

Program EVALPLOT
(Version 2021-1)

by

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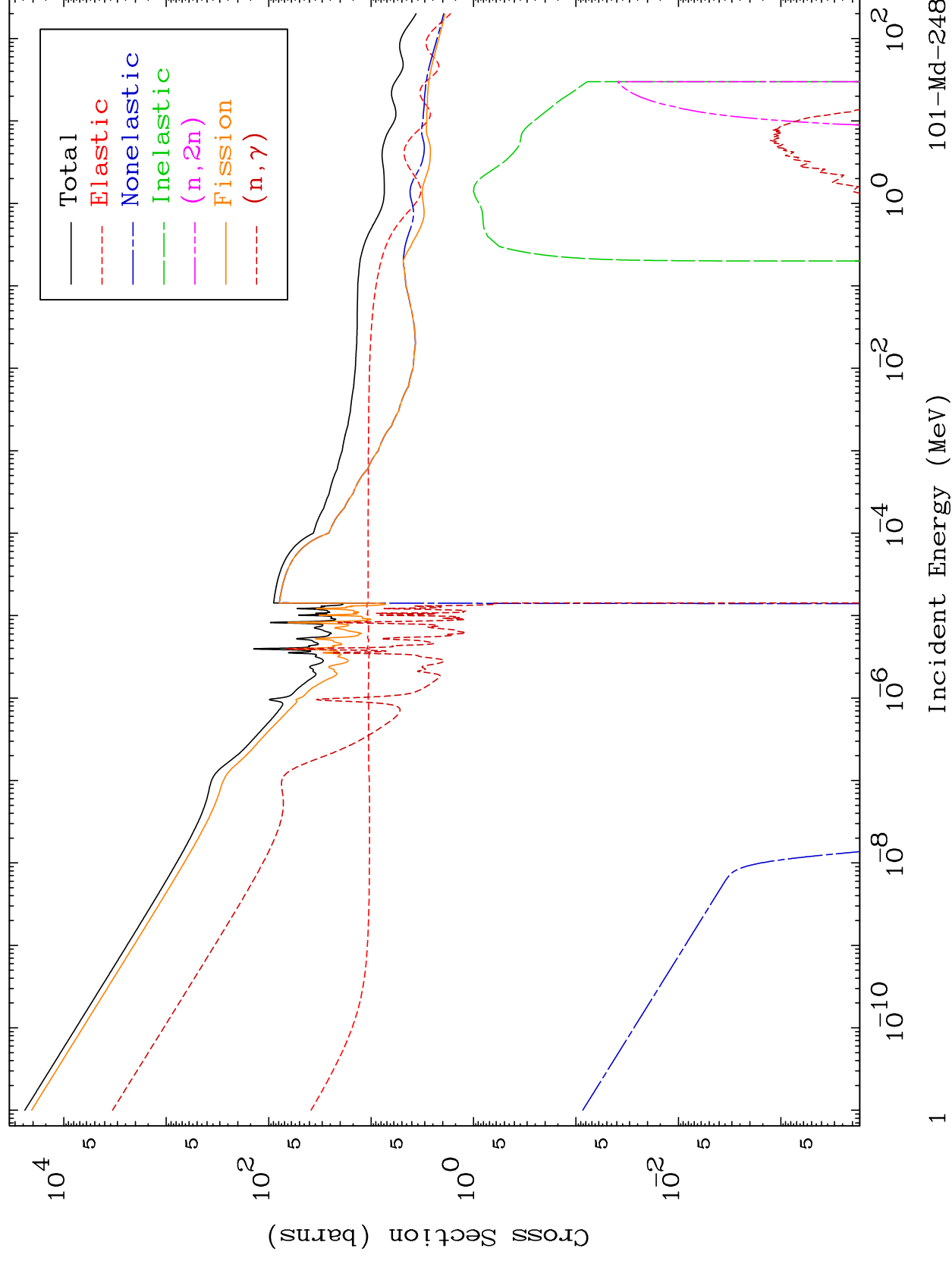
Press Mouse Button to Start

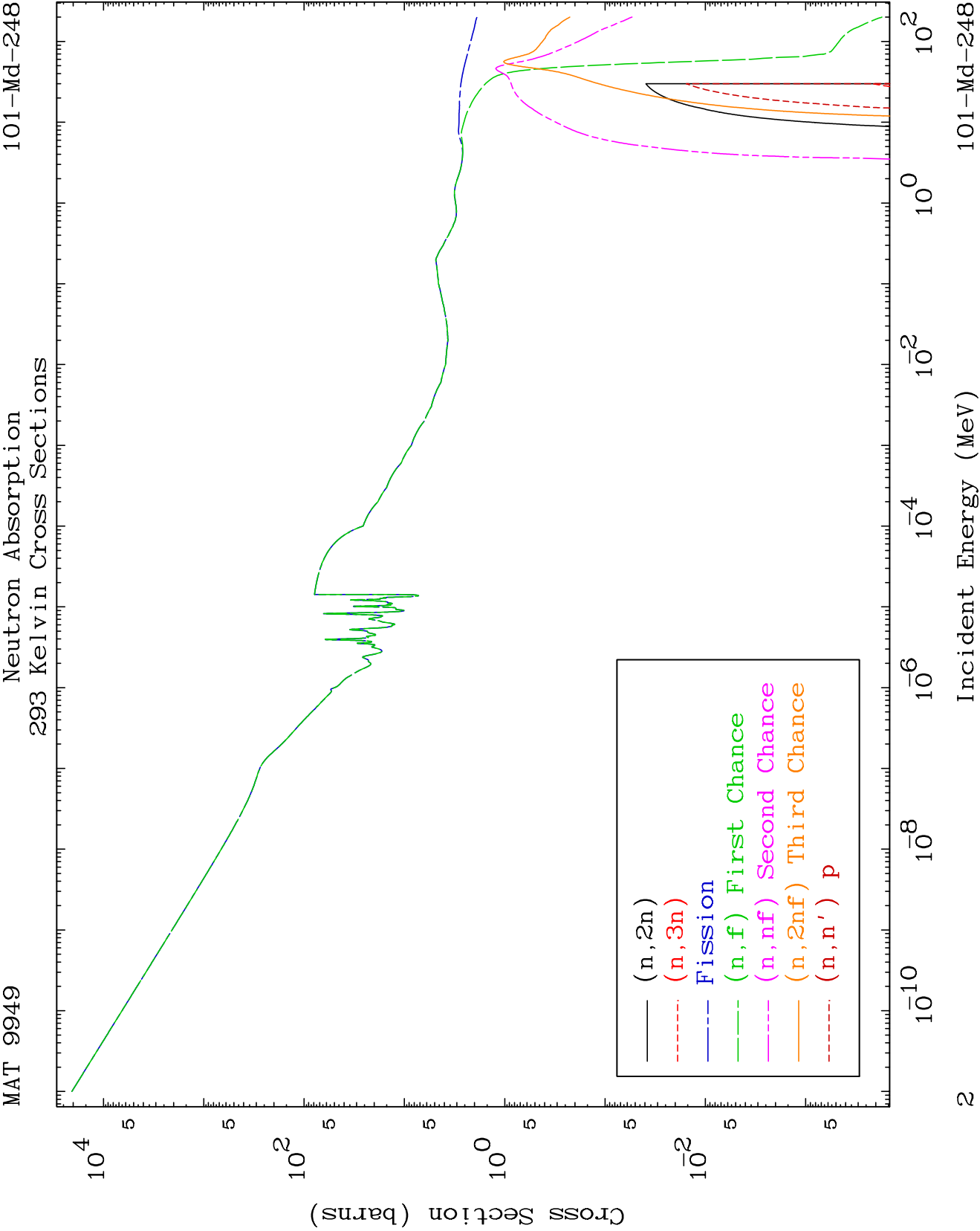
MAT 9949

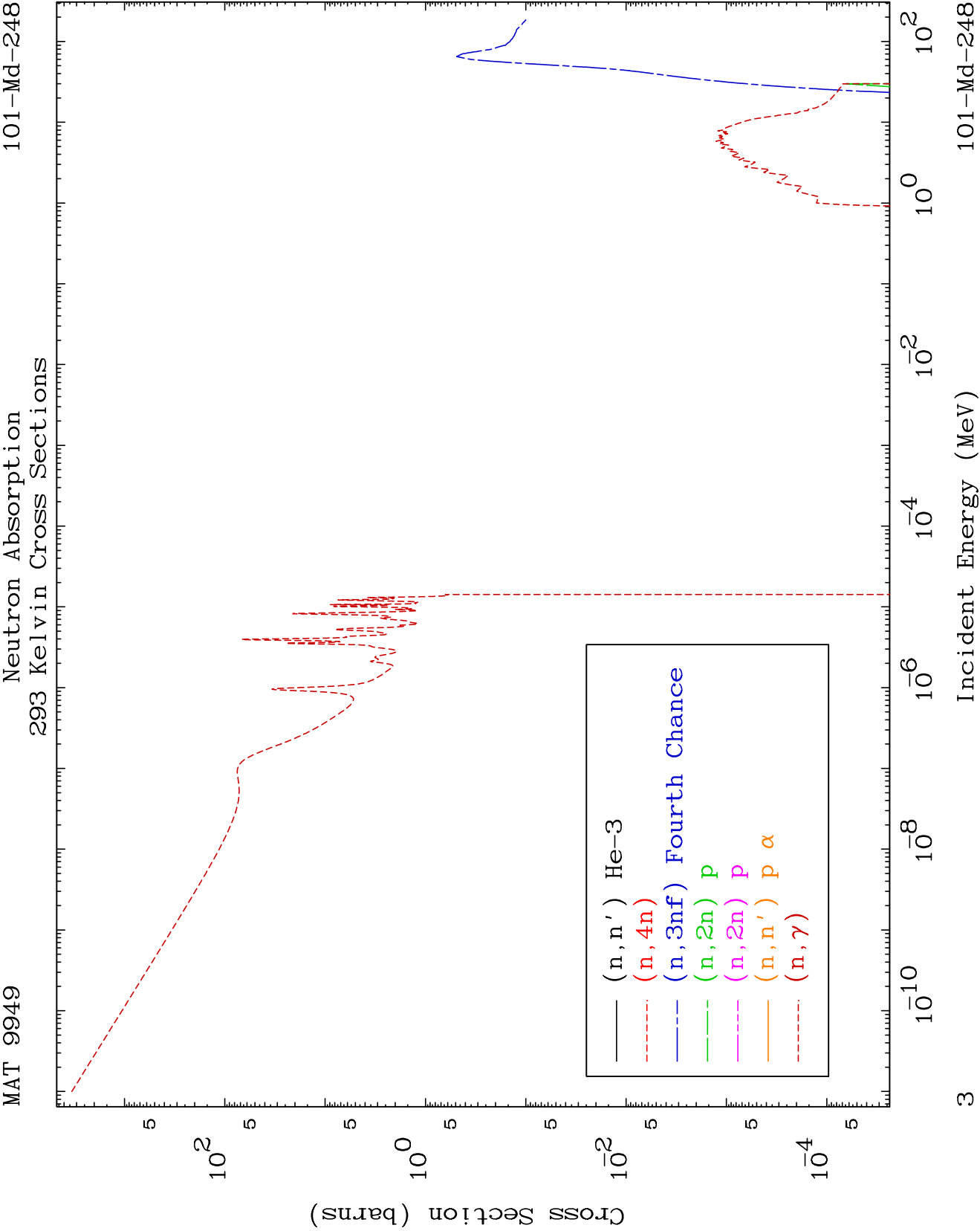
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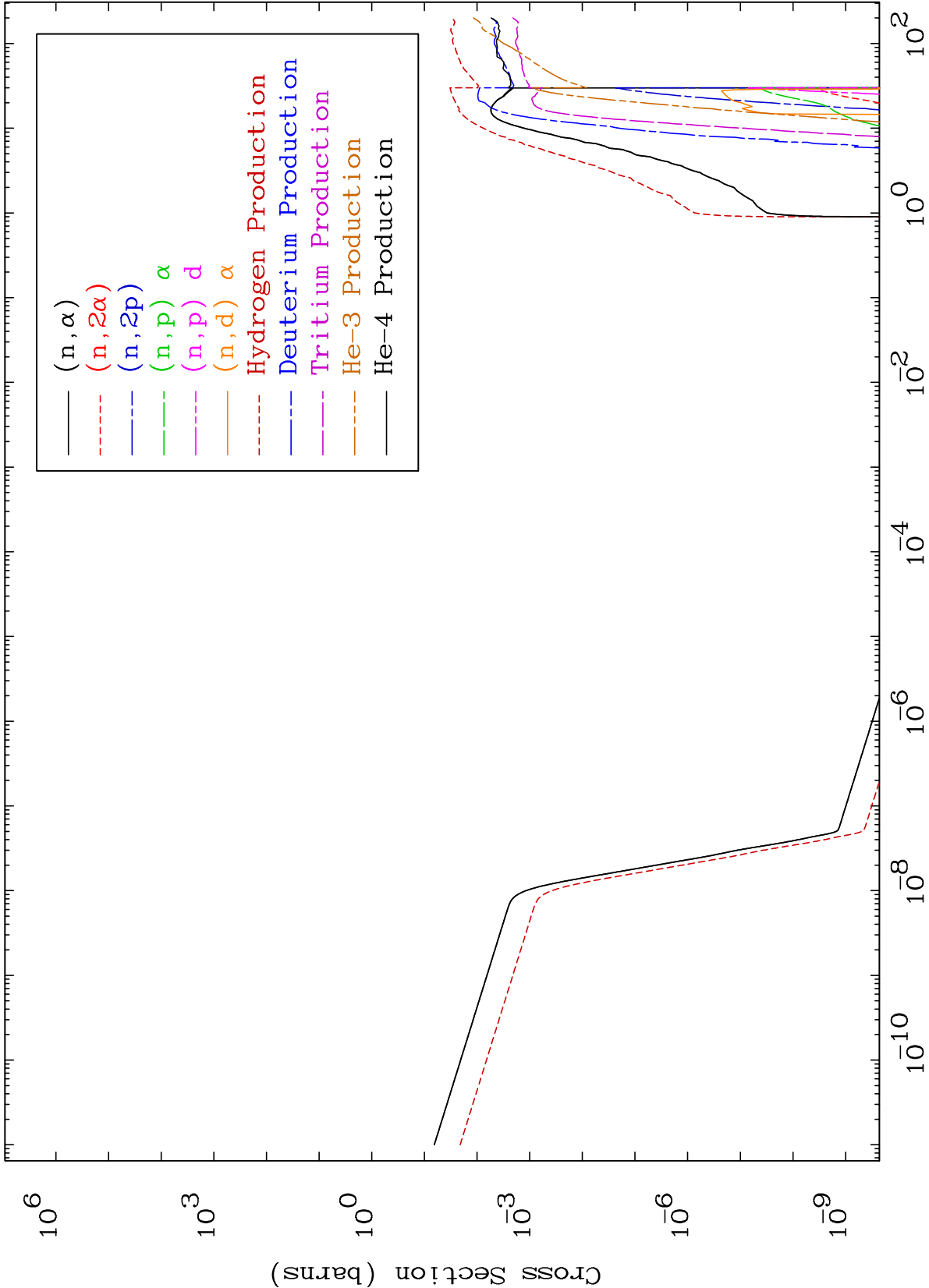
293 Kelvin Cross Sections

