

Program EVALPLOT
(Version 2021-1)

by

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(Present Contact Information)

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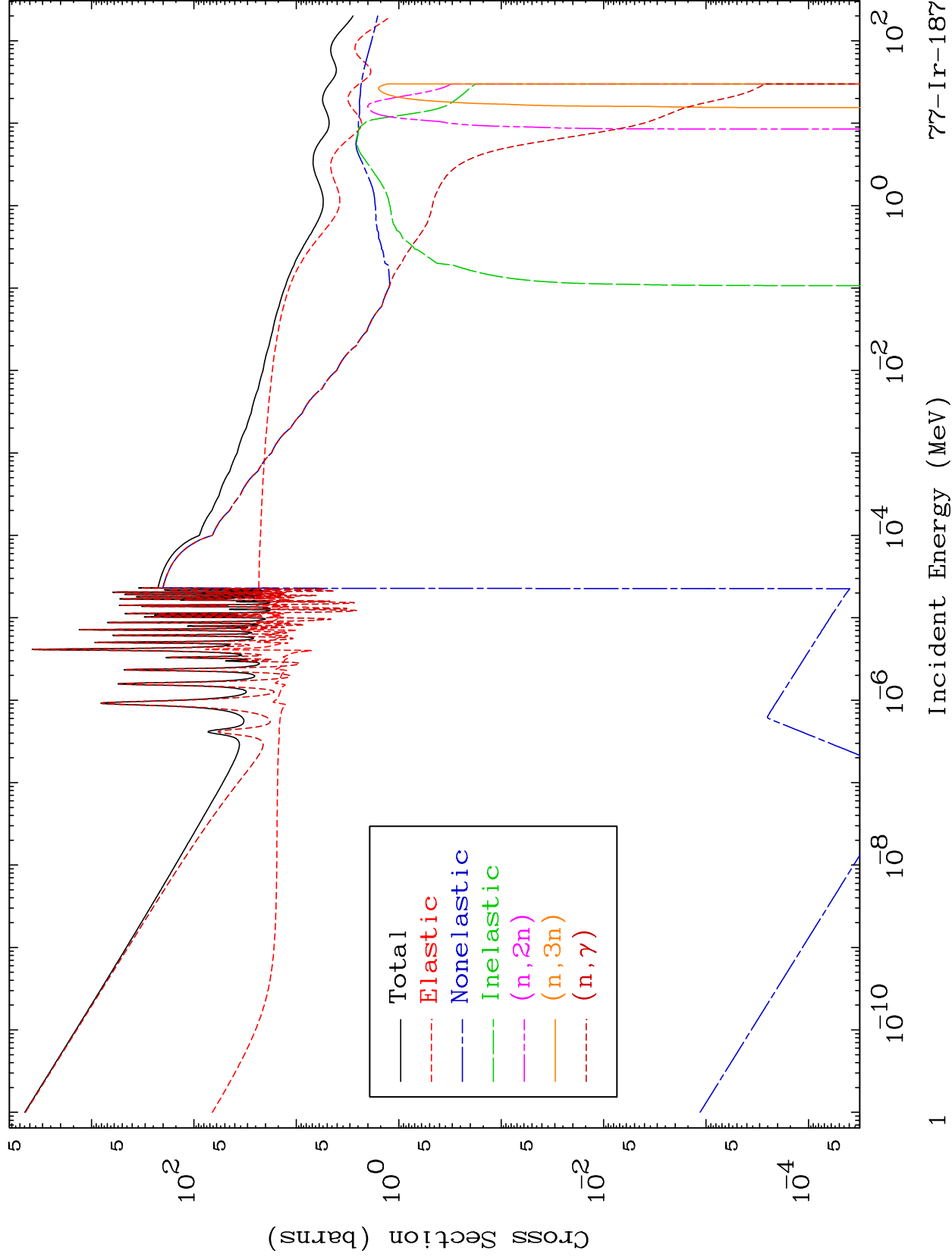
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Web:redcullen1.net/HOMEPAGE.NEW

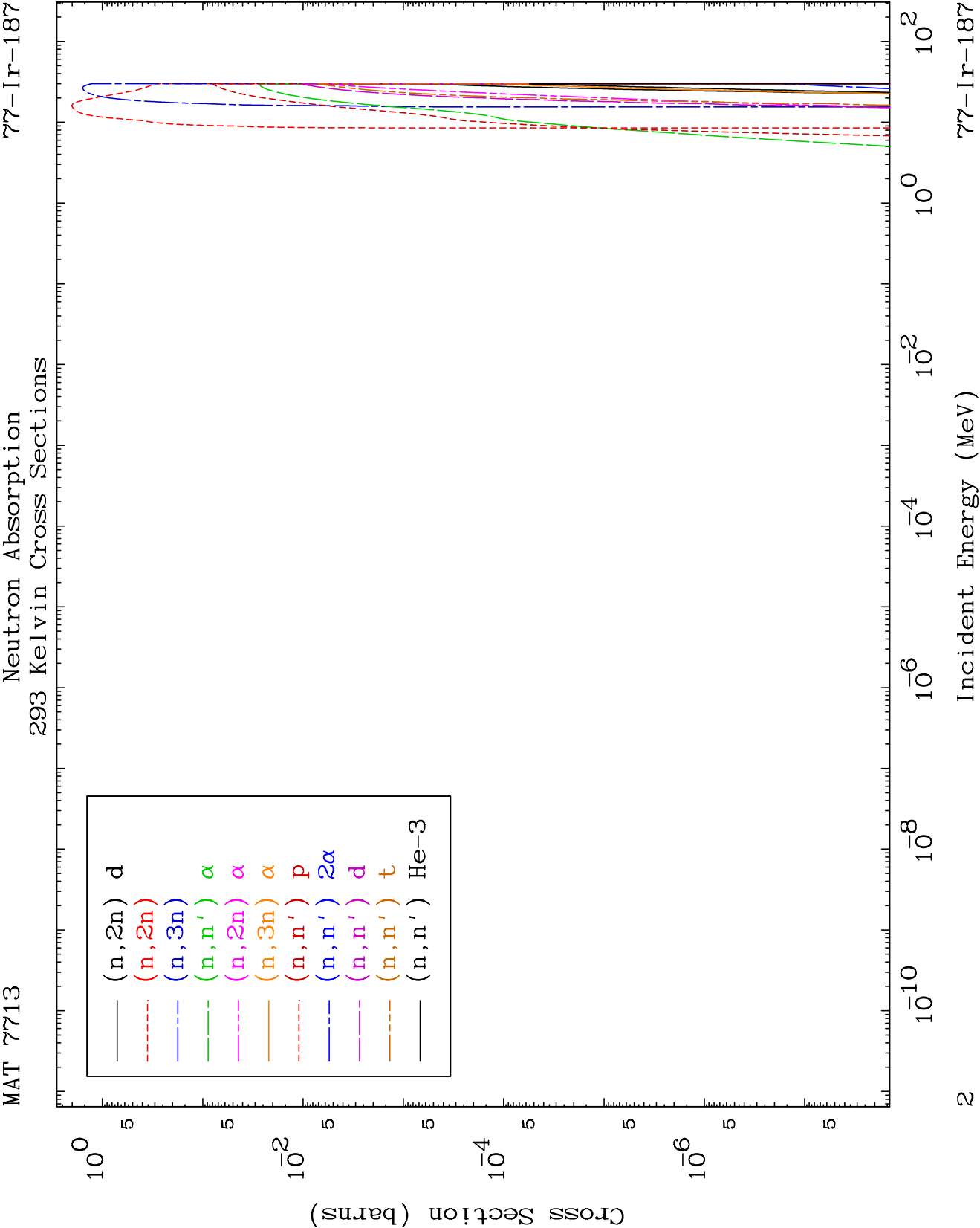
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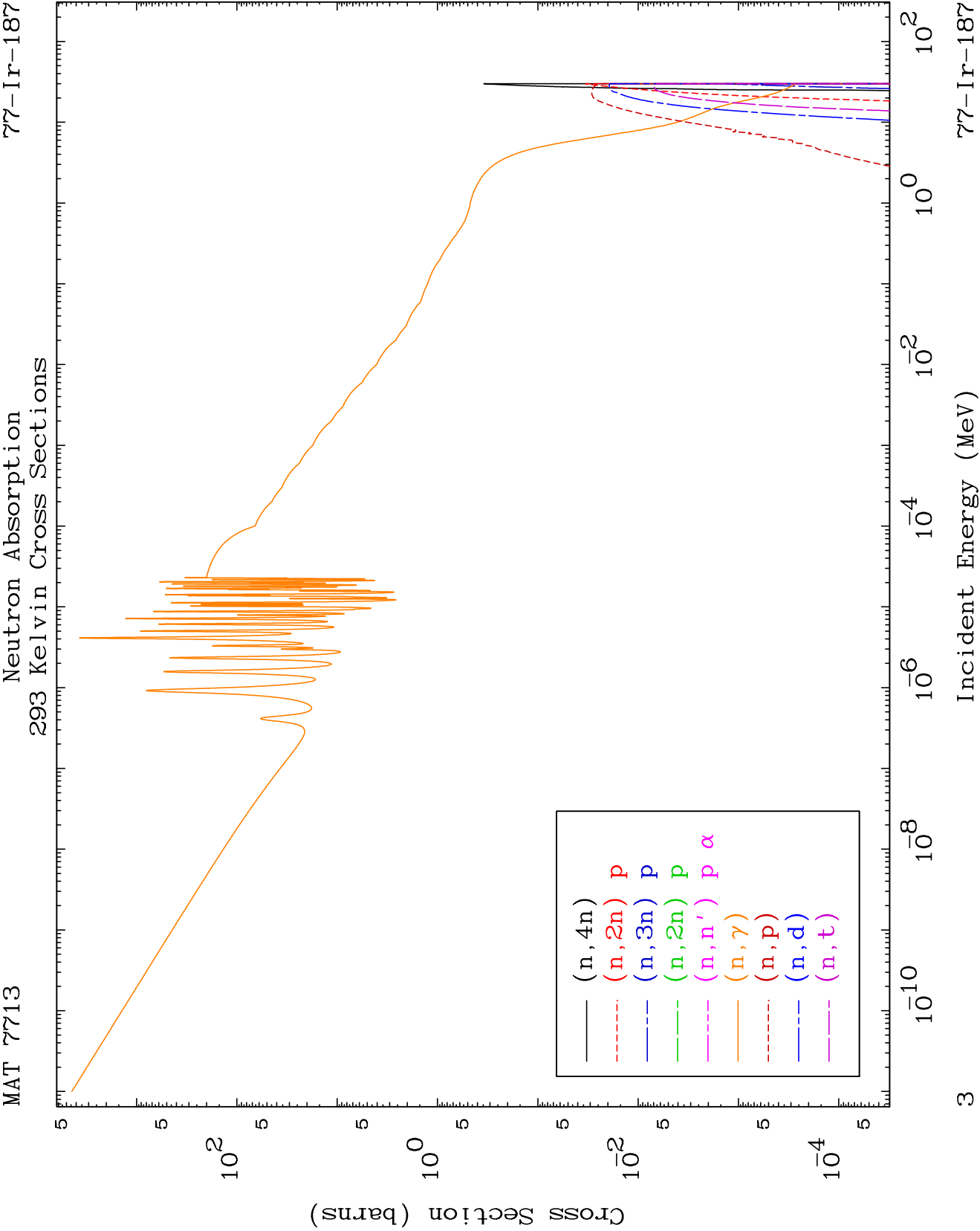
MAT 7713

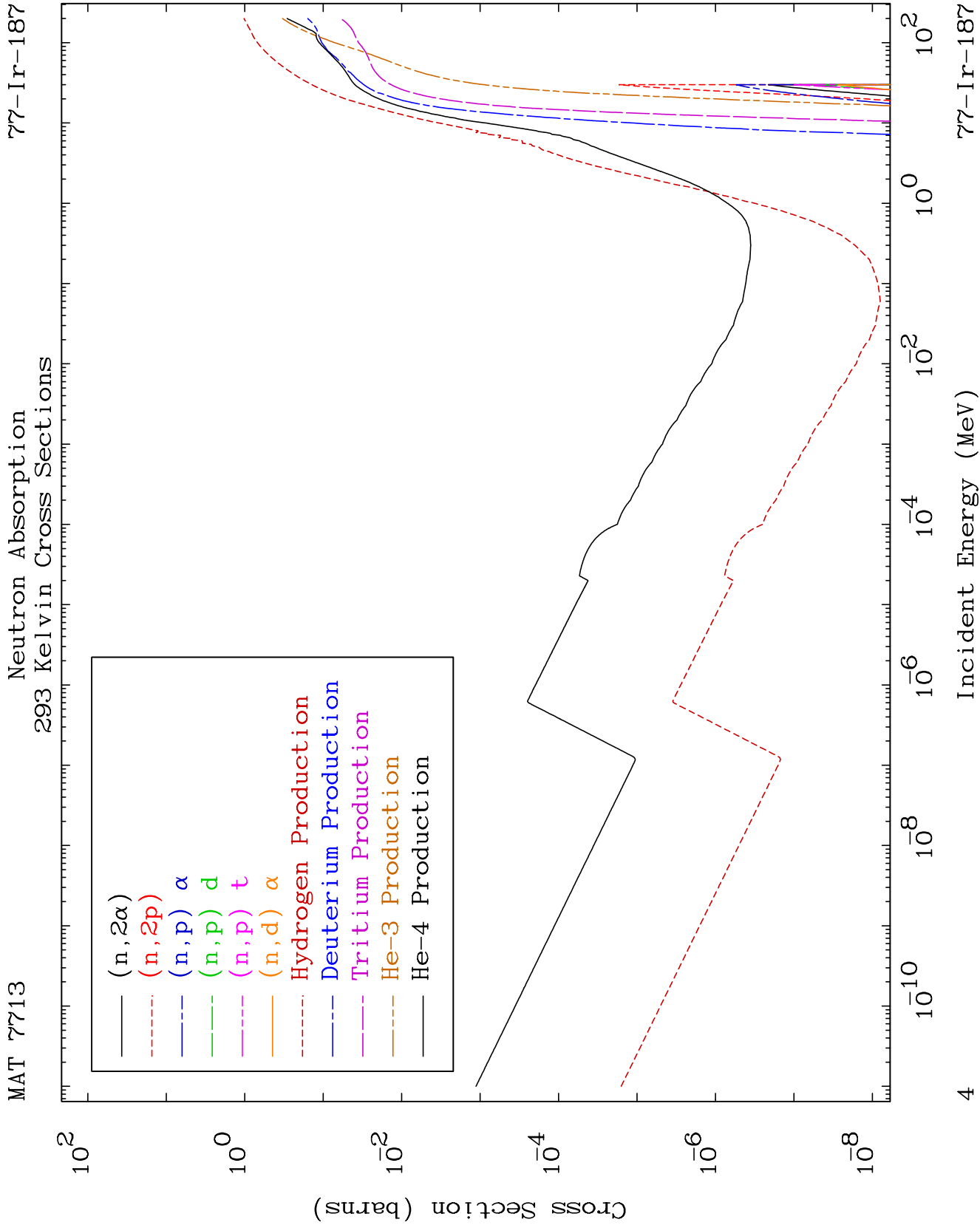
Neutron Major
293 Kelvin Cross Sections

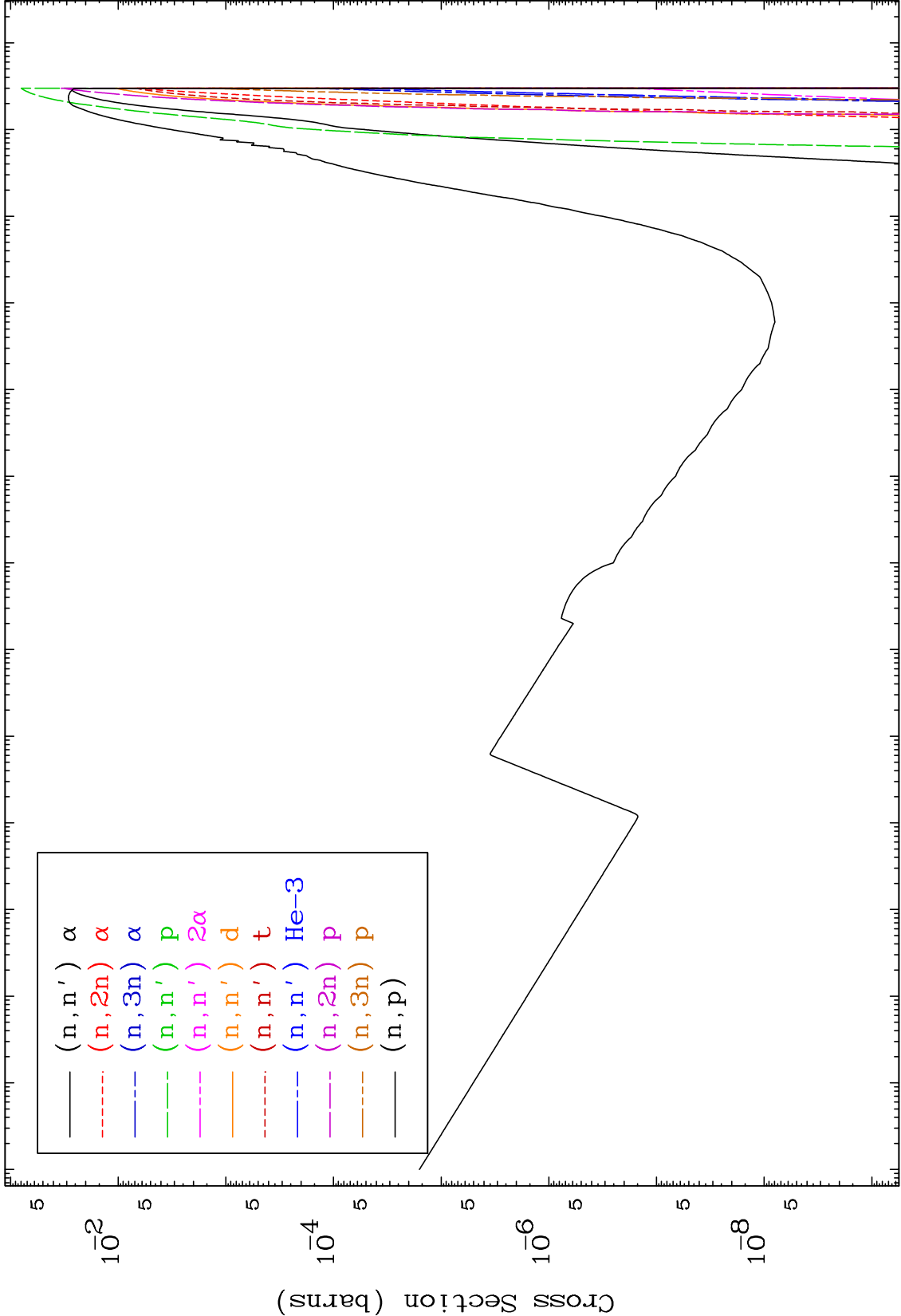
77-Ir-187

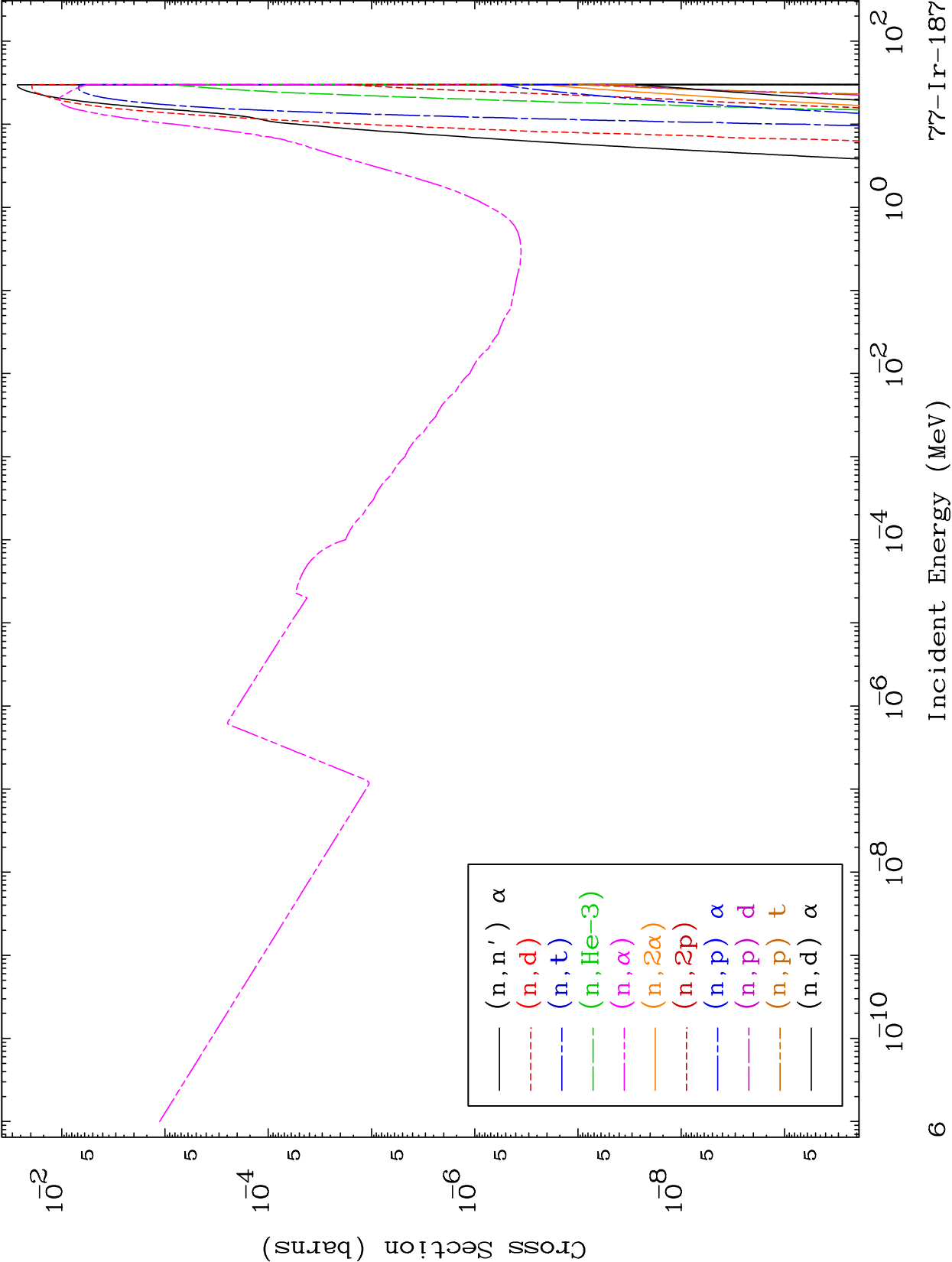


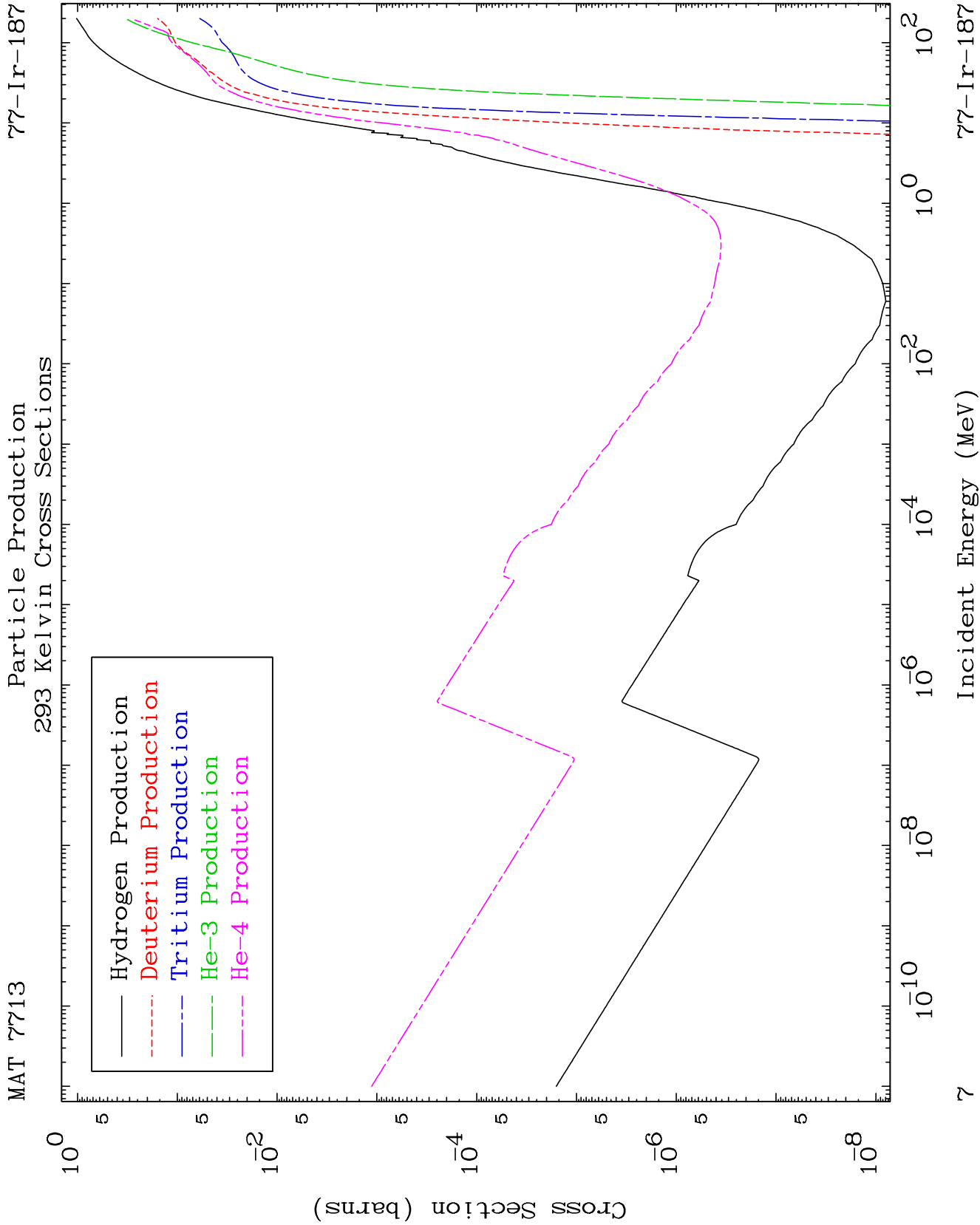








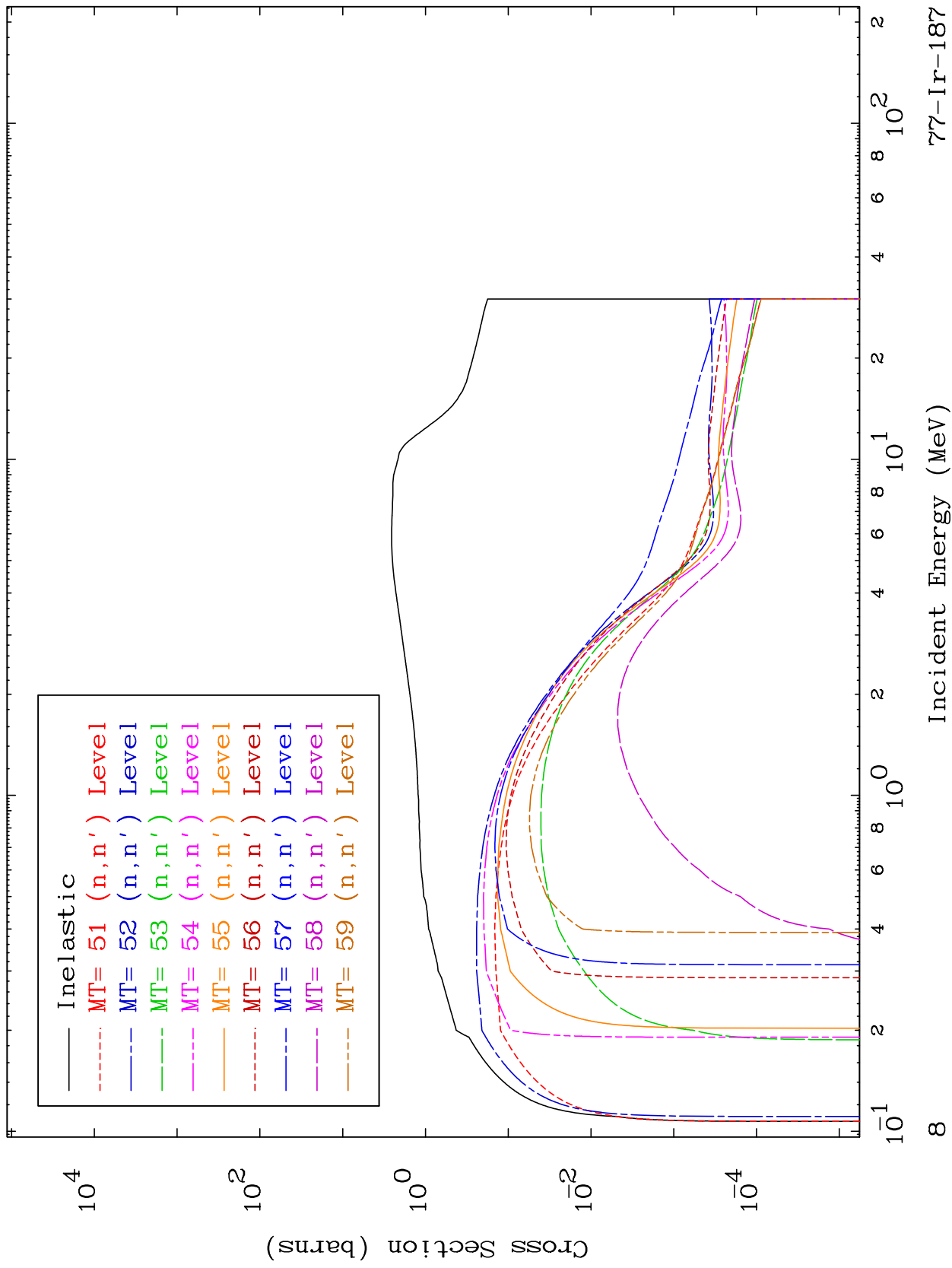


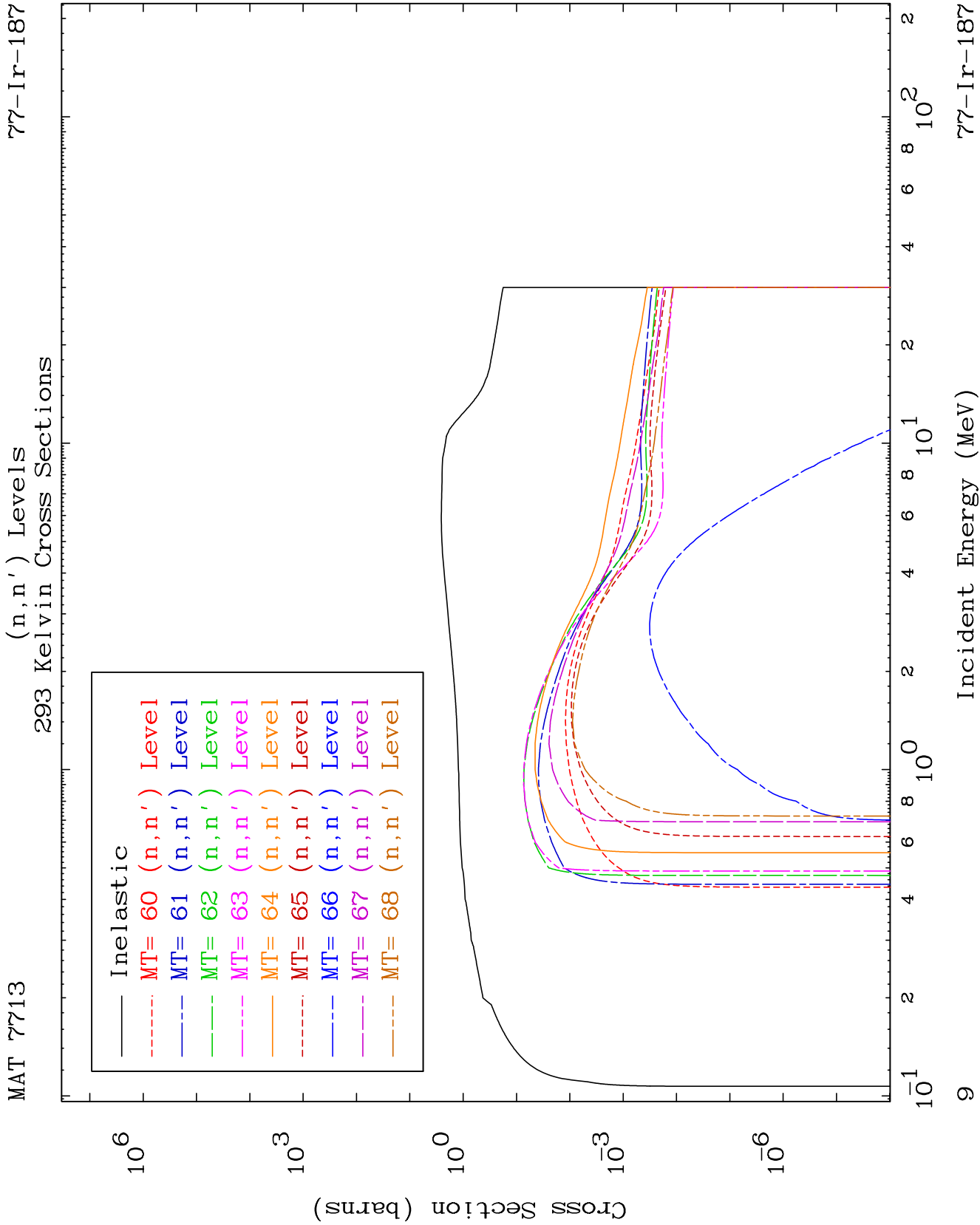


MAT 7713

77-Ir-187

(n,n') Levels
293 Kelvin Cross Sections

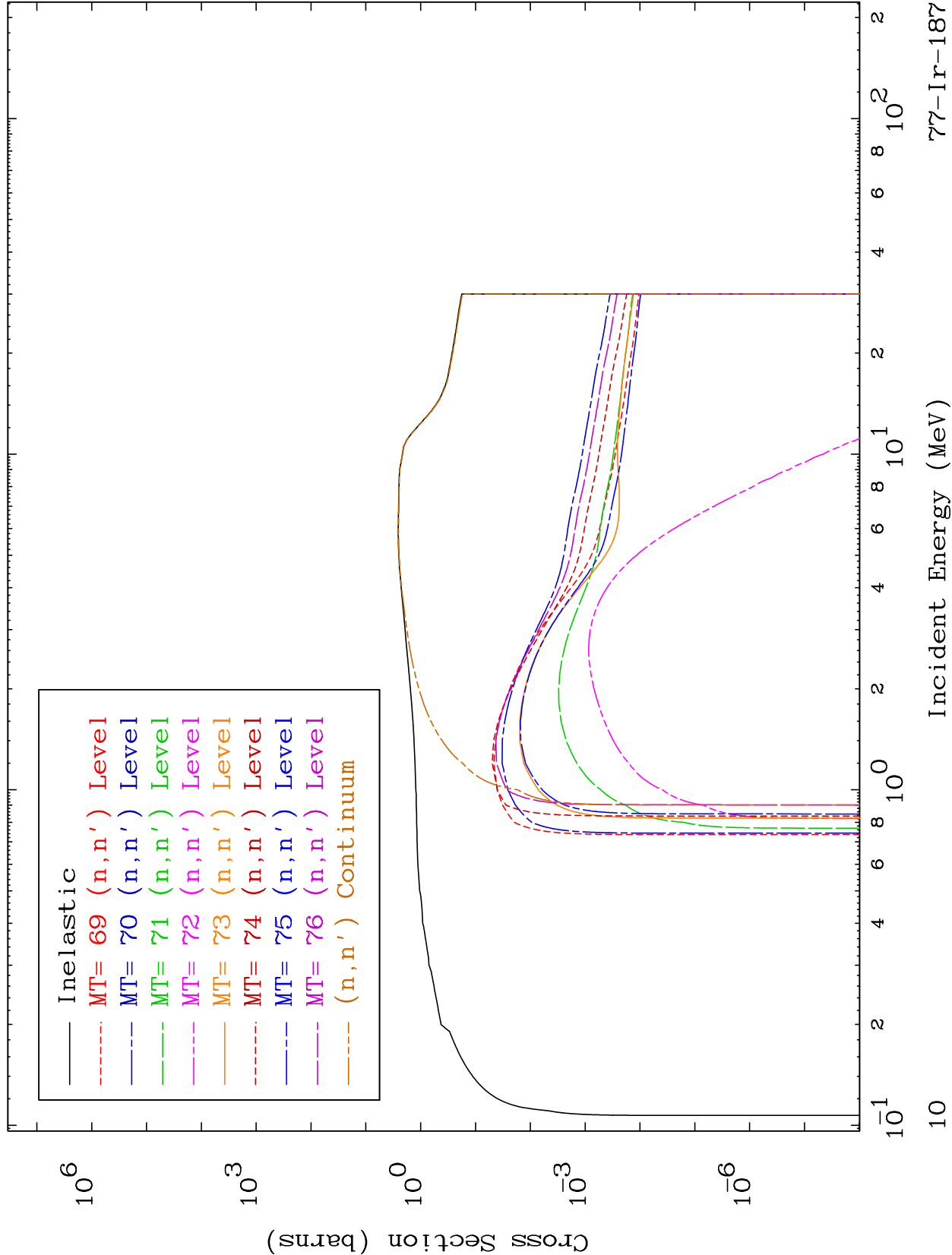




MAT 7713

(n,n') Levels
293 Kelvin Cross Sections

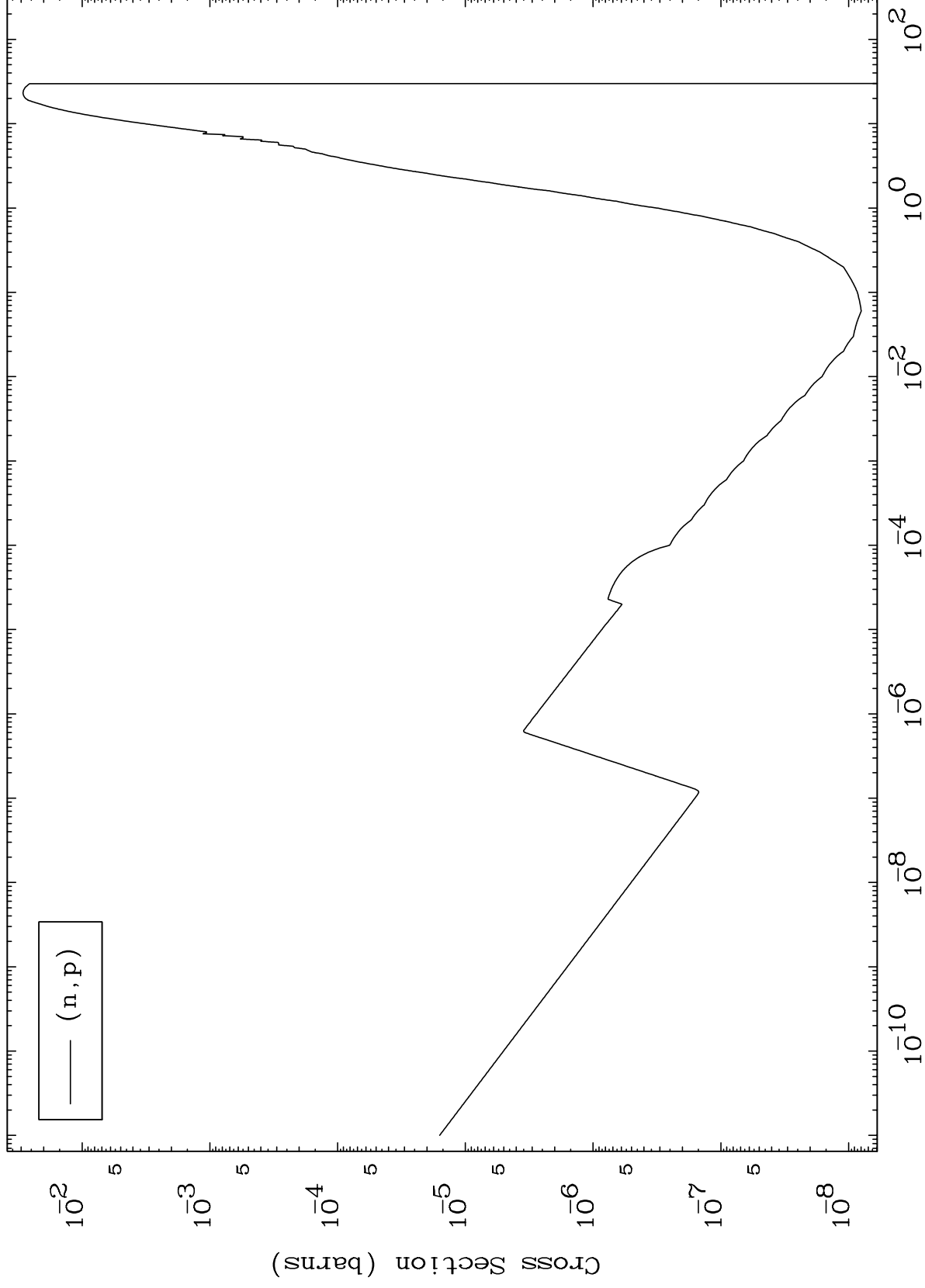
77-Ir-187



MAT 7713

(n,p) Levels
293 Kelvin Cross Sections

77-Ir-187



11

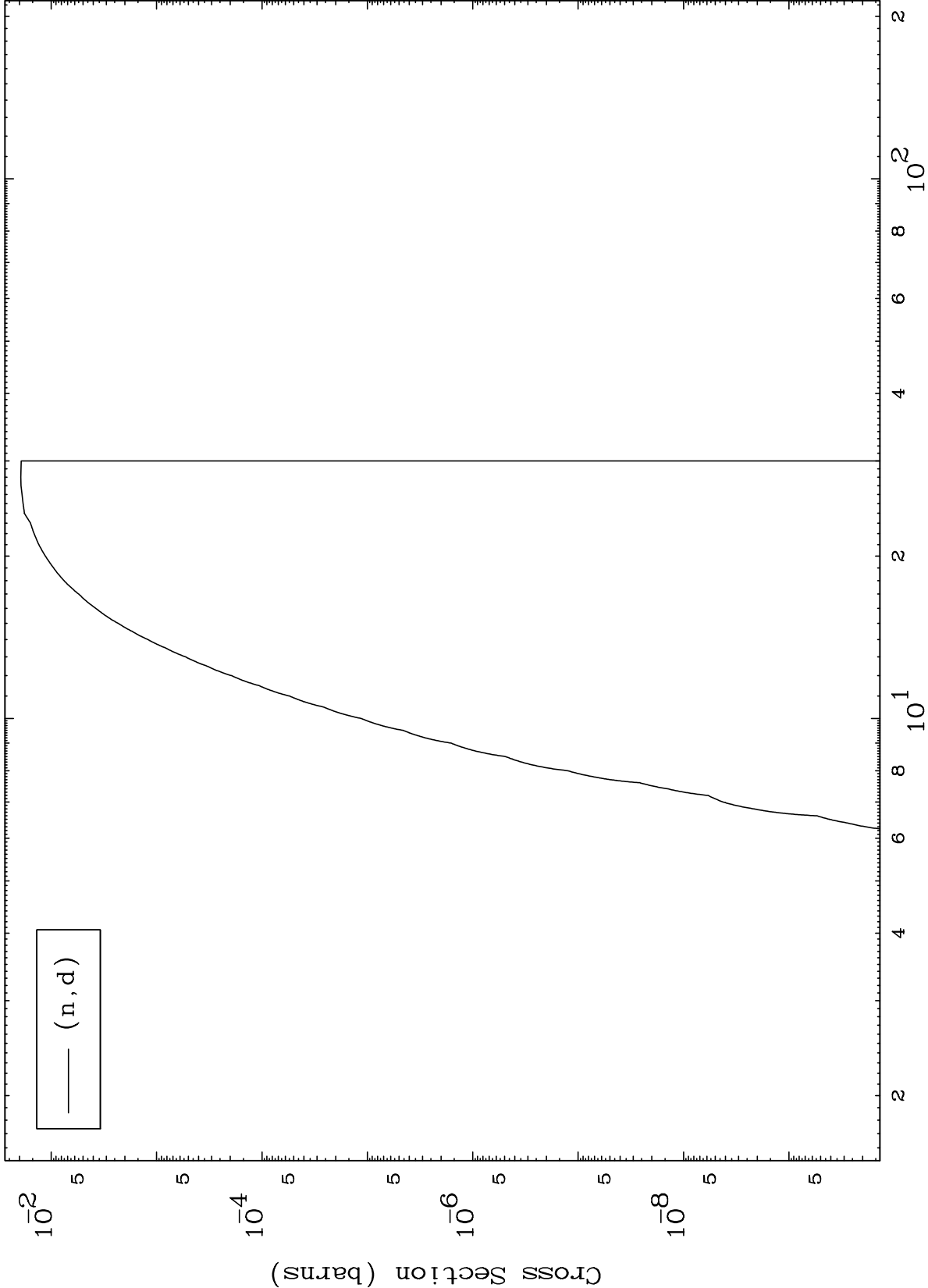
Incident Energy (MeV)

77-Ir-187

MAT 7713

(n,d) Levels
293 Kelvin Cross Sections

77-Ir-187



12

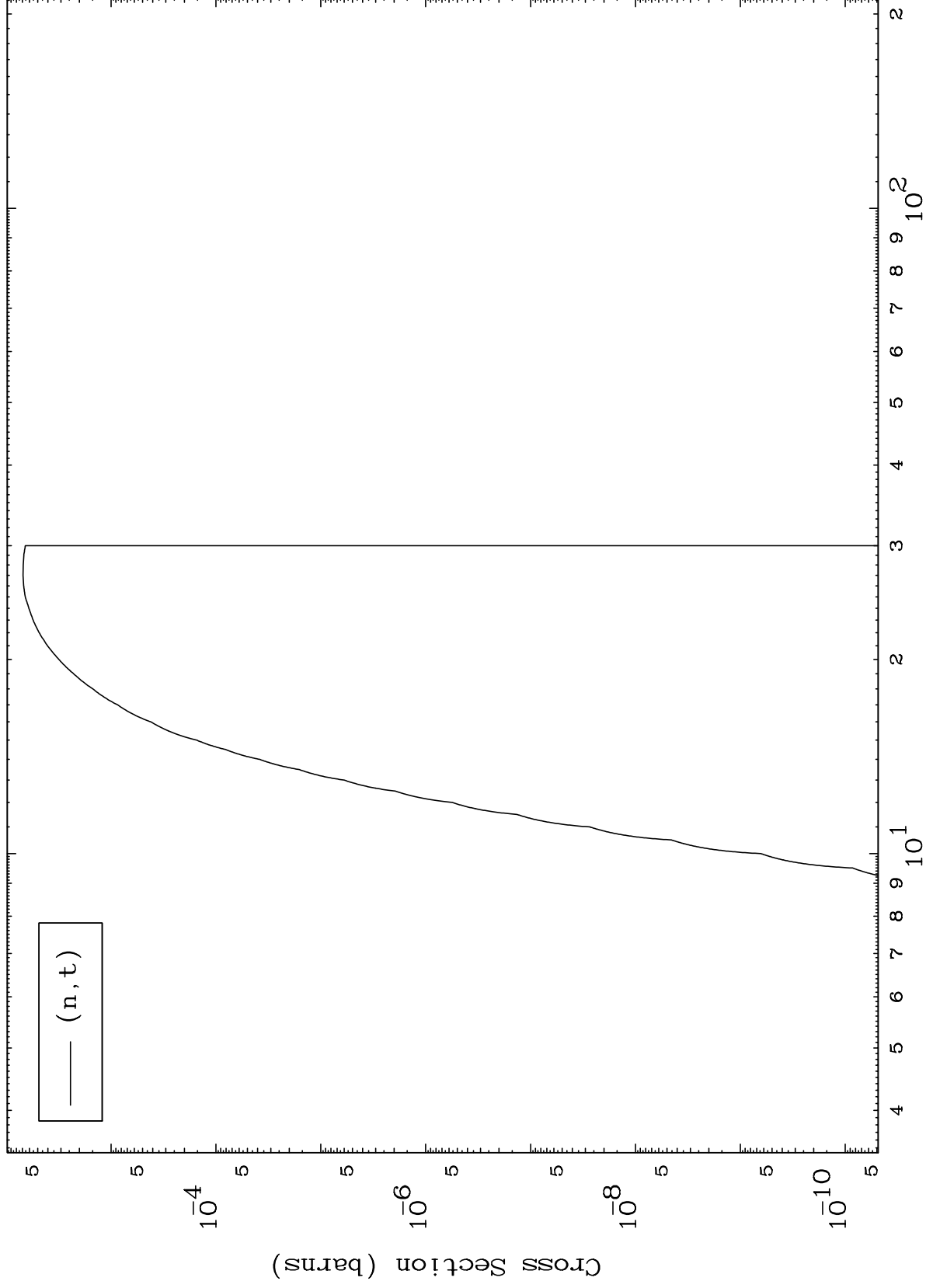
Incident Energy (MeV)

