

Program EVALPLOT
(Version 2021-1)

by

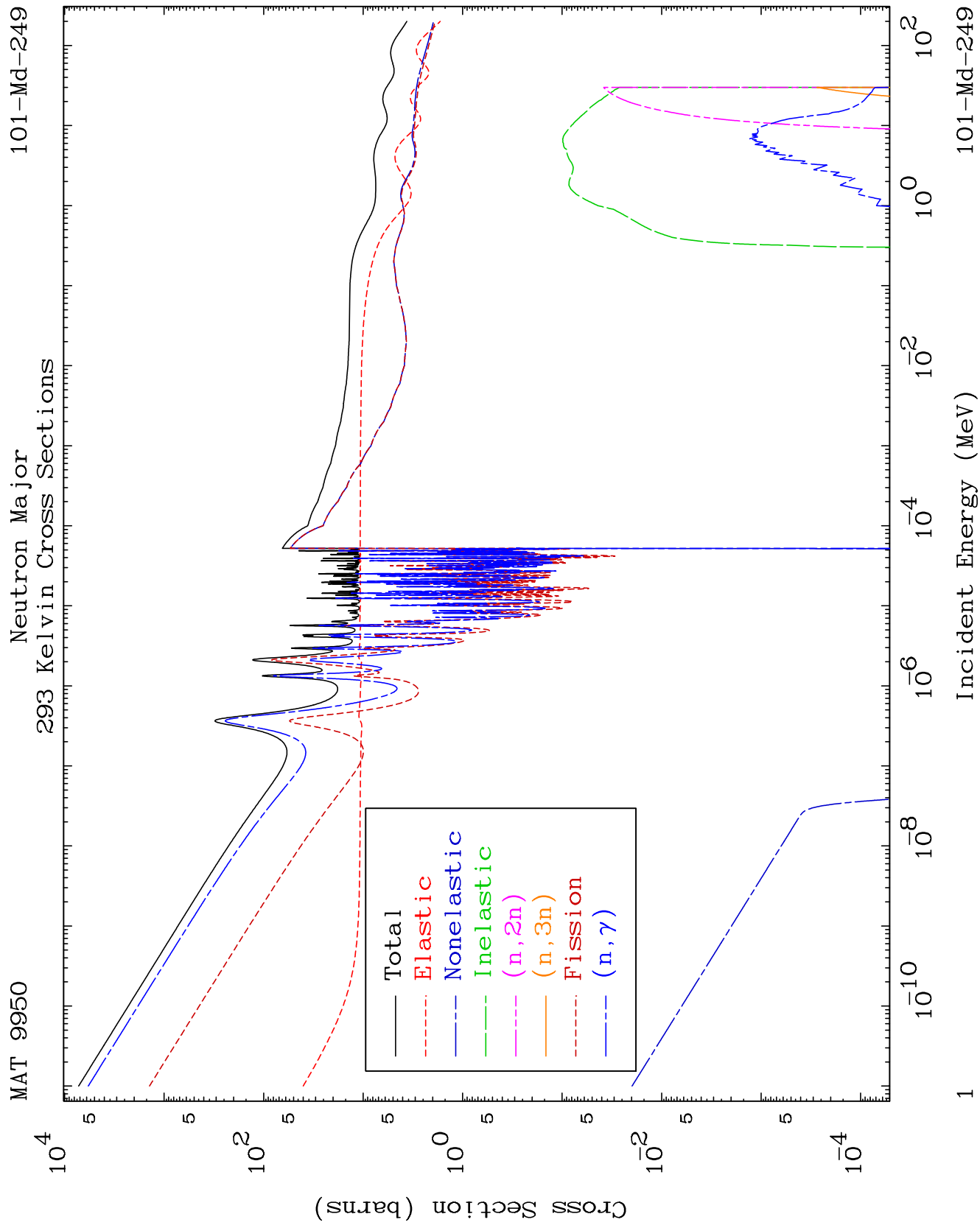