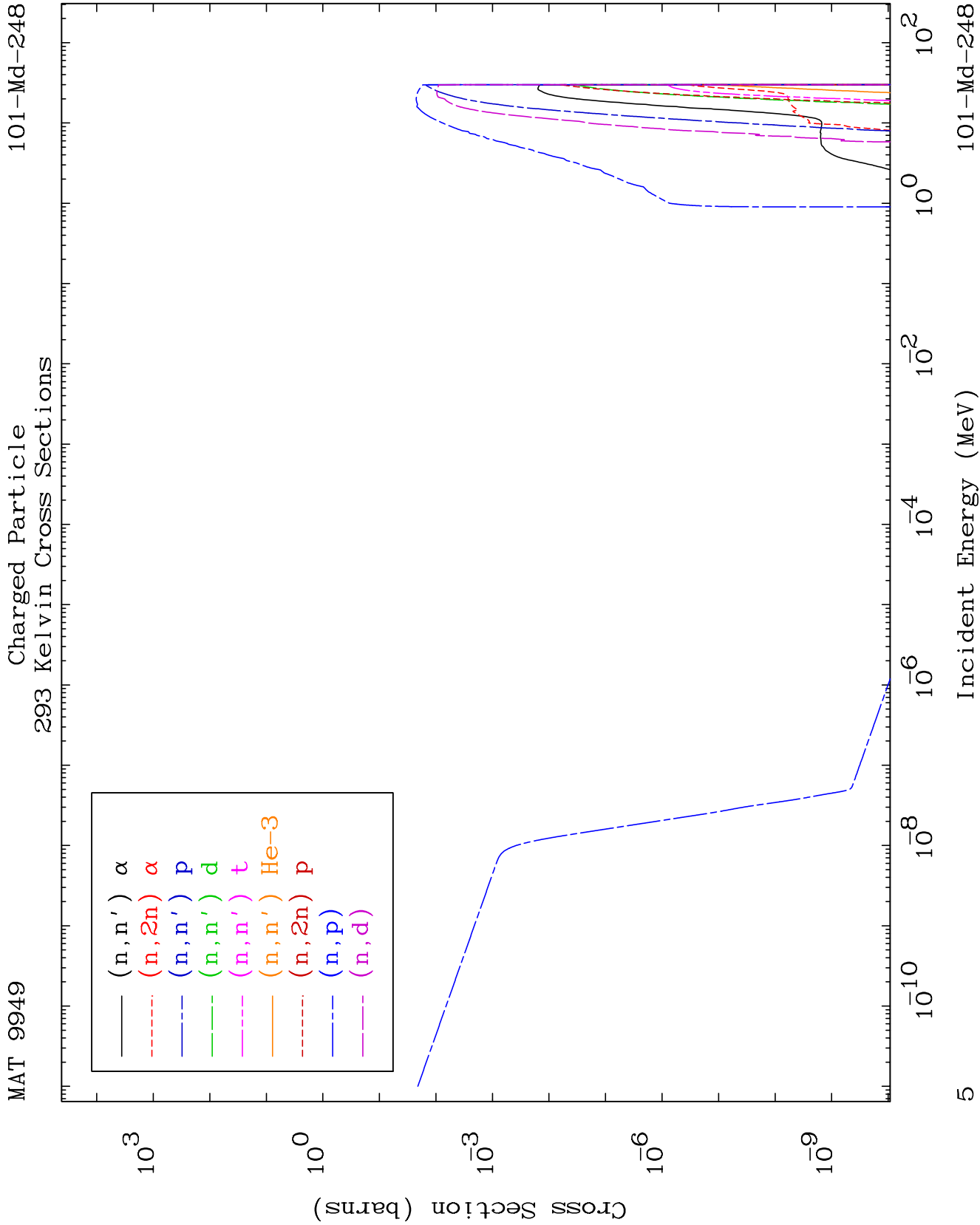
101-Md-248

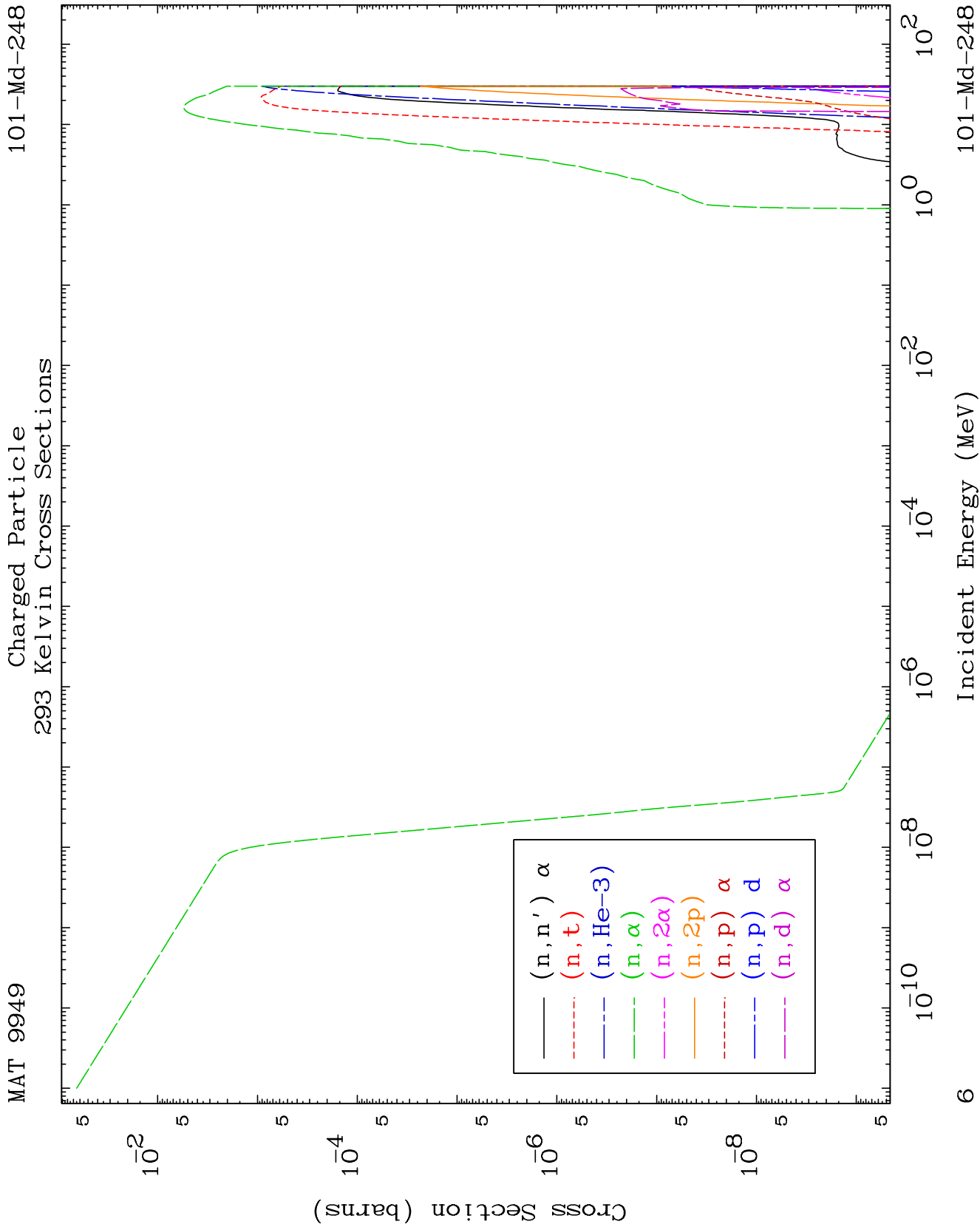








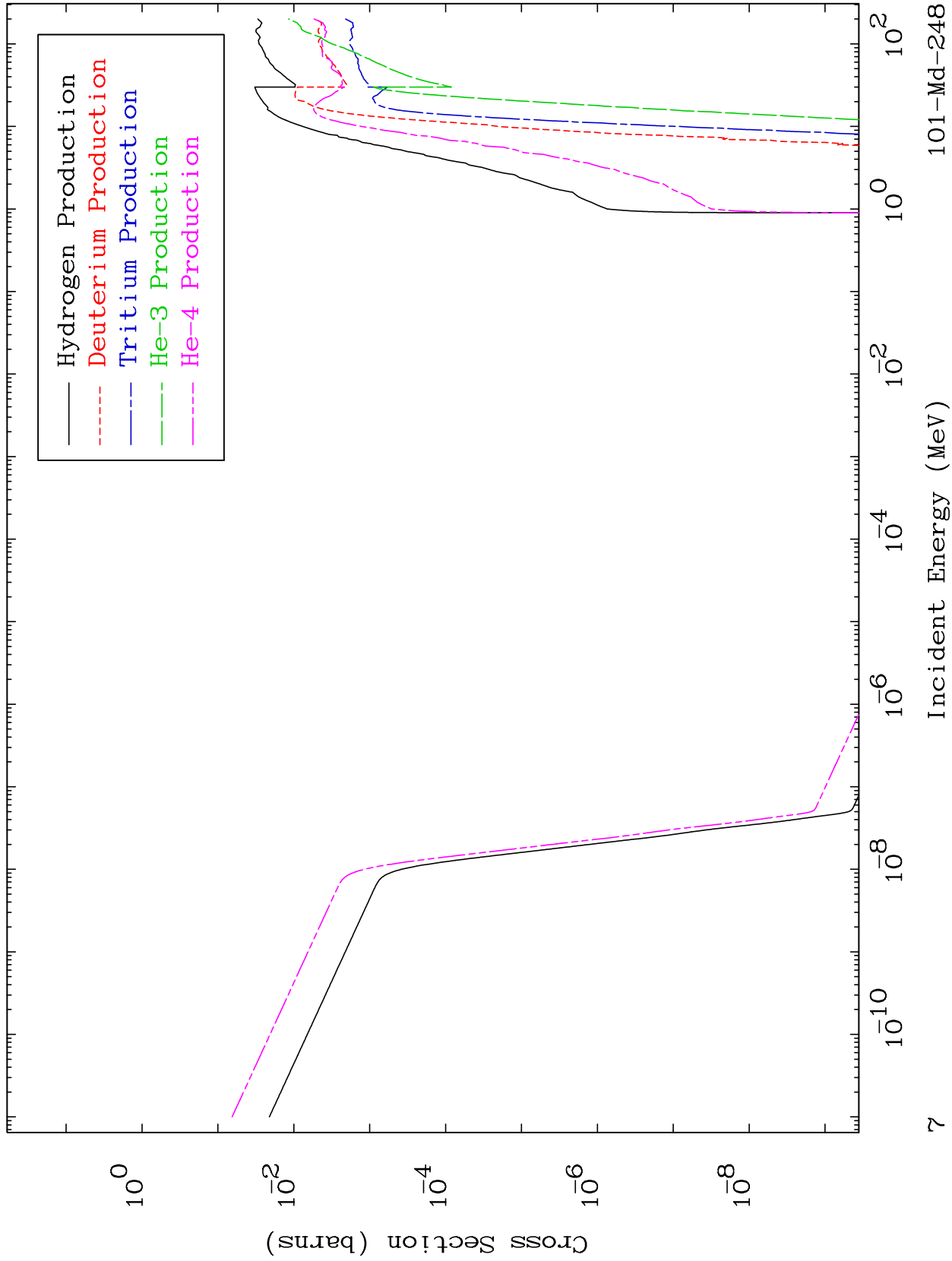


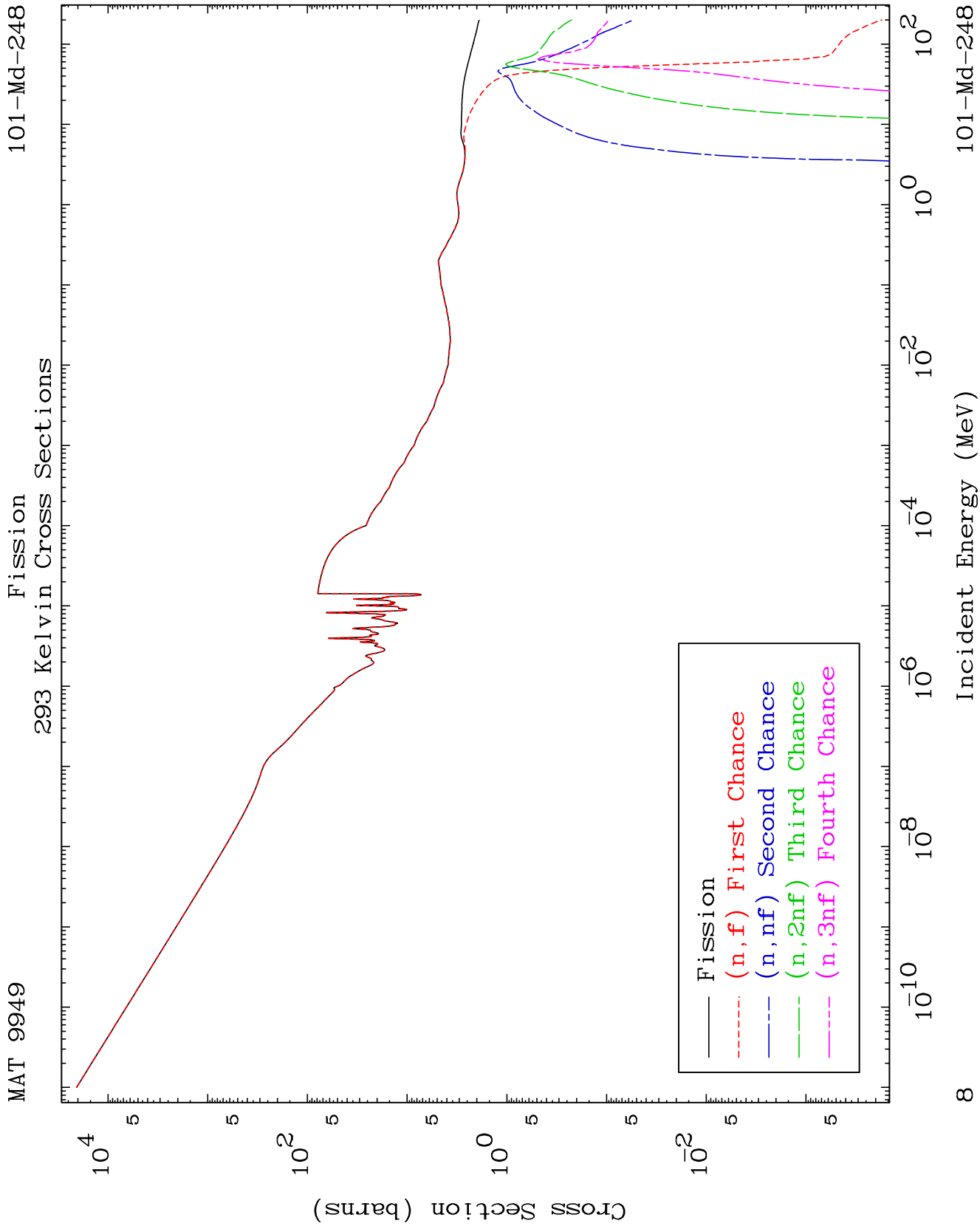


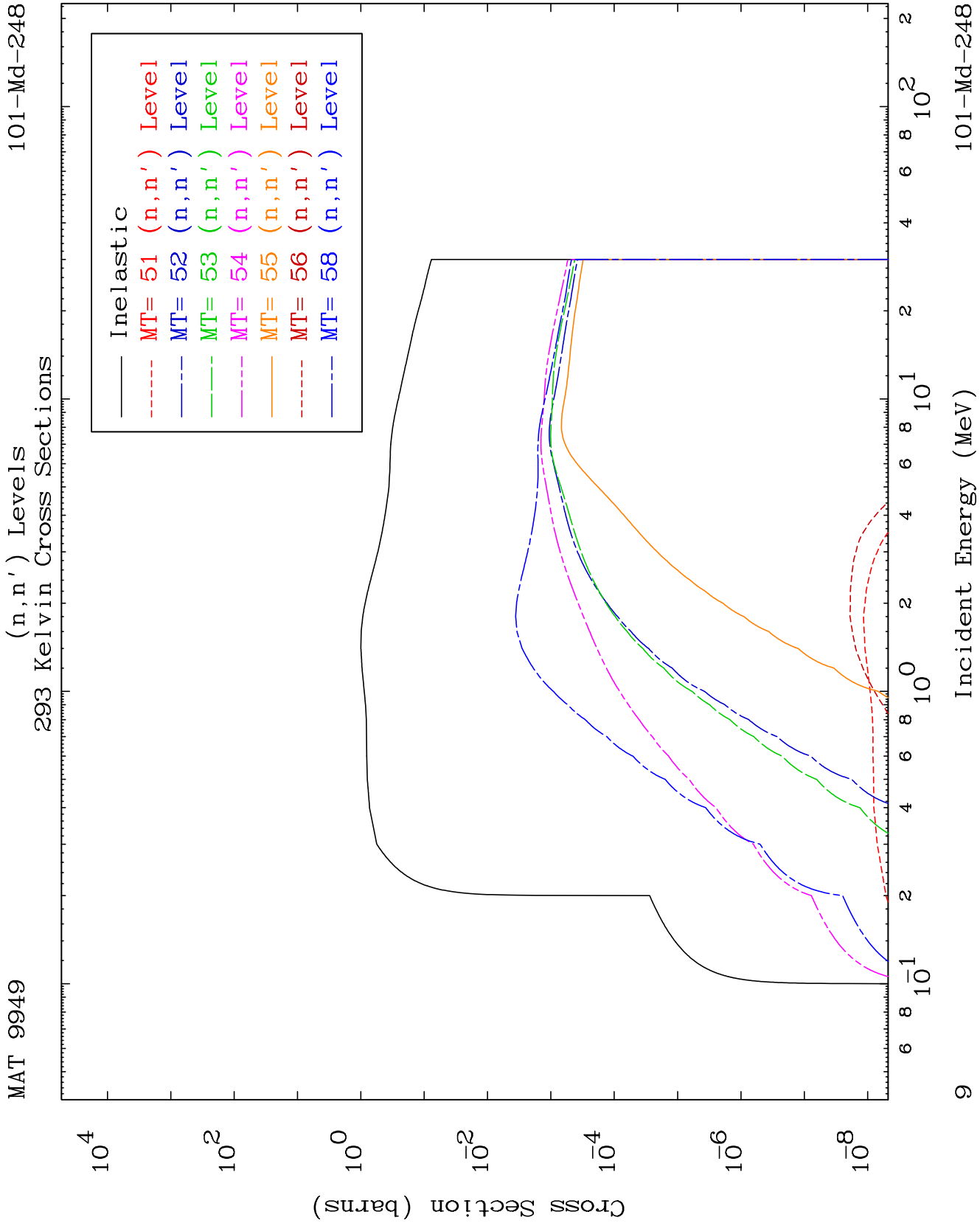
MAT 9949

Particle Production
293 Kelvin Cross Sections

