

Program EVALPLOT
(Version 2021-1)

by

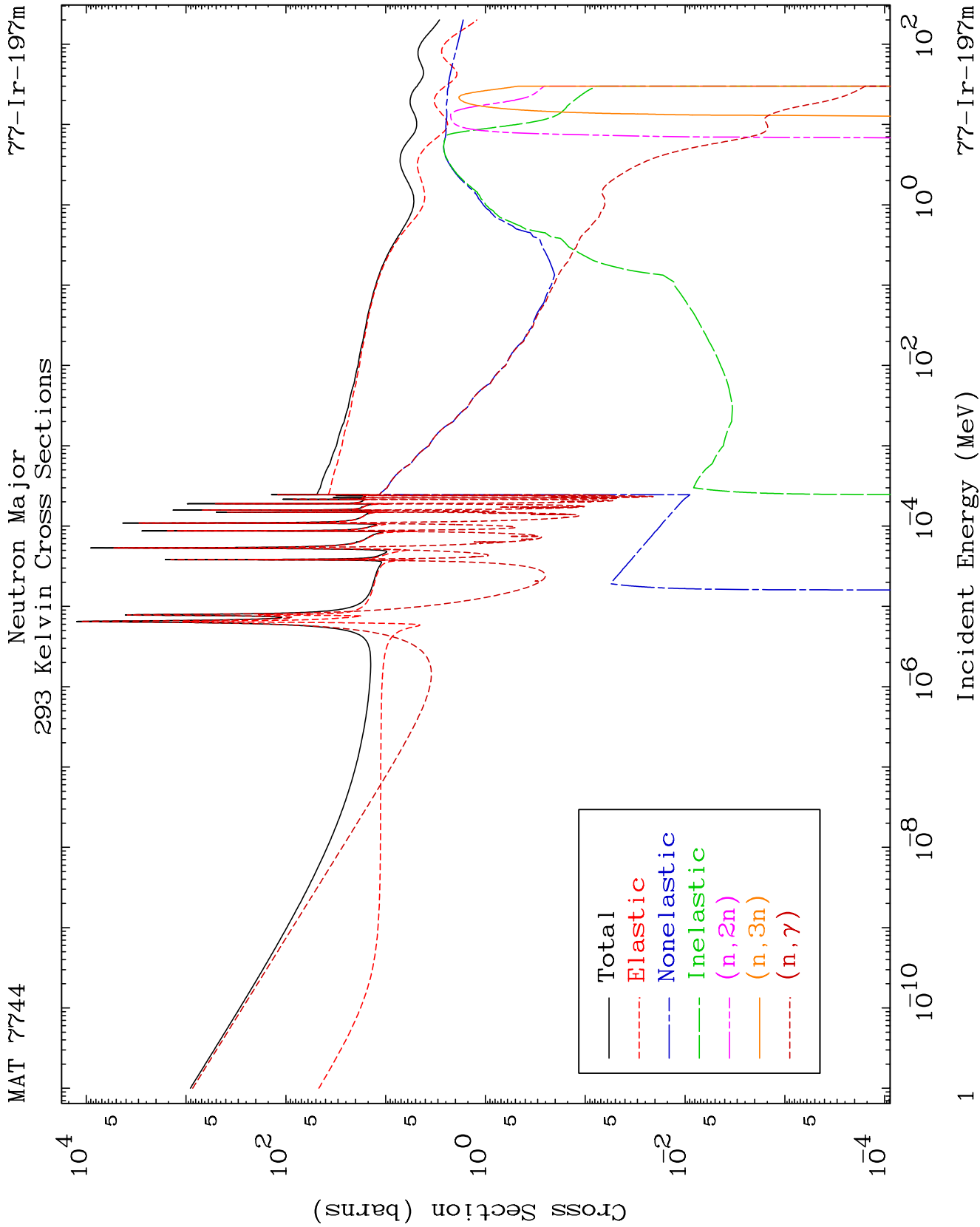
Dermott E. Cullen
(Present Contact Information)

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Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

