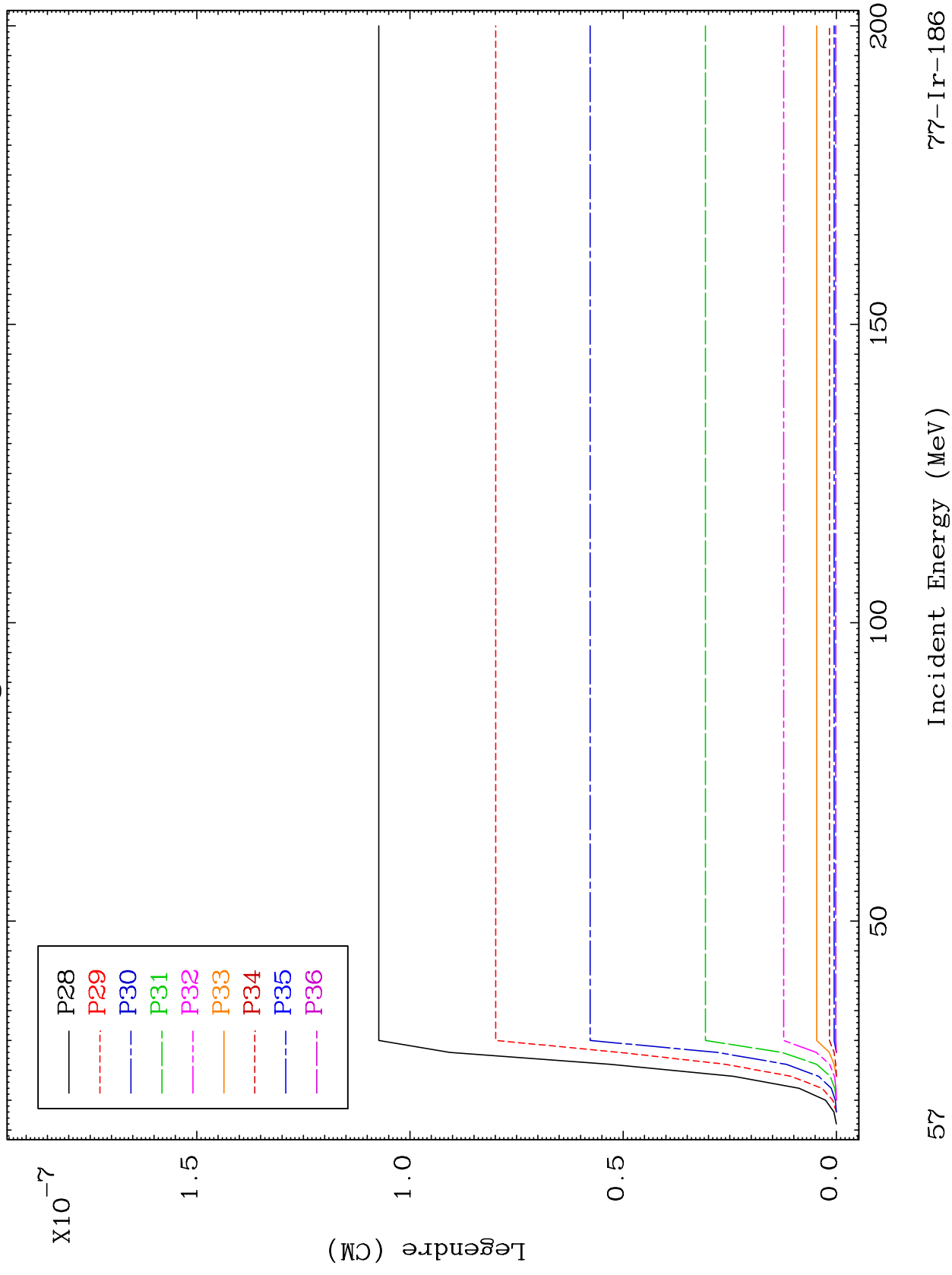




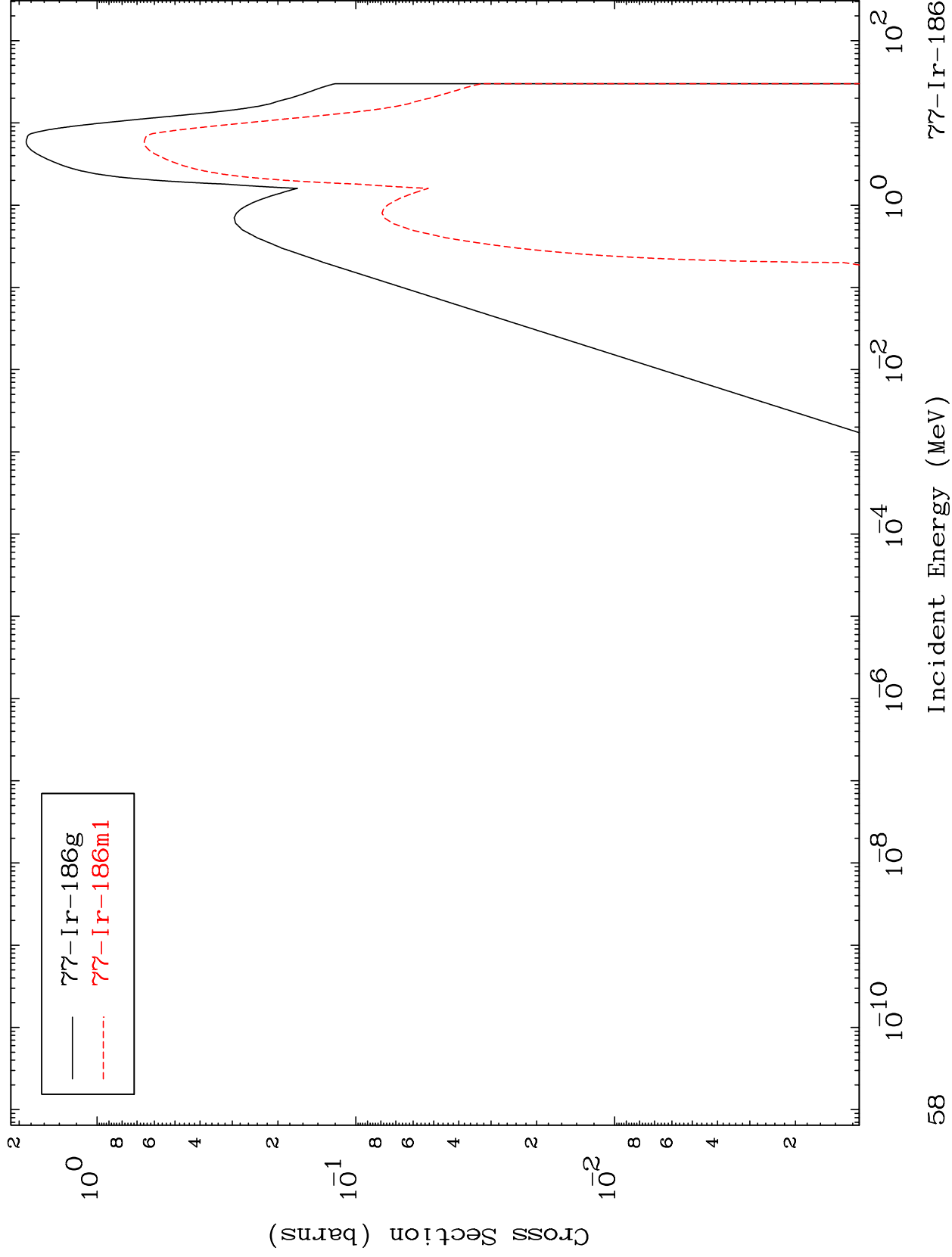
MAT 2710