101-Md-248





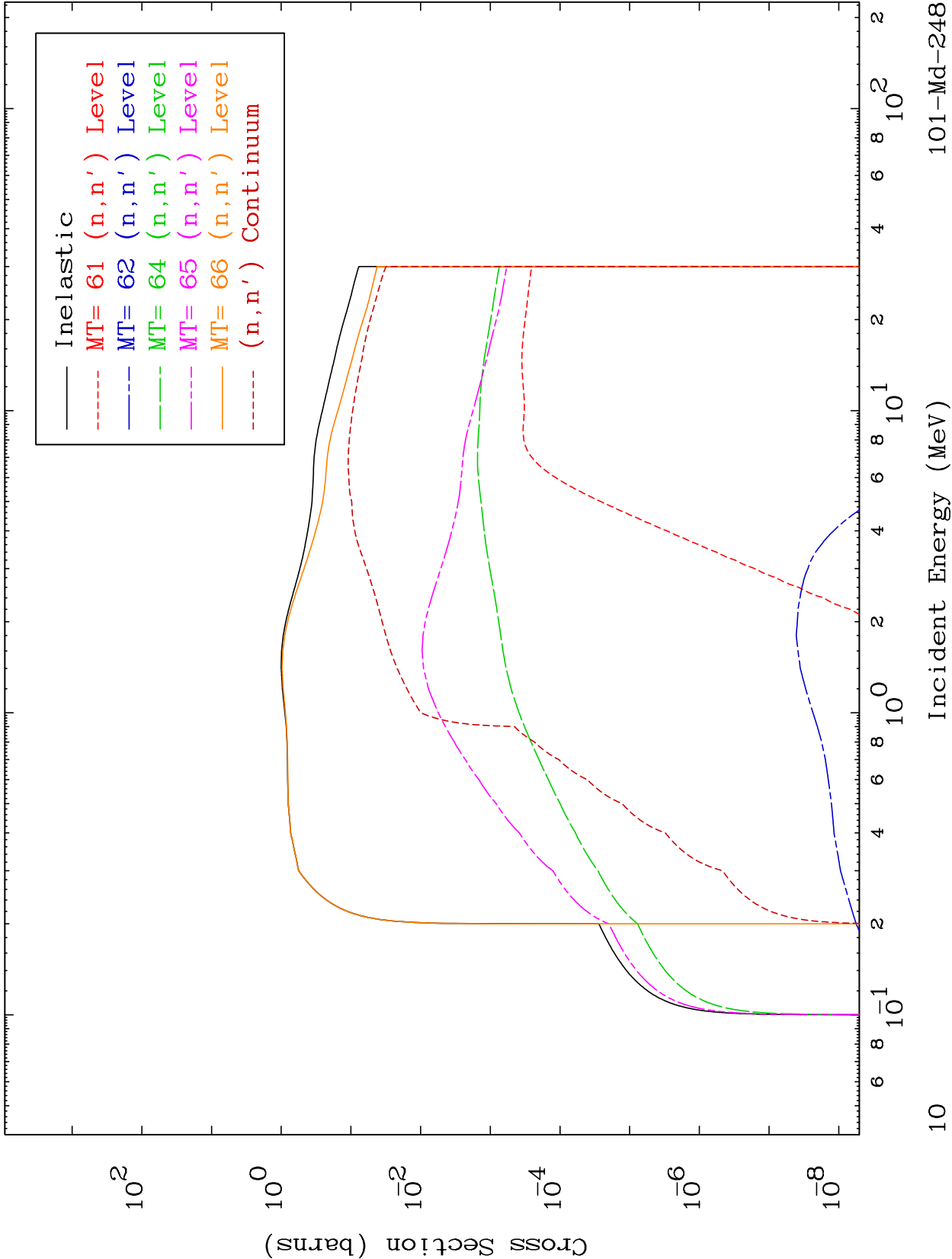


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(n,n') Levels

101-Md-248

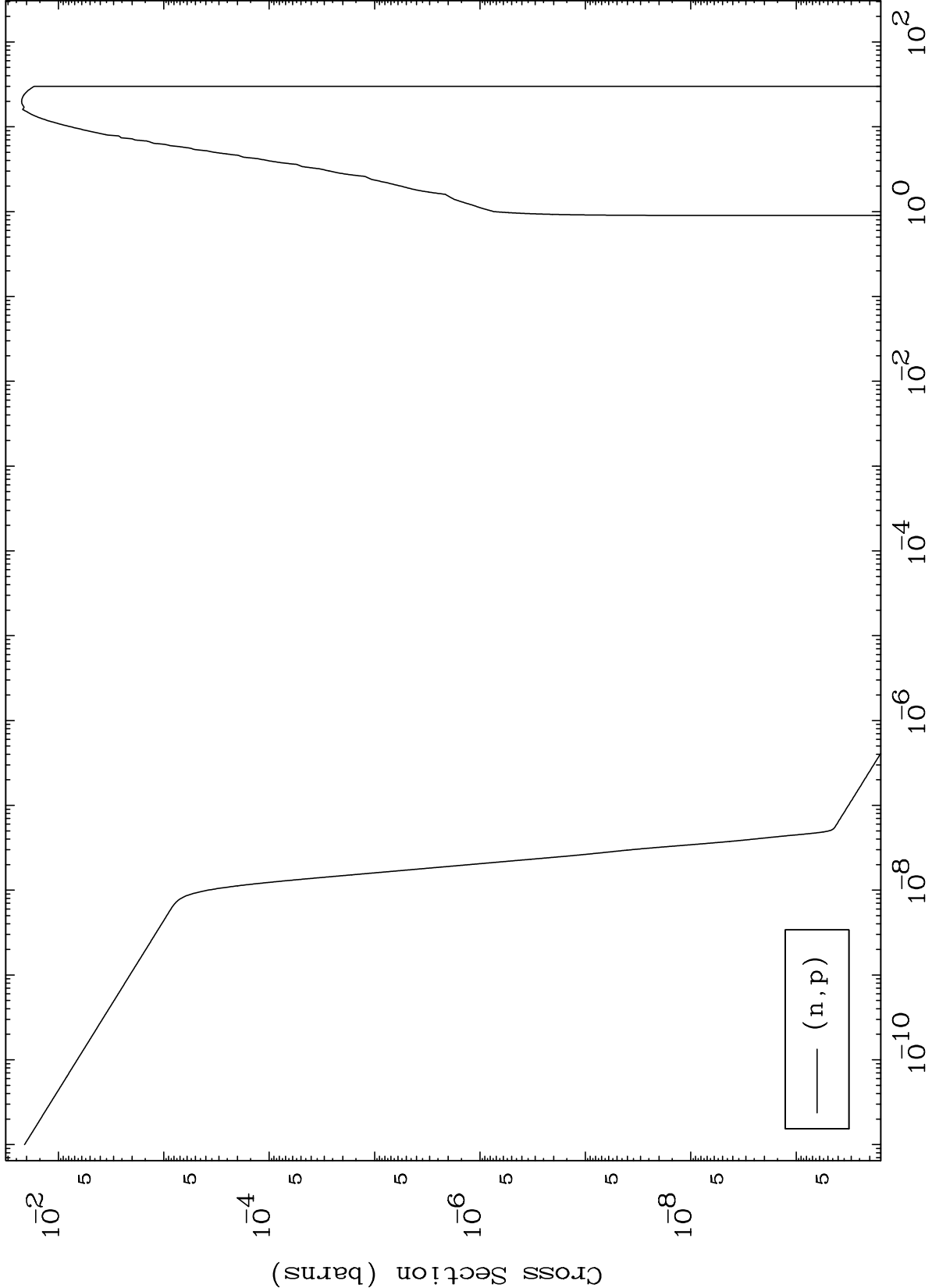
293 Kelvin Cross Sections



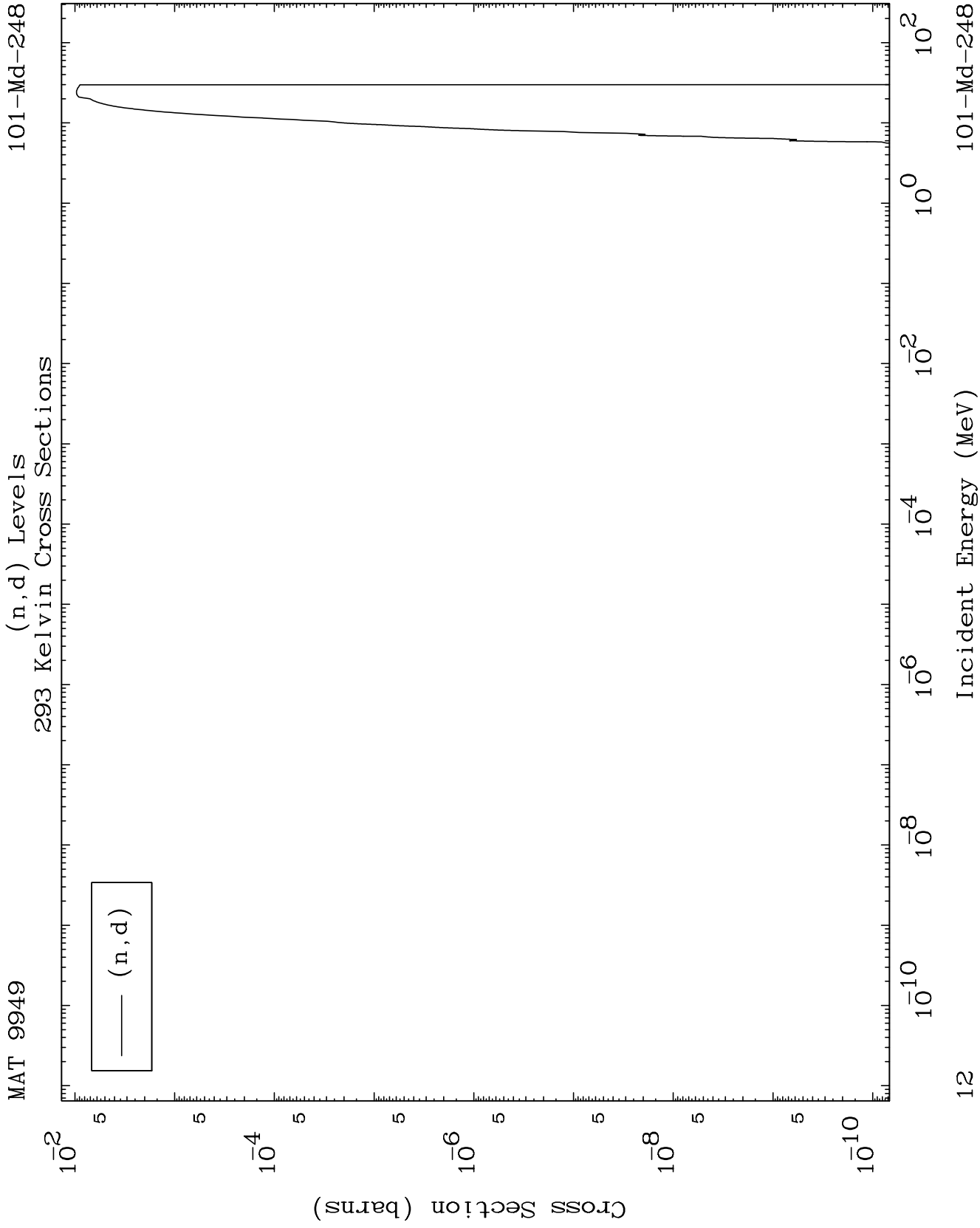
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(n,p) Levels