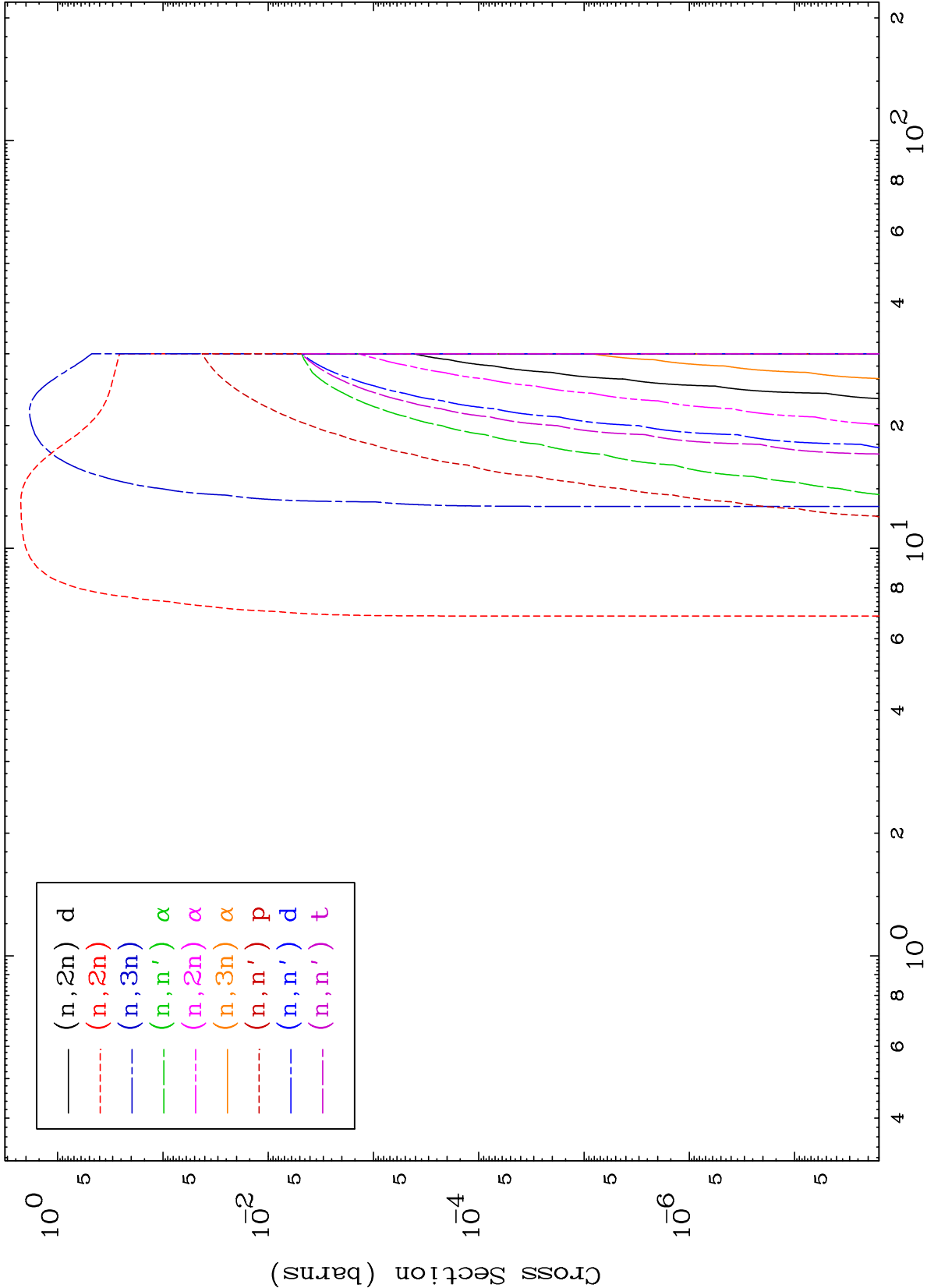


MAT 7744

Neutron Absorption

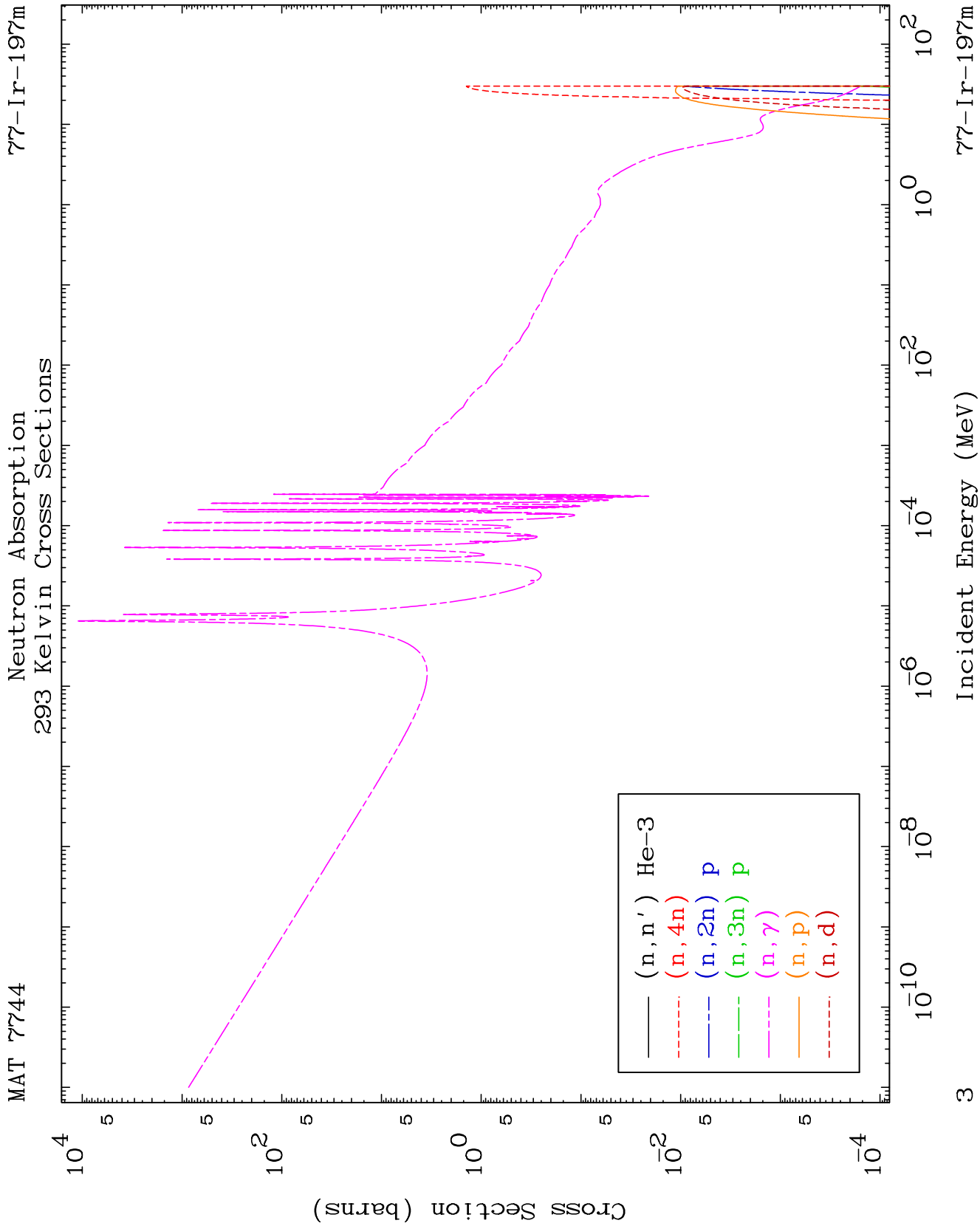
77-Ir-197m

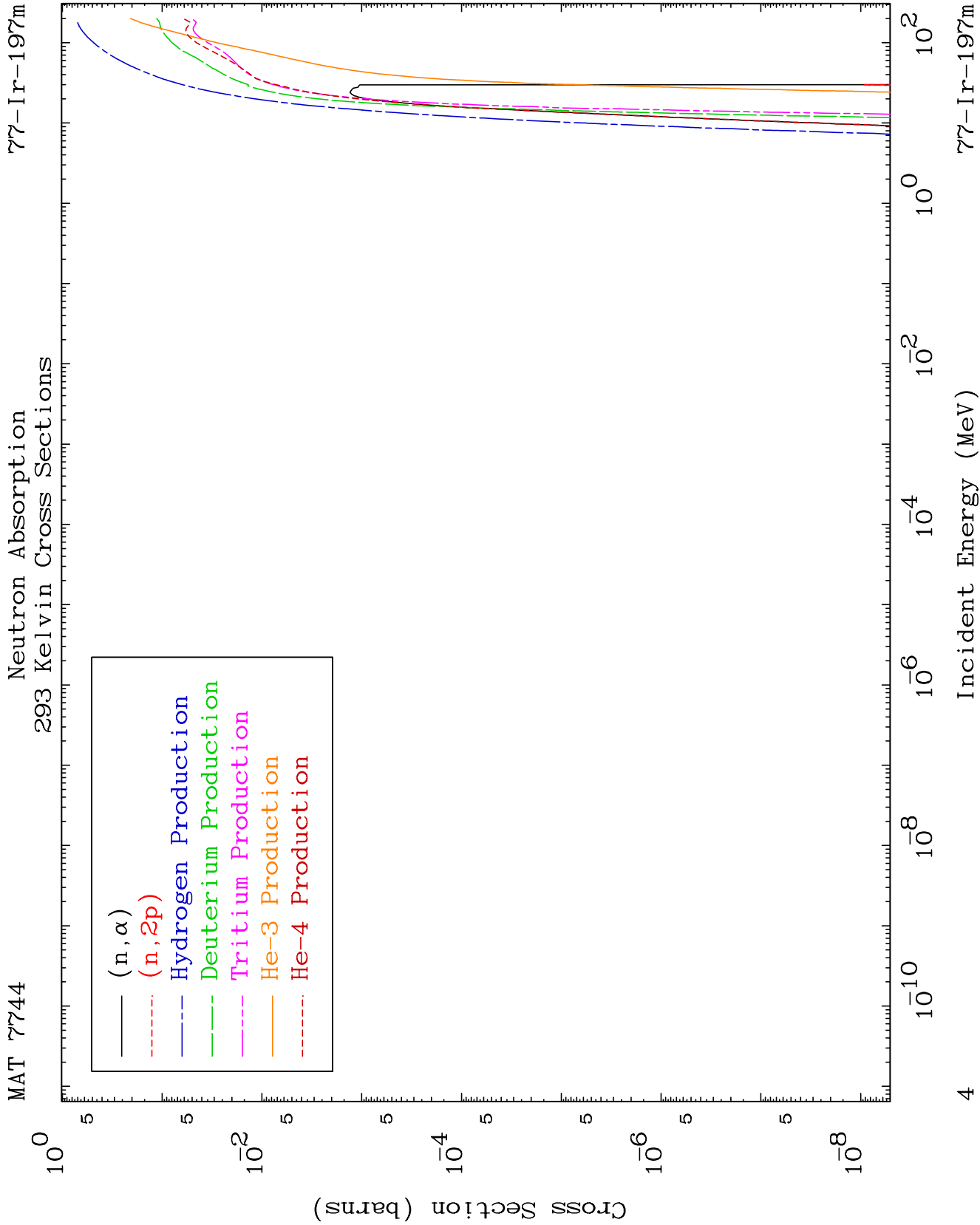
293 Kelvin Cross Sections

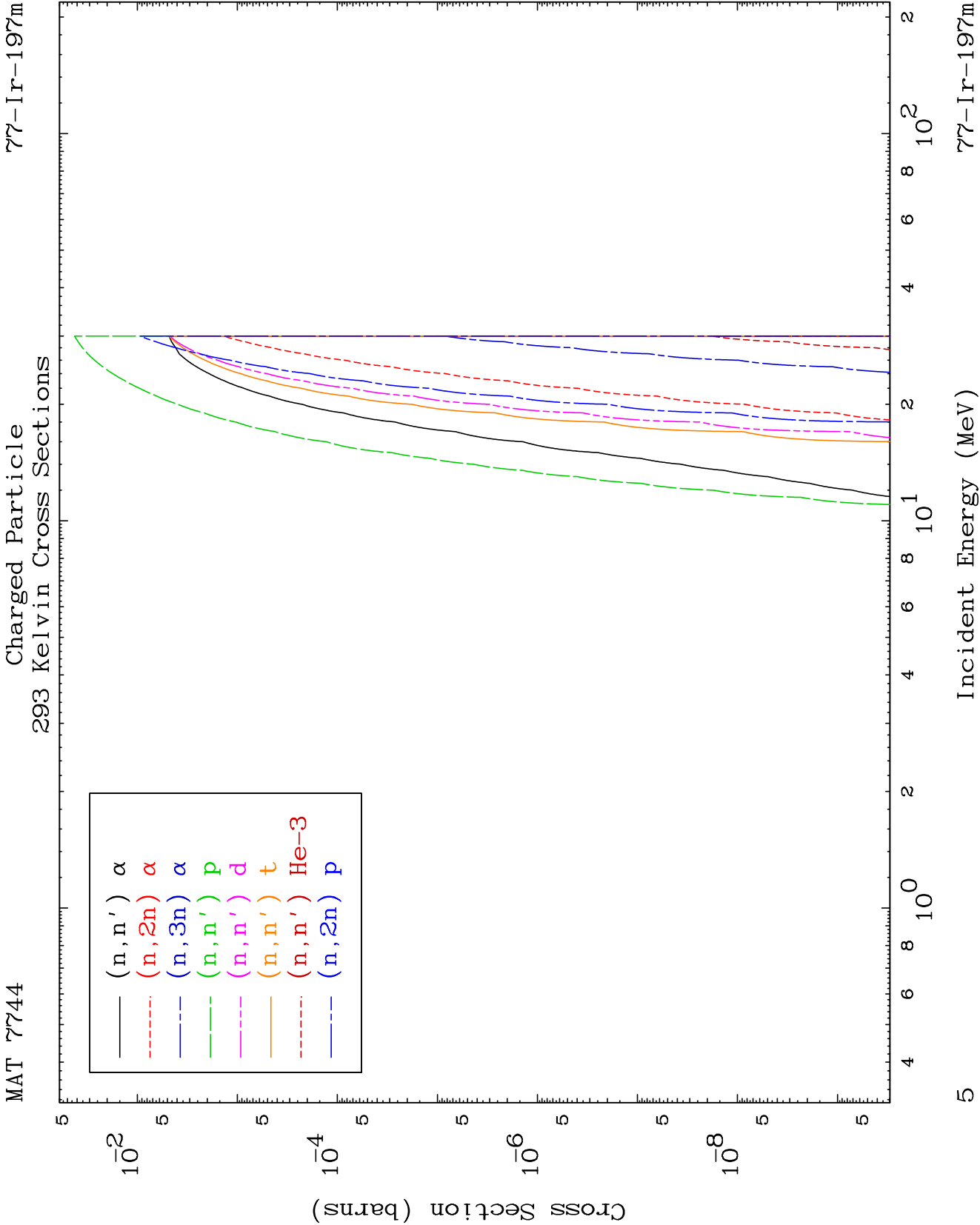