77-Ir-187

MAT 7713

77-Ir-187

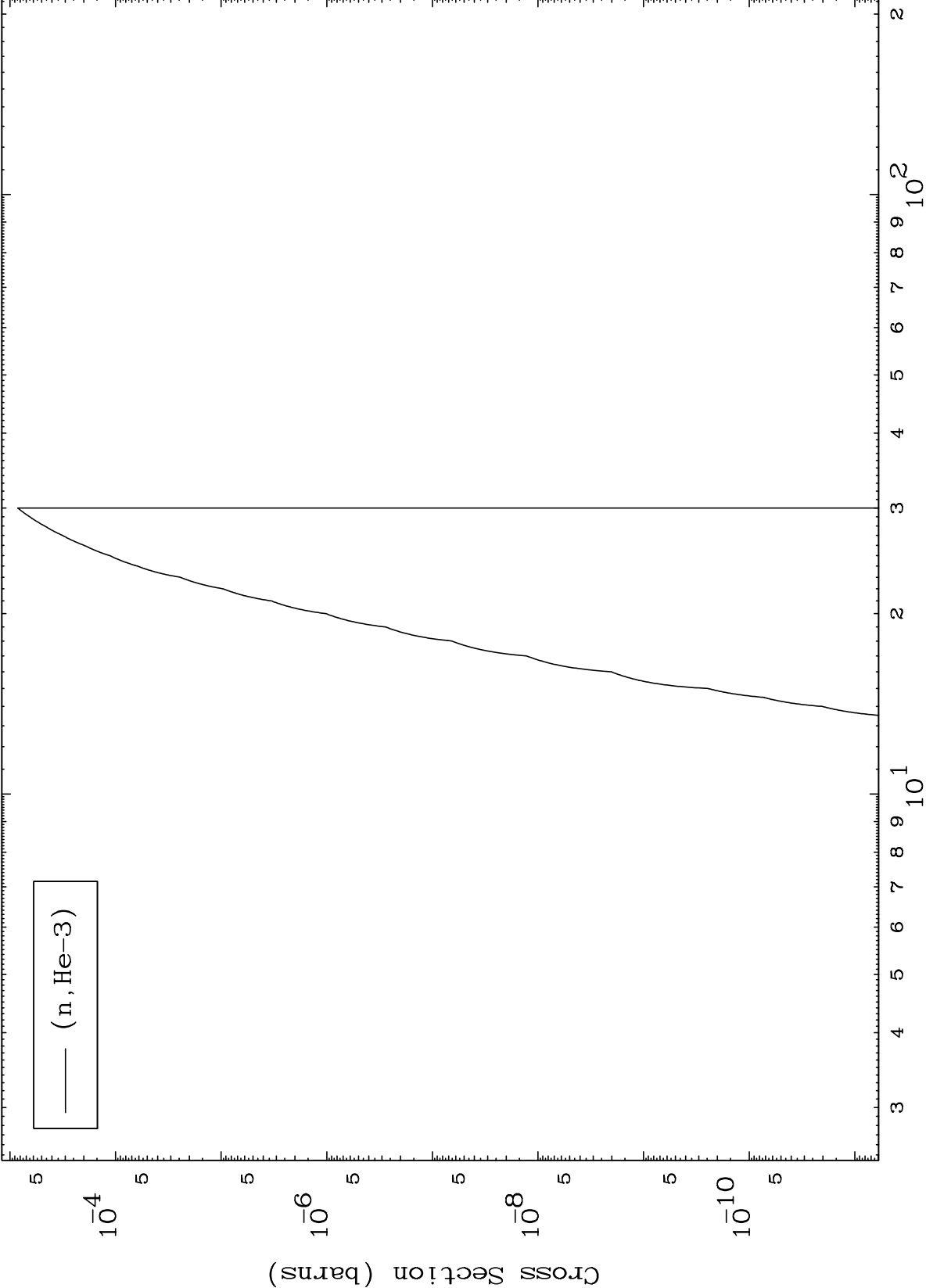
(n,t) Levels
293 Kelvin Cross Sections

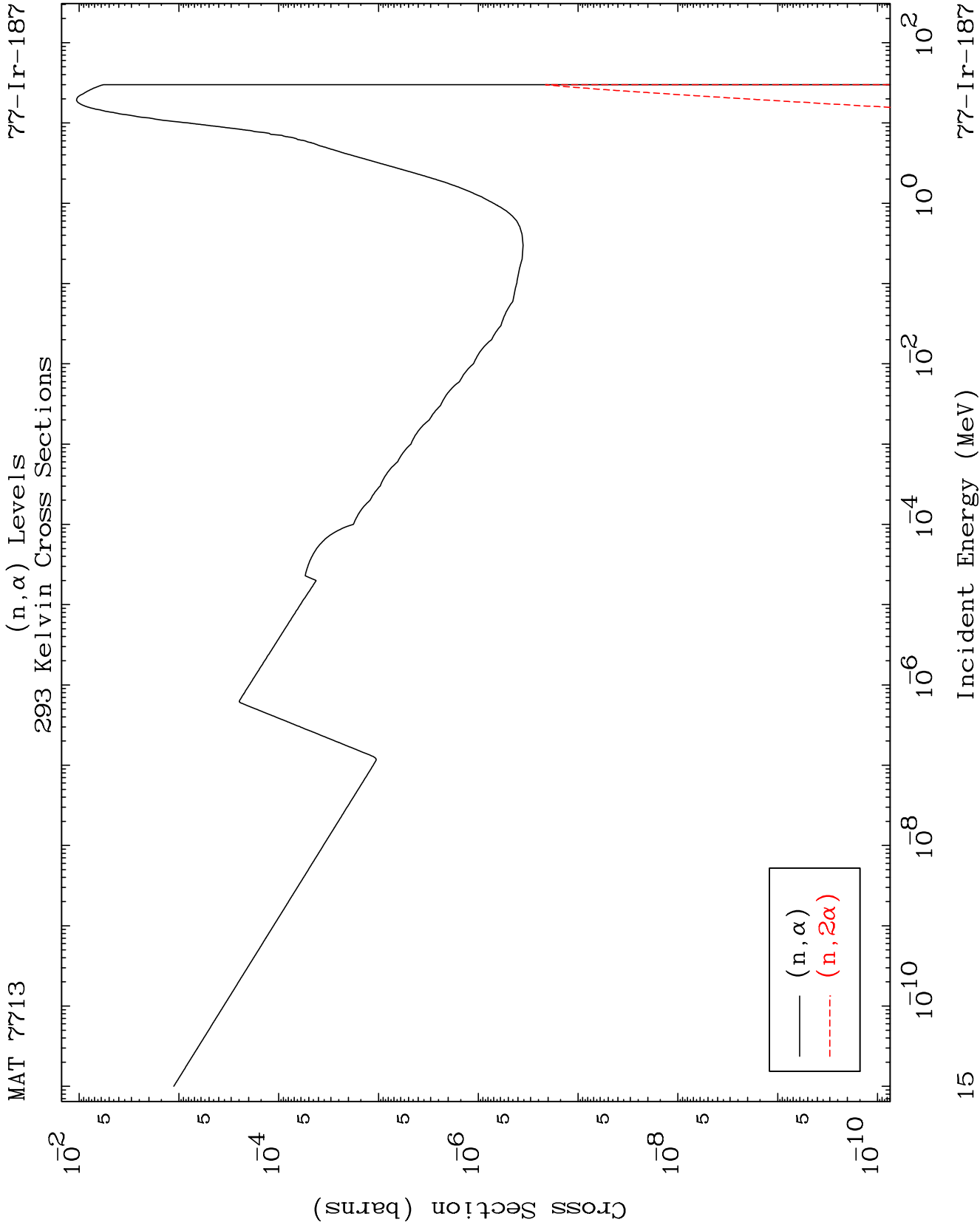


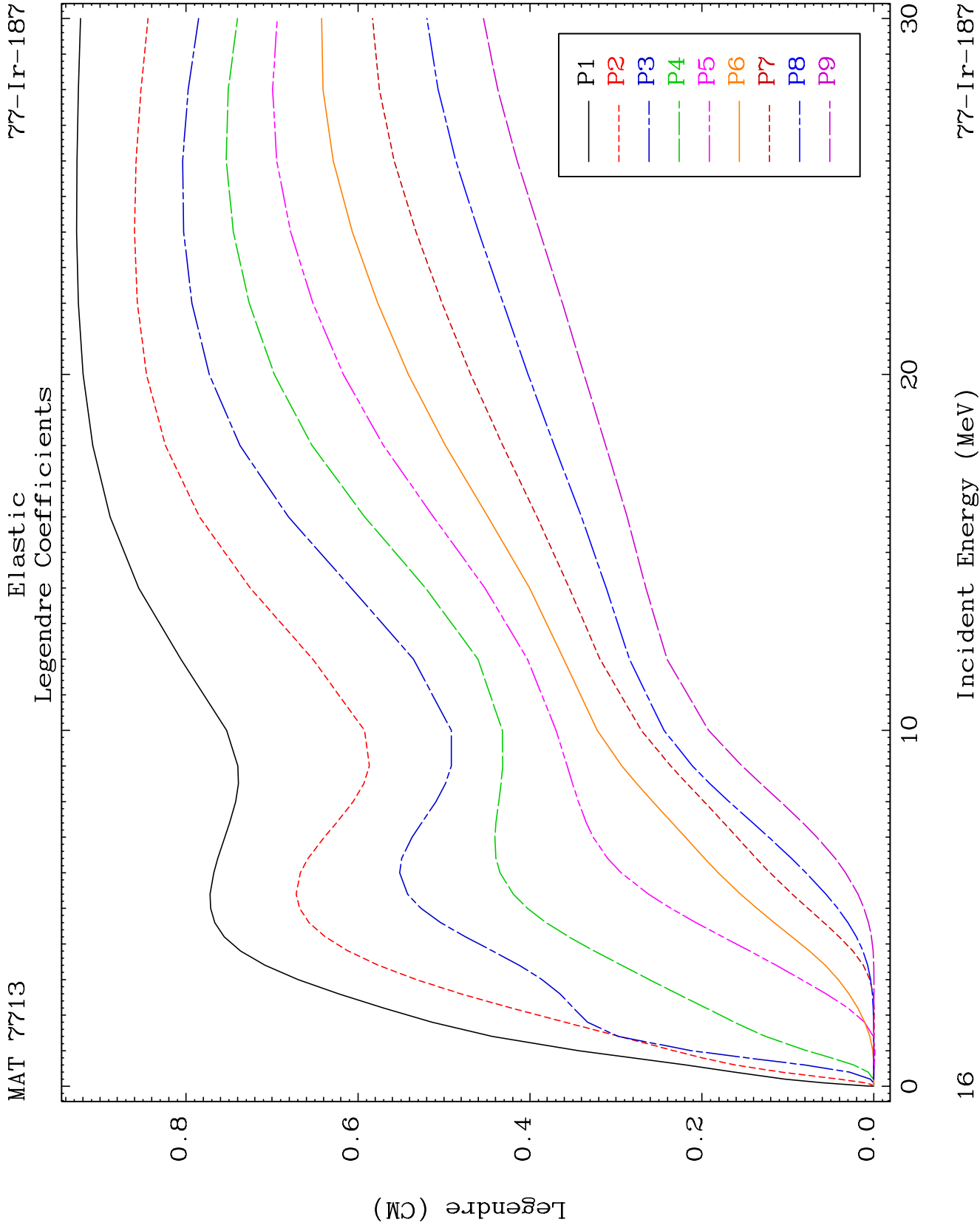
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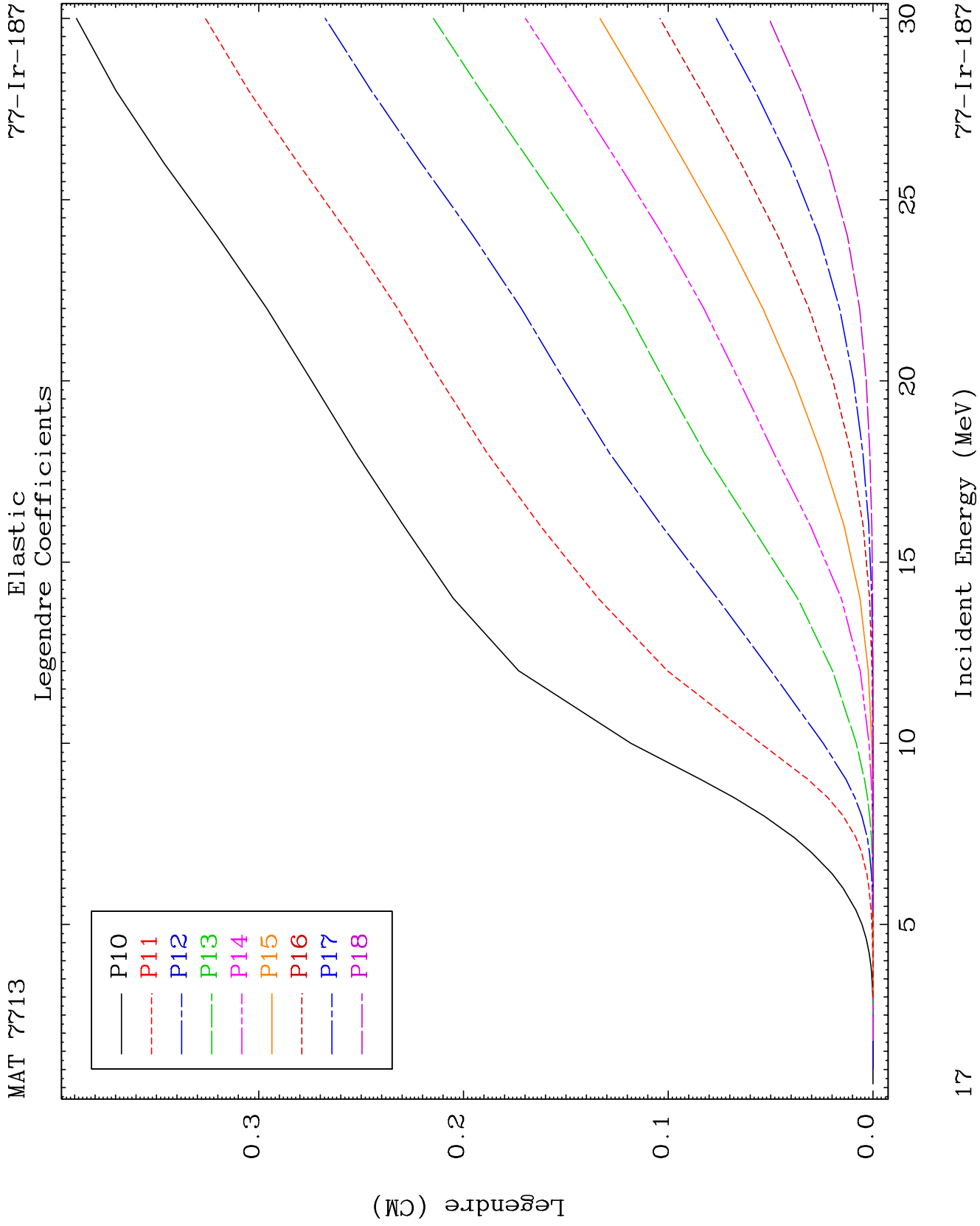
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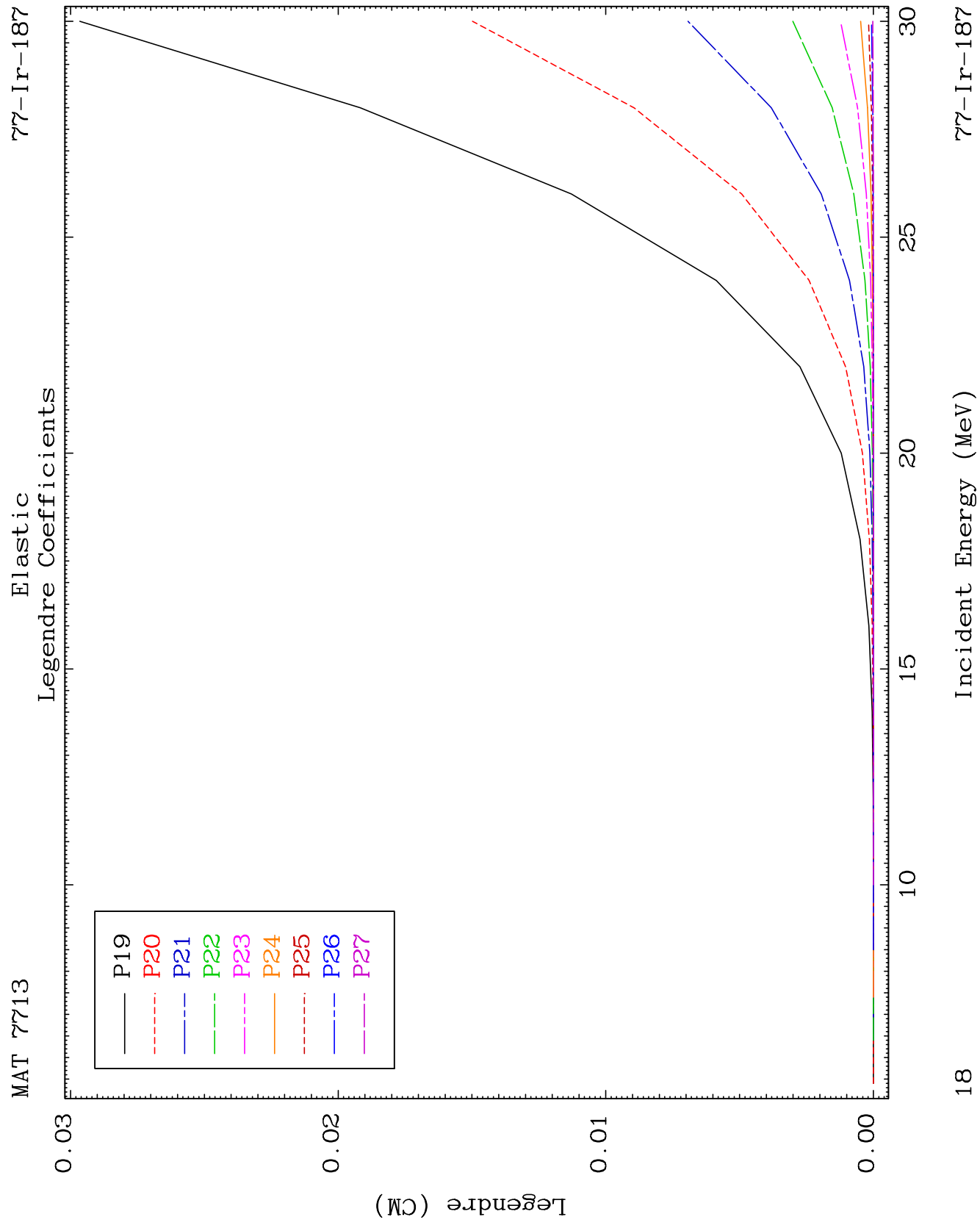
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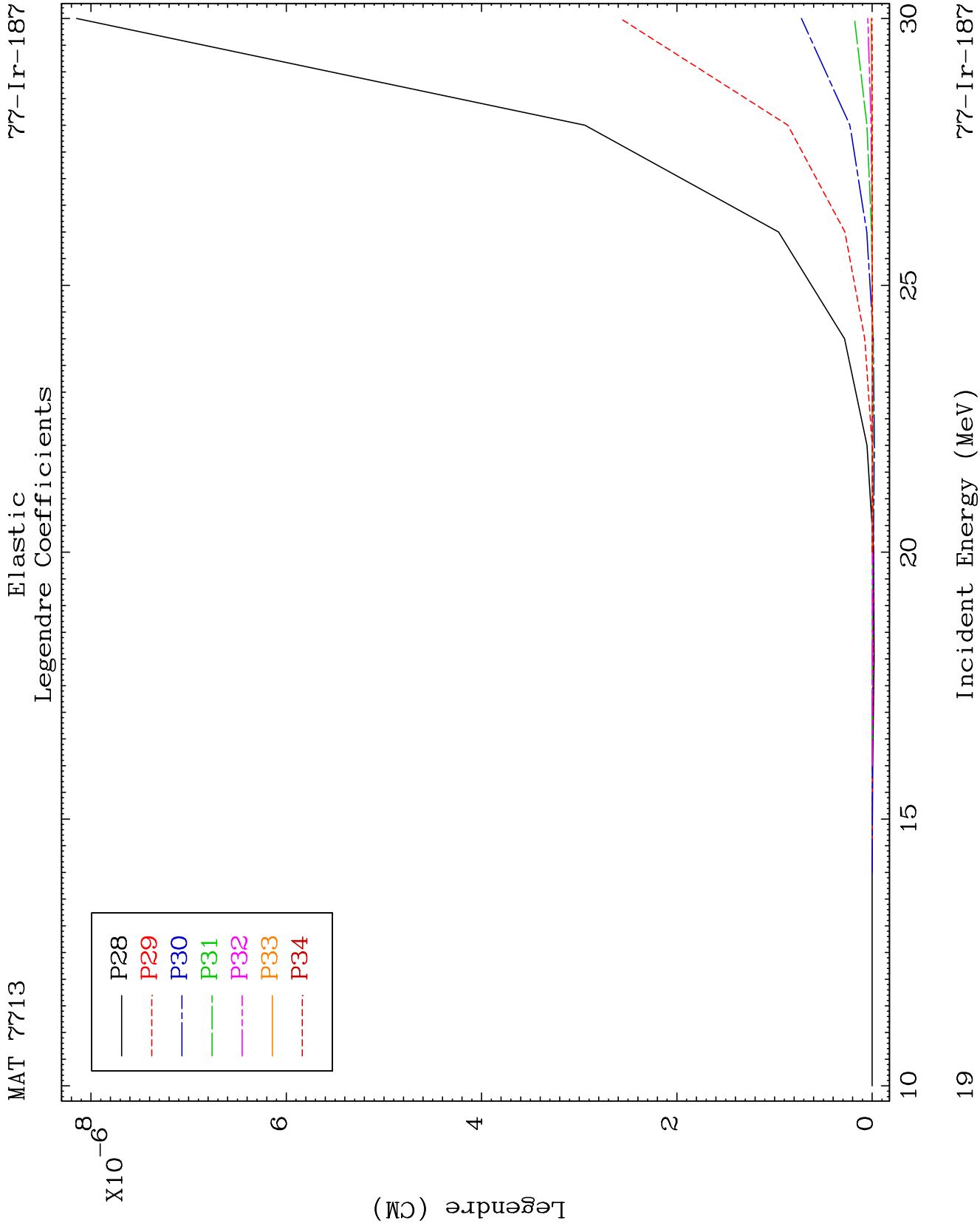


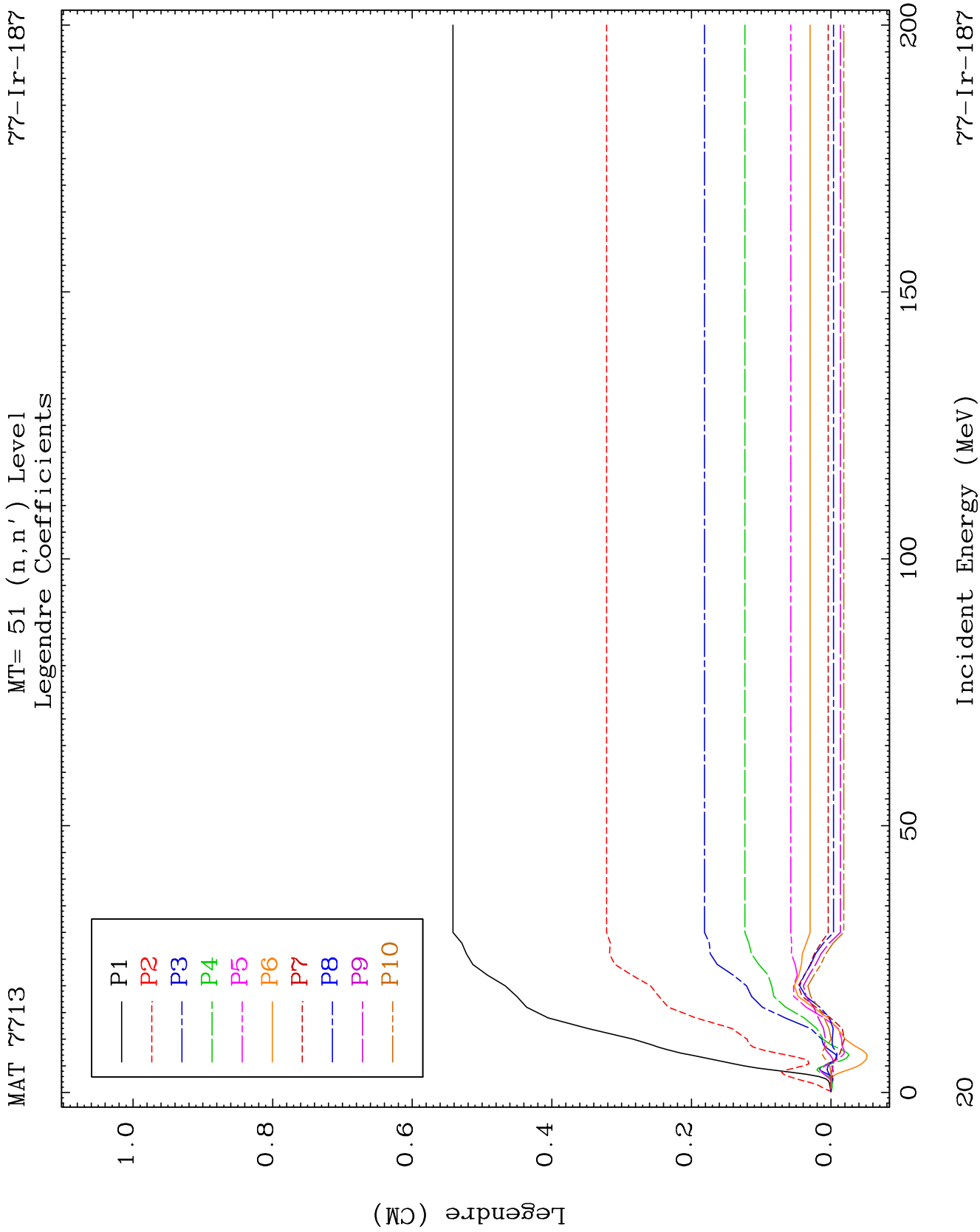


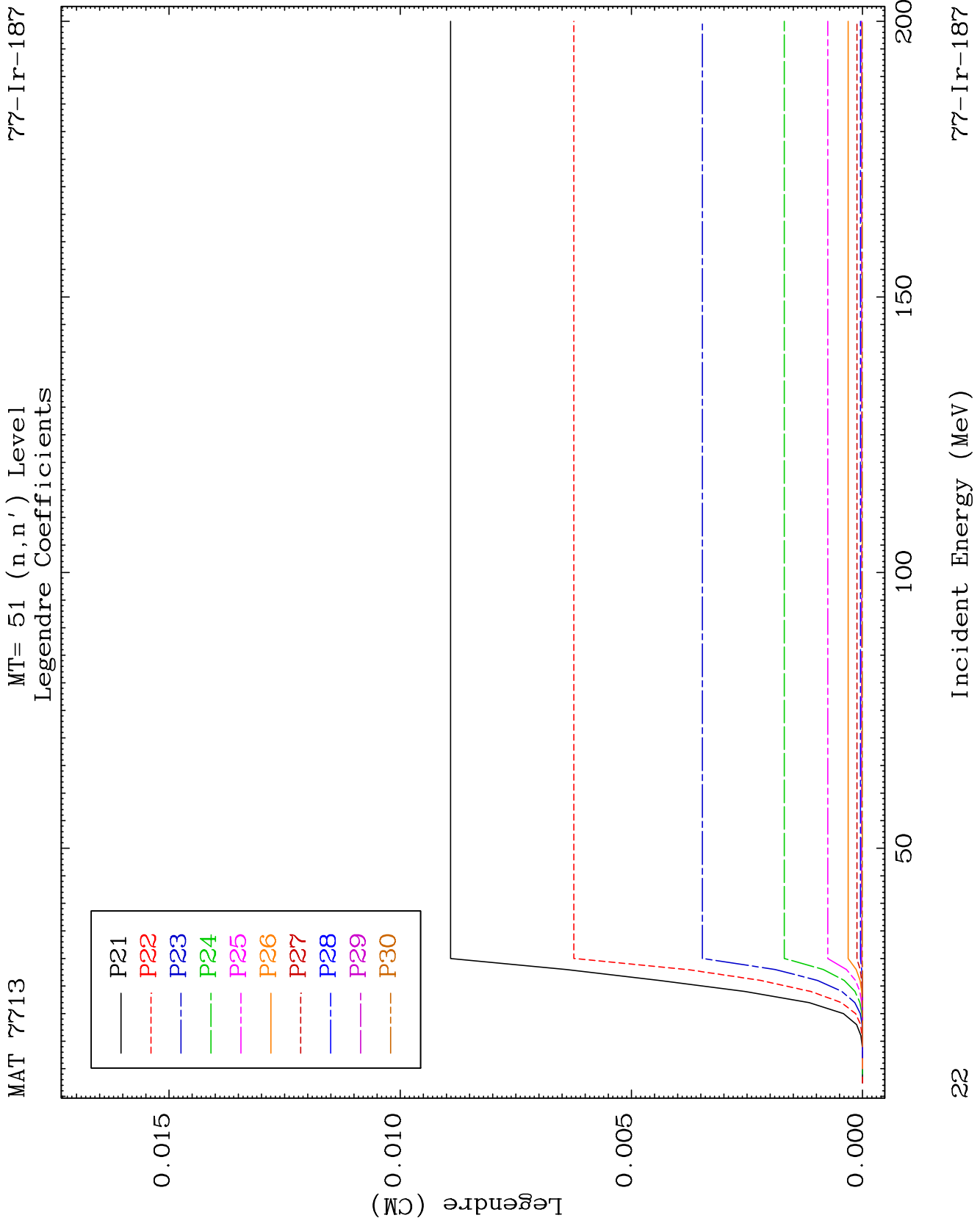








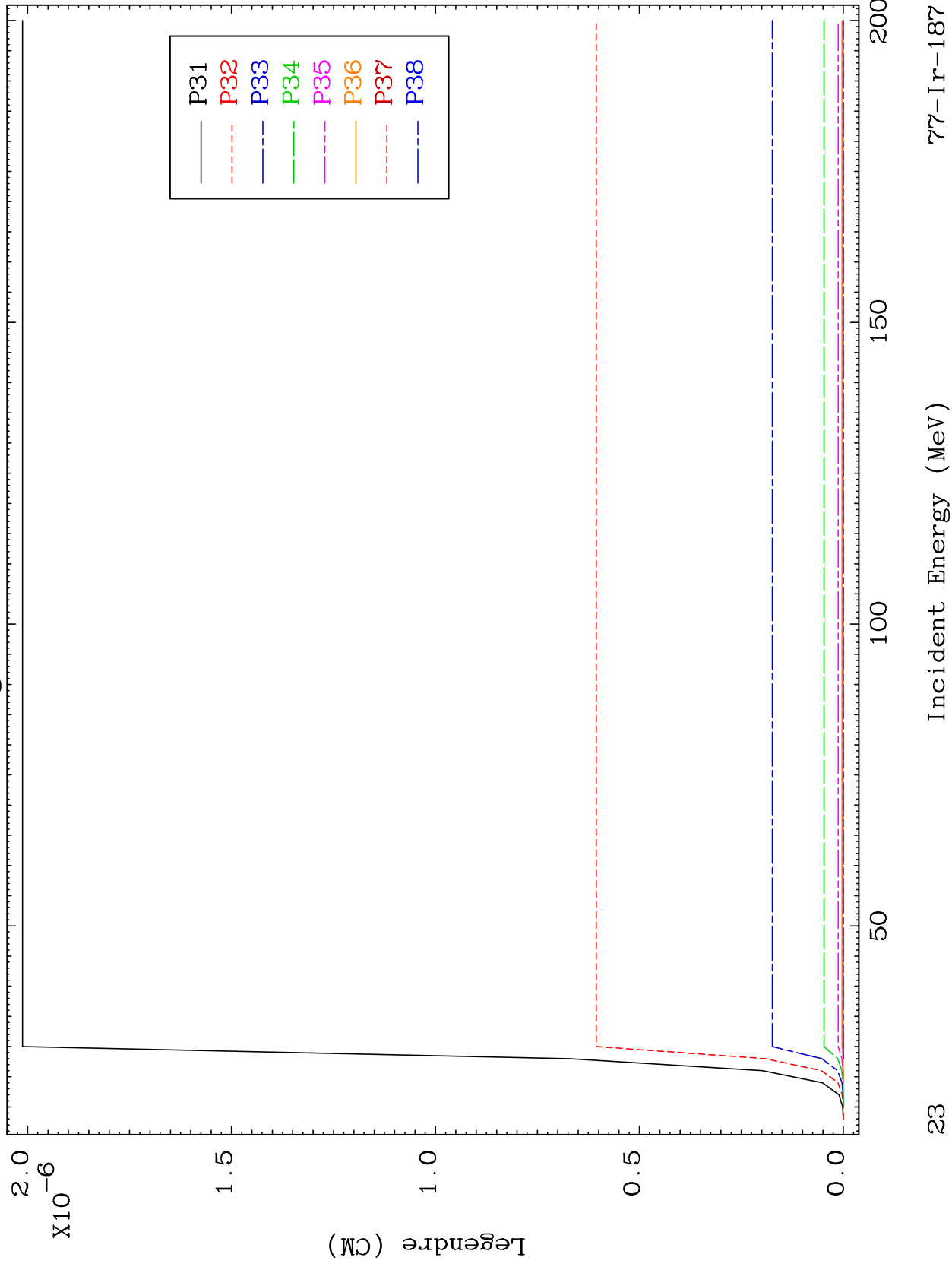


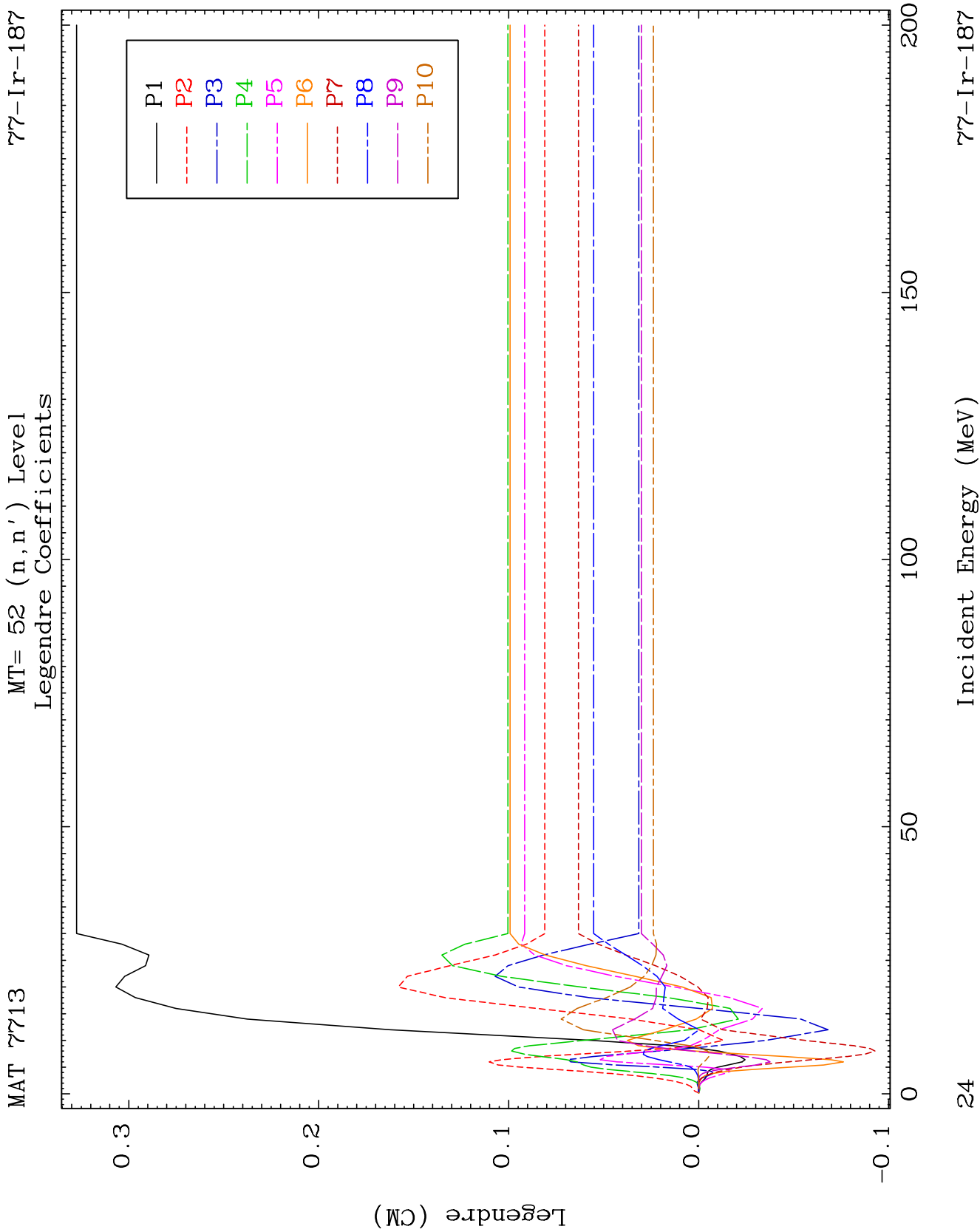


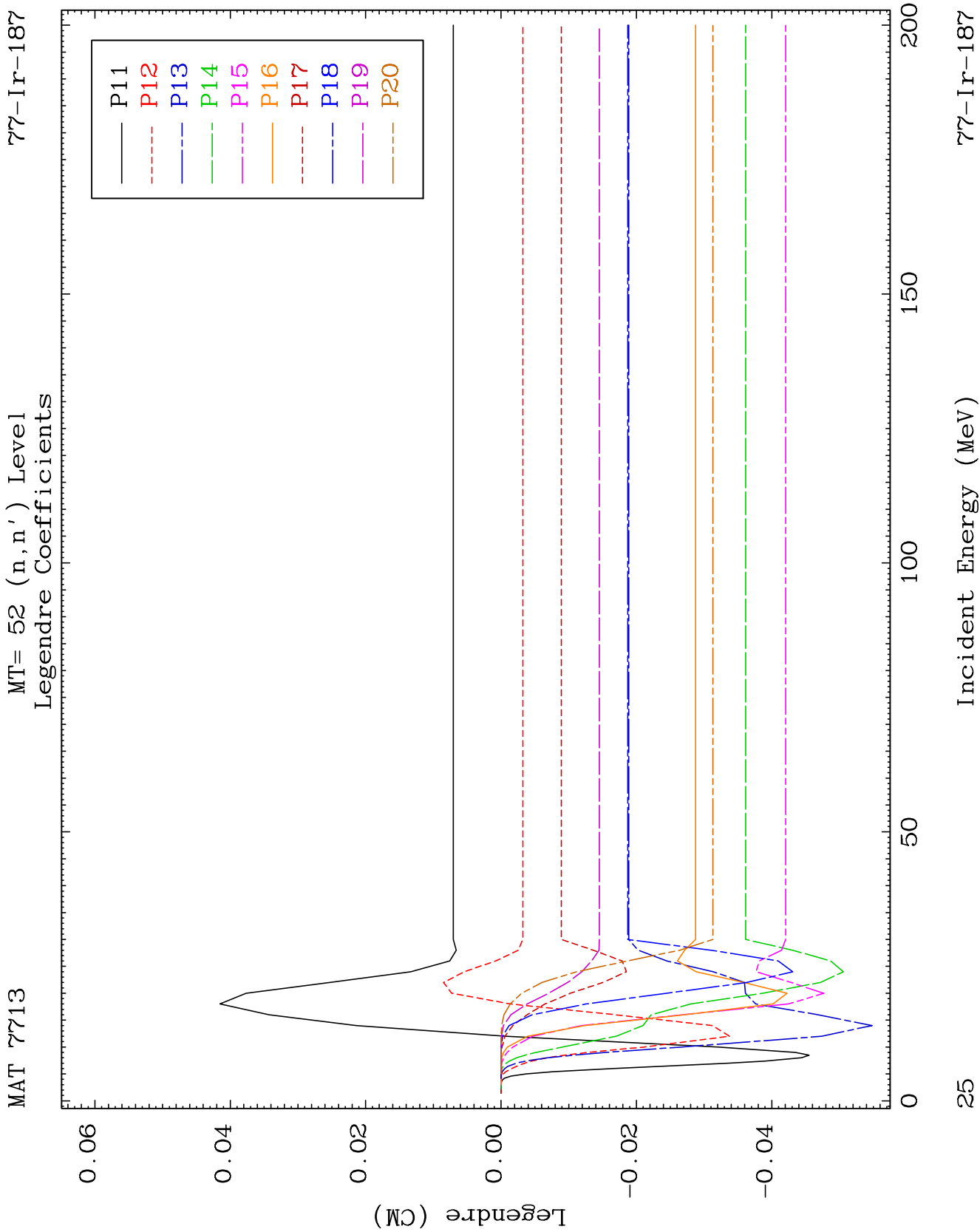
MAT 7713

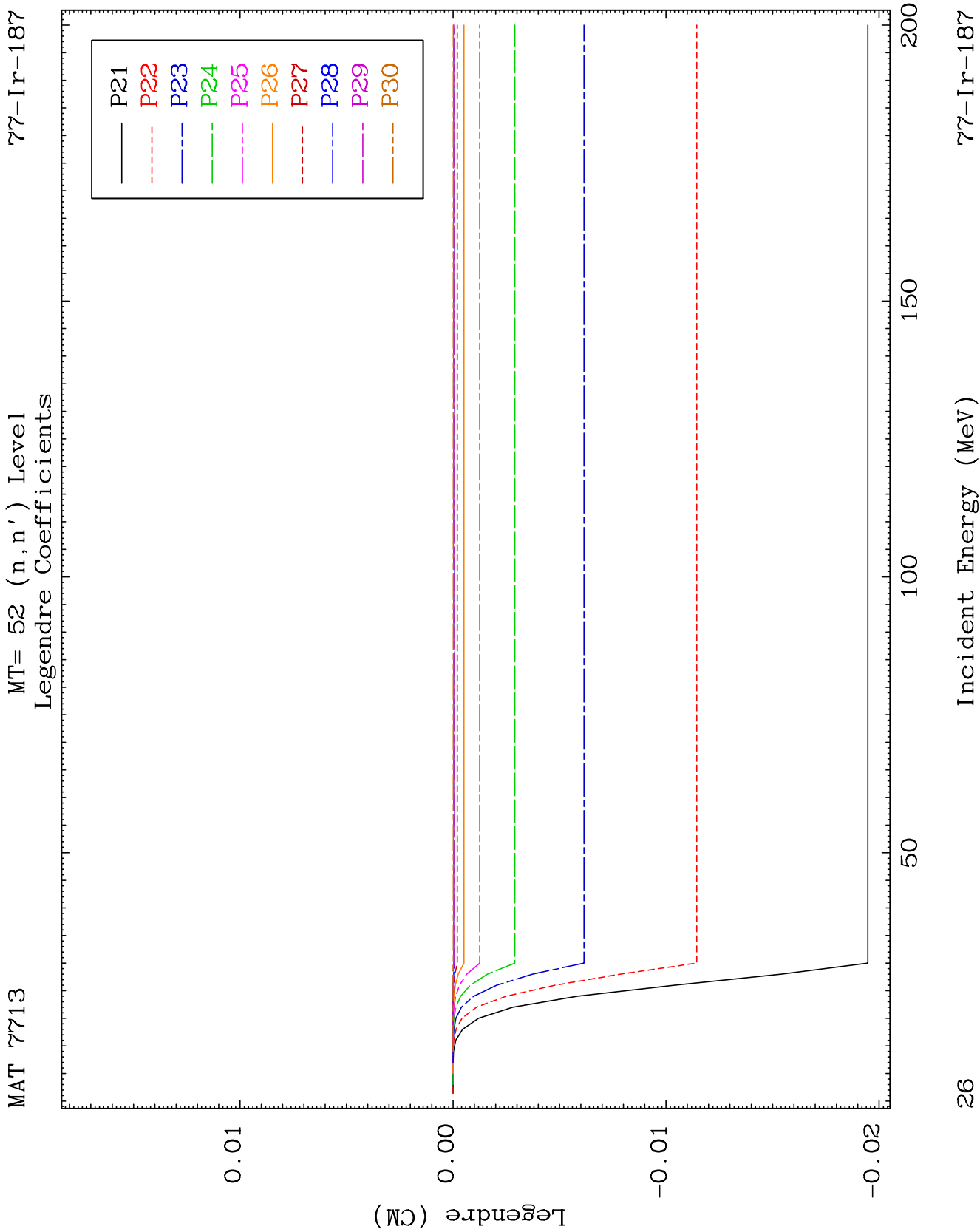
MT= 51 (n, n') Level
Legendre Coefficients

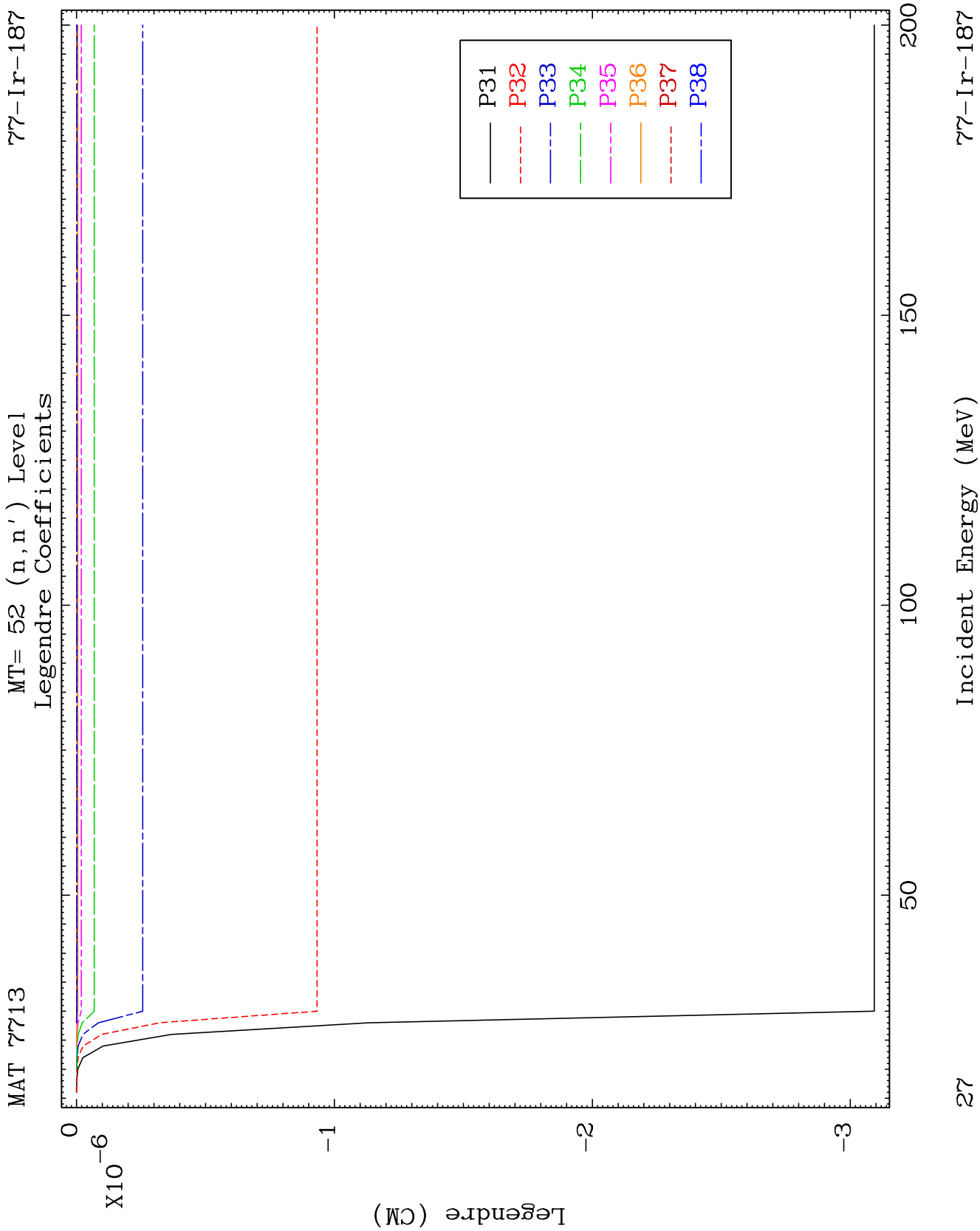
22-Ir-187

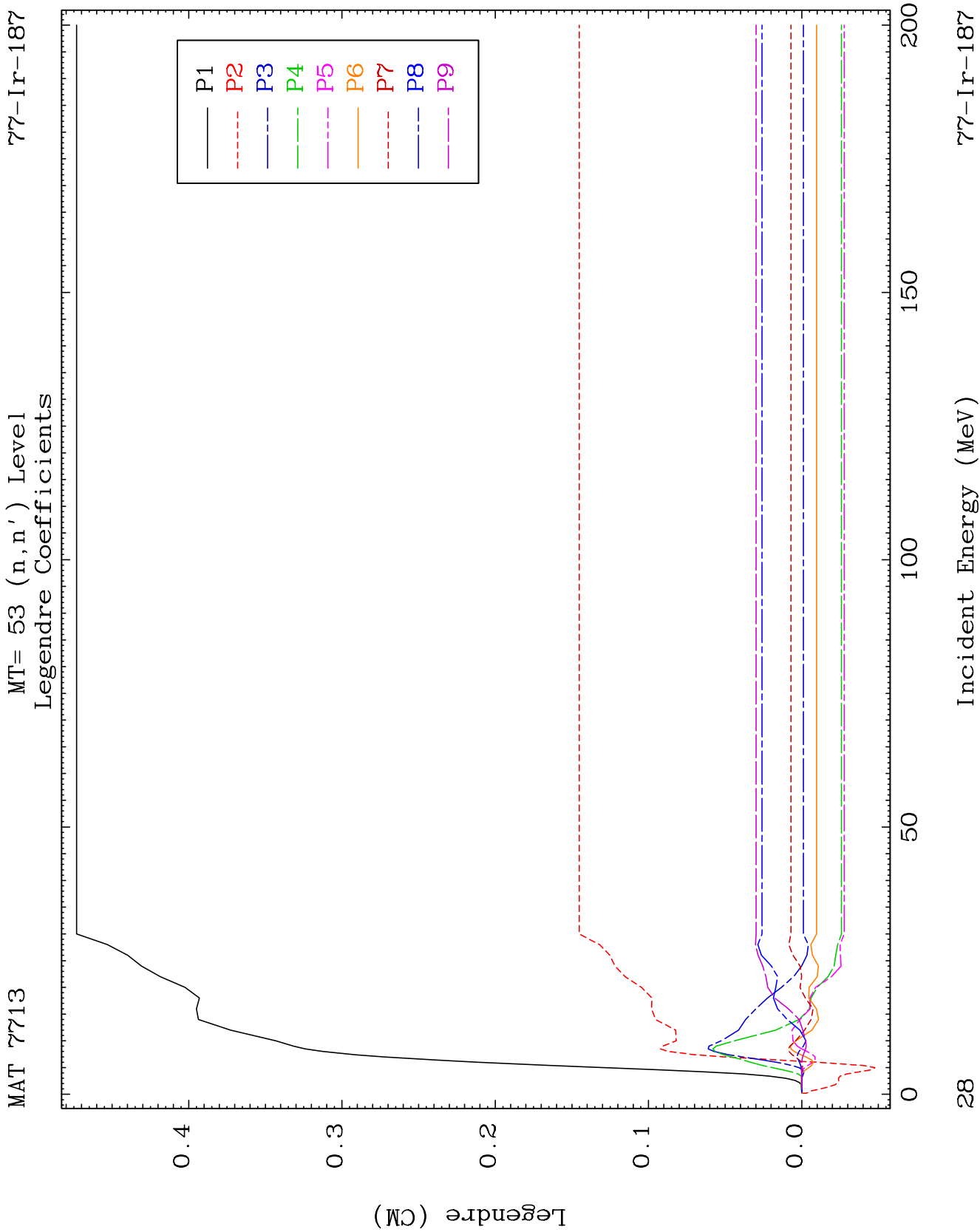


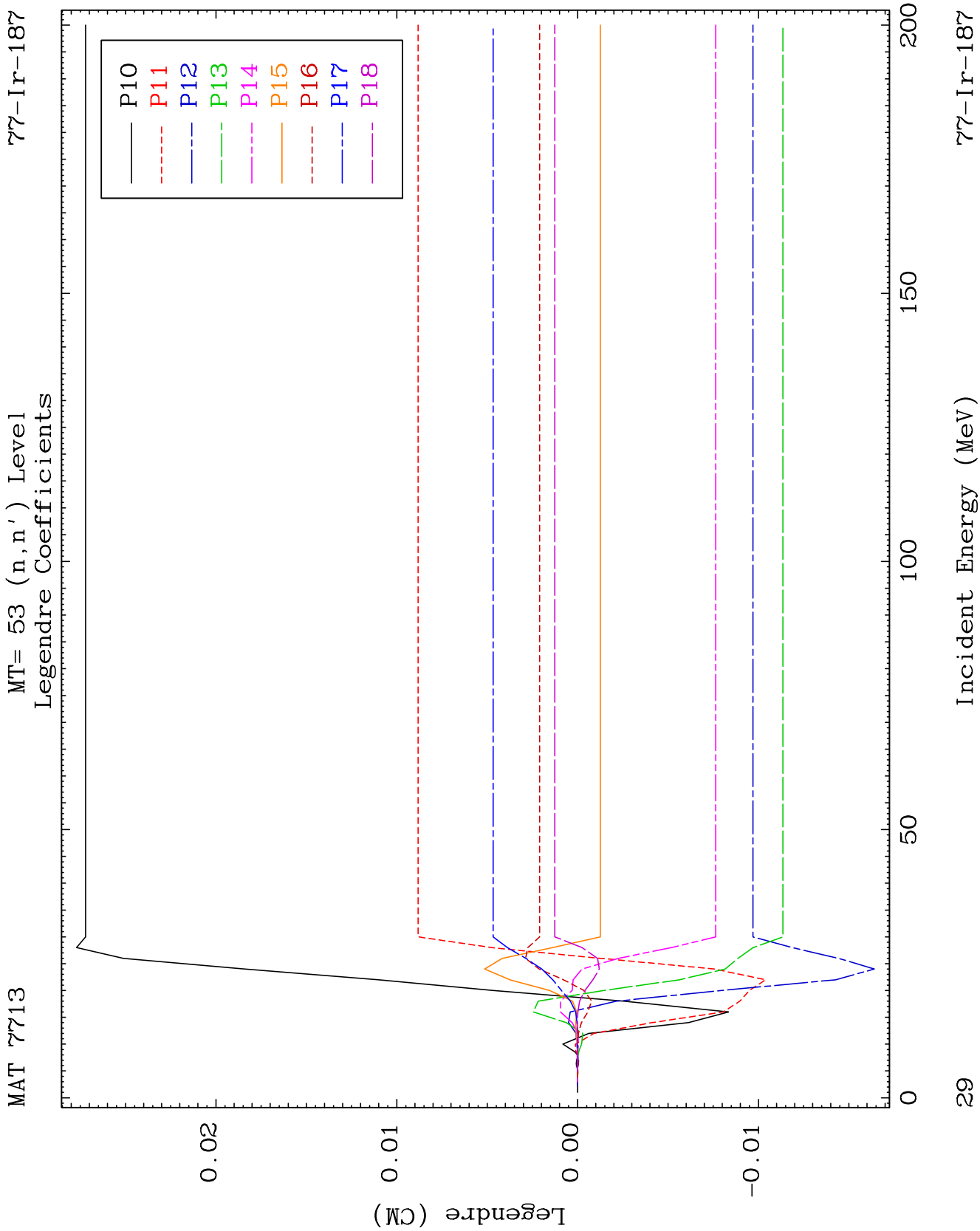








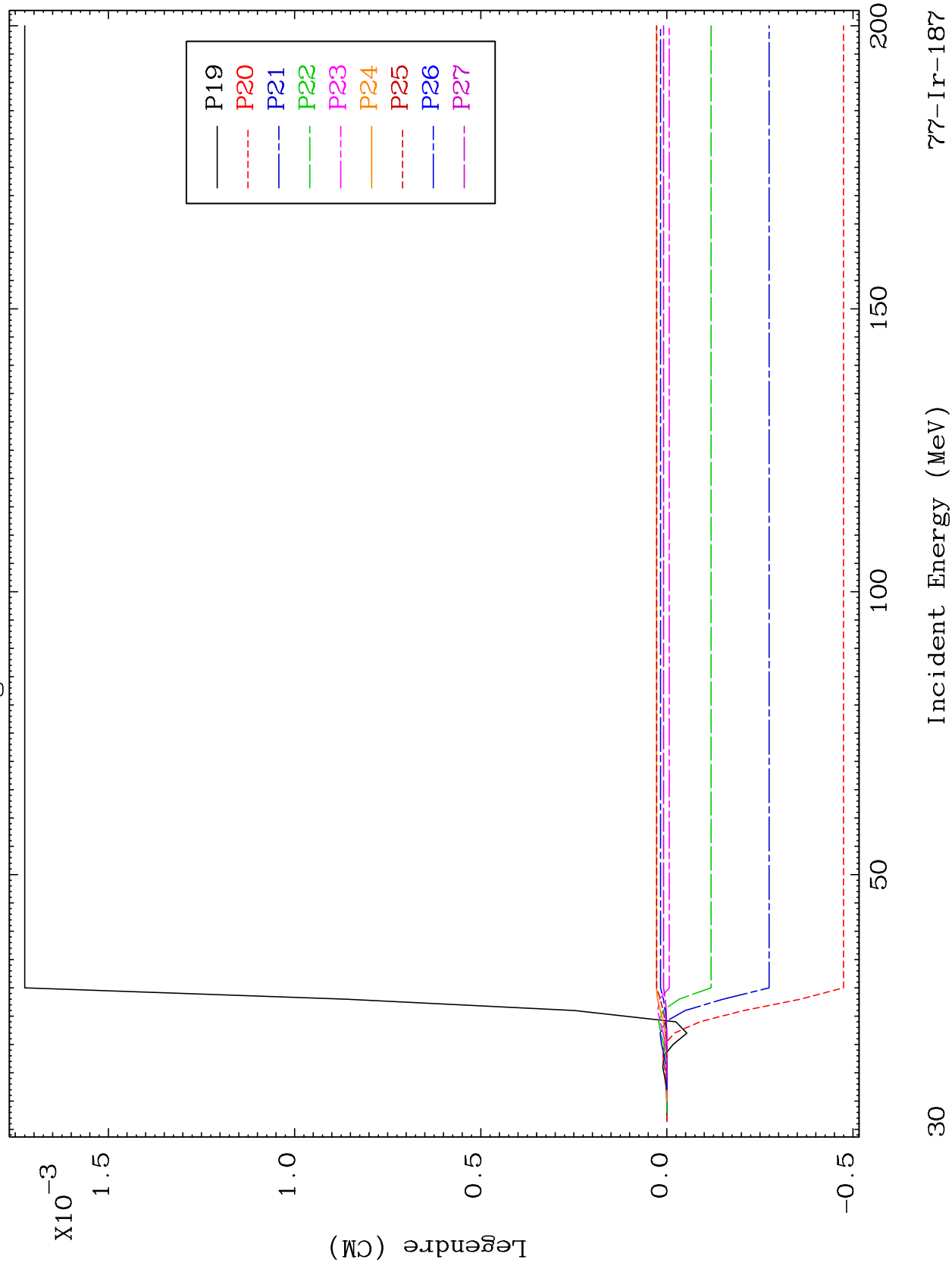


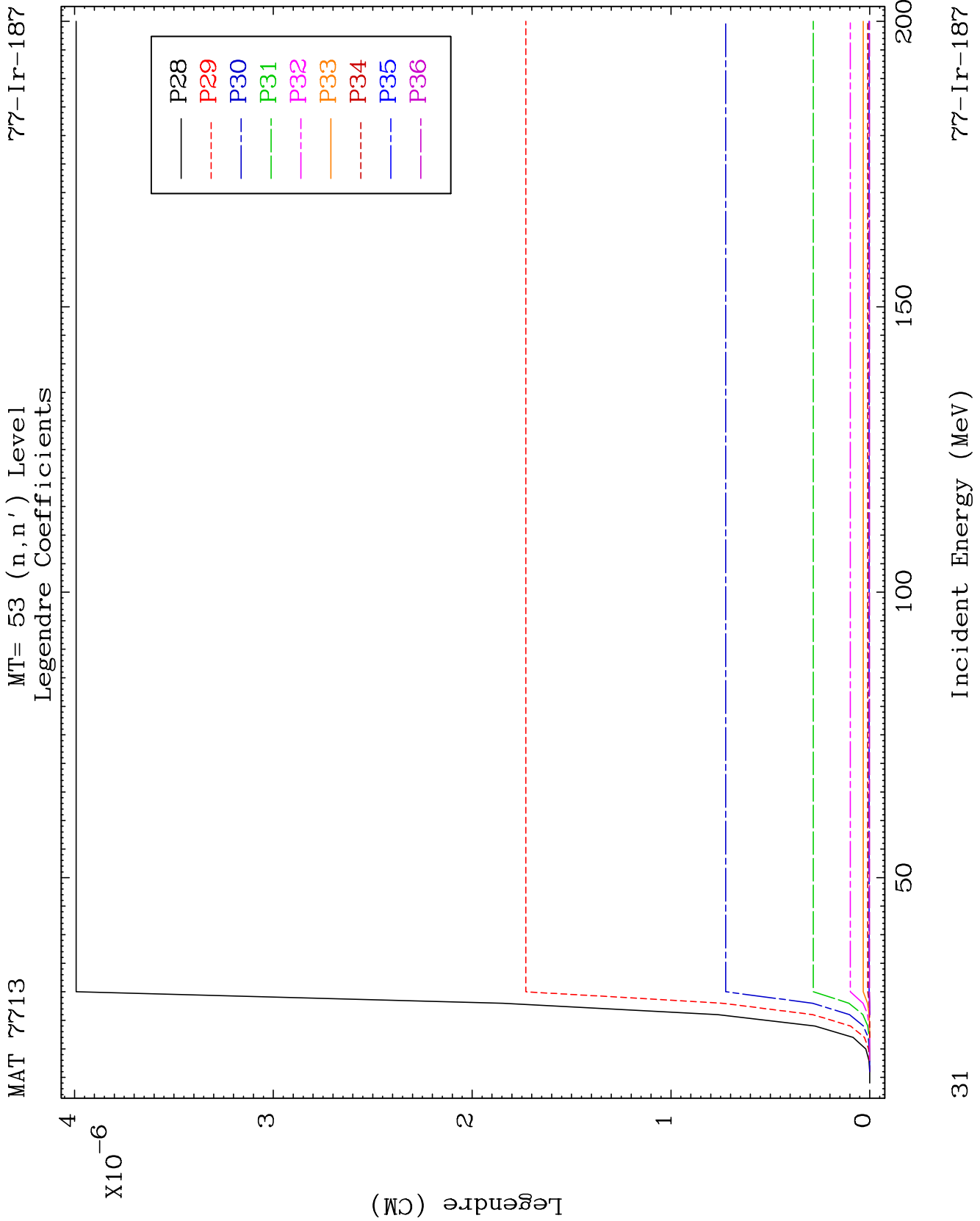


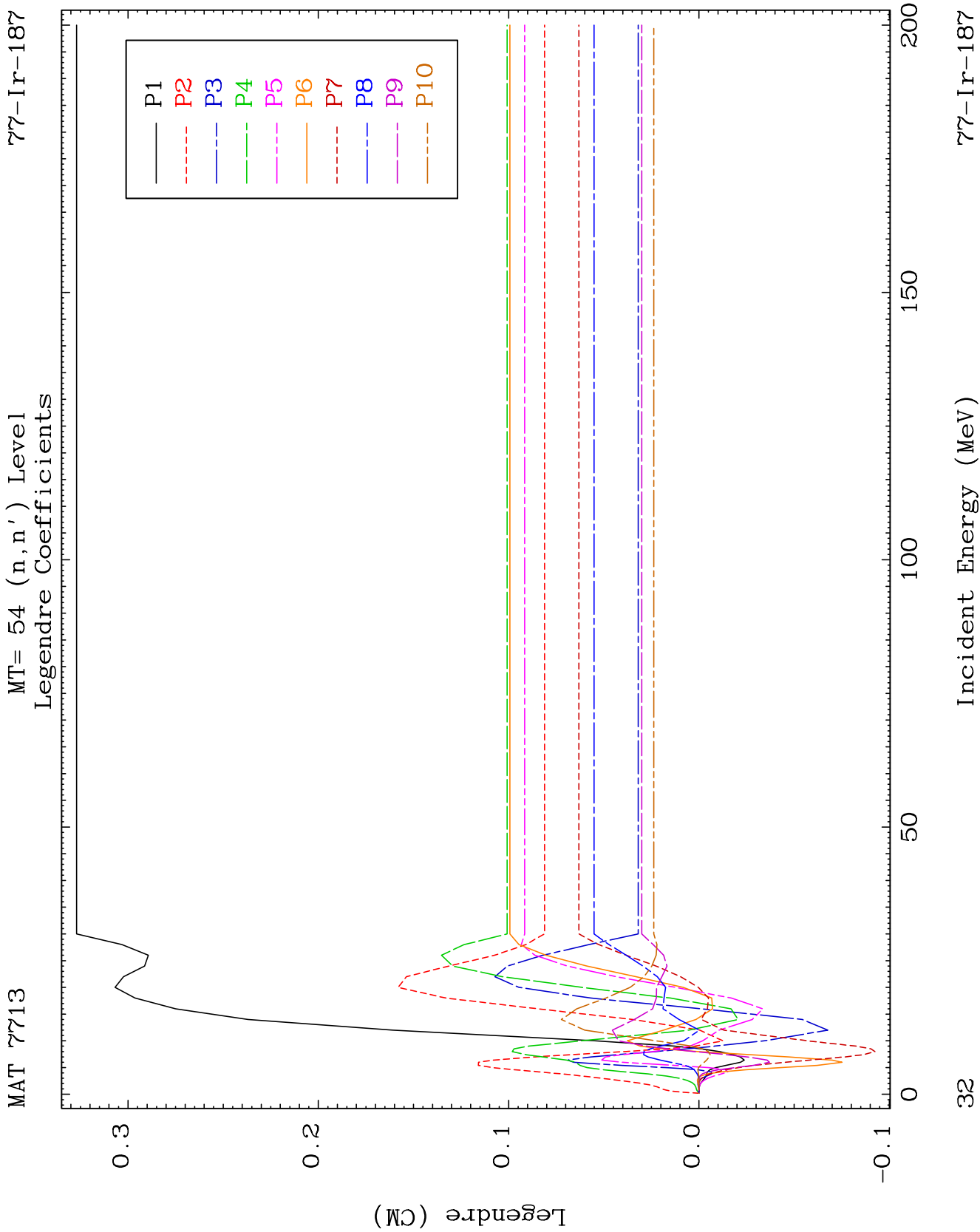
MAT 7713

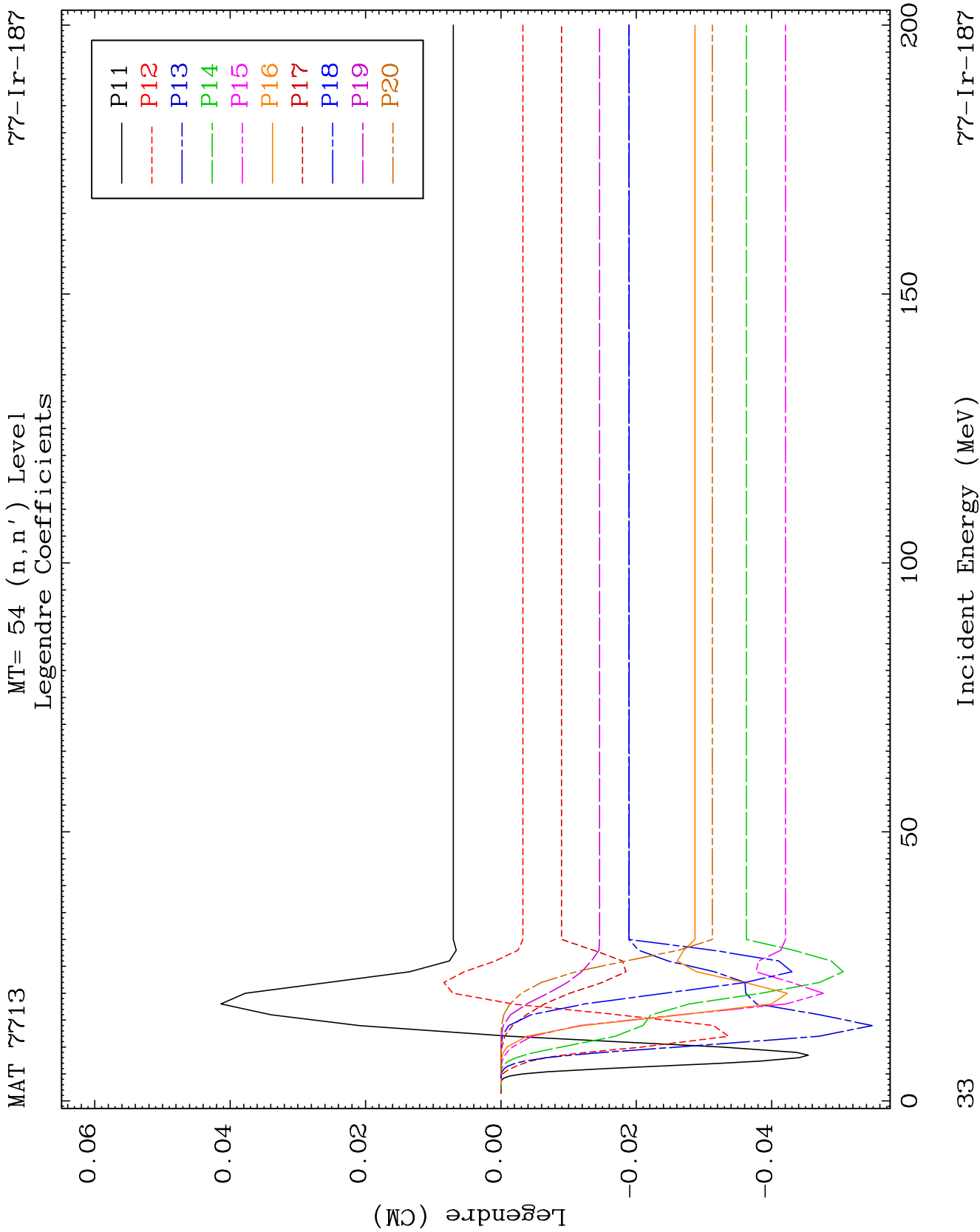
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Legendre Coefficients