293 Kelvin Cross Sections

101-Md-248



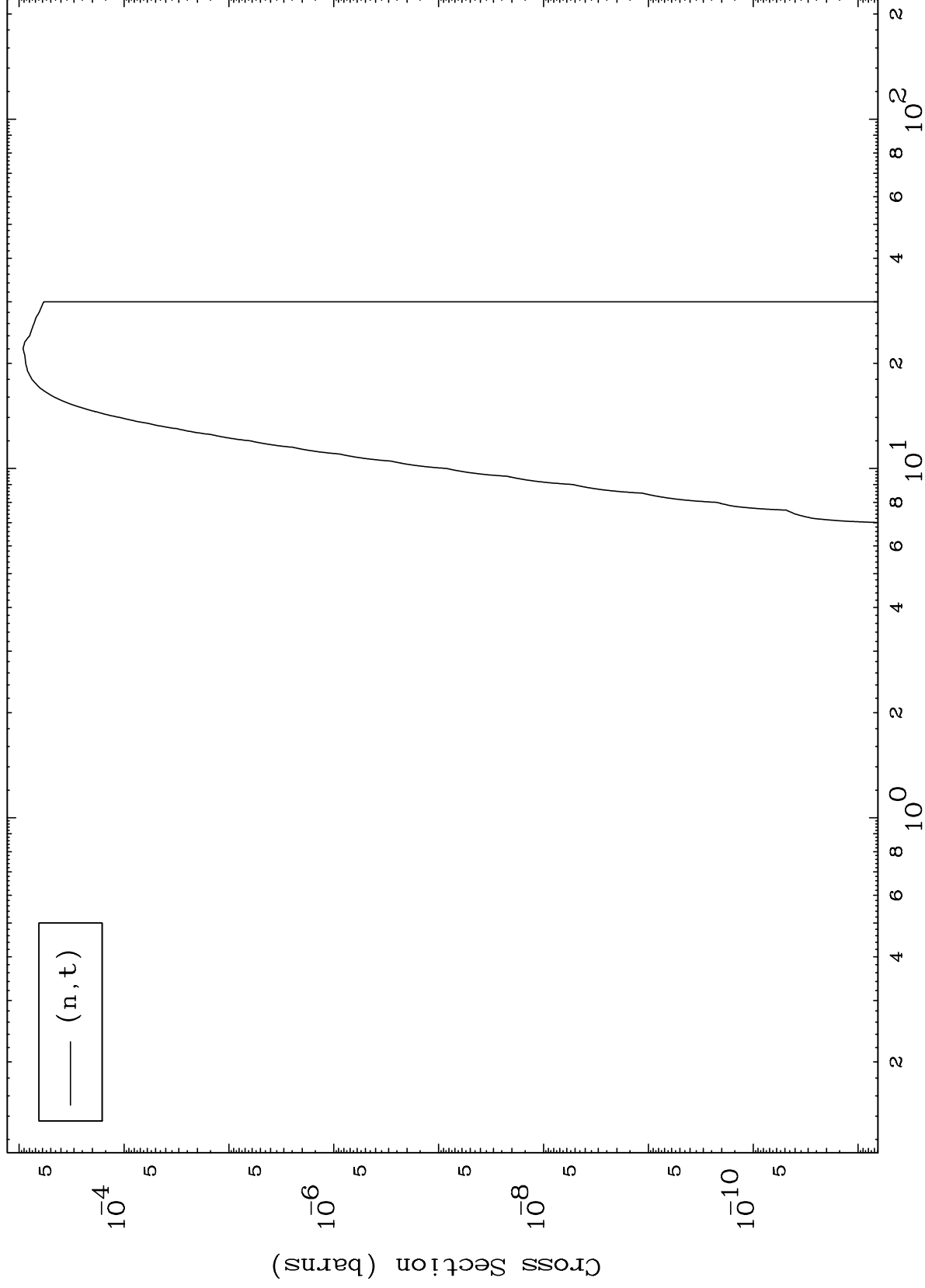
— (n,p)



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101-Md-248

(n,t) Levels
293 Kelvin Cross Sections



13

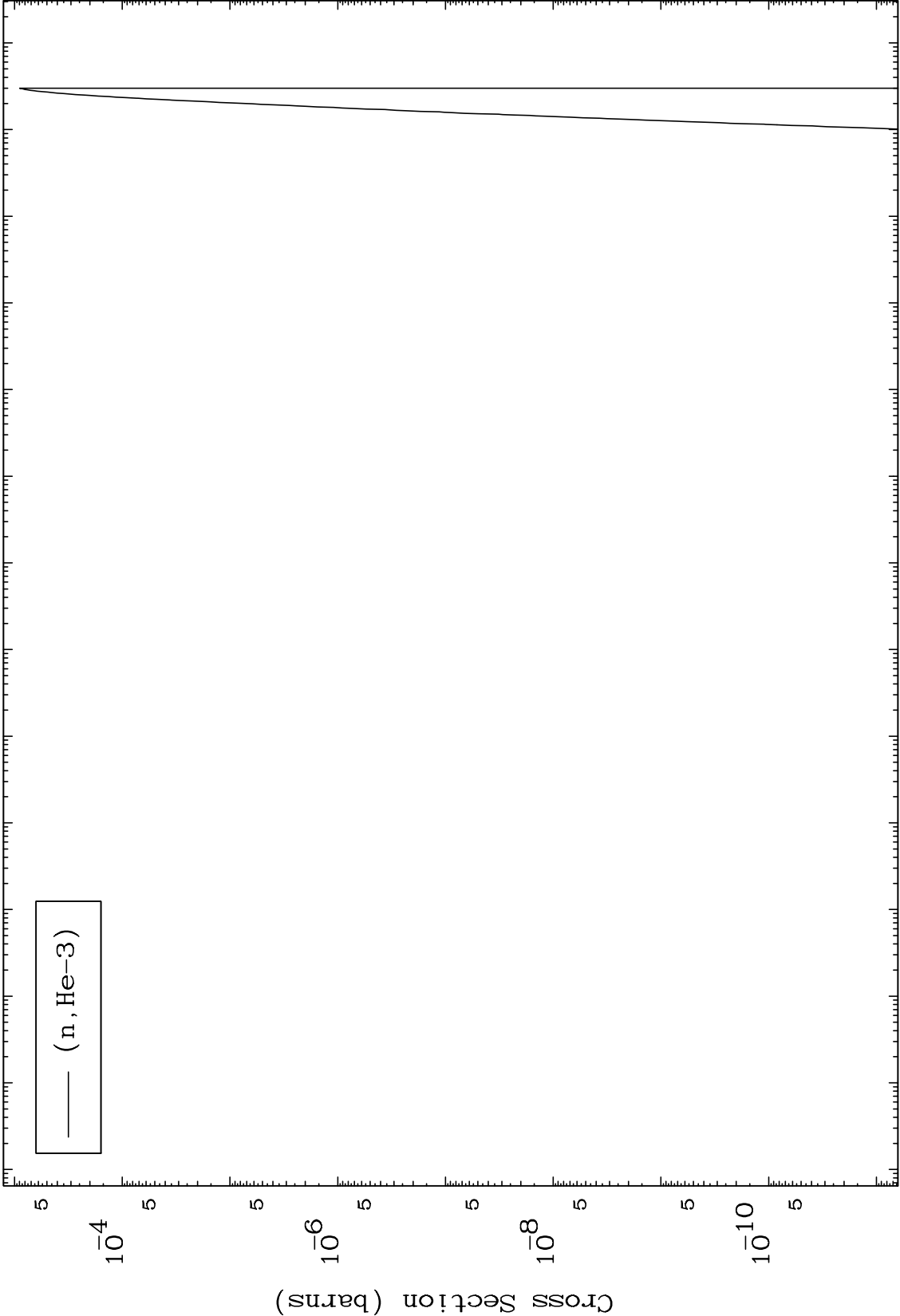
Incident Energy (MeV)

