

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

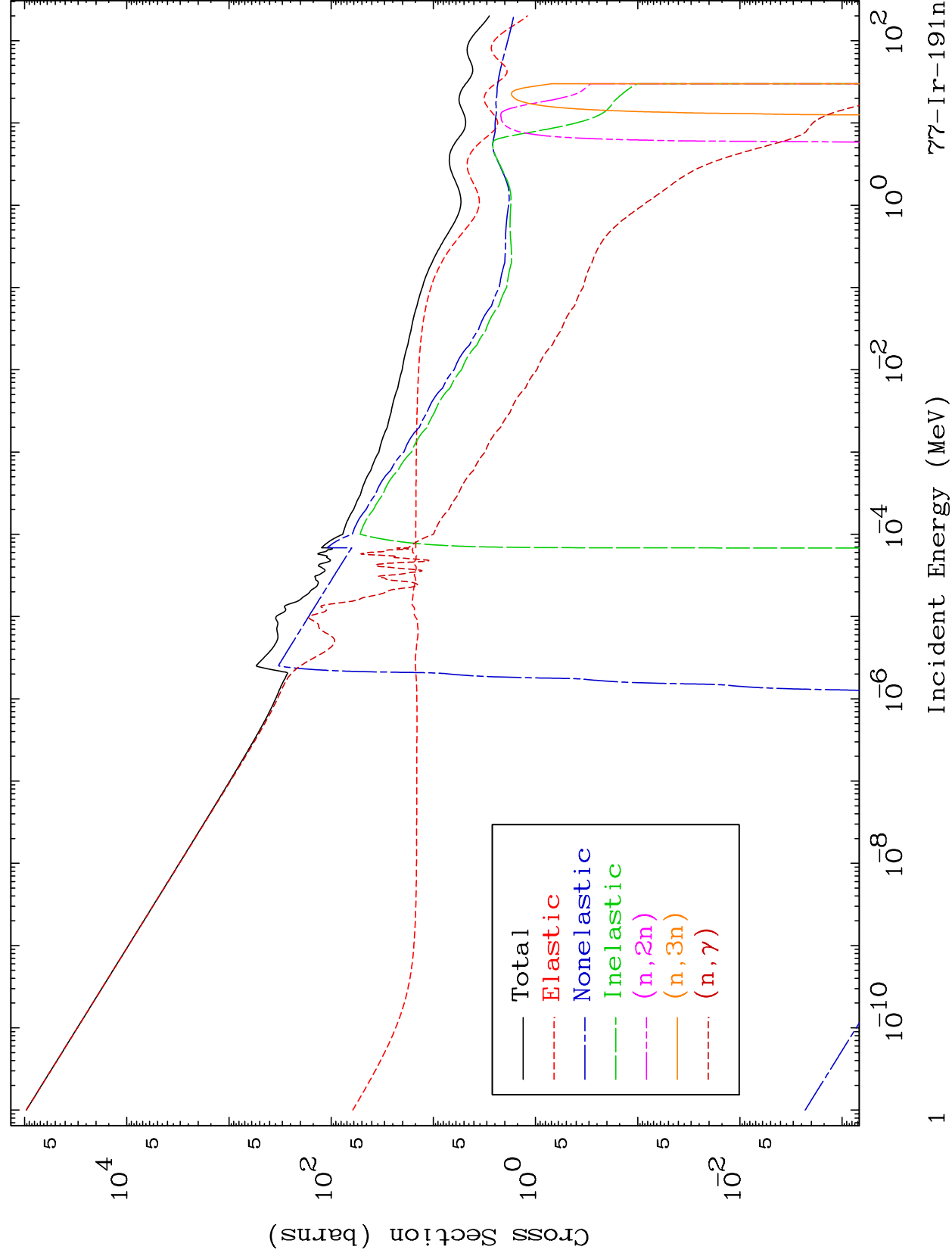
Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7727

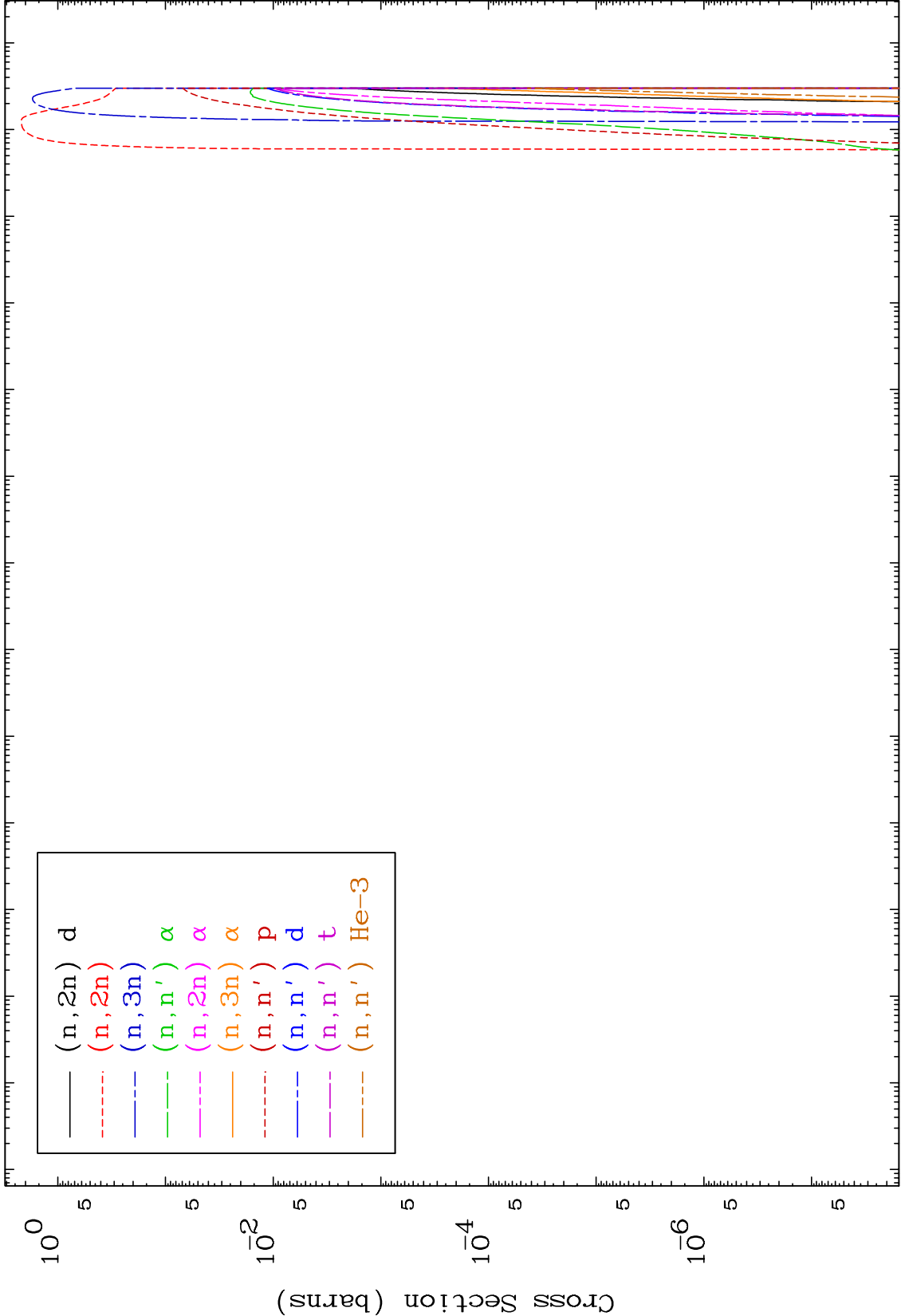
Neutron Major
293 Kelvin Cross Sections

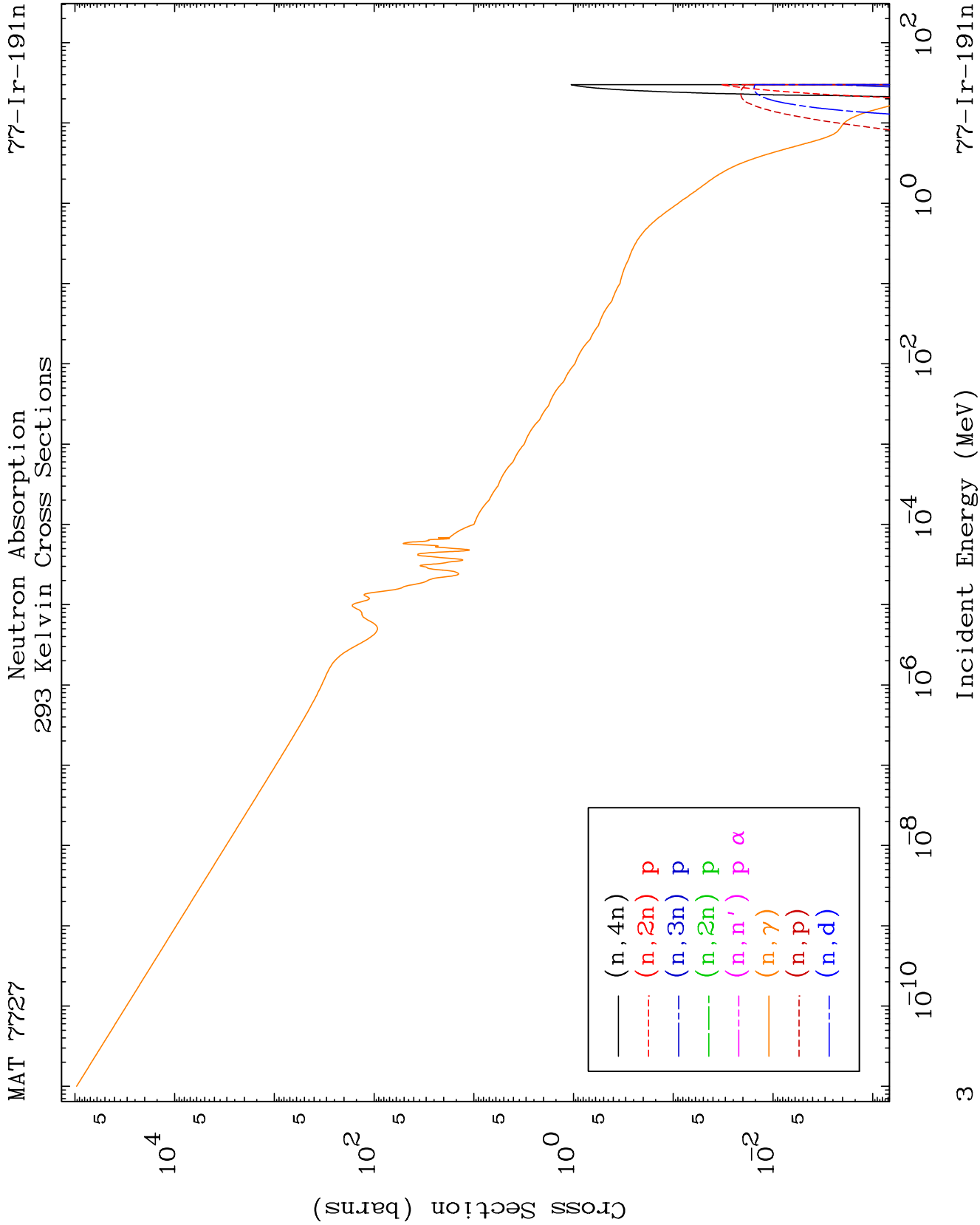
²²⁷Ir-191n

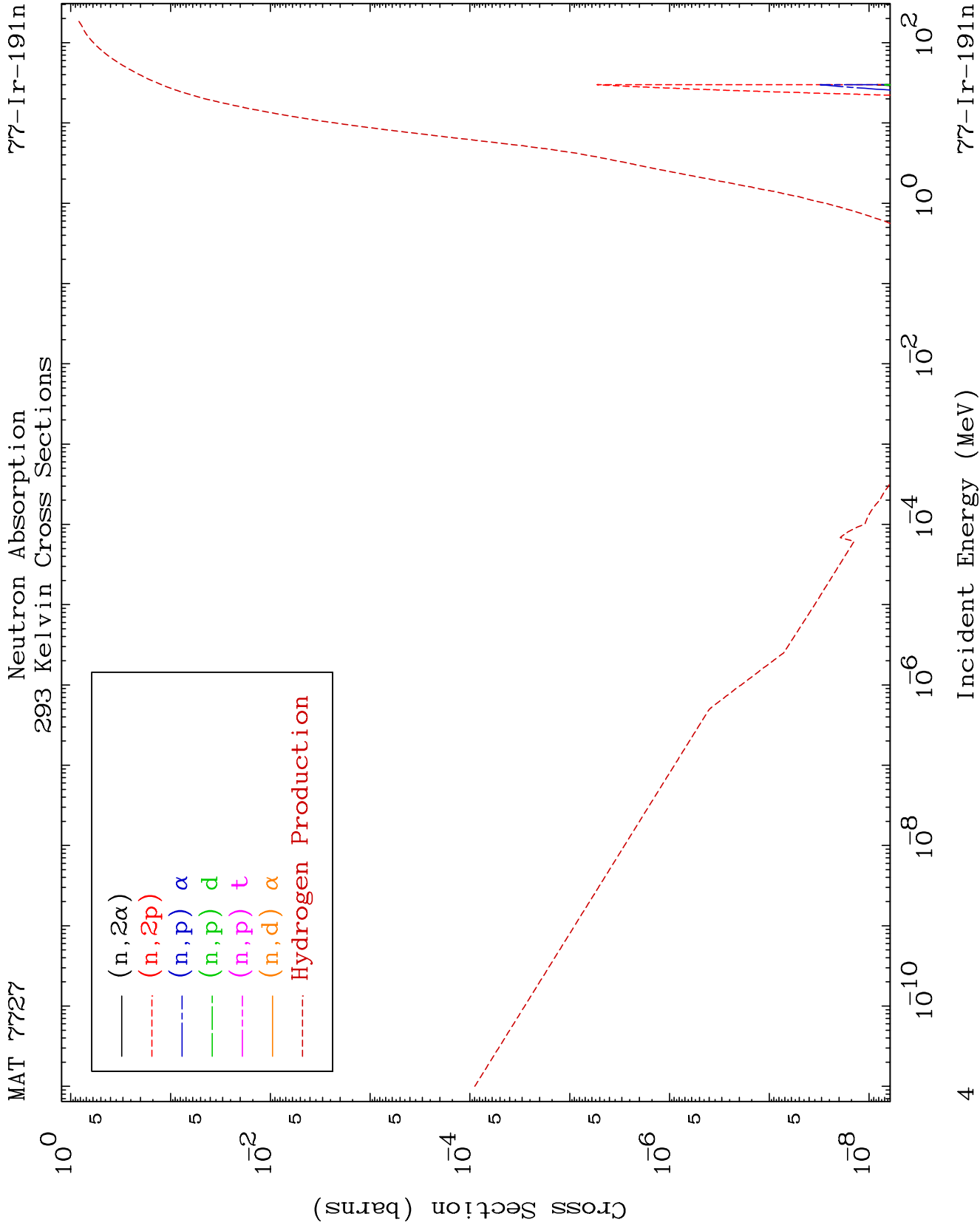
MAT 7727

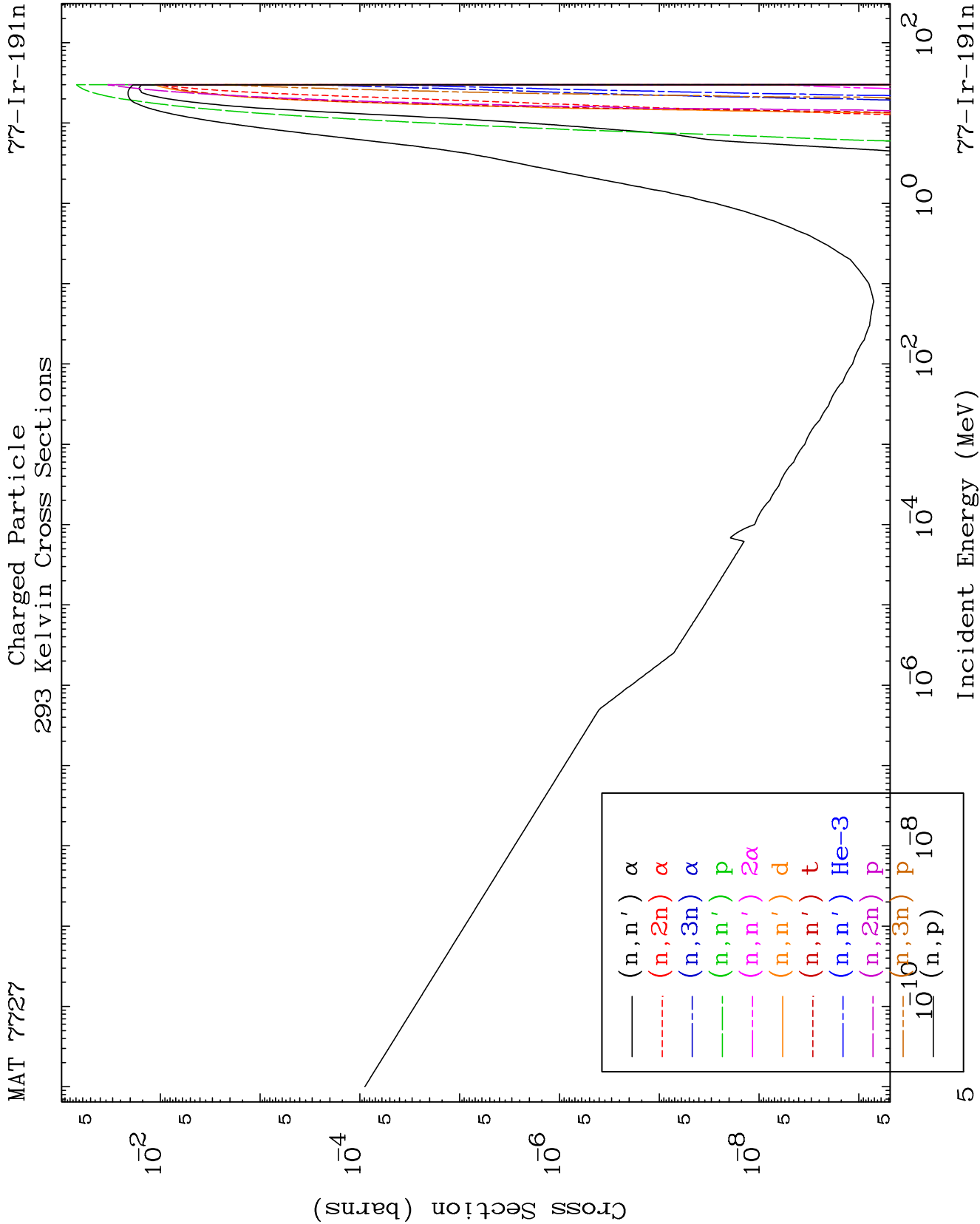
Neutron Absorption
293 Kelvin Cross Sections