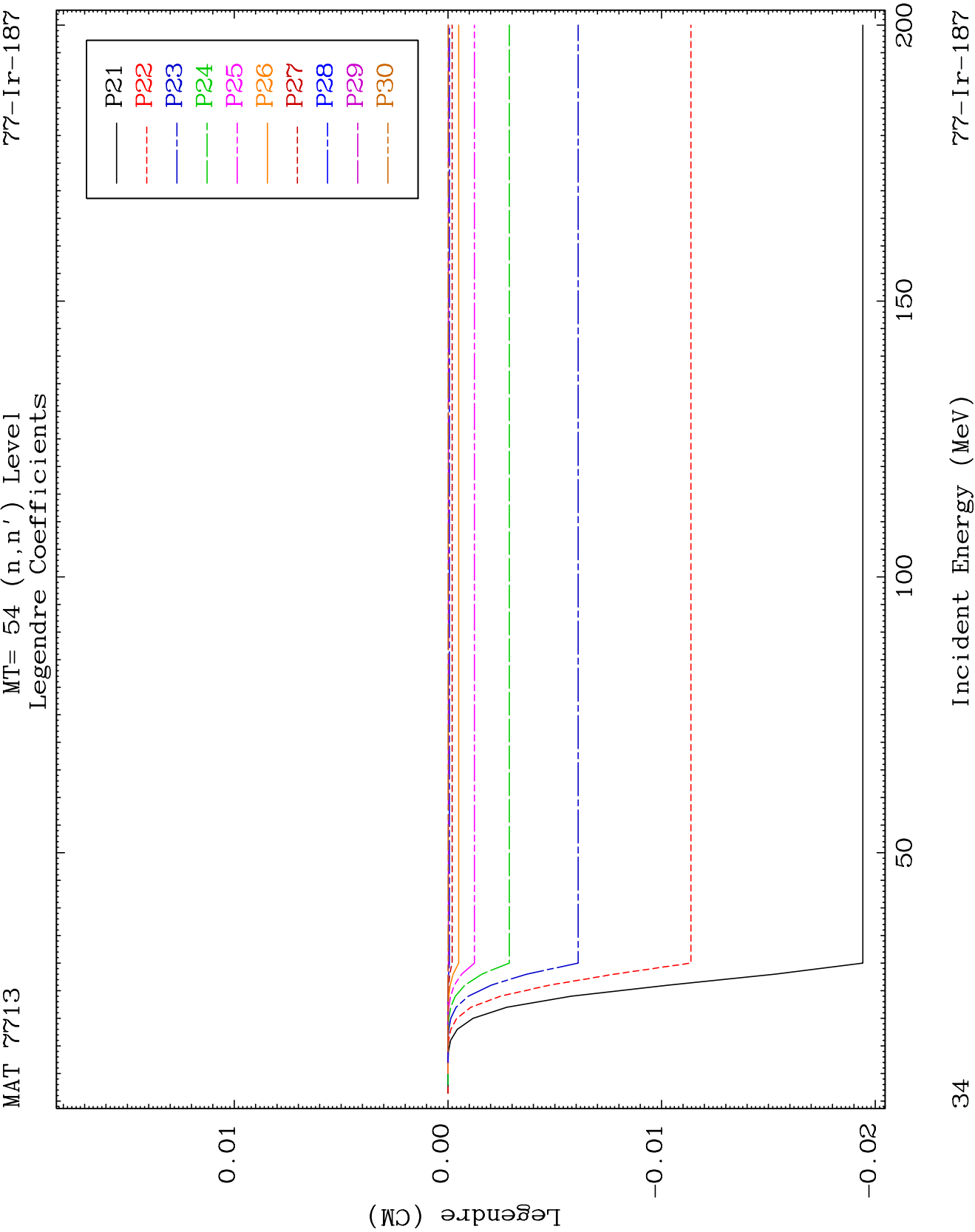
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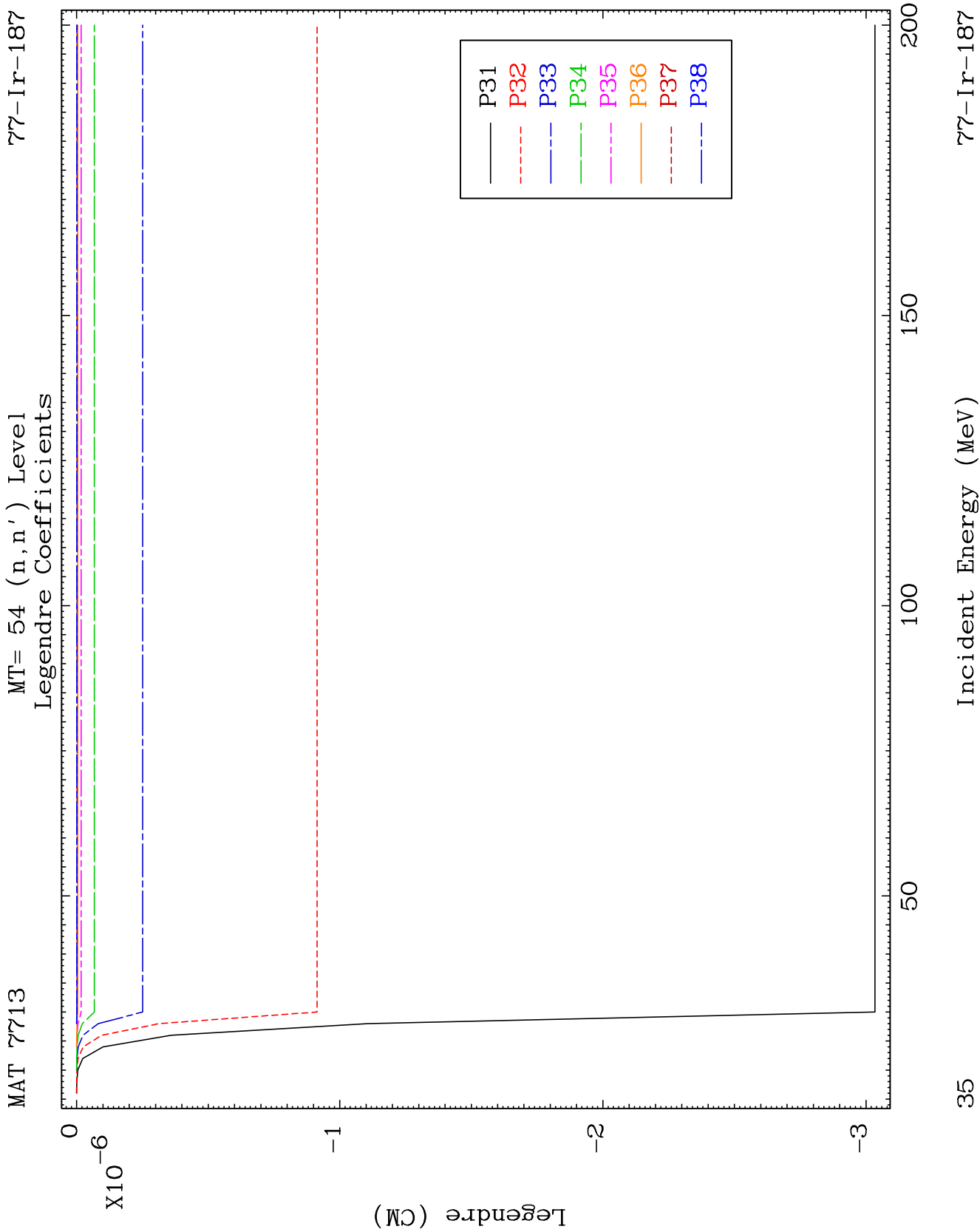


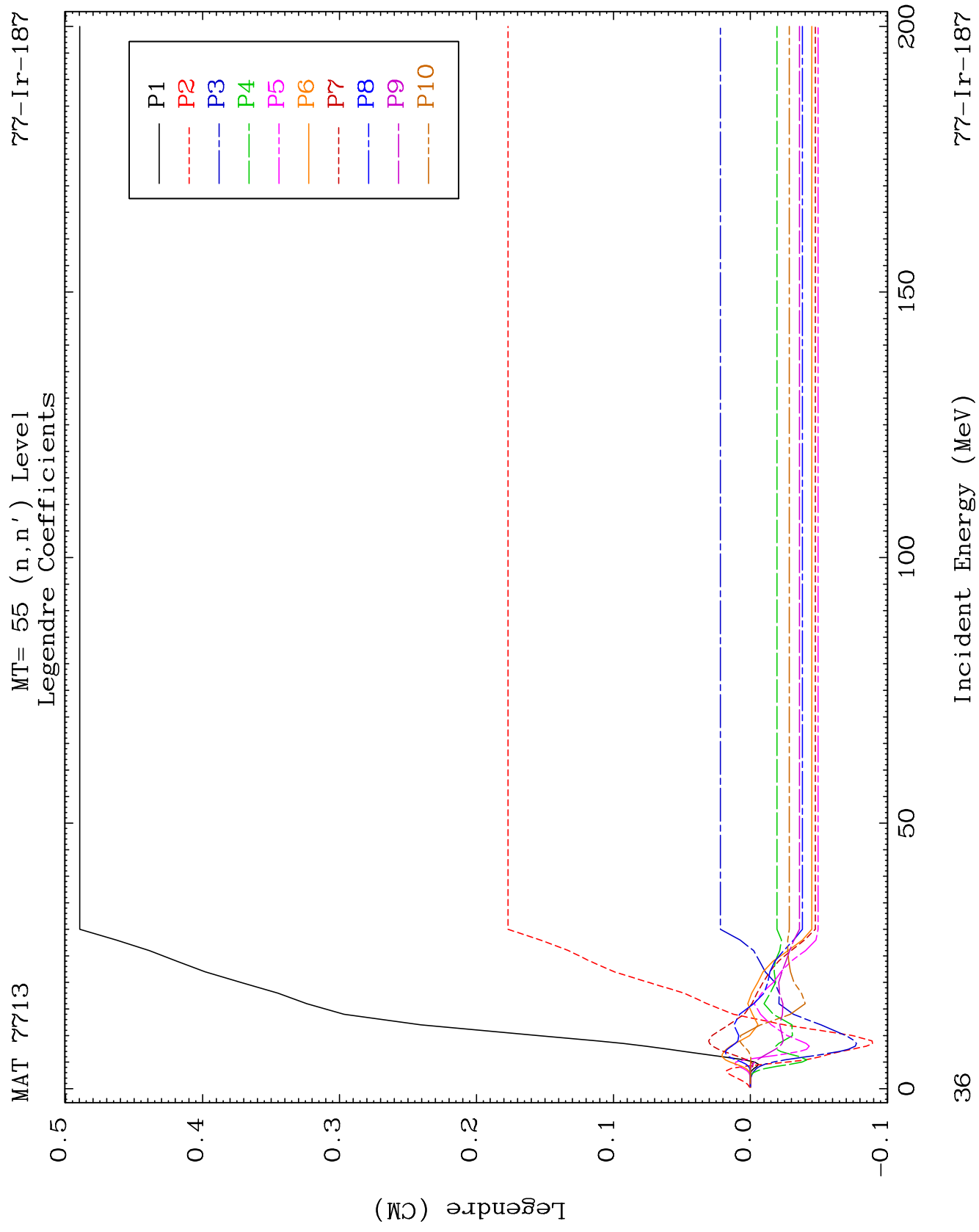


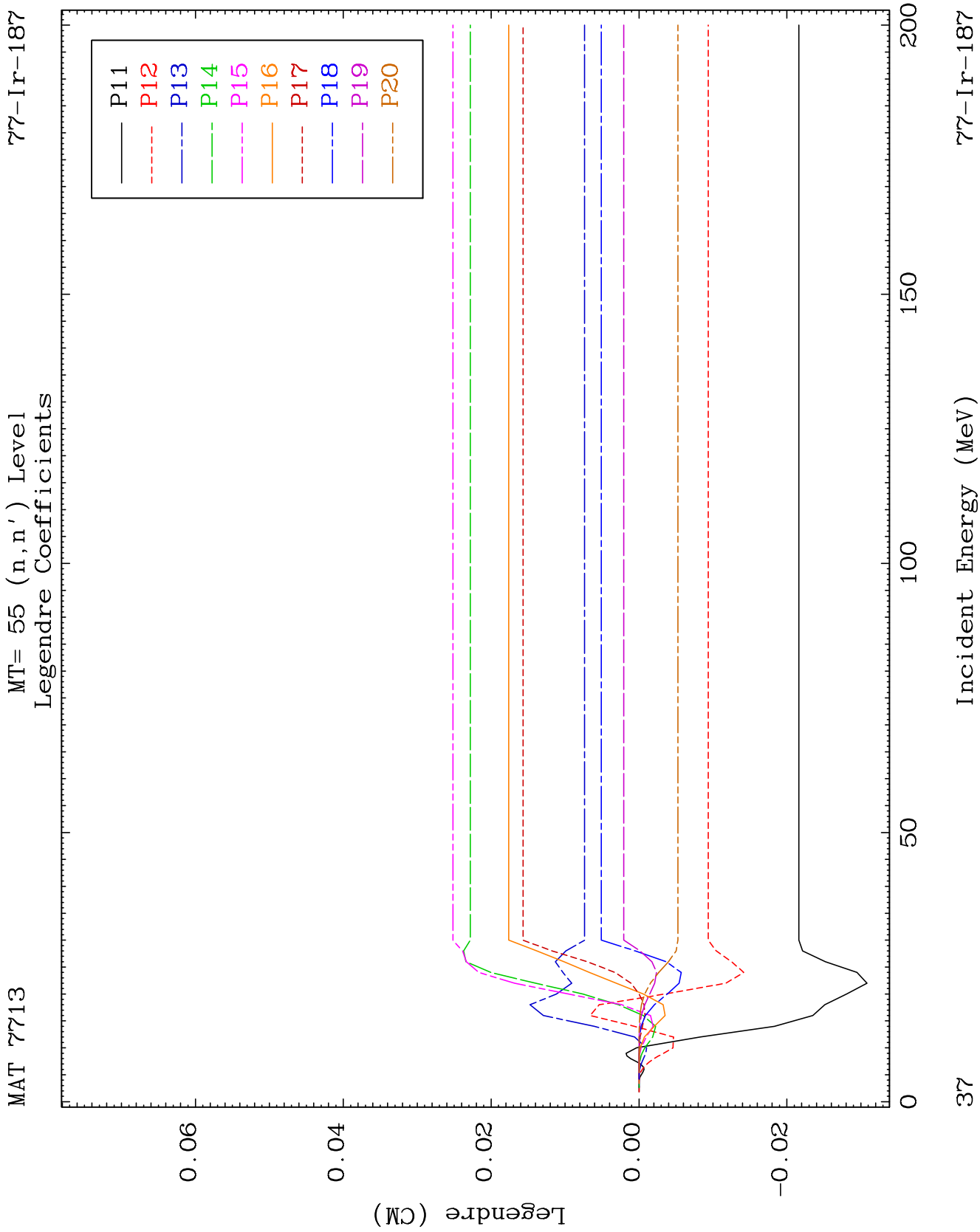


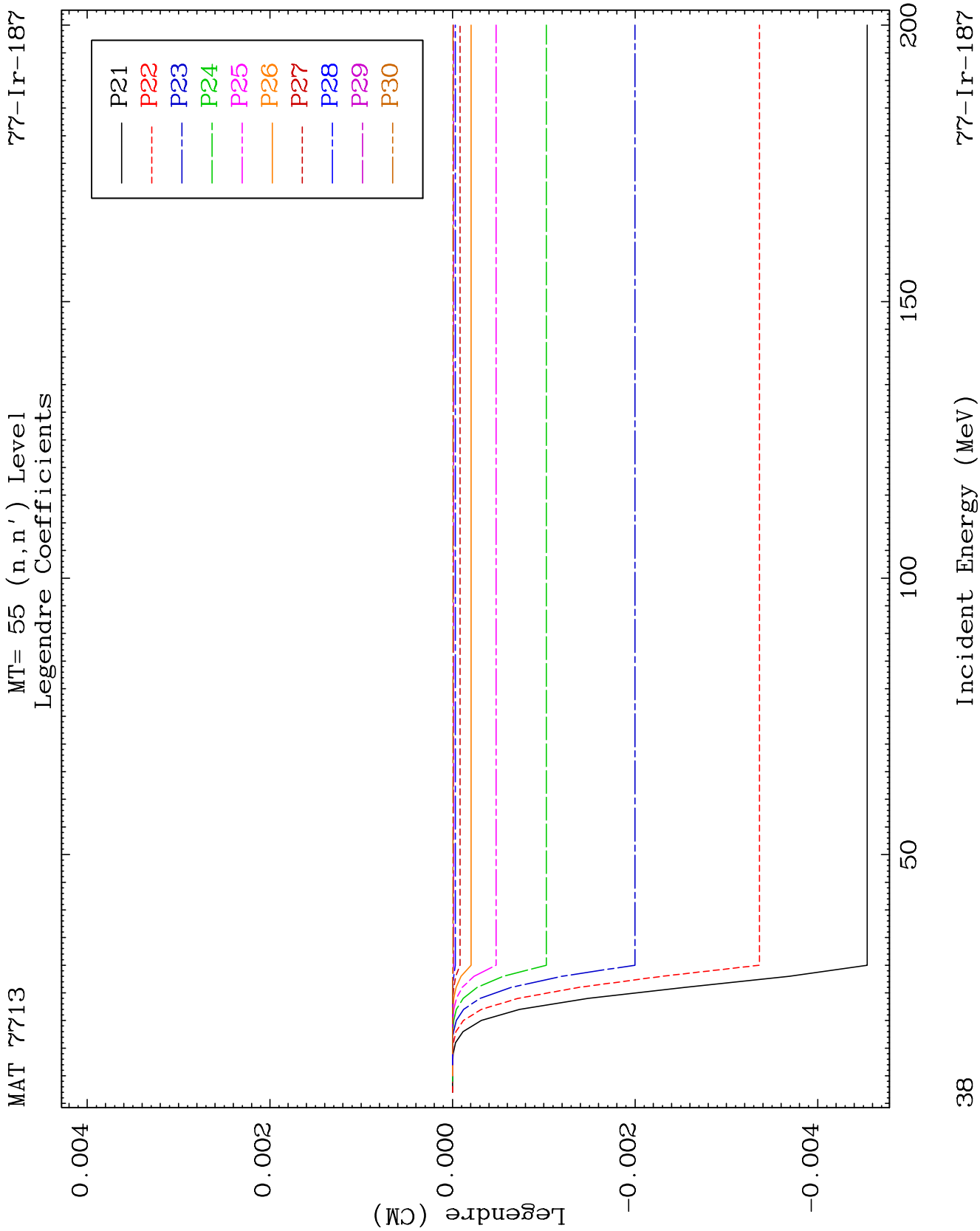


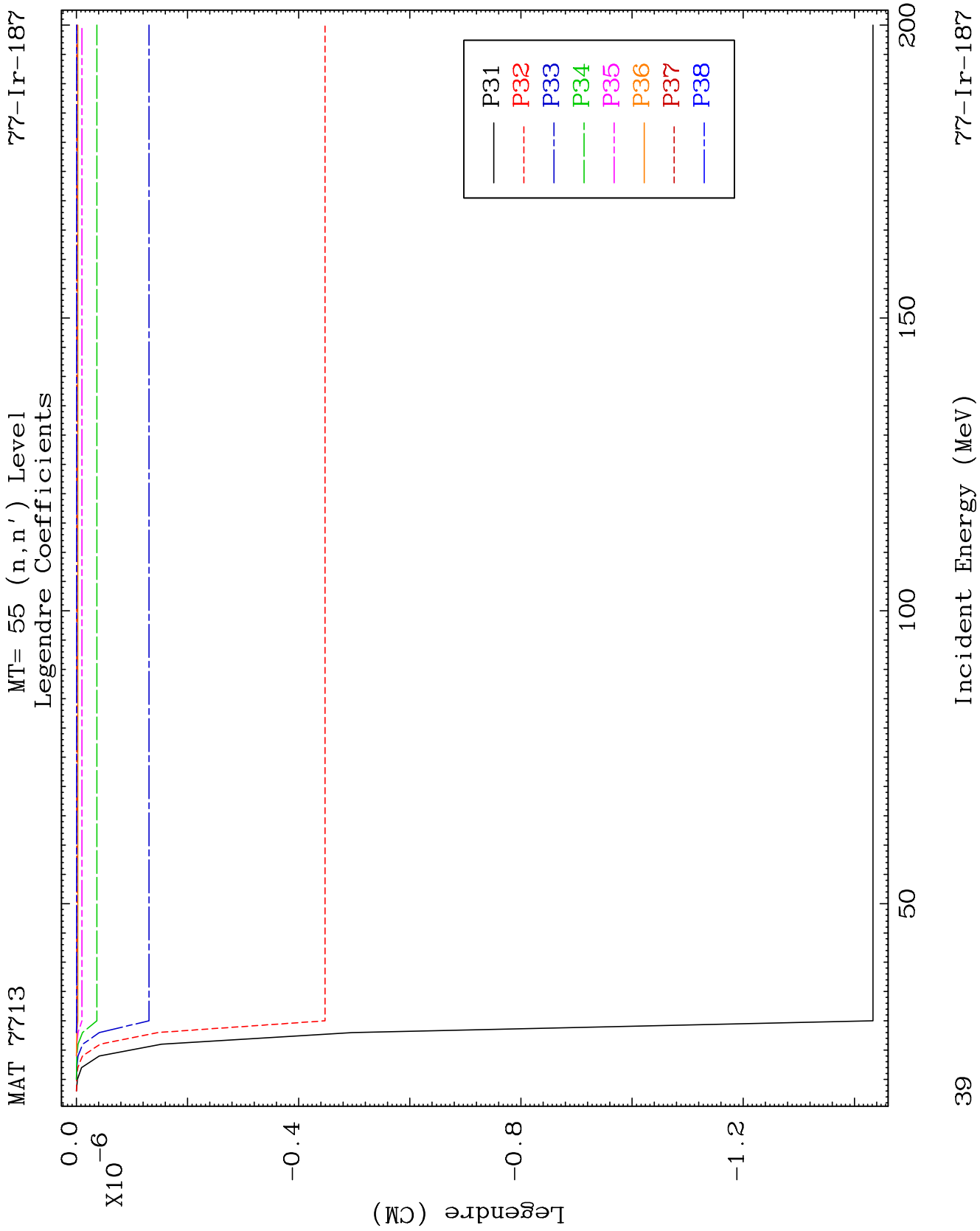


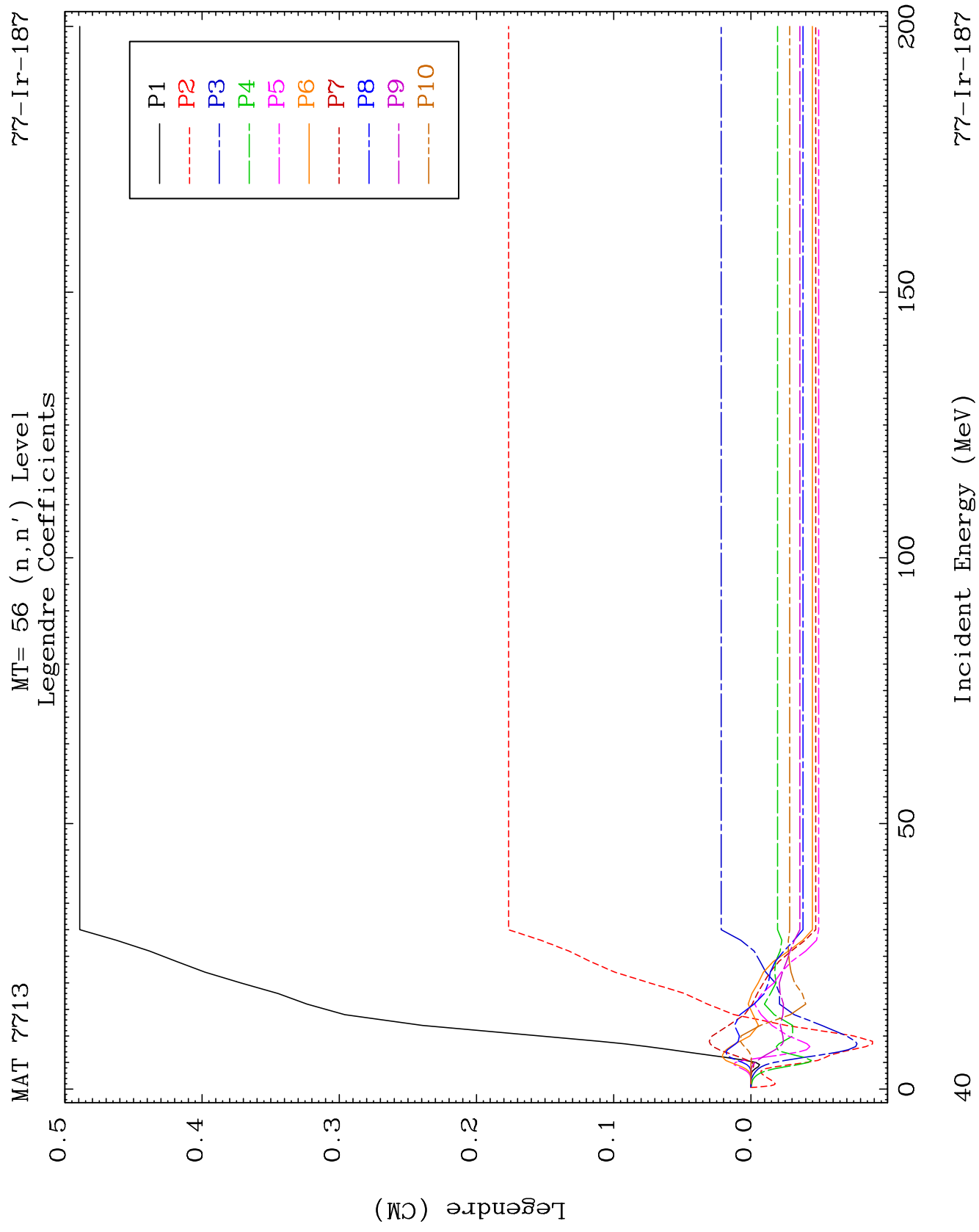


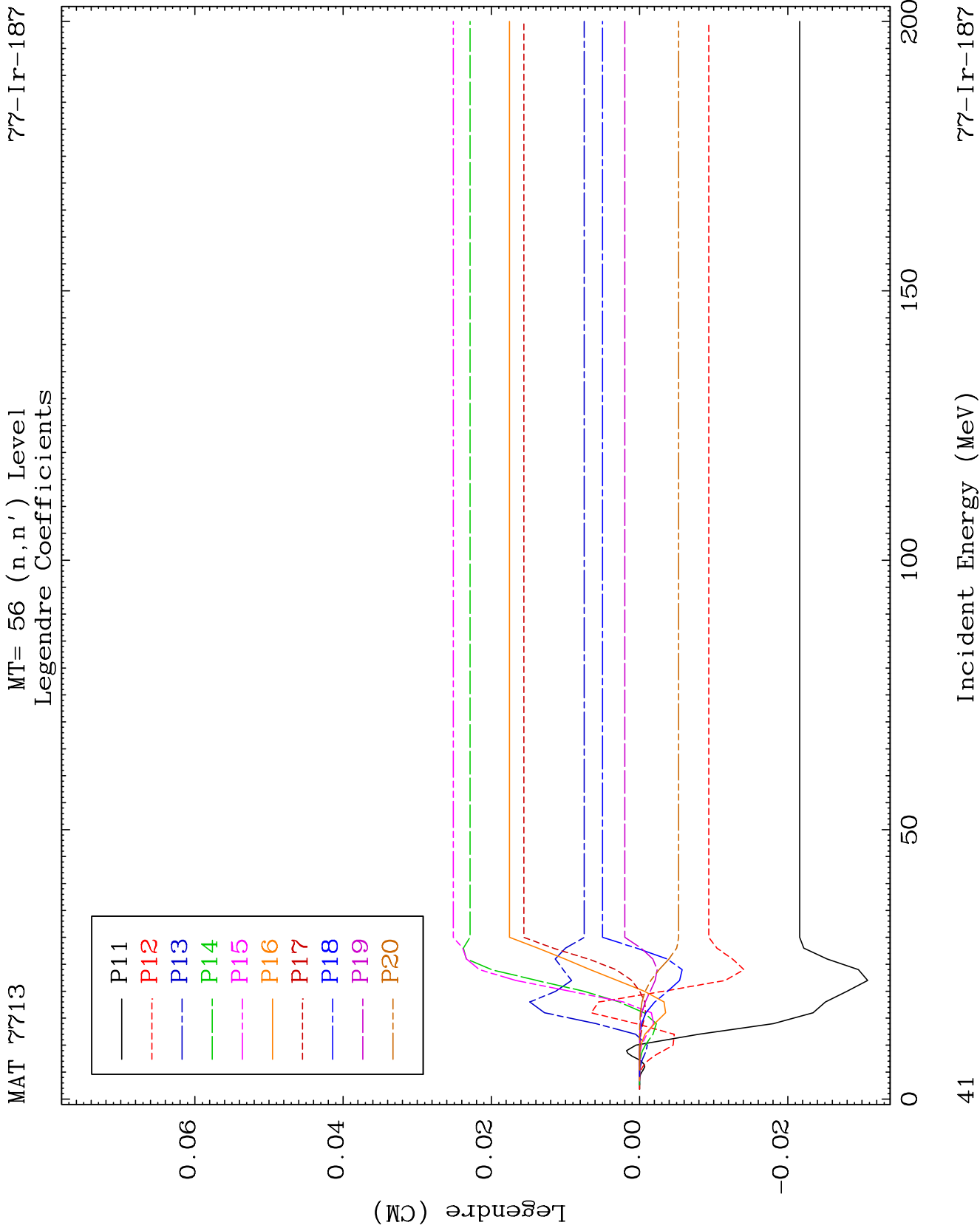








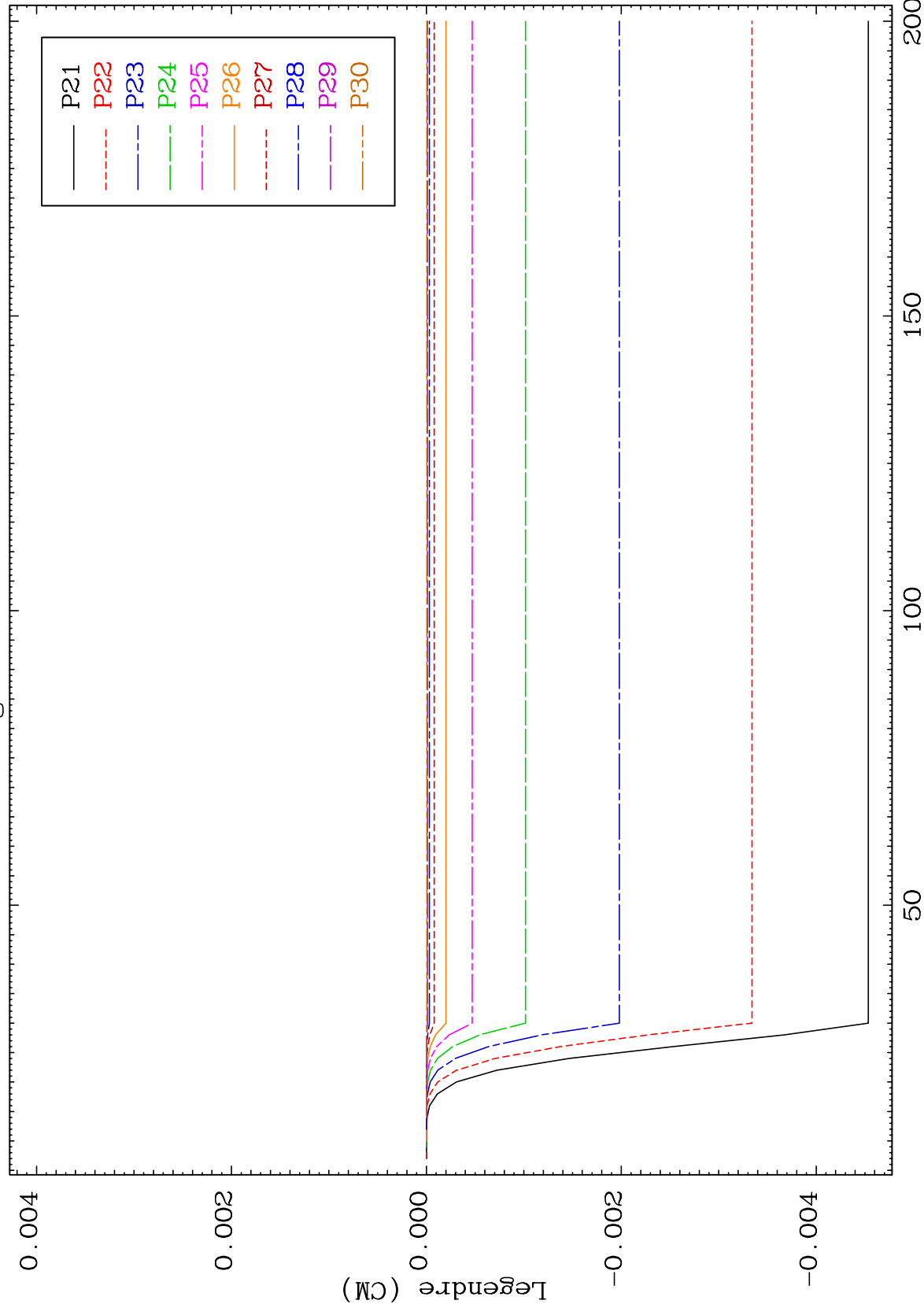




MAT 7713

MT= 56 (n, n') Level
Legendre Coefficients

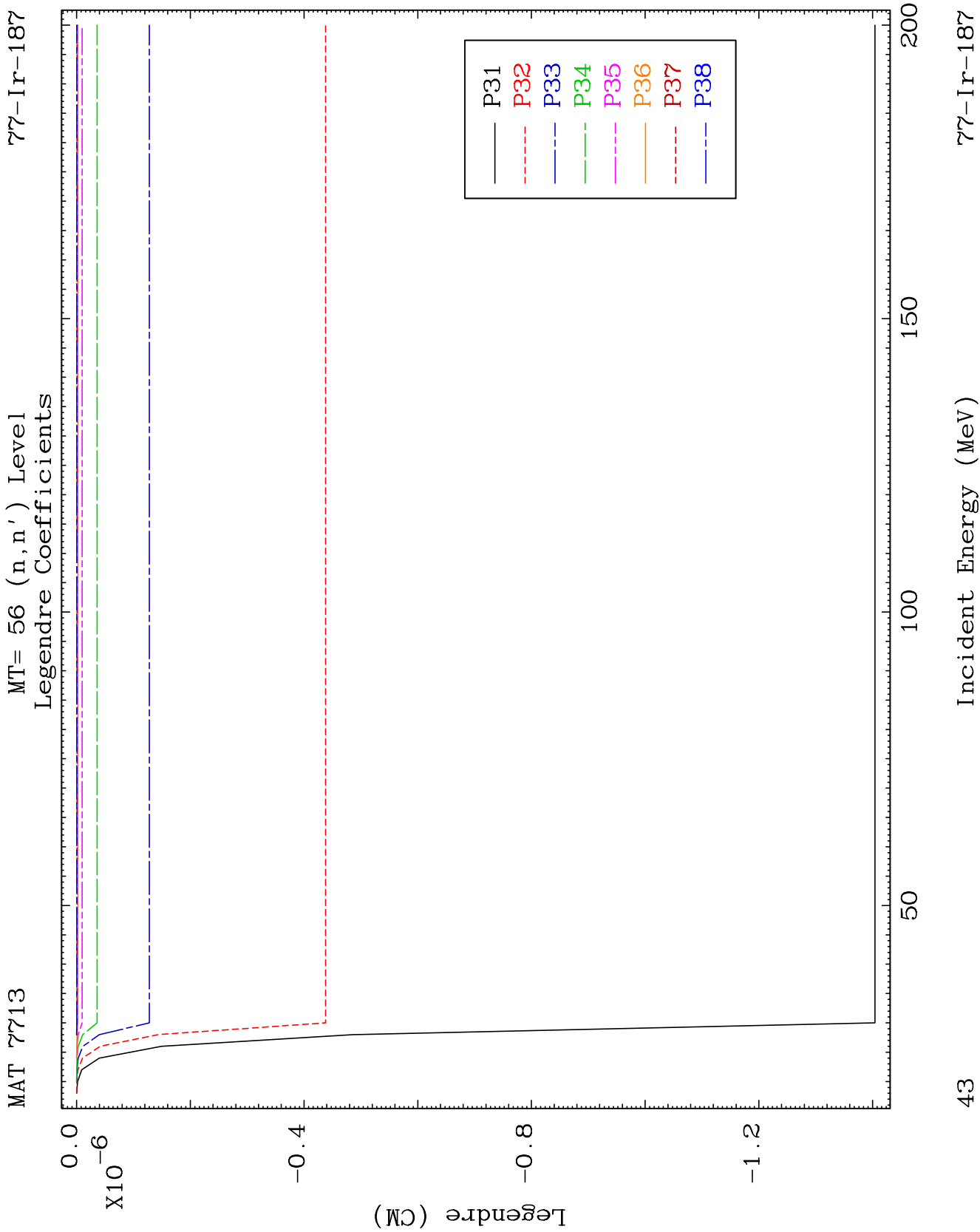
22-1r-187

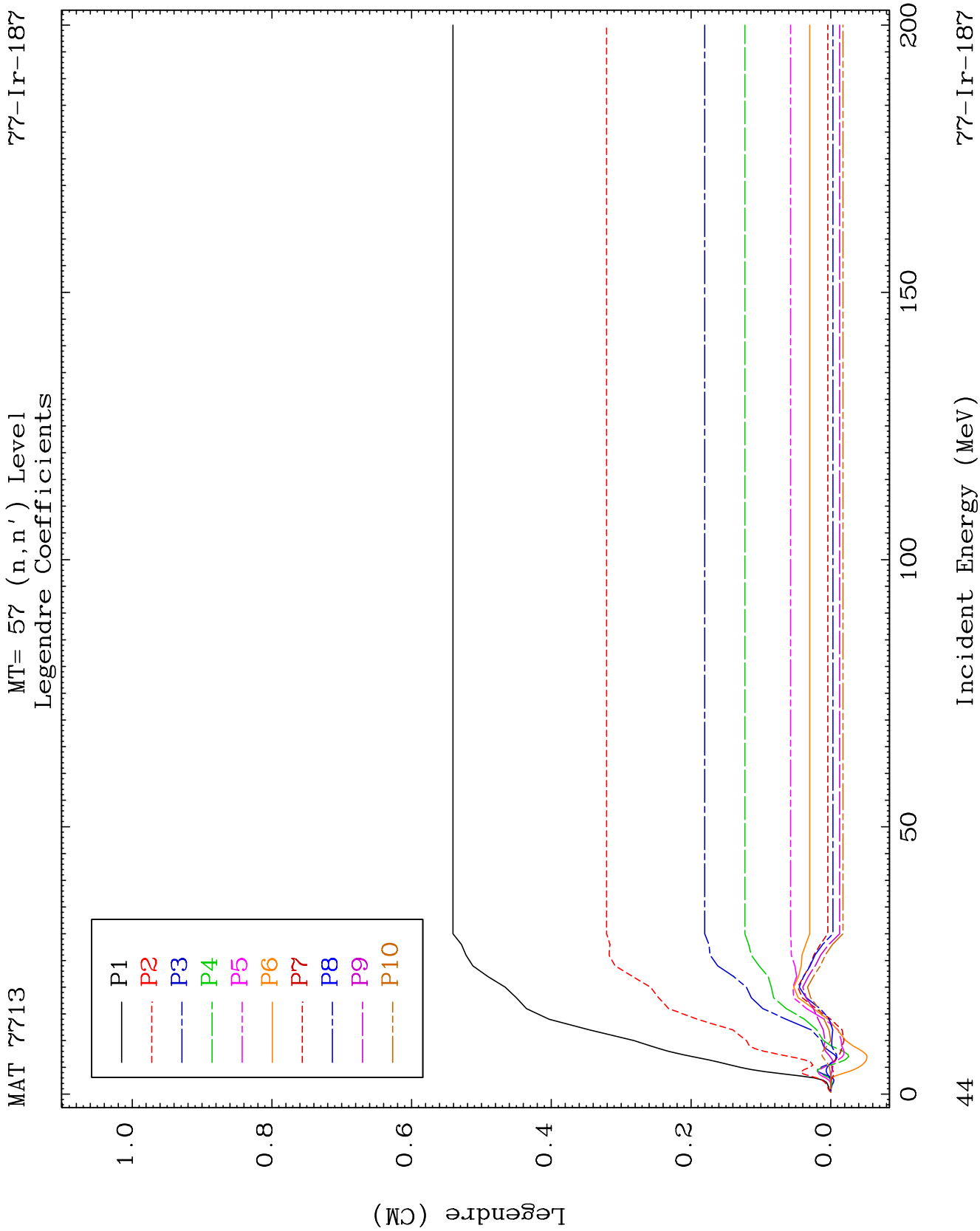


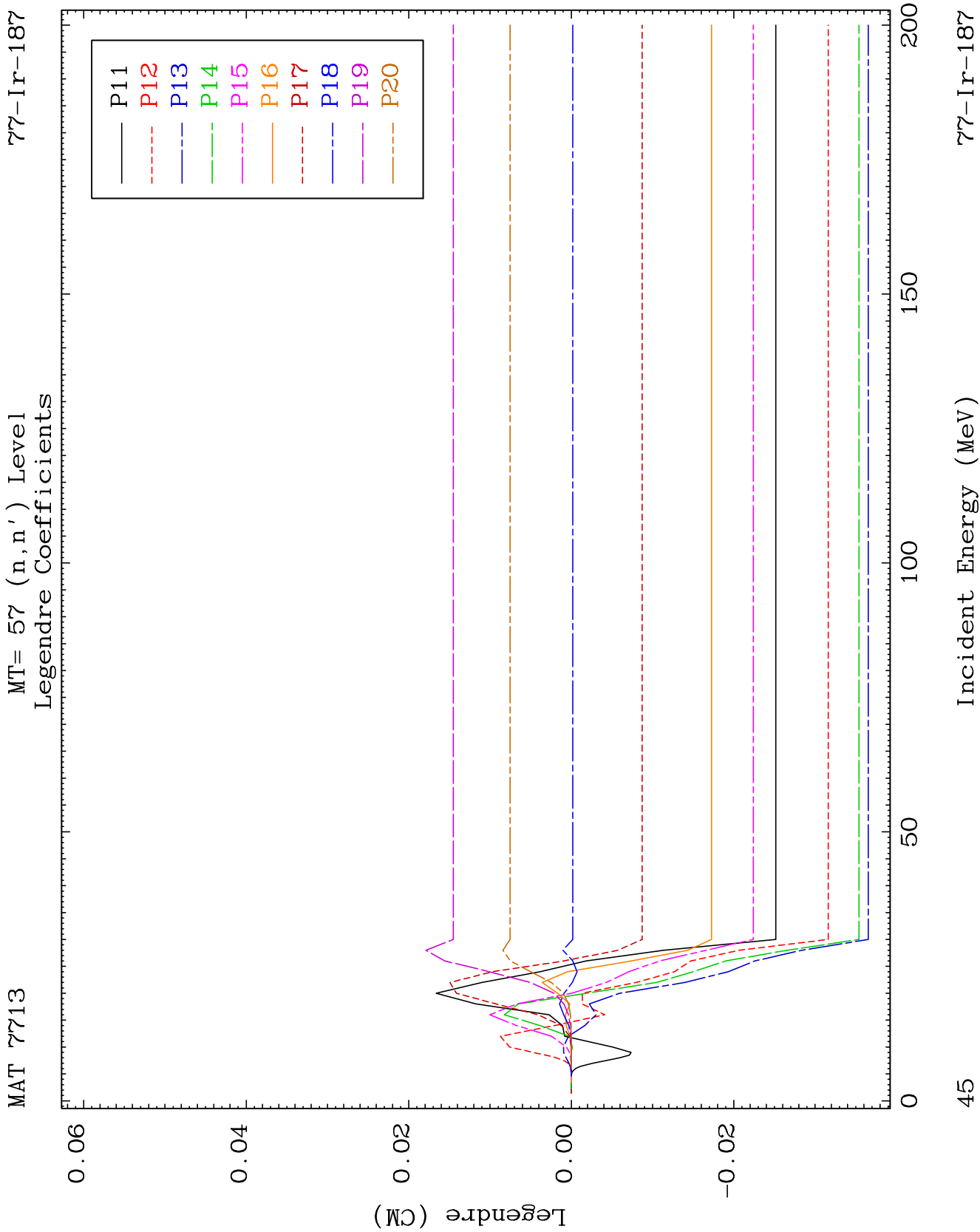
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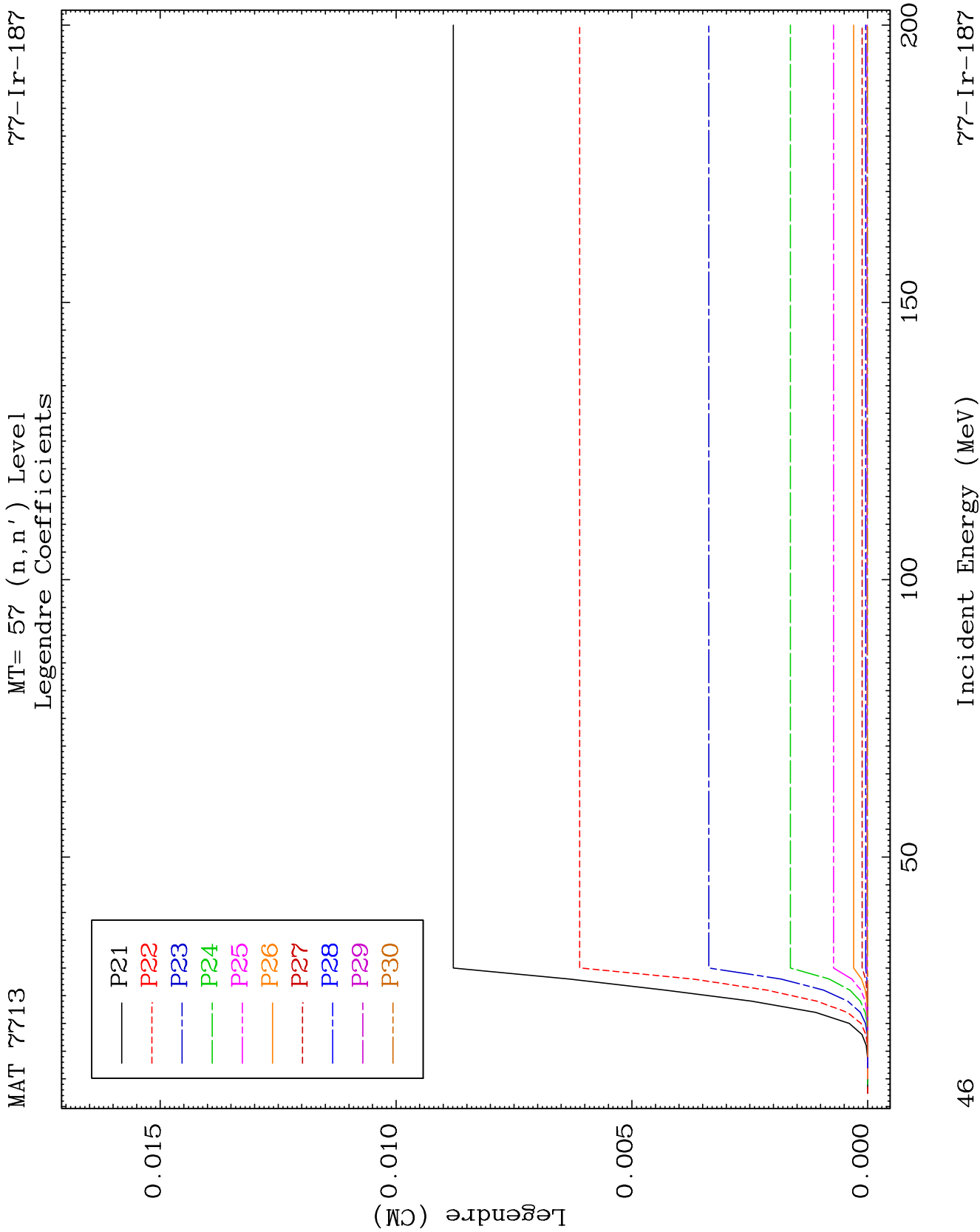
Incident Energy (MeV)