101-Md-248

MAT 9949

(n,He3) Levels
293 Kelvin Cross Sections

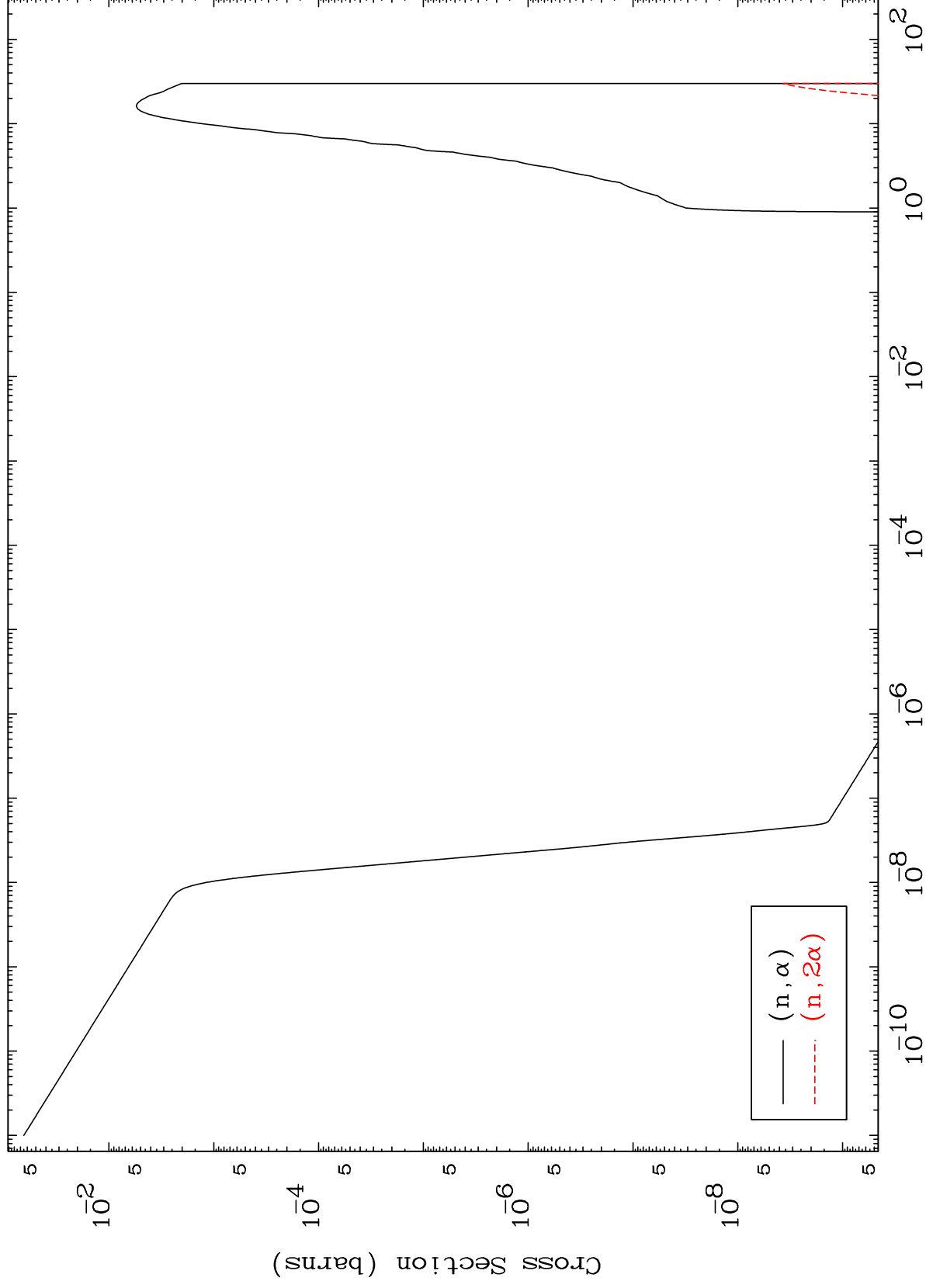
101-Md-248



MAT 9949

(n, α) Levels
293 Kelvin Cross Sections

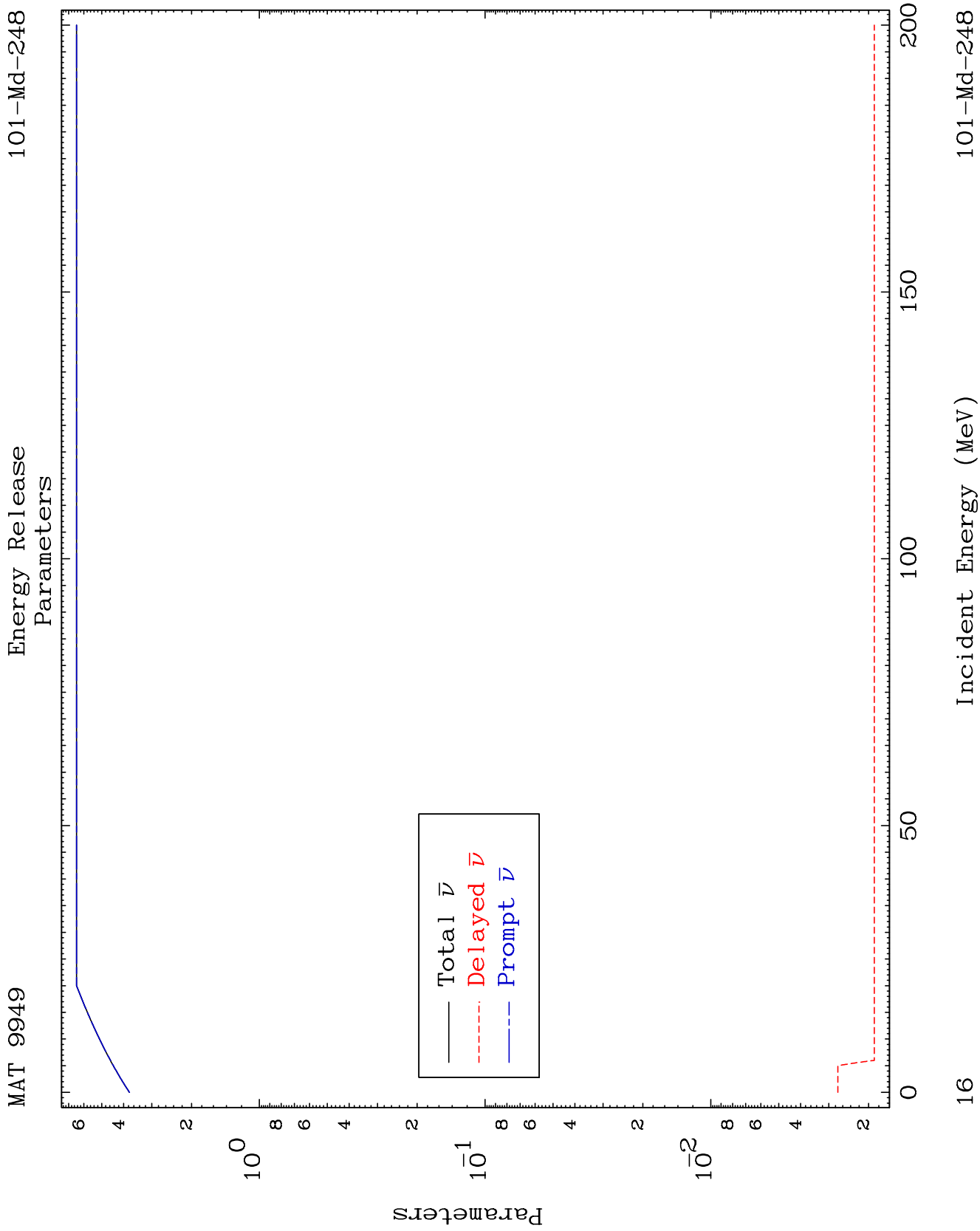
101-Md-248



15

Incident Energy (MeV)

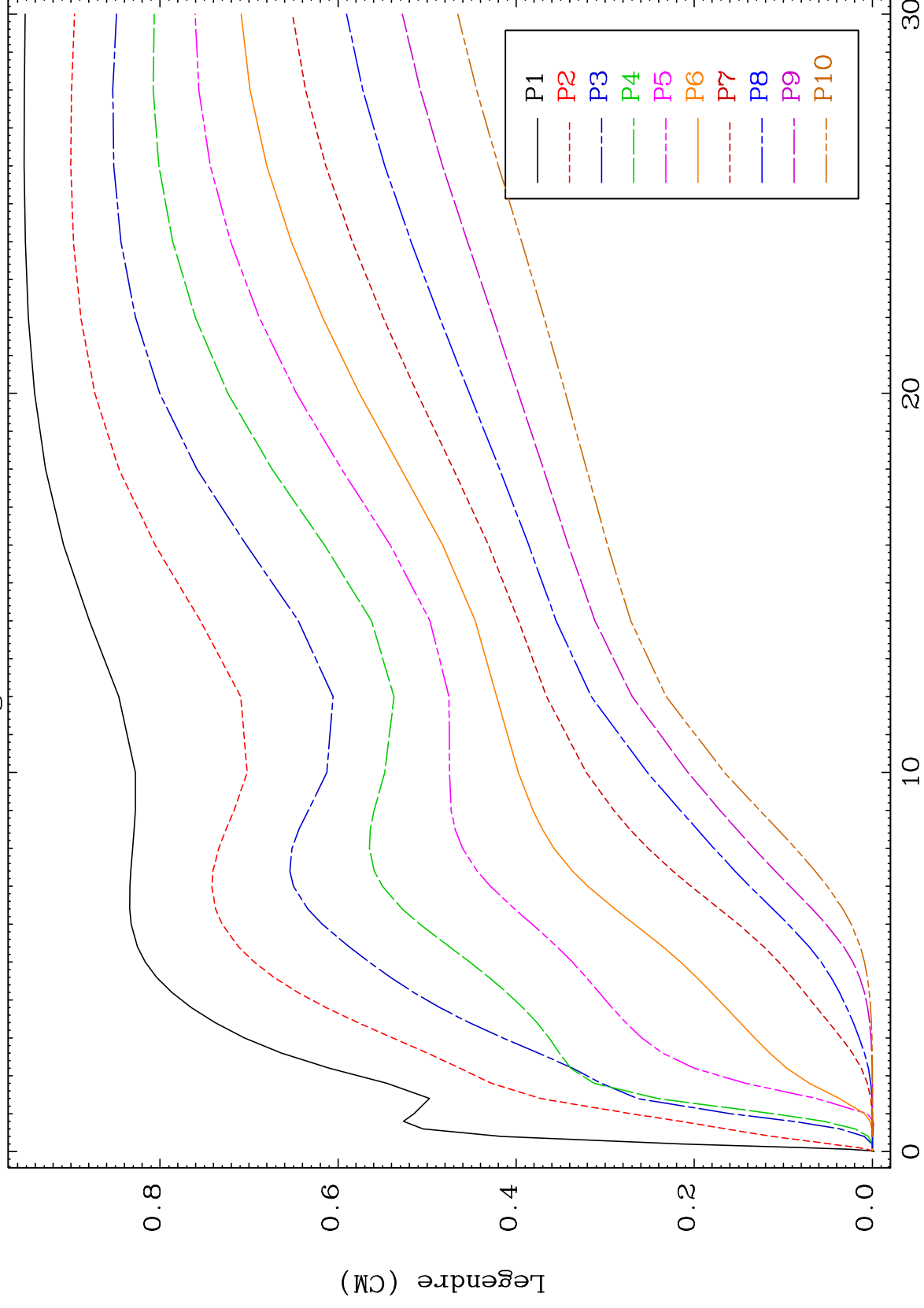
101-Md-248



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101-Md-248

Elastic Legendre Coefficients



21

Incident Energy (MeV)

101-Md-248

30

20

10

0

P1

-----P2

— P.3

-----P4

P5

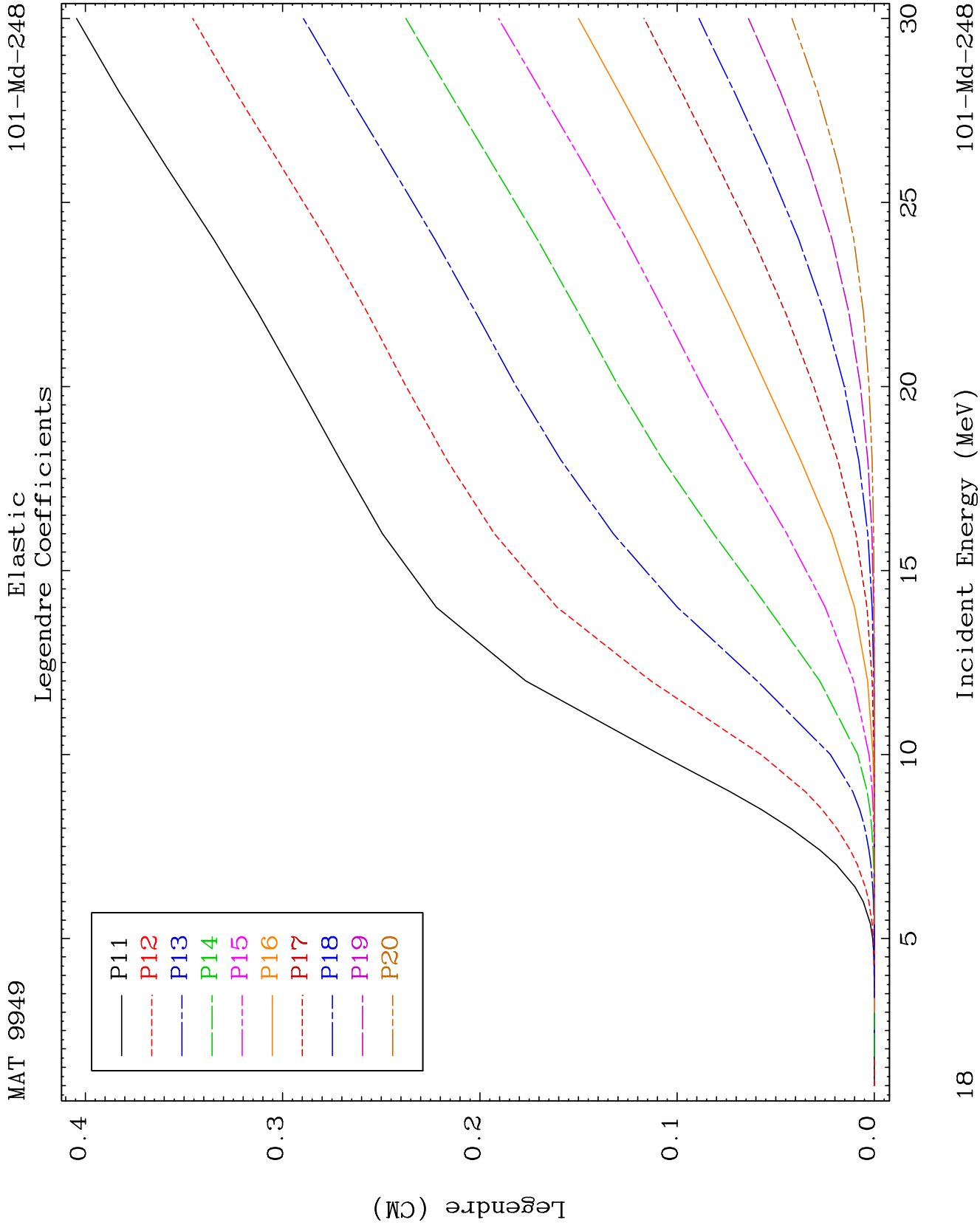
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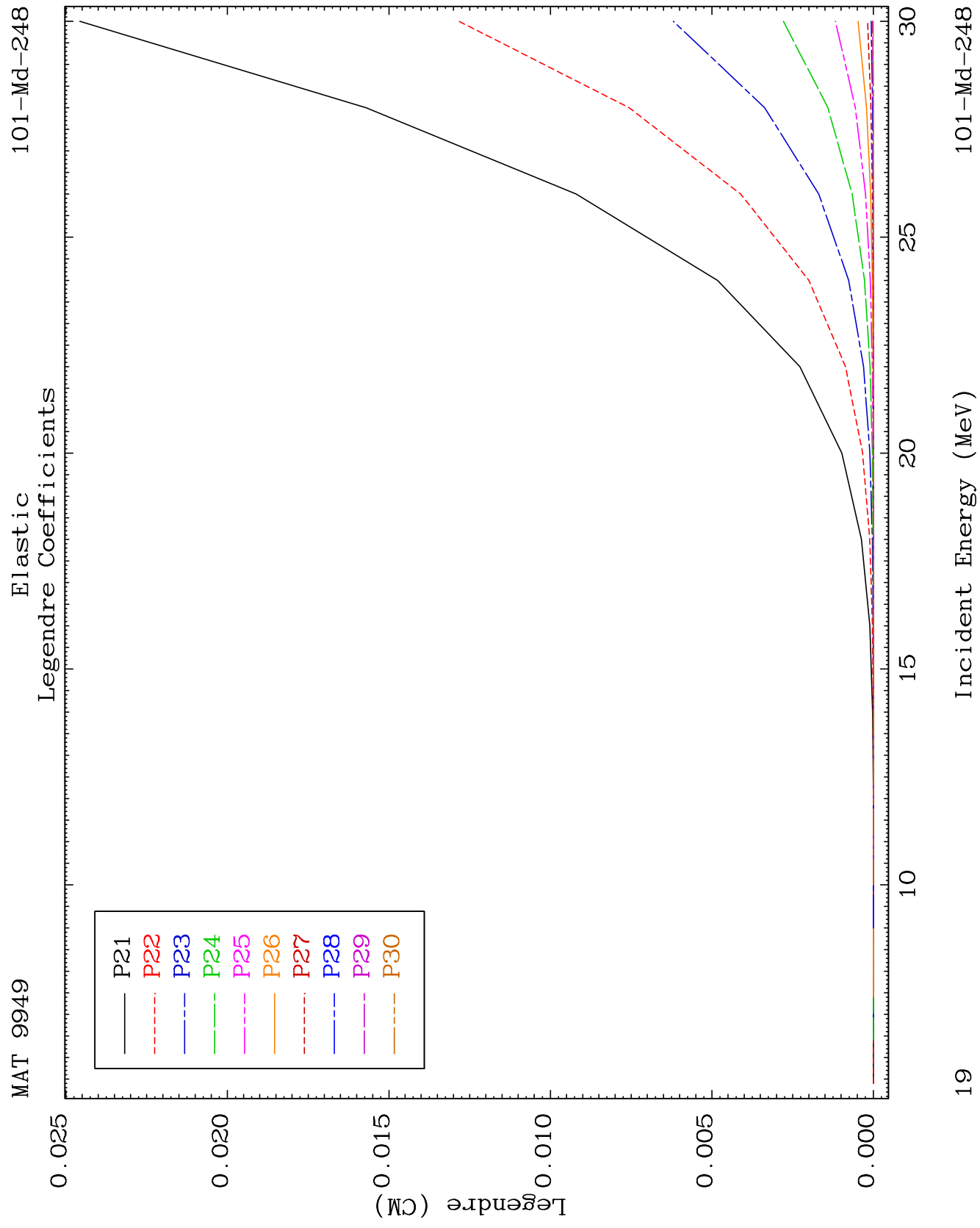
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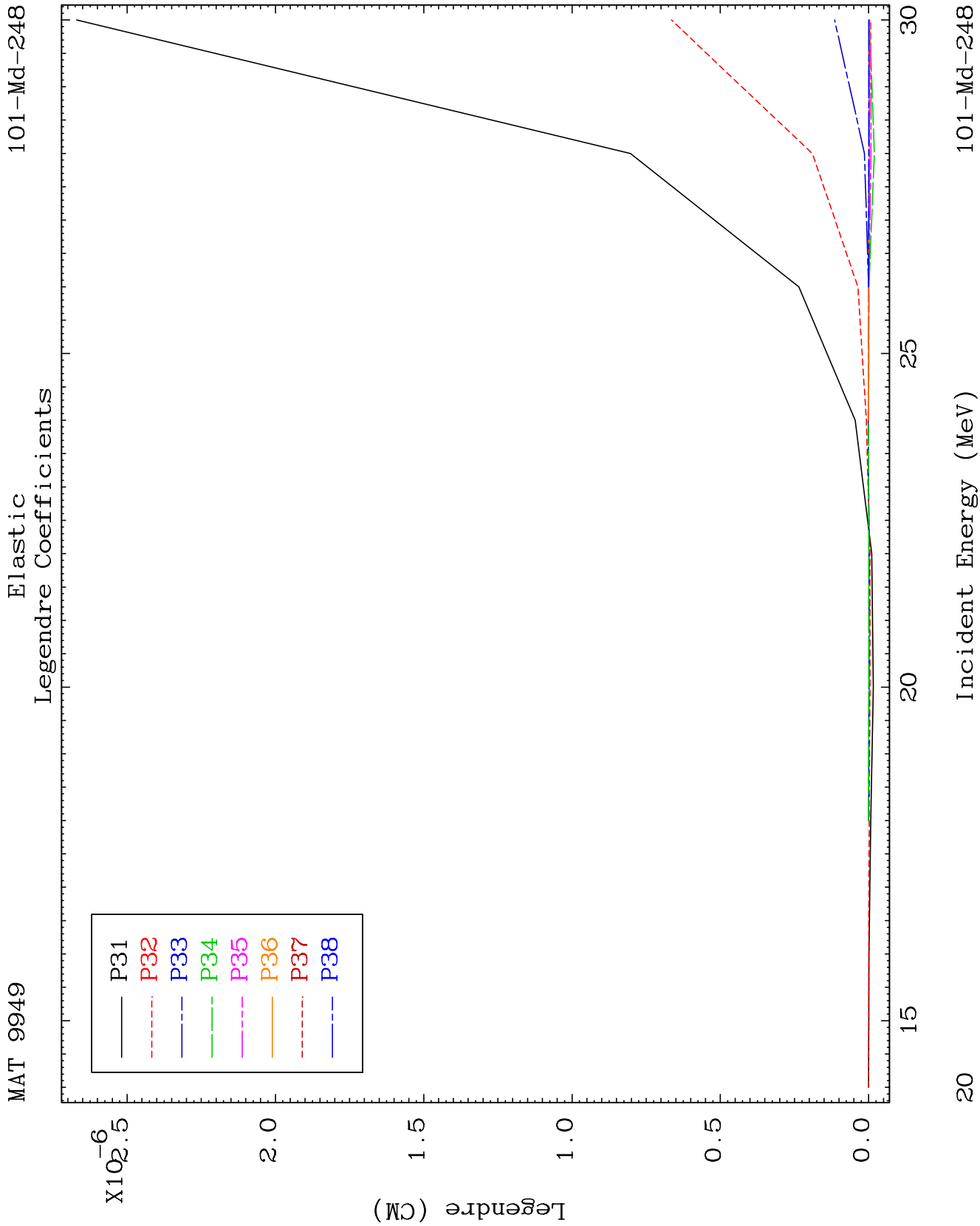
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PC