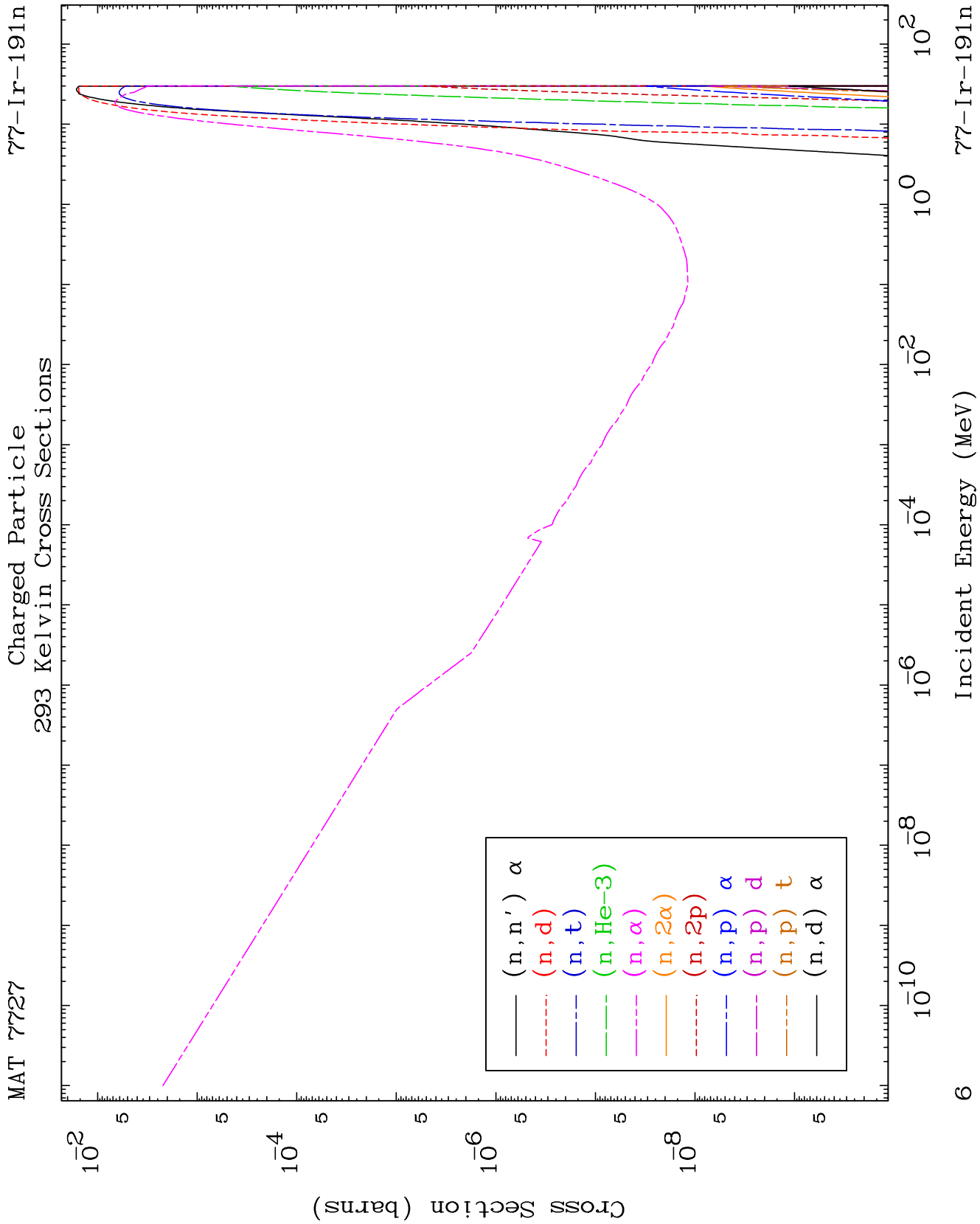
77-Ir-191n

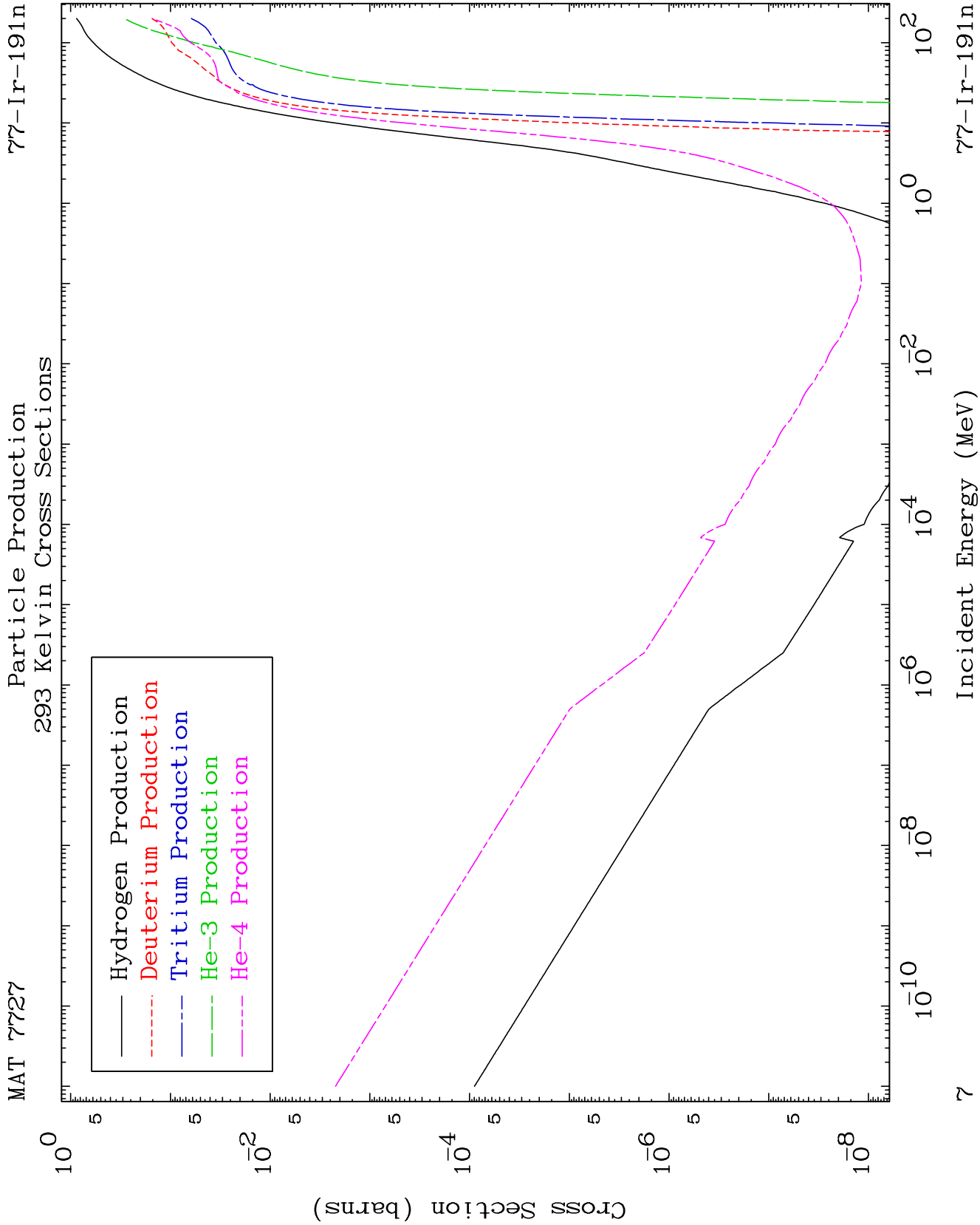


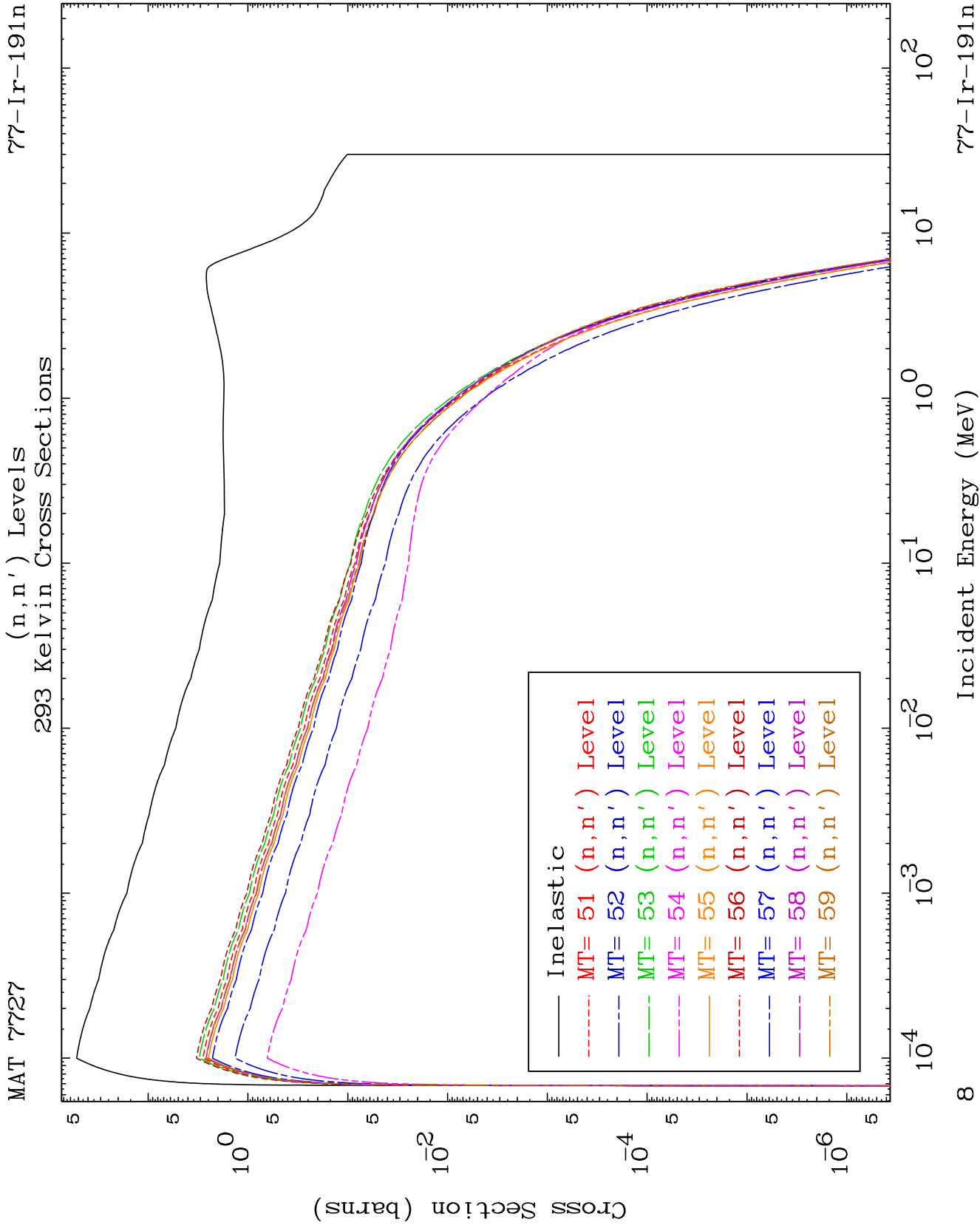








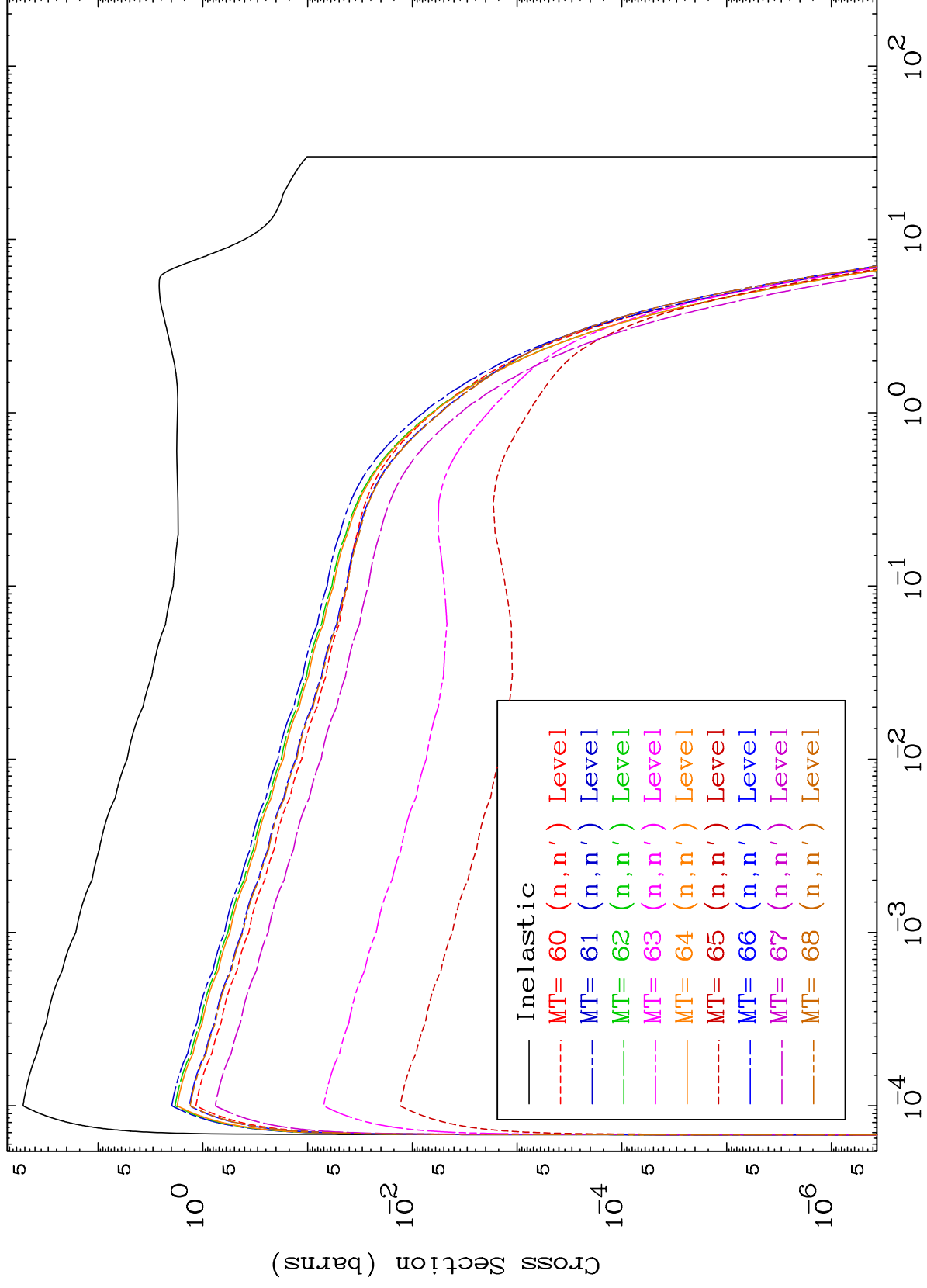


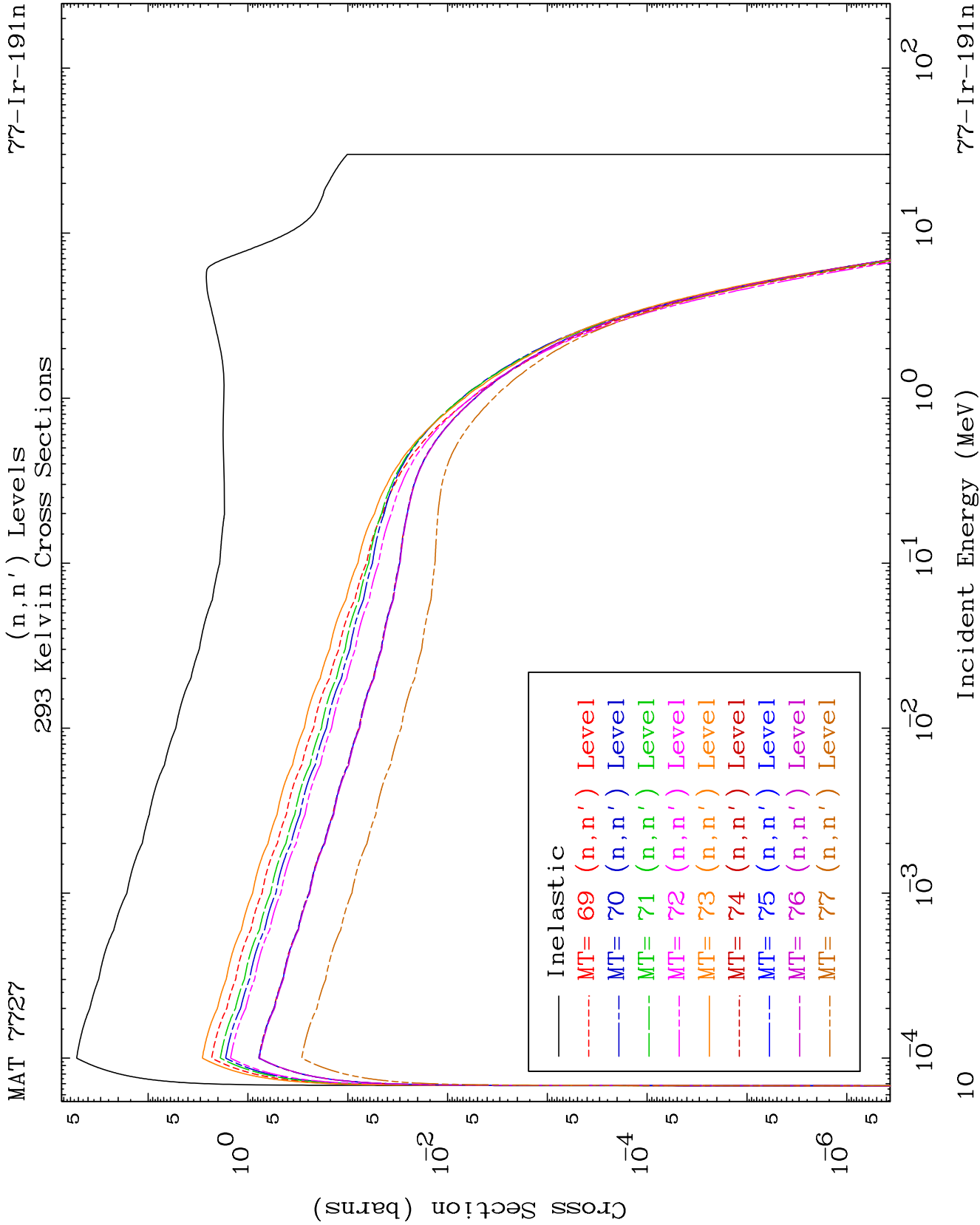


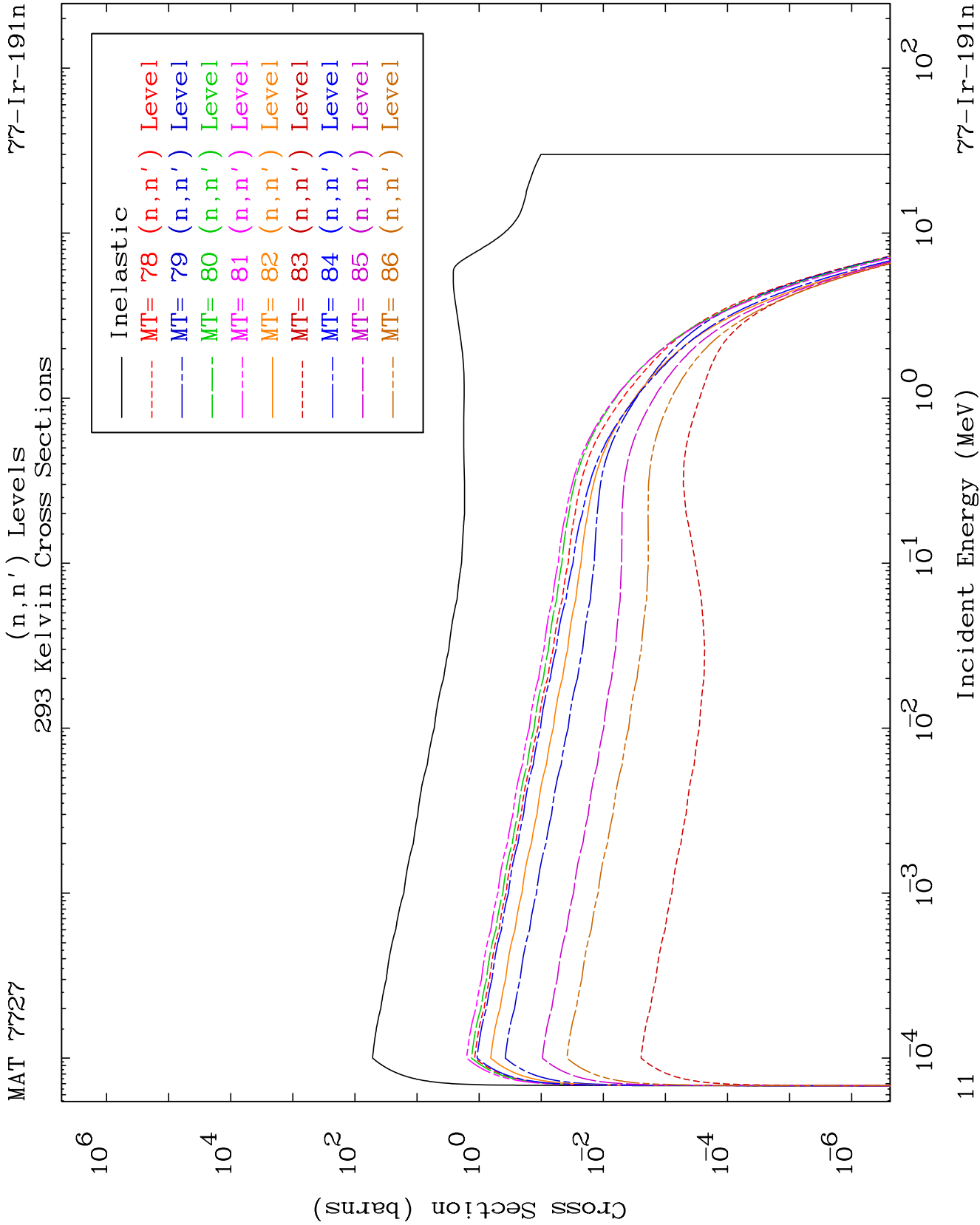