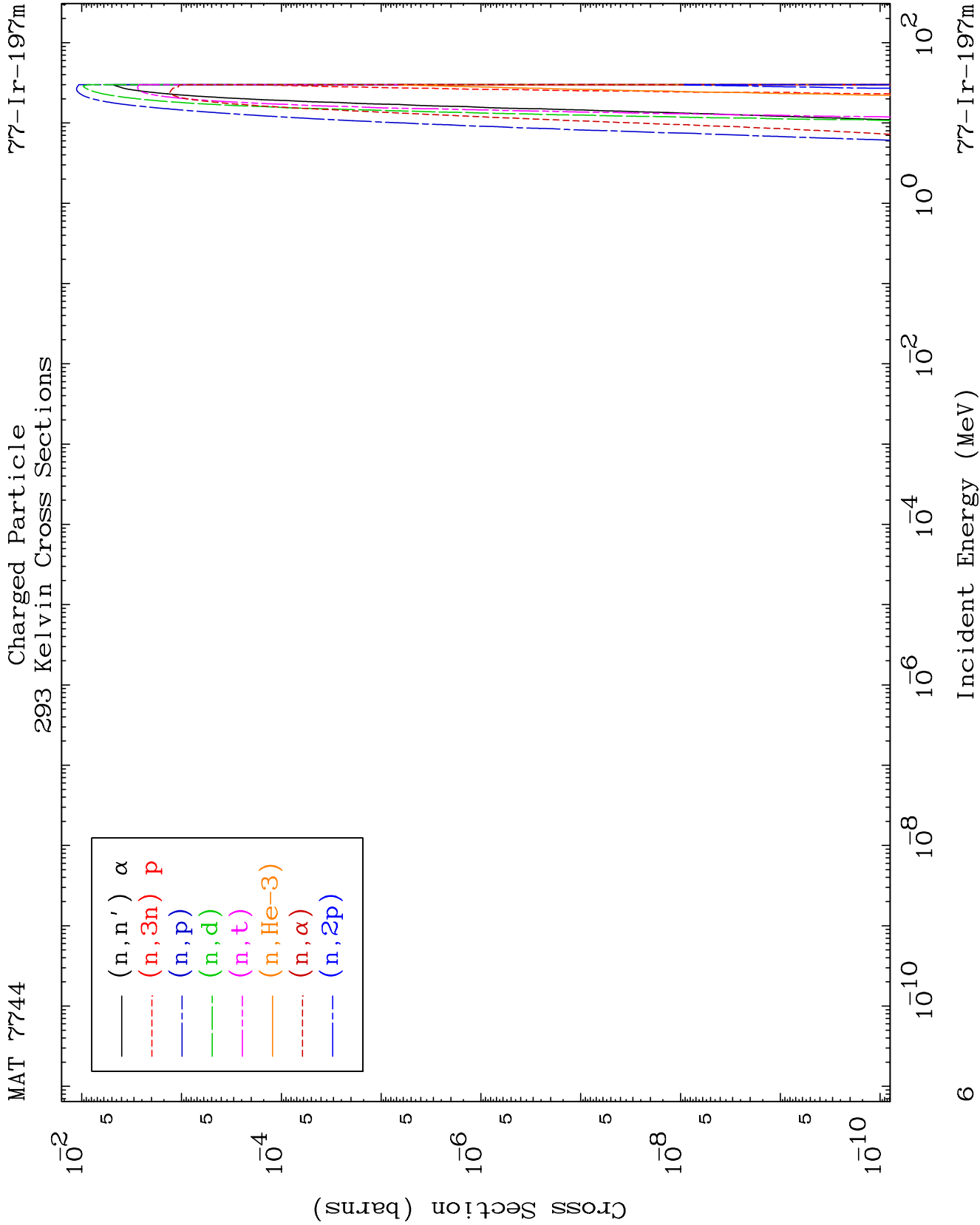
Incident Energy (MeV)

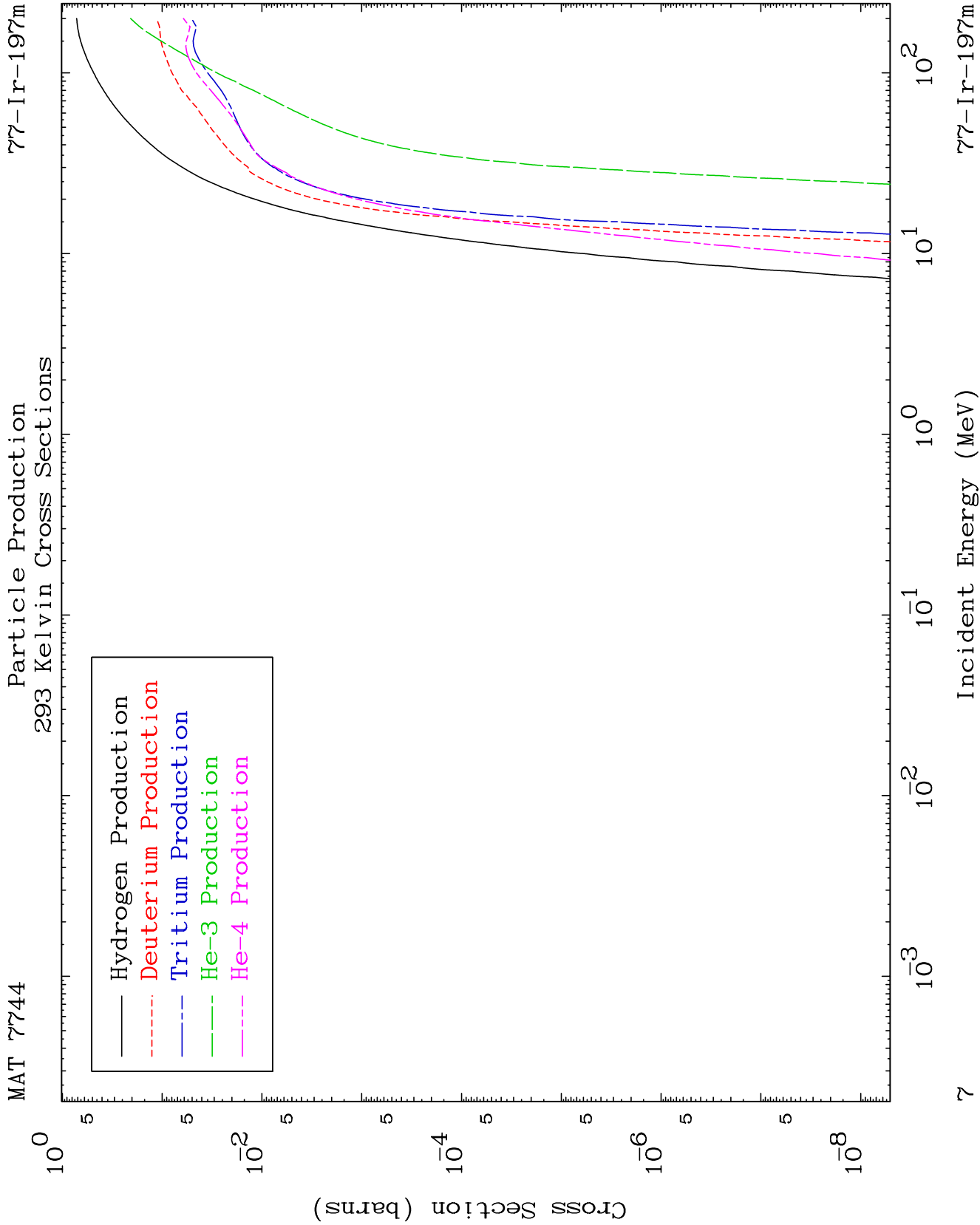
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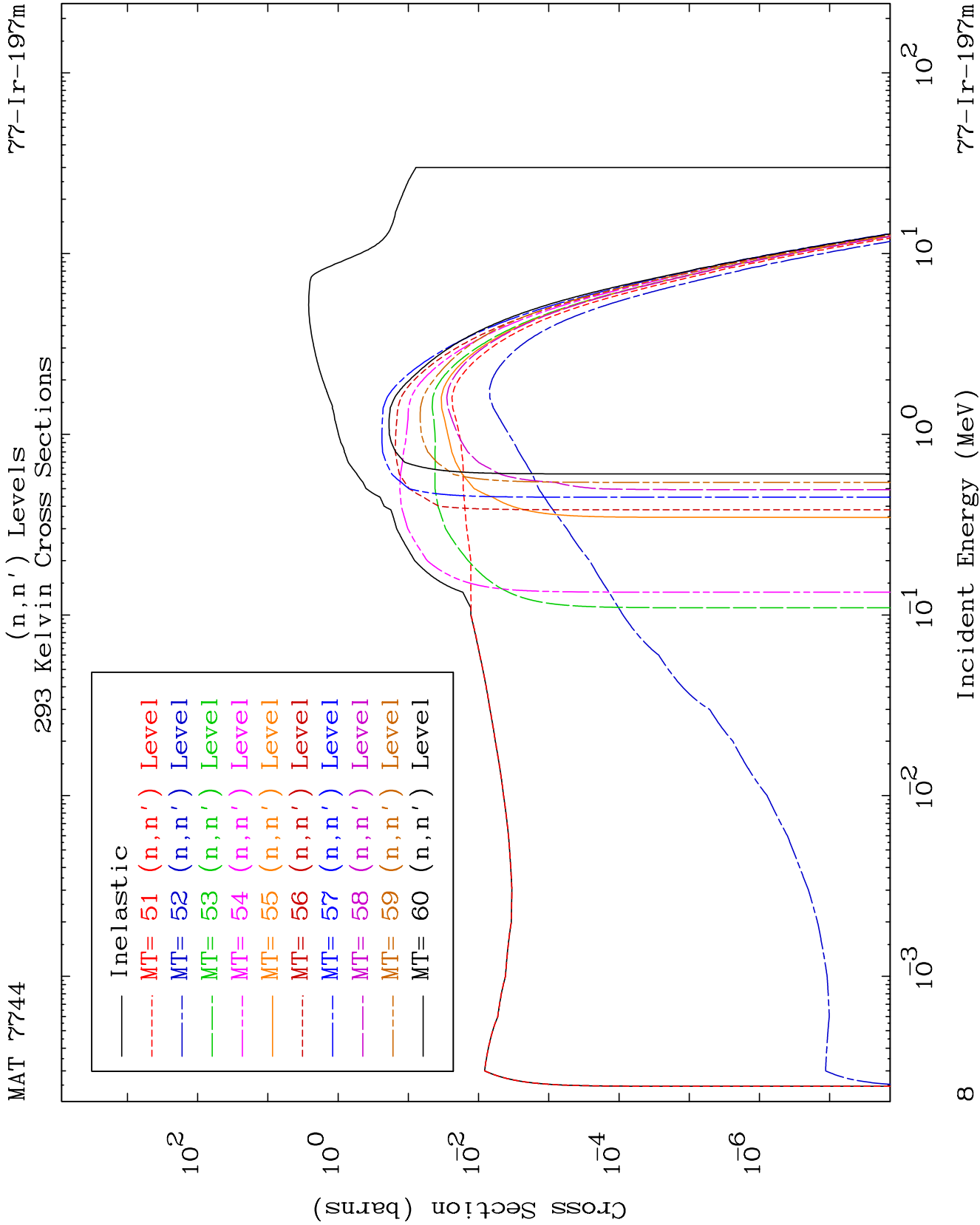