Dermott E. Cullen
(Present Contact Information)

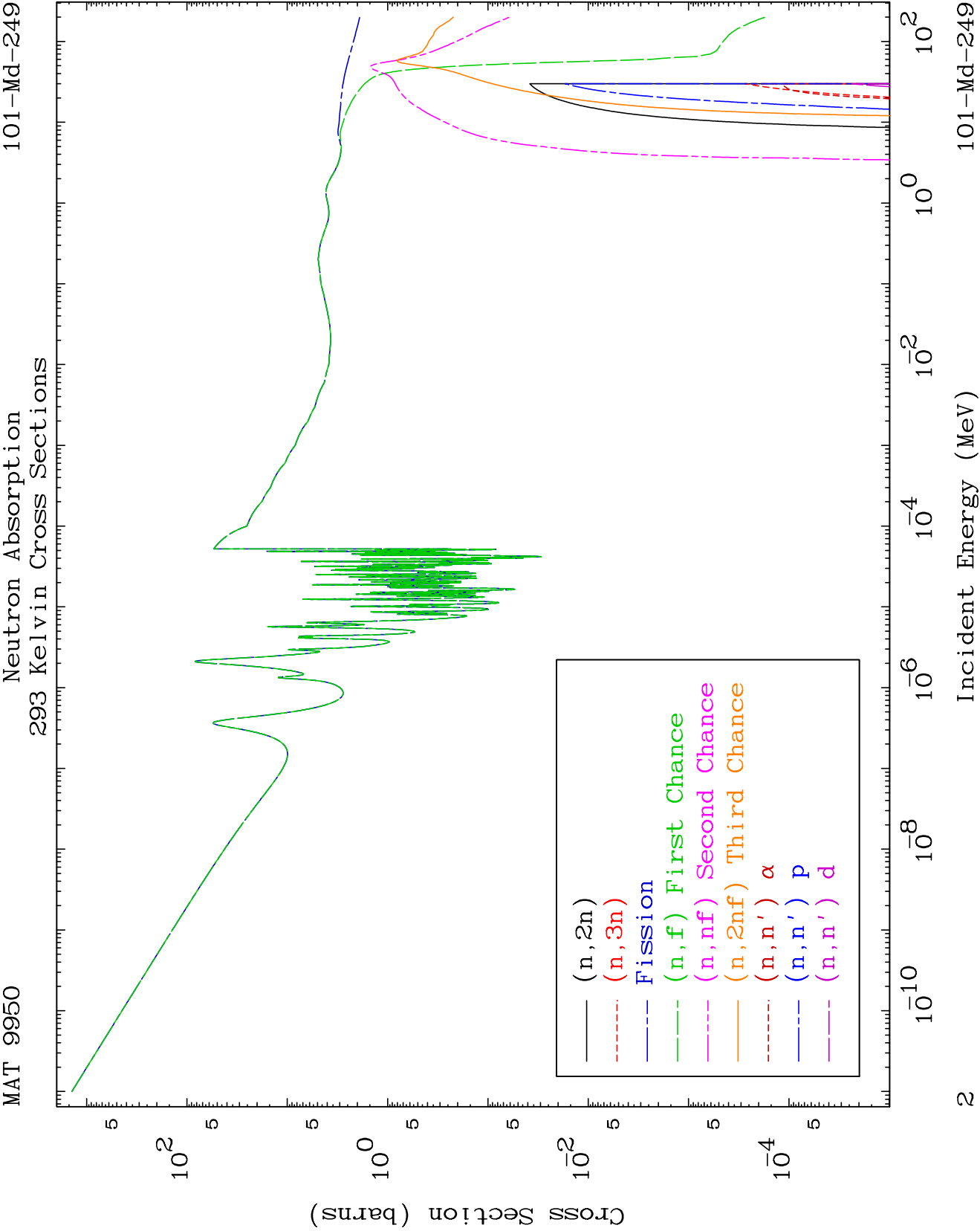