MAT 7727

(n,n') Levels
293 Kelvin Cross Sections

⁷⁷Ir-191n



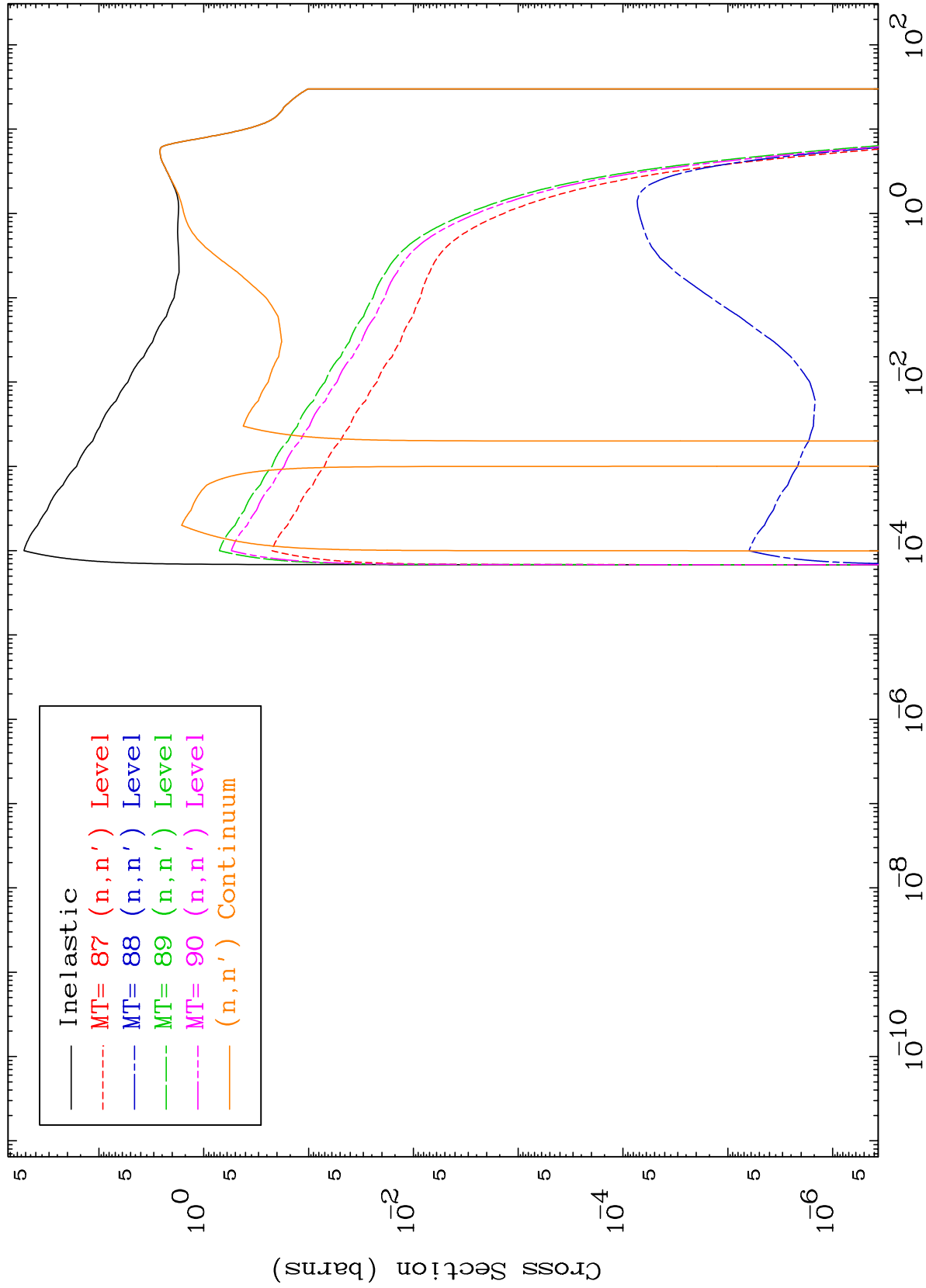




MAT 7727

(n,n') Levels
293 Kelvin Cross Sections

⁷⁷Ir-191n



12

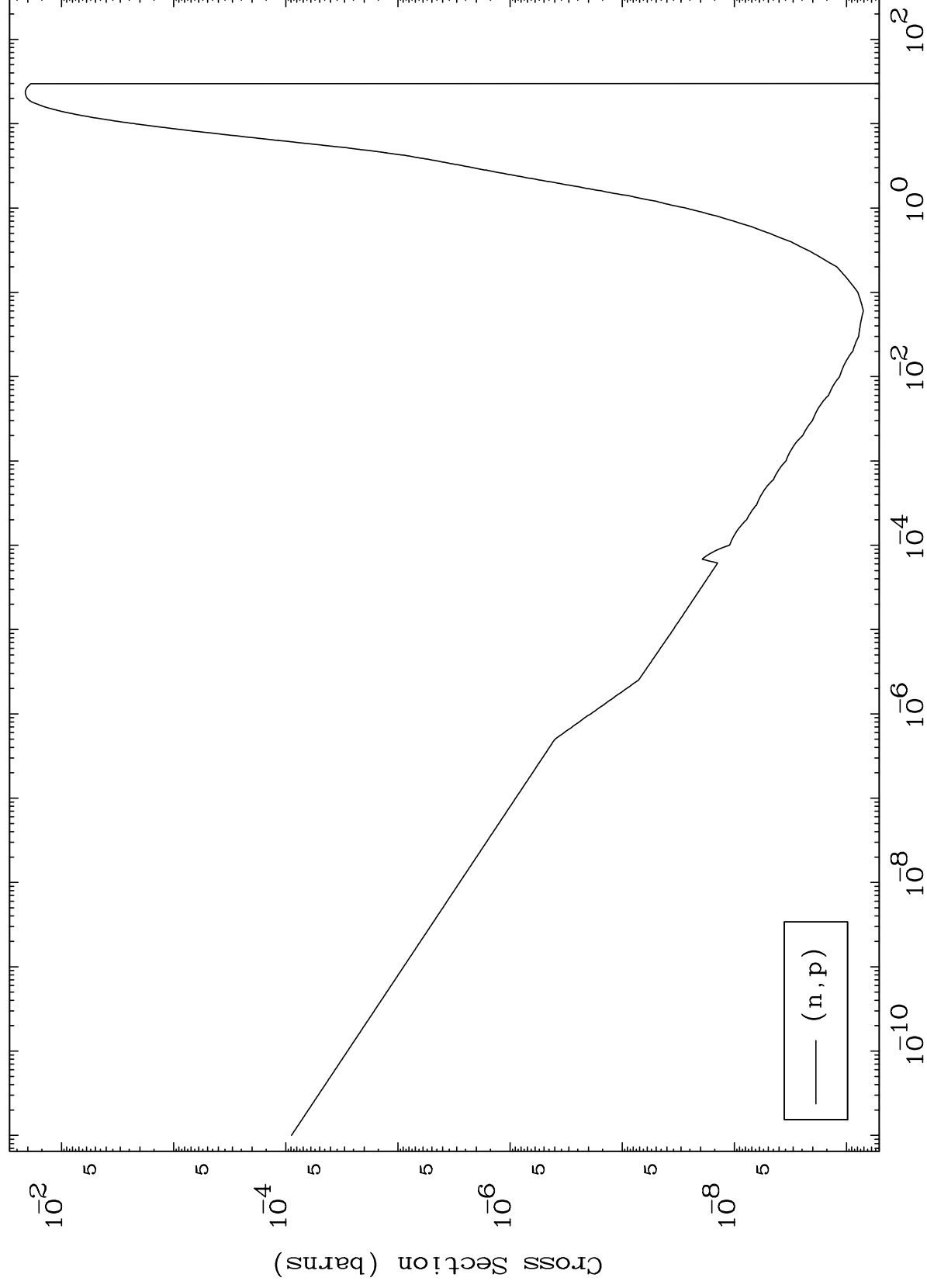
Incident Energy (MeV)