22-Ir-187





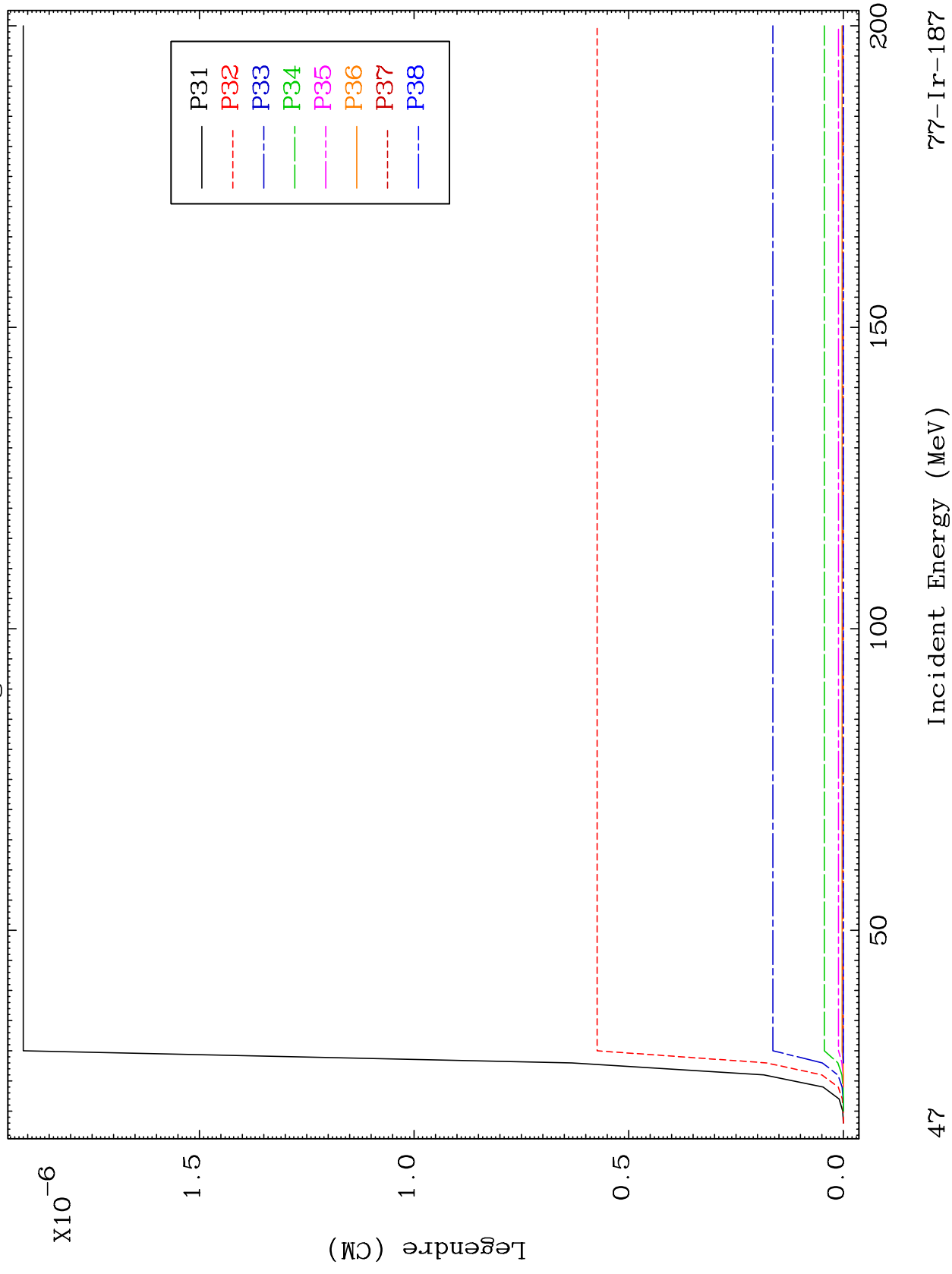


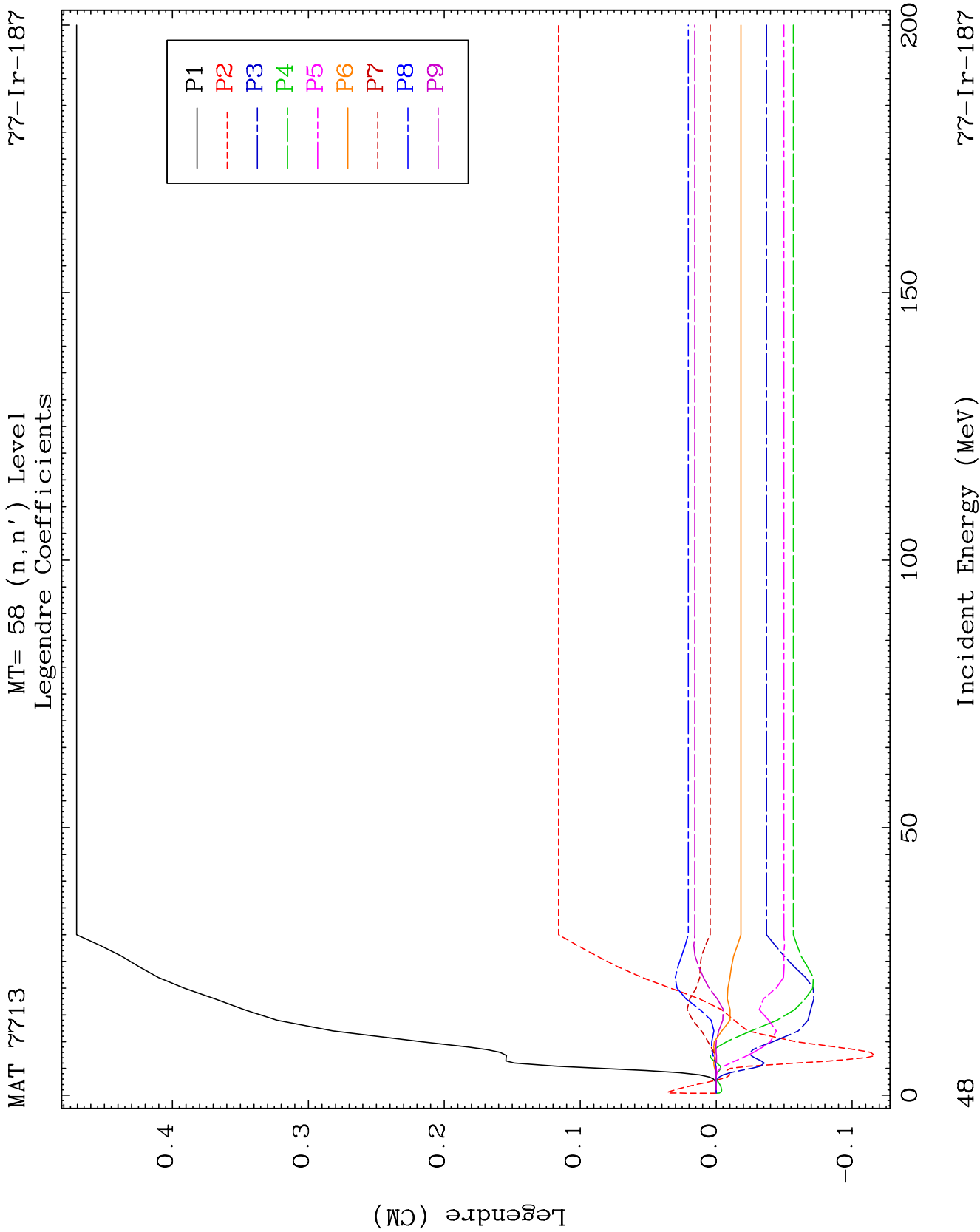


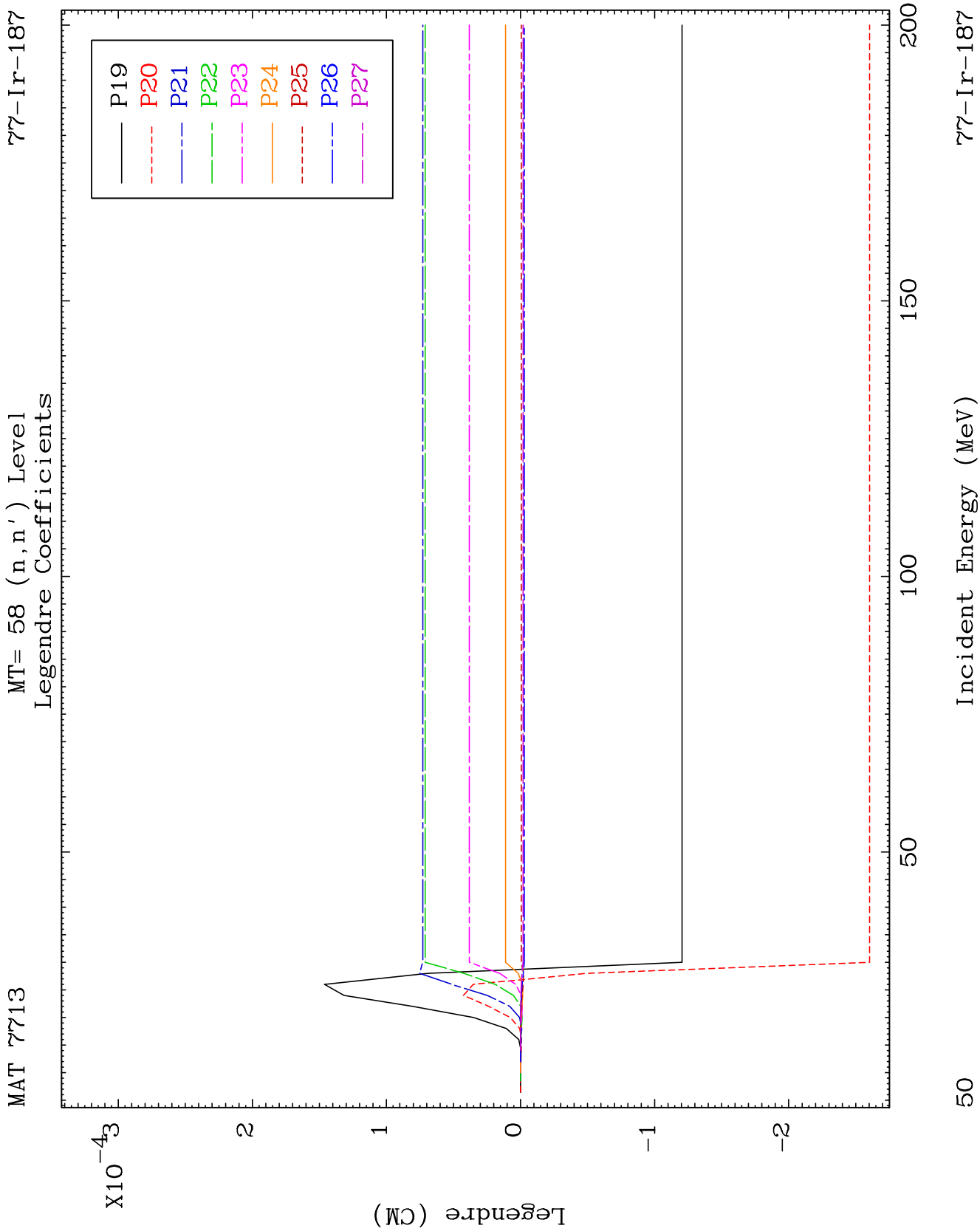
MAT 7713

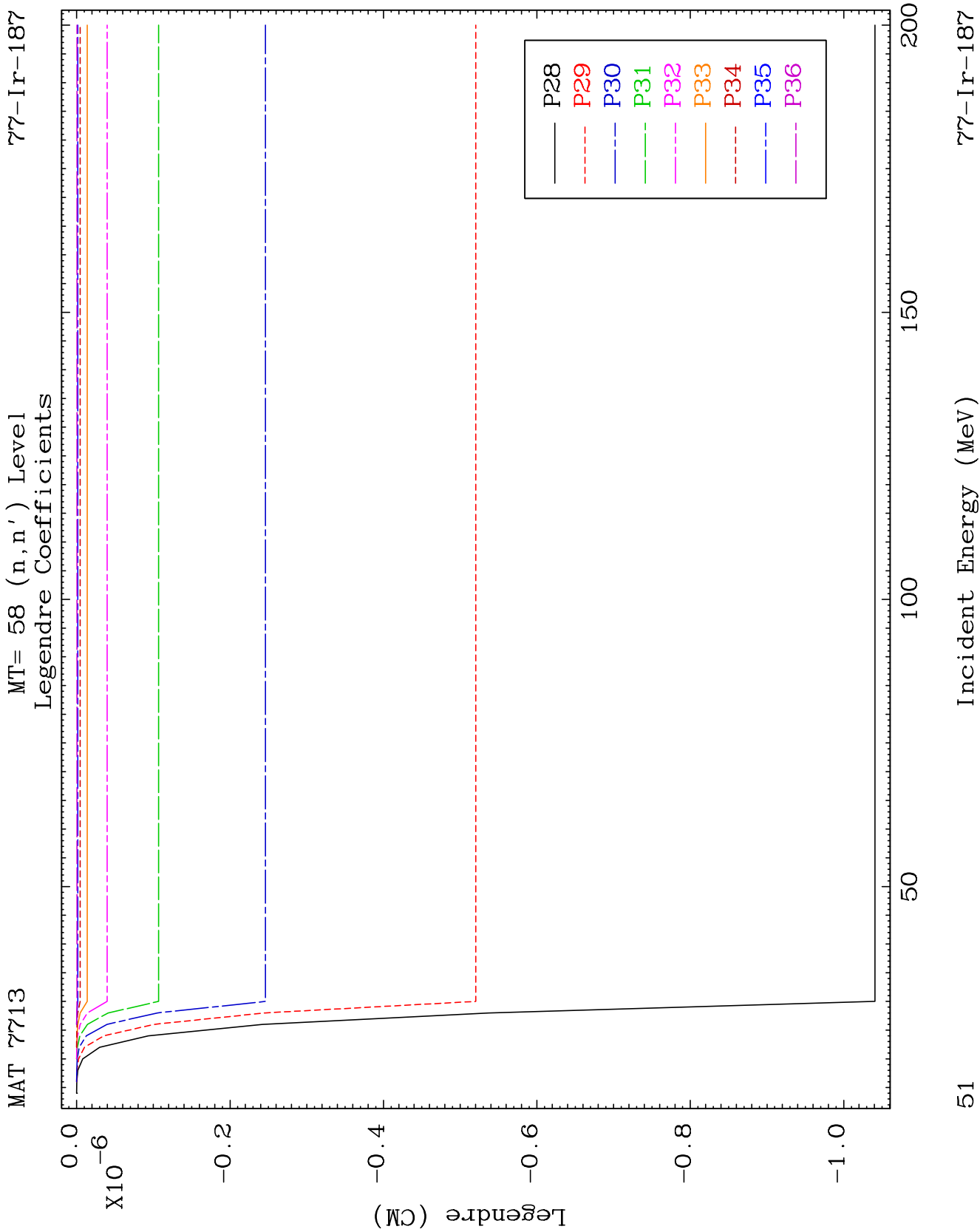
MT= 57 (n, n') Level
Legendre Coefficients

22-1r-187





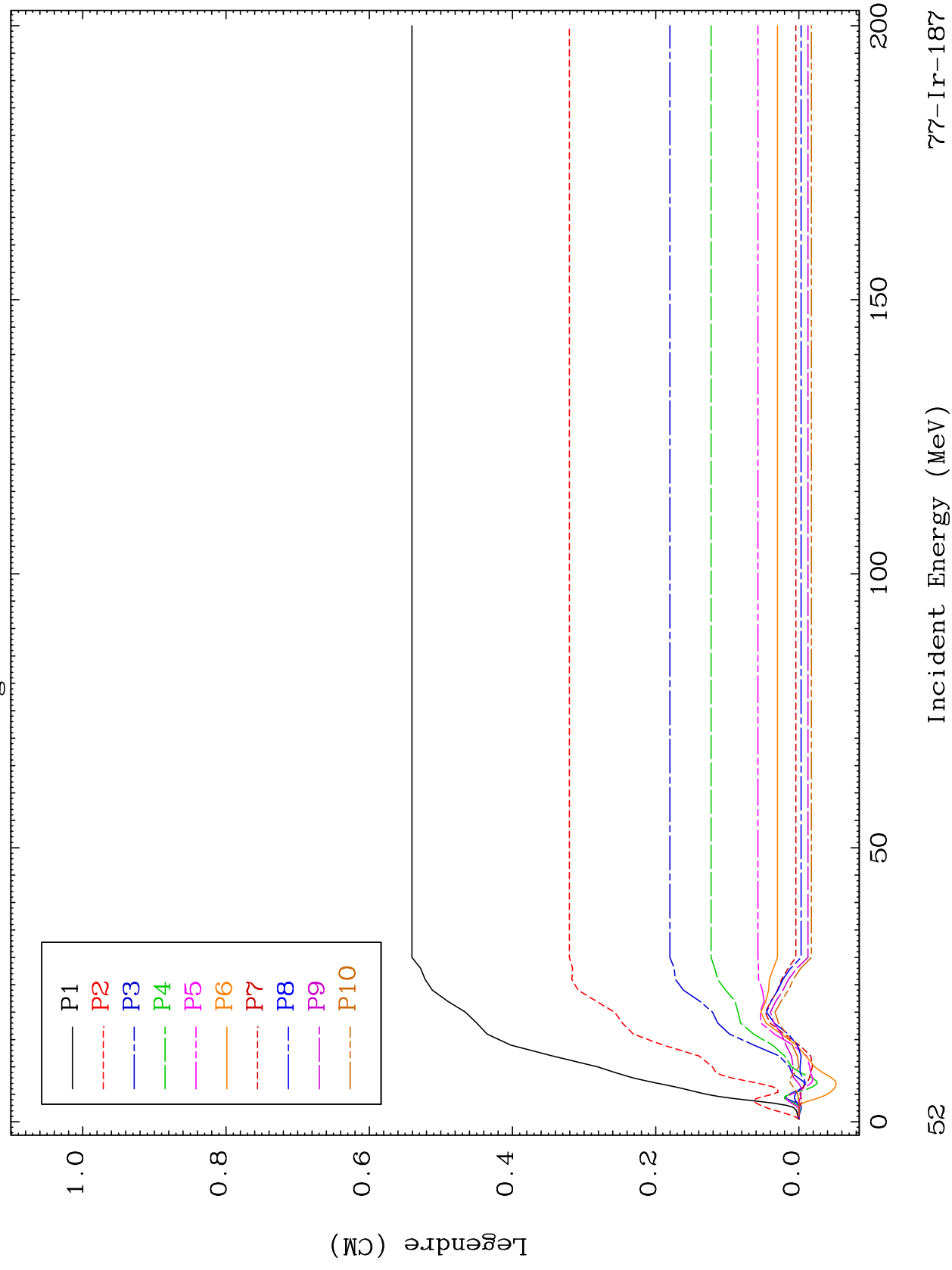


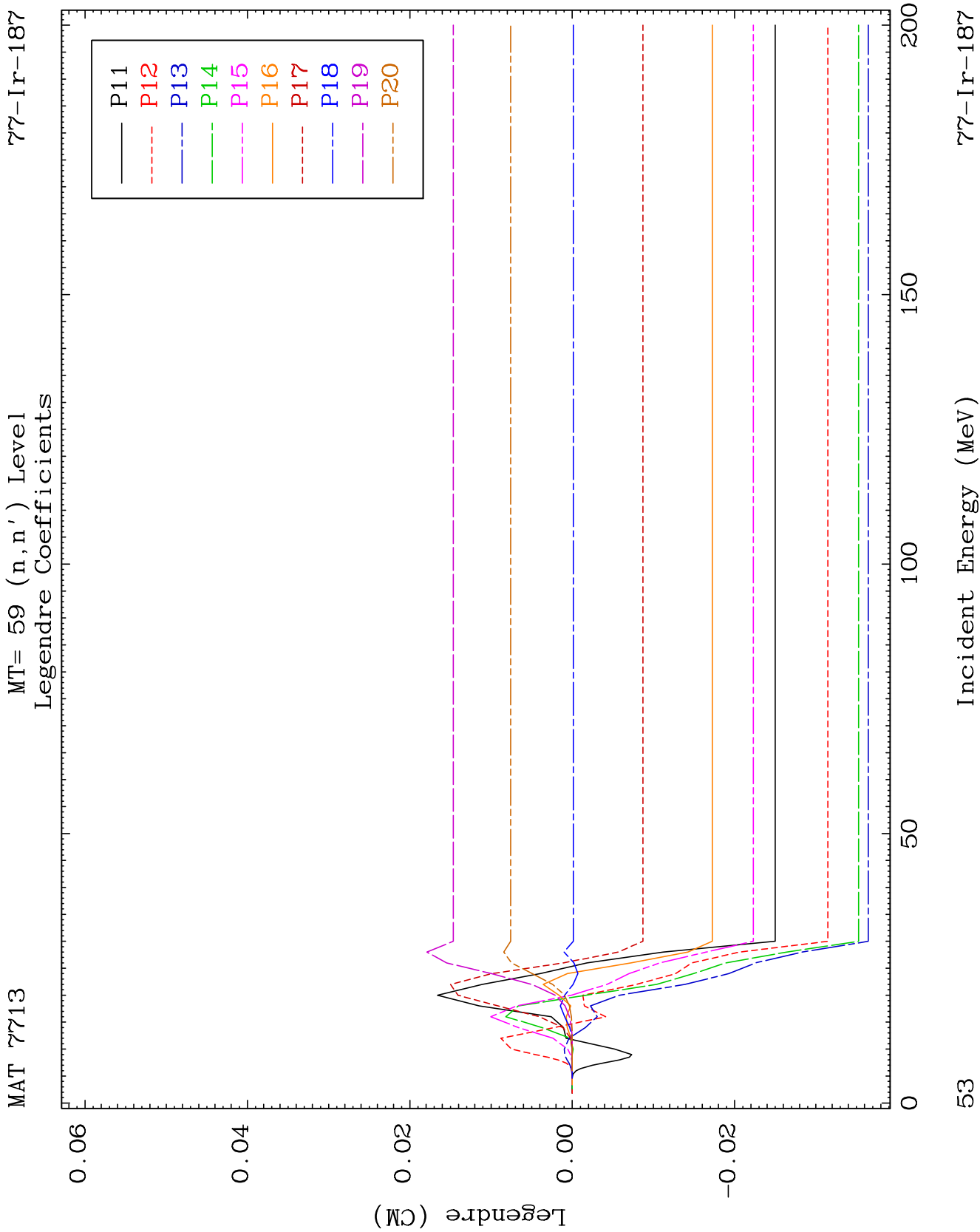


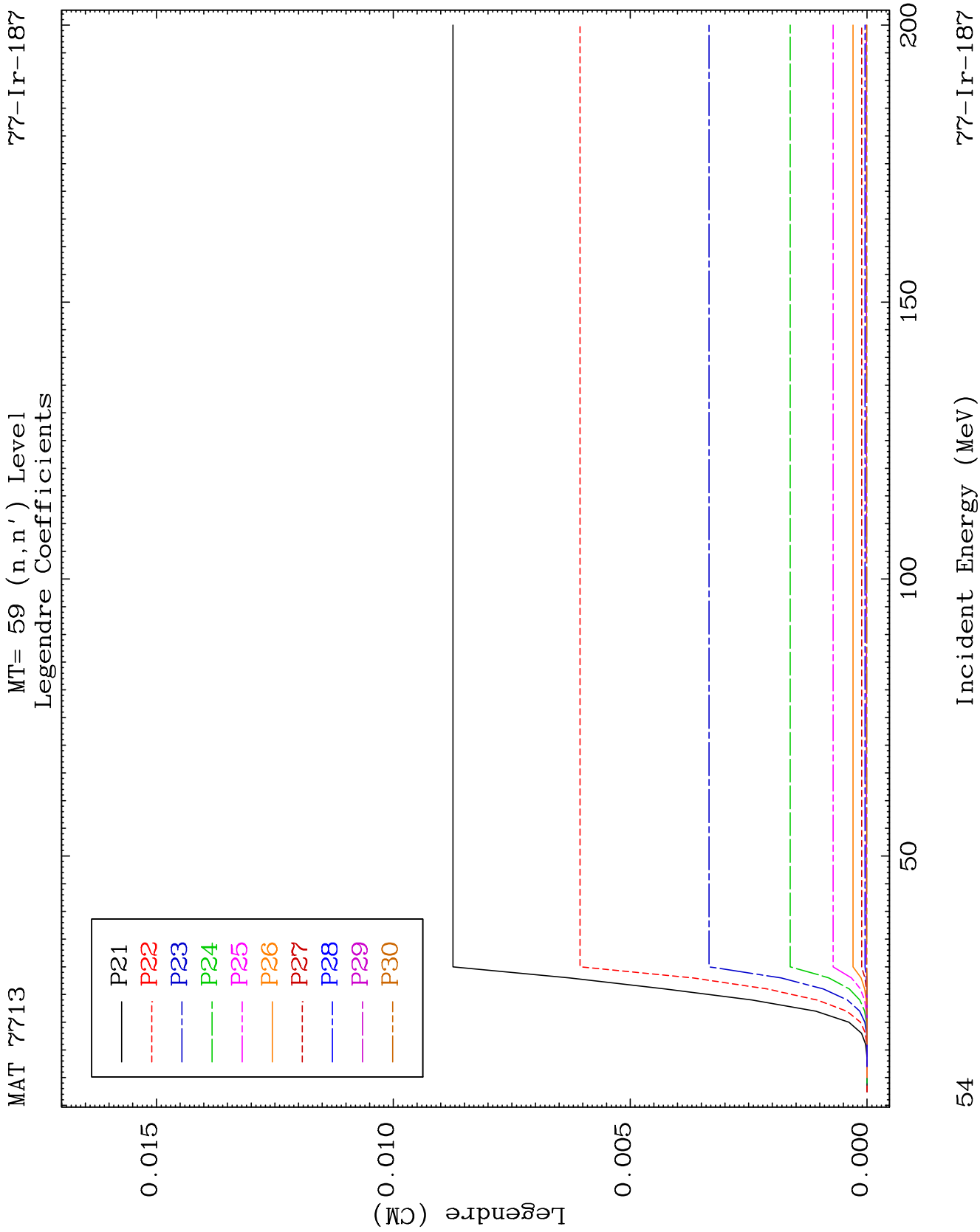
MAT 7713

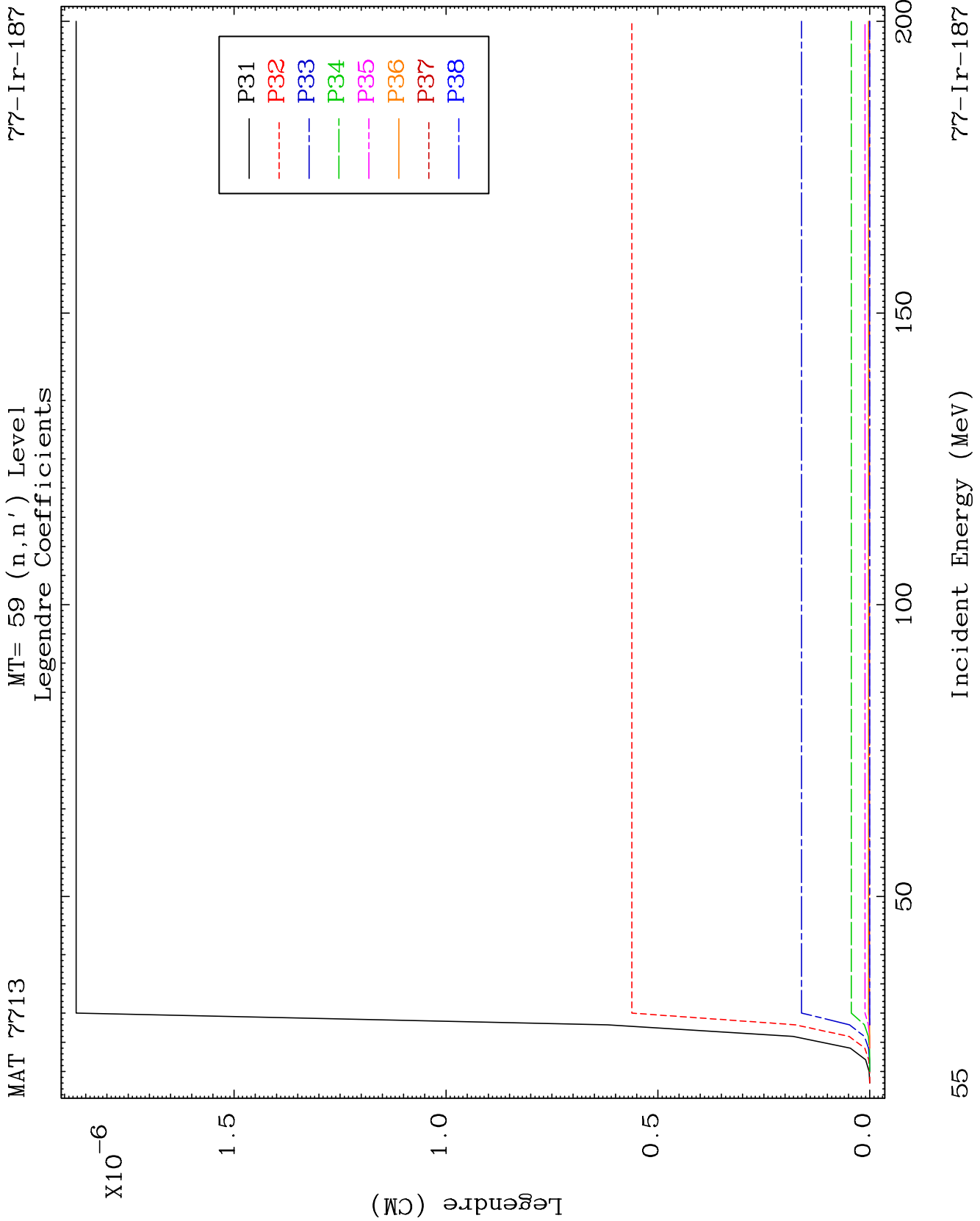
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Legendre Coefficients

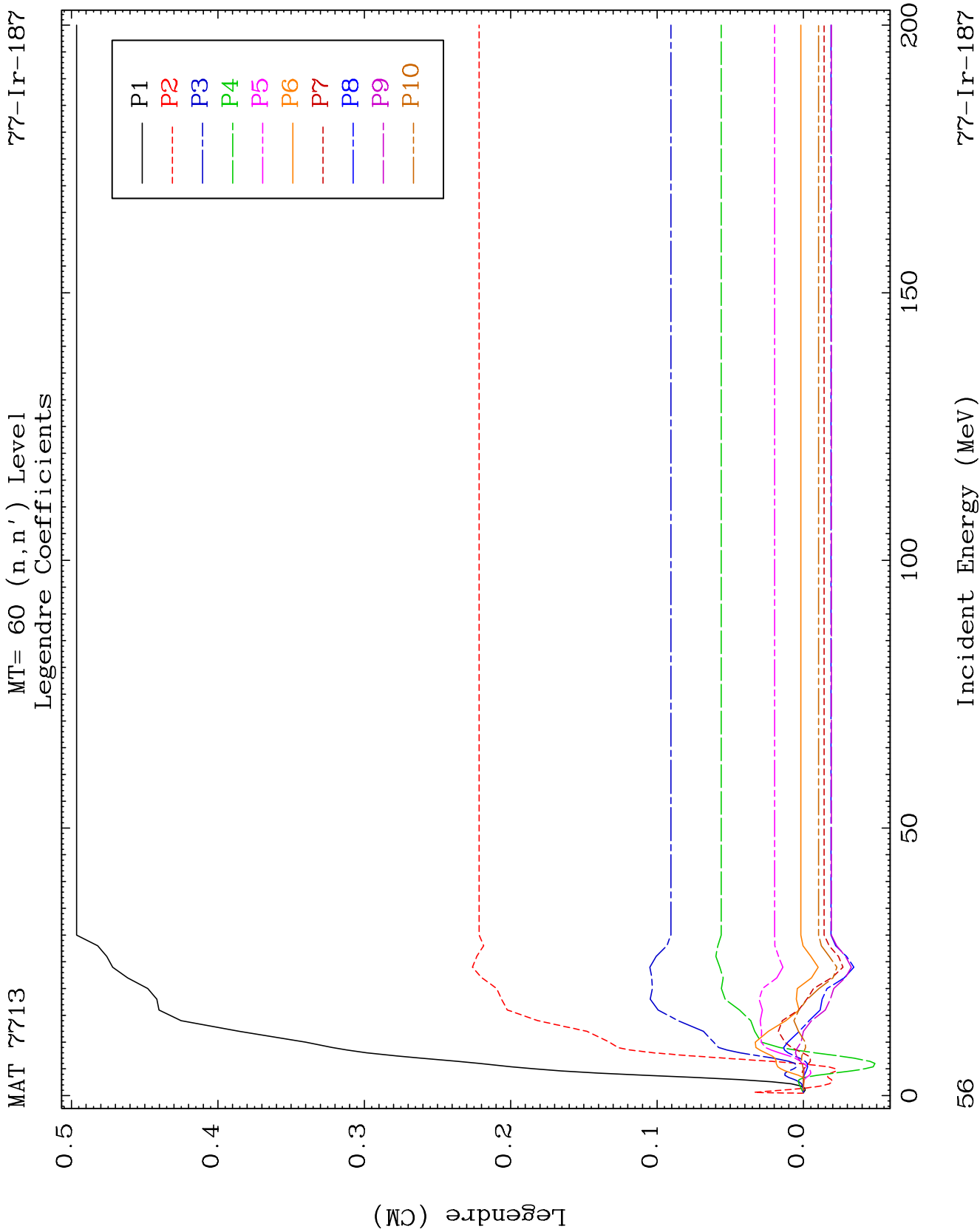
27-1r-187







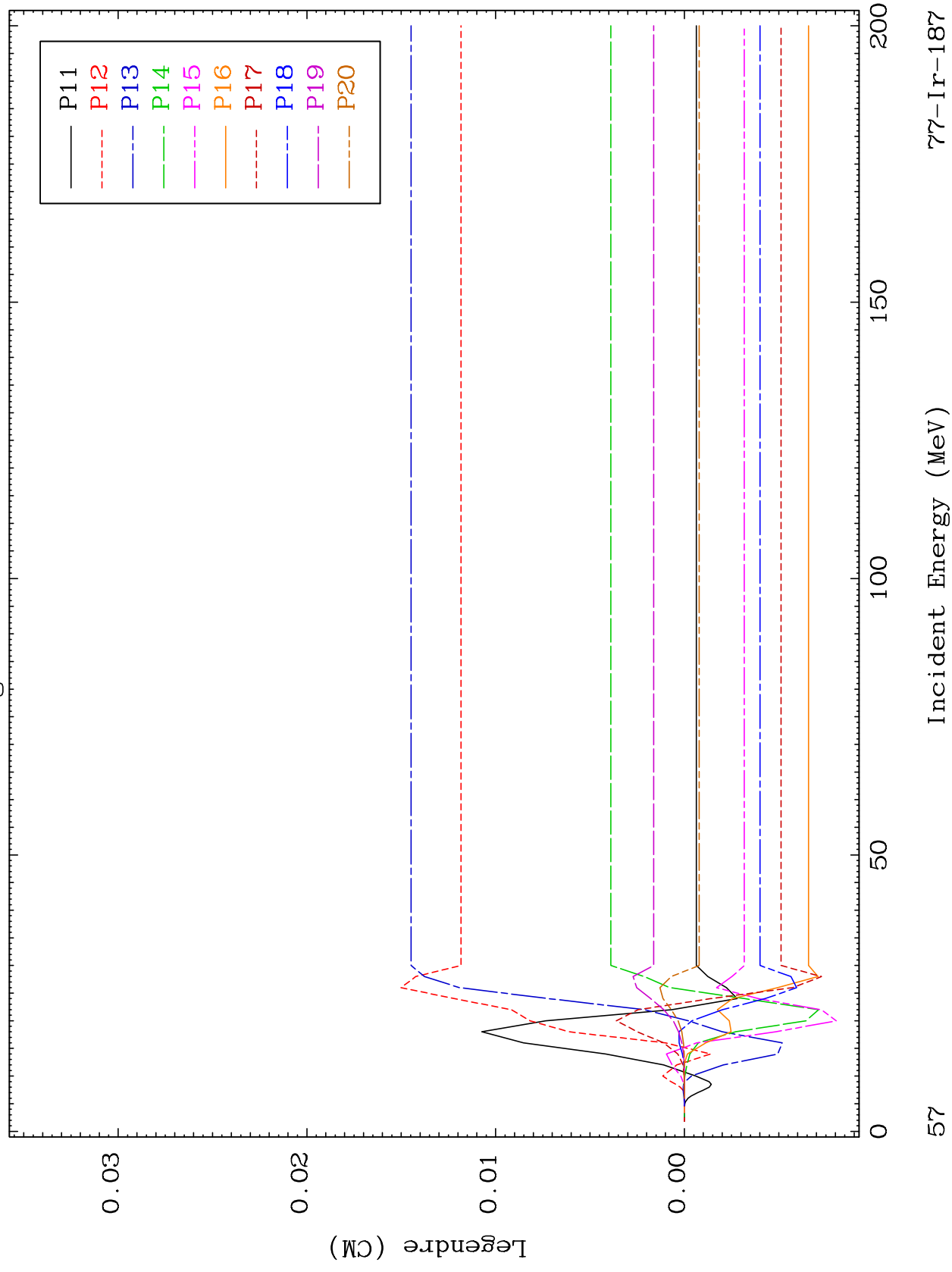


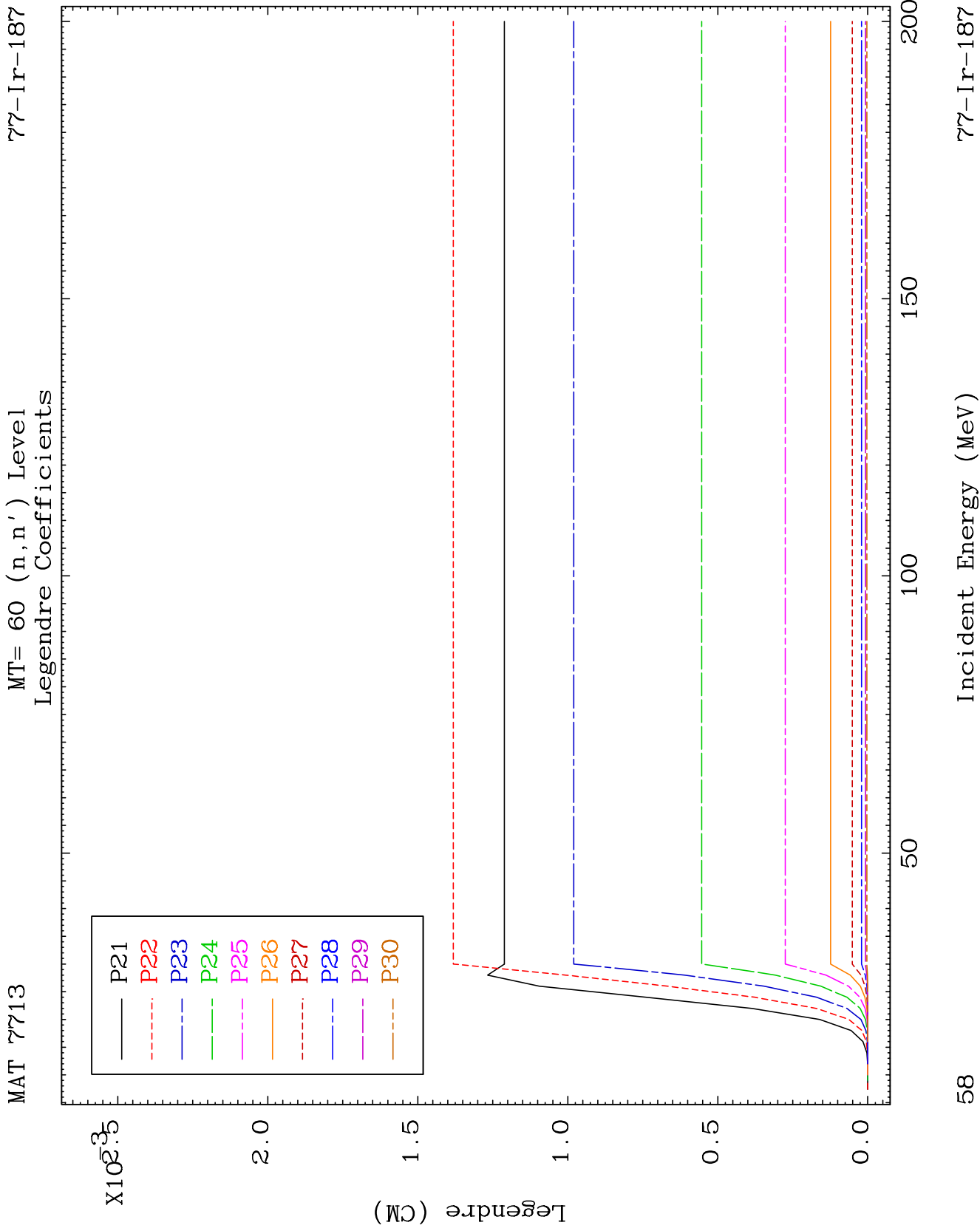


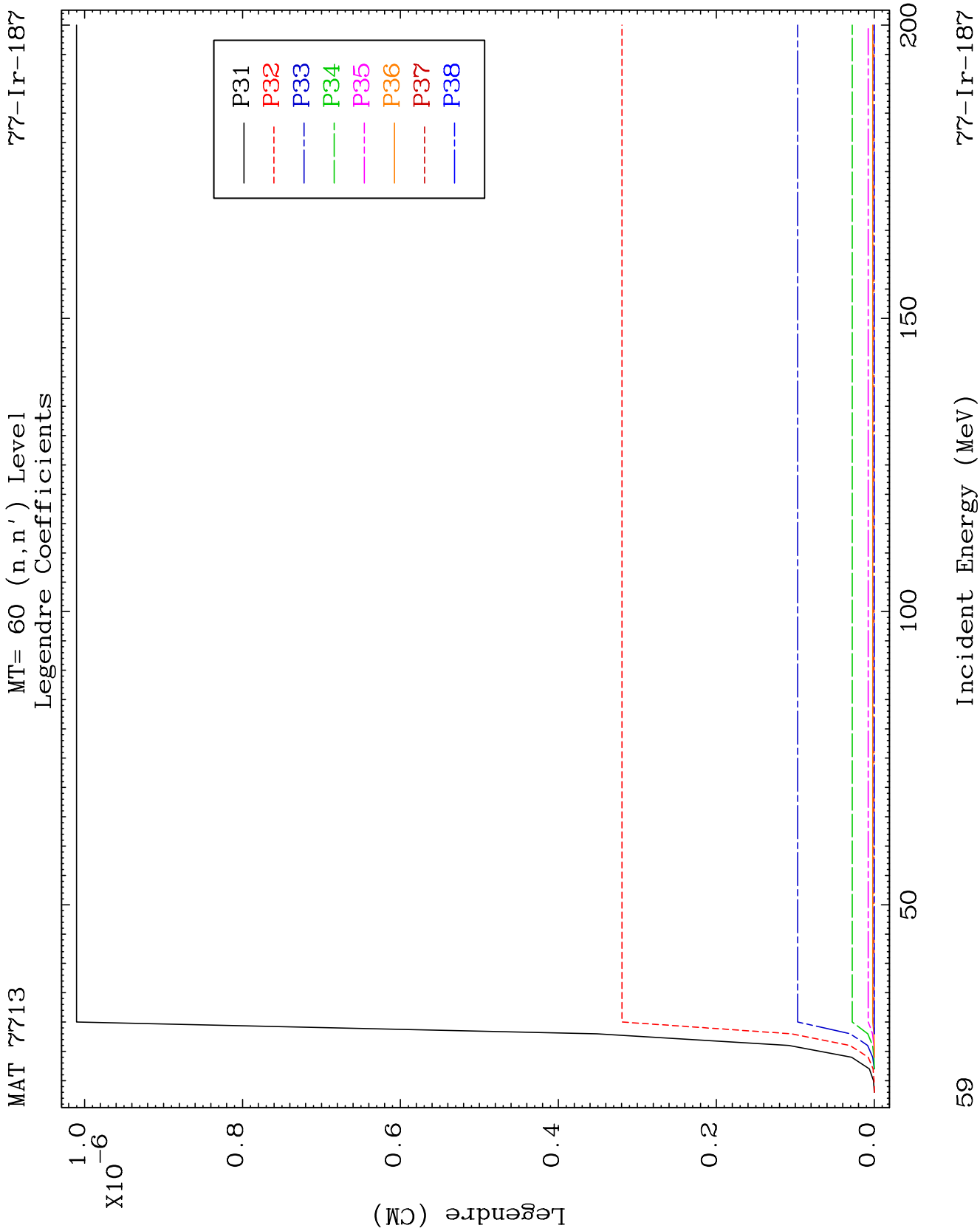
MAT 7713

MT= 60 (n, n') Level
Legendre Coefficients

22-1r-187



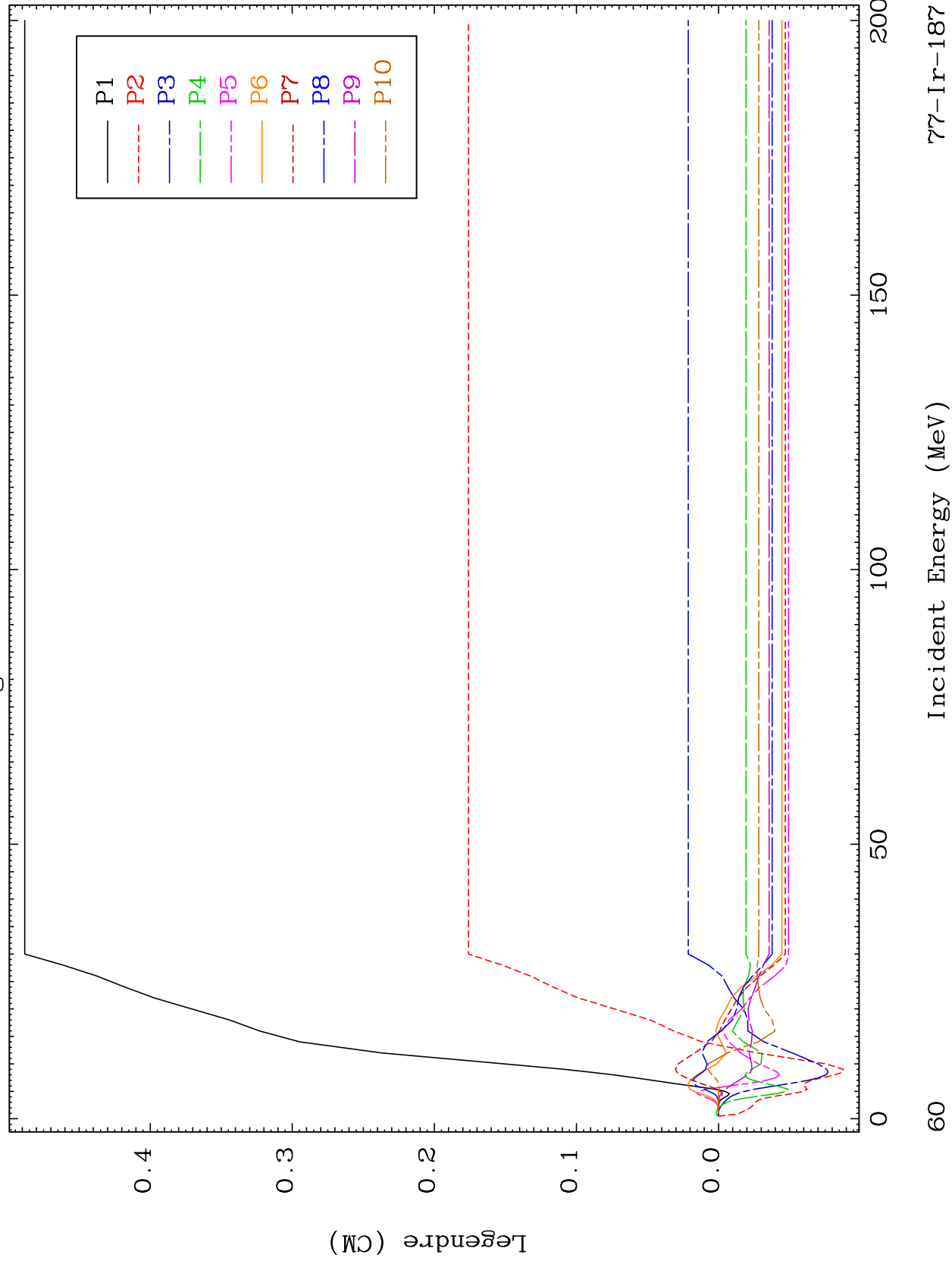


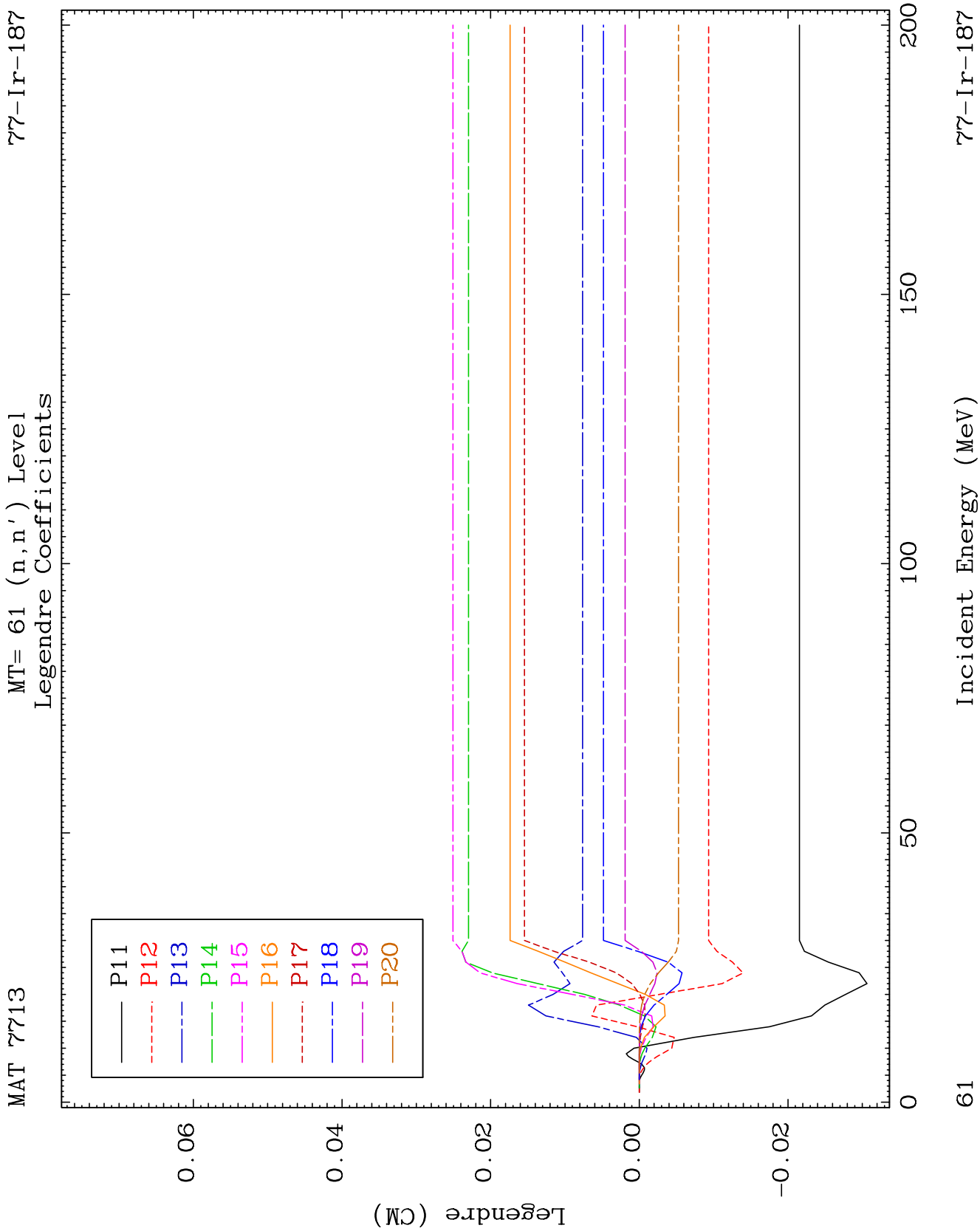


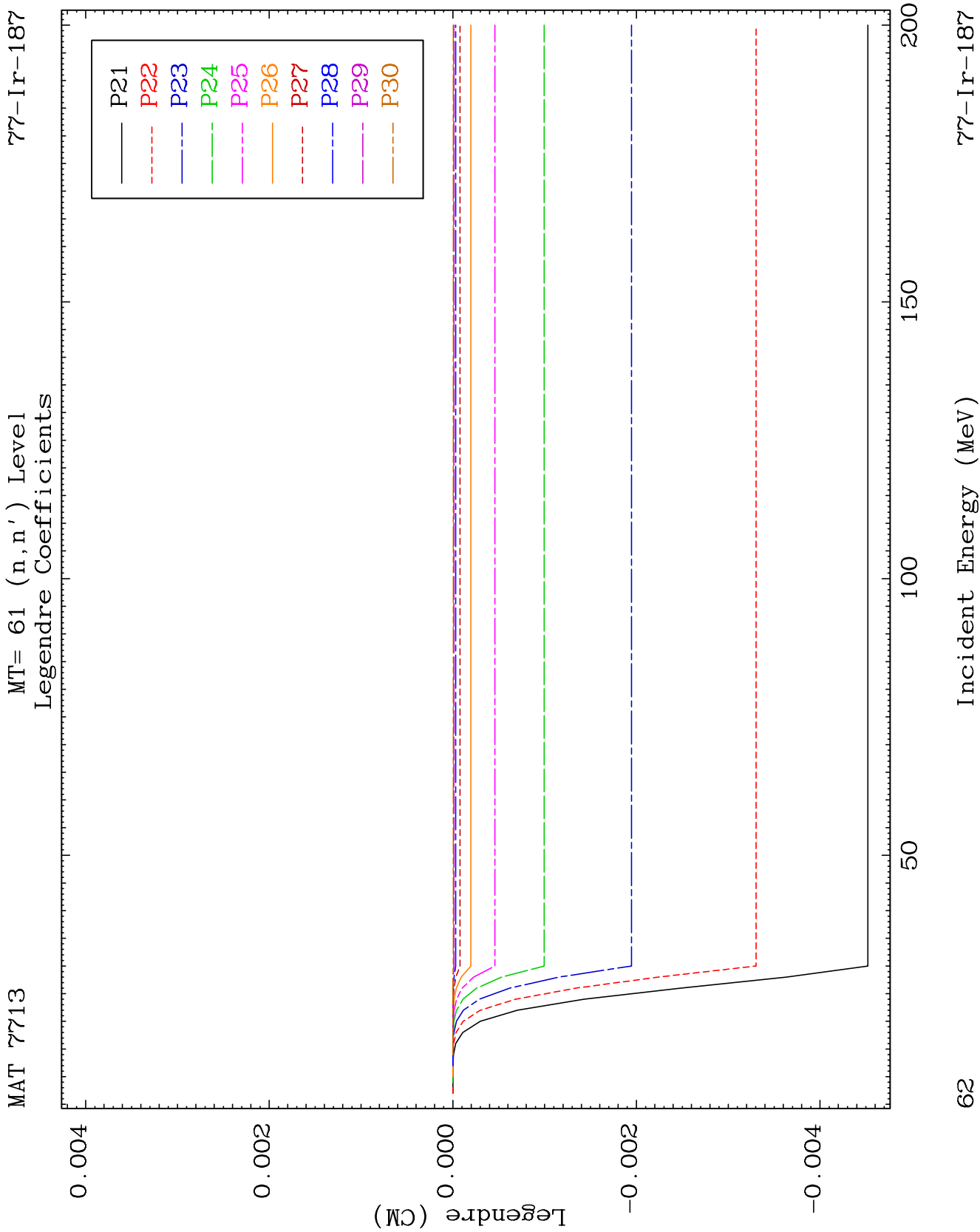
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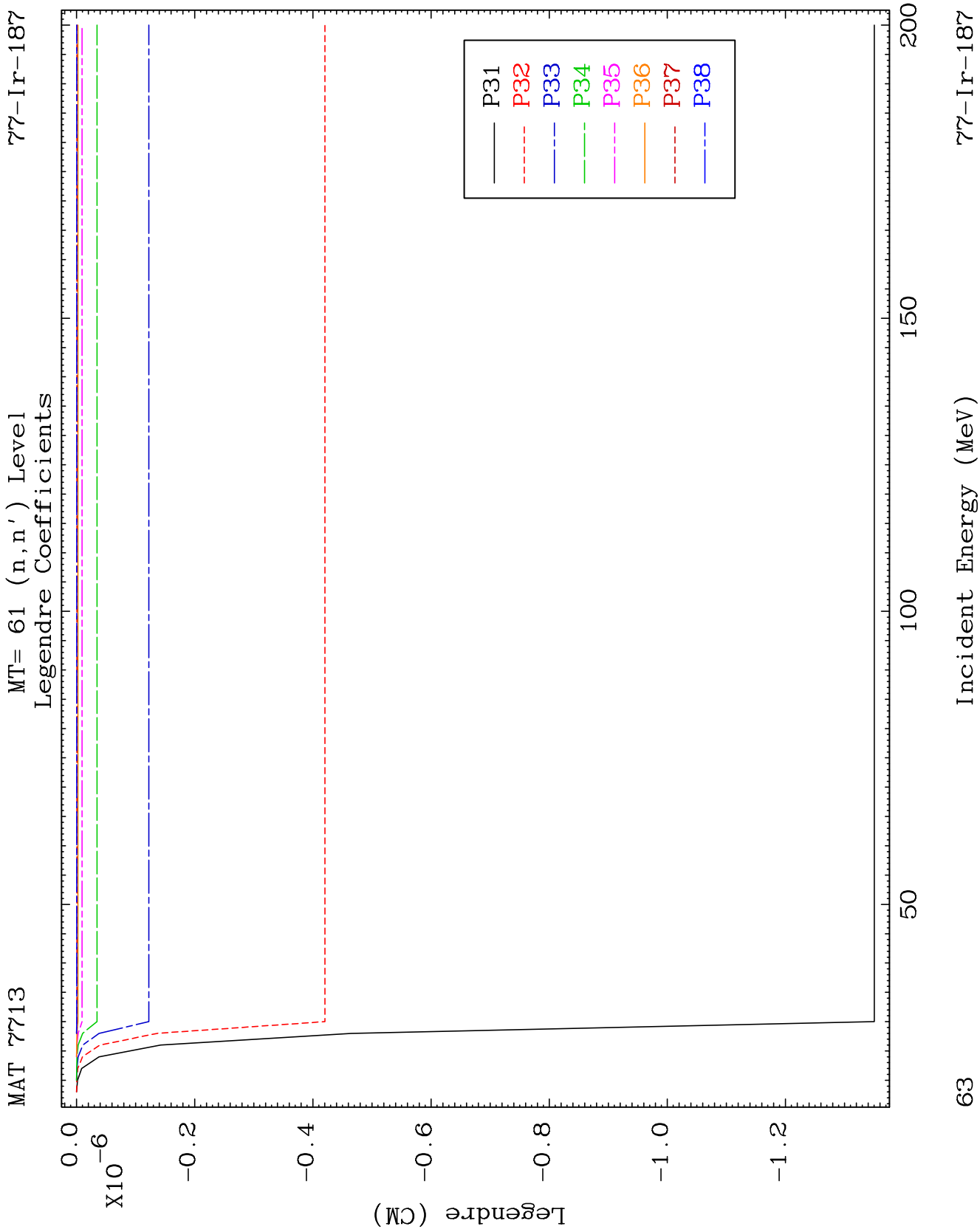
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Legendre Coefficients