D10



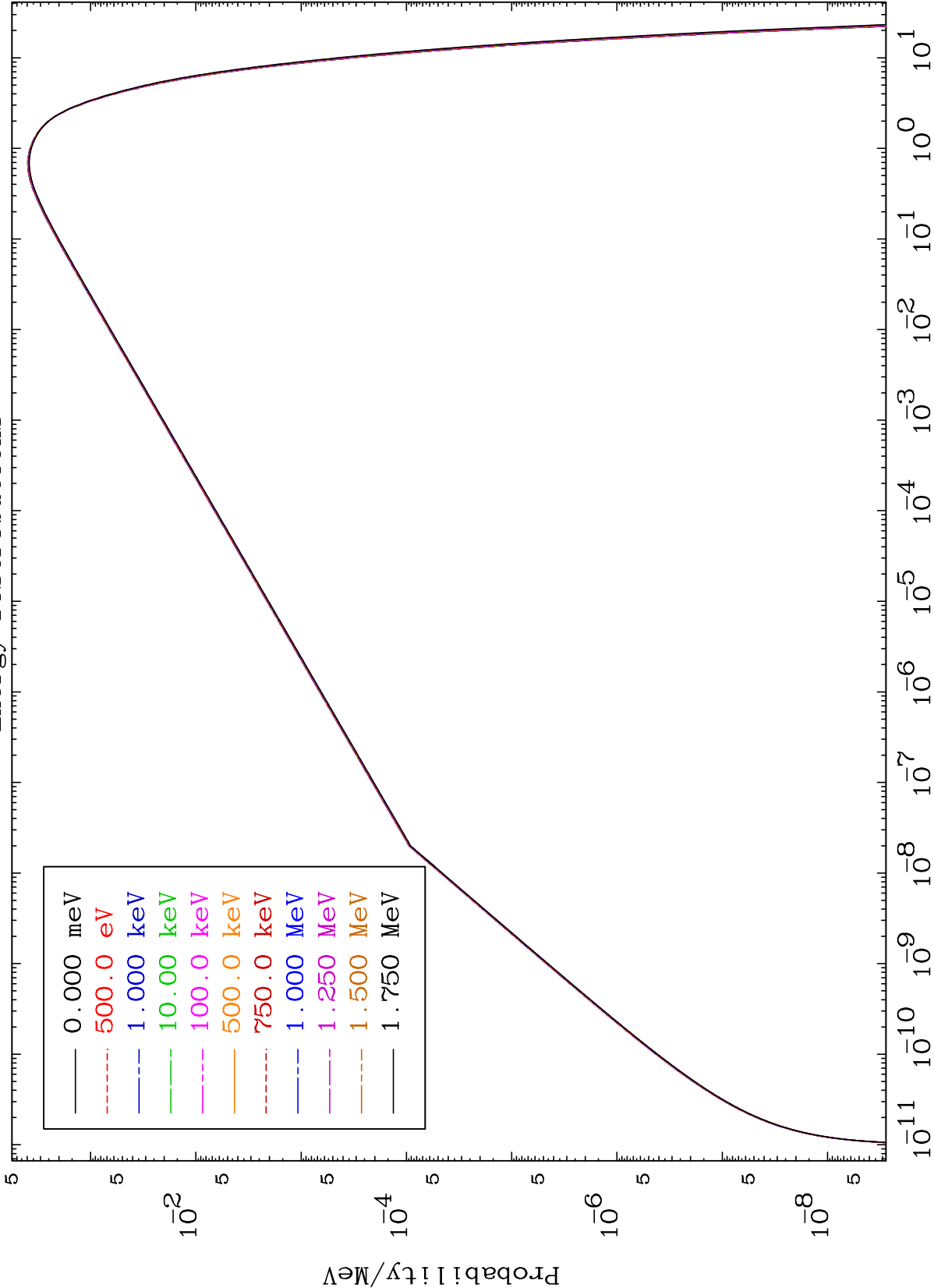


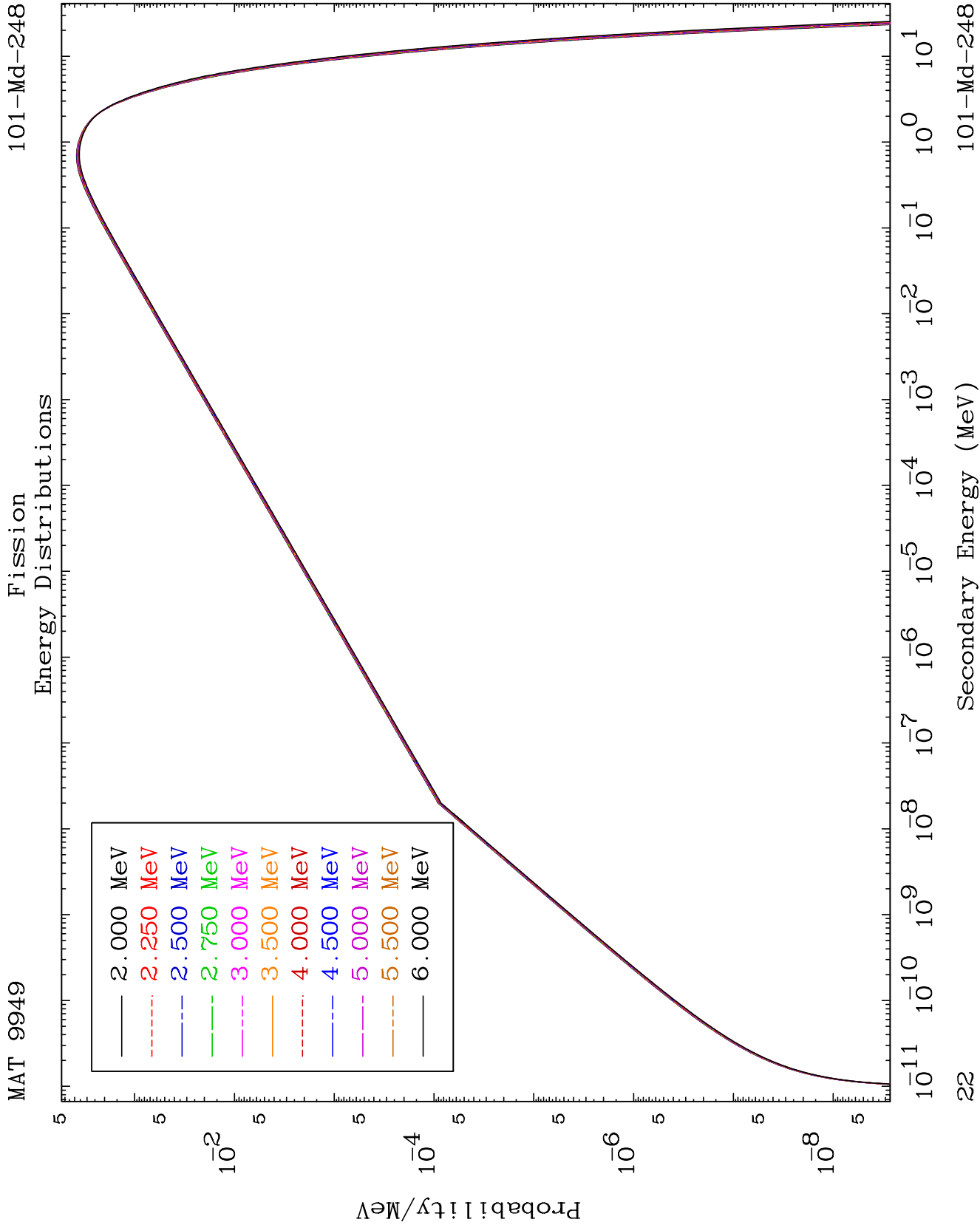


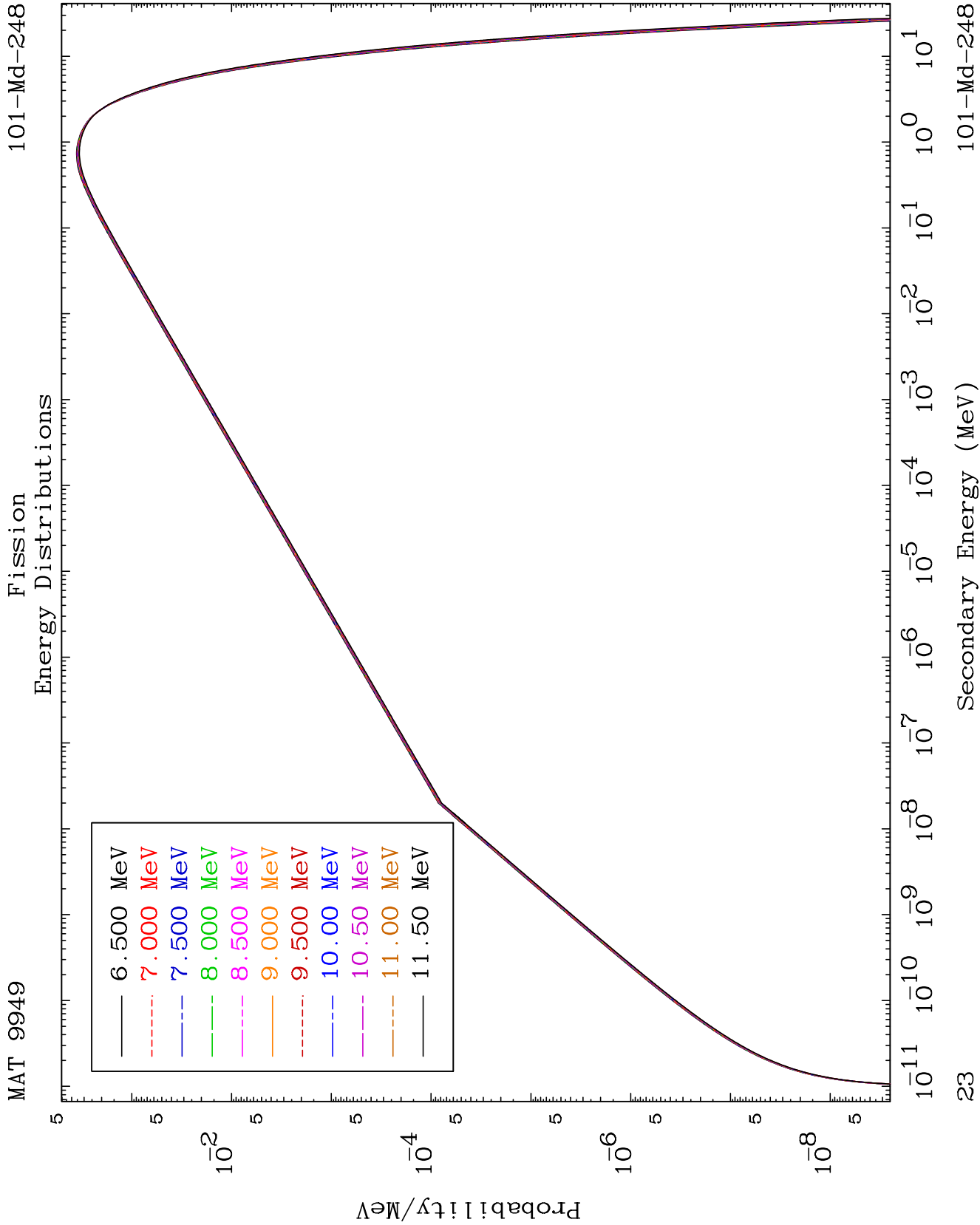
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Fission
Energy Distributions