MT= 61 (n, n') Level  
Legendre Coefficients

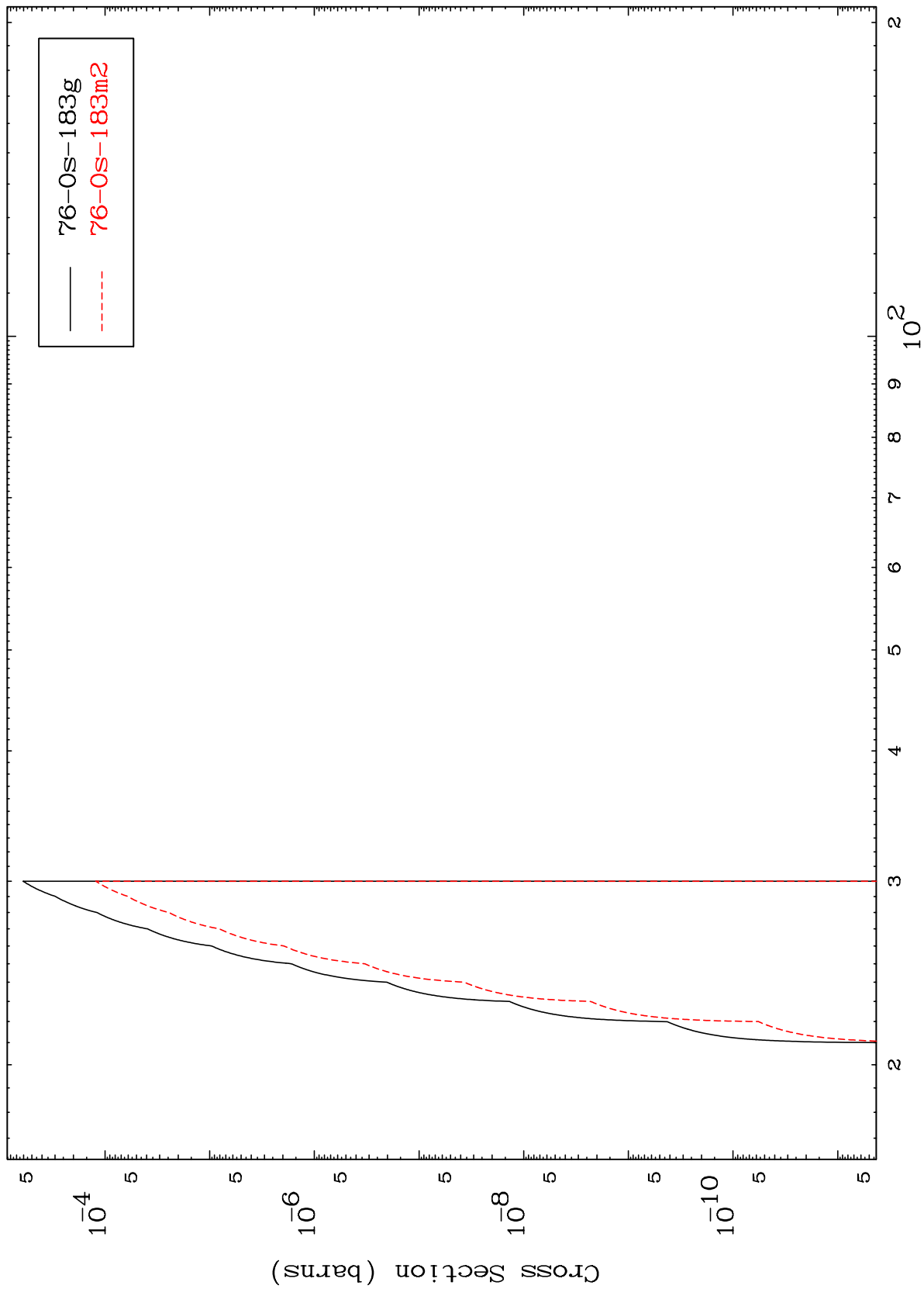