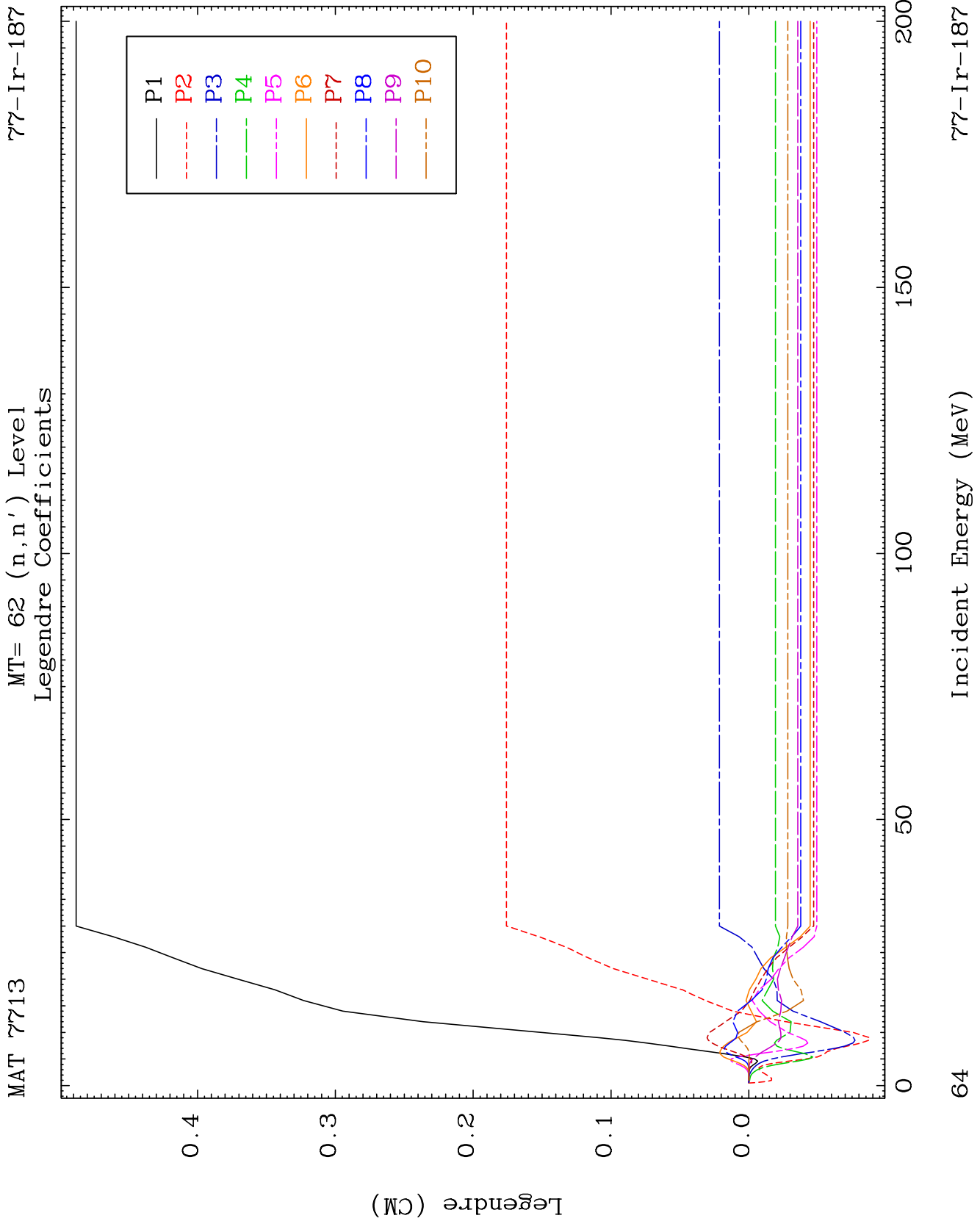
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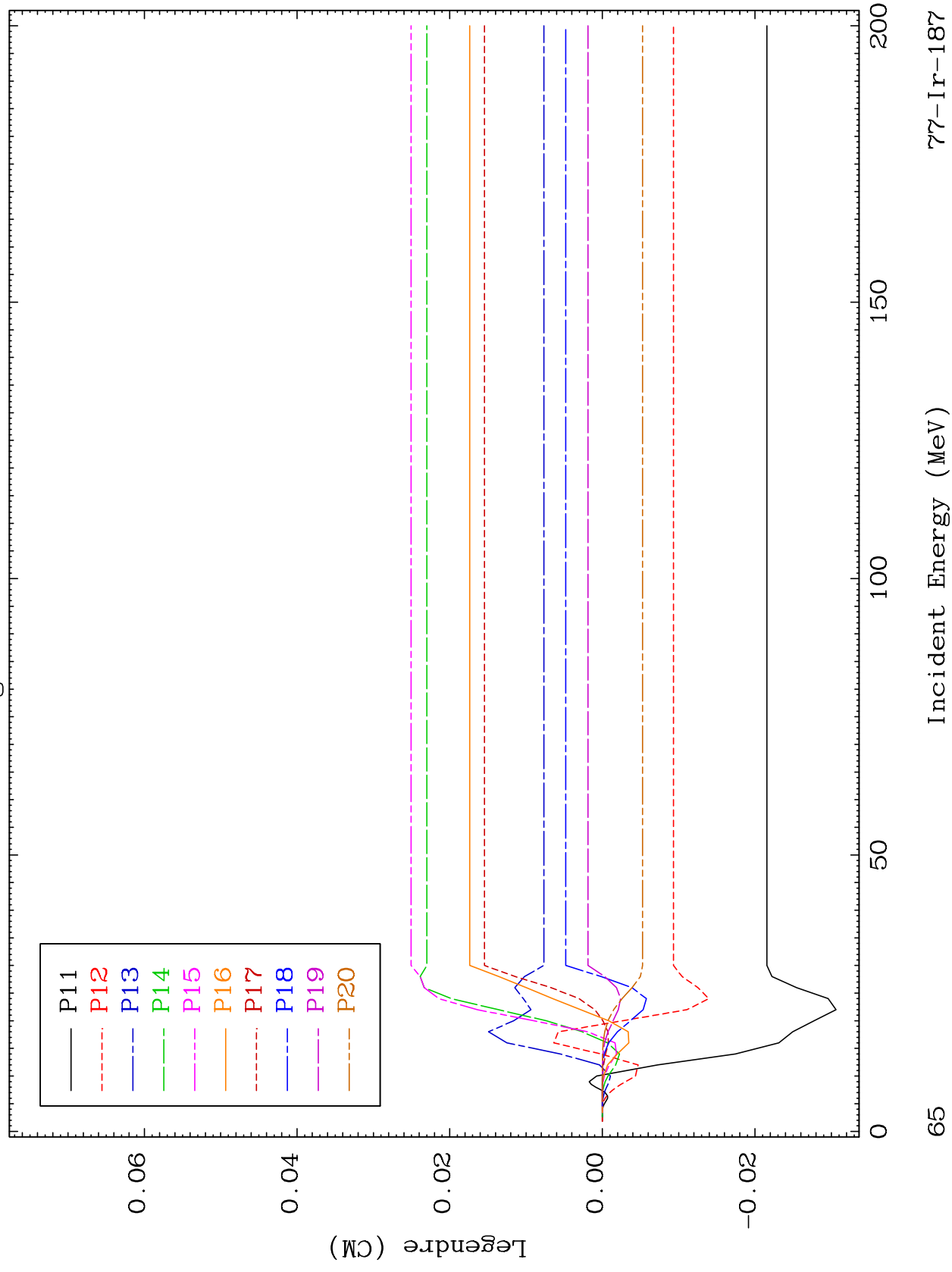


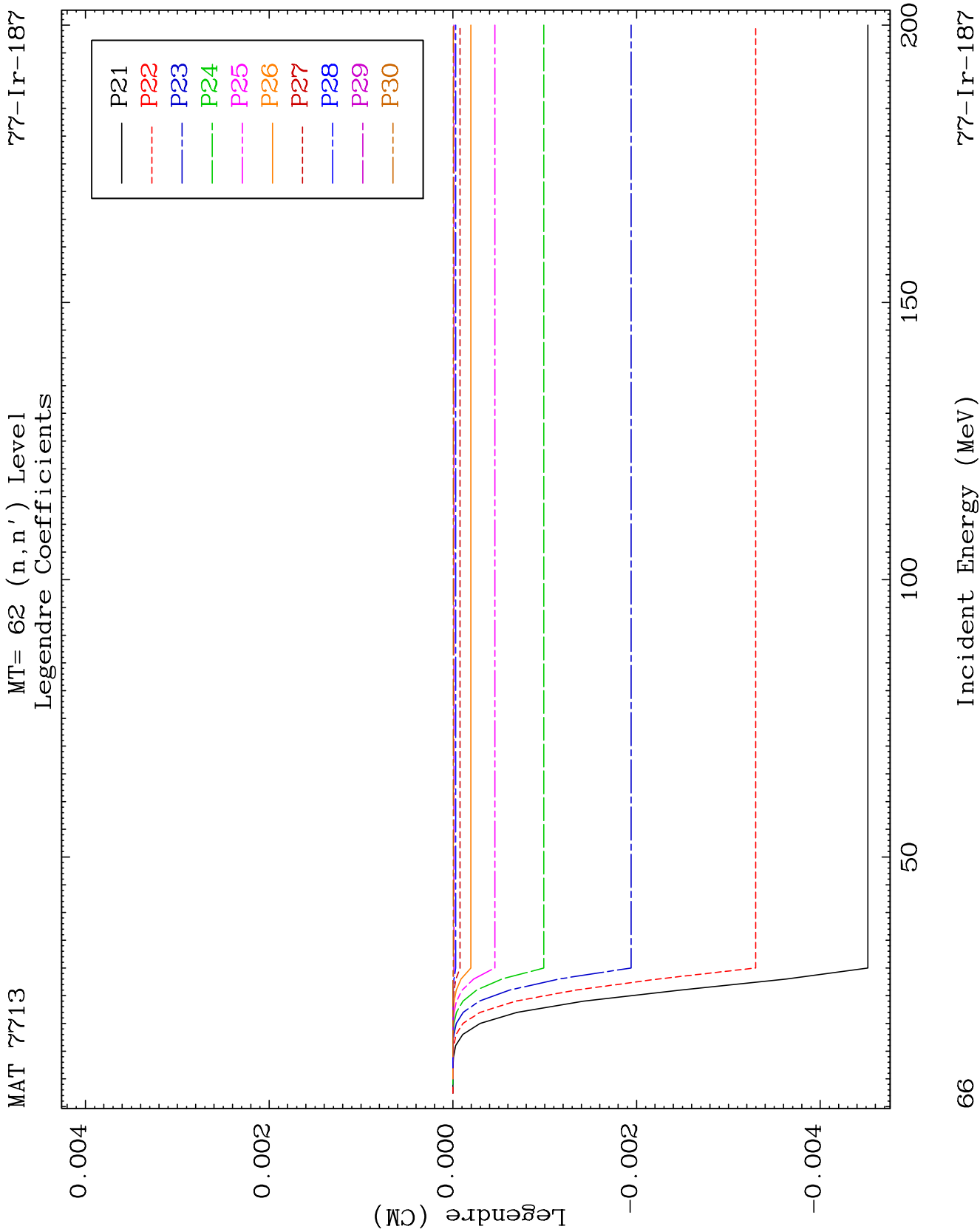


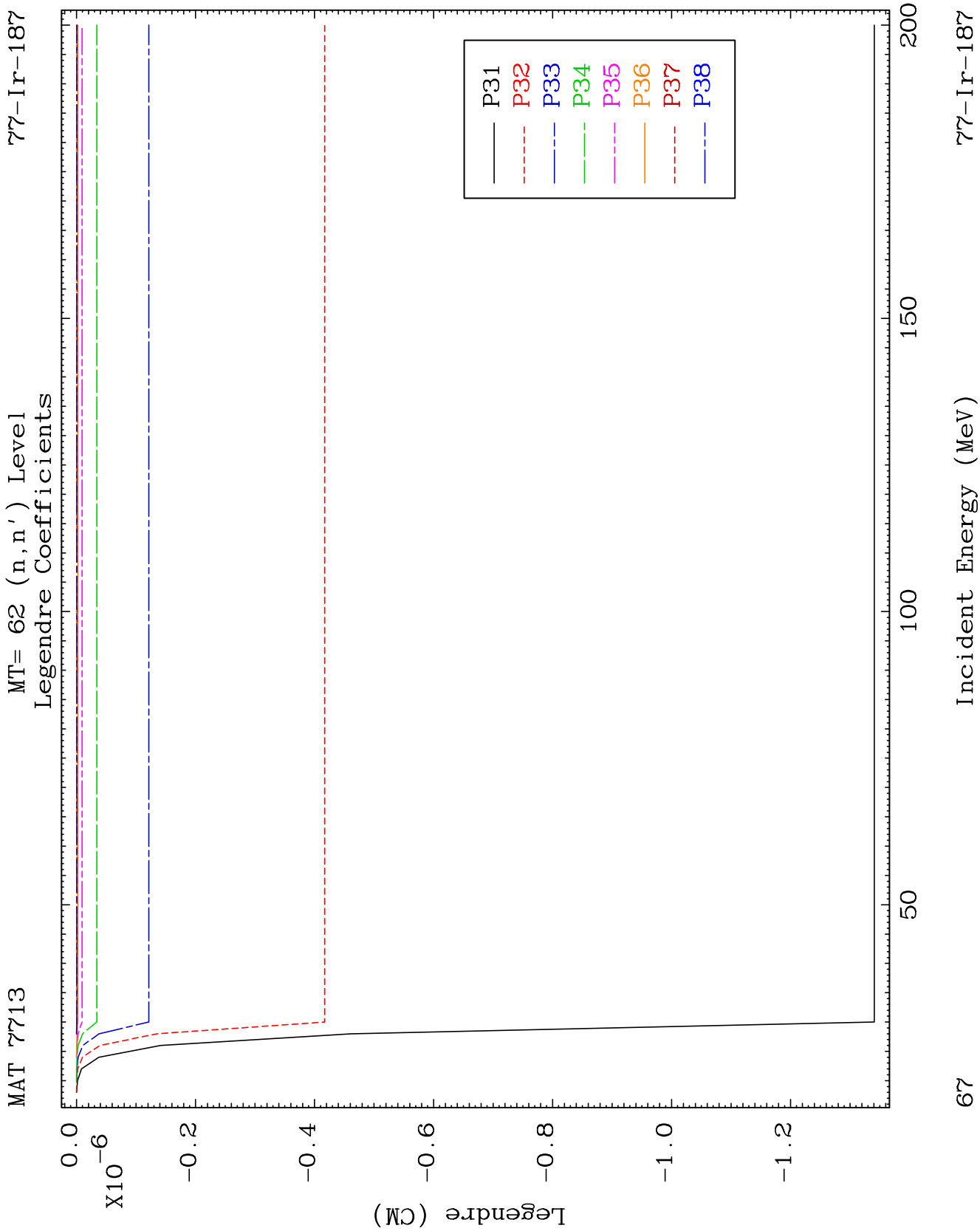
MAT 7713

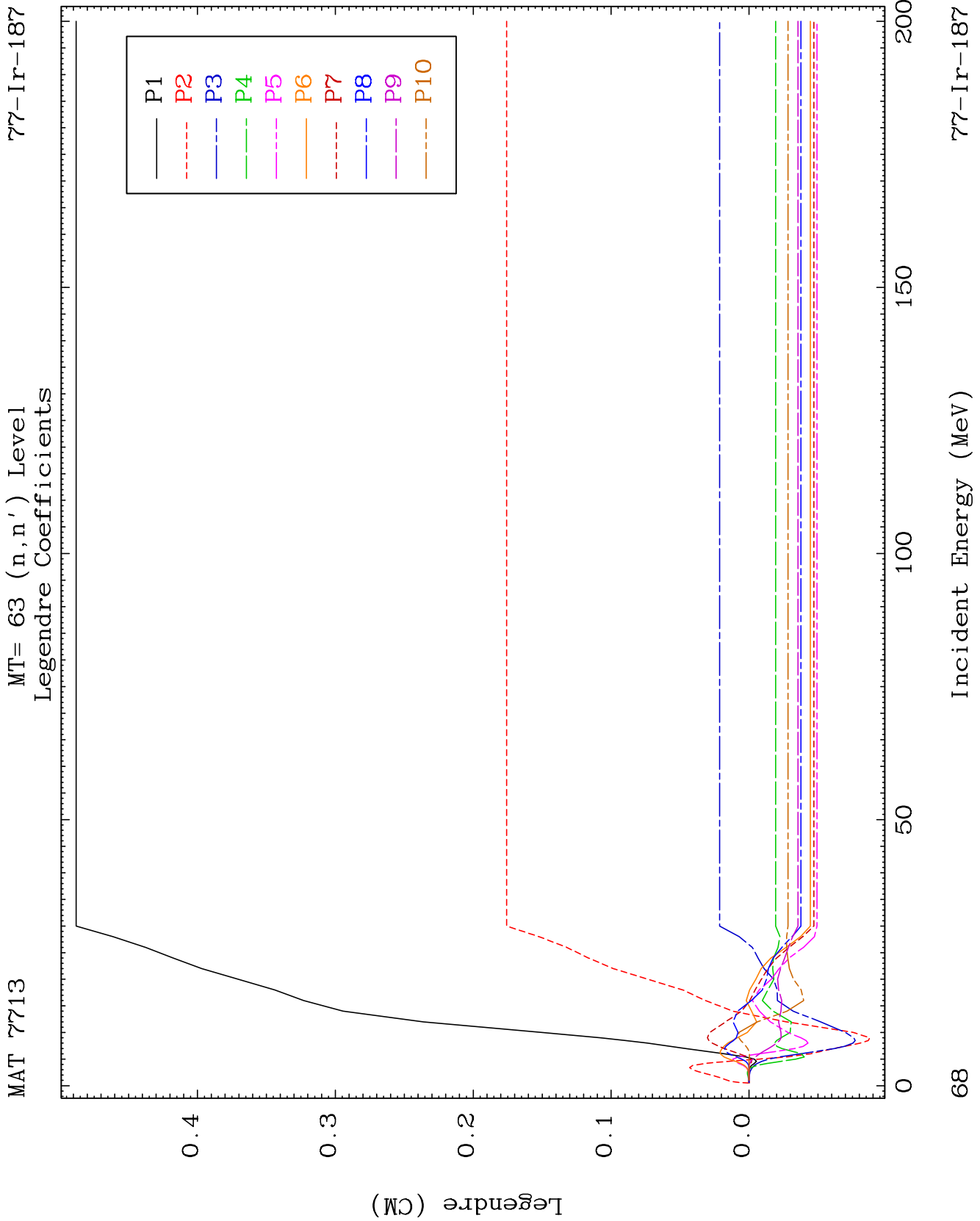
MT= 62 (n, n') Level
Legendre Coefficients

22-Ir-187





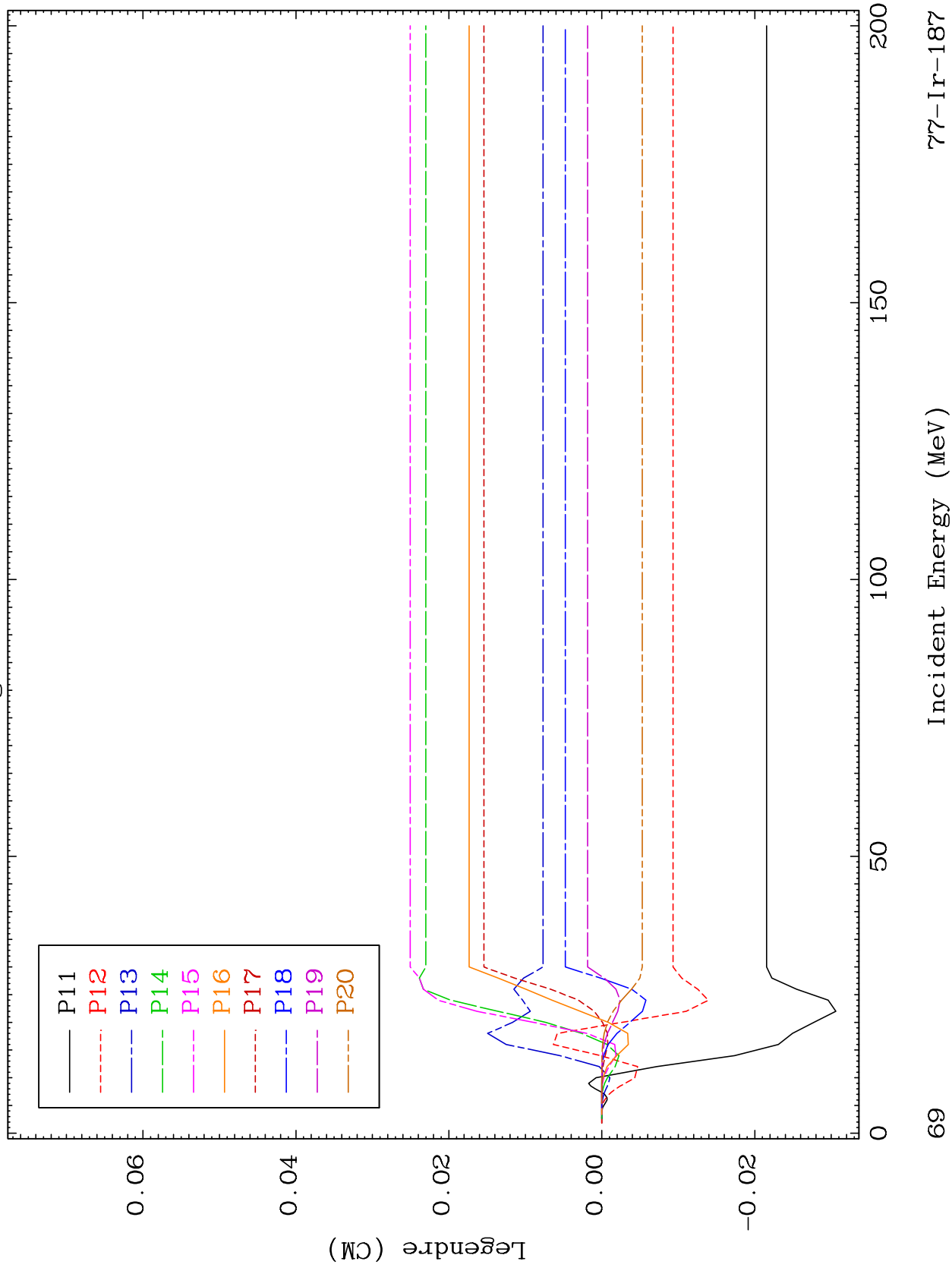


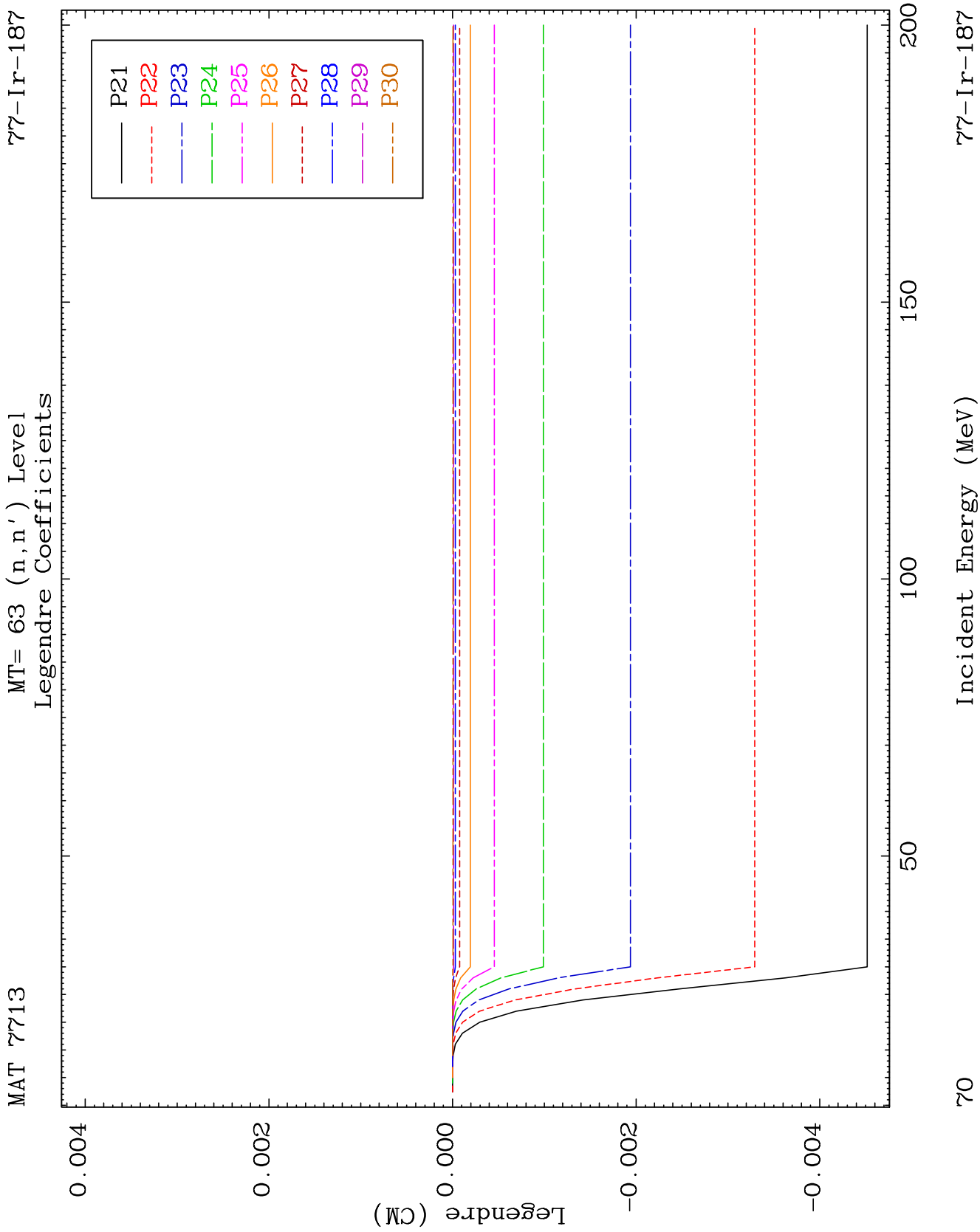


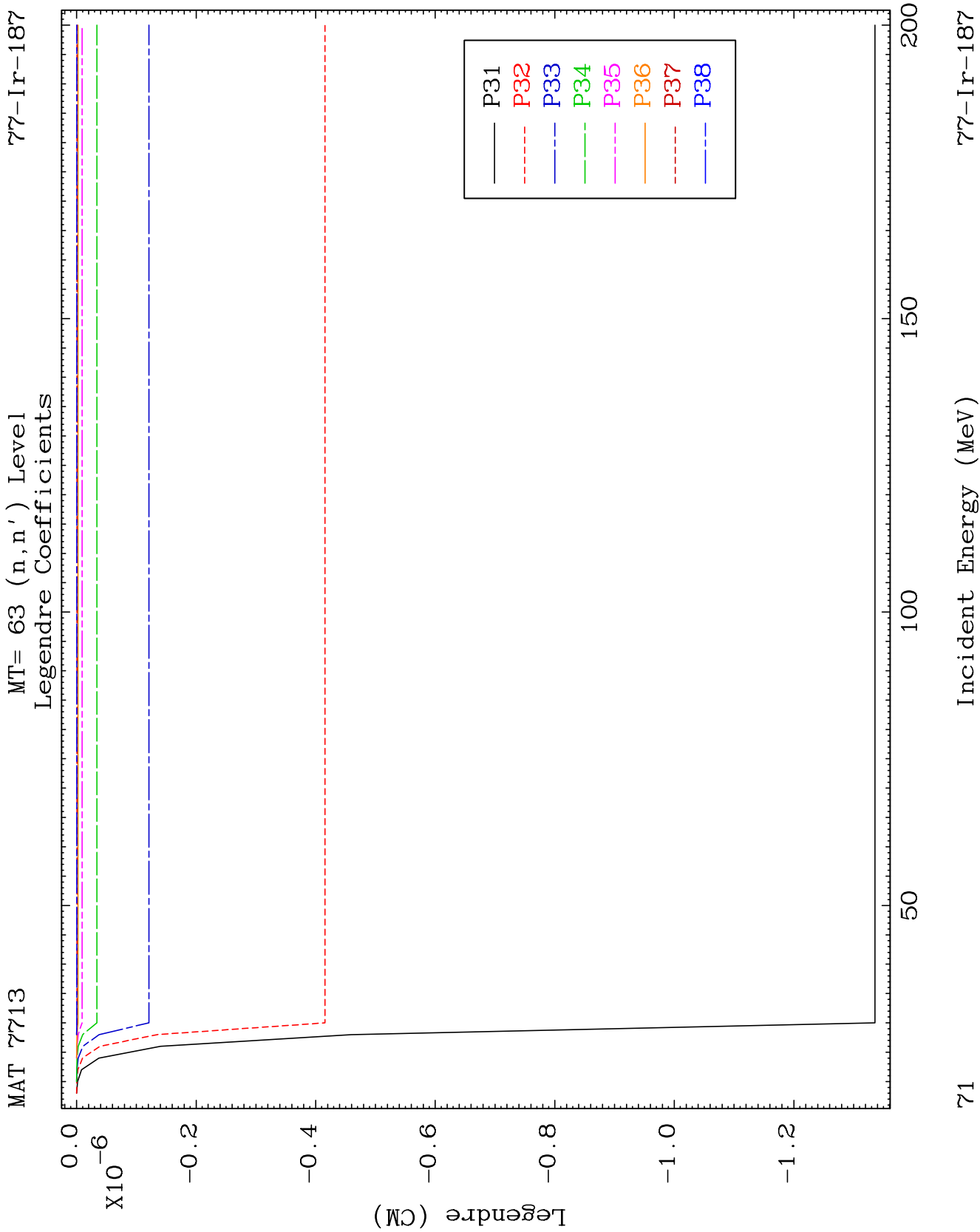
MAT 7713

MT= 63 (n, n') Level
Legendre Coefficients

22-Ir-187



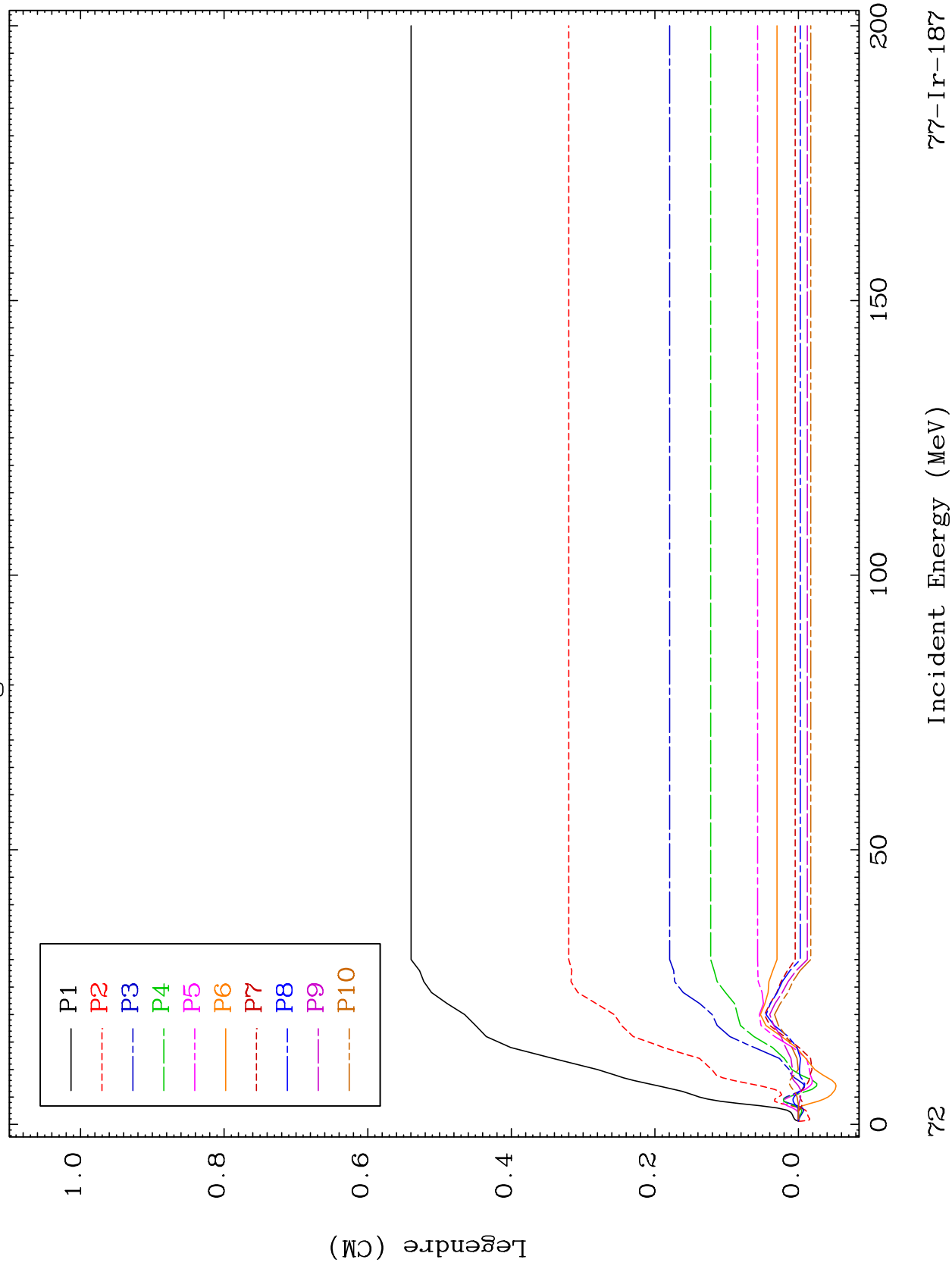




MAT 7713

MT= 64 (n, n') Level
Legendre Coefficients

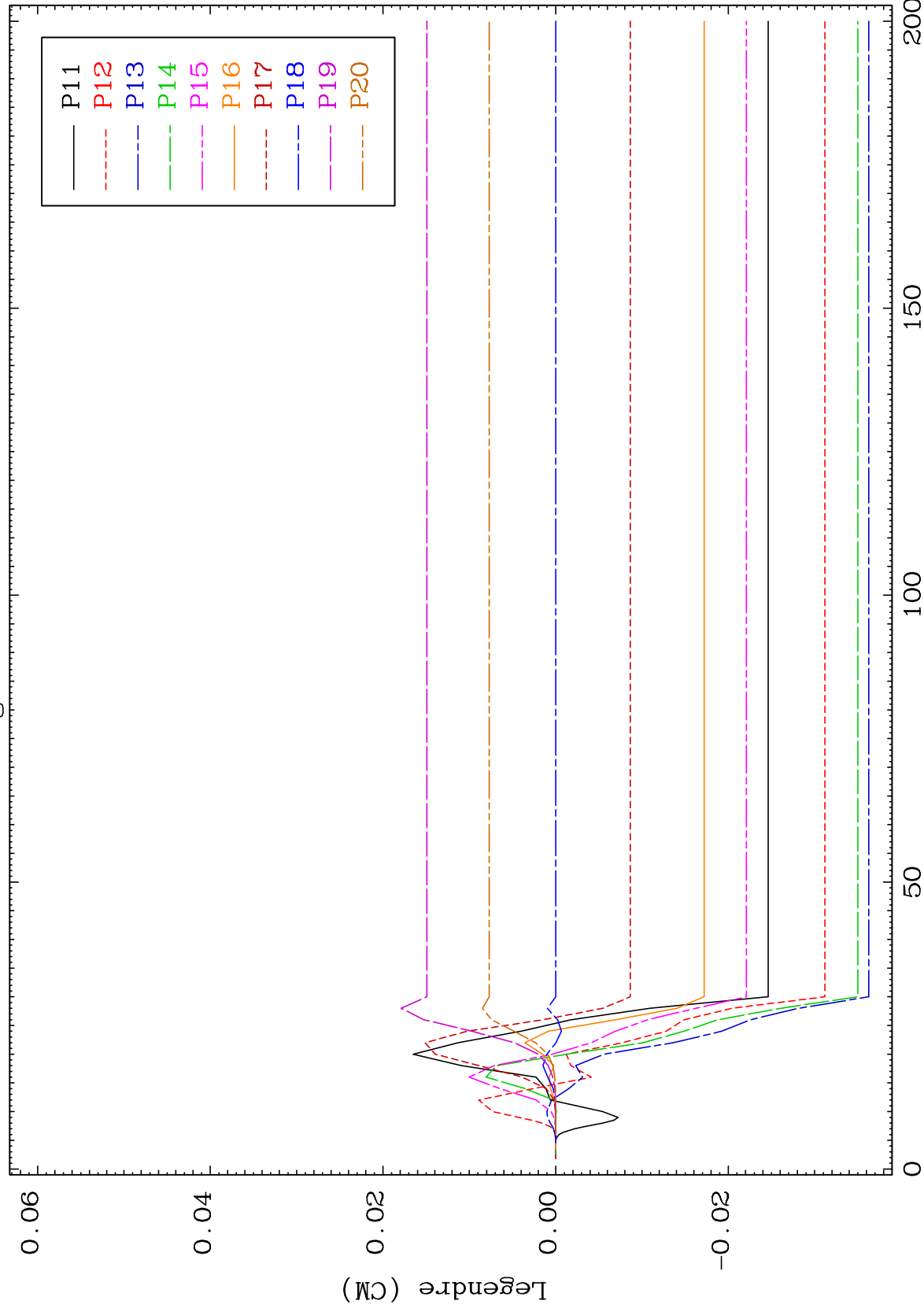
27-1r-187



MAT 7713

MT= 64 (n, n') Level
Legendre Coefficients

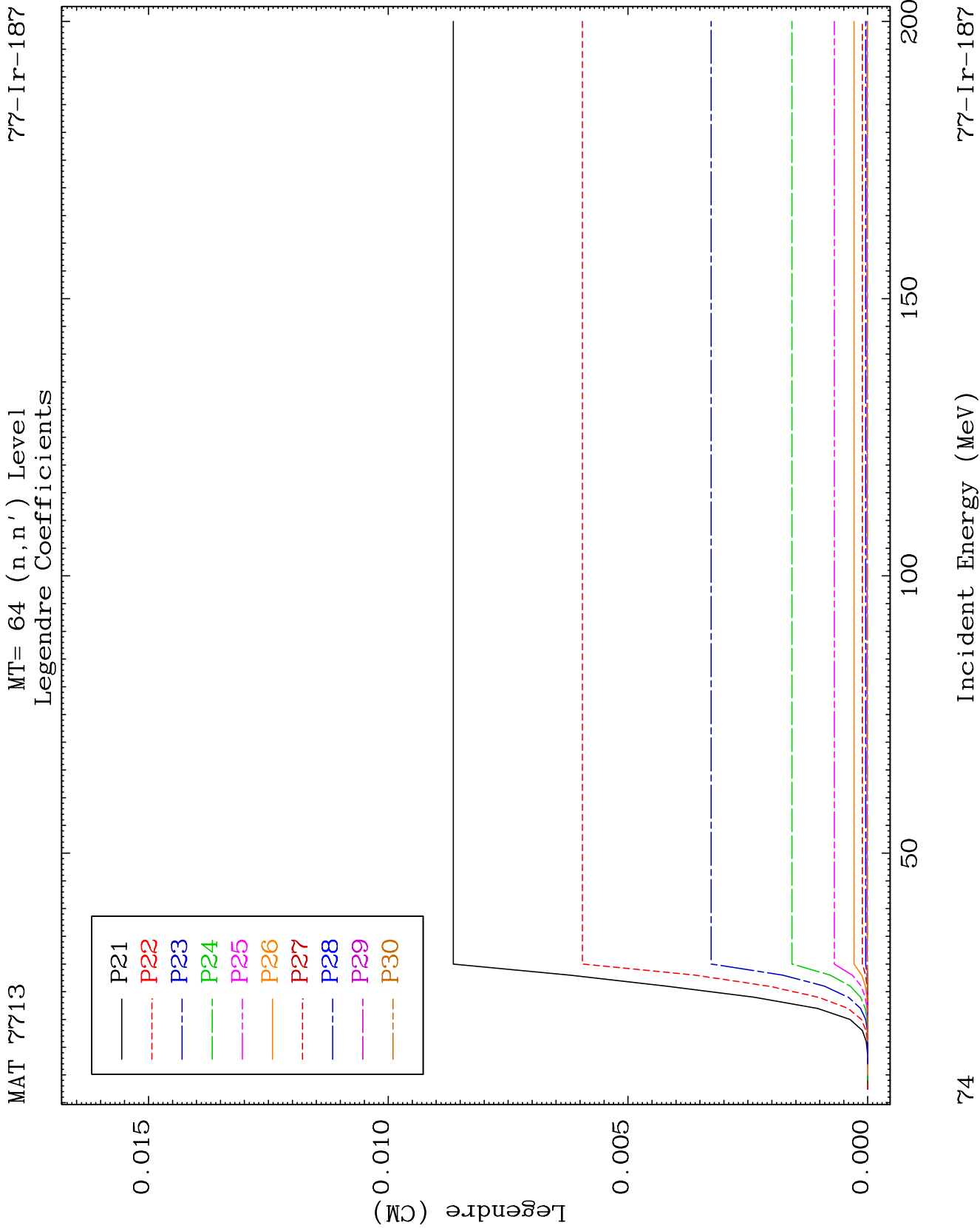
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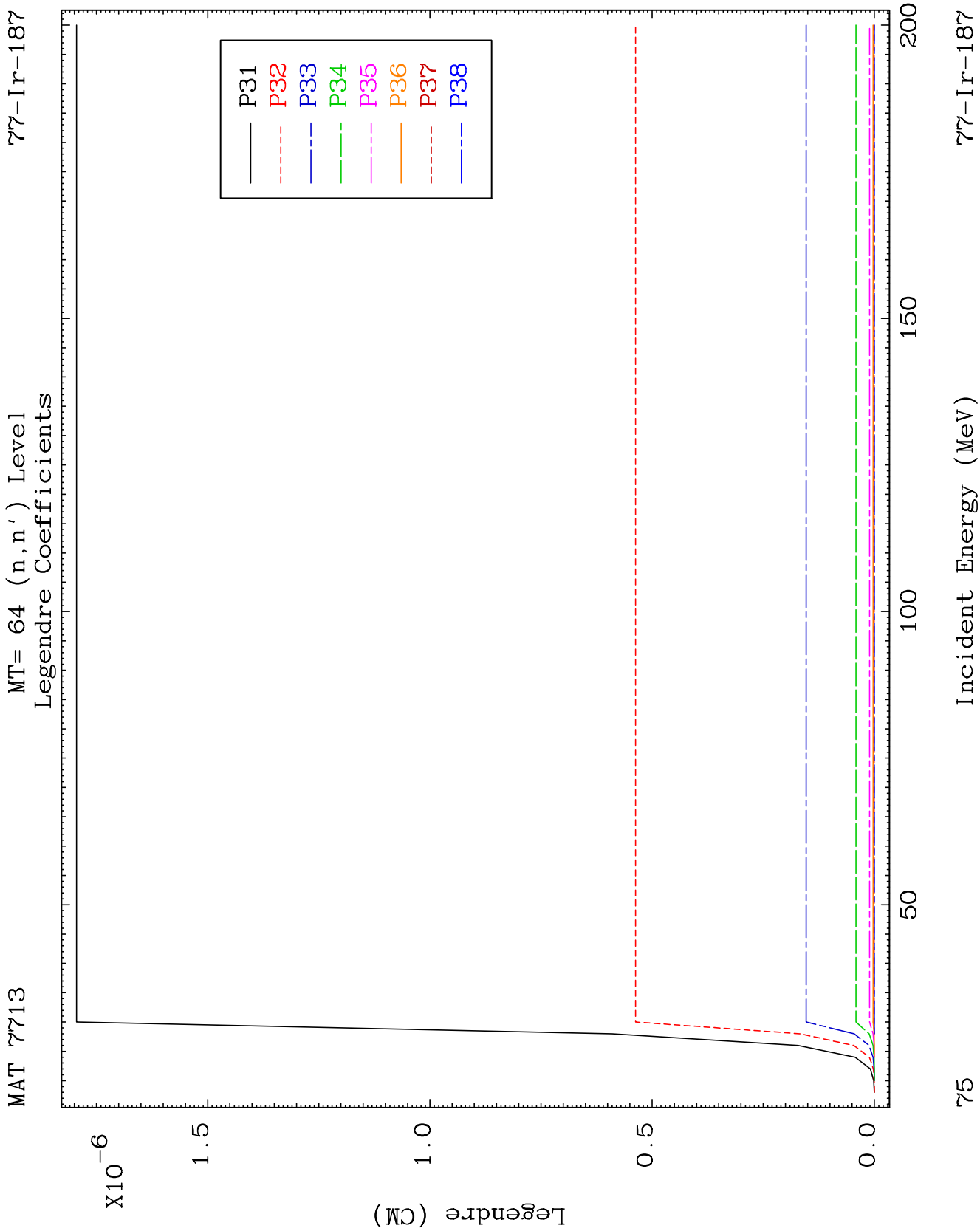


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Incident Energy (MeV)