101-Md-248



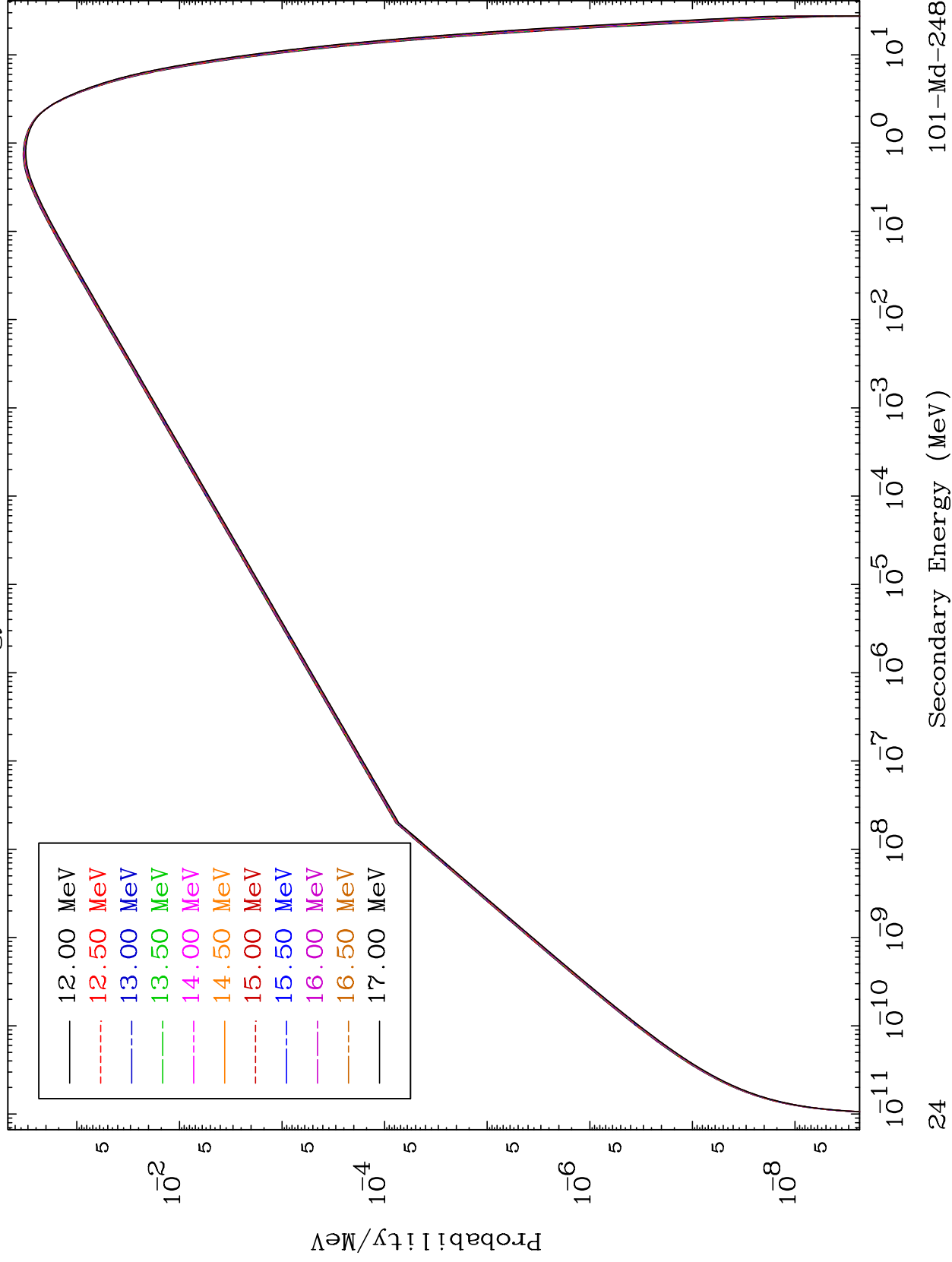




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Fission
Energy Distributions

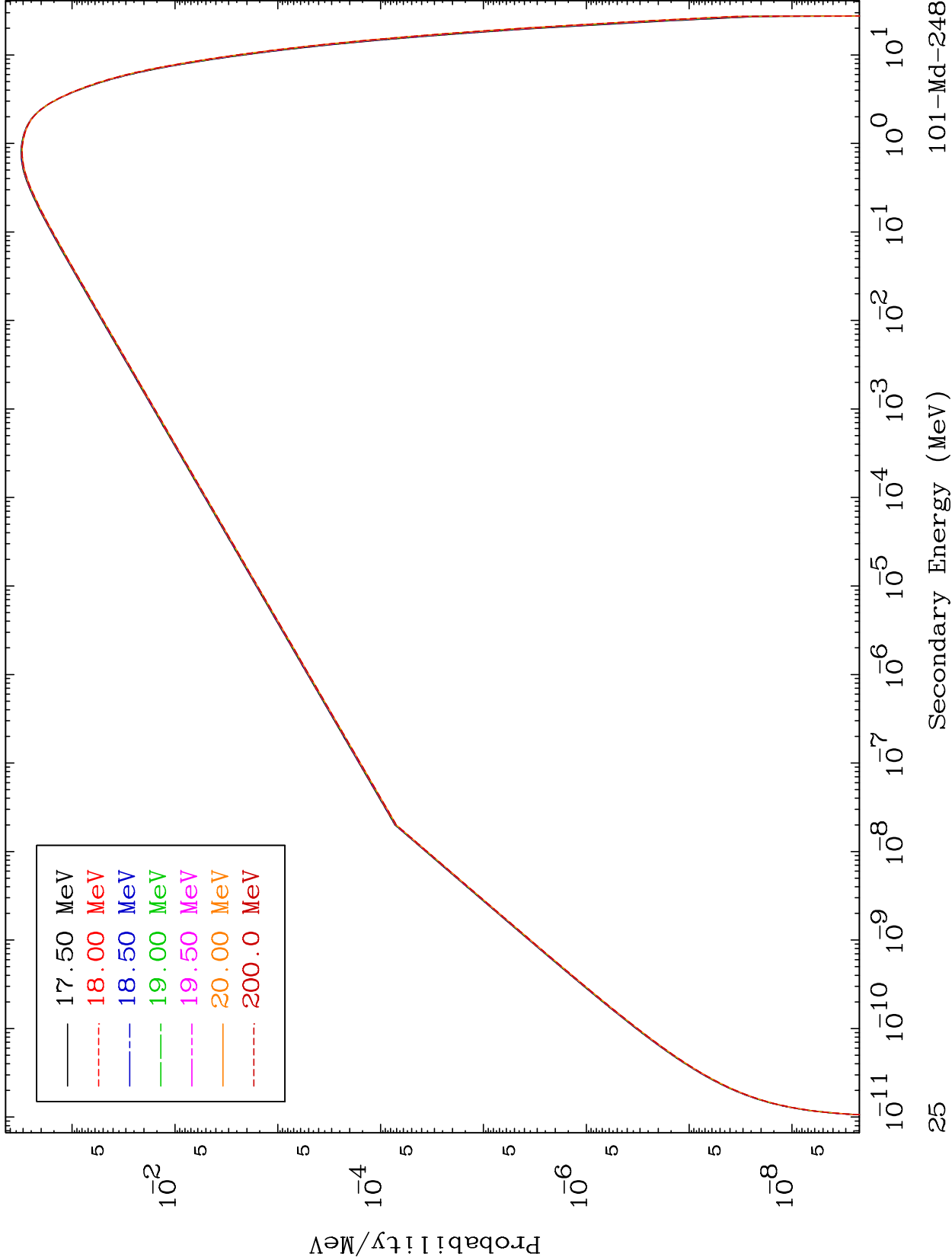
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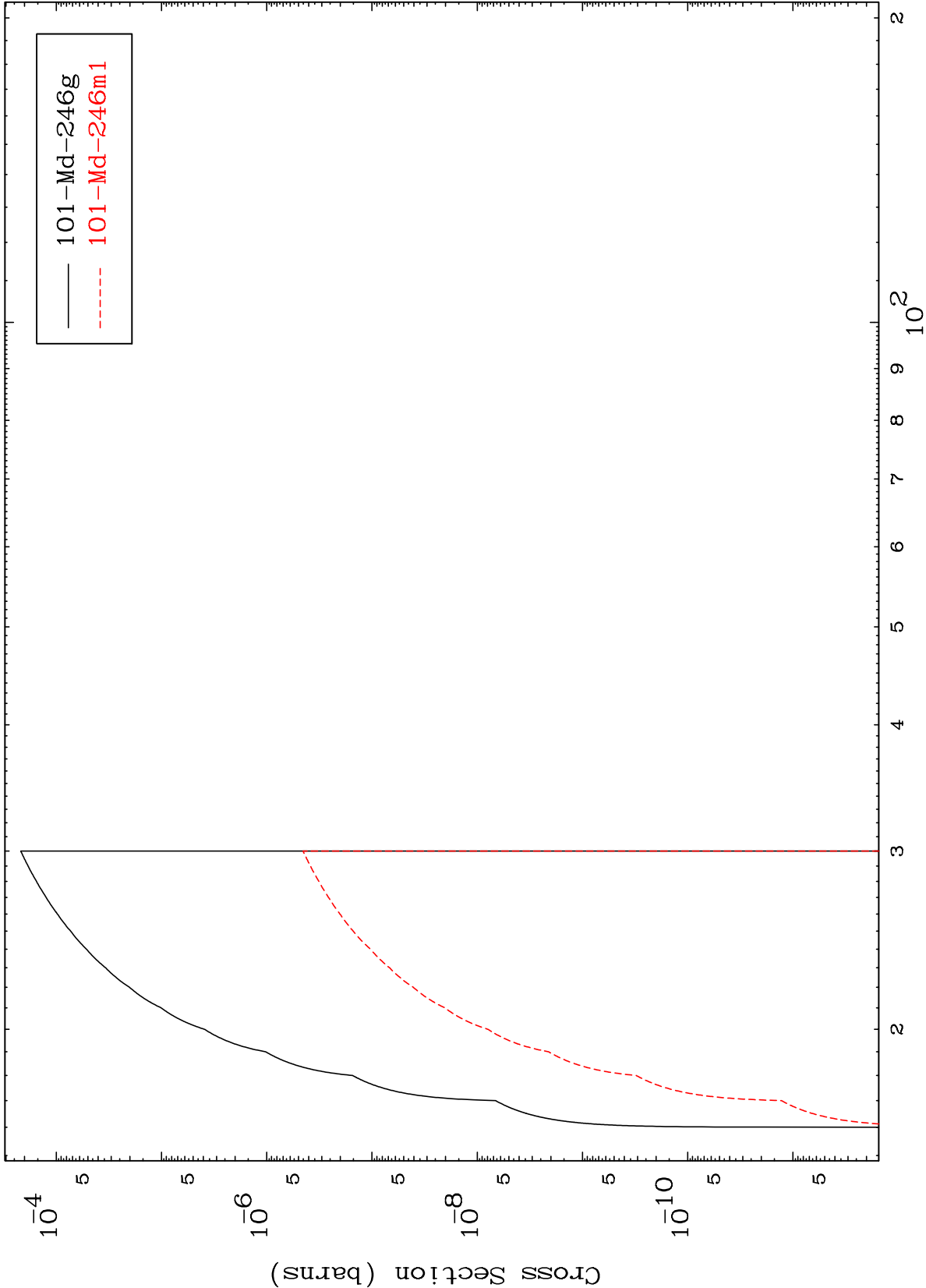
MAT 9949