⁷⁷Ir-191n

MAT 7727

(n,p) Levels
293 Kelvin Cross Sections

$^{77}\text{Ir-191n}$



13

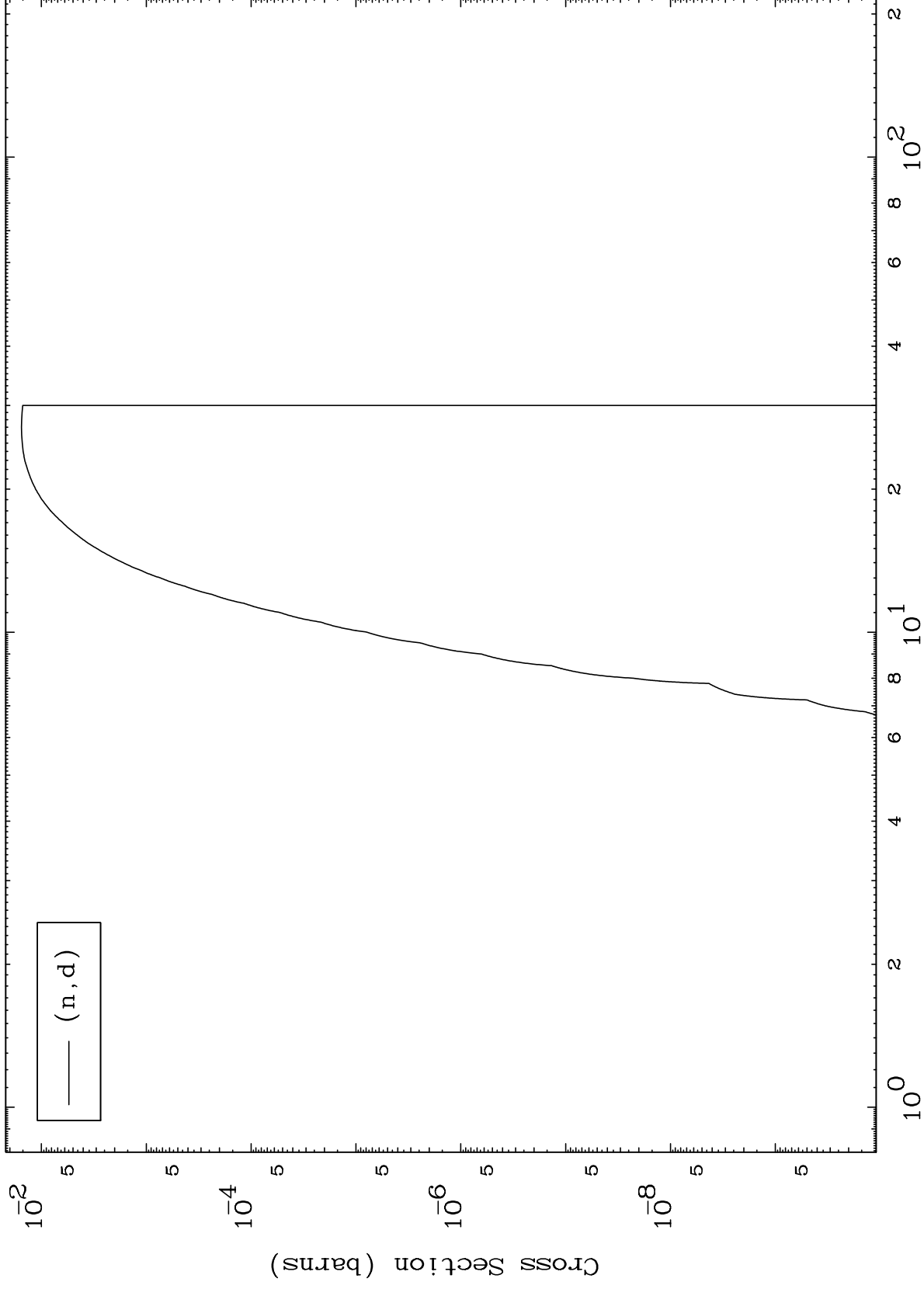
Incident Energy (MeV)

$^{77}\text{Ir-191n}$

MAT 7727

(n,d) Levels
293 Kelvin Cross Sections

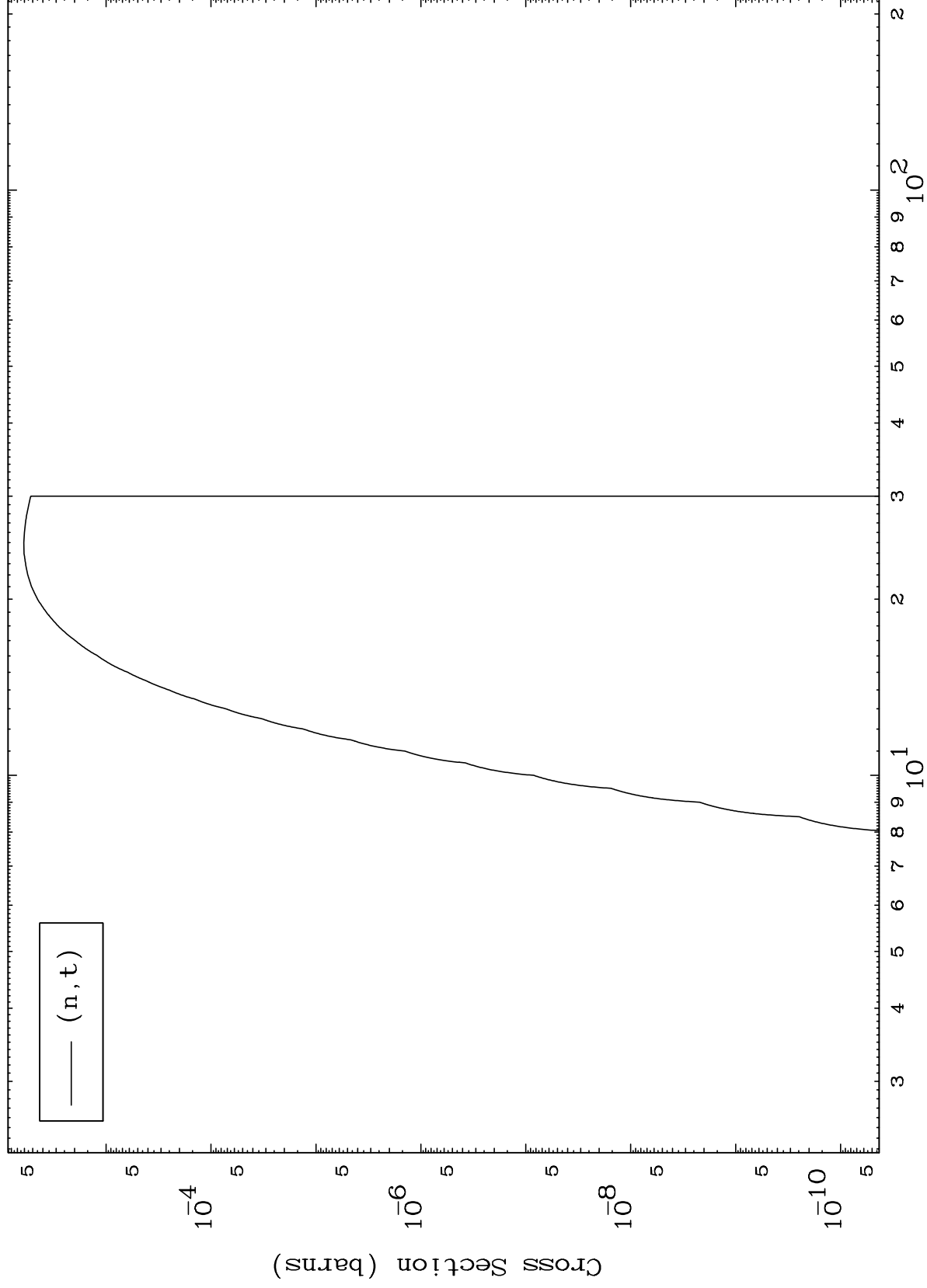
77-Ir-191n



MAT 7727

$^{77}\text{Ir-191n}$

(n,t) Levels
293 Kelvin Cross Sections



15

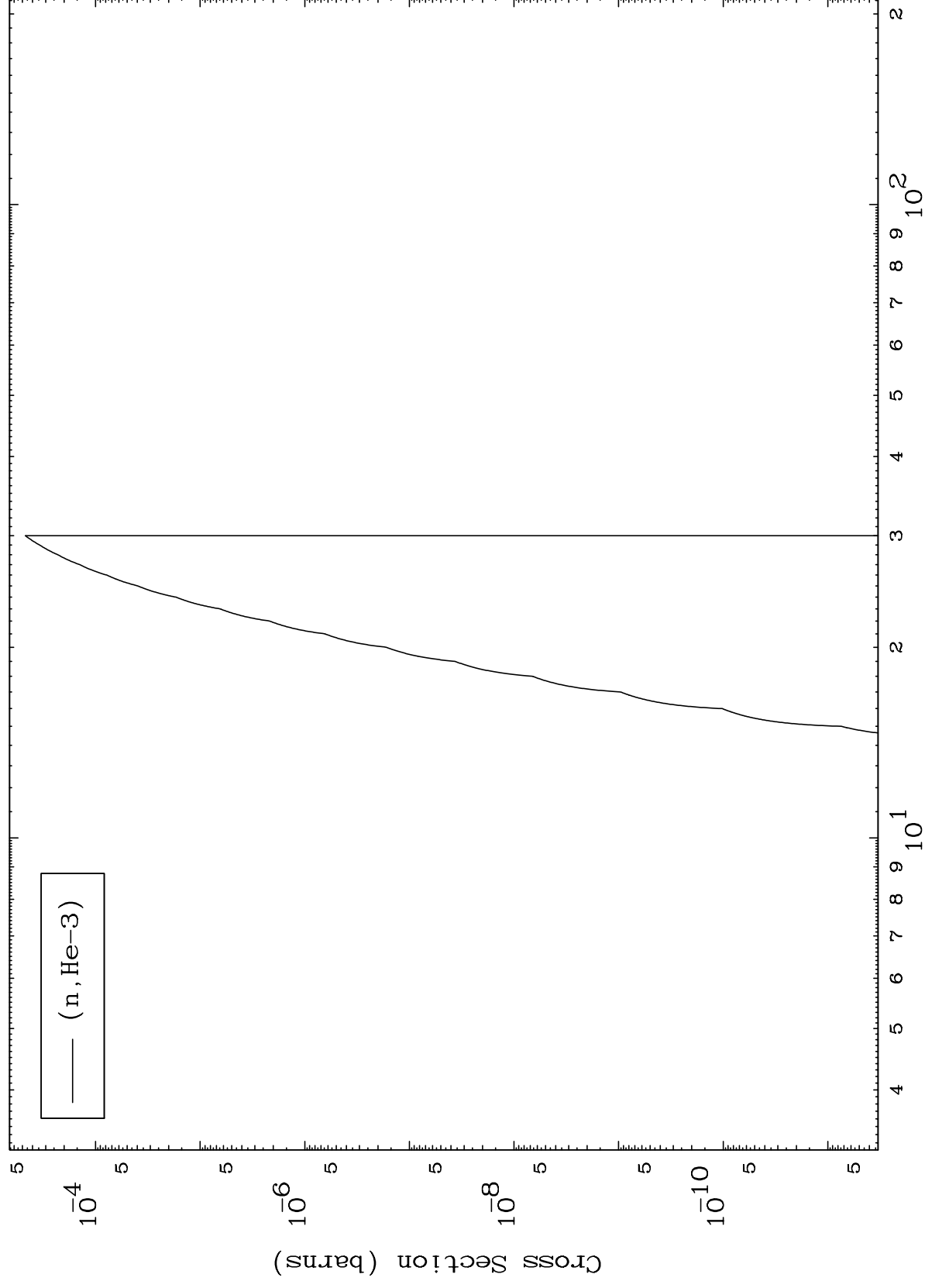
Incident Energy (MeV)