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1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

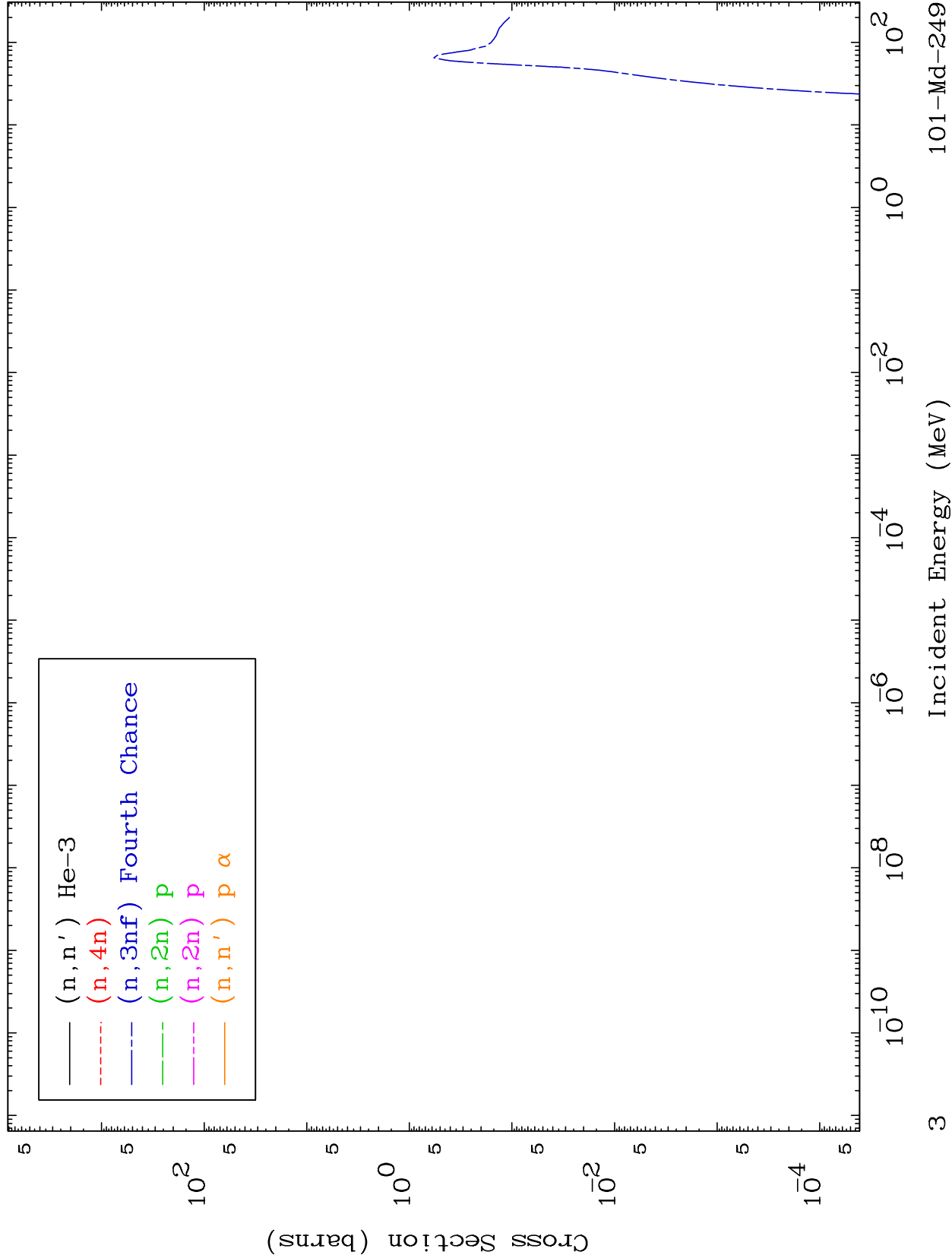