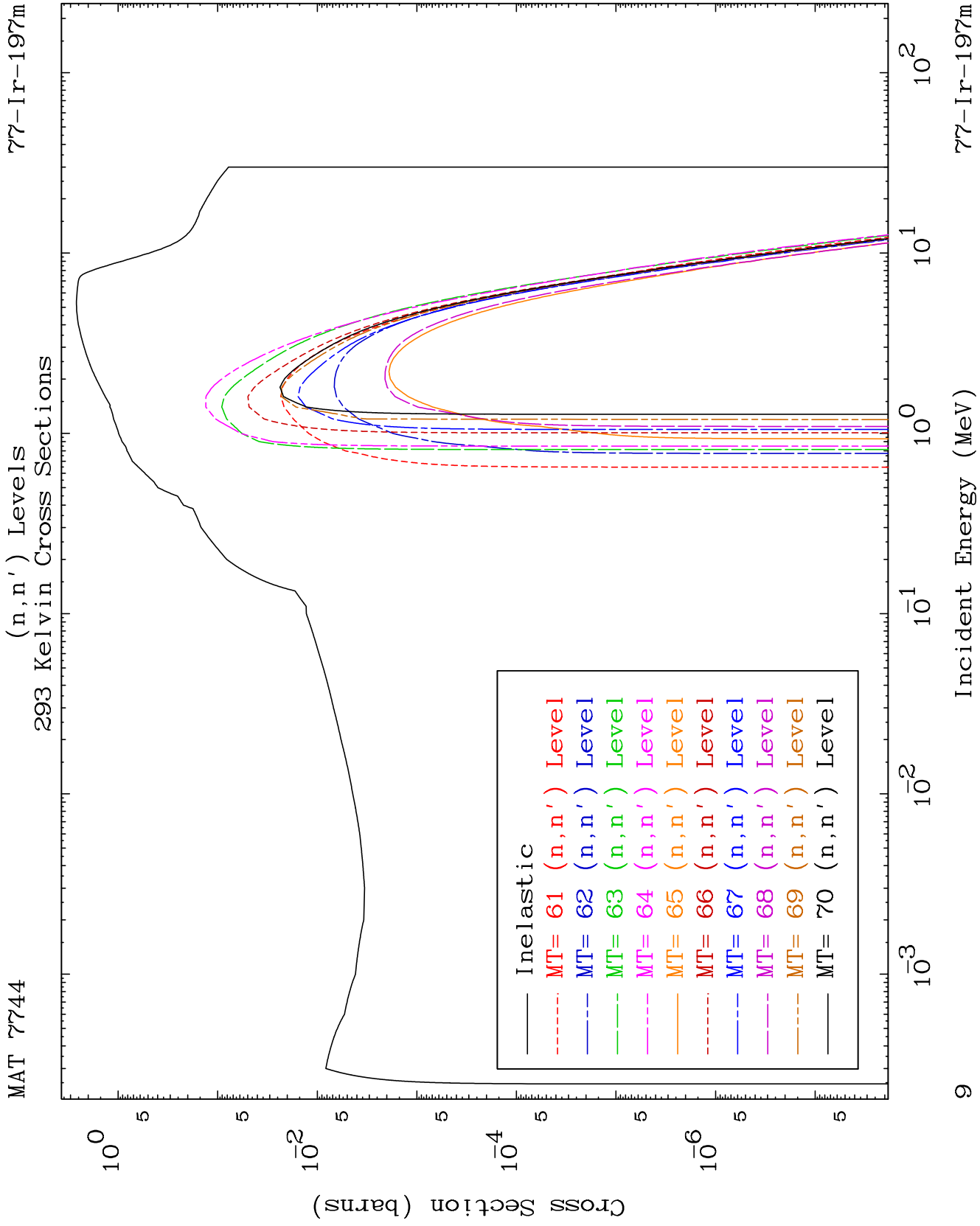


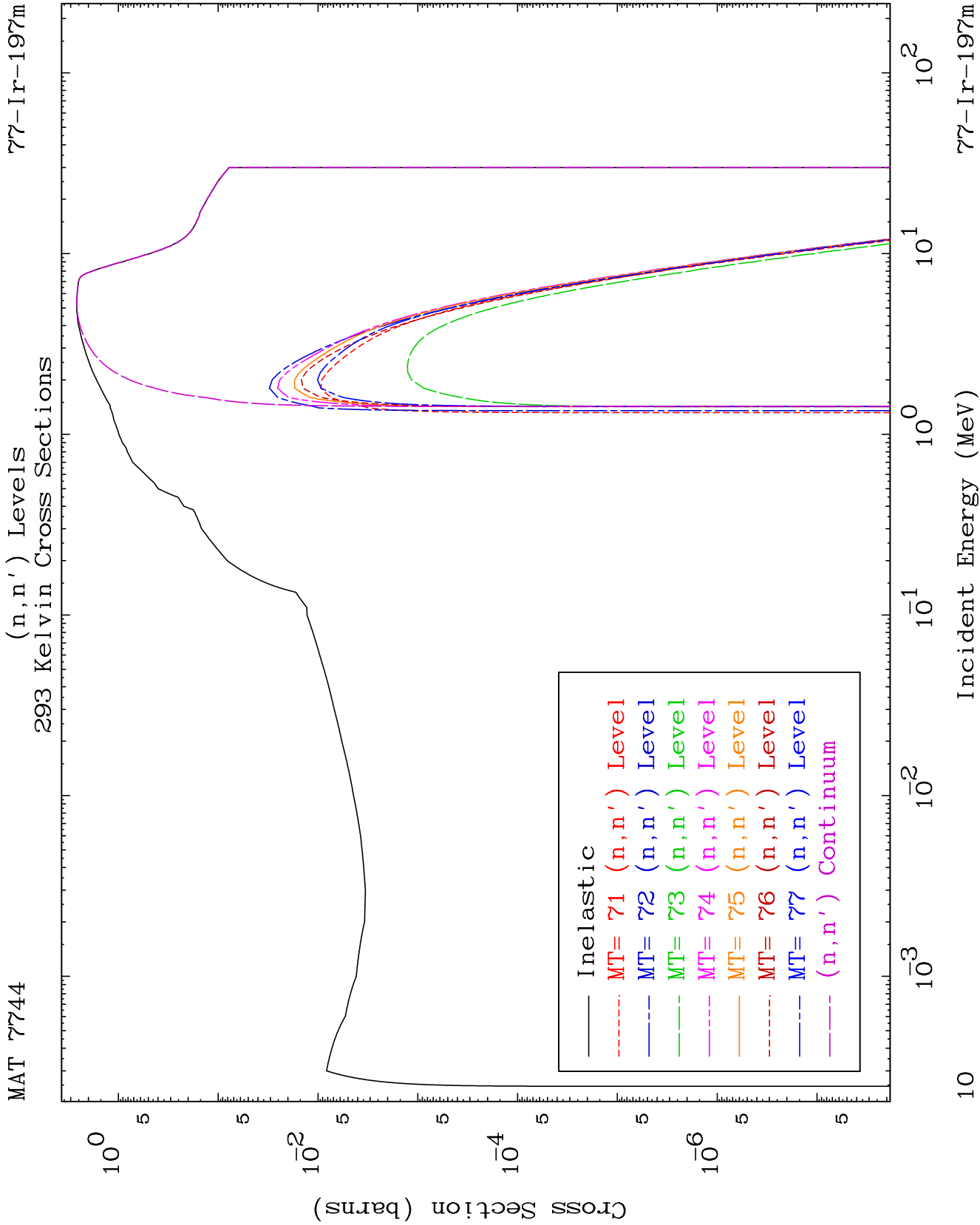


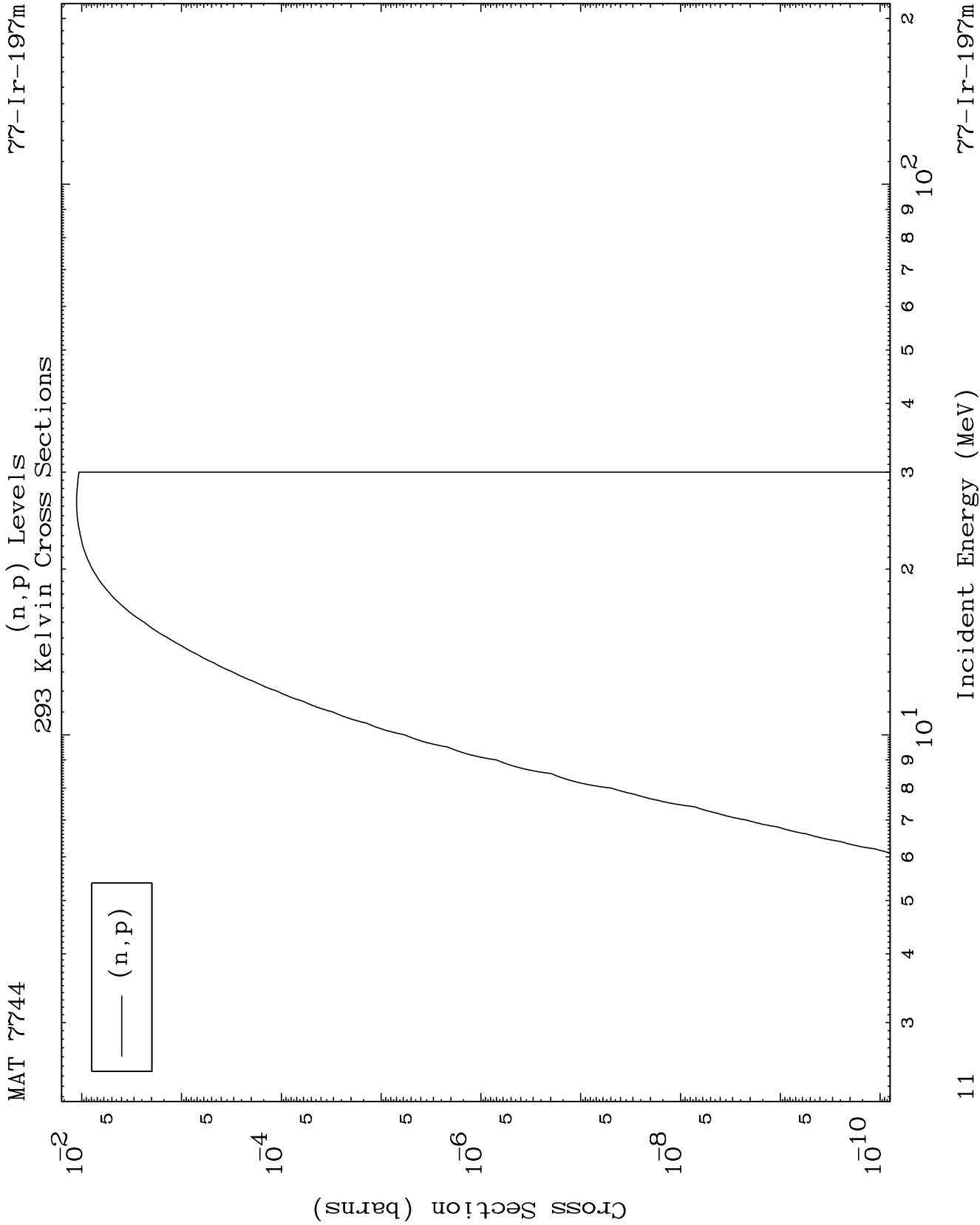










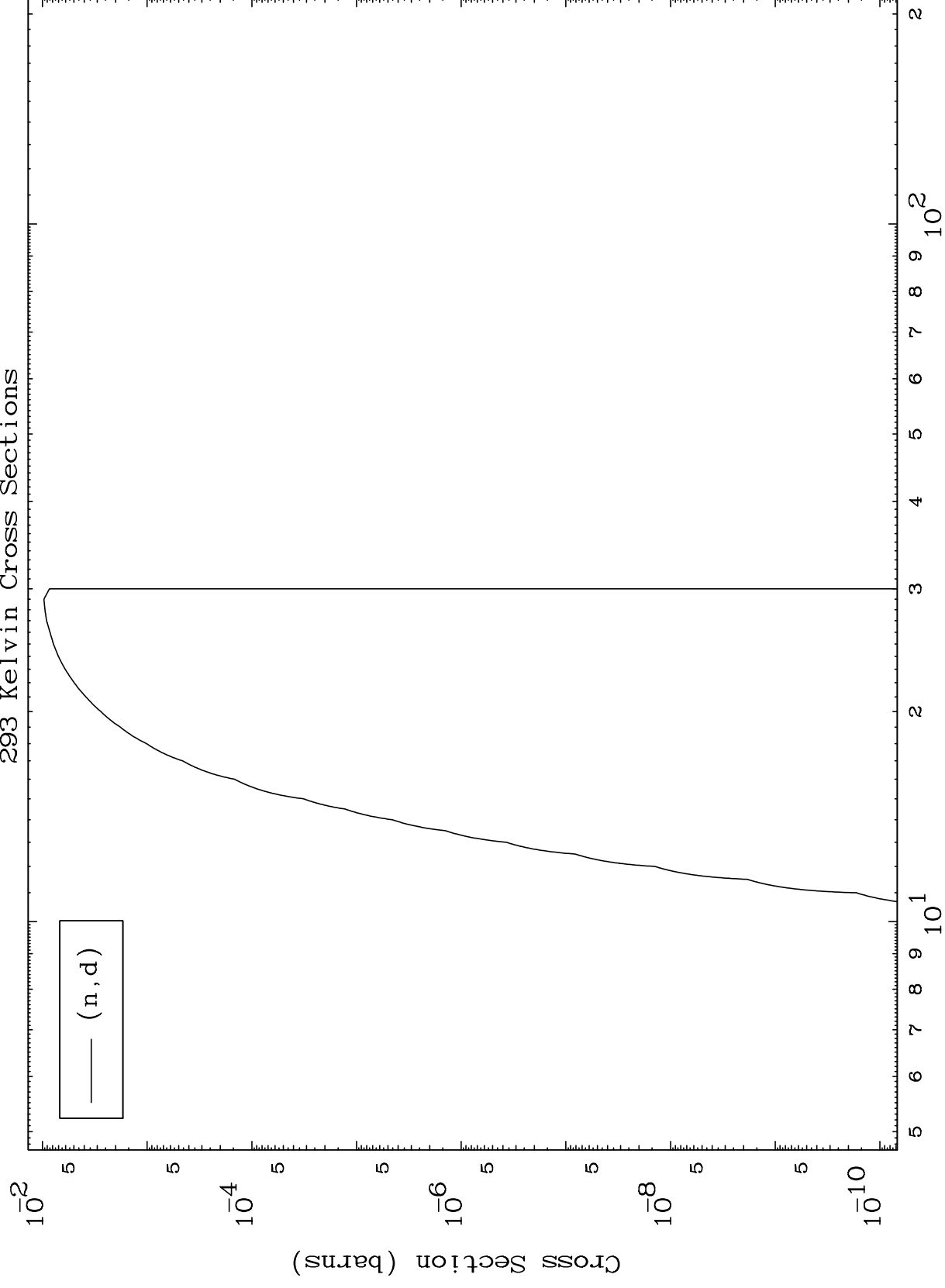


MAT 2244

(n, d) Levels

27-Ir-197m

293 Kelvin Cross Sections



12

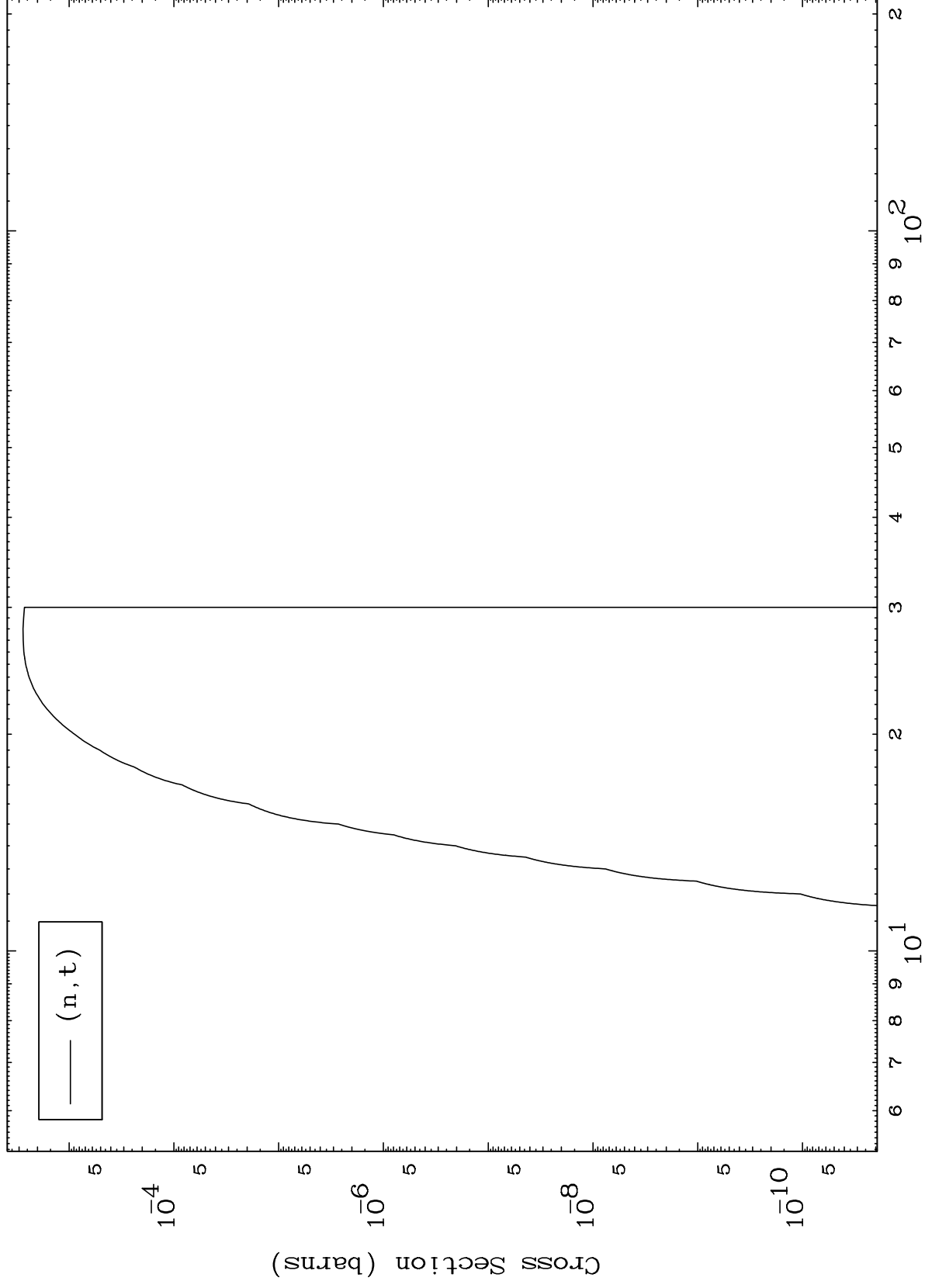
Incident Energy (MeV)