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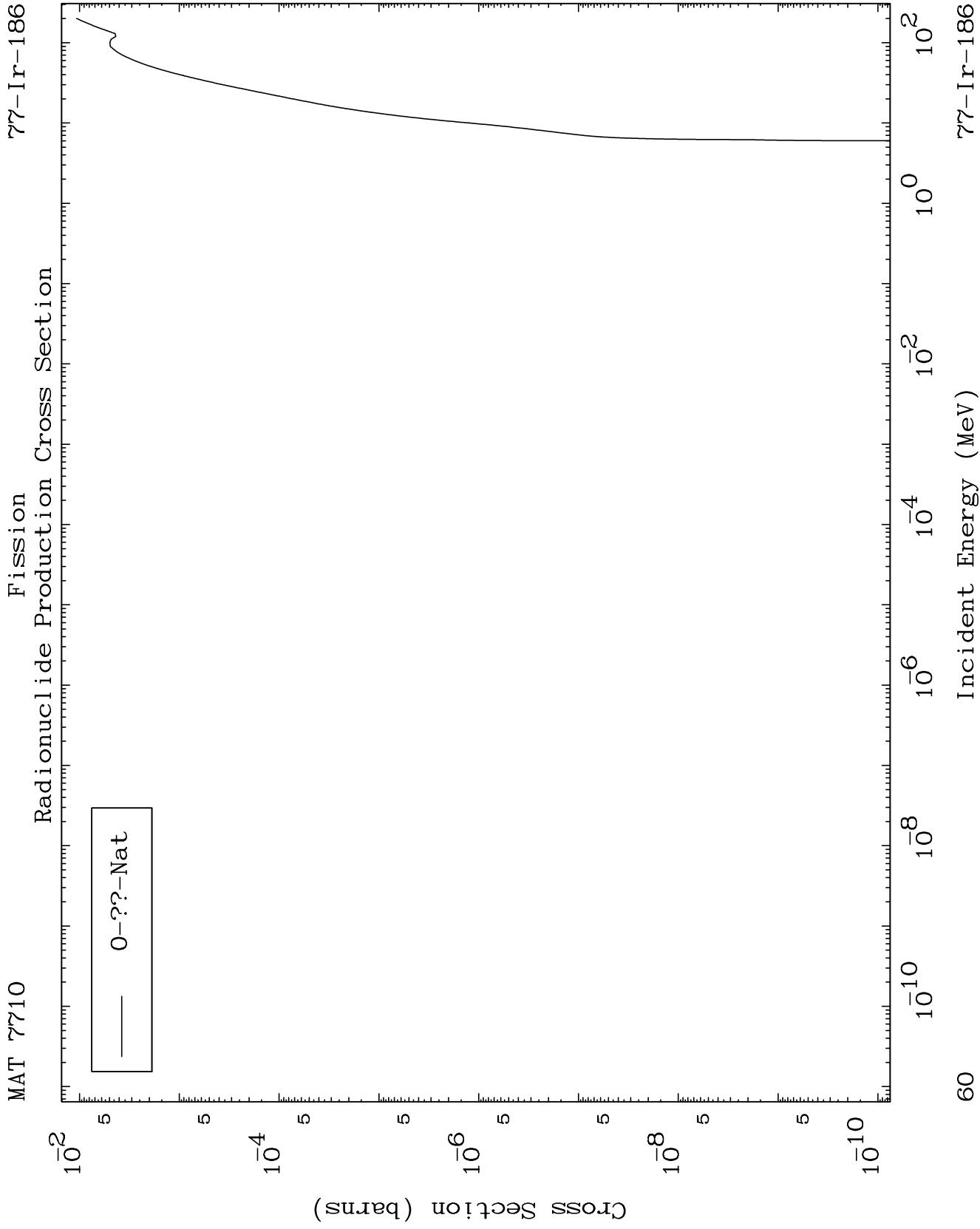