Fission
Energy Distributions

101-Md-248



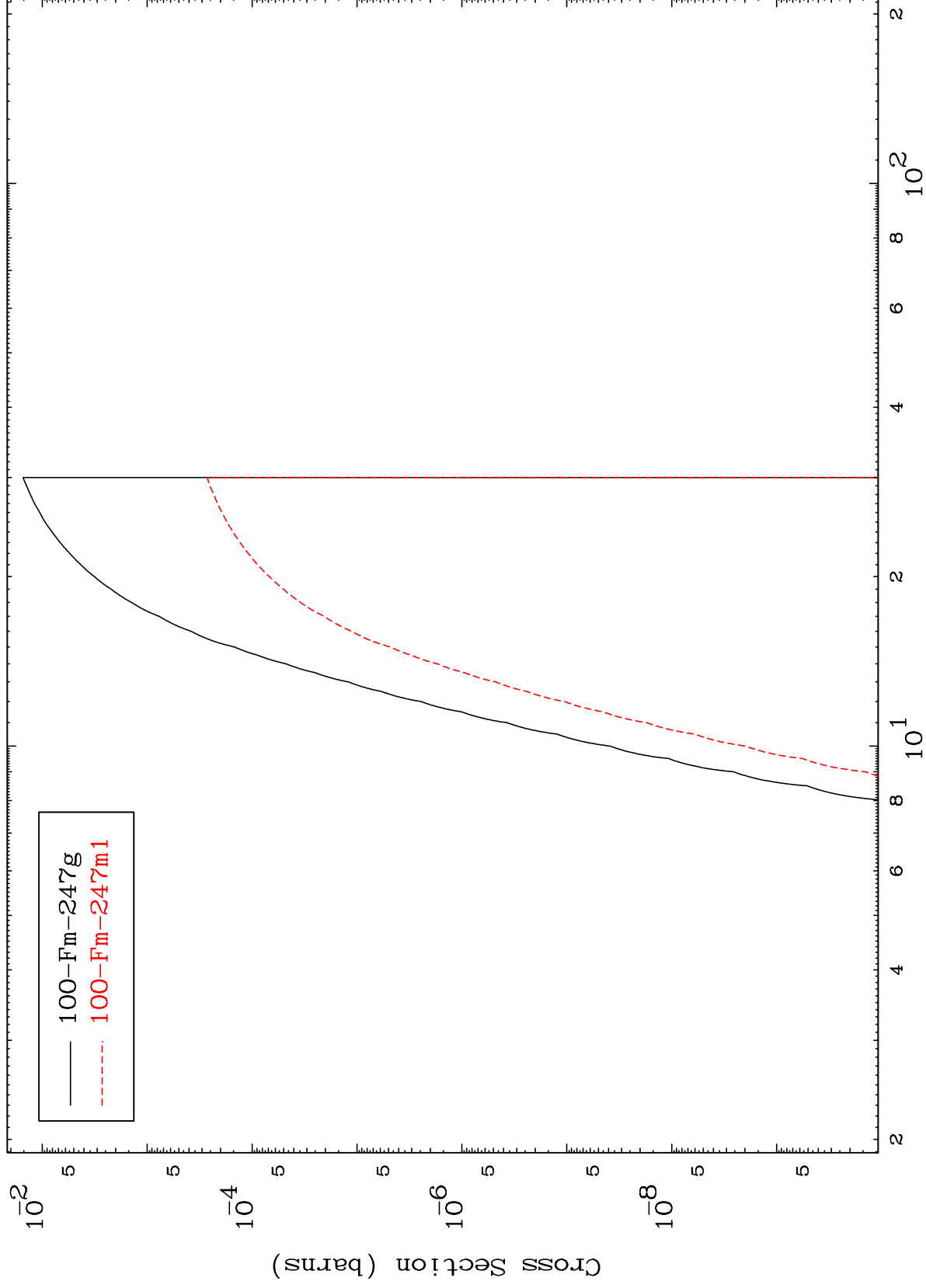
(n,3n)
Radionuclide Production Cross Section



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101-Md-248

(n,n') p
Radionuclide Production Cross Section



27

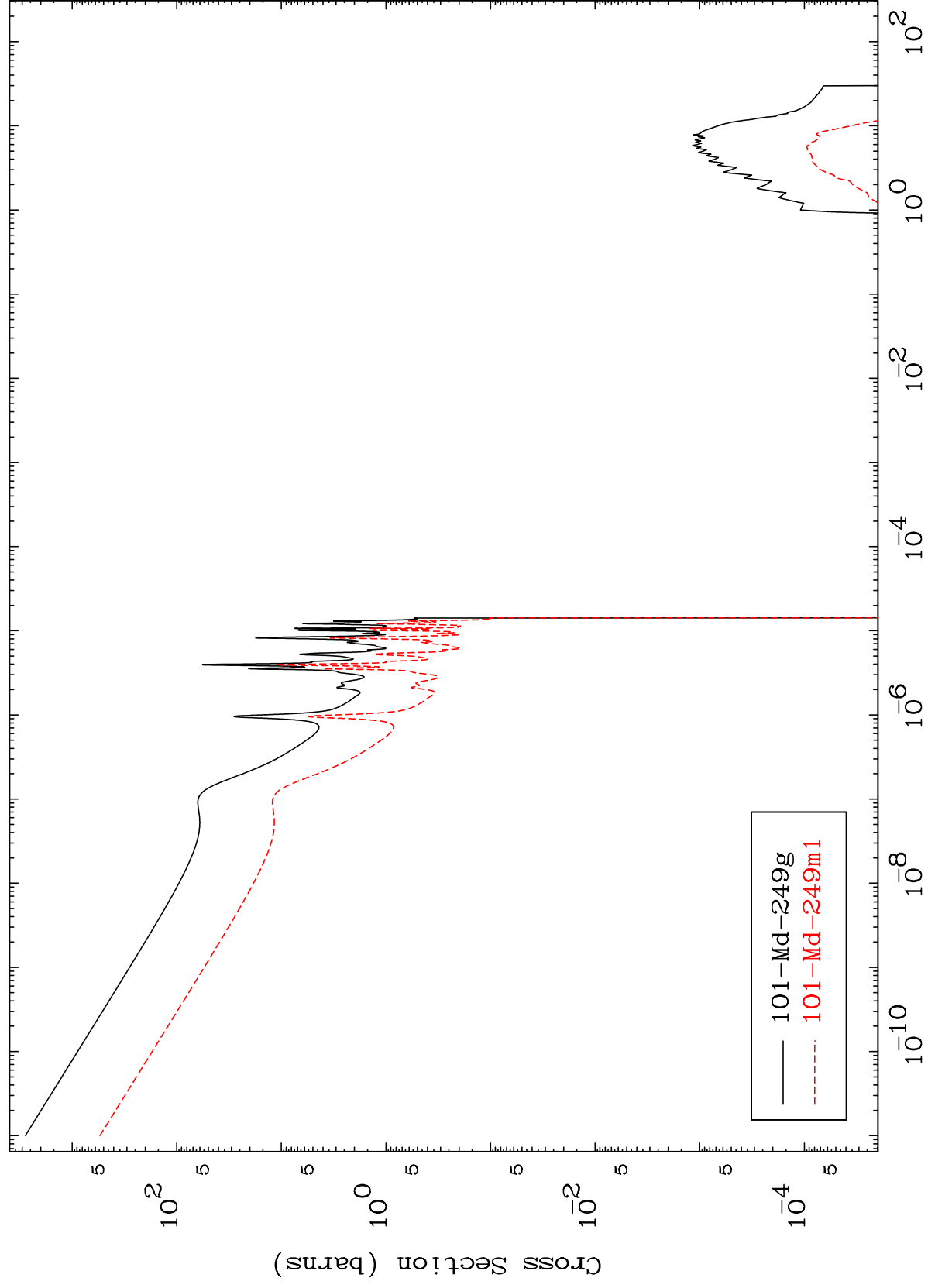
Incident Energy (MeV)

101-Md-248

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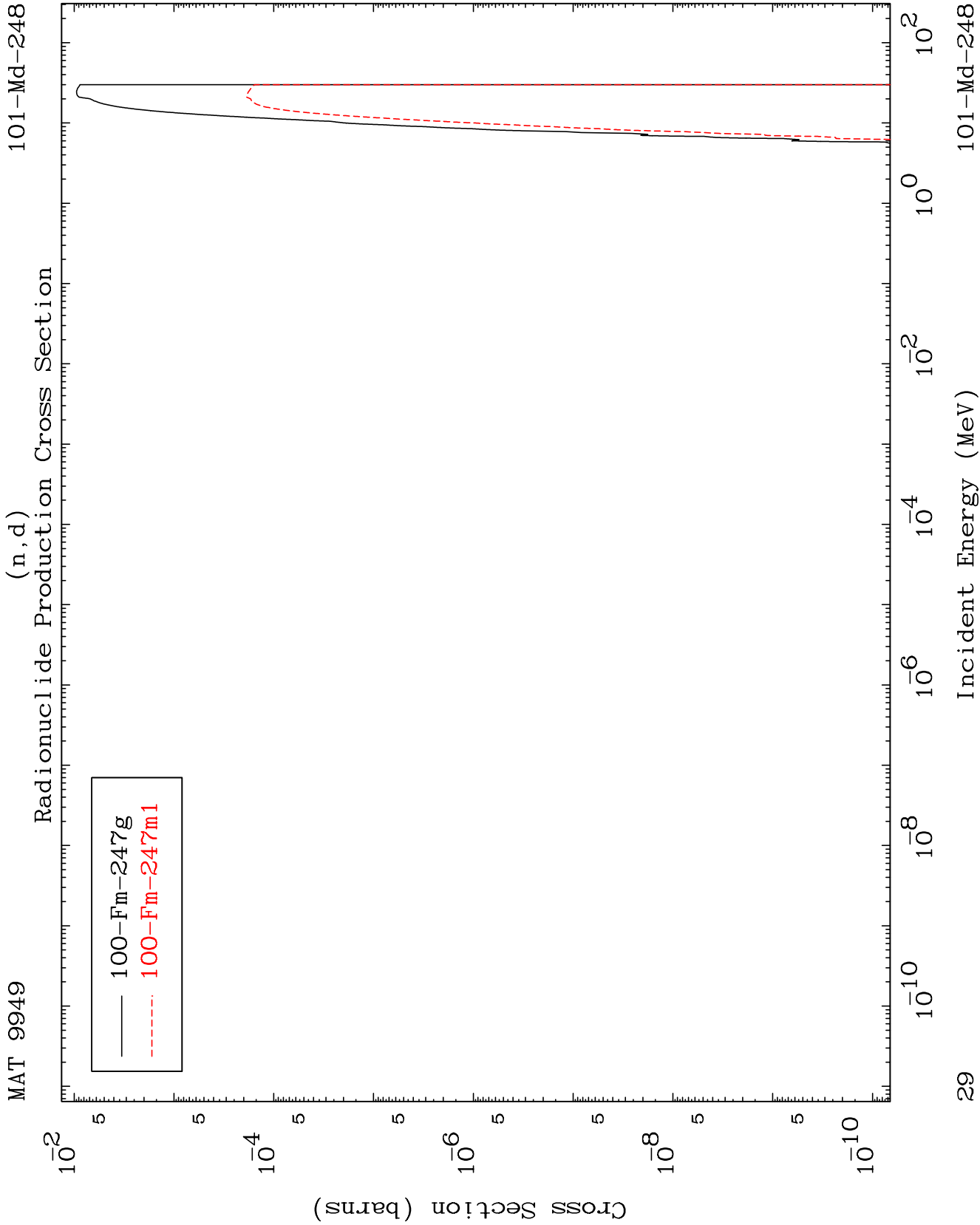
101-Md-248

(n, γ)
Radionuclide Production Cross Section



101-Md-248

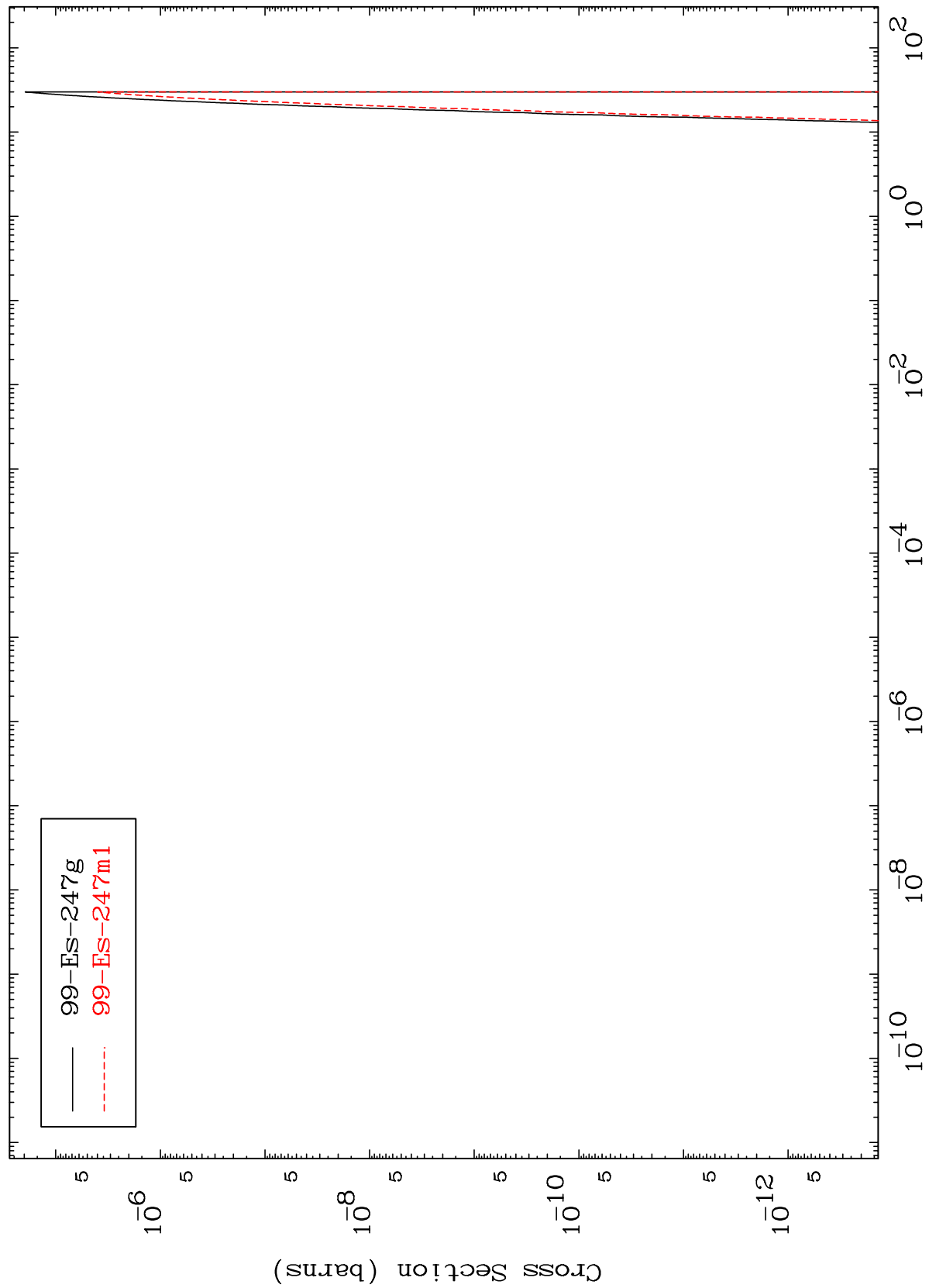
Incident Energy (MeV)



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101-Md-248

(n,2p)
Radionuclide Production Cross Section



30

101-Md-248