22-1r-197m

MAT 7744

(n,t) Levels
293 Kelvin Cross Sections

77-Ir-197m



13

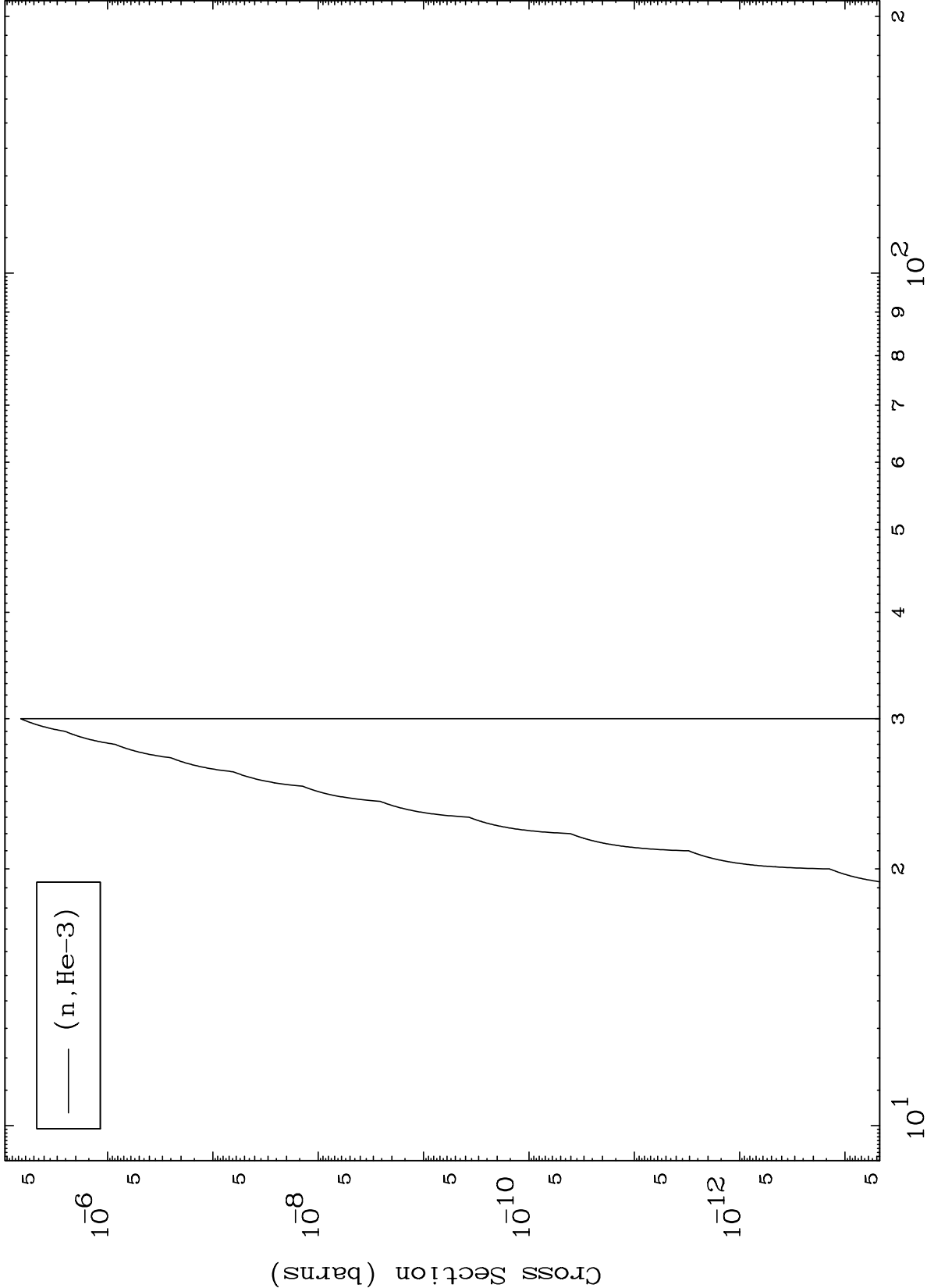
Incident Energy (MeV)

77-Ir-197m

MAT 7744

(n,He3) Levels
293 Kelvin Cross Sections

77-Ir-197m



77-Ir-197m

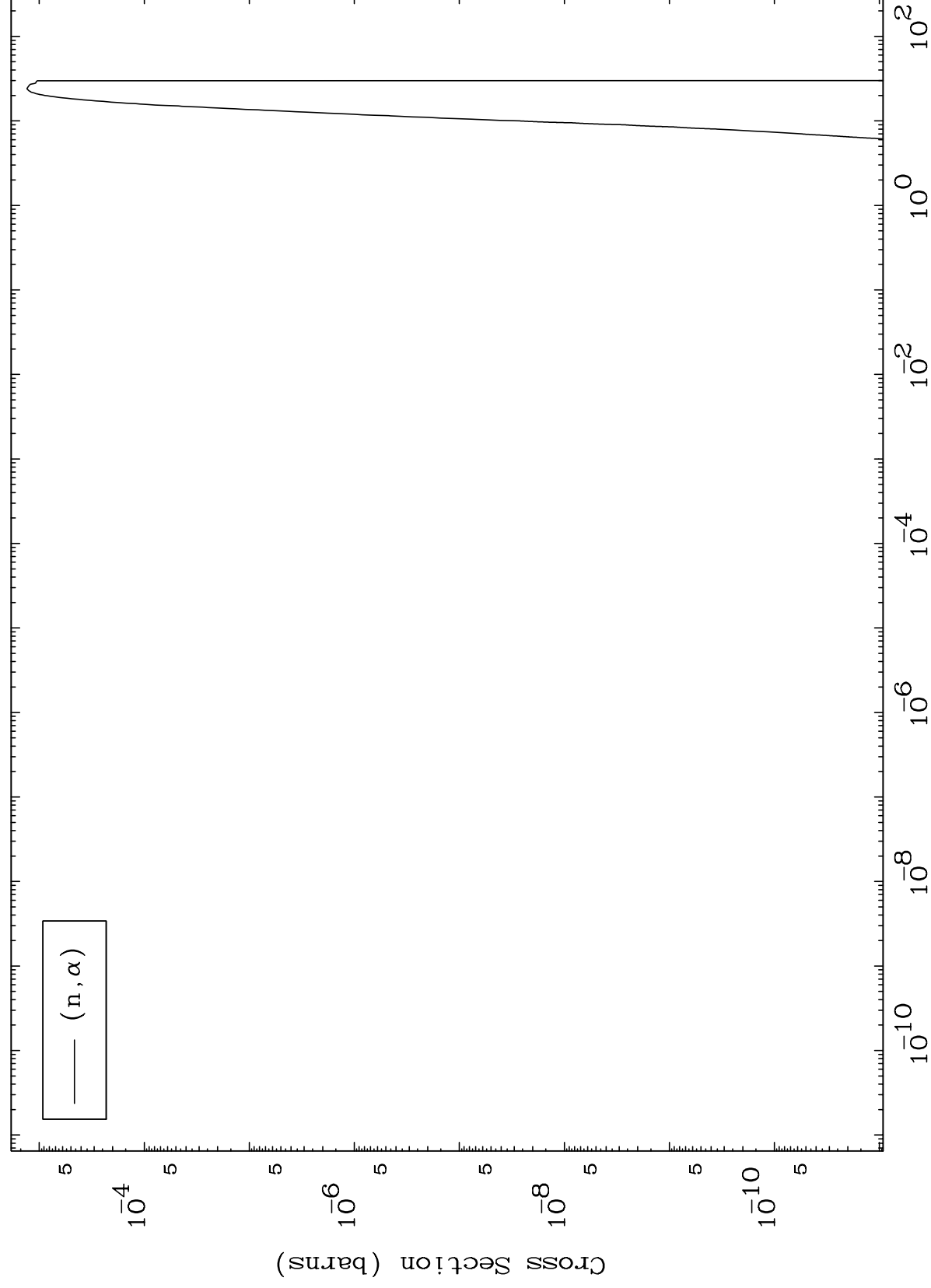
Incident Energy (MeV)