$^{77}\text{Ir-191n}$

MAT 7727

⁷⁷Ir-191n

(n,He3) Levels
293 Kelvin Cross Sections

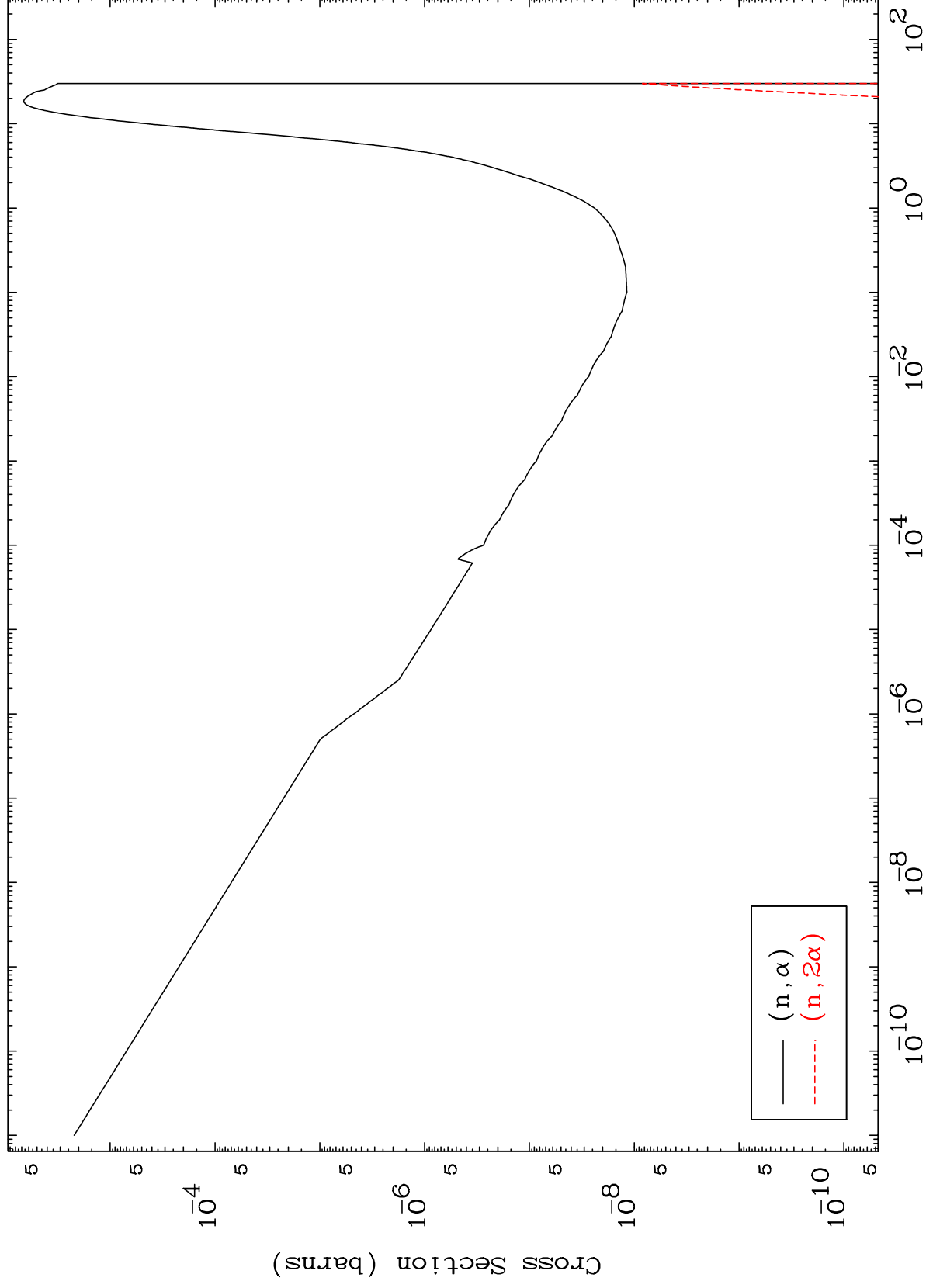


77-Ir-191n

MAT 7727

(n, α) Levels
293 Kelvin Cross Sections

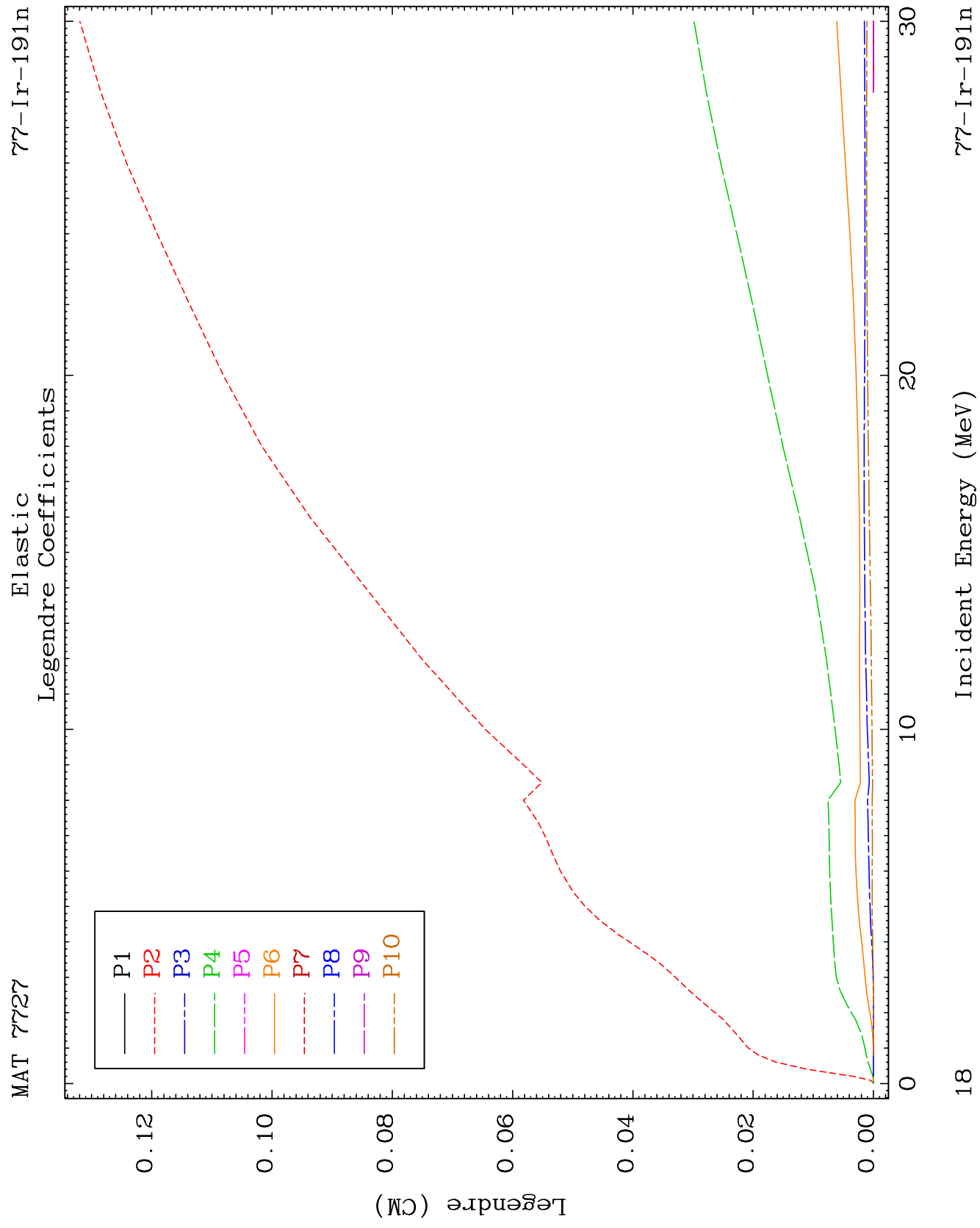
⁷⁷Ir-191n

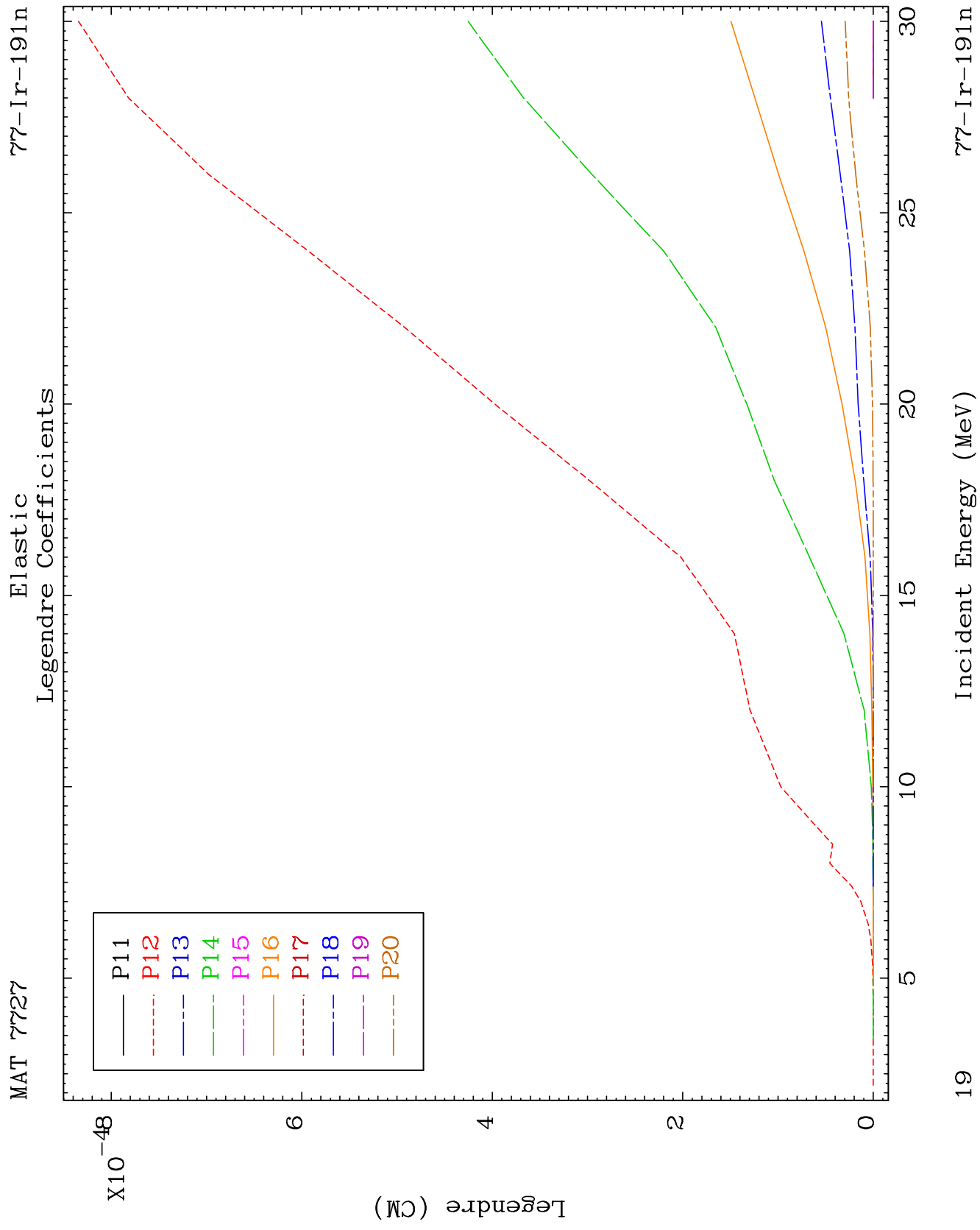


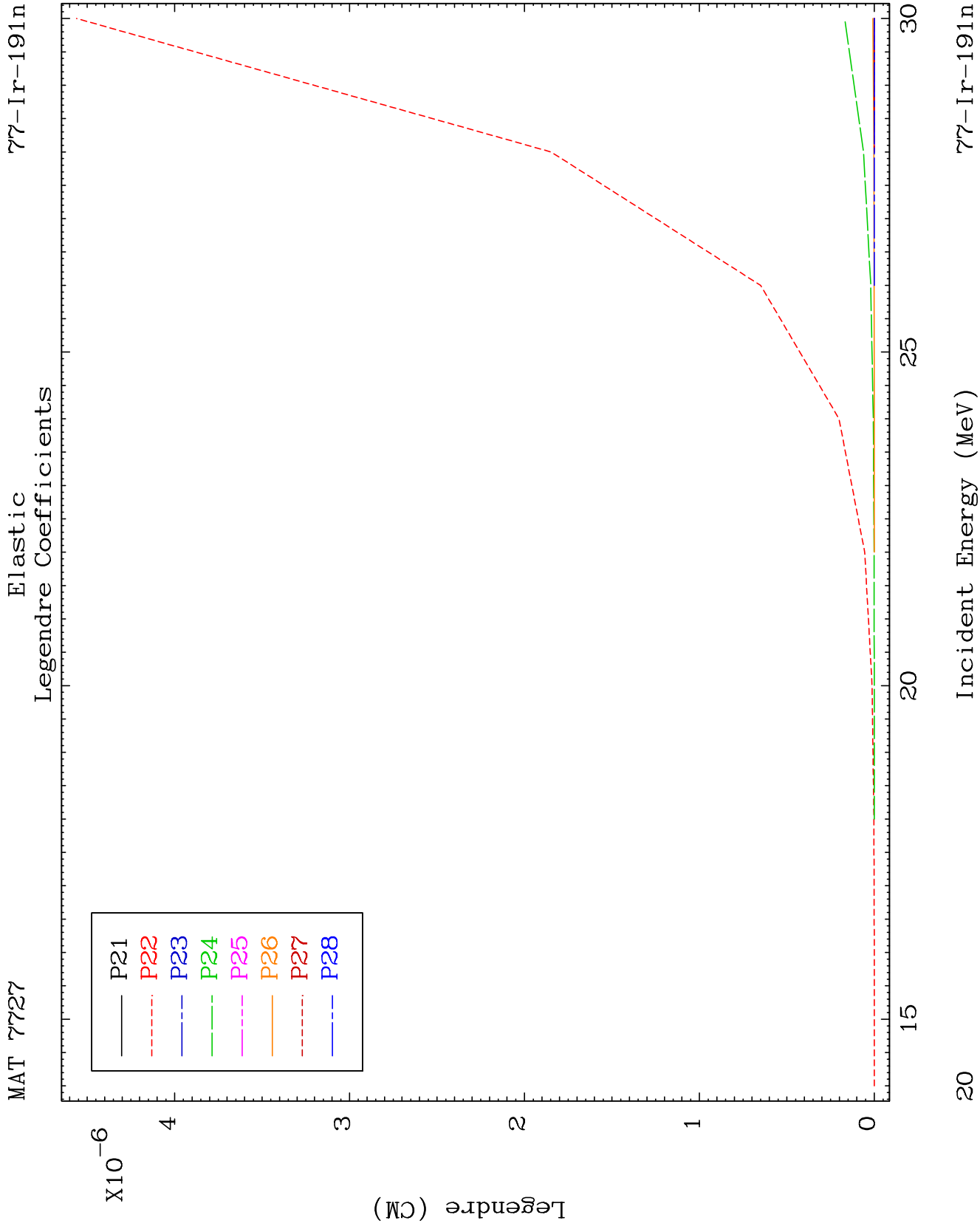
17

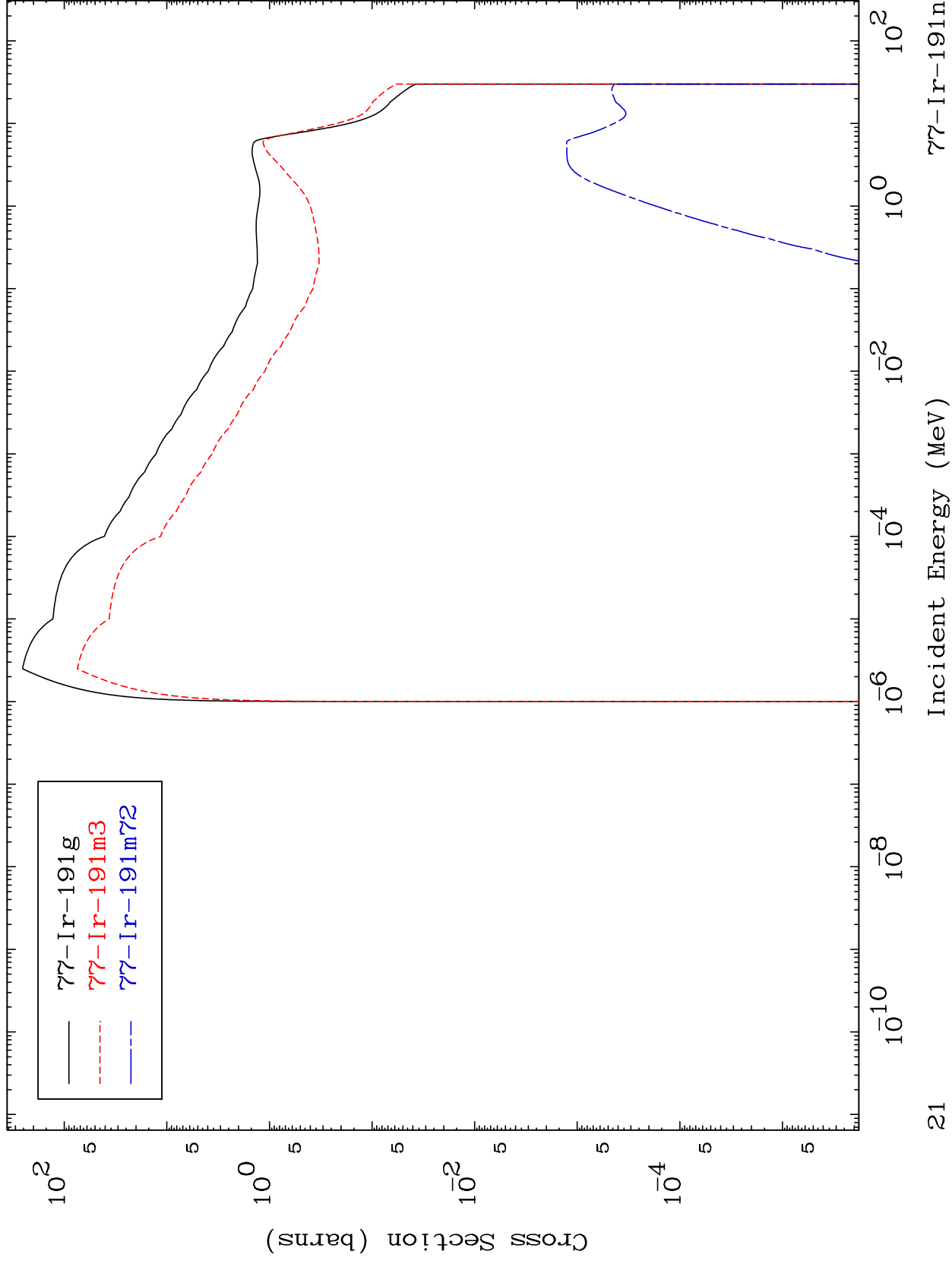
Incident Energy (MeV)