22-1r-187

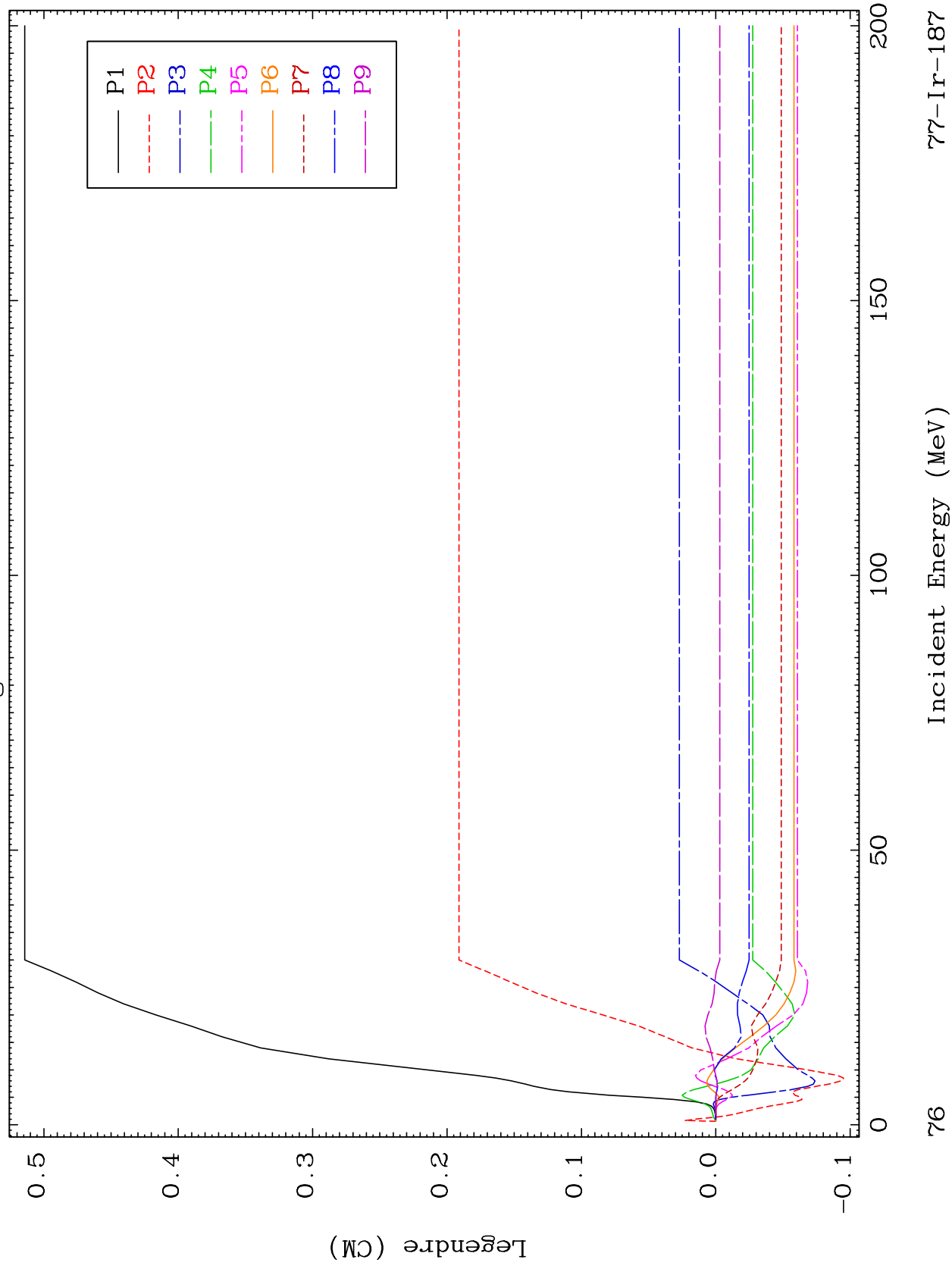


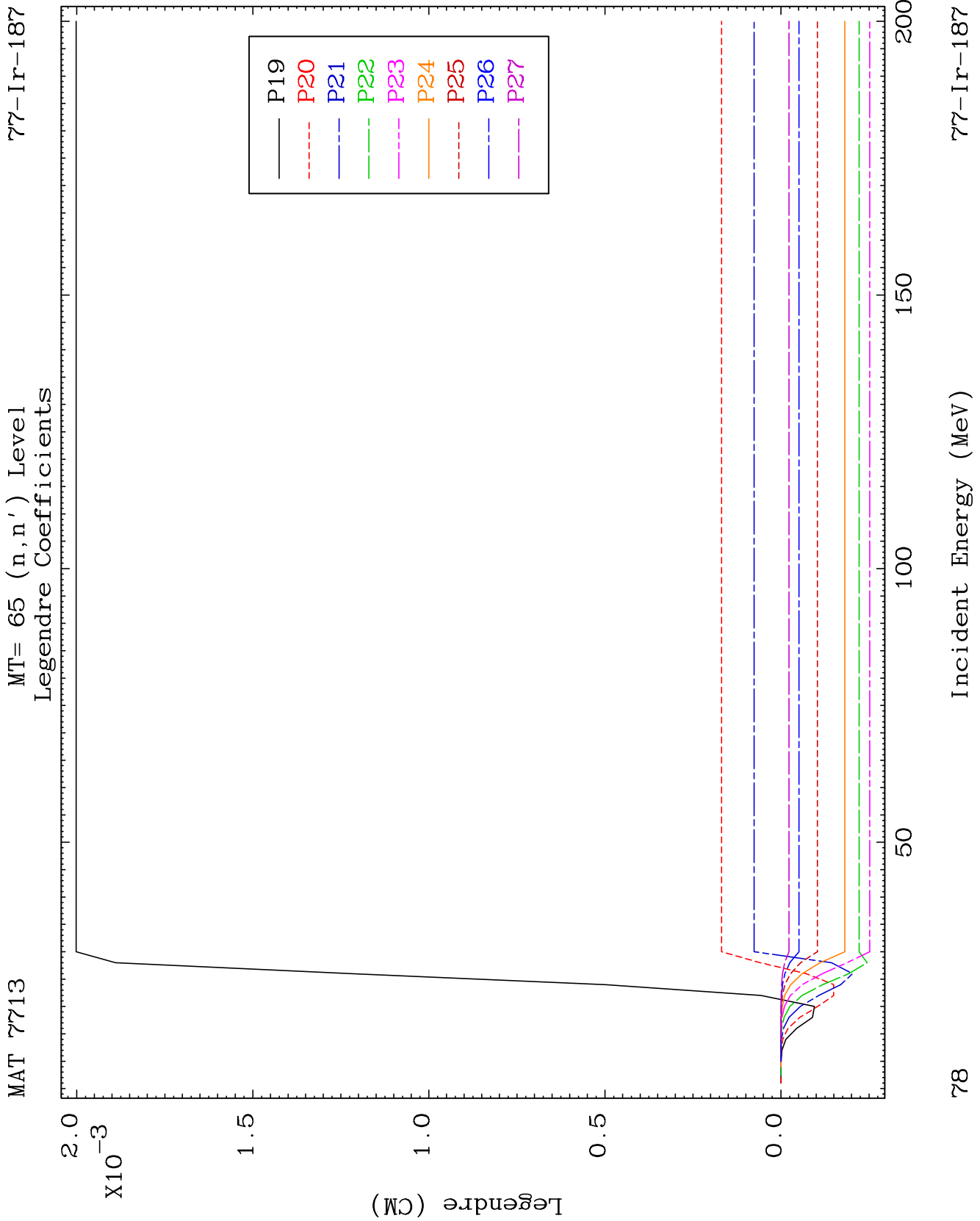


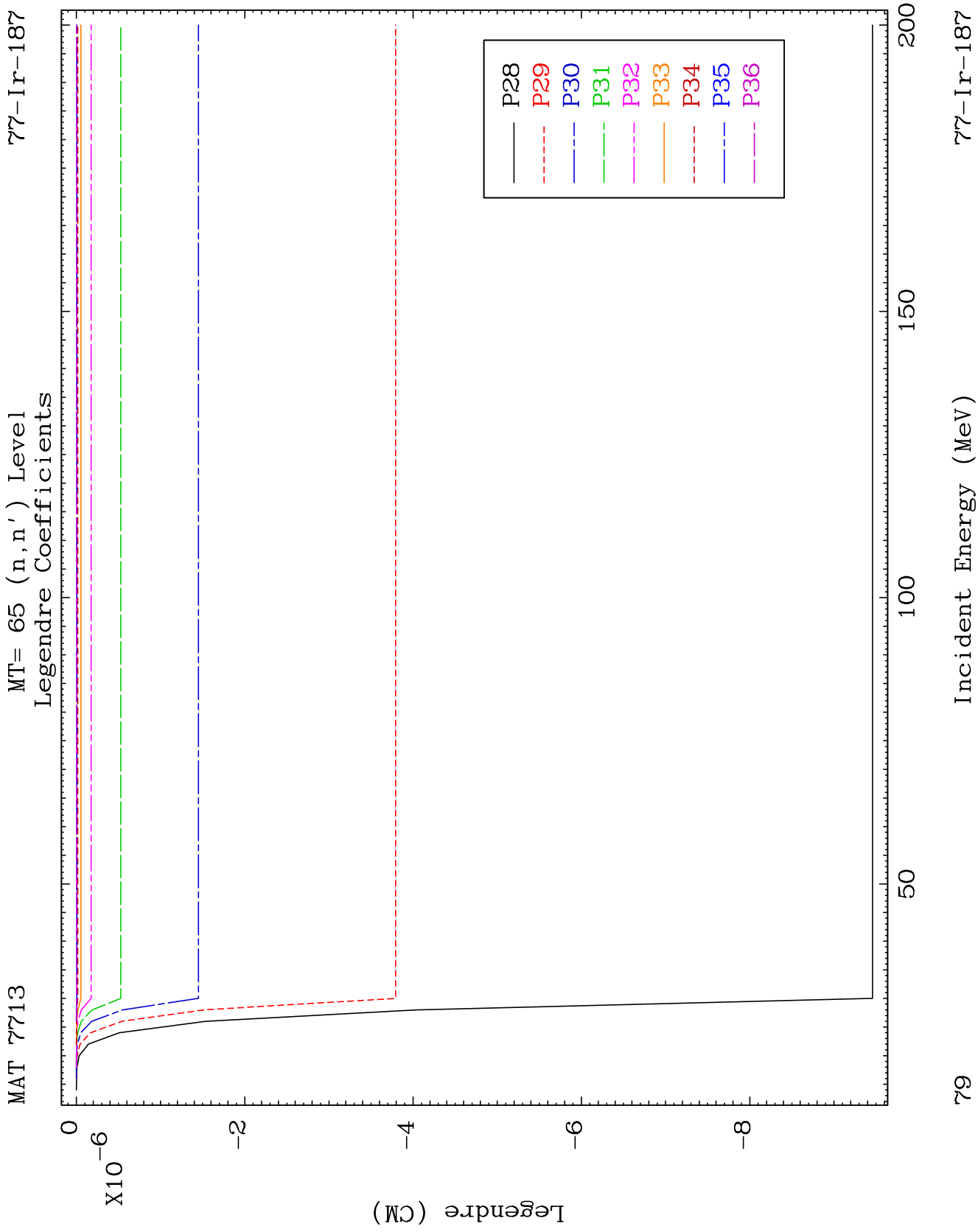
MAT 7713

MT= 65 (n, n') Level
Legendre Coefficients

22-Ir-187



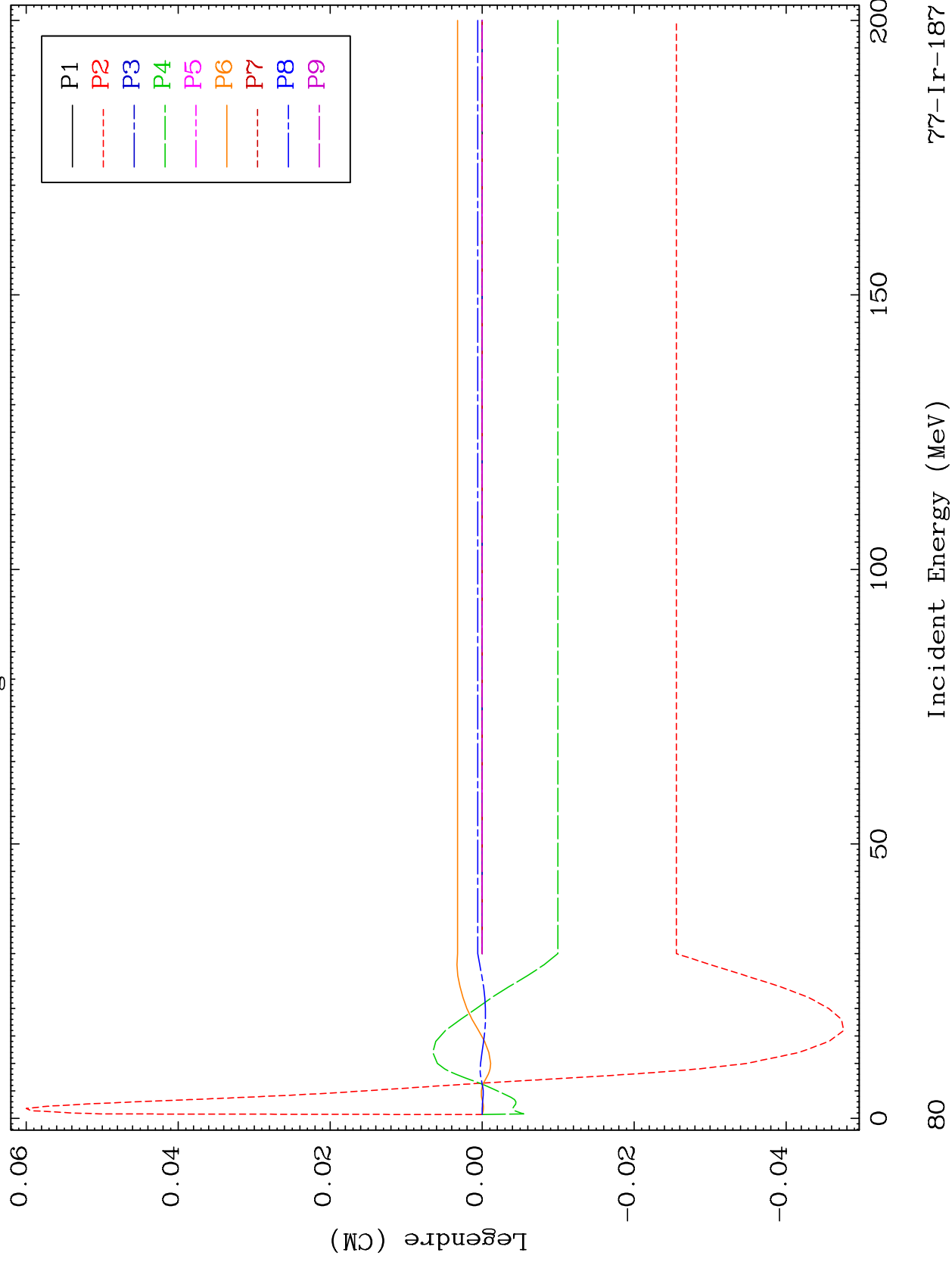


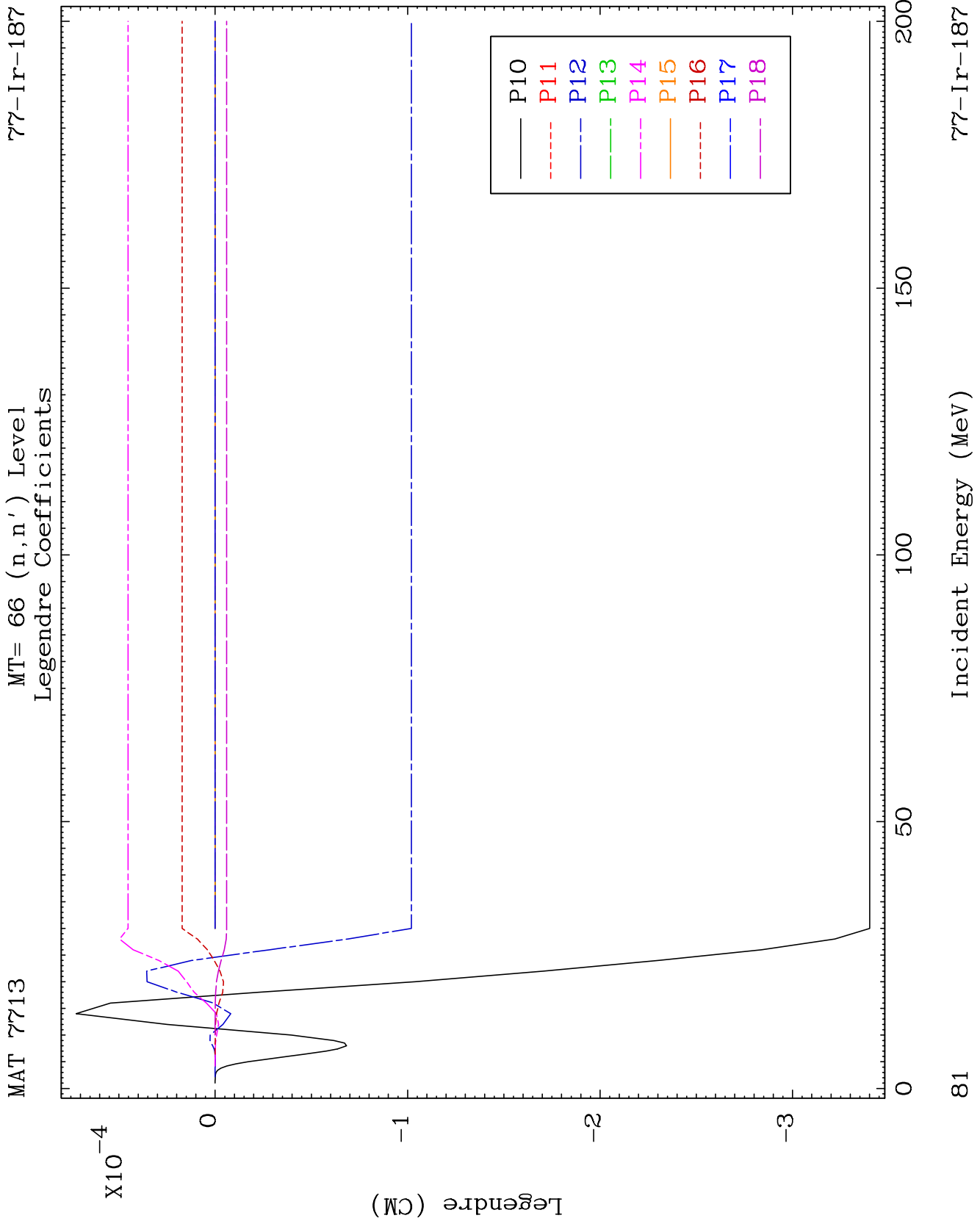


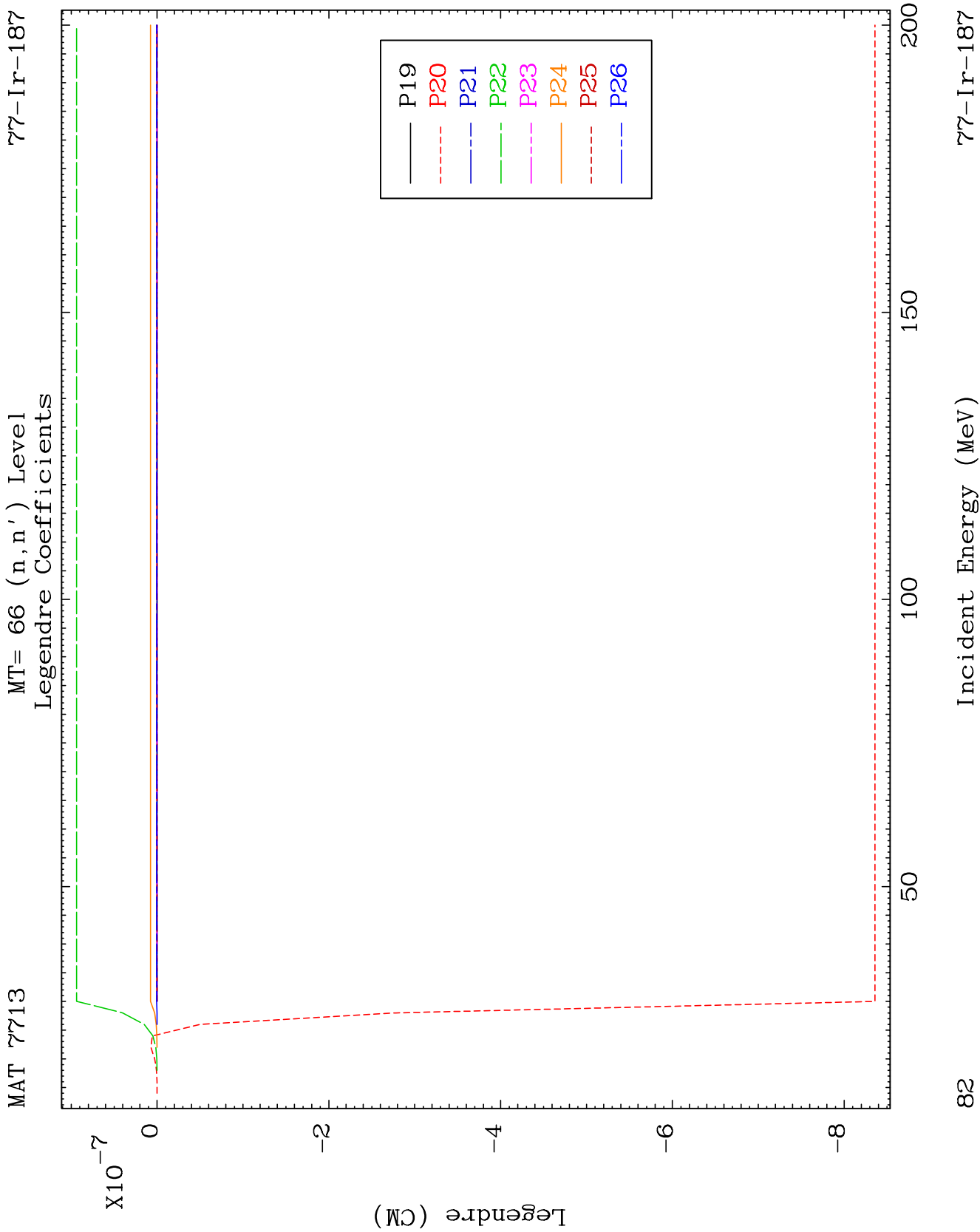
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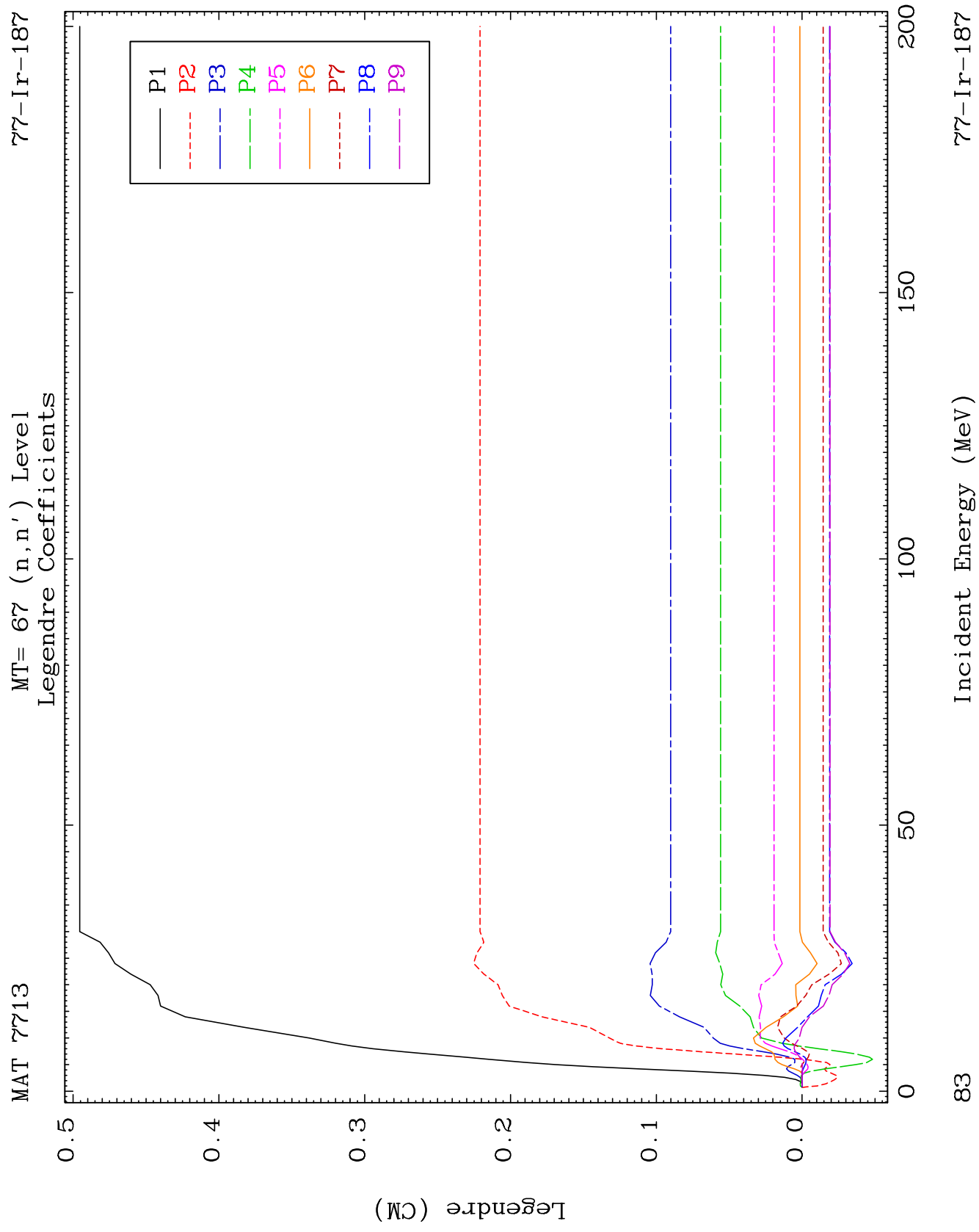
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Legendre Coefficients

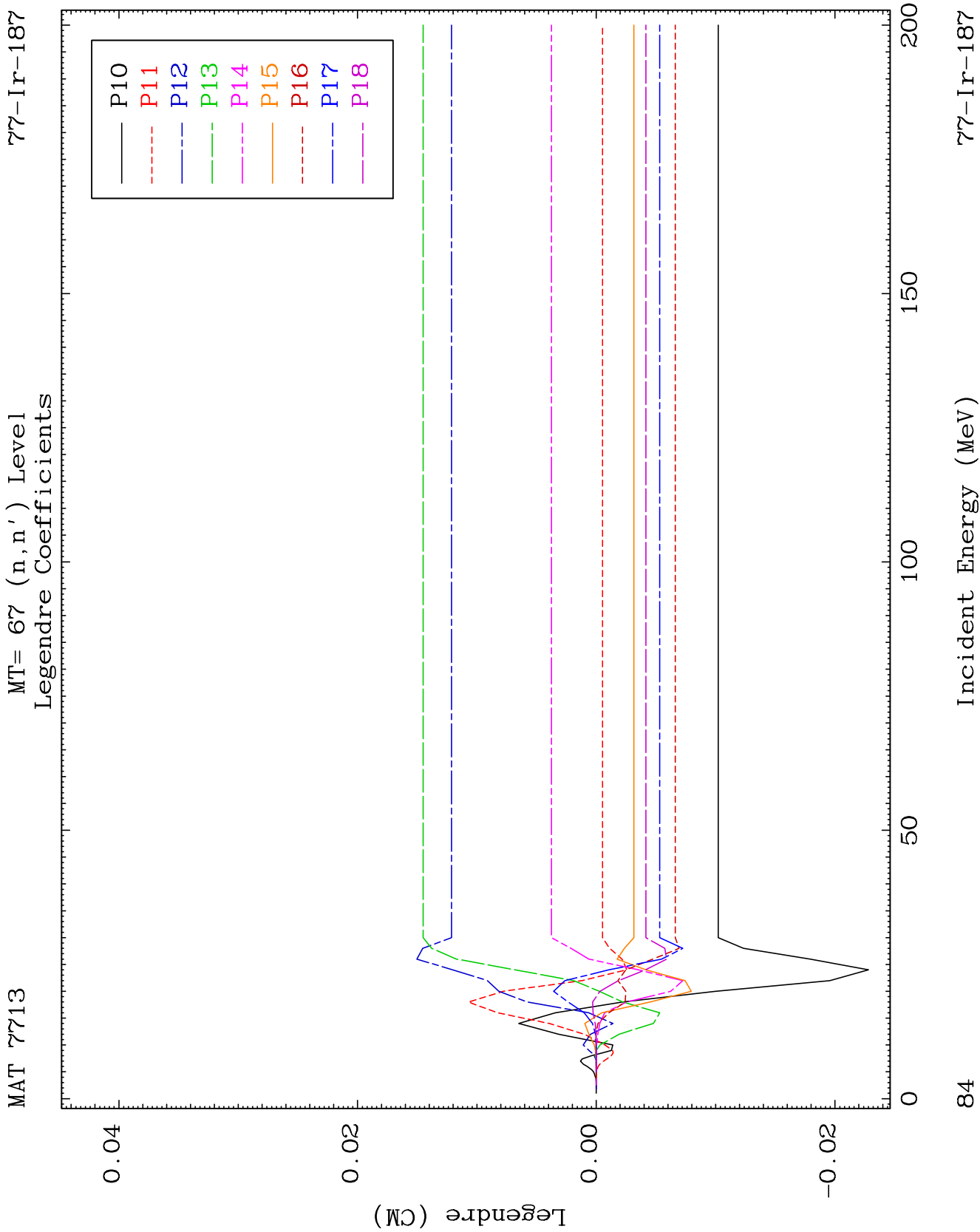
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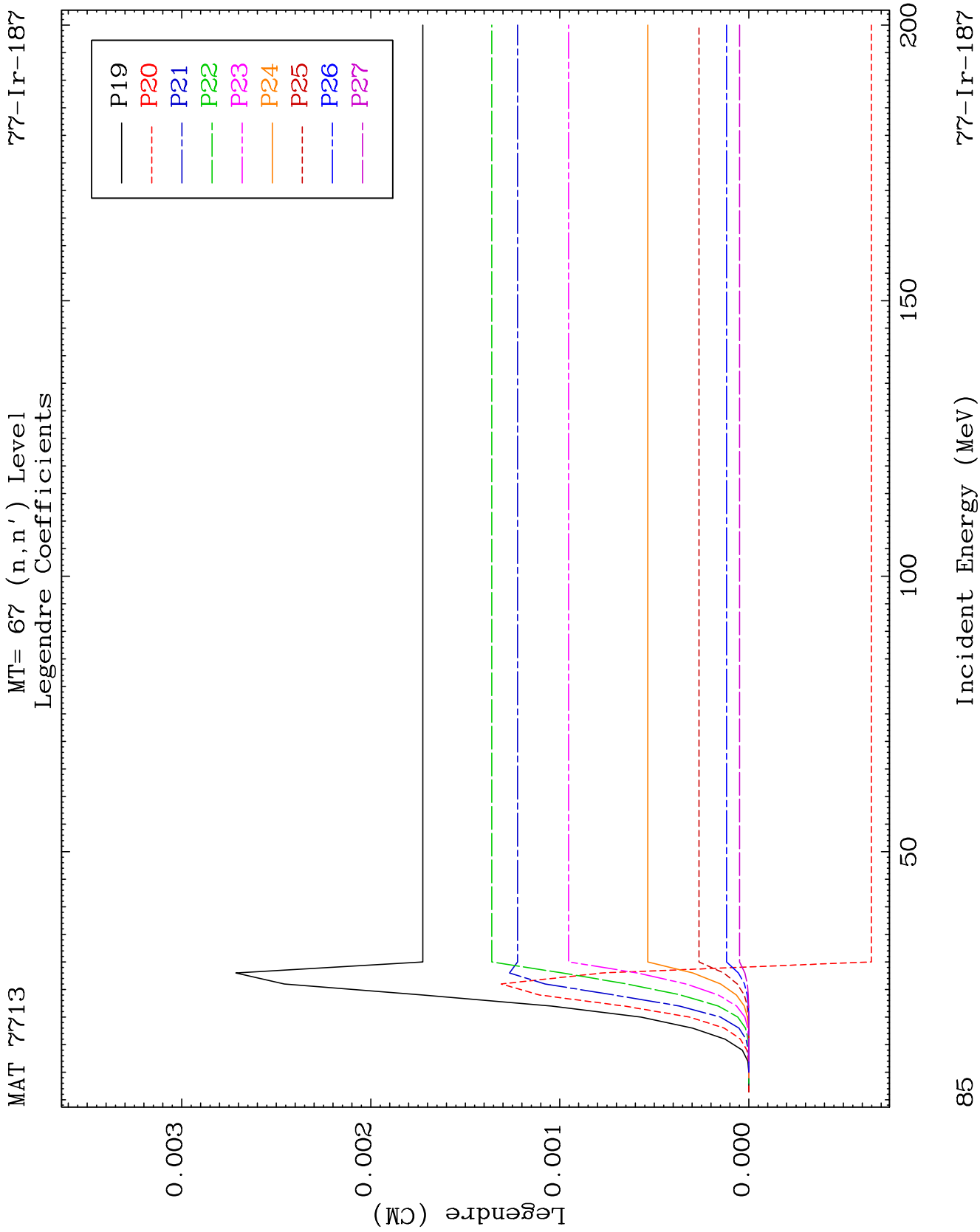


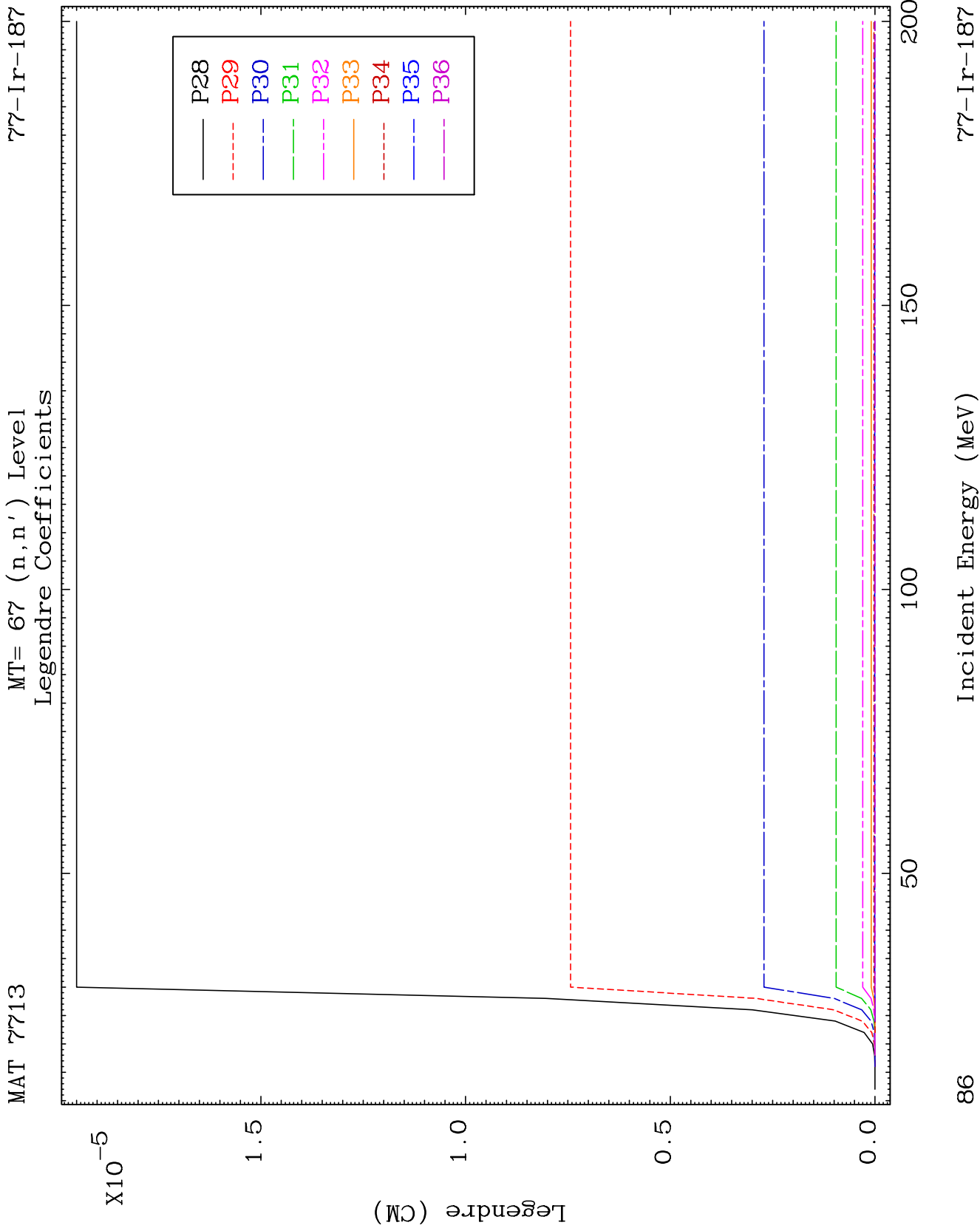


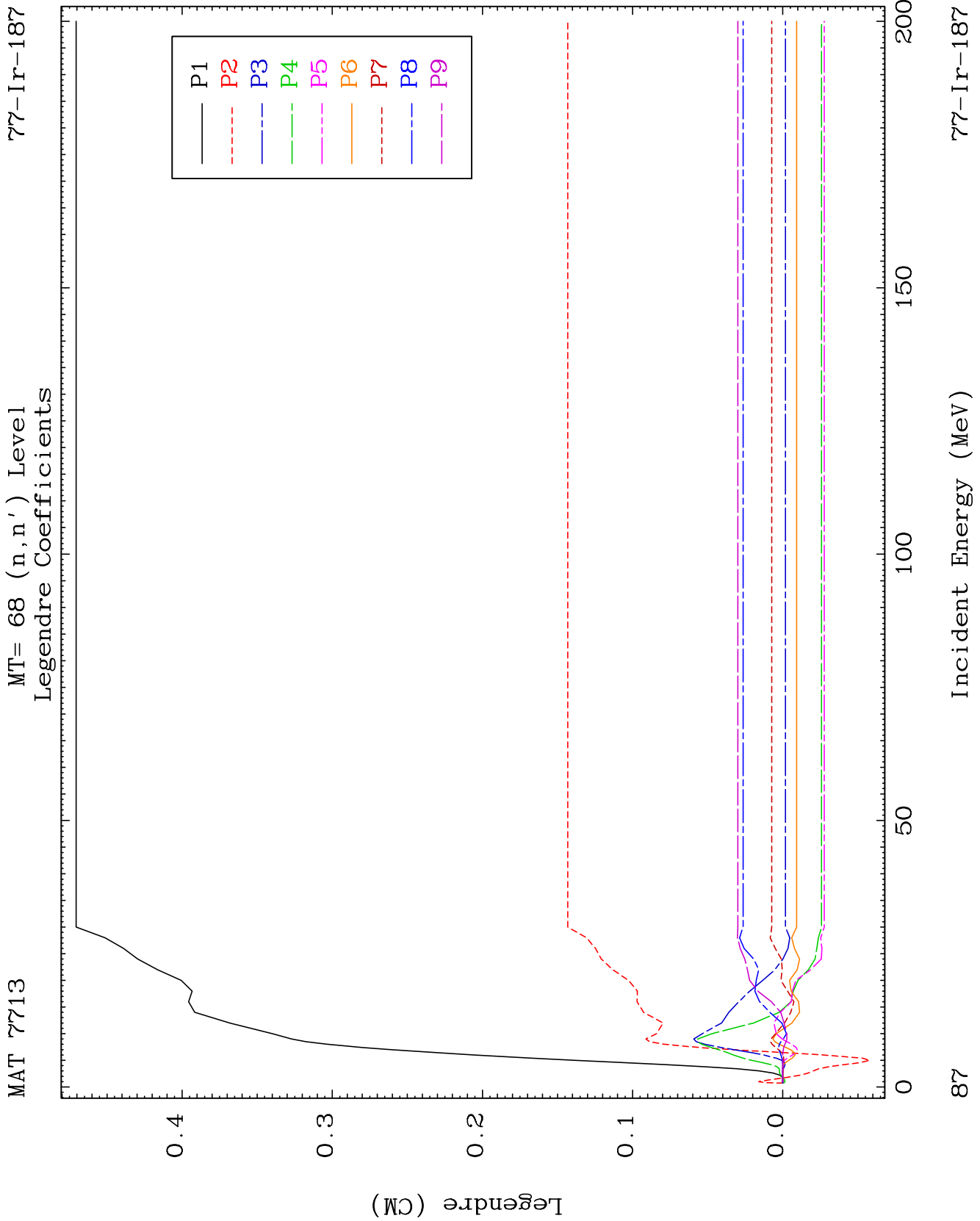


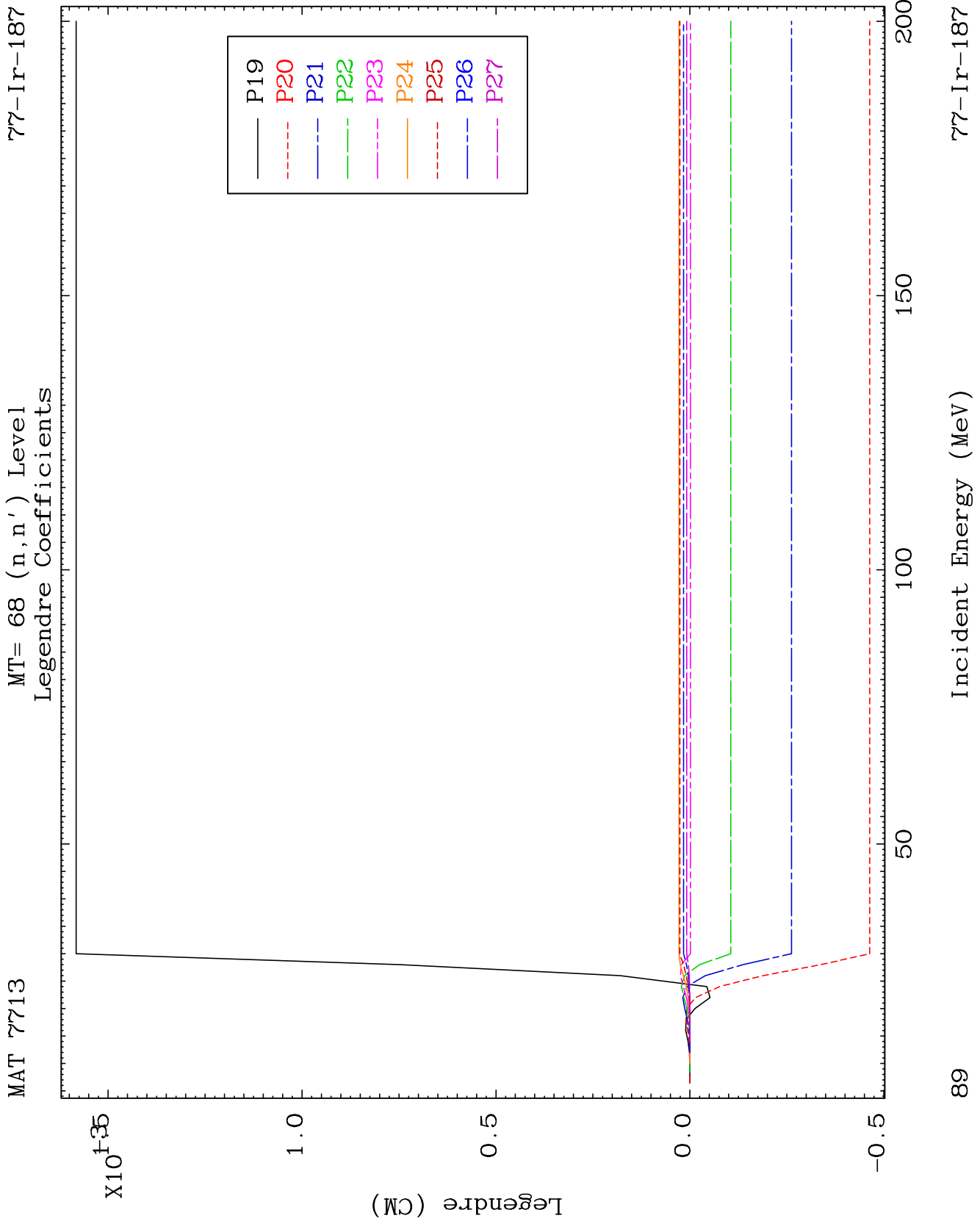


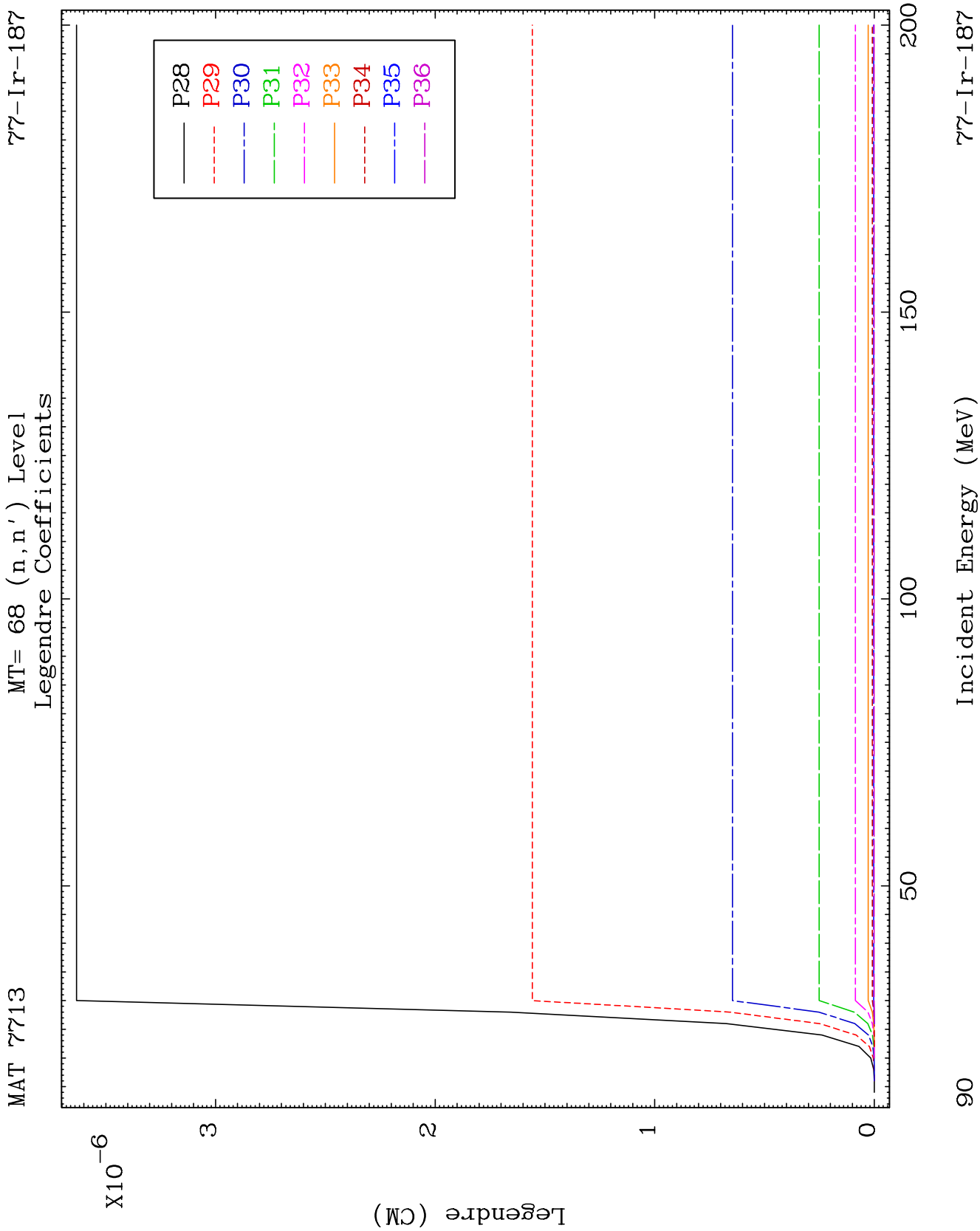








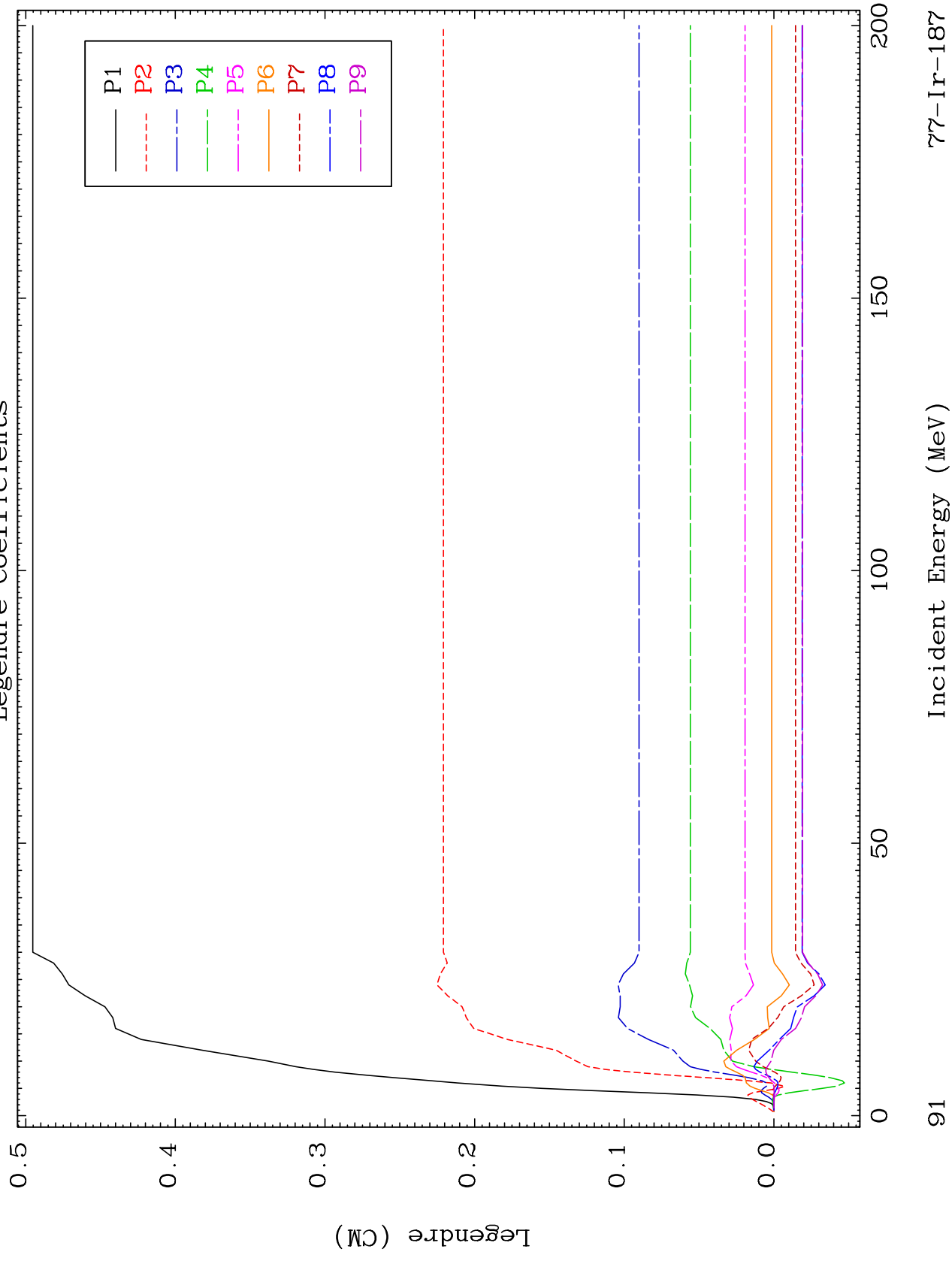




MAT 7713

MT= 69 (n, n') Level
Legendre Coefficients

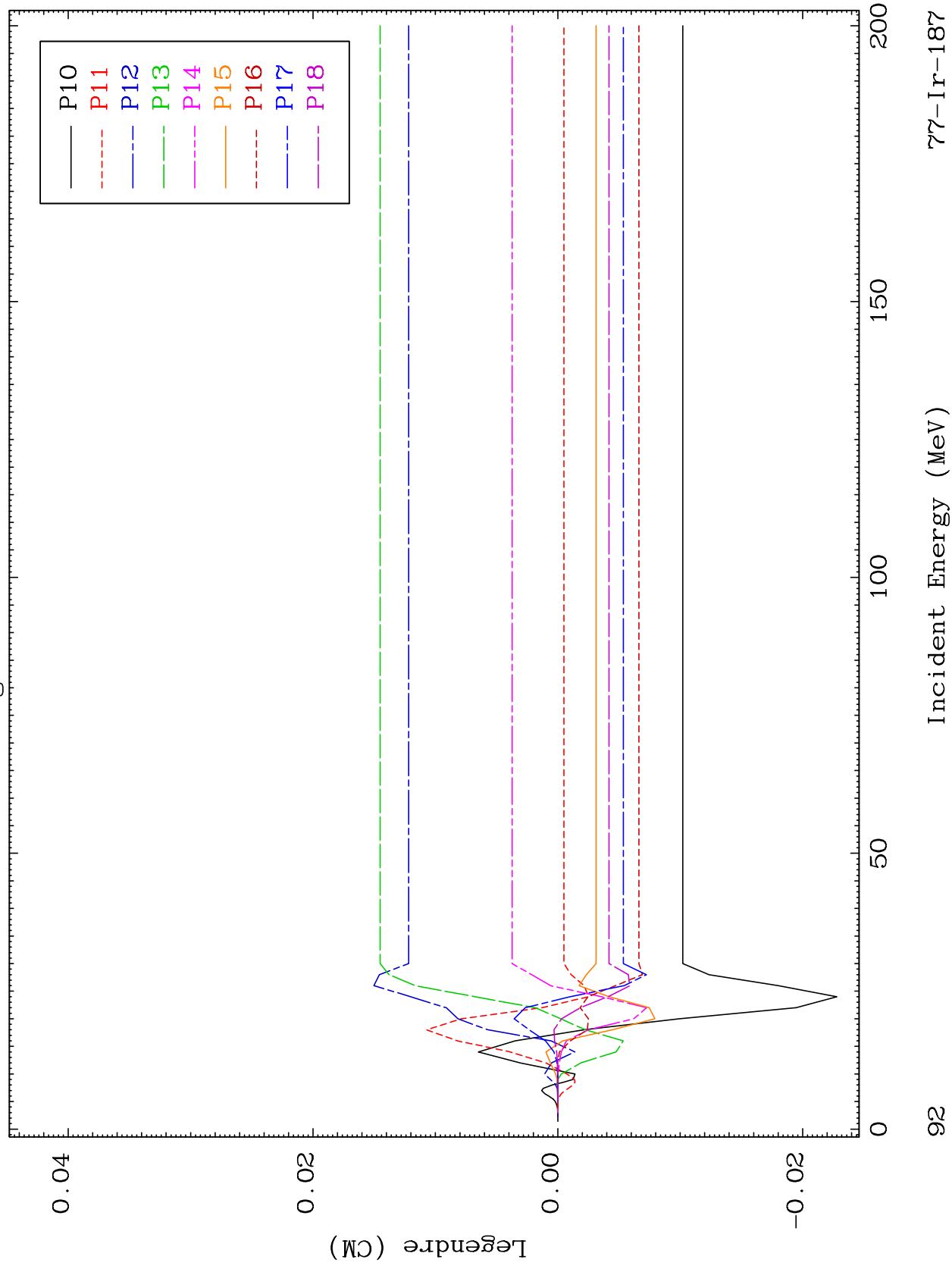
22-Ir-187

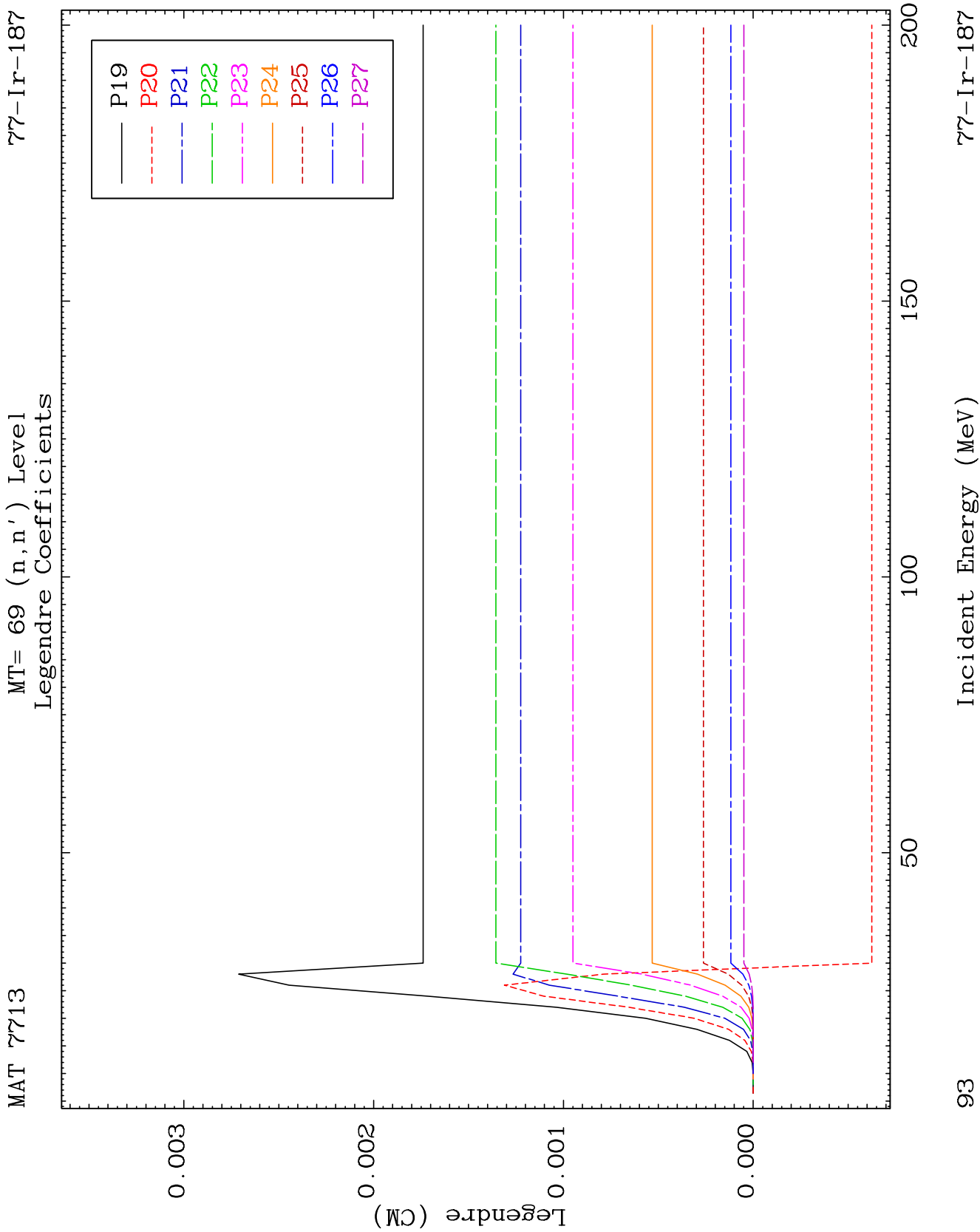


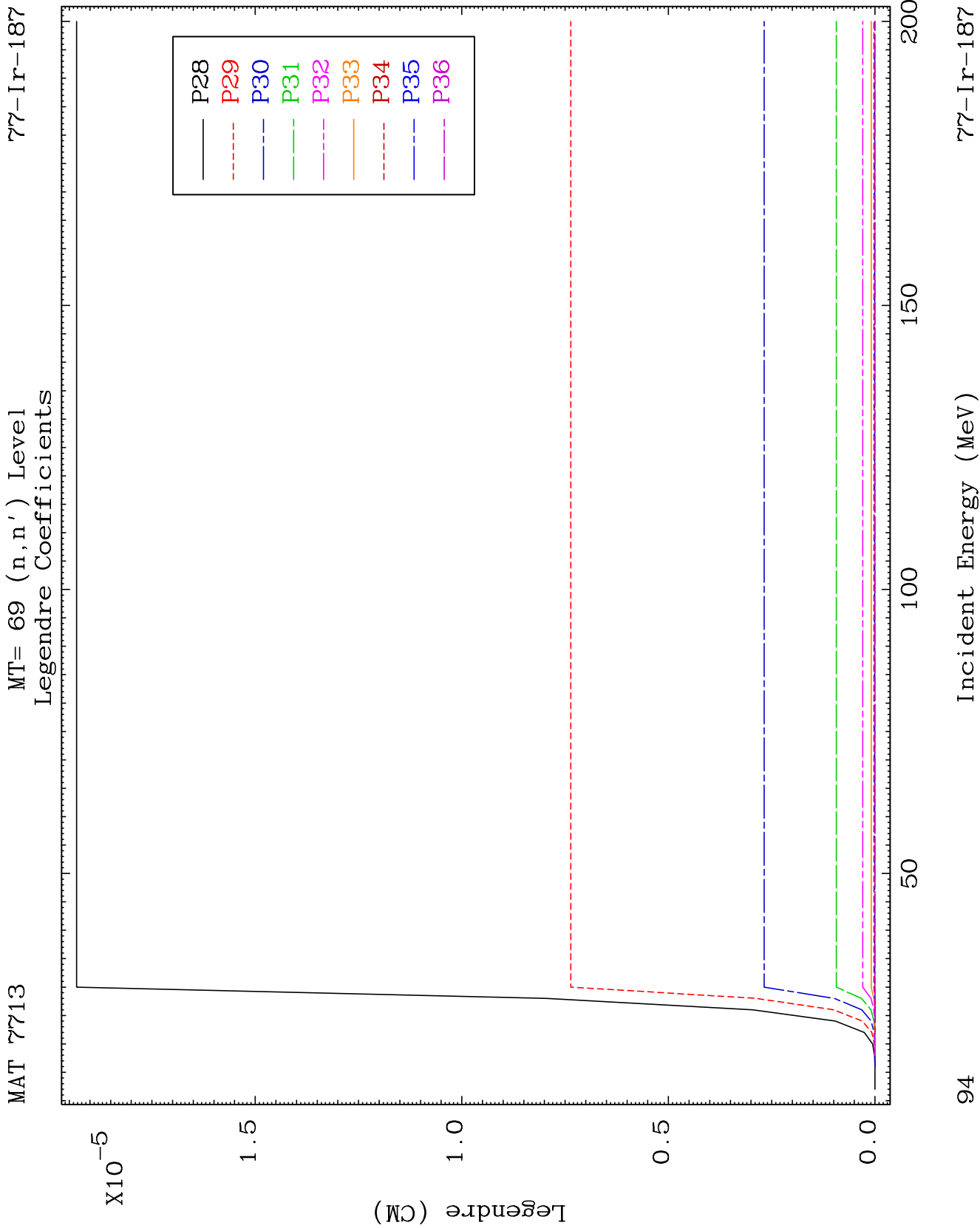
MAT 7713

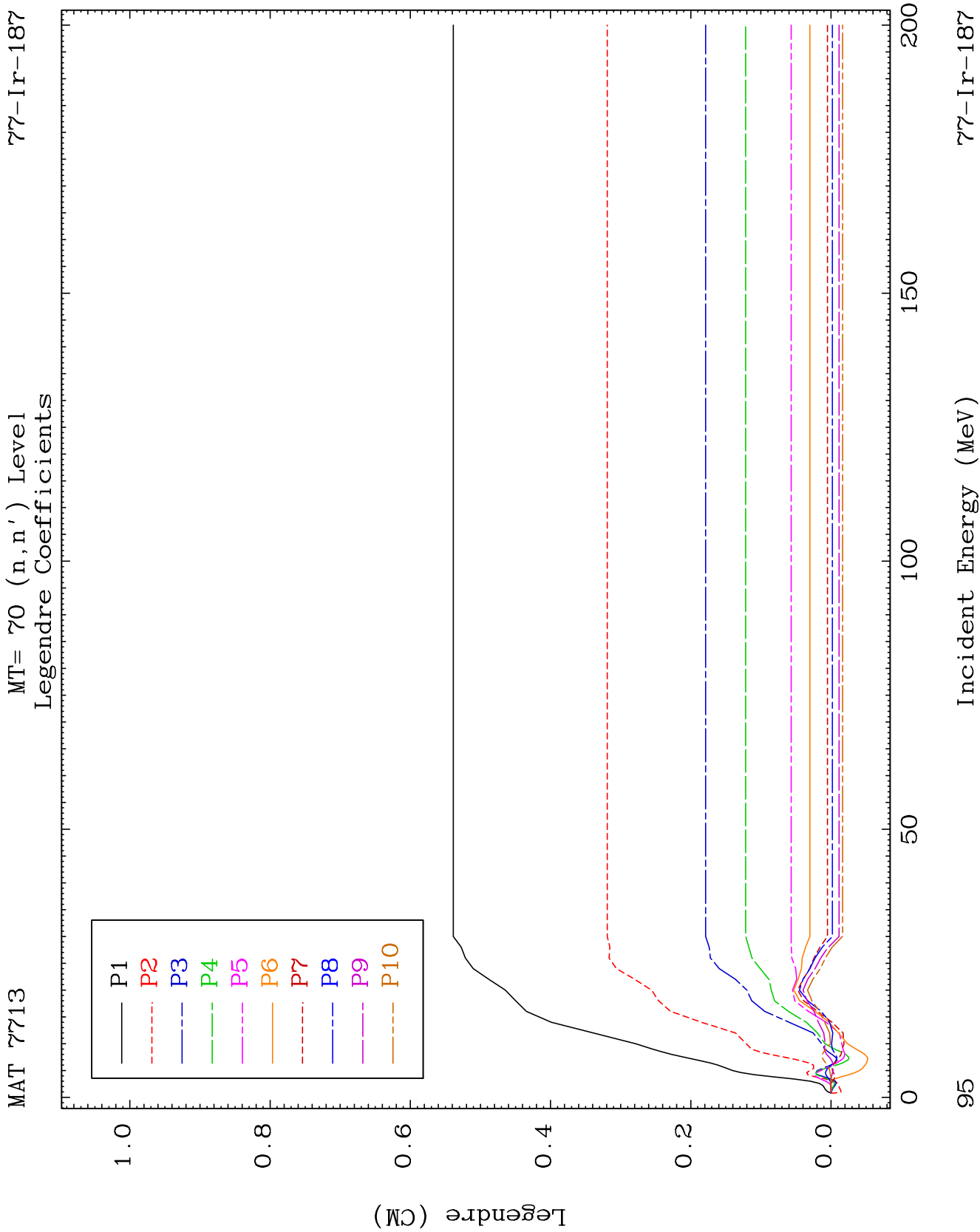
MT= 69 (n, n') Level
Legendre Coefficients