14

MAT 7744

(n, α) Levels
293 Kelvin Cross Sections

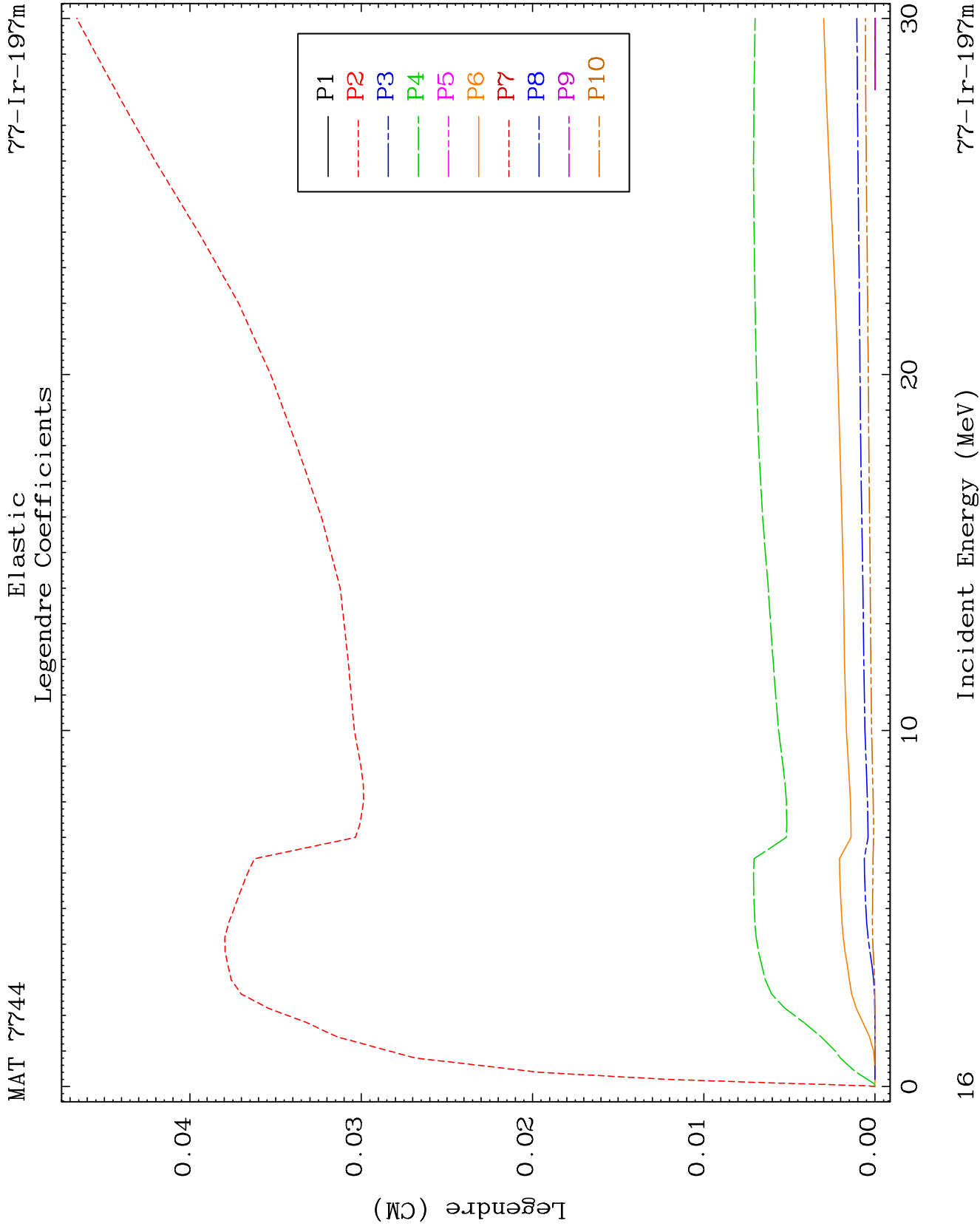
77-Ir-197m

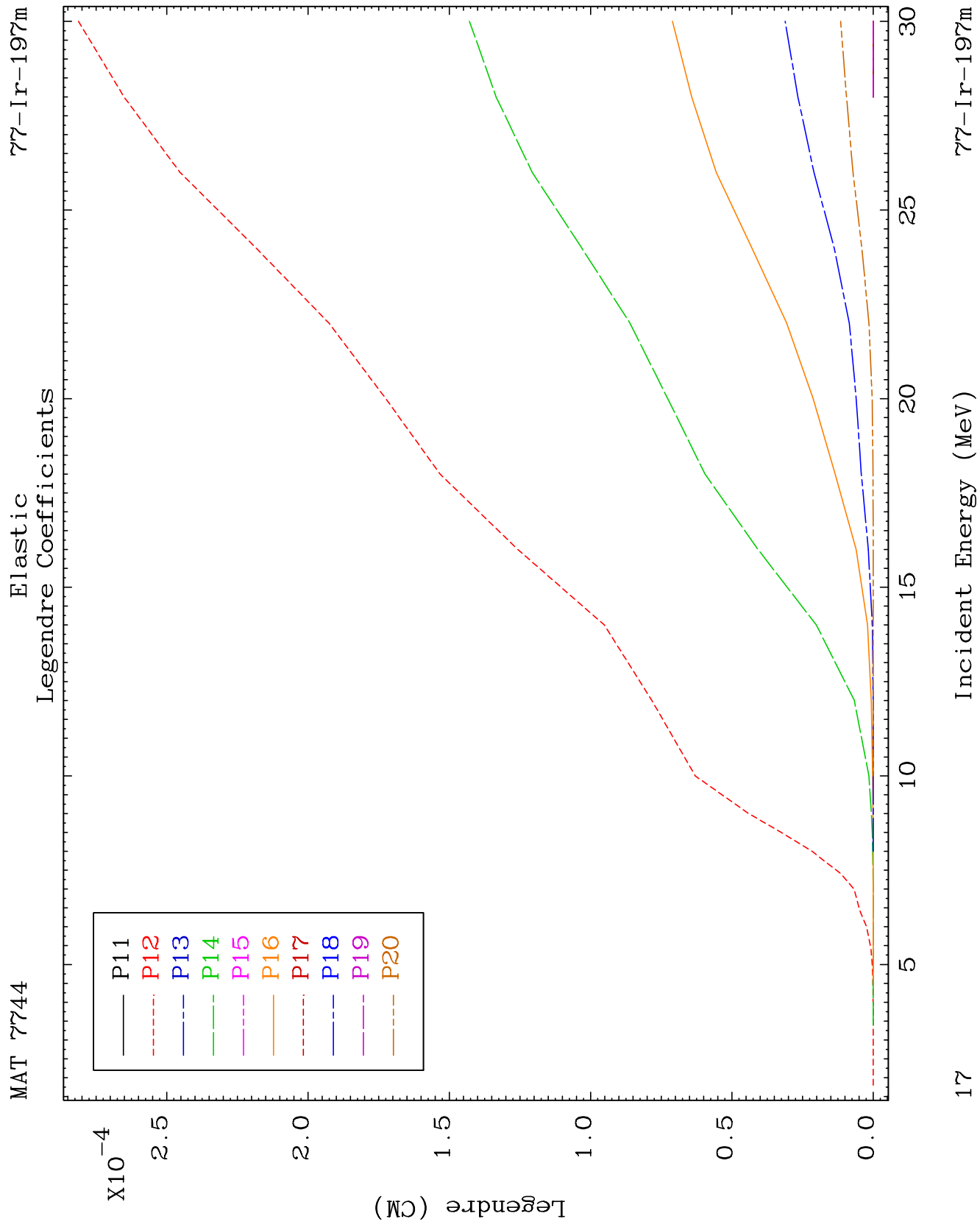


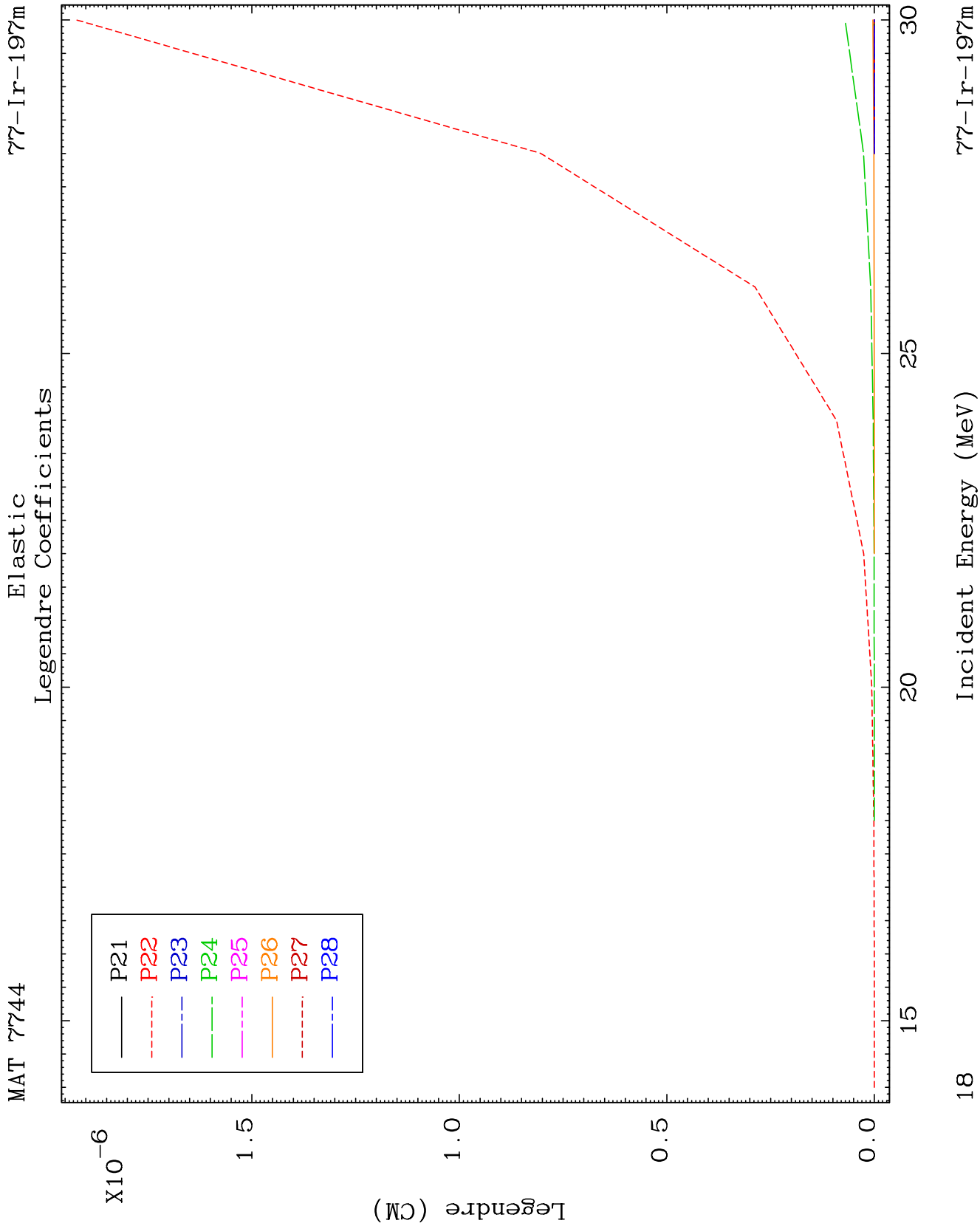
15

Incident Energy (MeV)