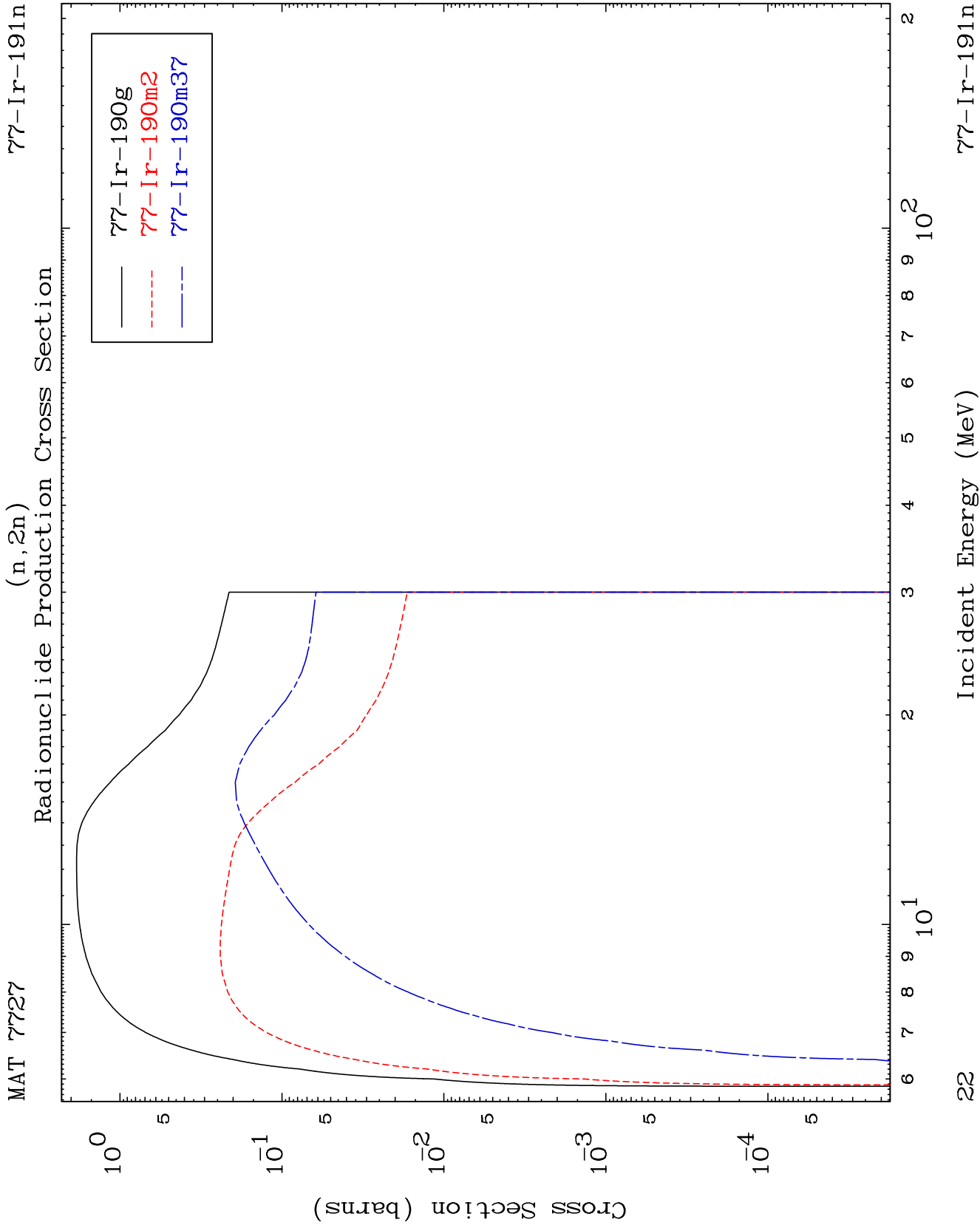
⁷⁷Ir-191n







Inelastic
Radionuclide Production Cross Section

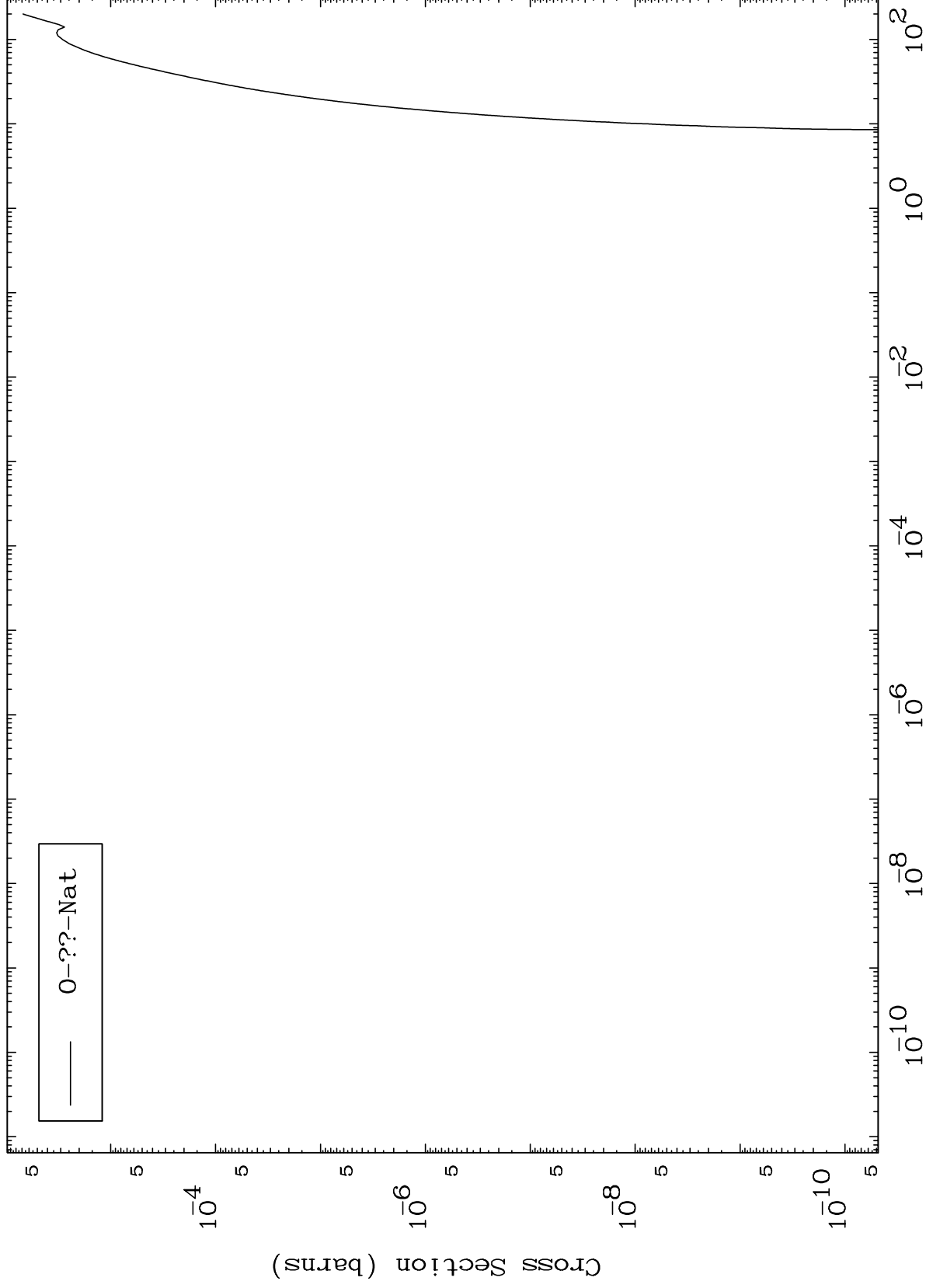


MAT 7727

Fission

$^{77}\text{Ir-191n}$

Radionuclide Production Cross Section

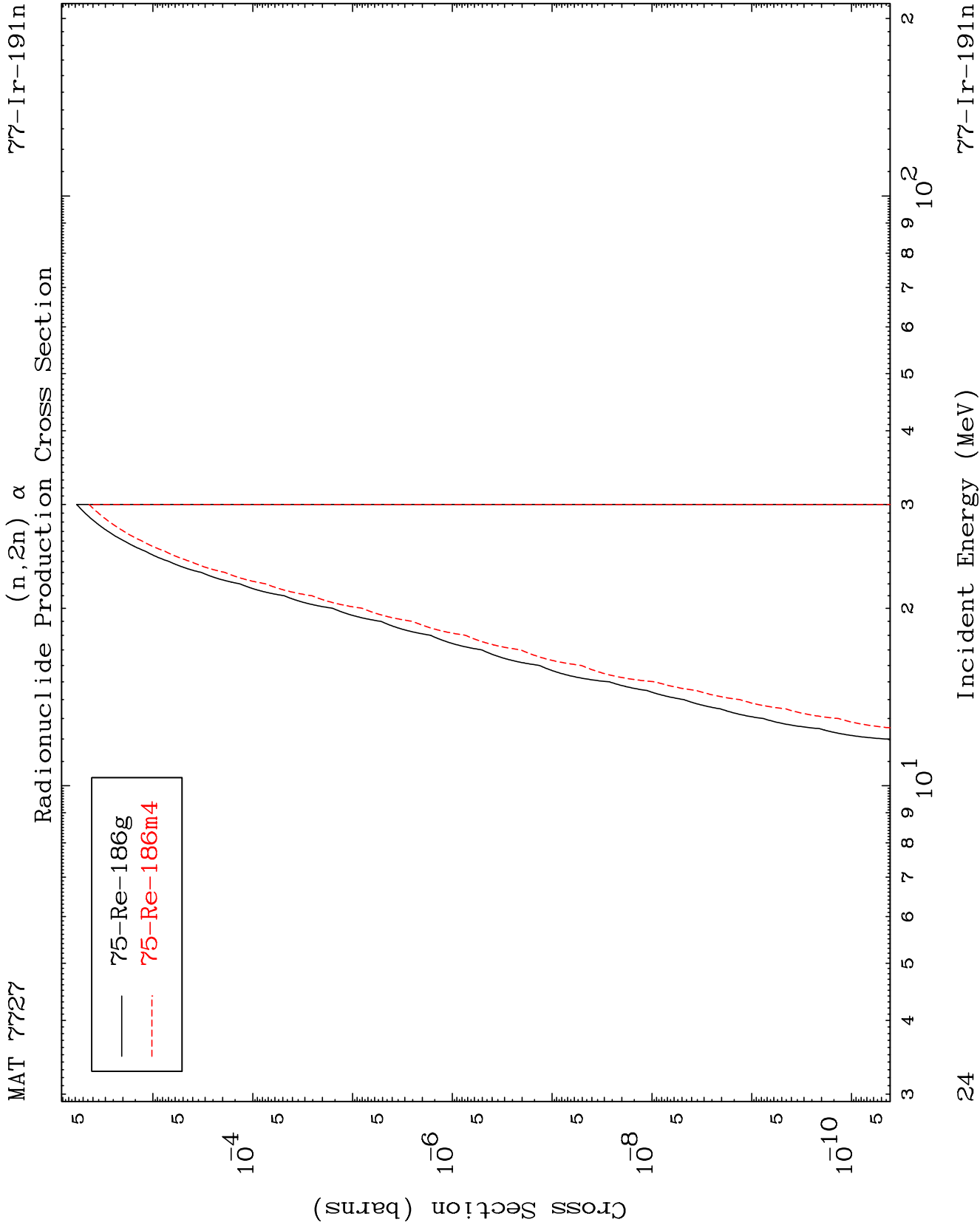


0-??-Nat

23

Incident Energy (MeV)

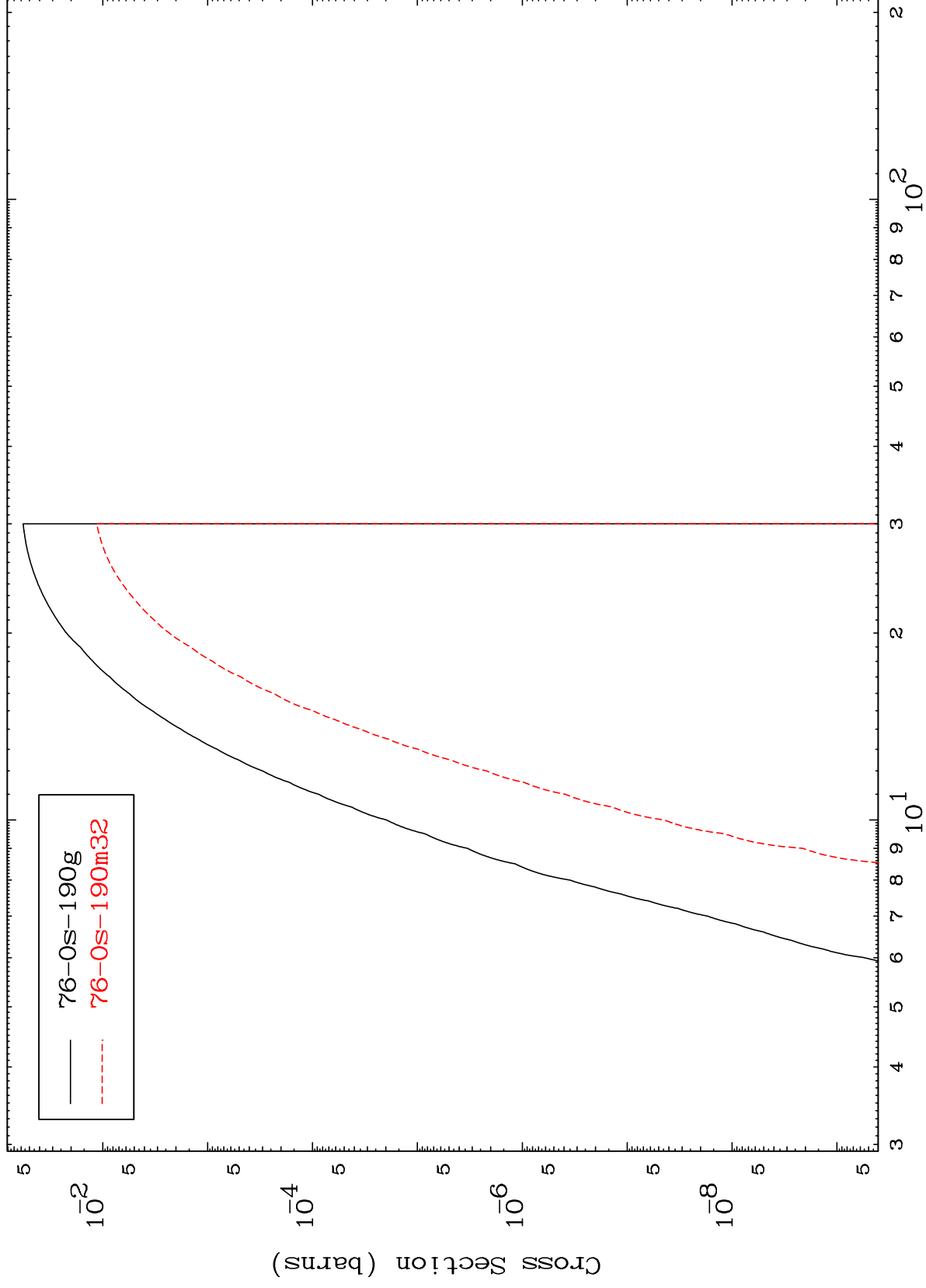
$^{77}\text{Ir-191n}$



MAT 7727

$^{77}\text{Ir-191n}$

$(n,n')\text{ p}$
Radionuclide Production Cross Section



25

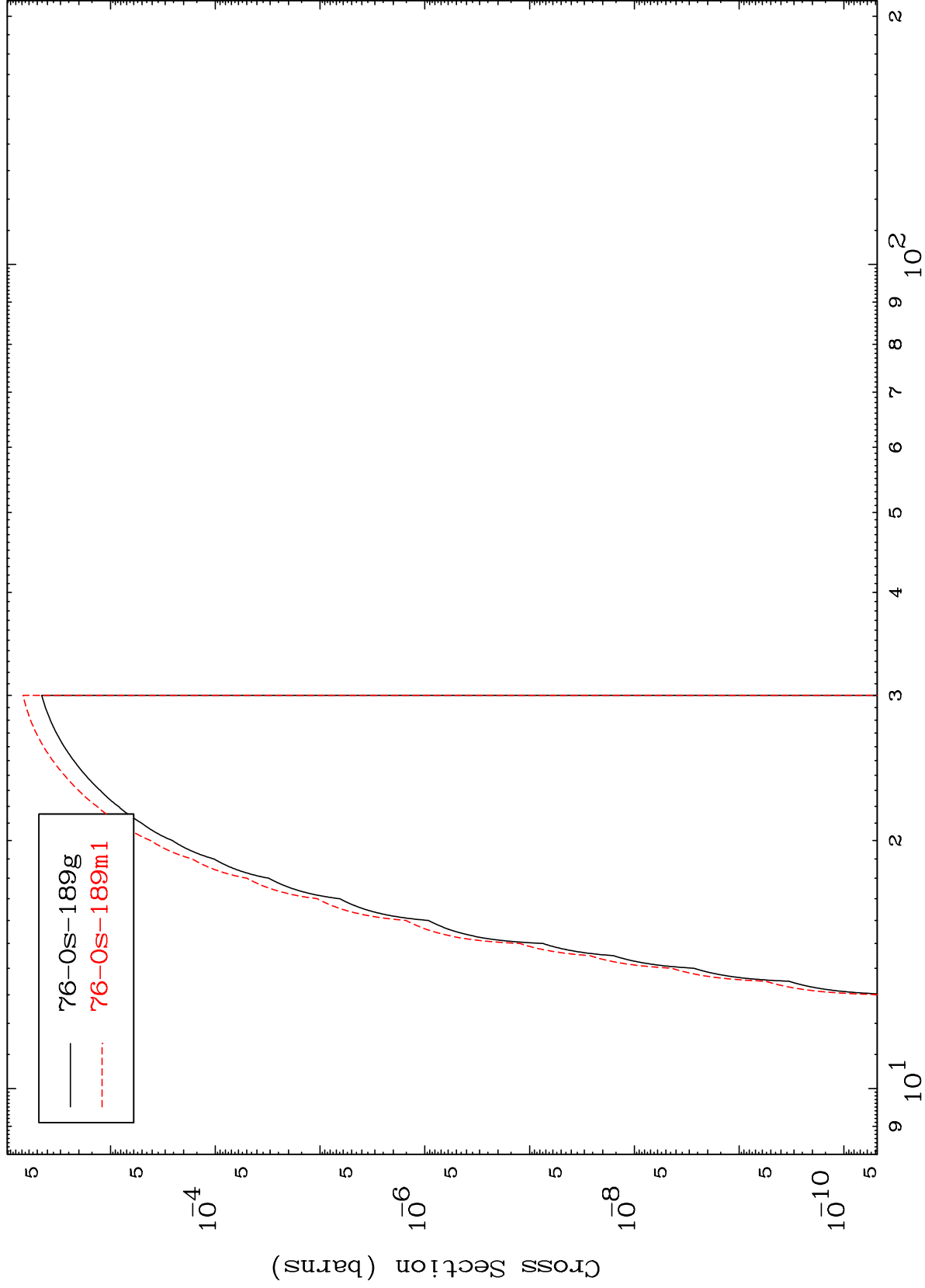
Incident Energy (MeV)