Radionuclide Production Cross Section



(n,2n) d  
Radionuclide Production Cross Section

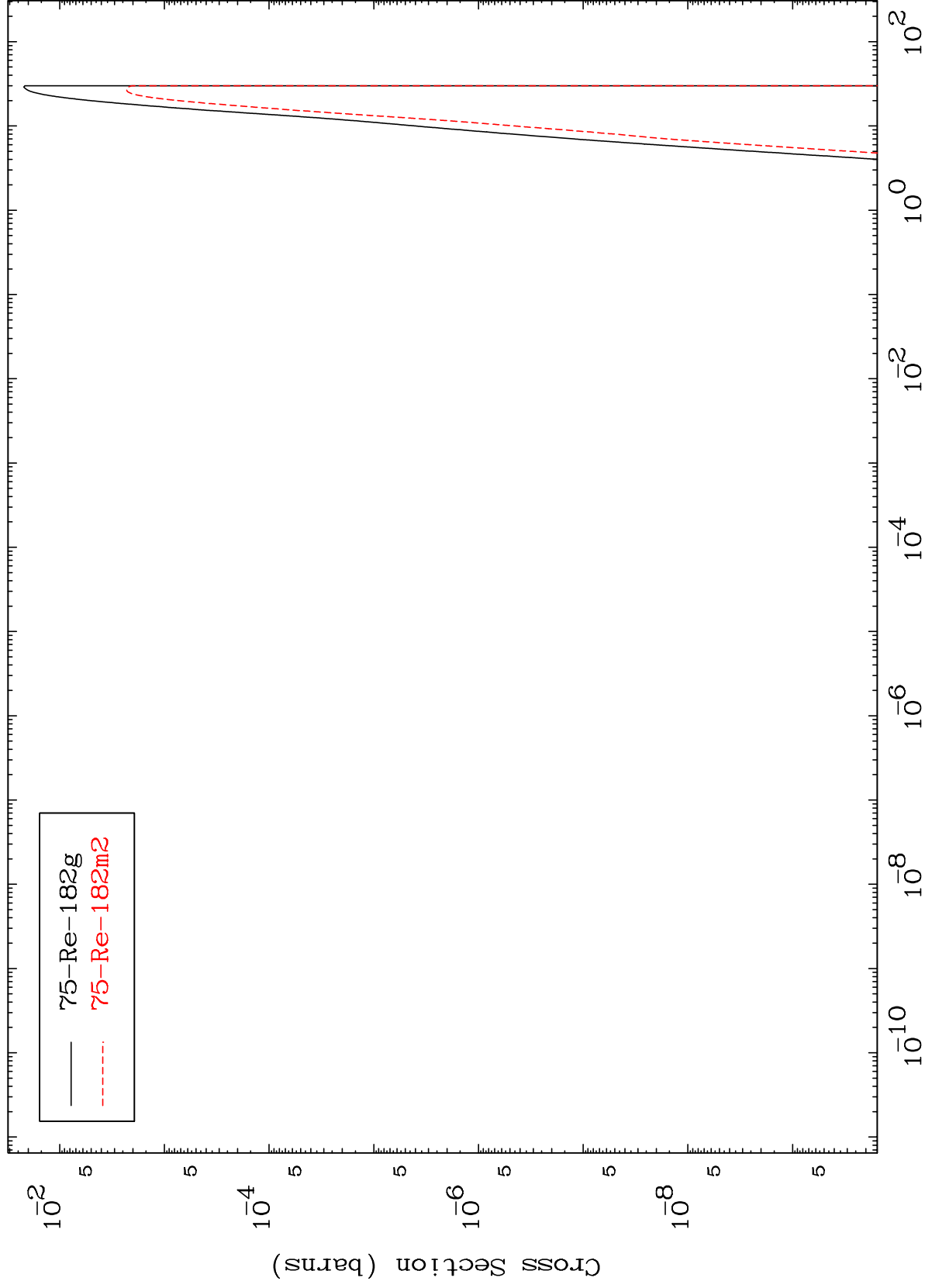




MAT 7710

77-Ir-186