77-Ir-197m



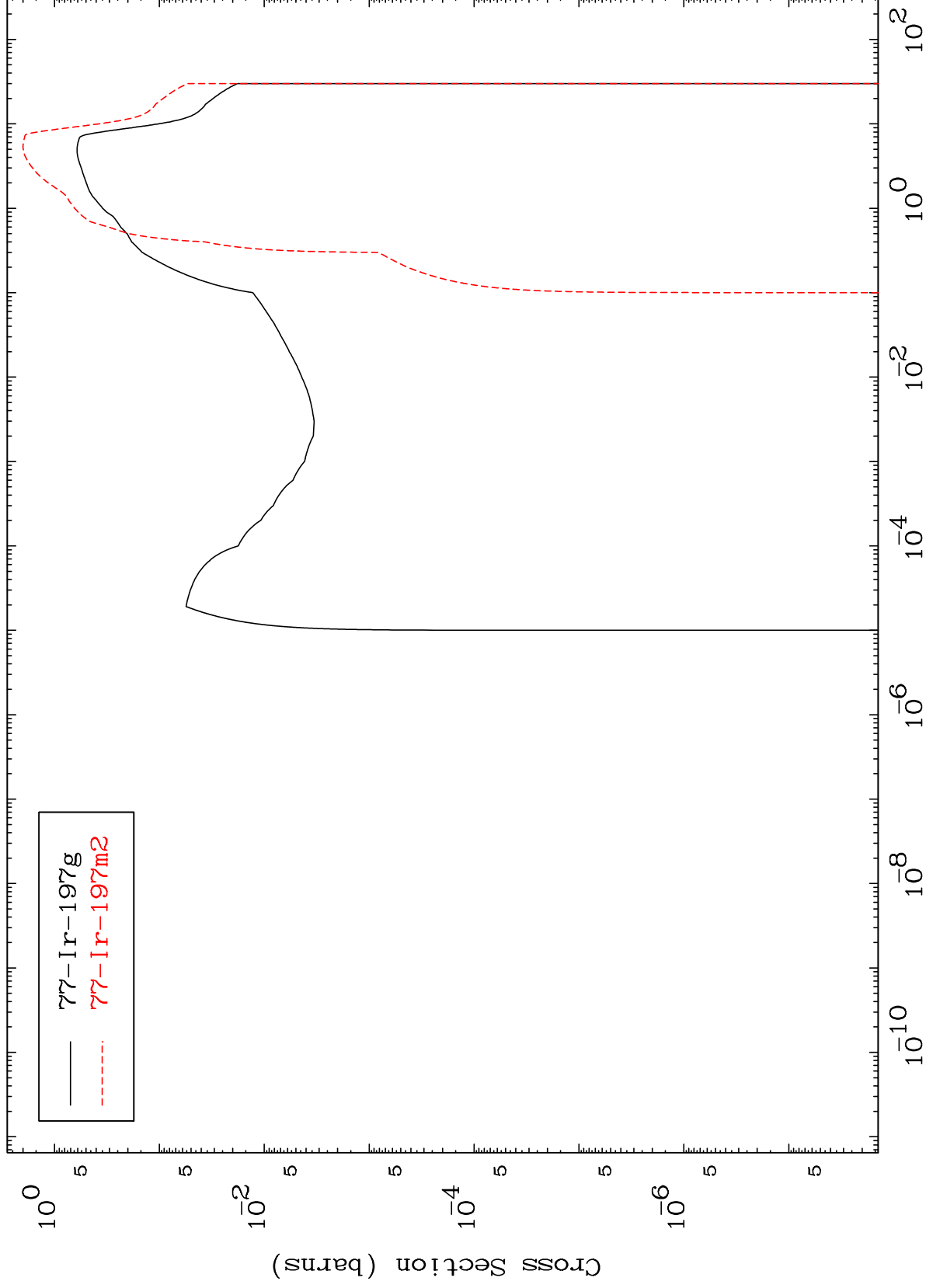


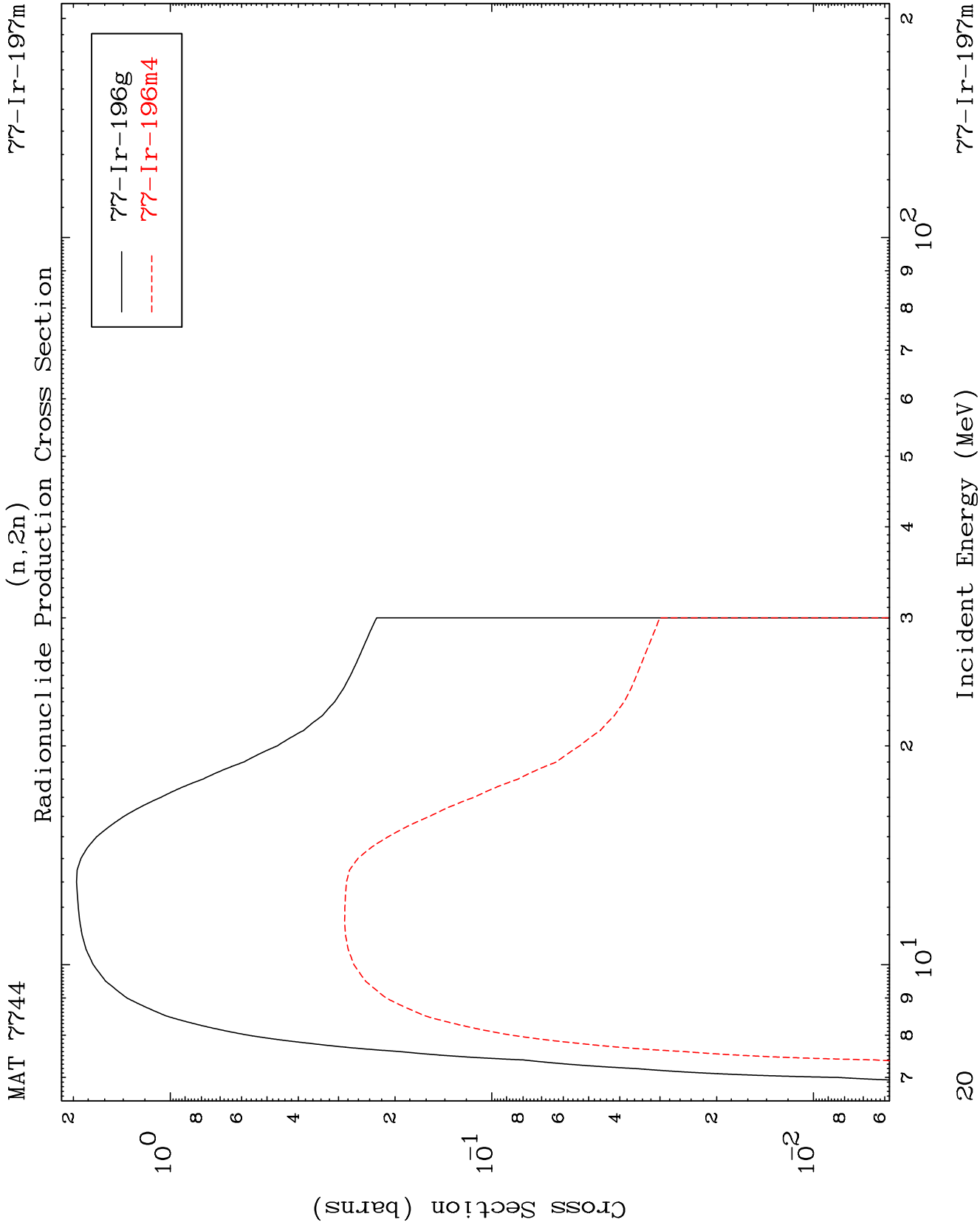


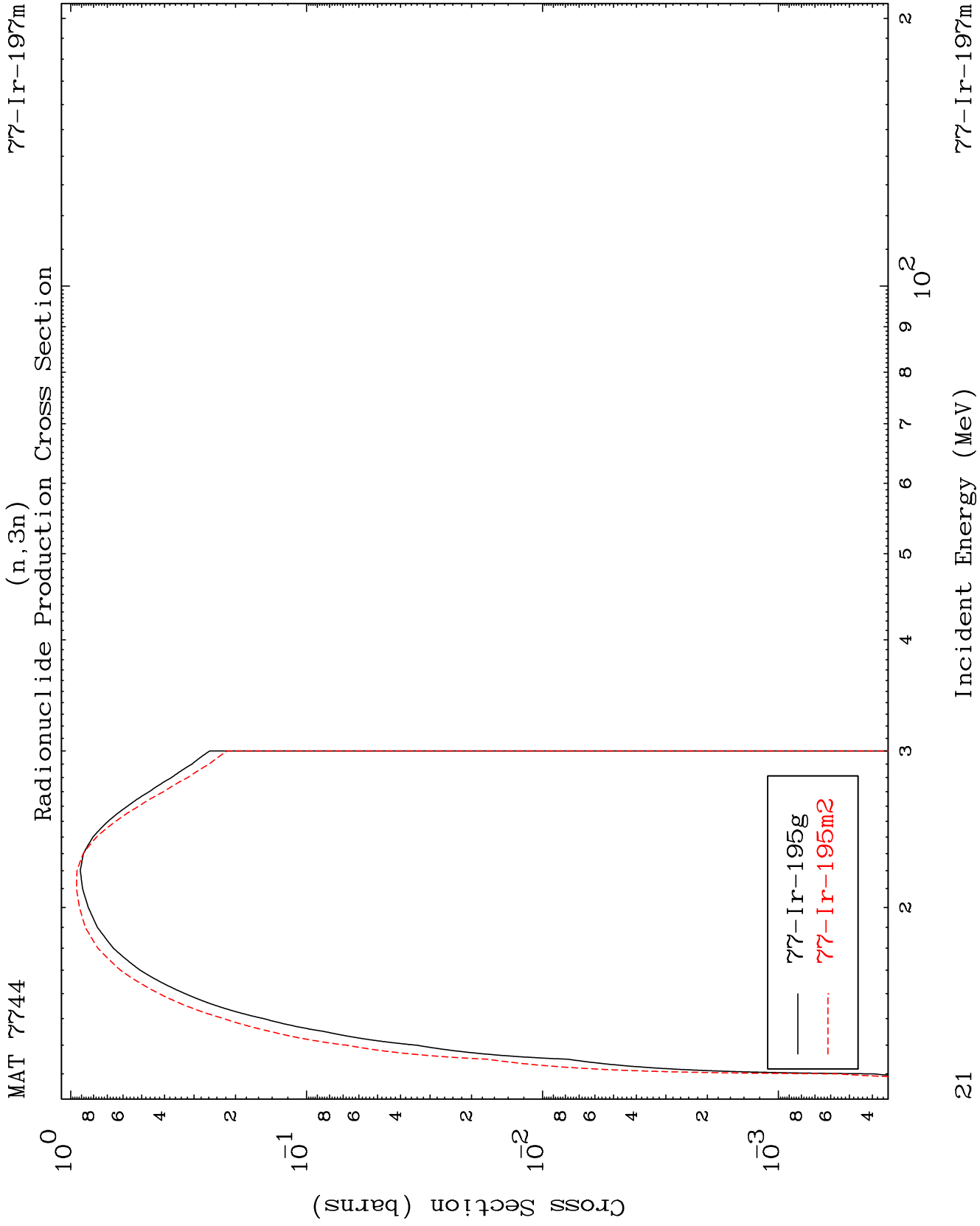
MAT 7744

Inelastic
Radionuclide Production Cross Section

$^{77}\text{Ir-197m}$





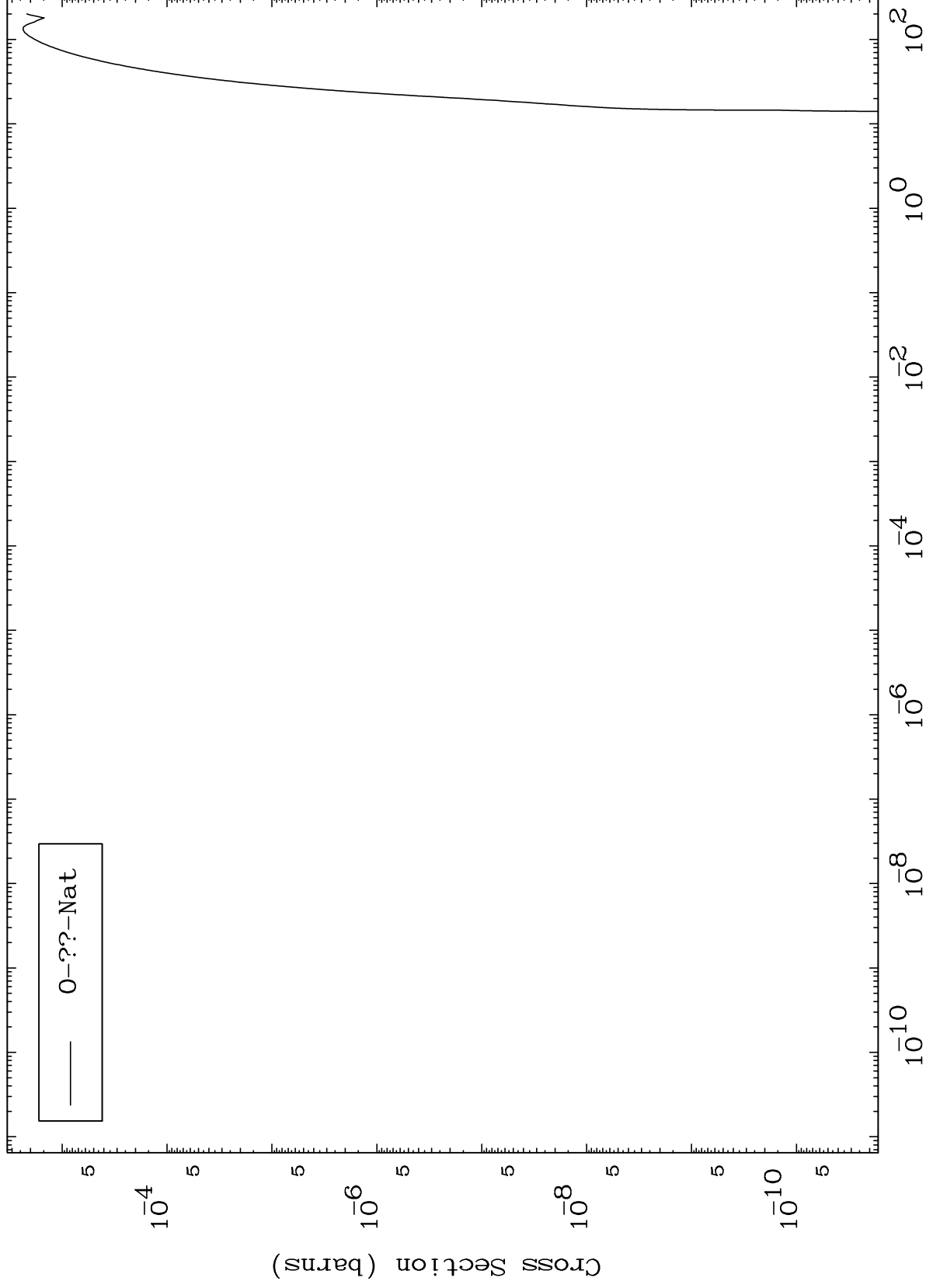


MAT 7744

Fission

77-Ir-197m

Radionuclide Production Cross Section



22

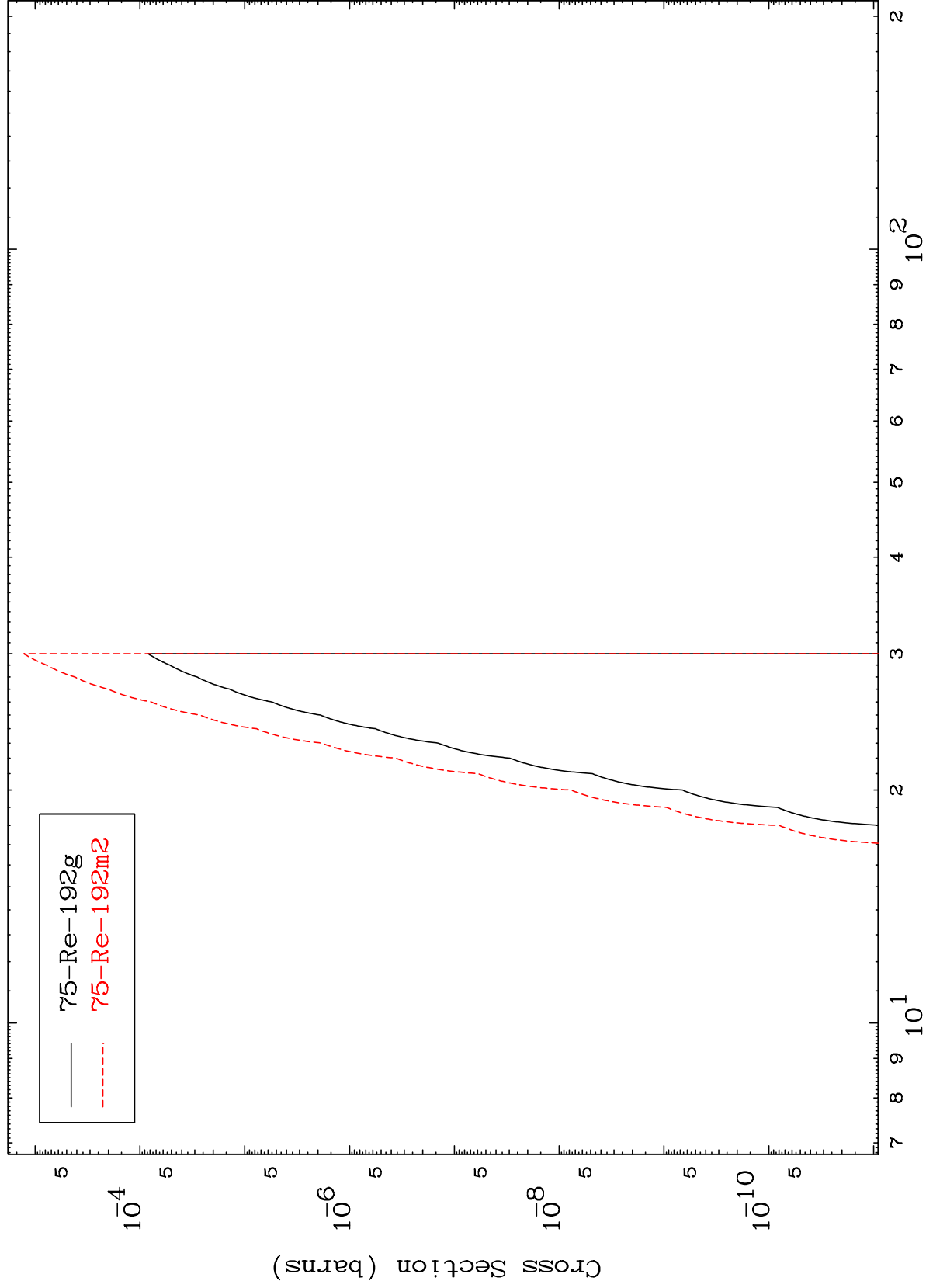
Incident Energy (MeV)

77-Ir-197m

MAT 7744

$^{77}\text{Ir-197m}$

Radionuclide Production Cross Section



Incident Energy (MeV)