$^{77}\text{Ir-191n}$

MAT 7727

$^{77}\text{Ir-191n}$

(n,n') d
Radionuclide Production Cross Section

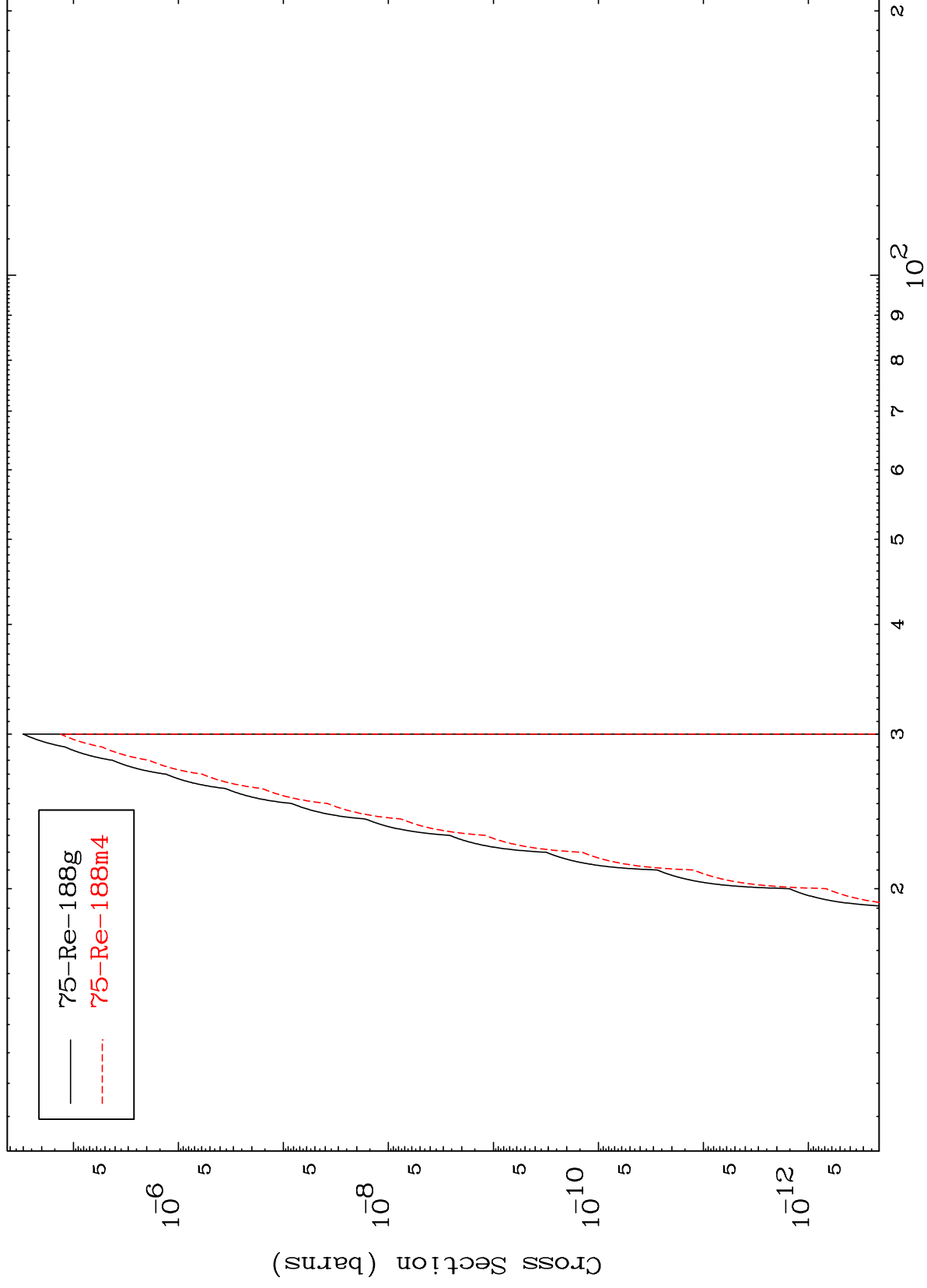


$^{77}\text{Ir-191n}$

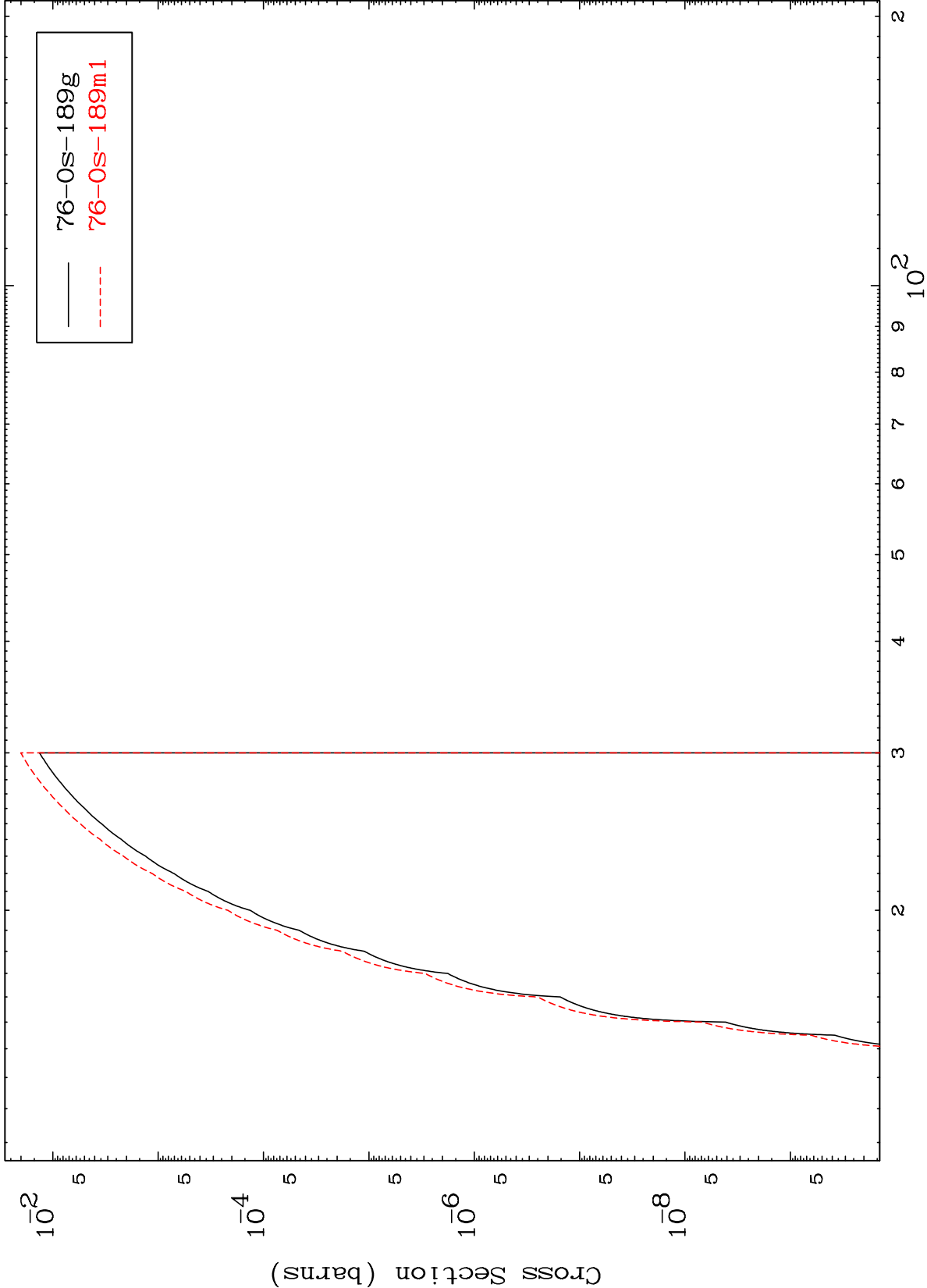
Incident Energy (MeV)