Radionuclide Production Cross Section  
(n,n')  $\alpha$



75-Re-182g  
75-Re-182m2

61

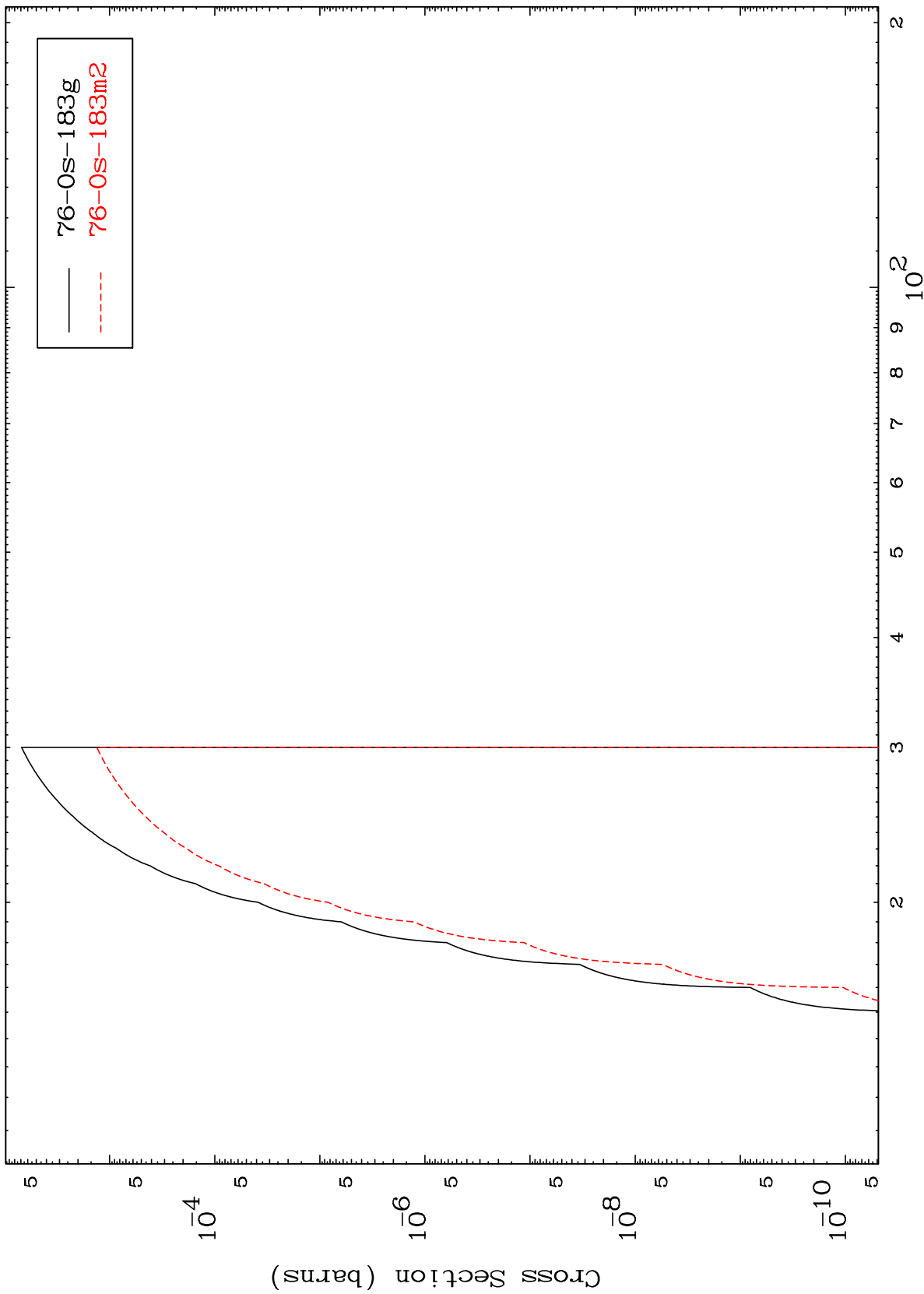
77-Ir-186

MAT 7710

(n,n') t