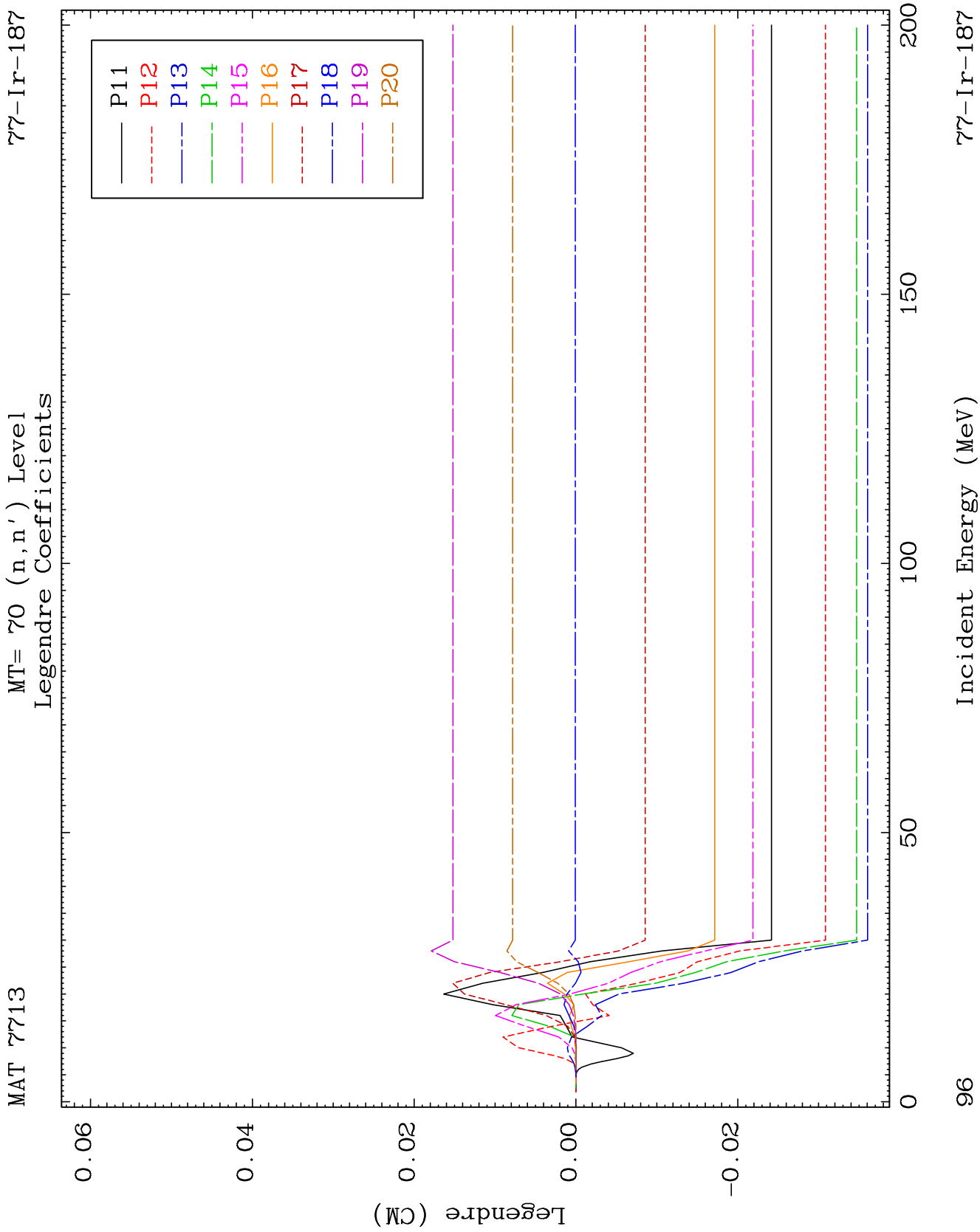
77-1r-187

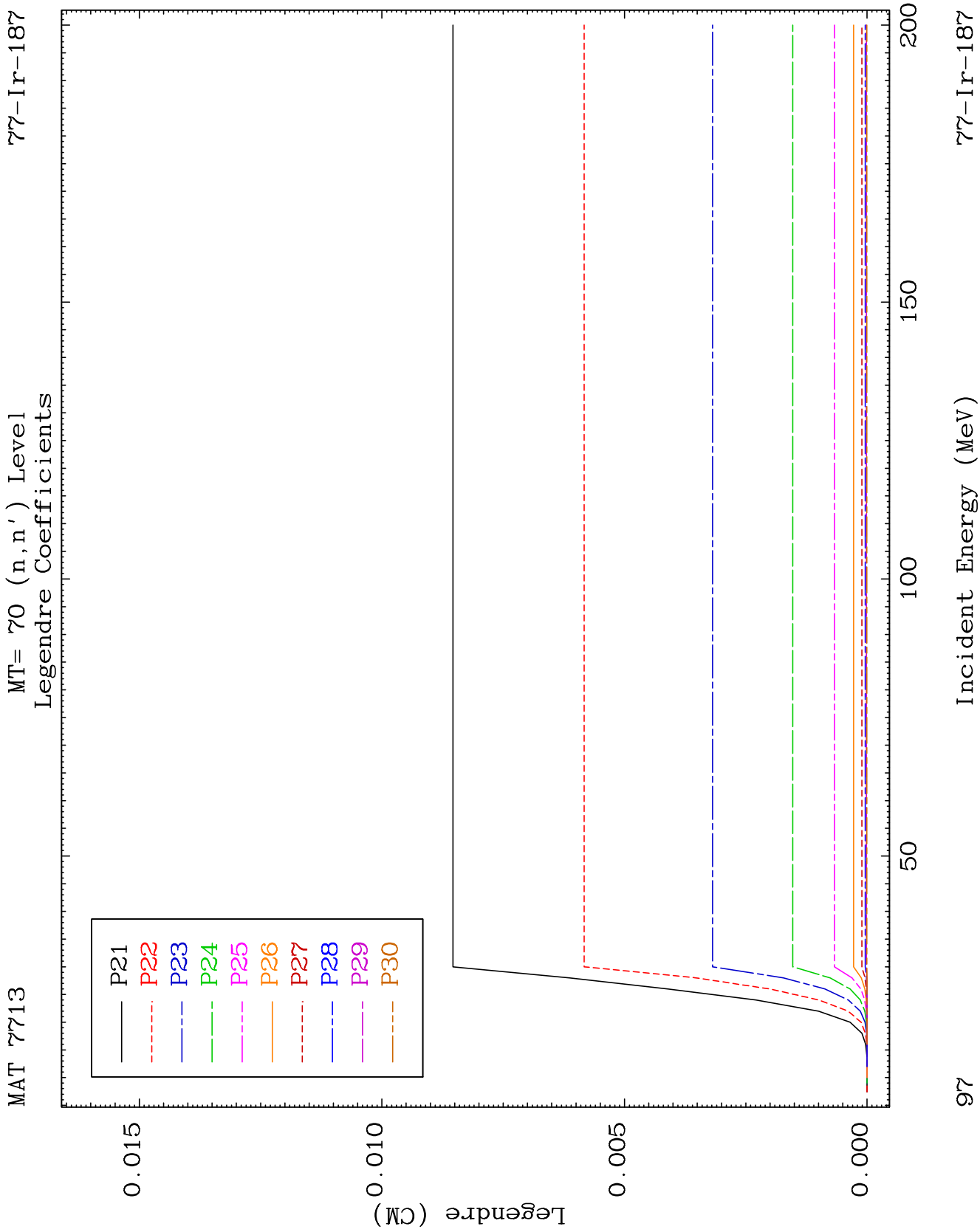


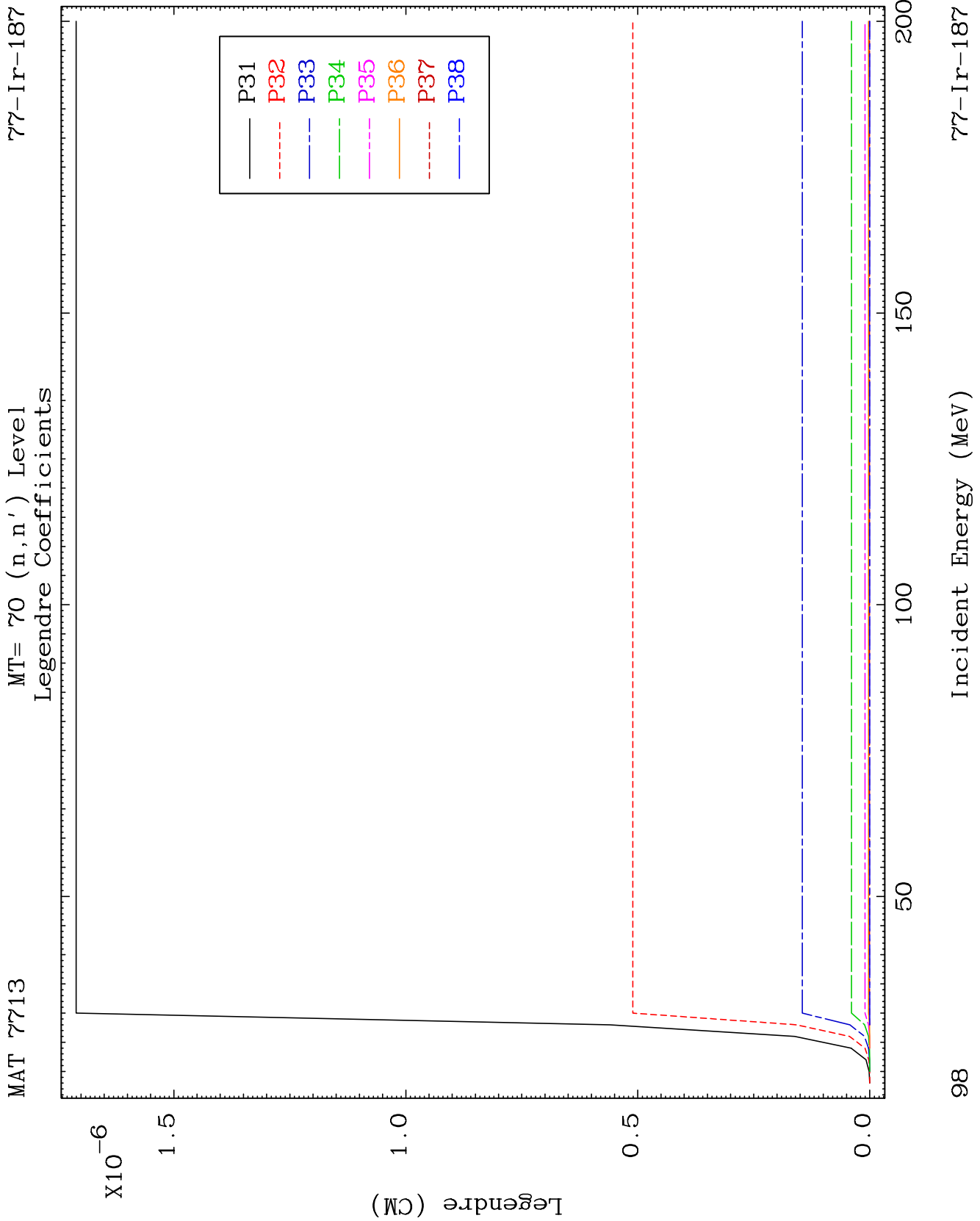


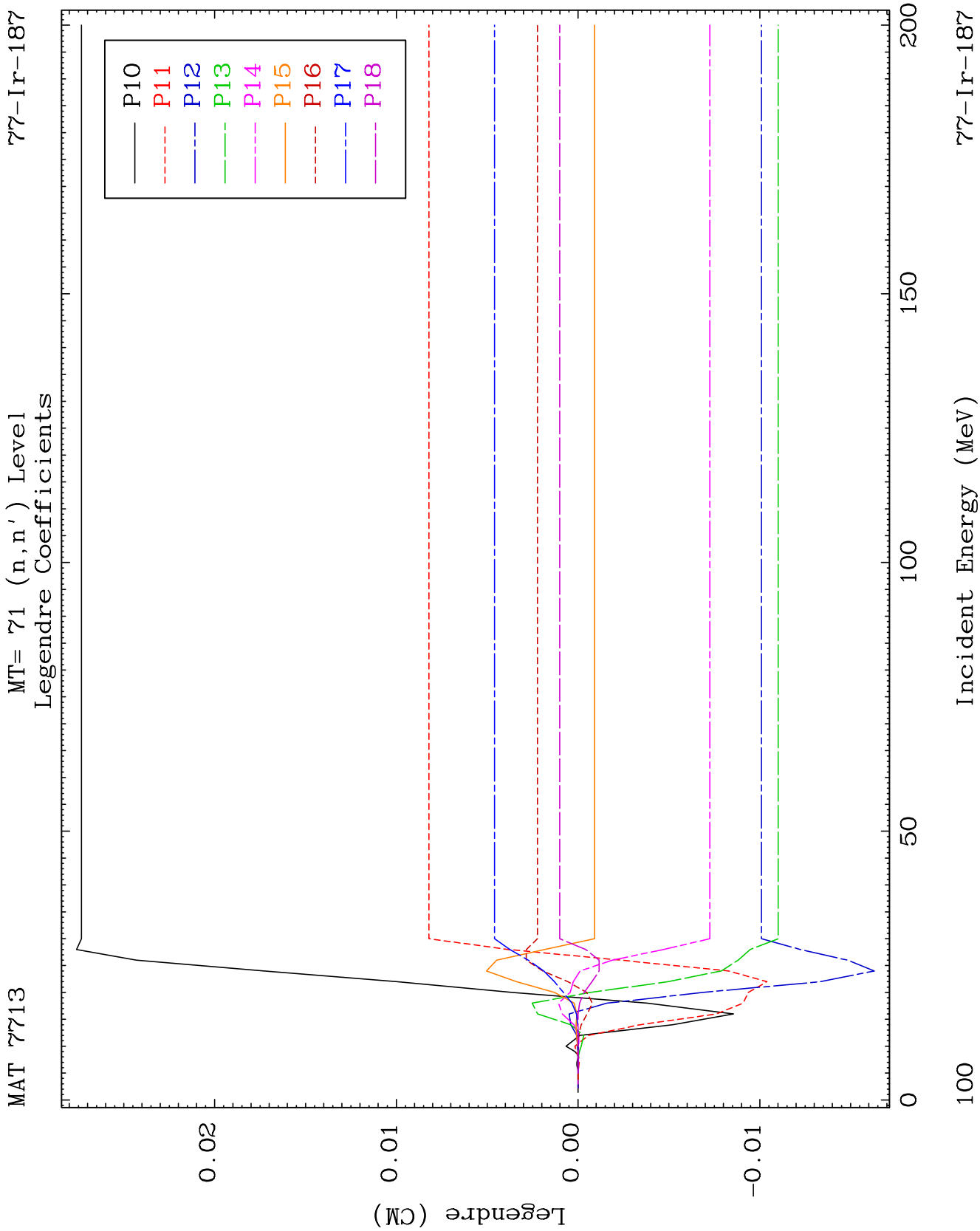


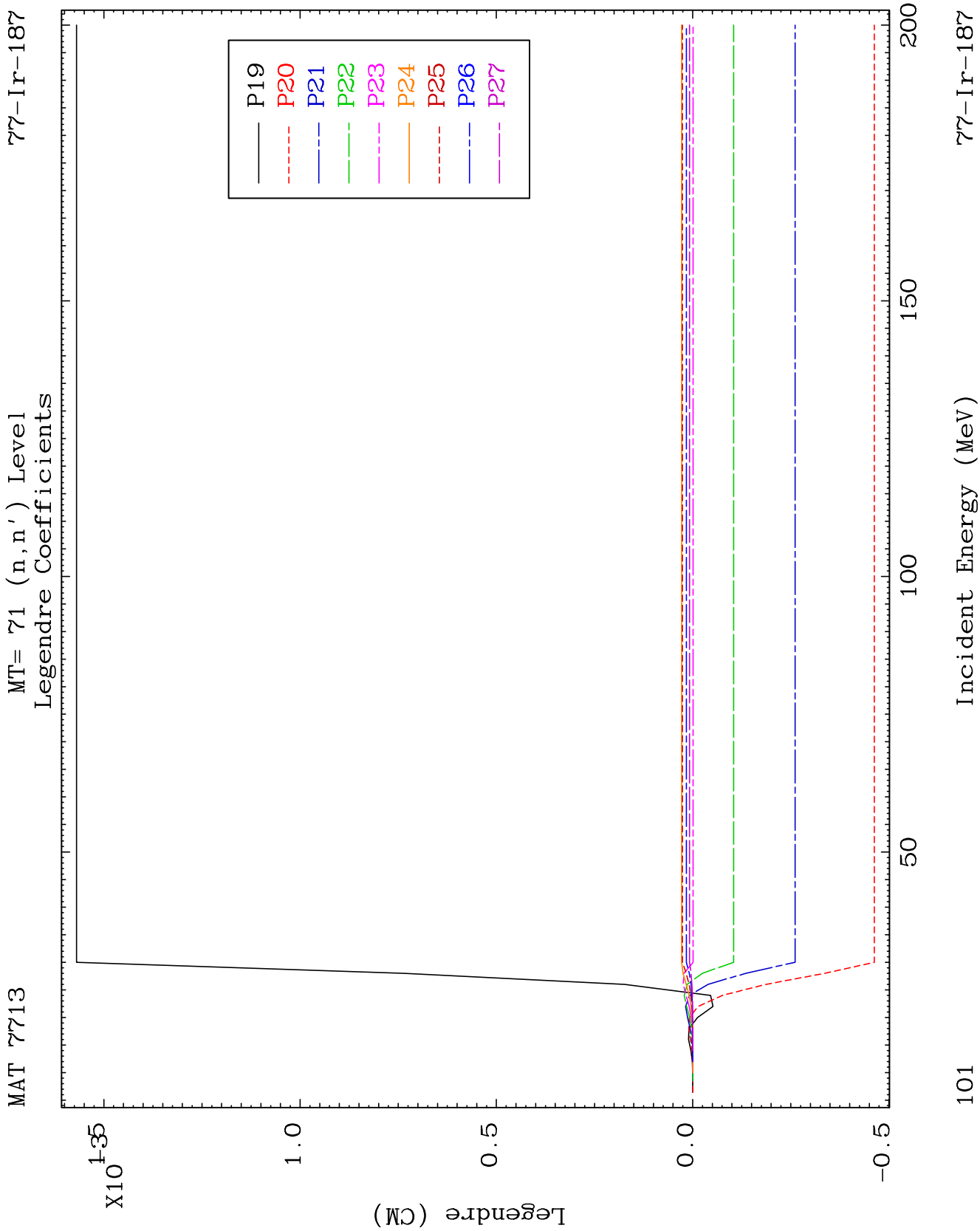


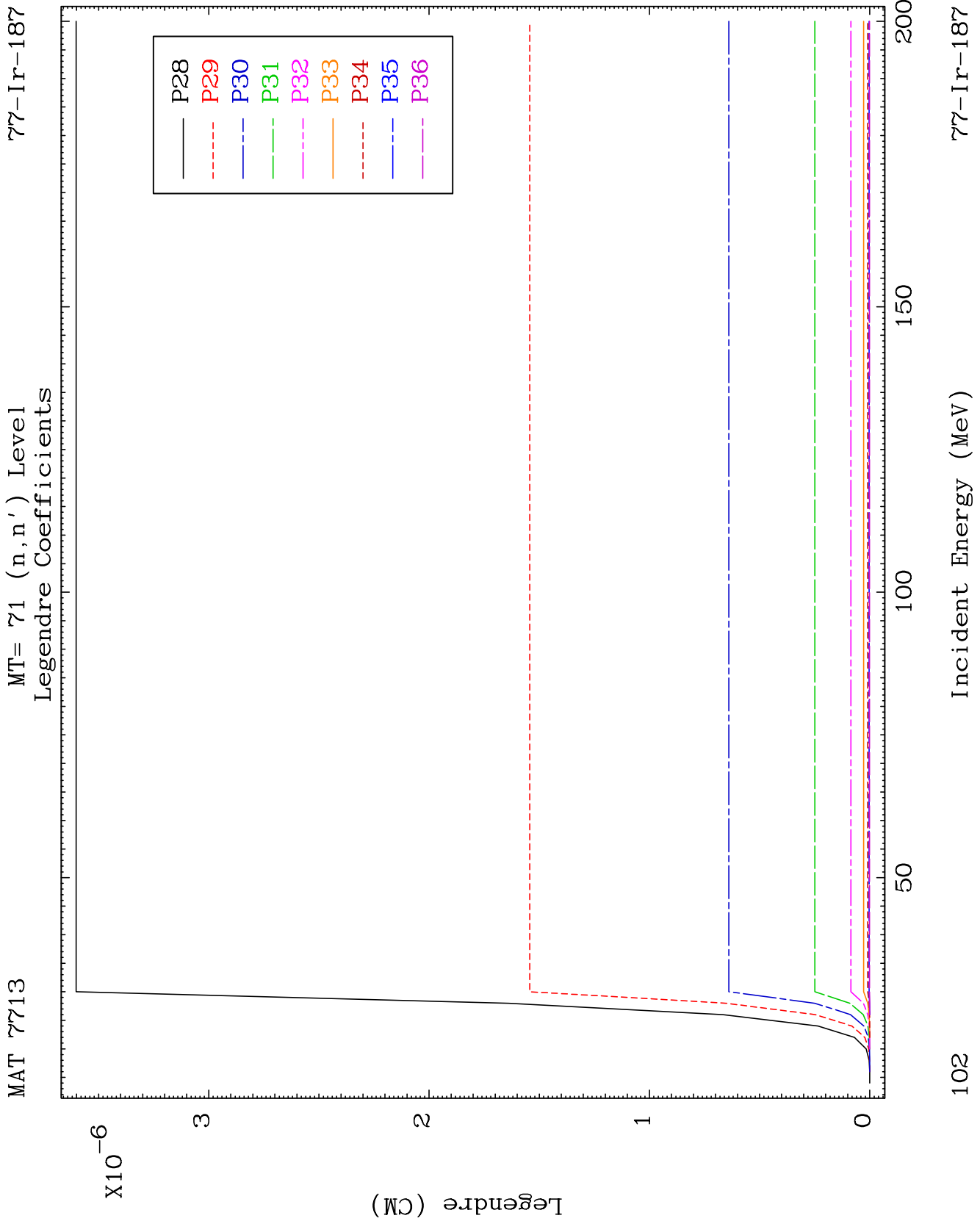








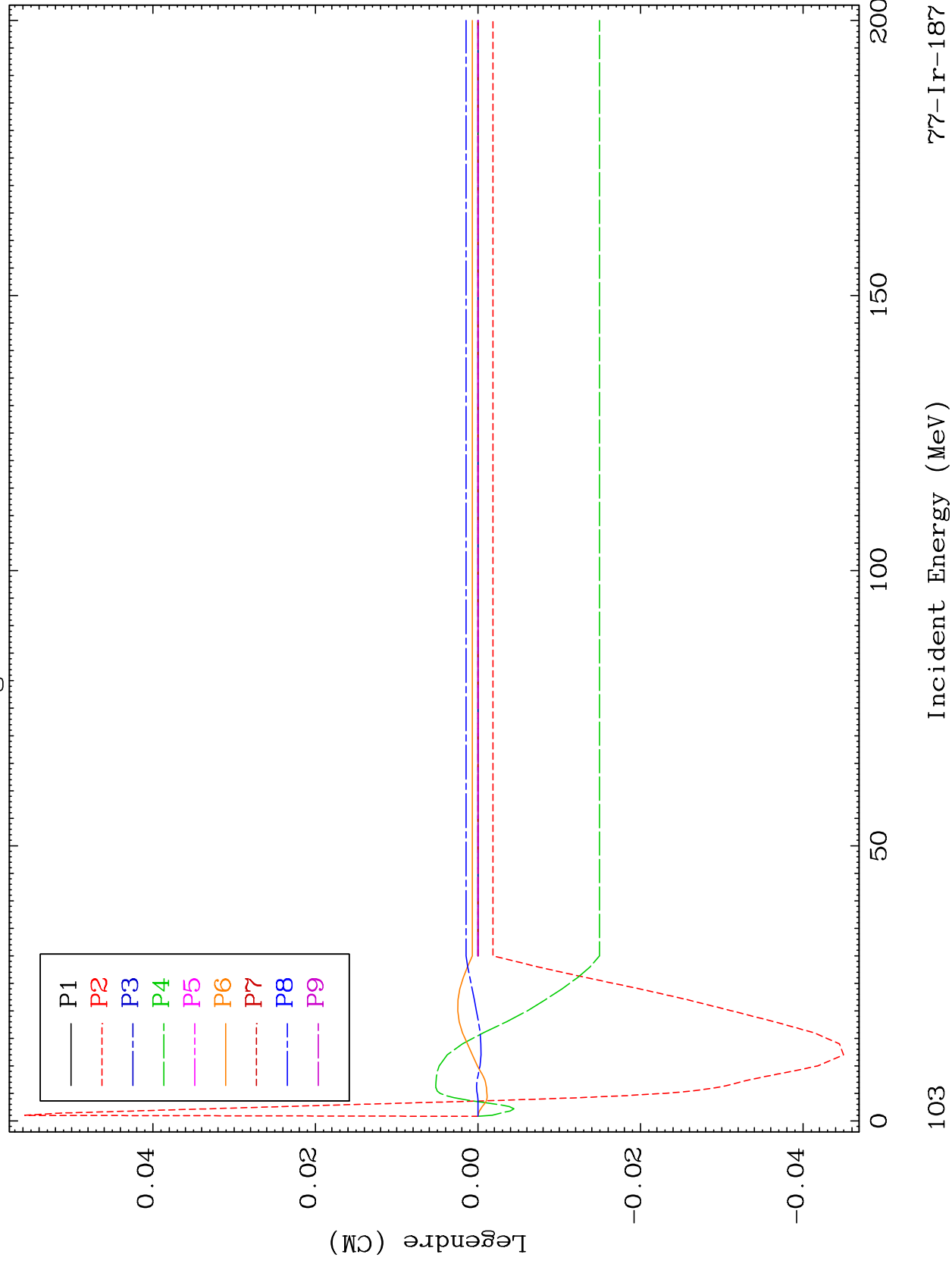


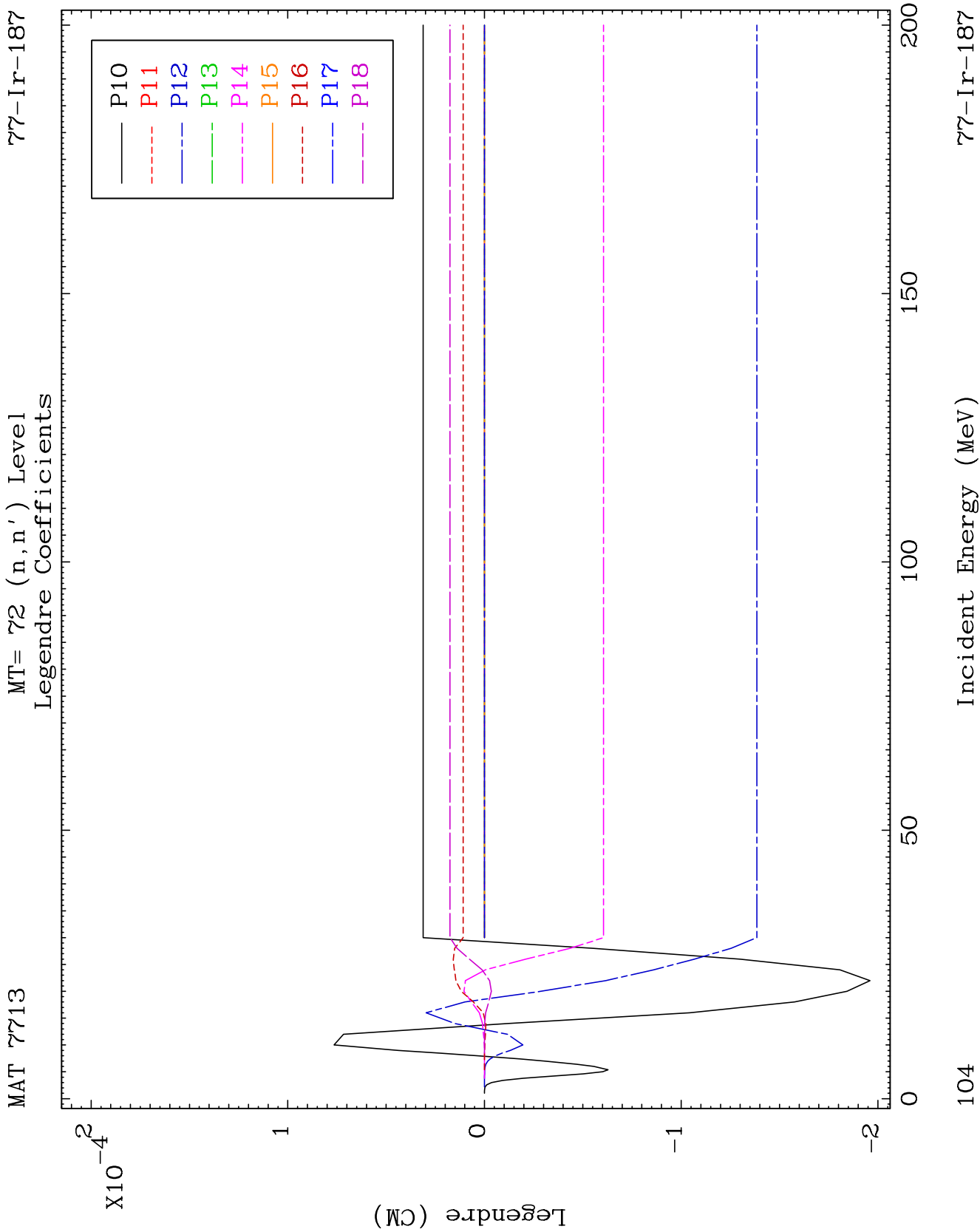


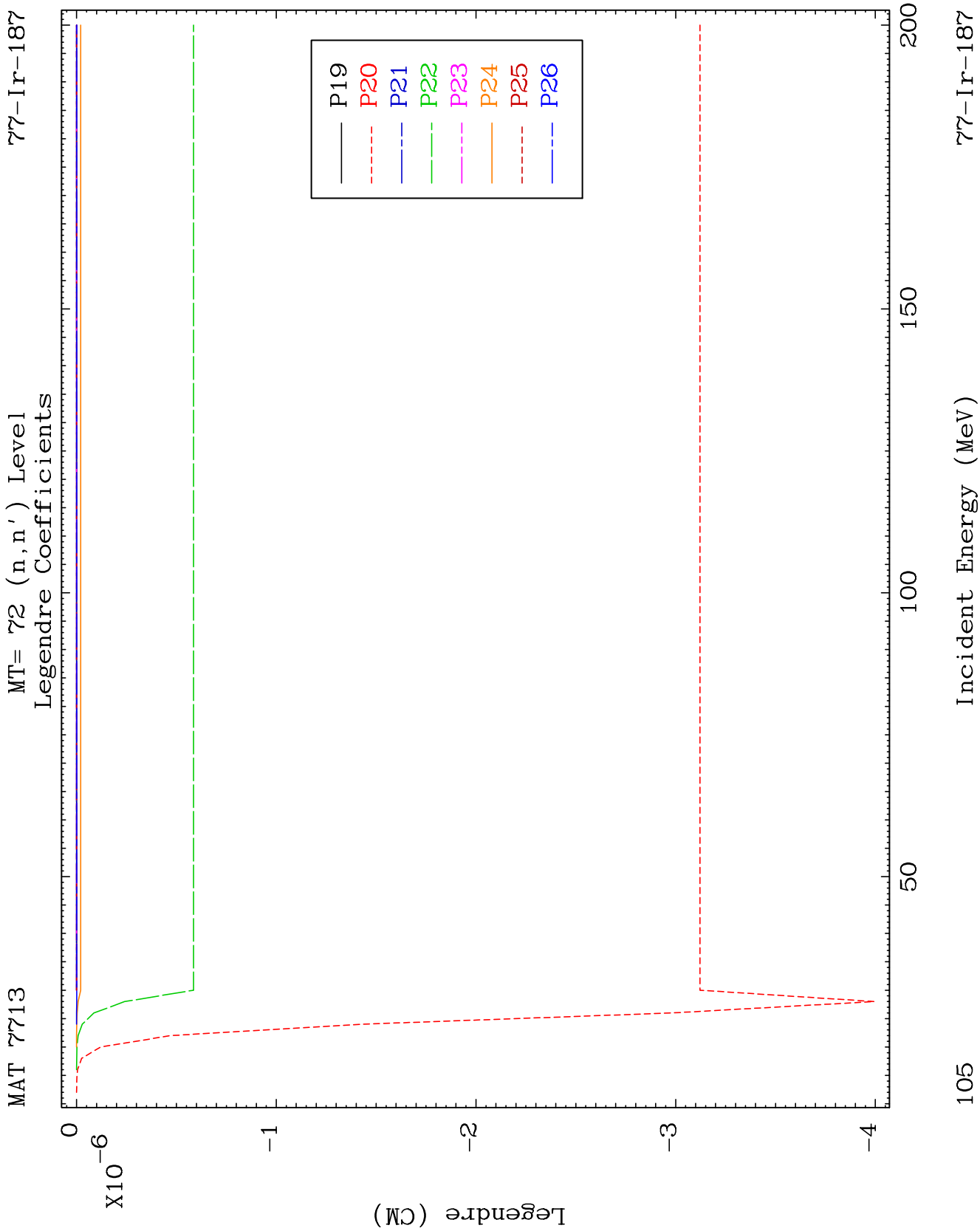
MAT 7713

MT= 72 (n, n') Level
Legendre Coefficients

22-Ir-187



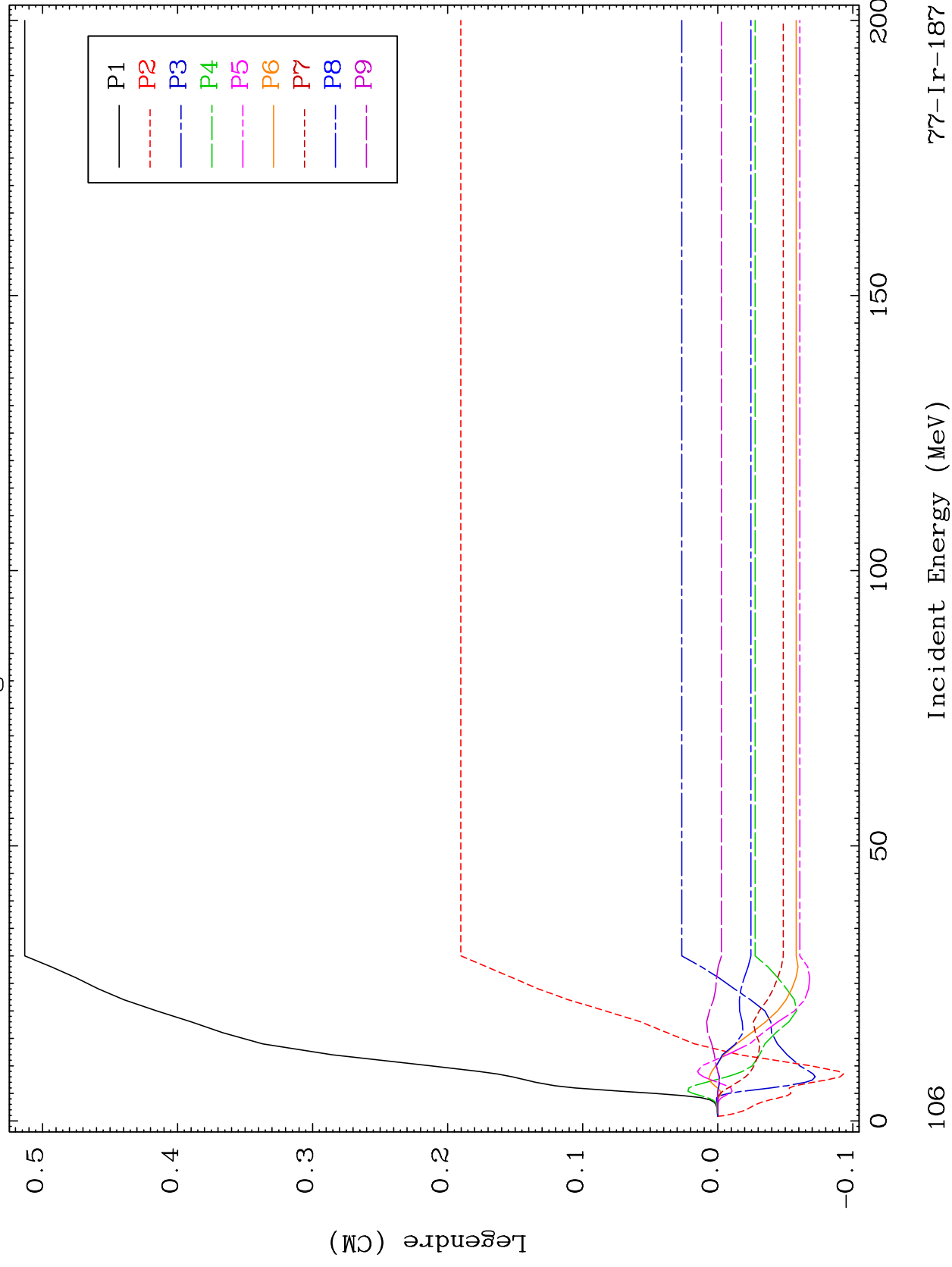




MAT 7713

MT= 73 (n, n') Level
Legendre Coefficients

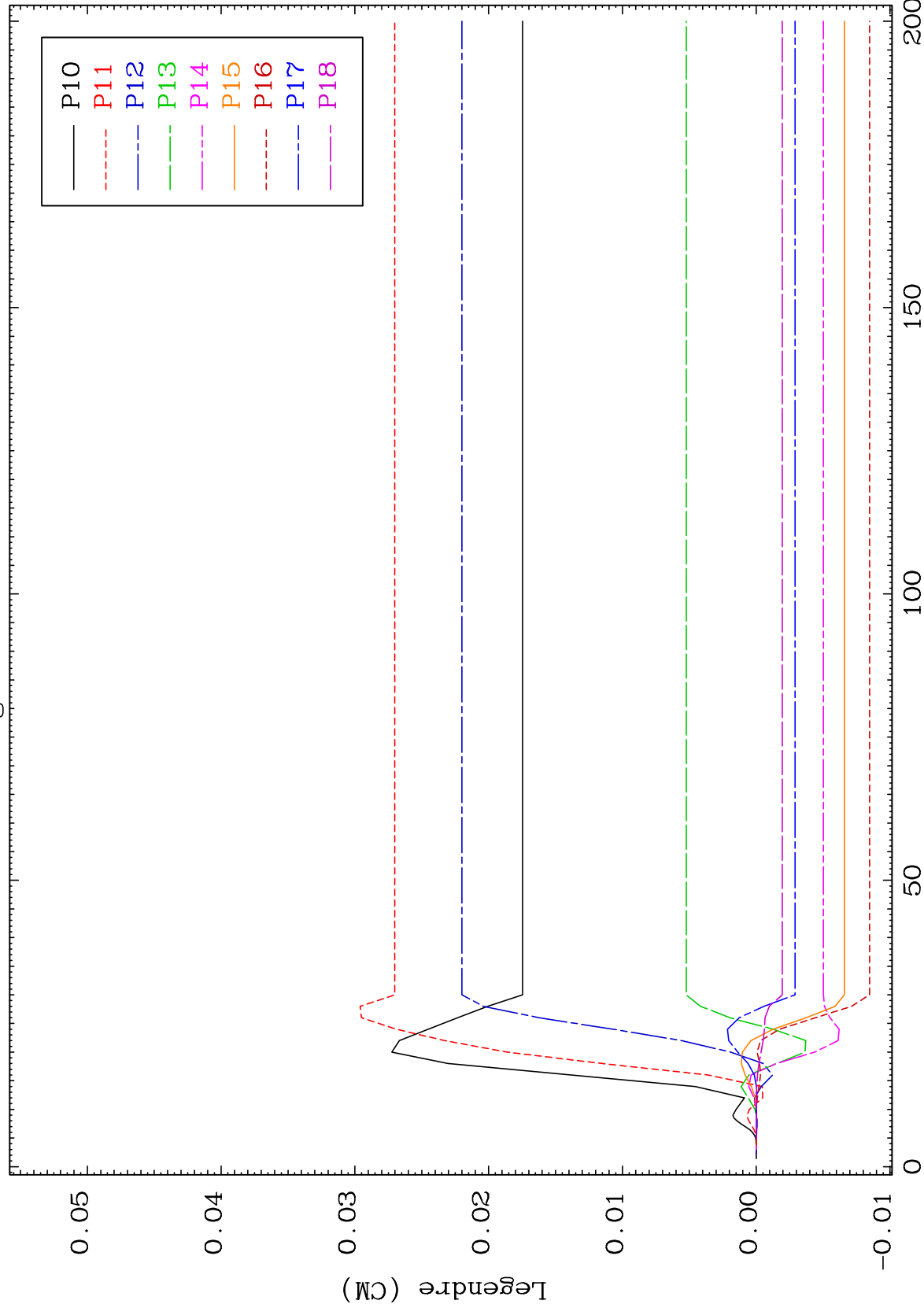
22-Ir-187



MAT 7713

MT= 73 (n, n') Level
Legendre Coefficients

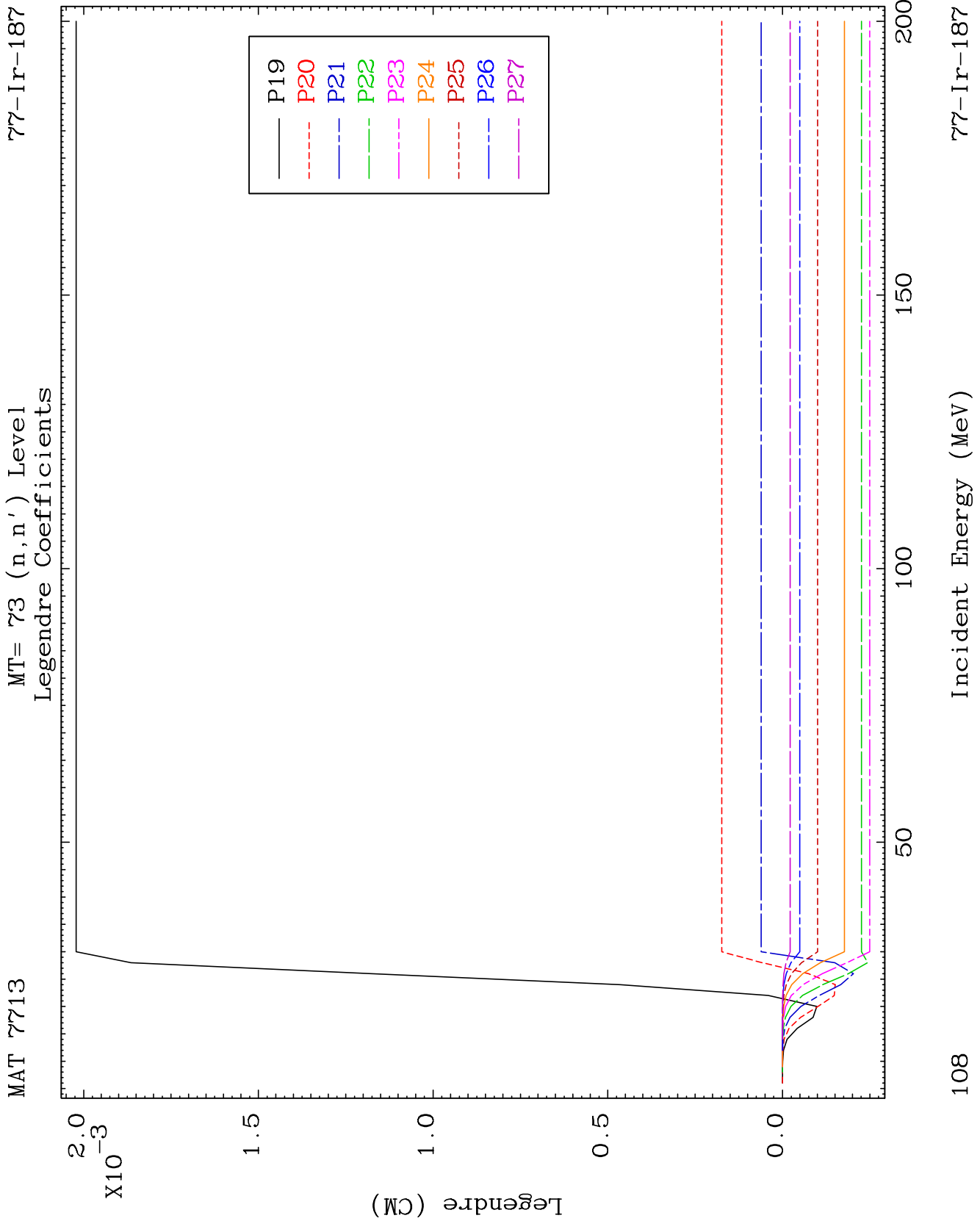
27-1r-187

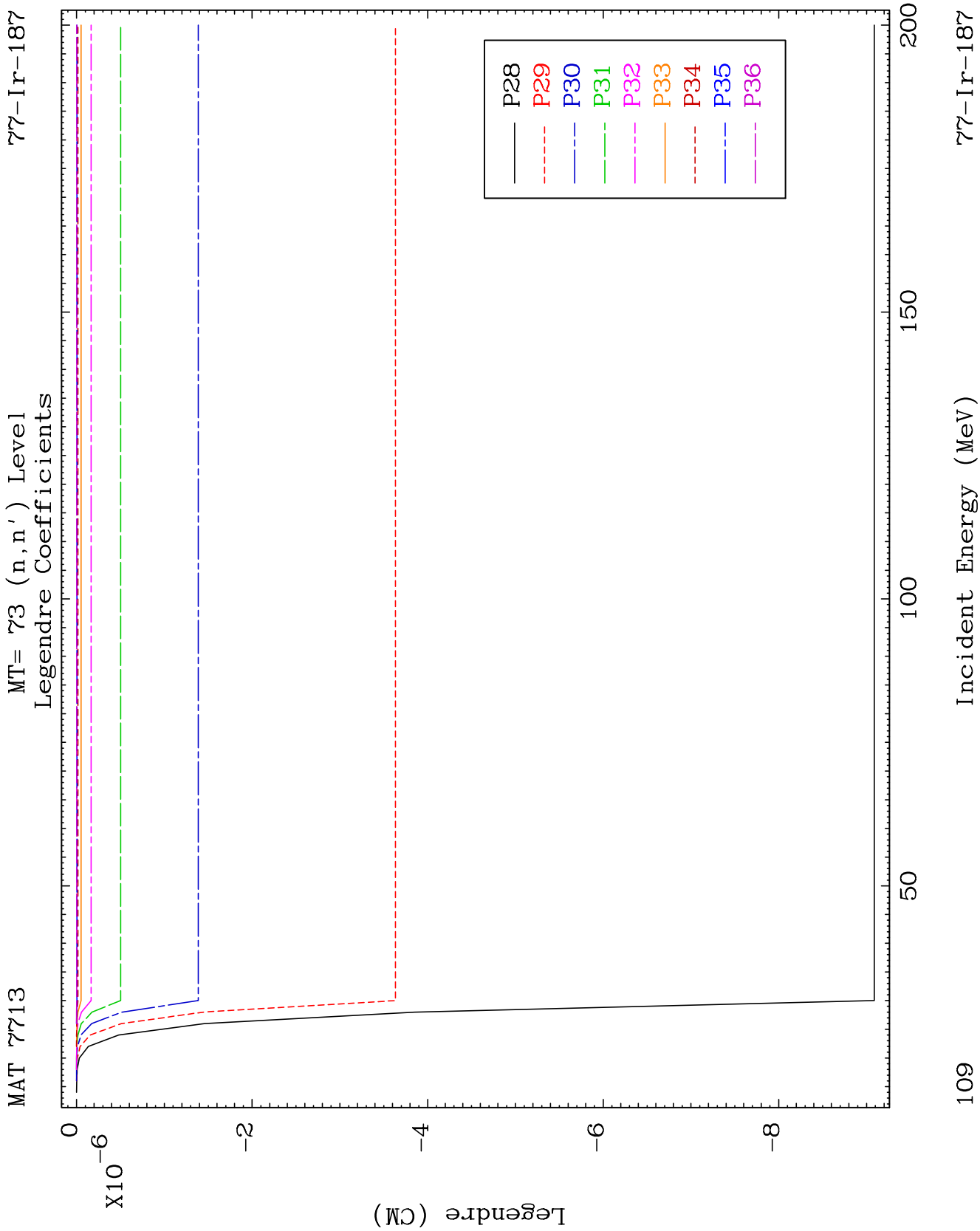


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Incident Energy (MeV)

27-1r-187

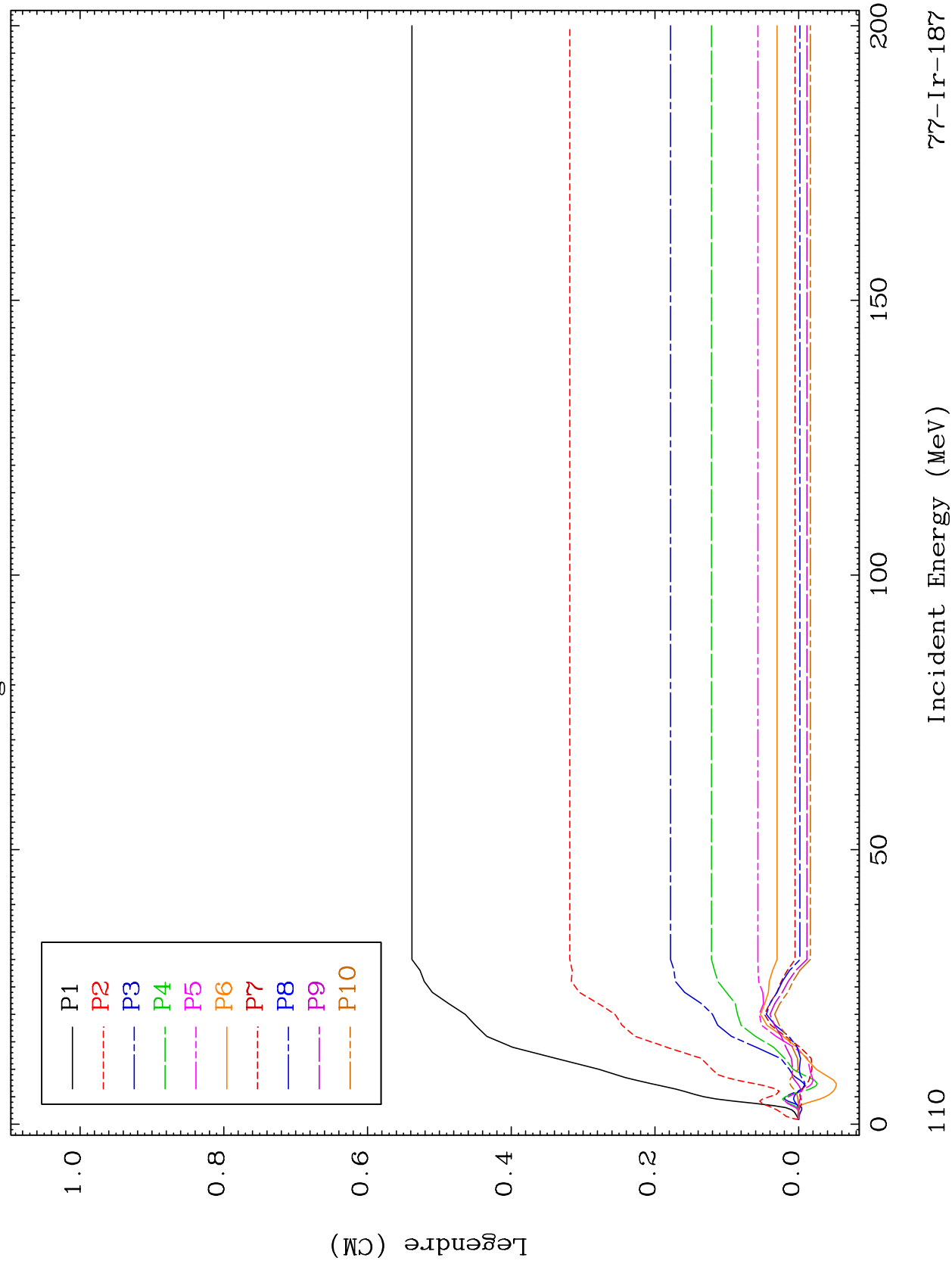


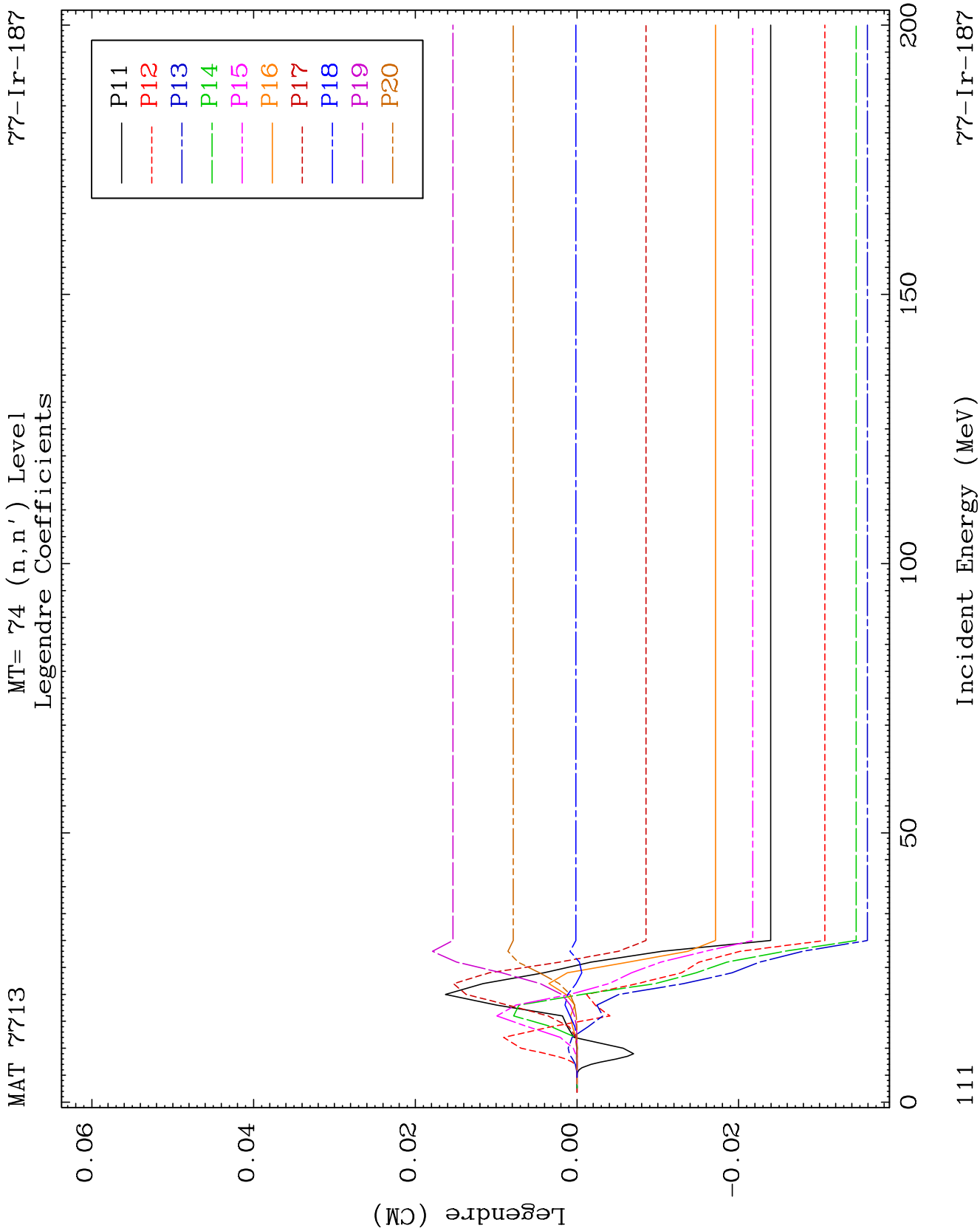


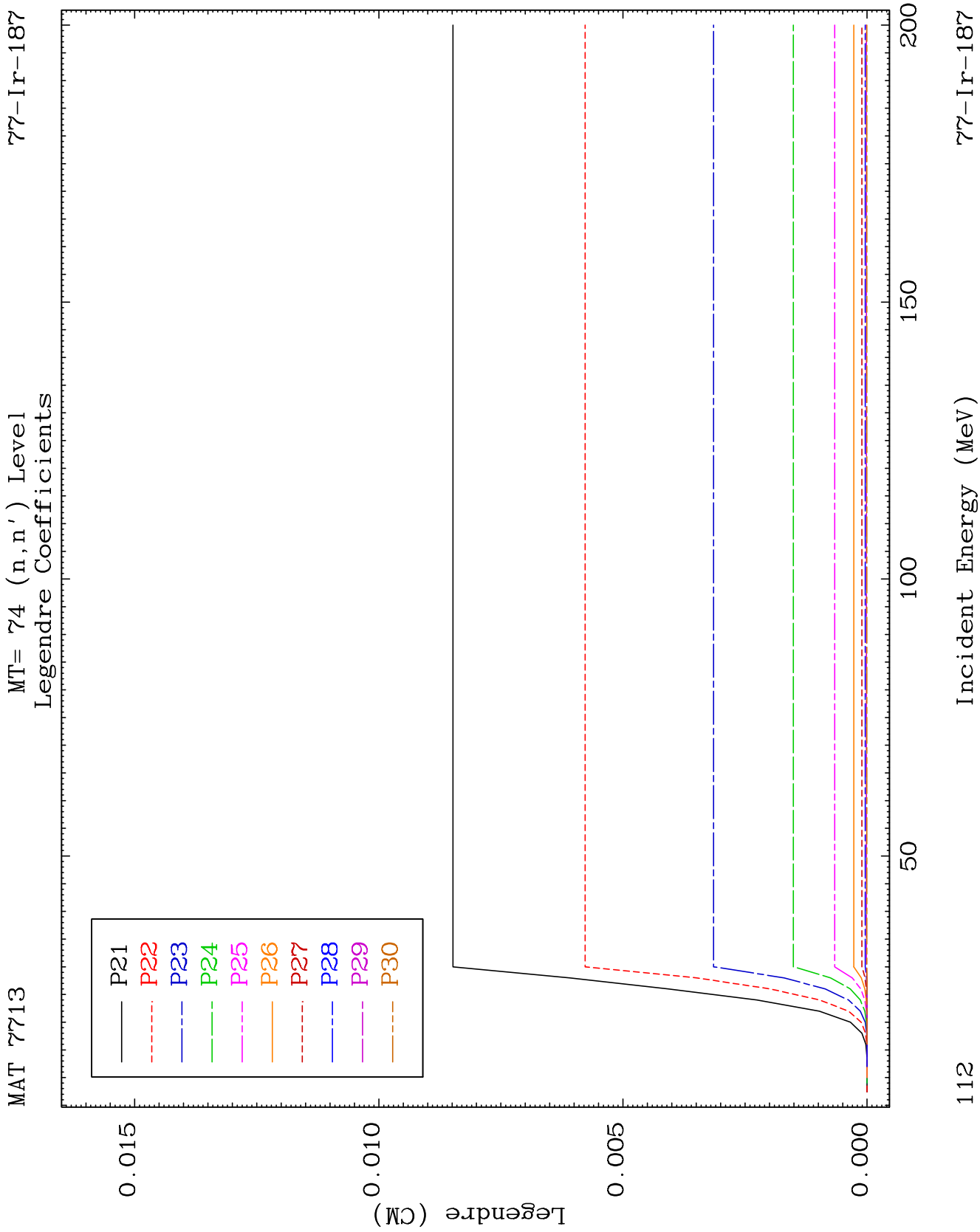
MAT 7713

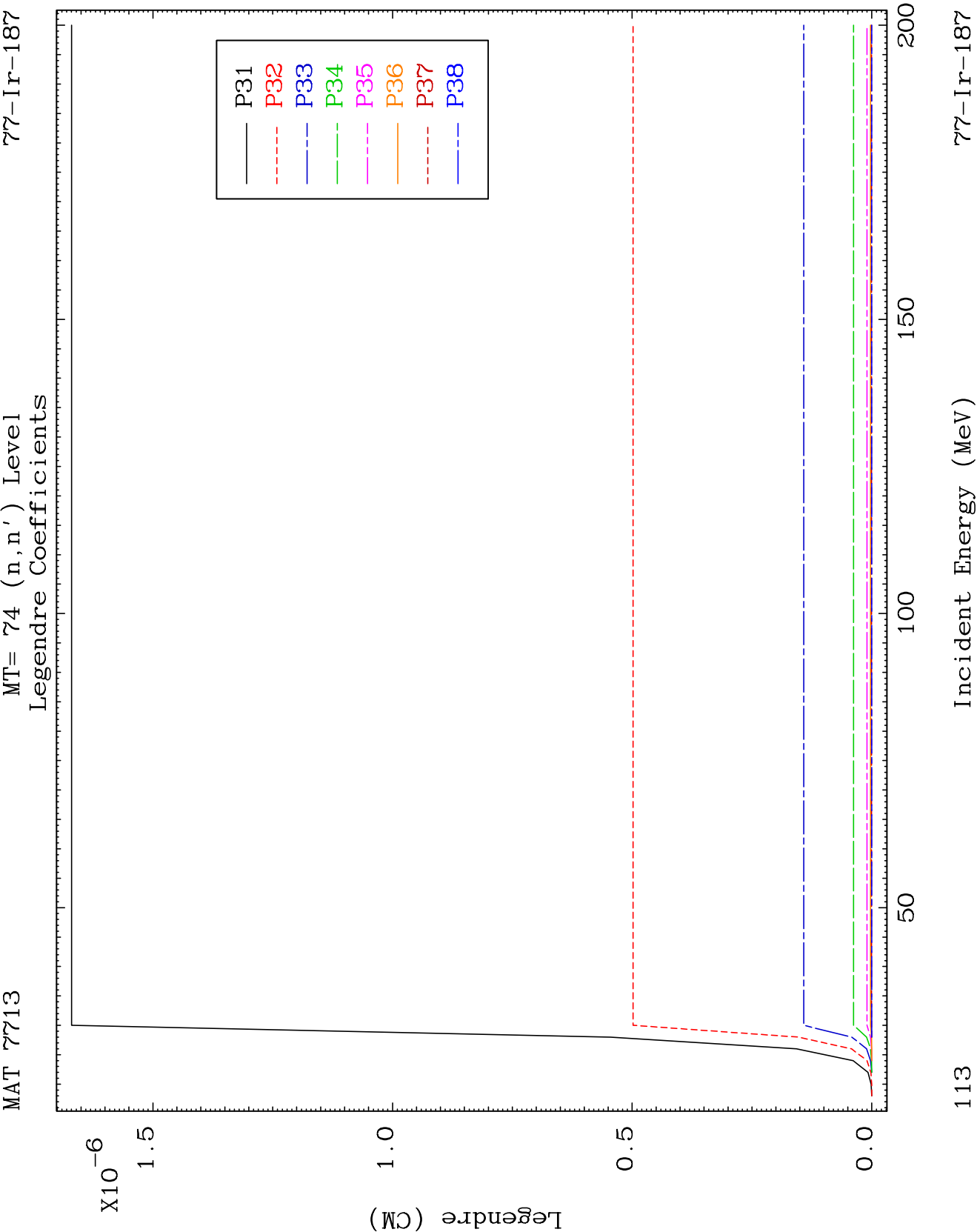
MT= 74 (n, n') Level
Legendre Coefficients