$^{77}\text{Ir-197m}$

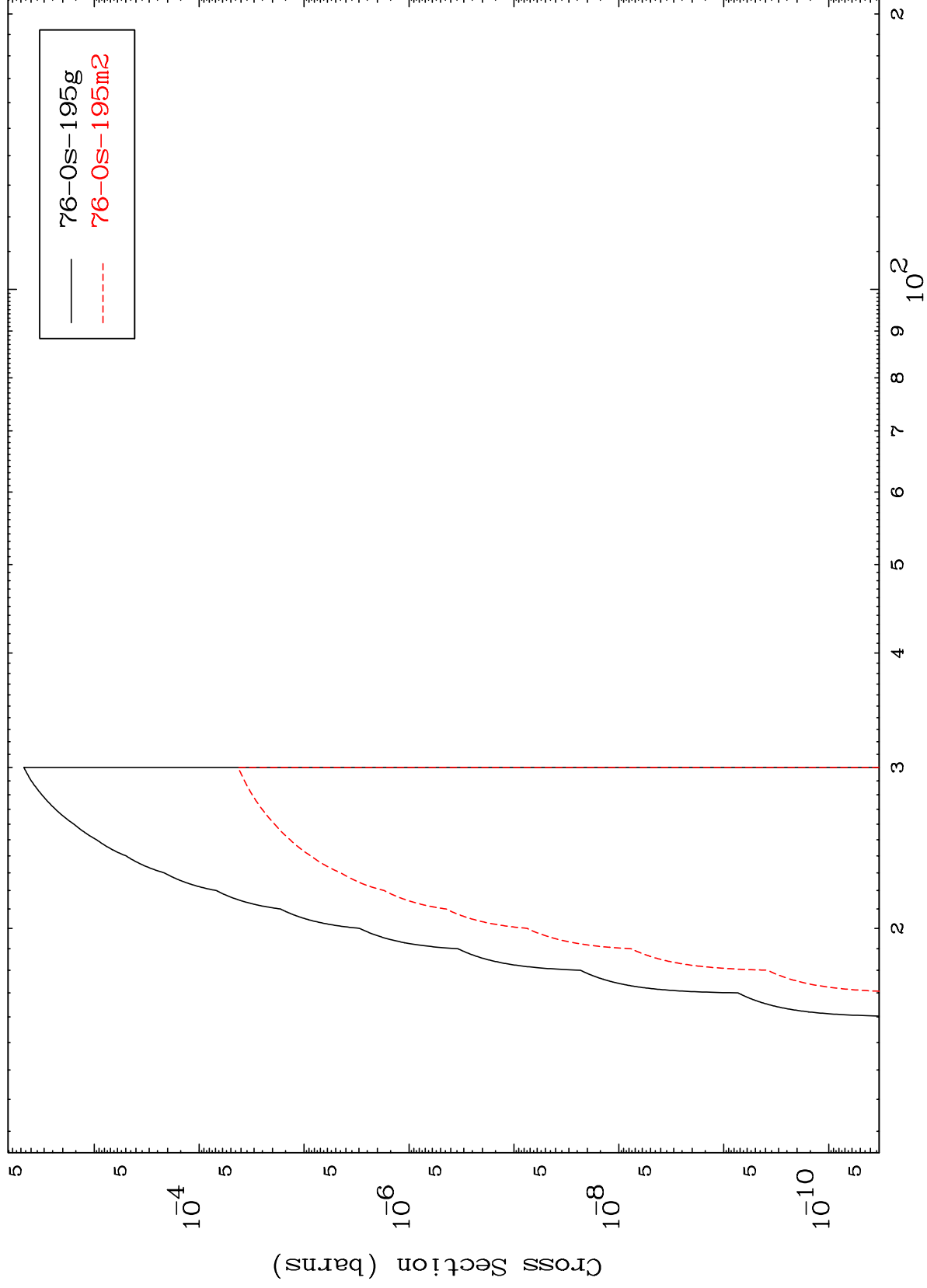
23

MAT 7744

(n,n') d

77-Ir-197m

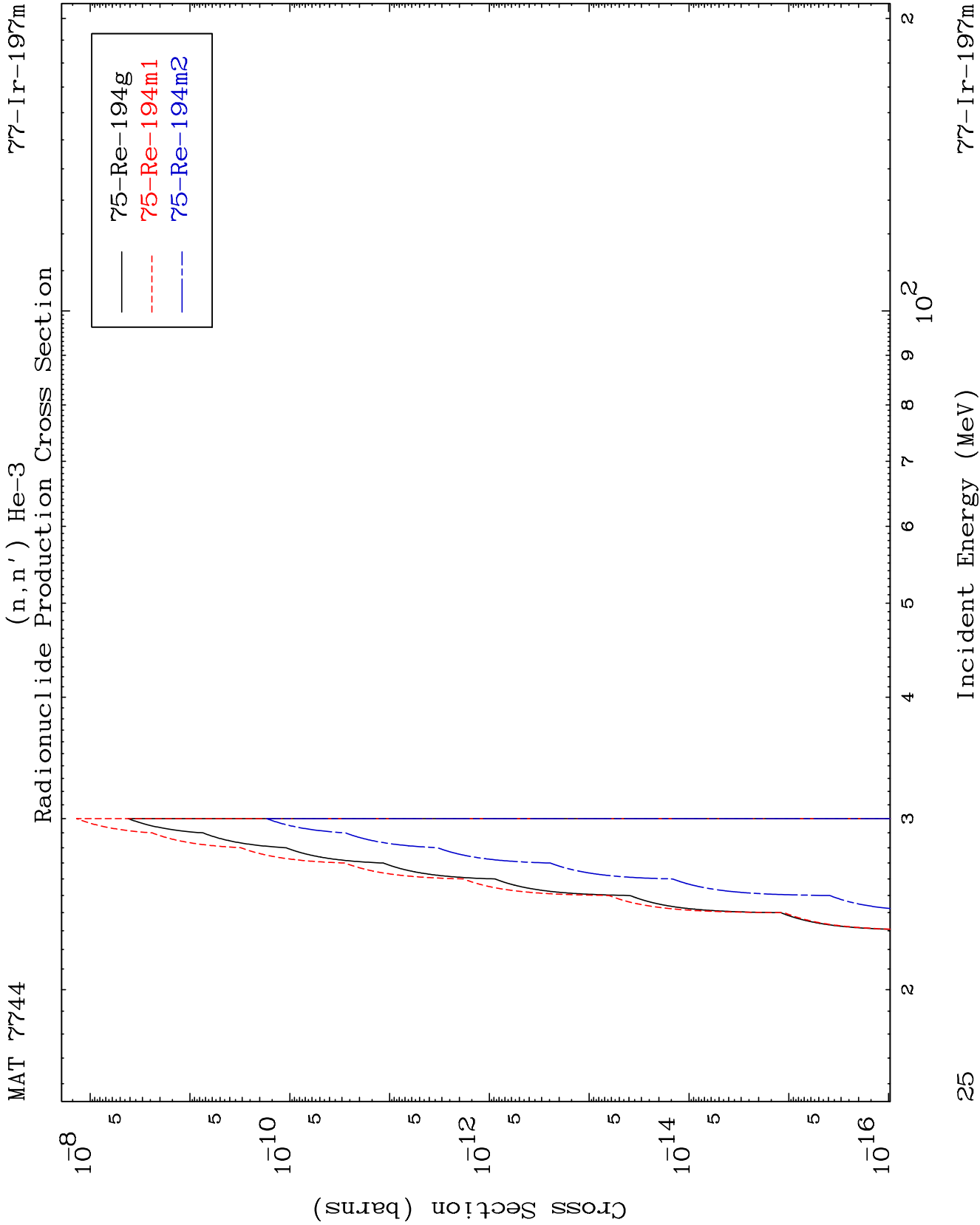
Radionuclide Production Cross Section

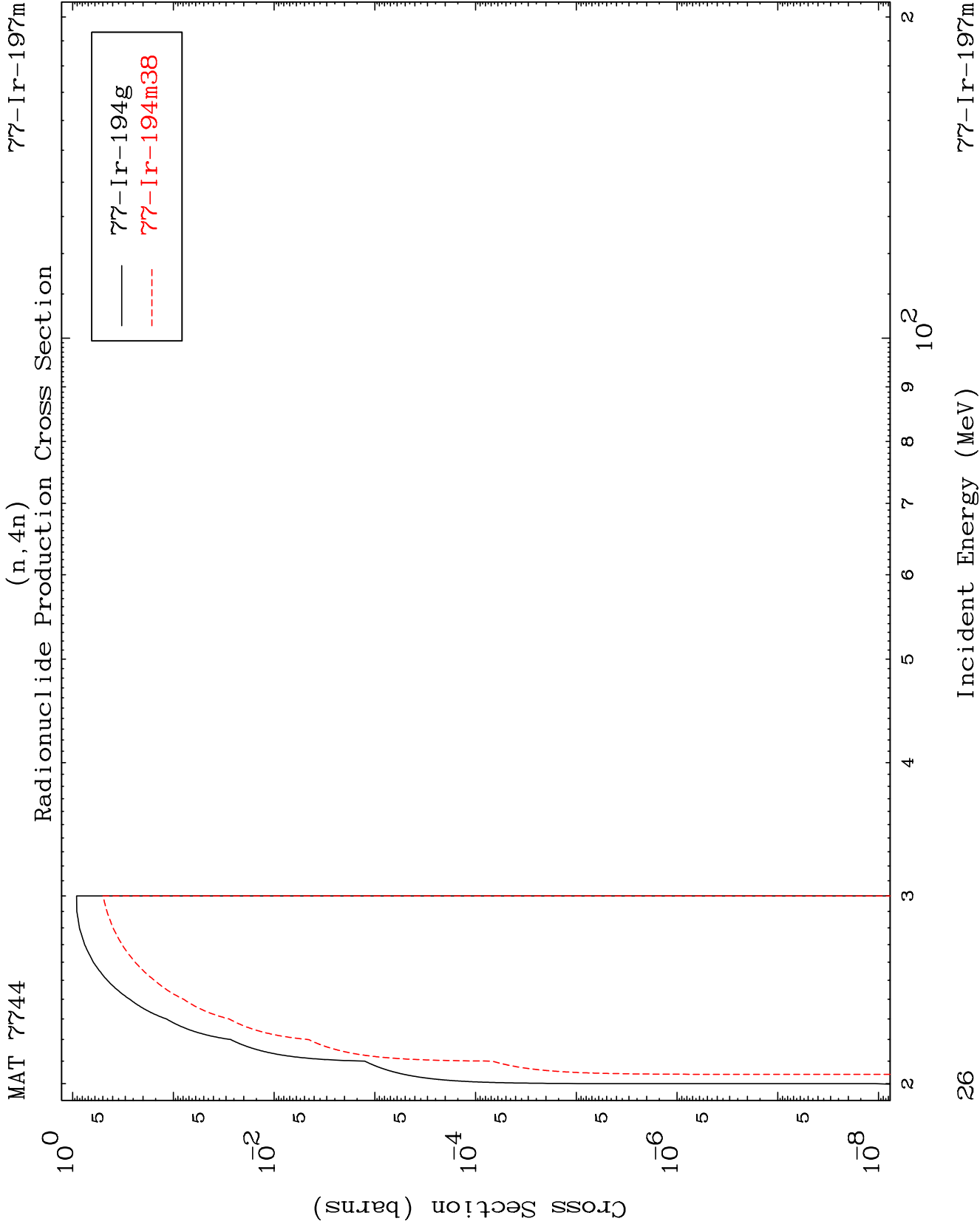


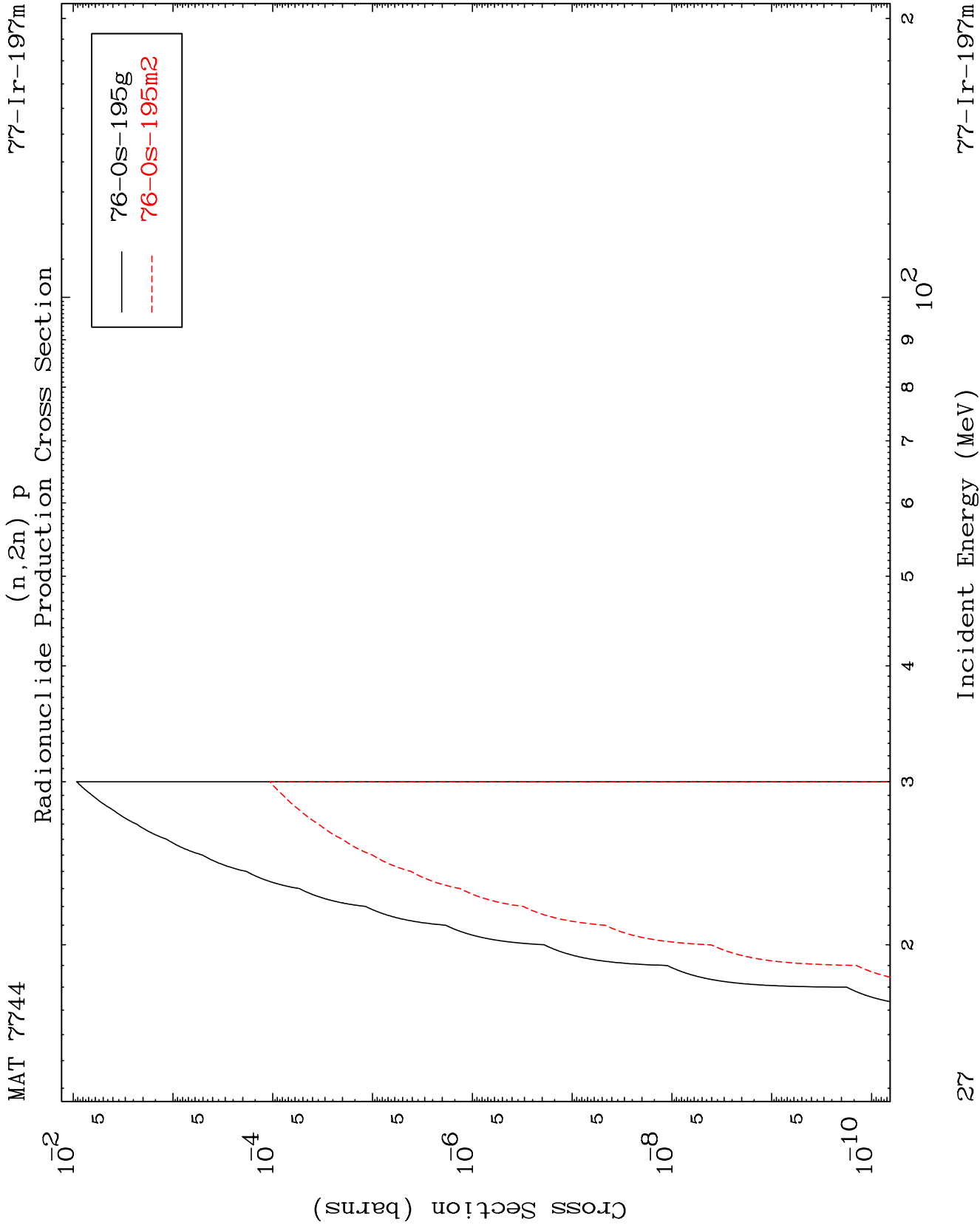
24

Incident Energy (MeV)

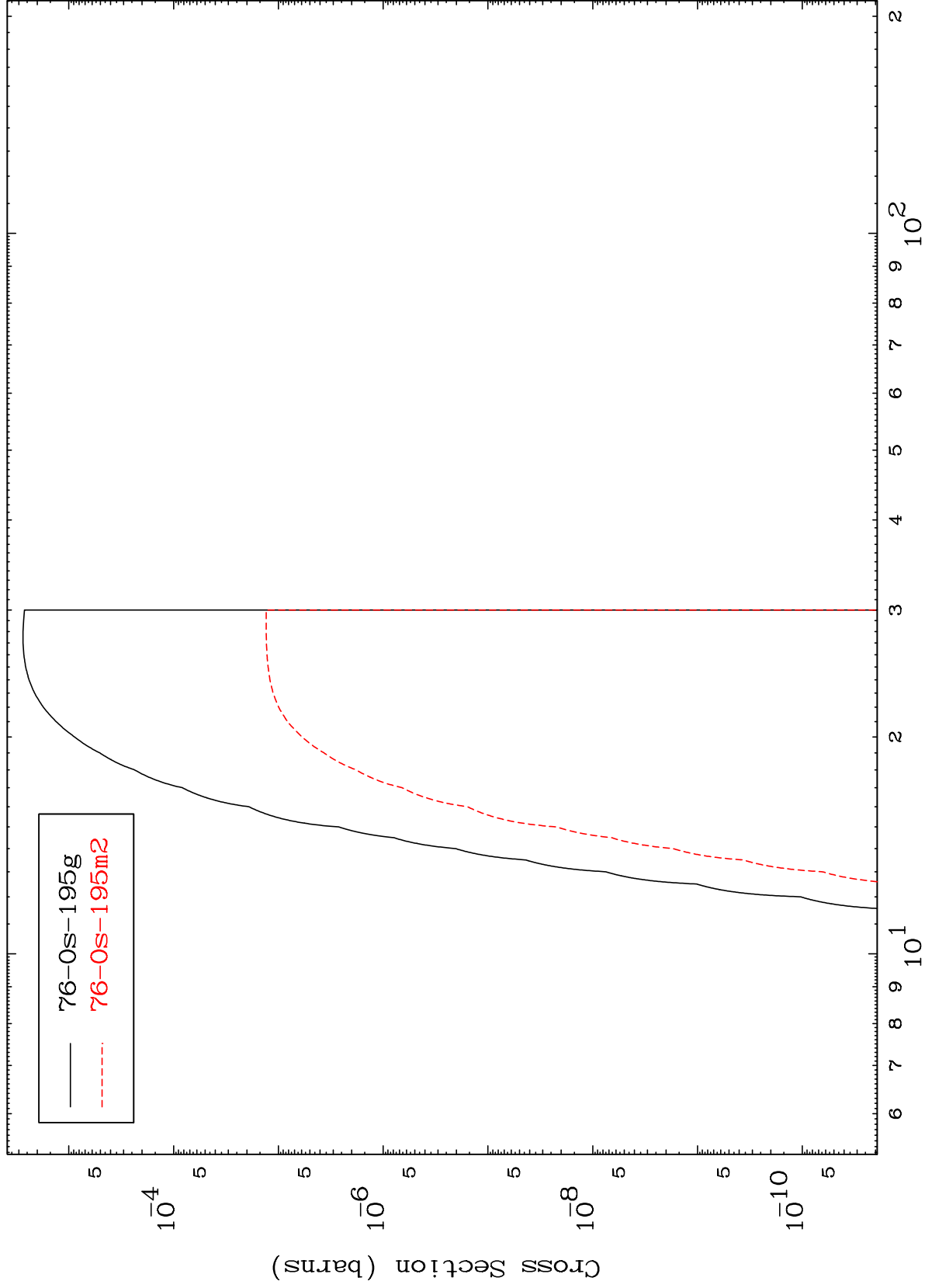
77-Ir-197m







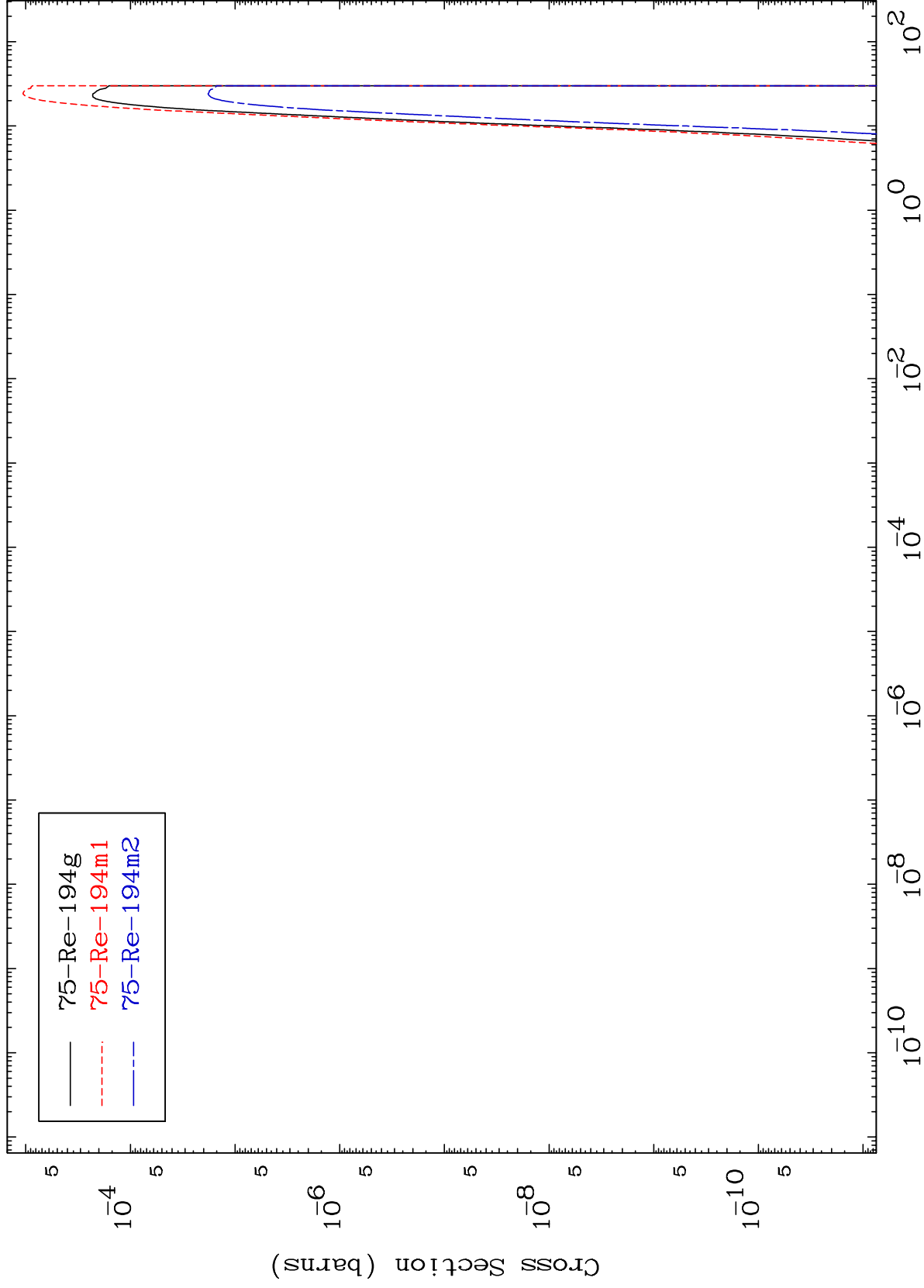
Radionuclide Production Cross Section



MAT 7744

$^{77}\text{Ir-197m}$

Radionuclide Production Cross Section



29

$^{77}\text{Ir-197m}$