77-Ir-186

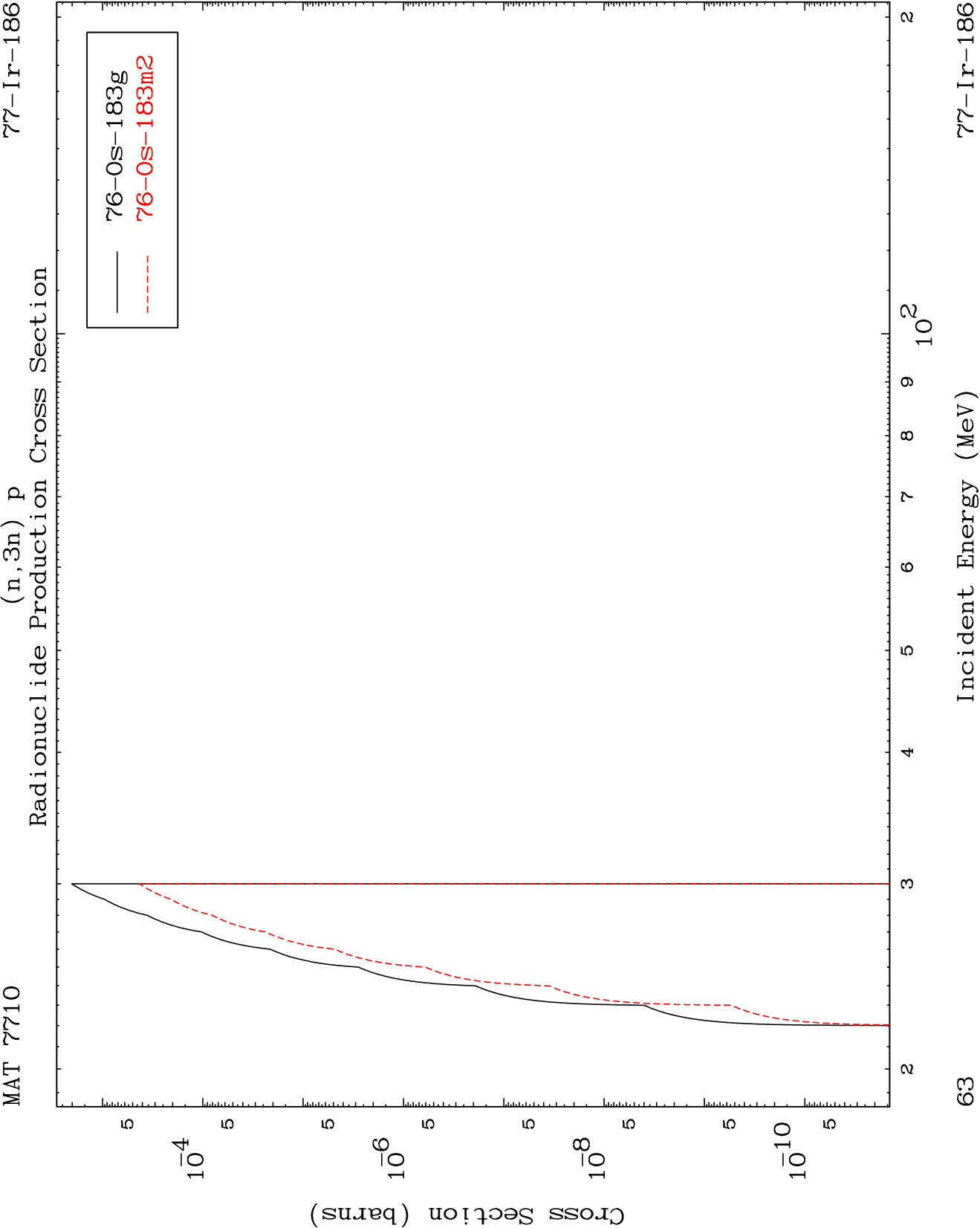
Radionuclide Production Cross Section



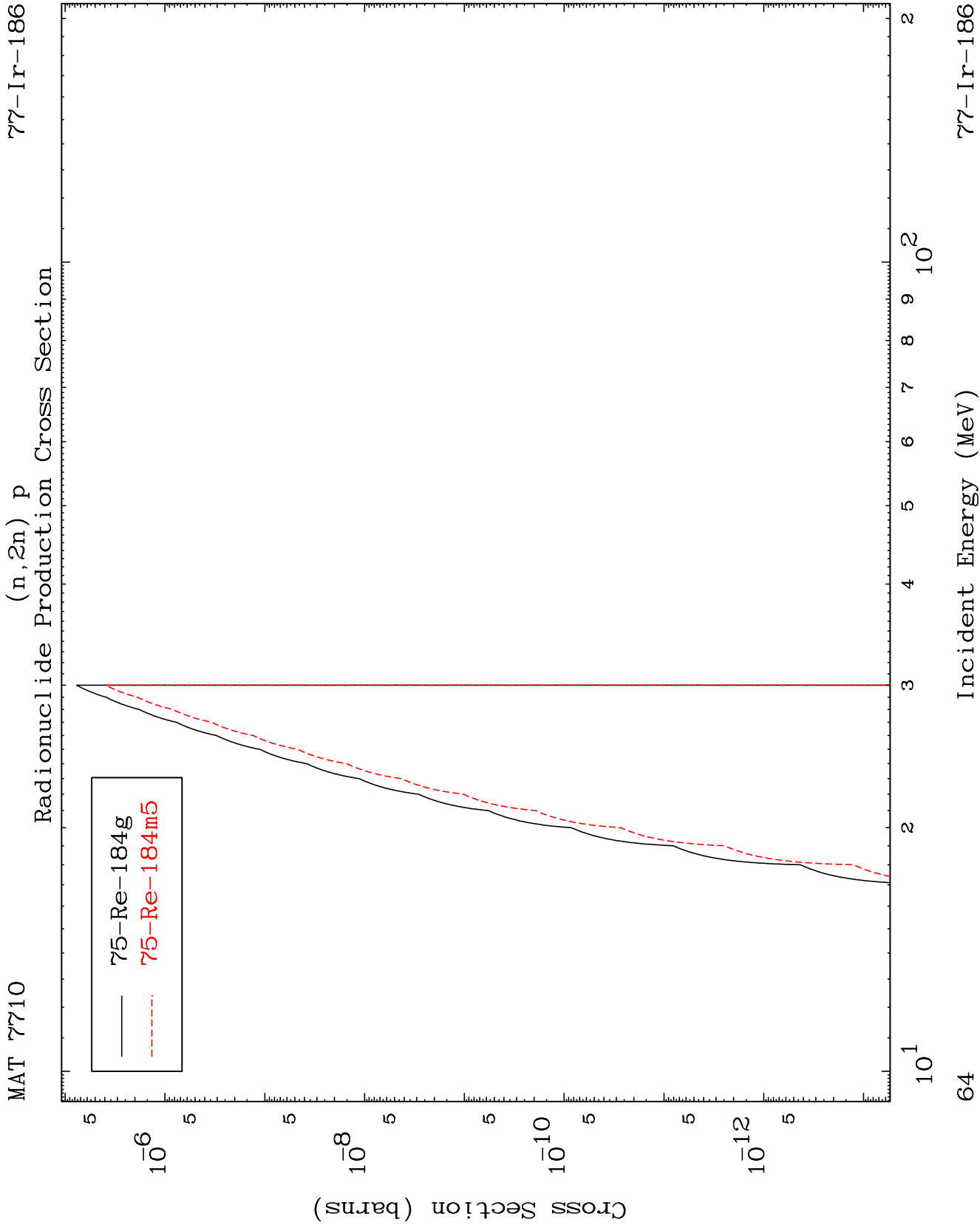
62

Incident Energy (MeV)