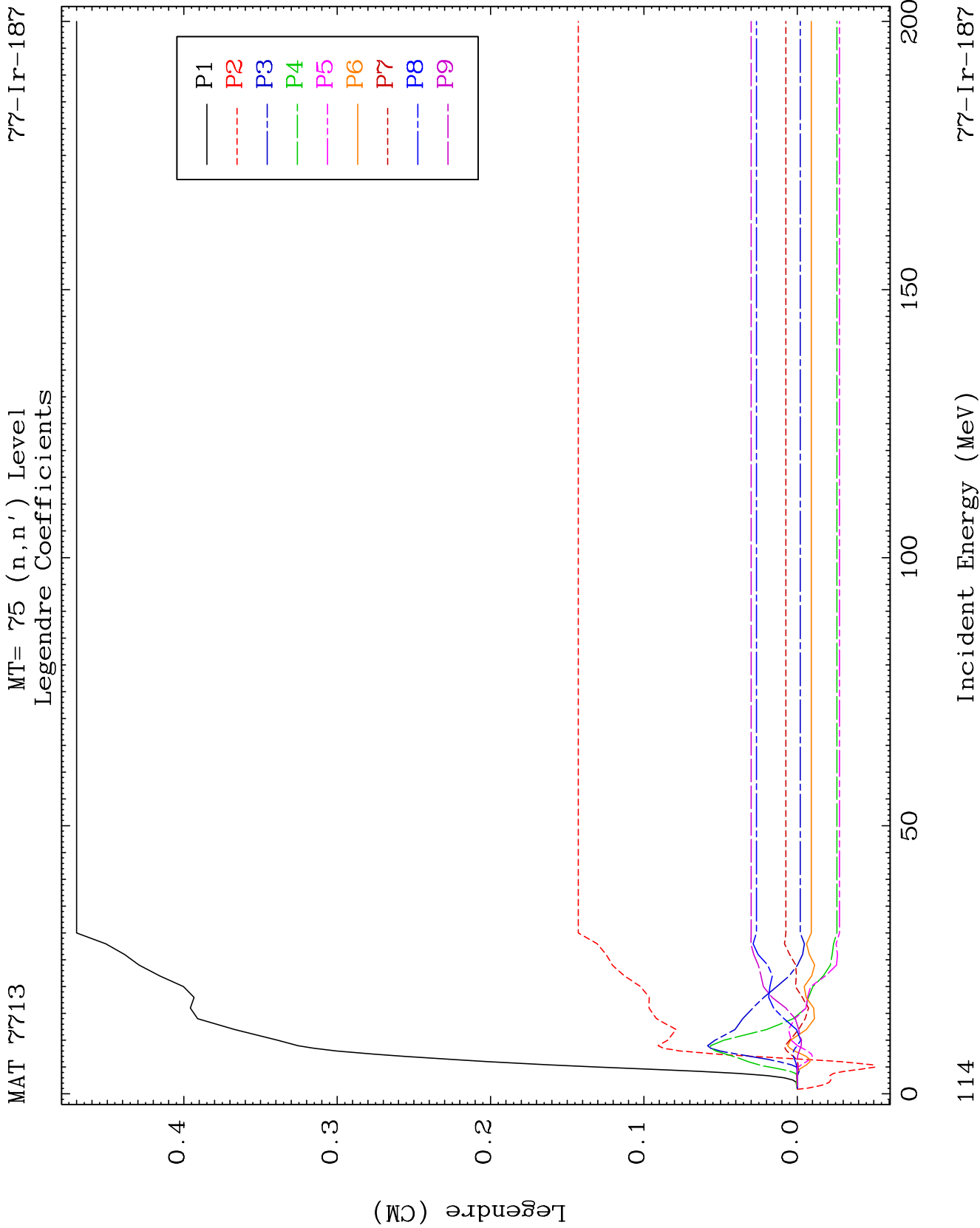
77-Ir-187

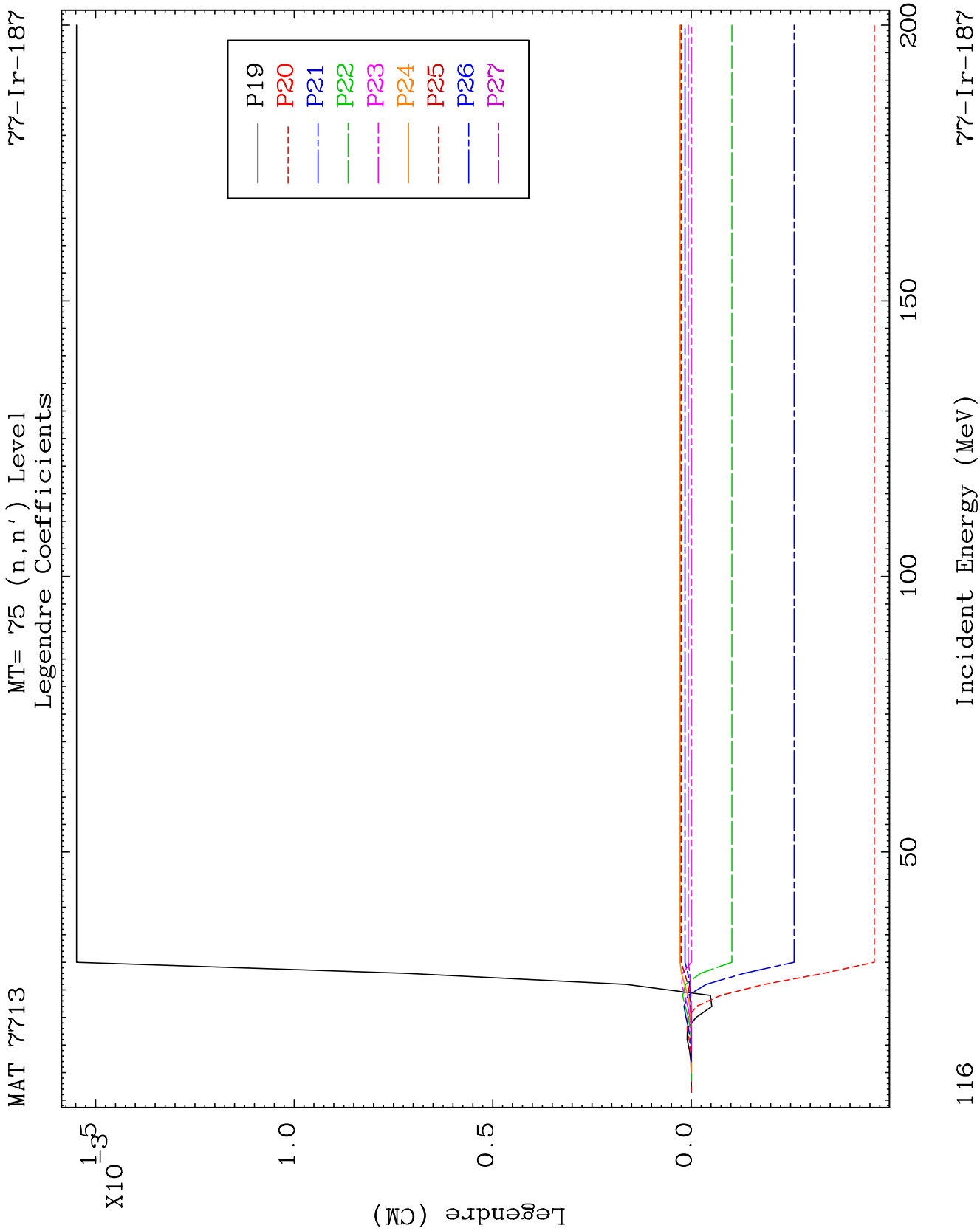


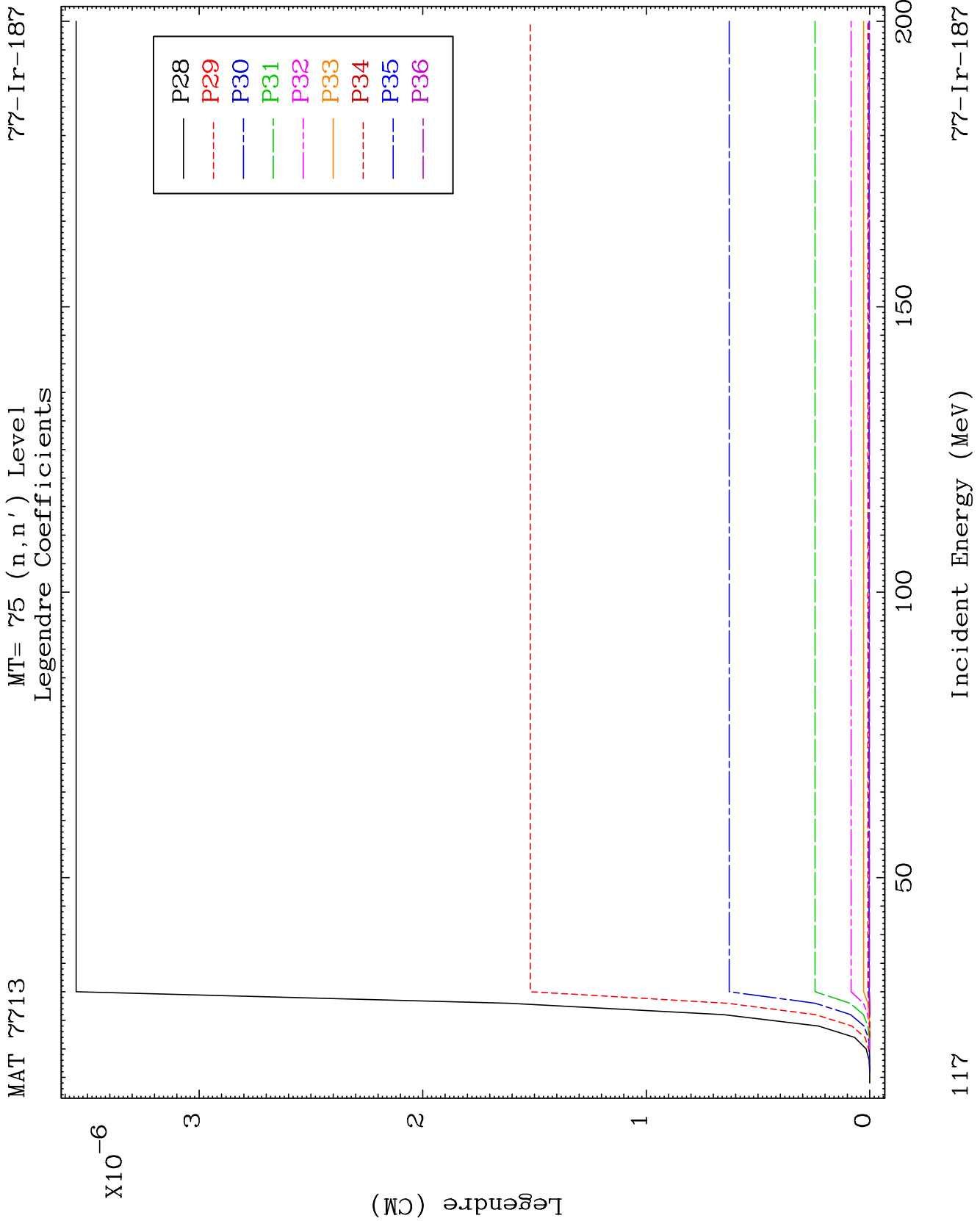


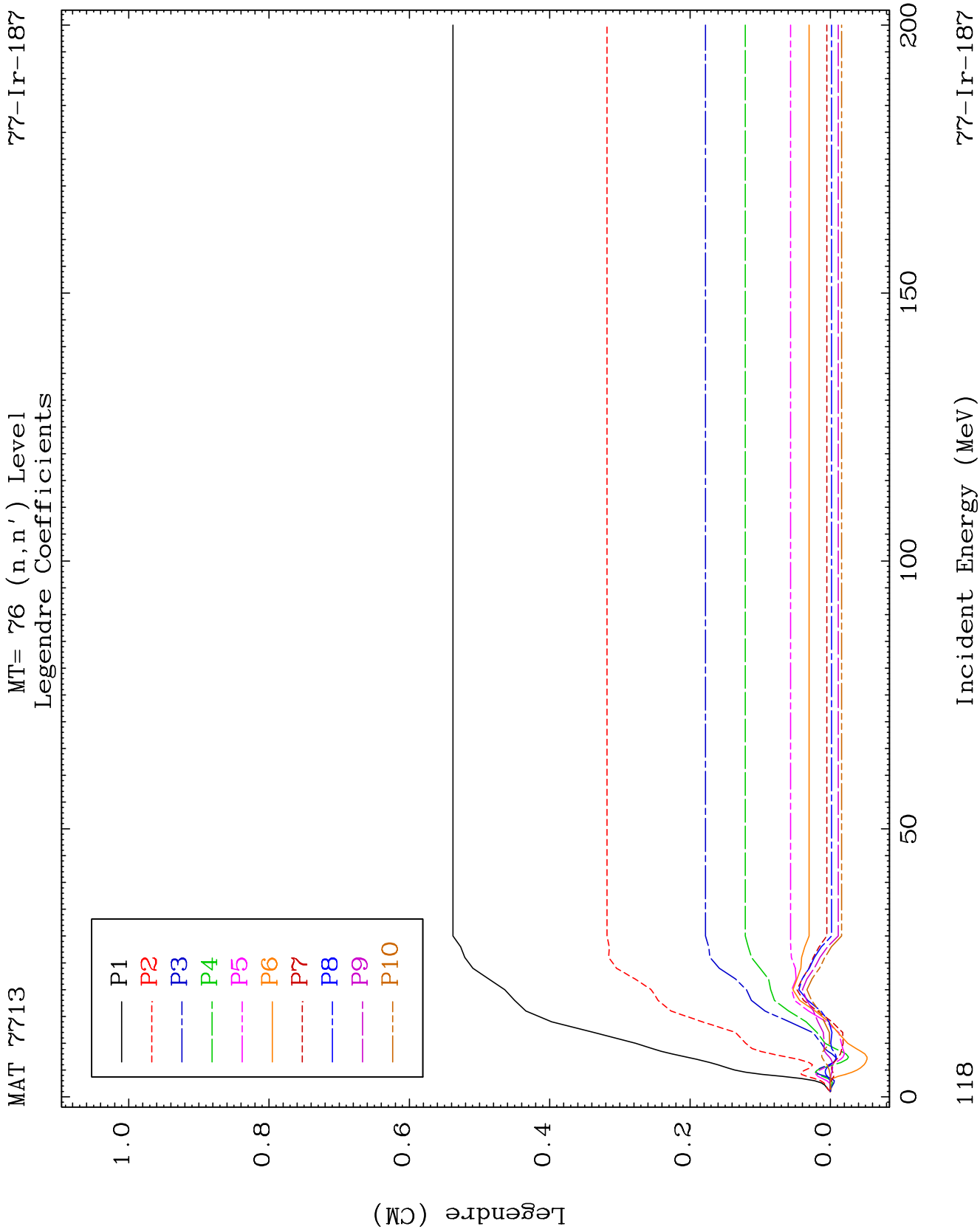








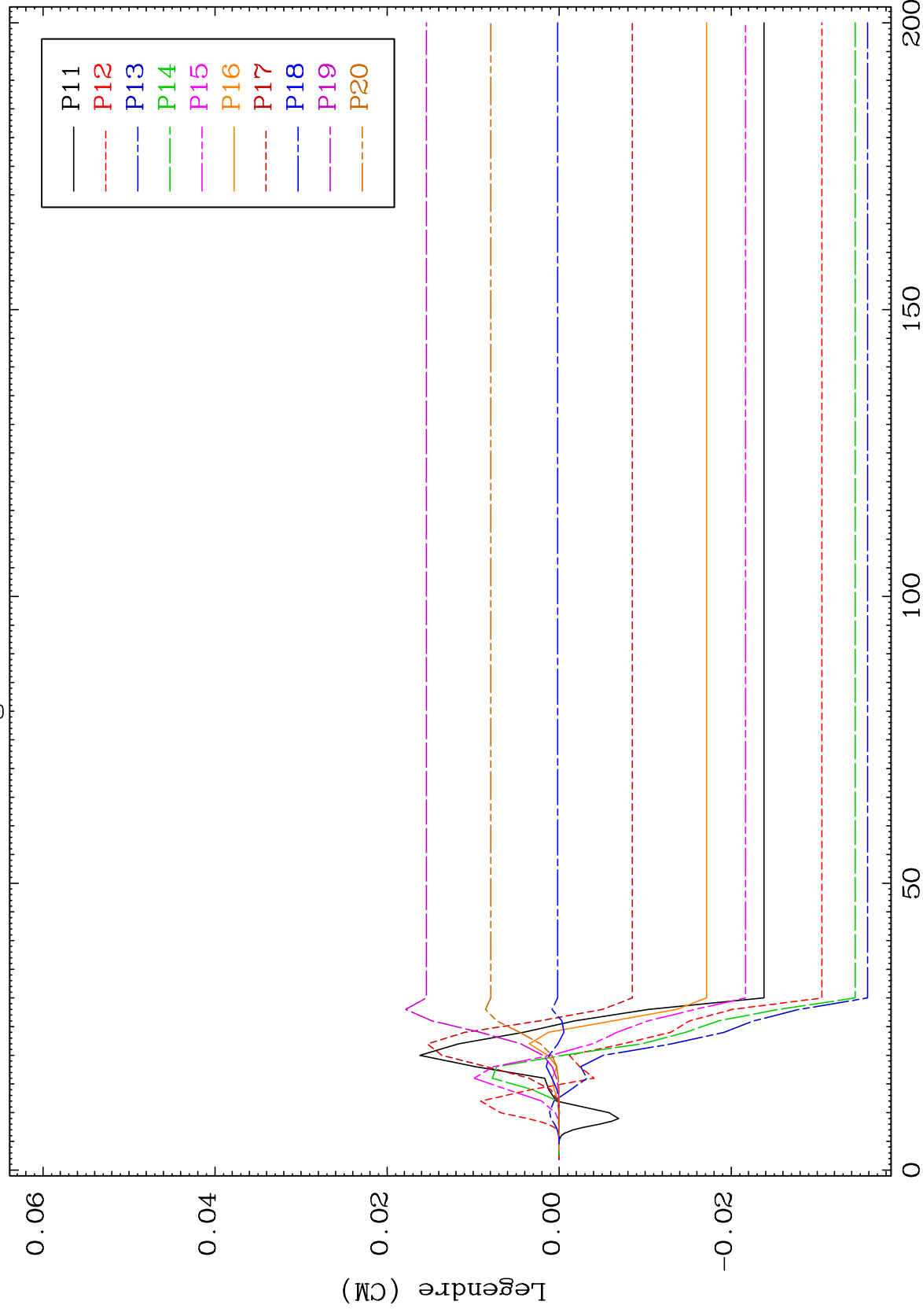




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MT= 76 (n, n') Level
Legendre Coefficients

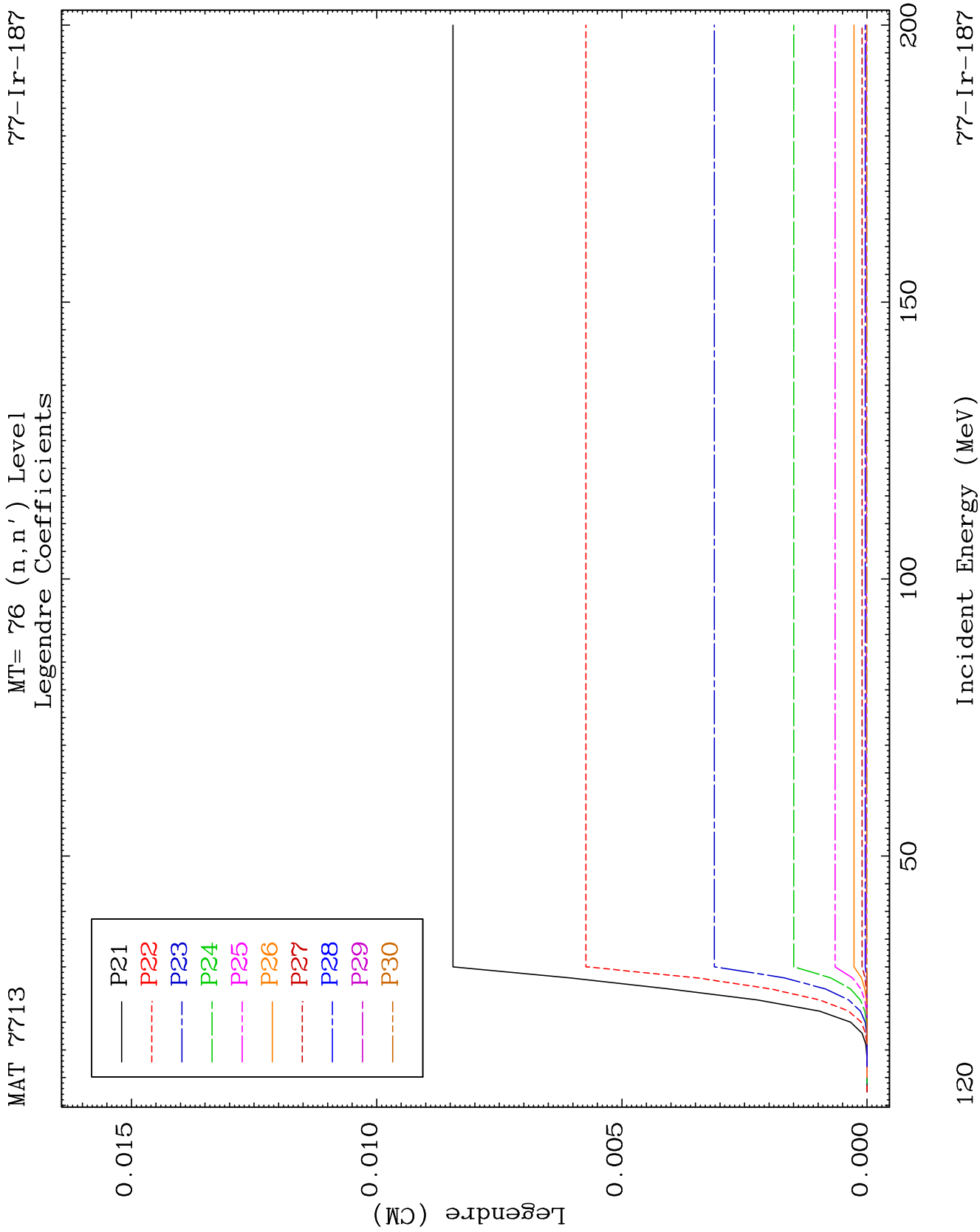
22-Ir-187

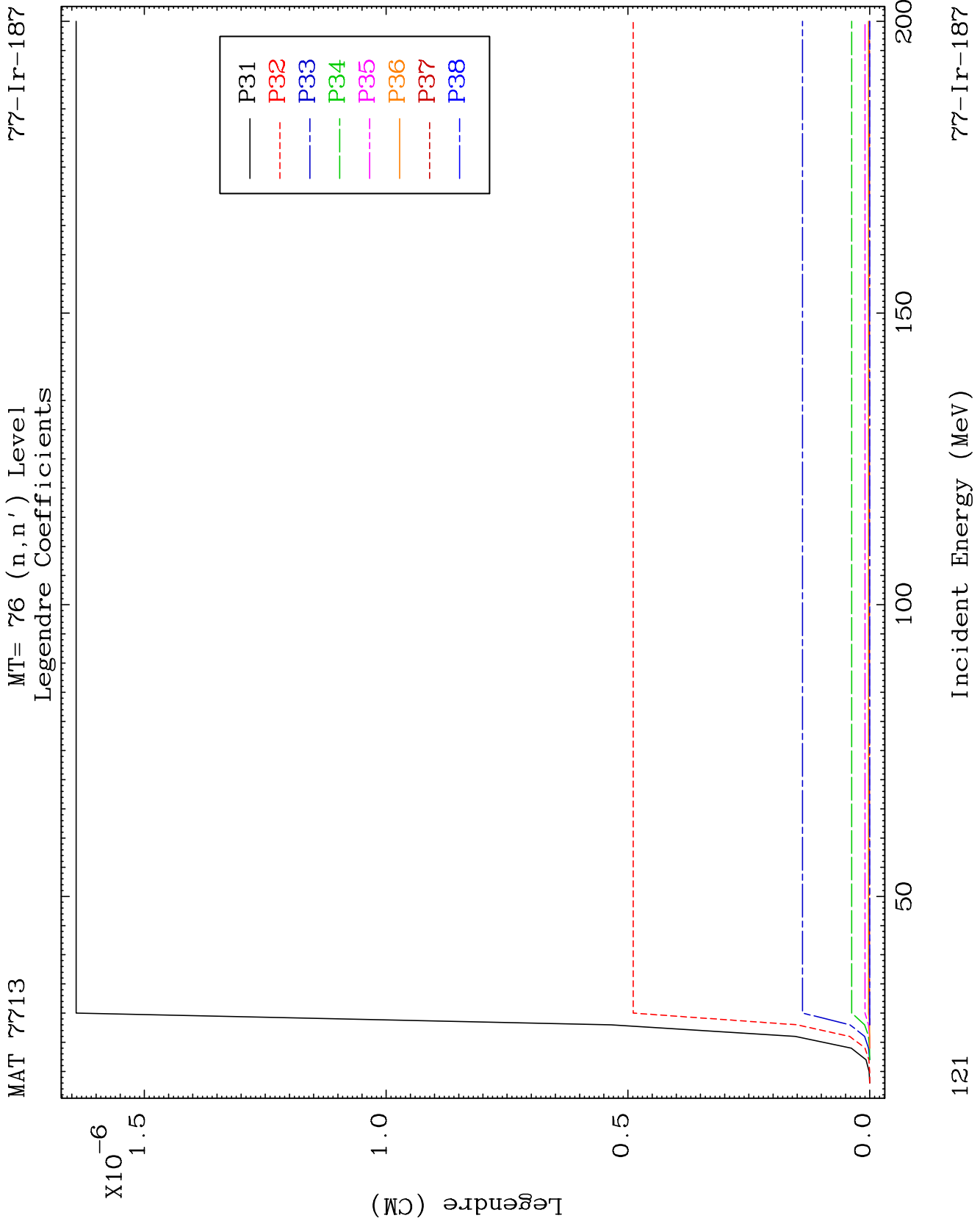


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Incident Energy (MeV)

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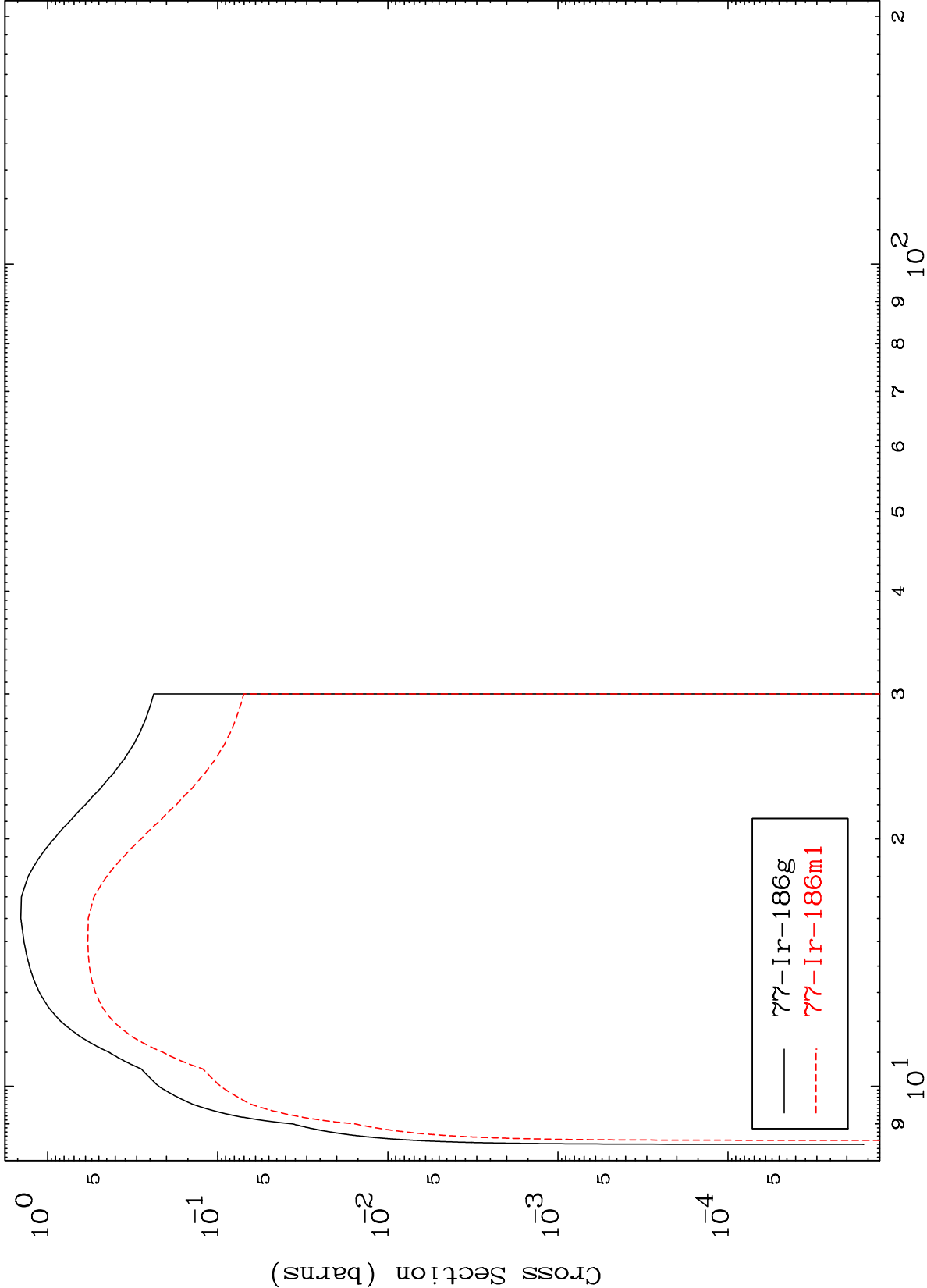




MAT 7713

77-Ir-187

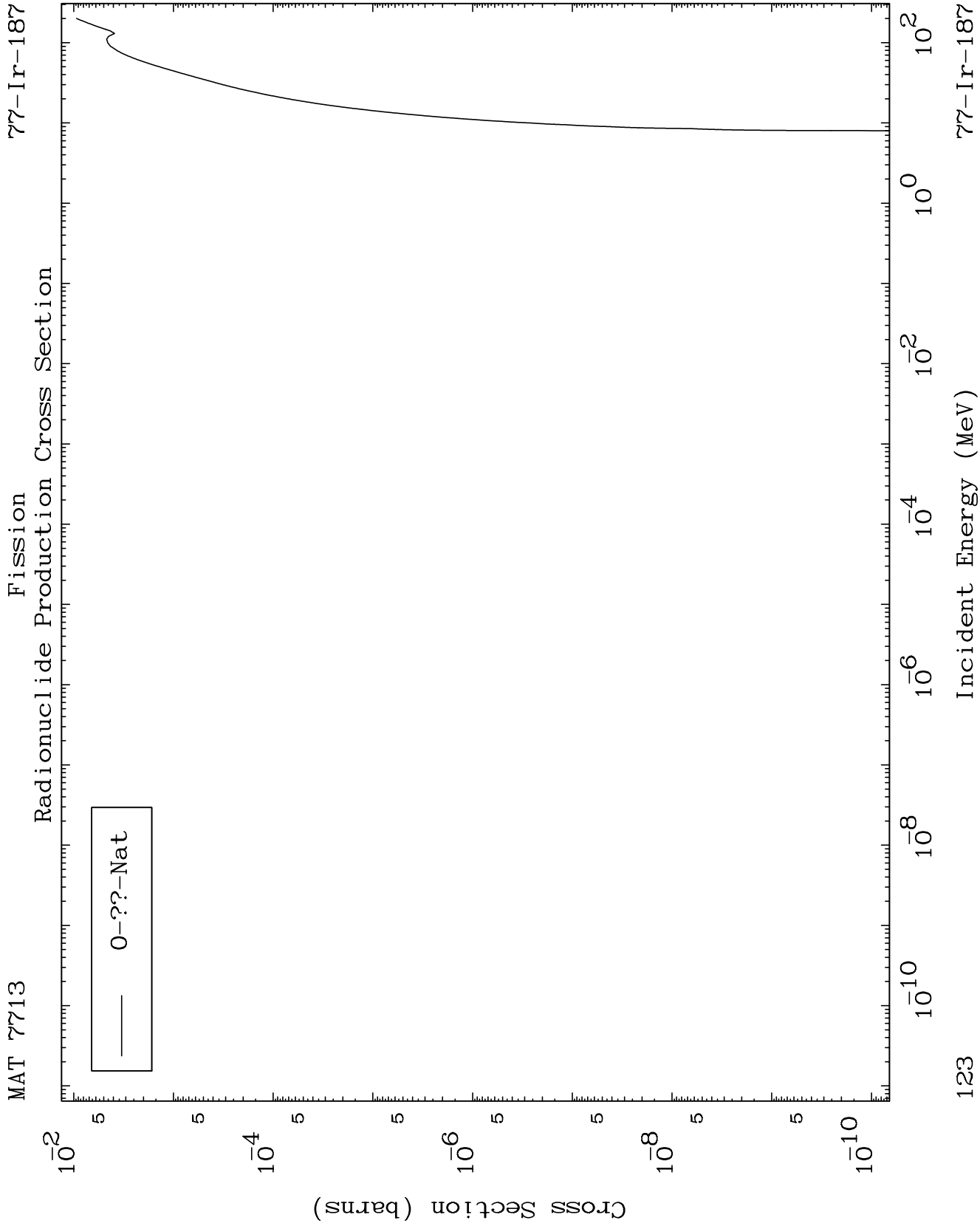
(n,2n)
Radionuclide Production Cross Section

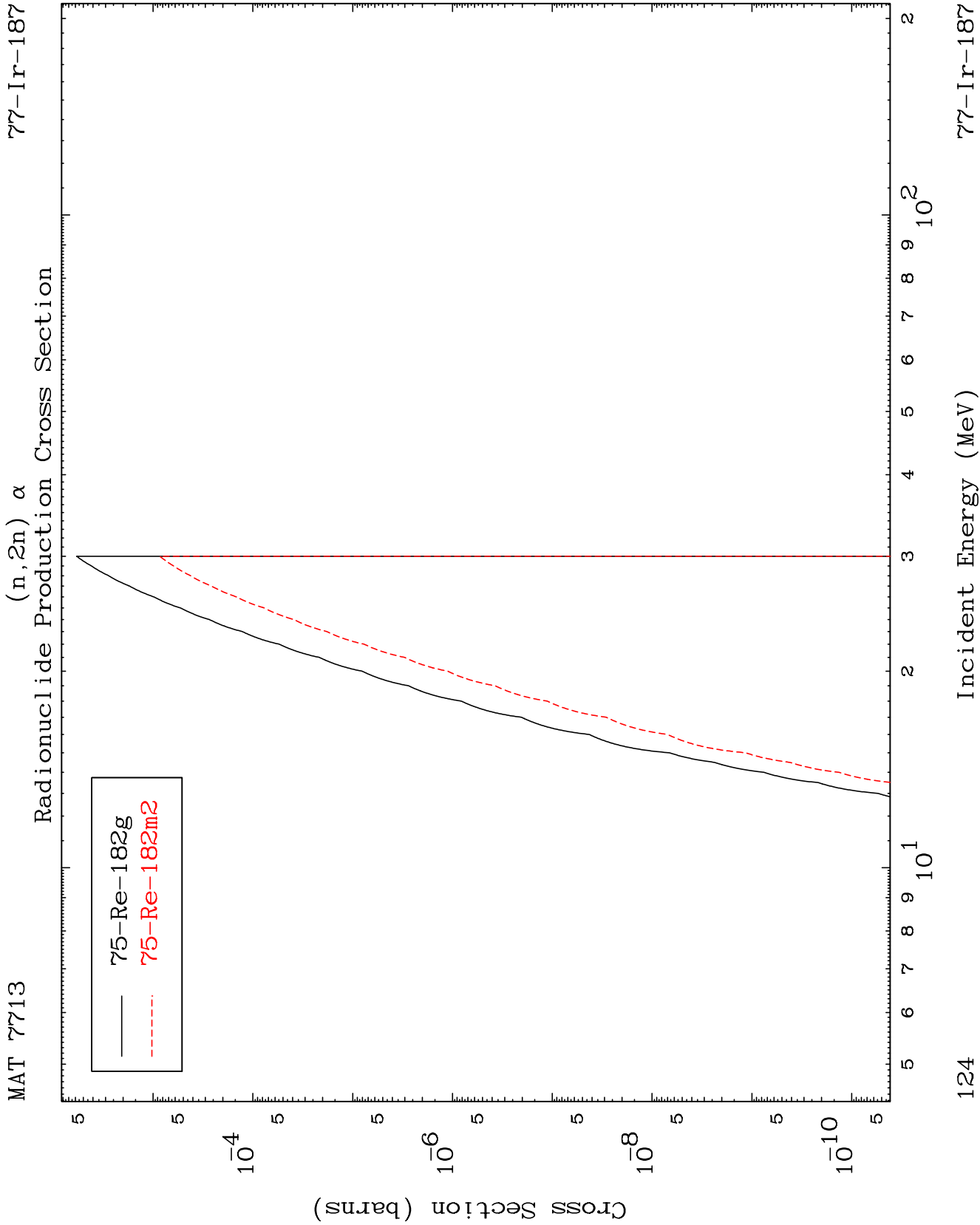


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Incident Energy (MeV)

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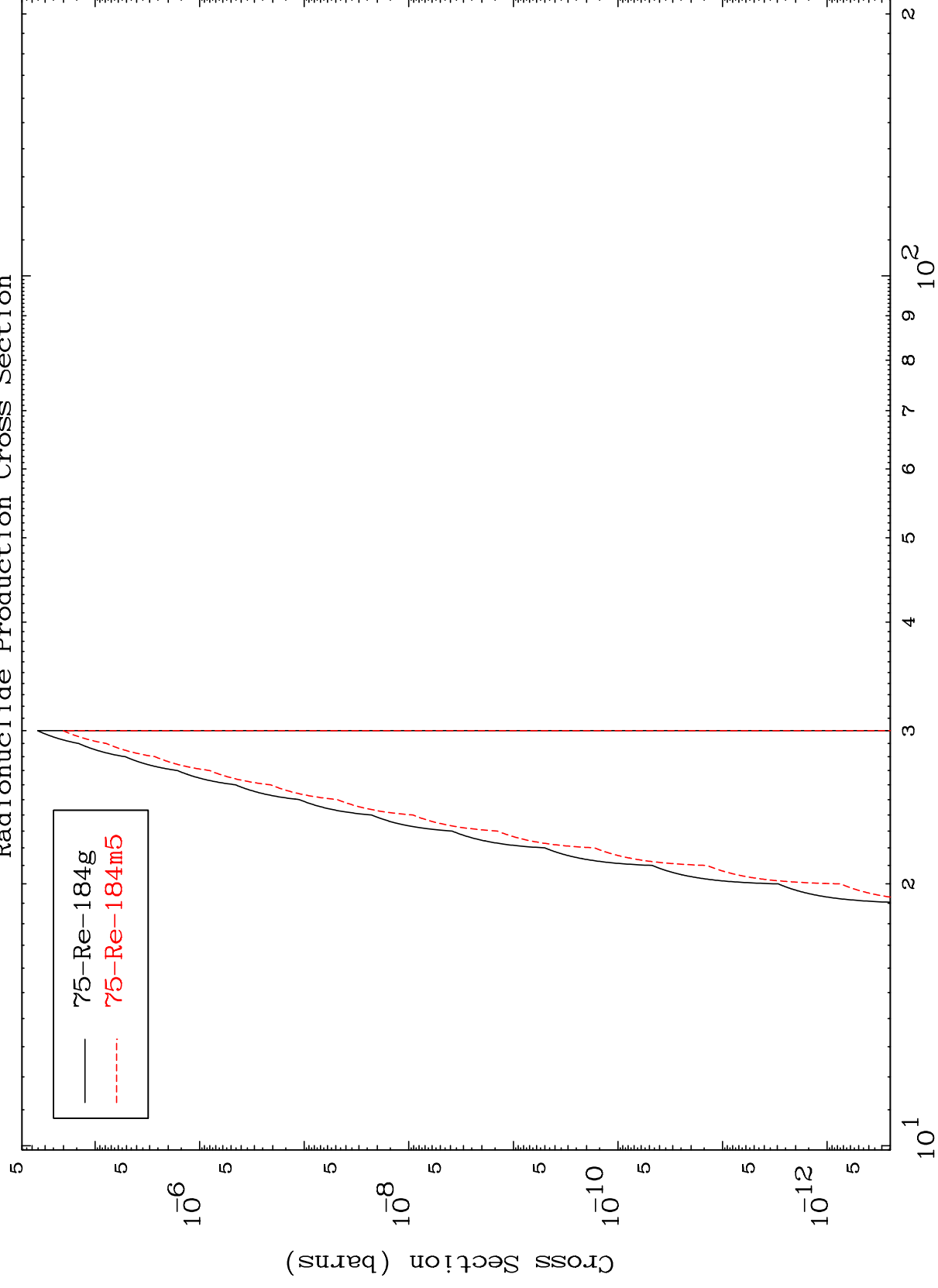


MAT 7713

 $(n, n') \text{ He-3}$

27-1r-187

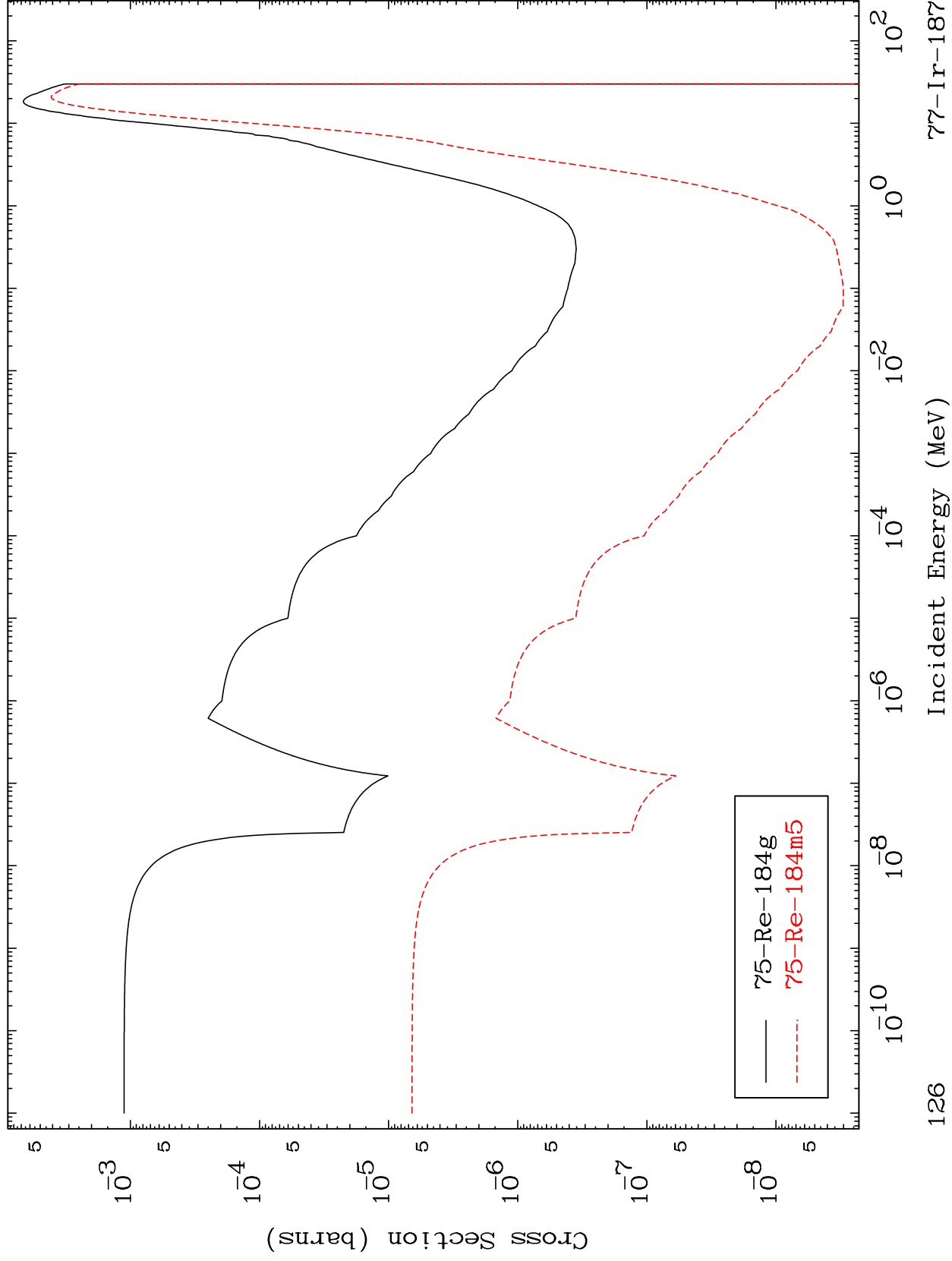
Radionuclide Production Cross Section



281-11-22

Incident Energy (MeV)

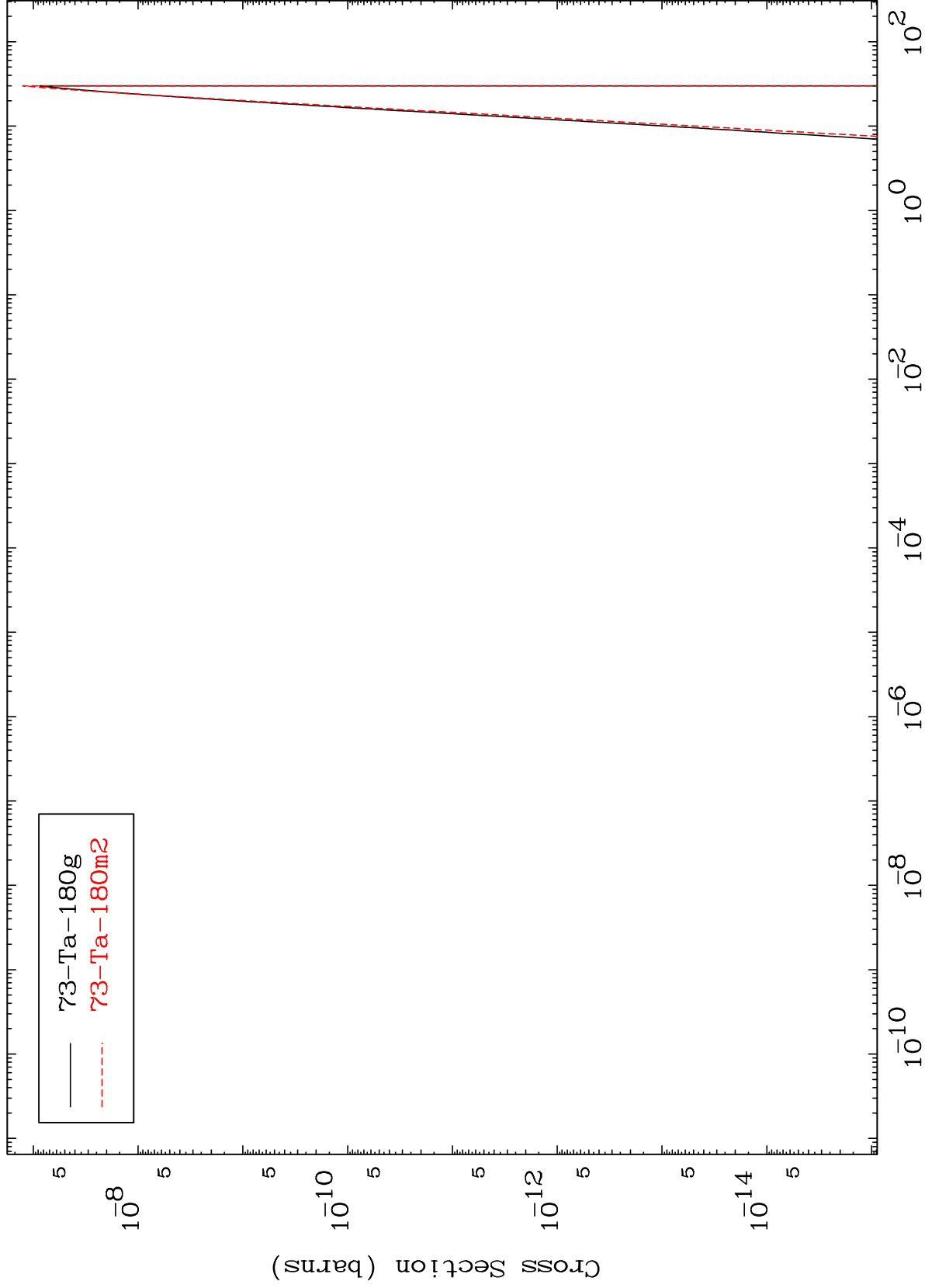
Radionuclide Production Cross Section
(n, α)



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$^{77}\text{Ir-187}$

Radionuclide Production Cross Section



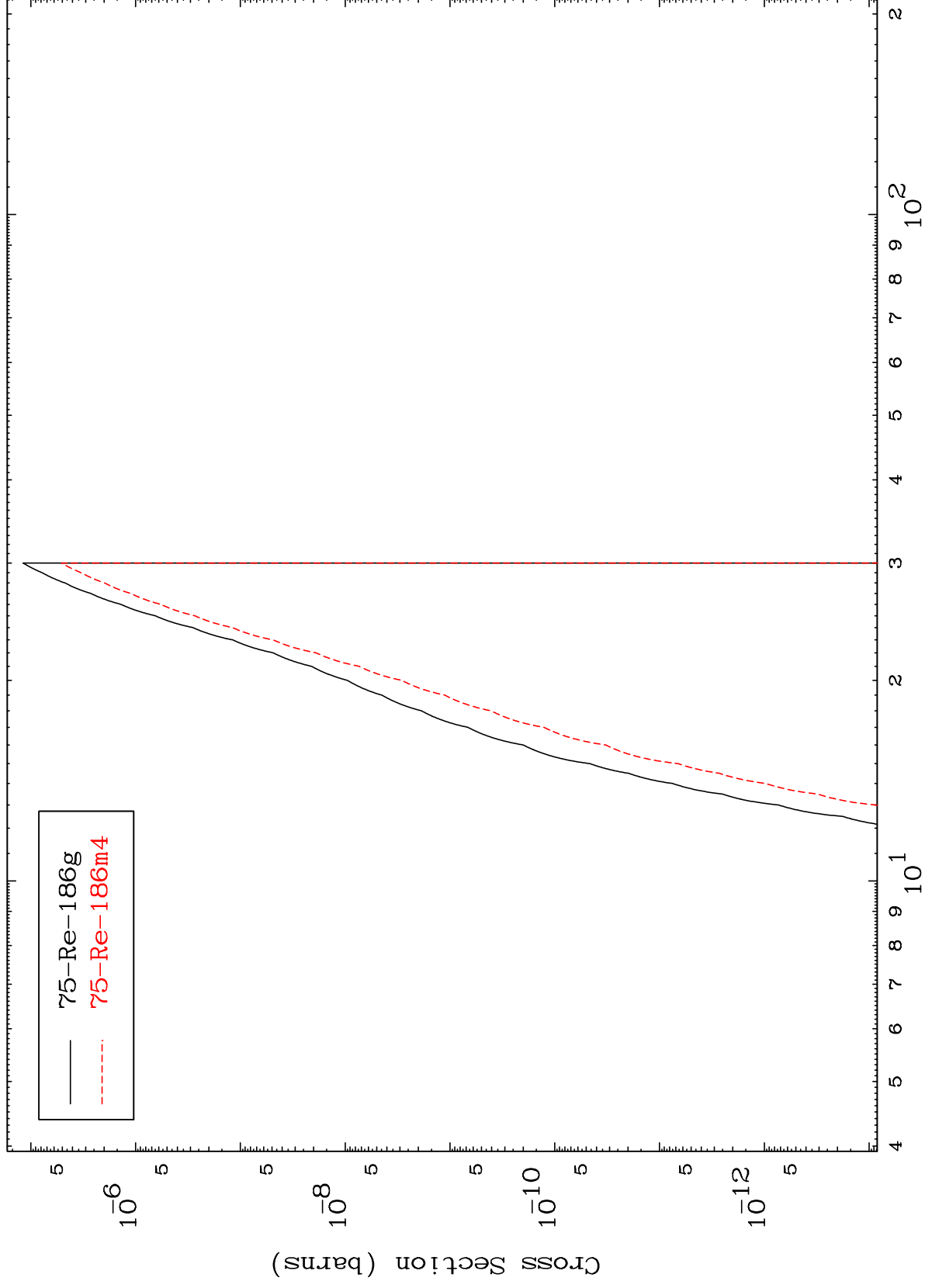
127

$^{77}\text{Ir-187}$

MAT 7713

⁷⁷Ir-187

Radionuclide Production Cross Section
(n,2p)



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Incident Energy (MeV)

⁷⁷Ir-187

Radionuclide Production Cross Section