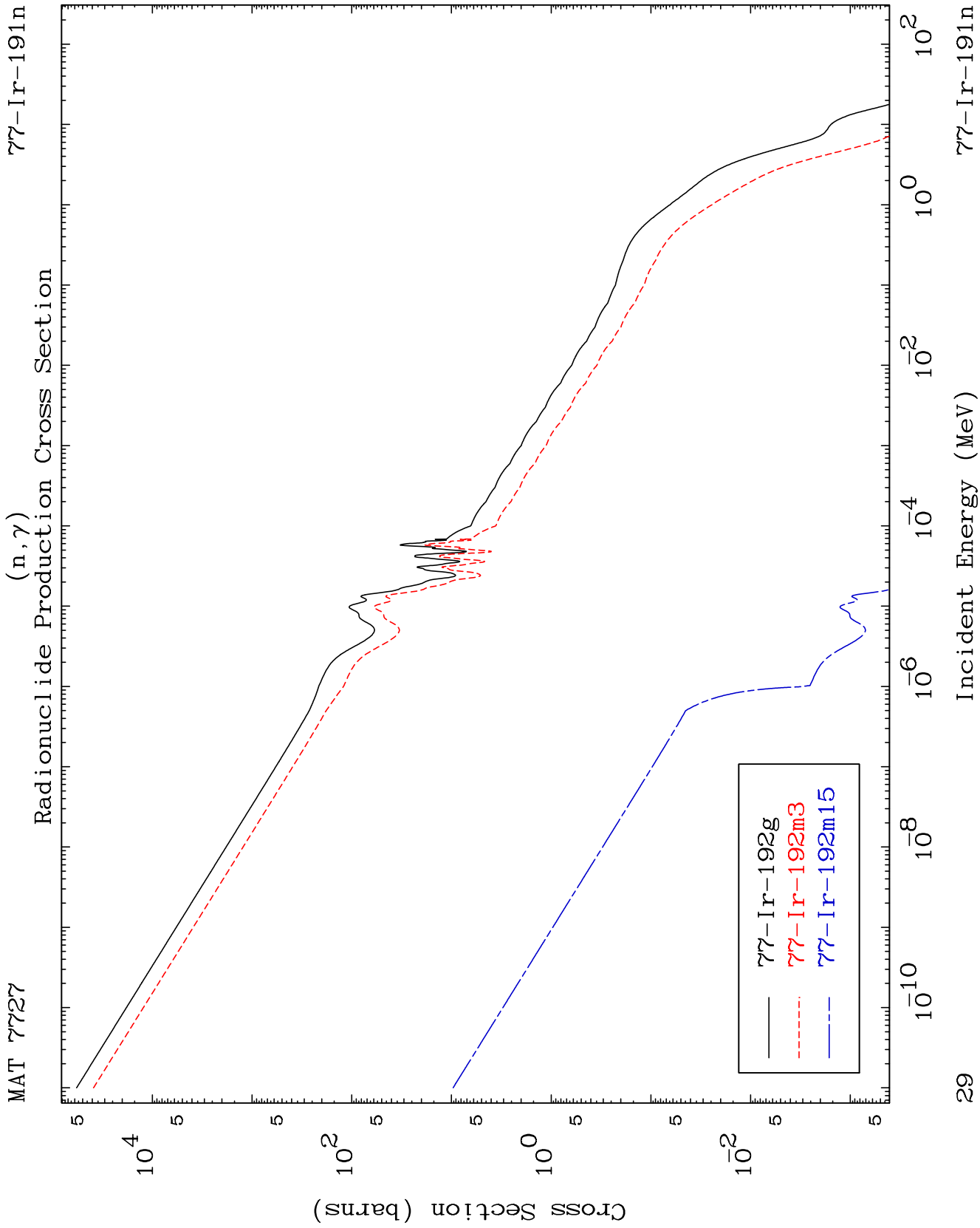
26

Radionuclide Production Cross Section



(n,2n) p
Radionuclide Production Cross Section

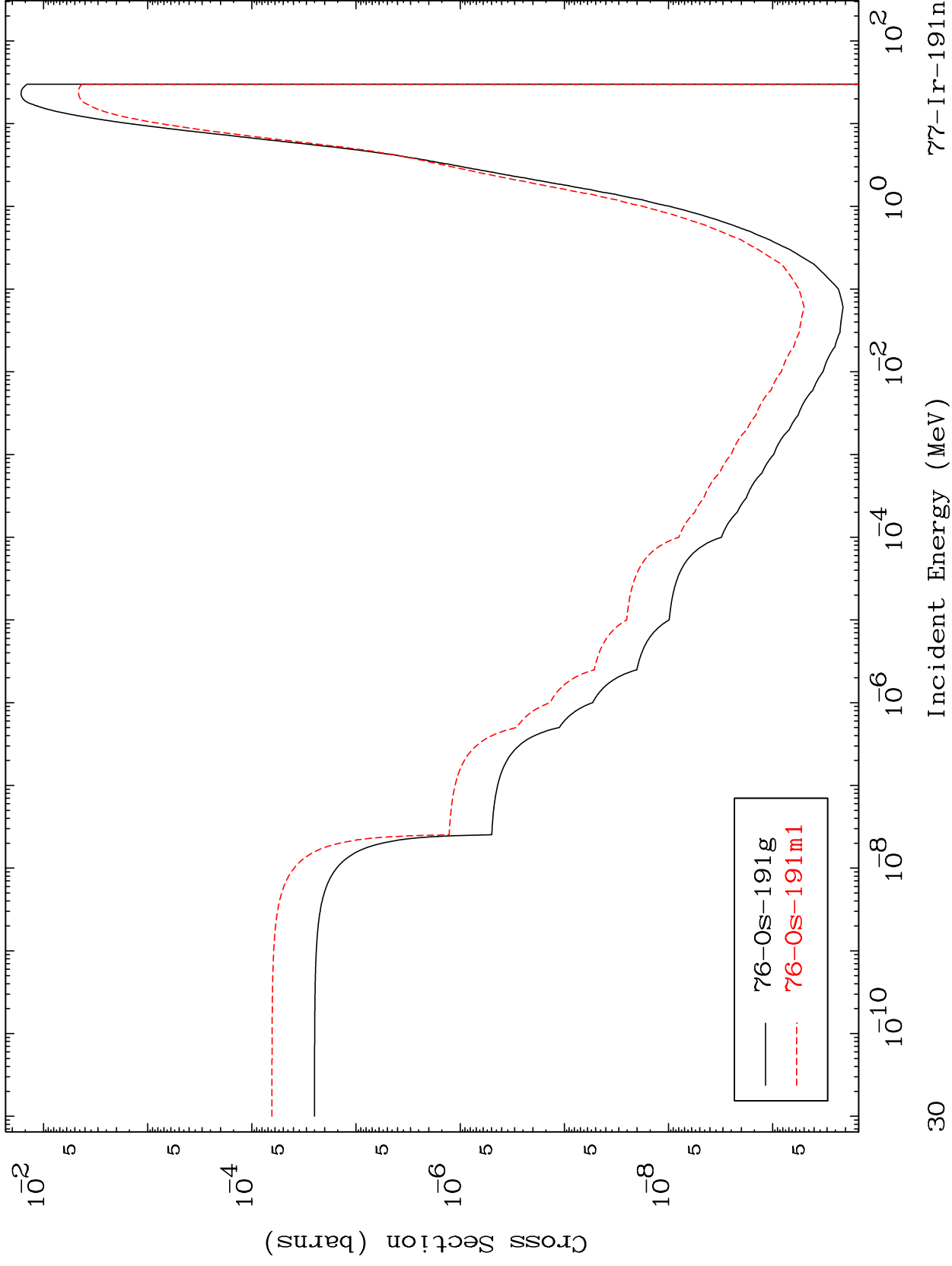




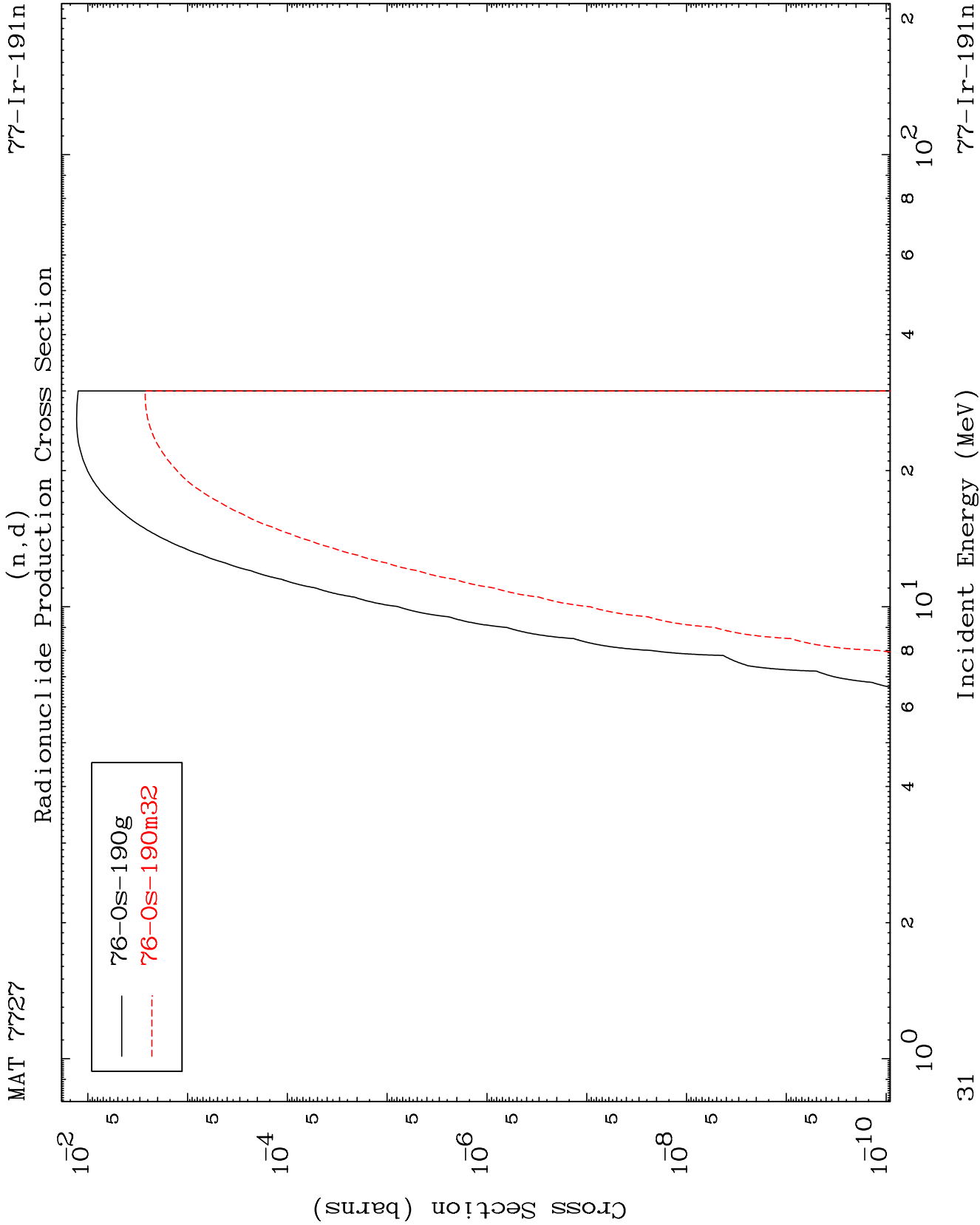
MAT 7727

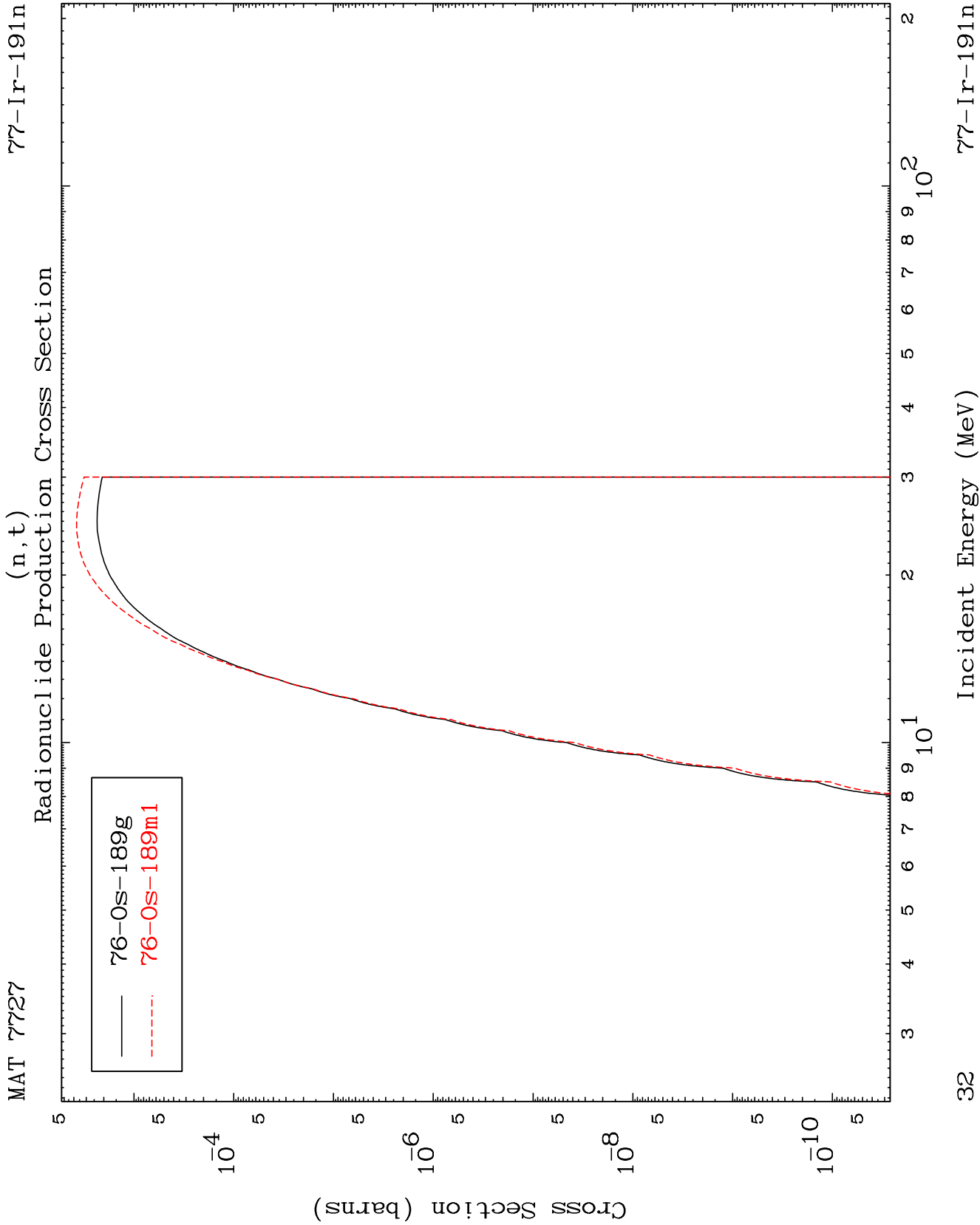
$^{77}\text{Ir-191n}$

Radionuclide Production Cross Section



30

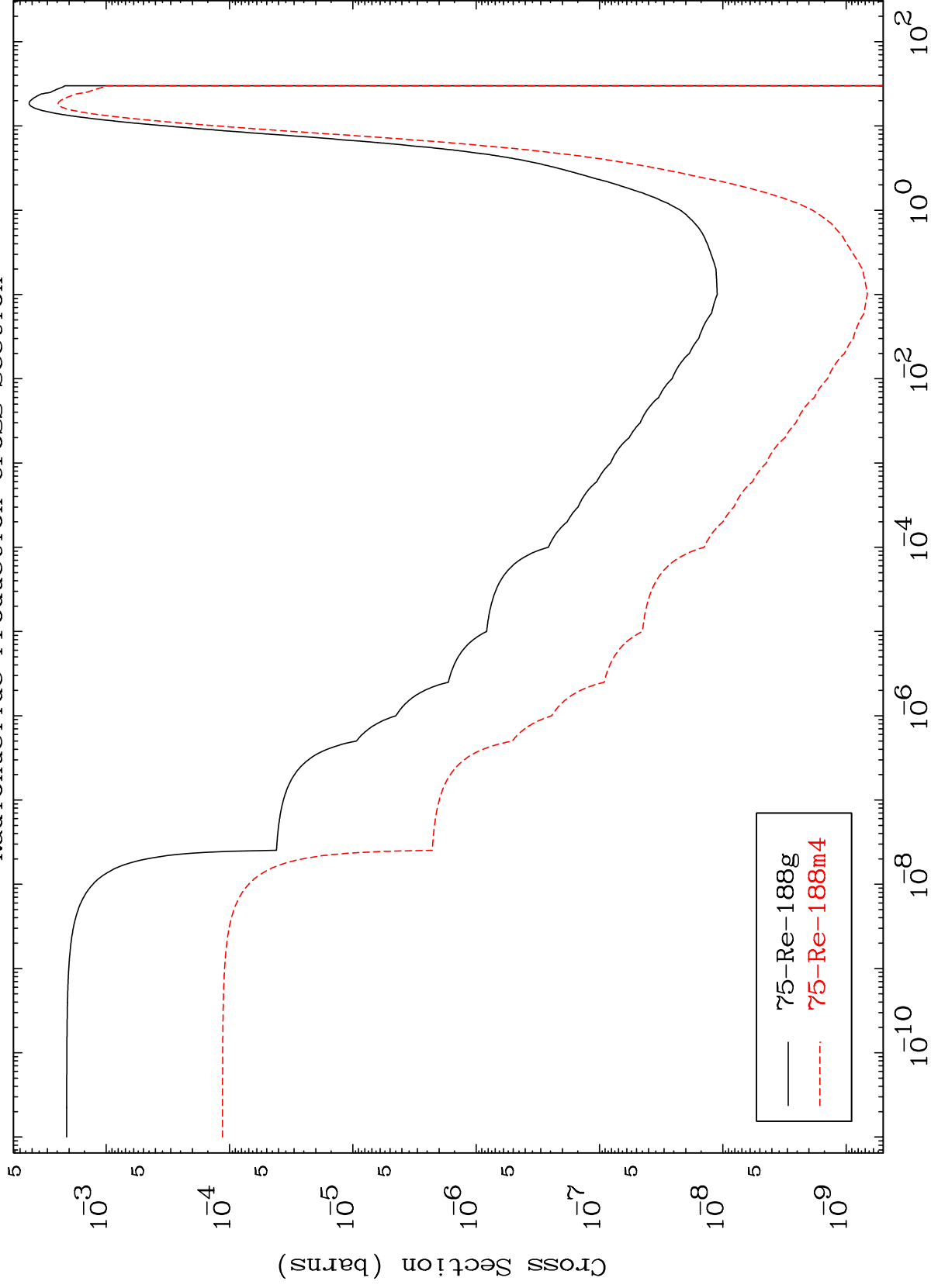




MAT 7727

$^{77}\text{Ir-191n}$

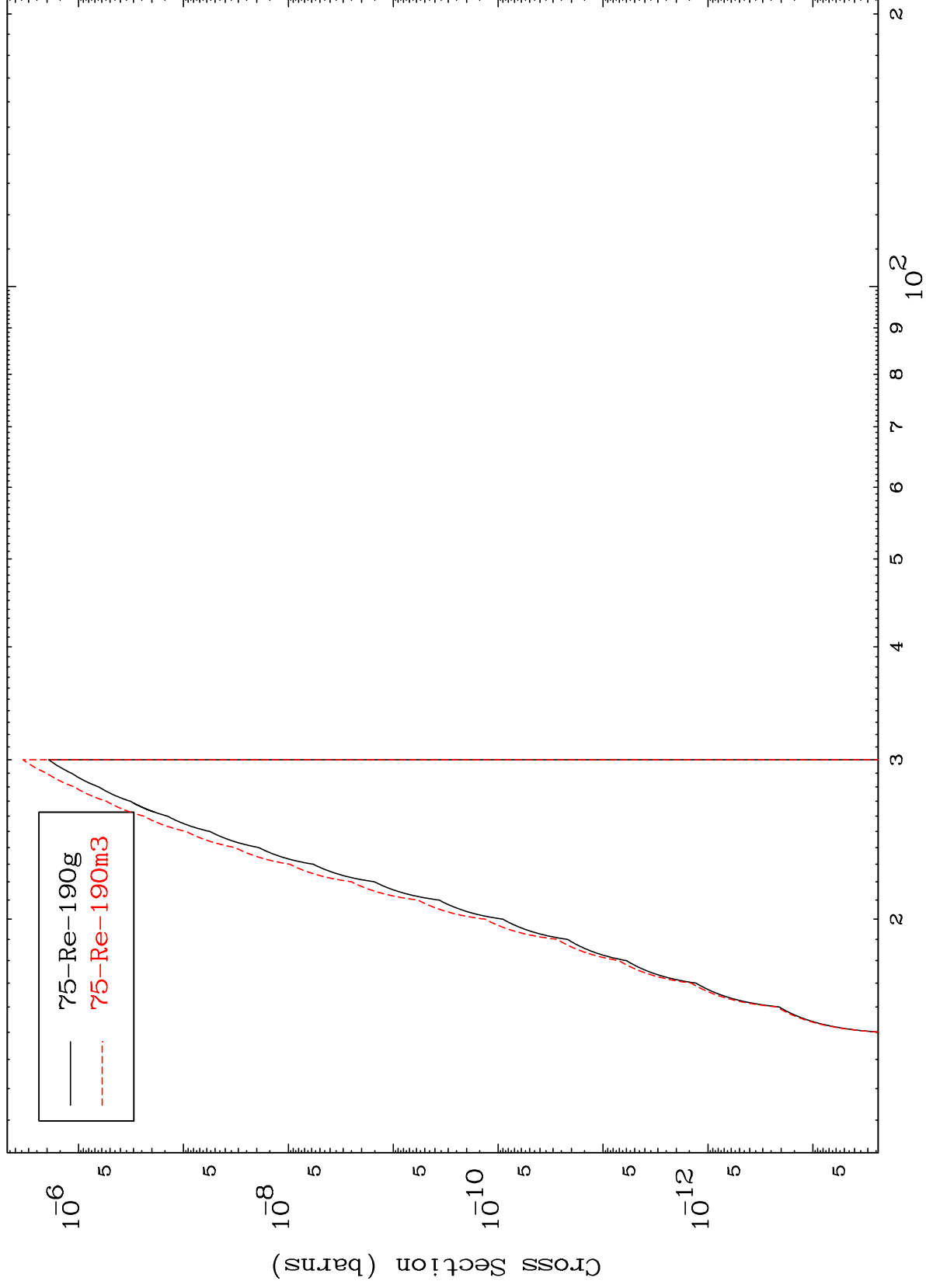
Radionuclide Production Cross Section



33

$^{77}\text{Ir-191n}$

Radionuclide Production Cross Section (n,2p)

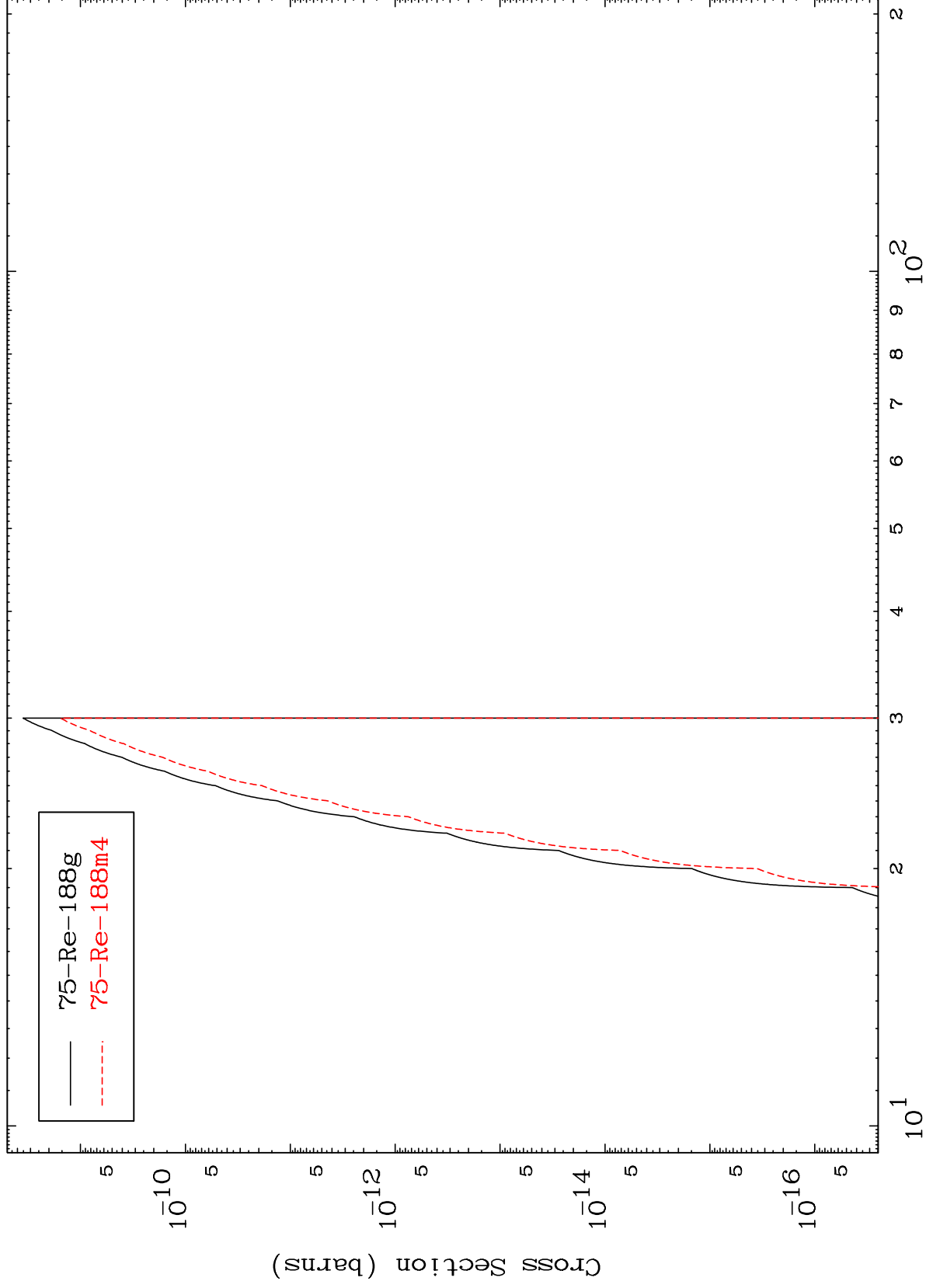


MAT 7727

(n,p) t

$^{77}\text{Ir-191n}$

Radionuclide Production Cross Section



Incident Energy (MeV)

$^{77}\text{Ir-191n}$

35