77-Ir-186



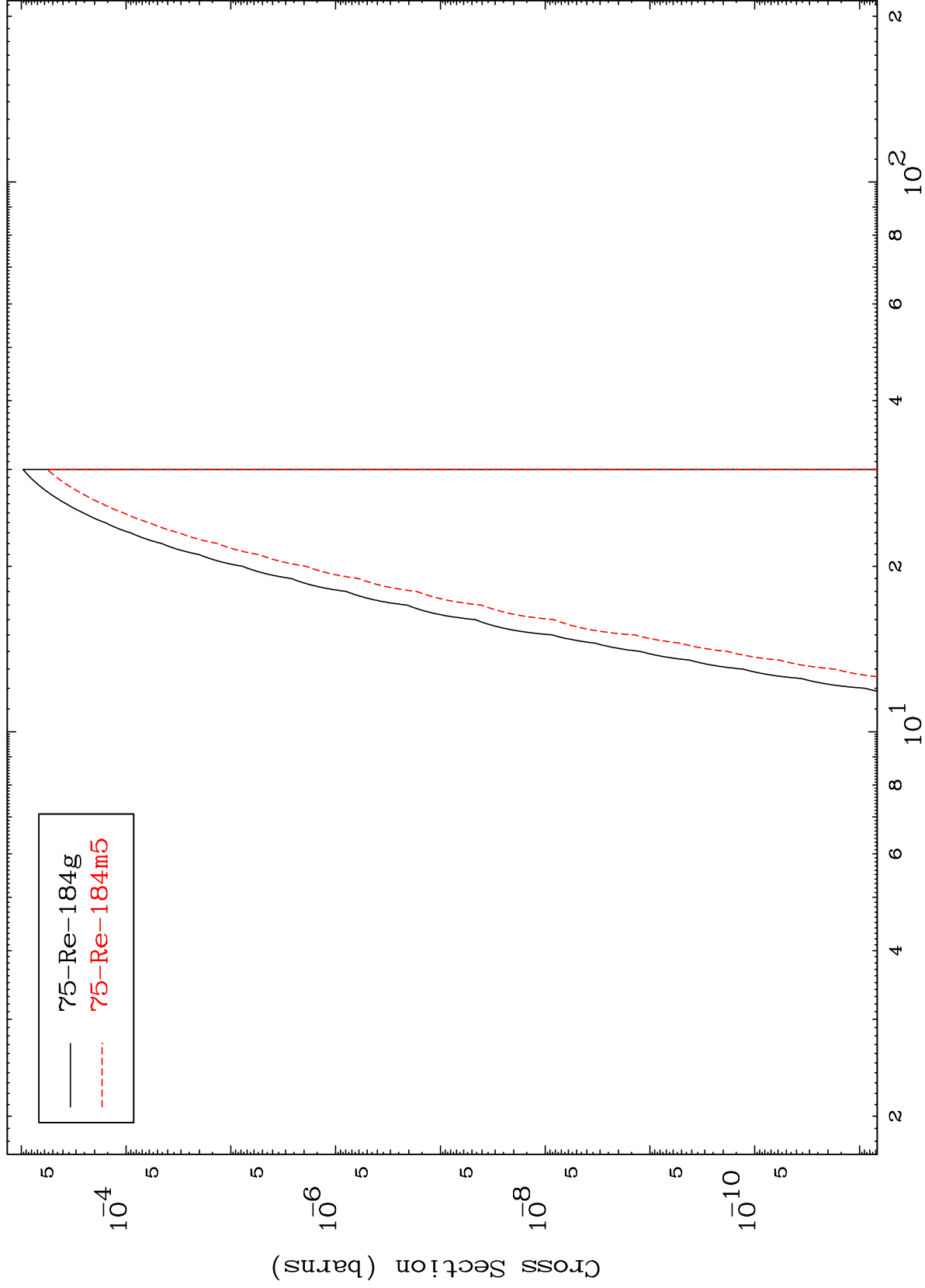




MAT 7710

77-Ir-186

Radionuclide Production Cross Section



65

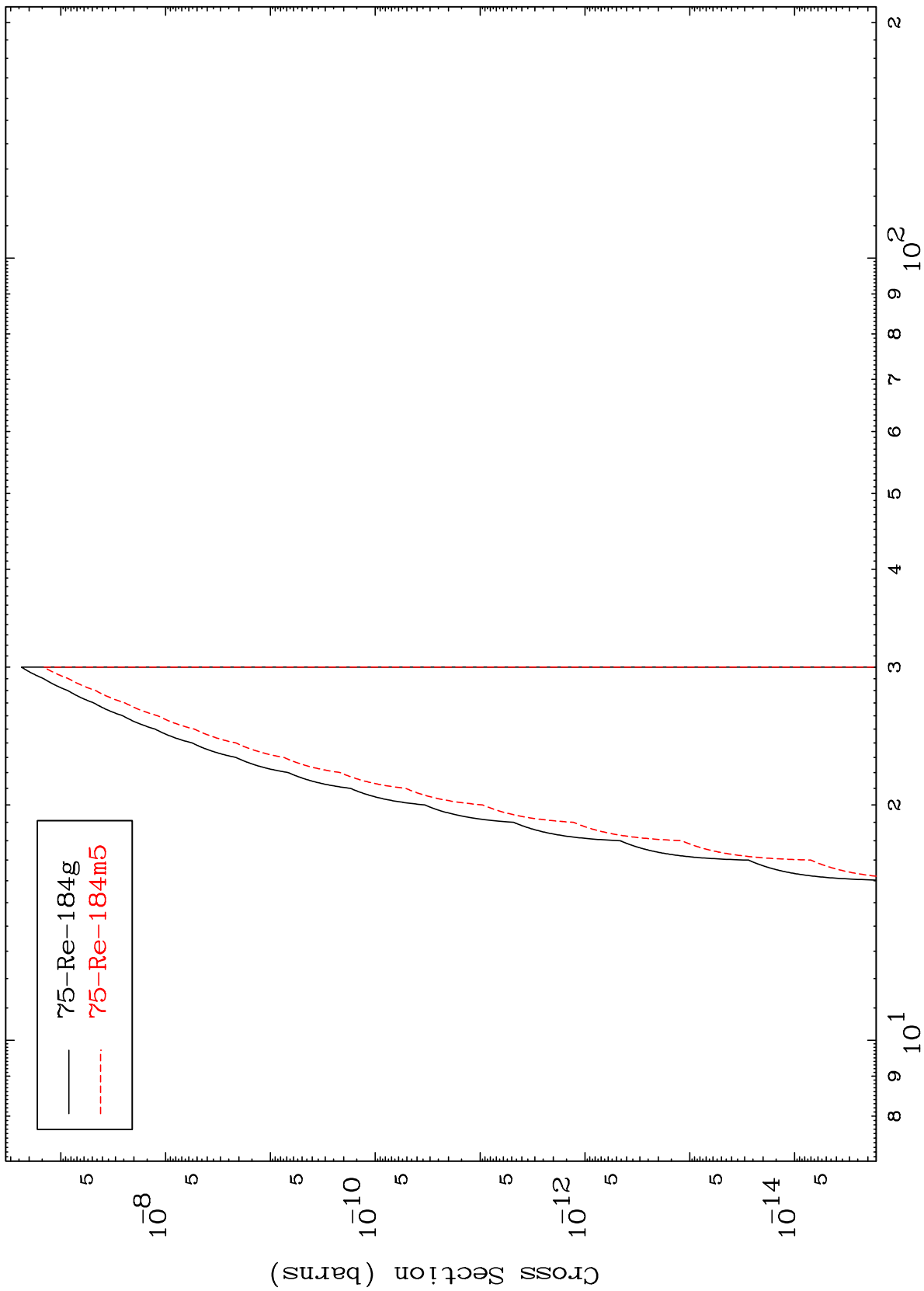
Incident Energy (MeV)

77-Ir-186

MAT 7710

77-Ir-186

(n,p) d  
Radionuclide Production Cross Section



66

Incident Energy (MeV)

77-Ir-186