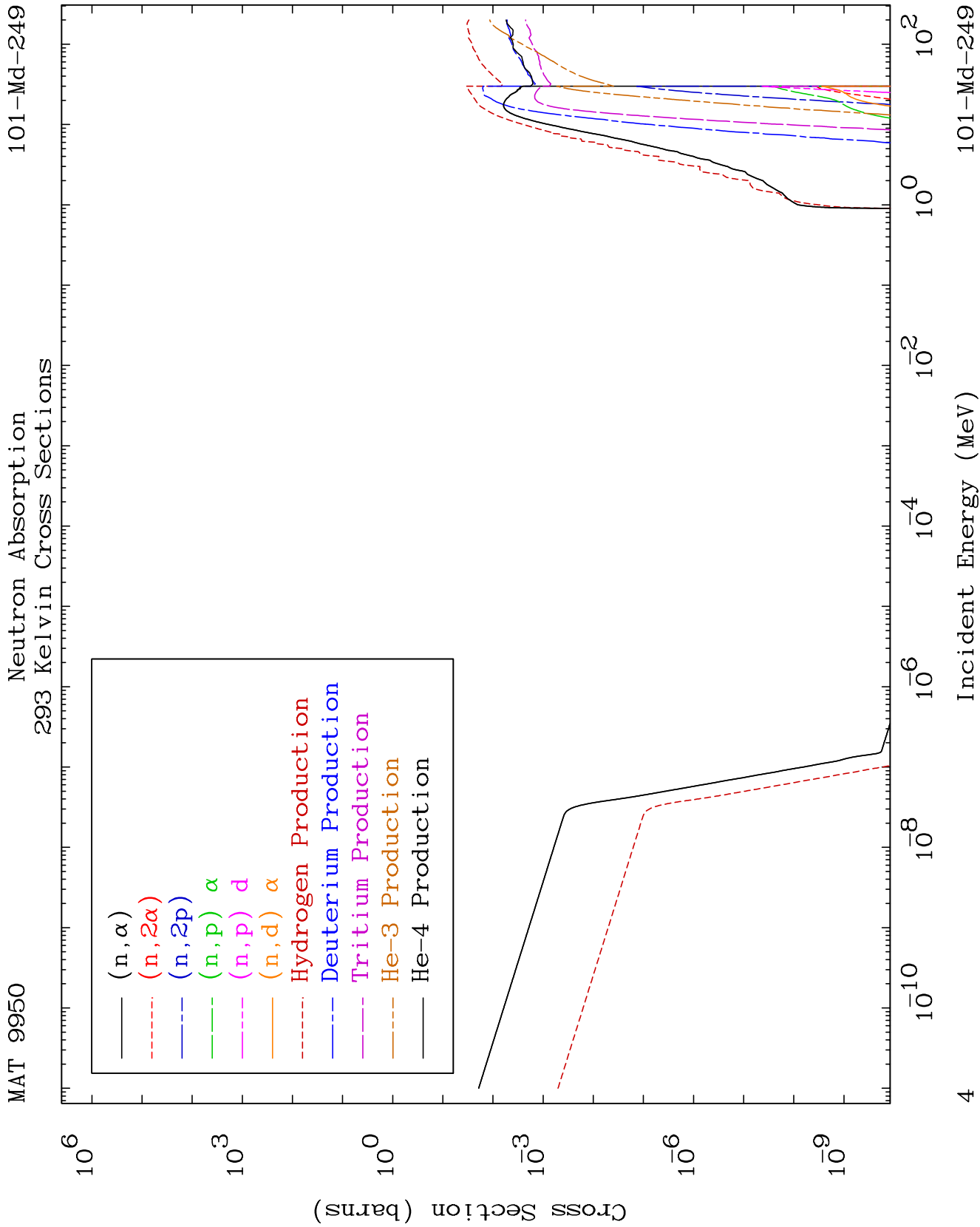


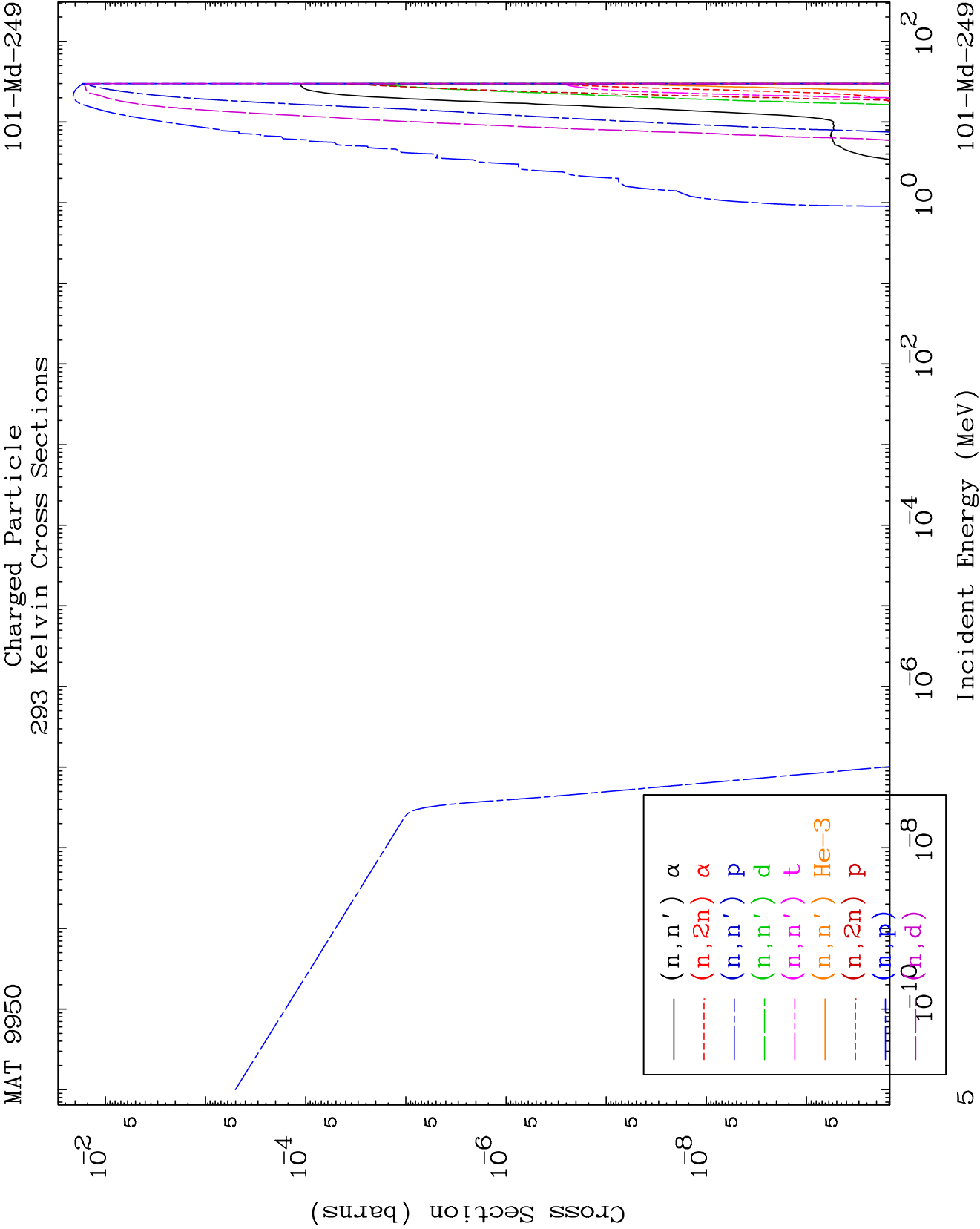
MAT 9950

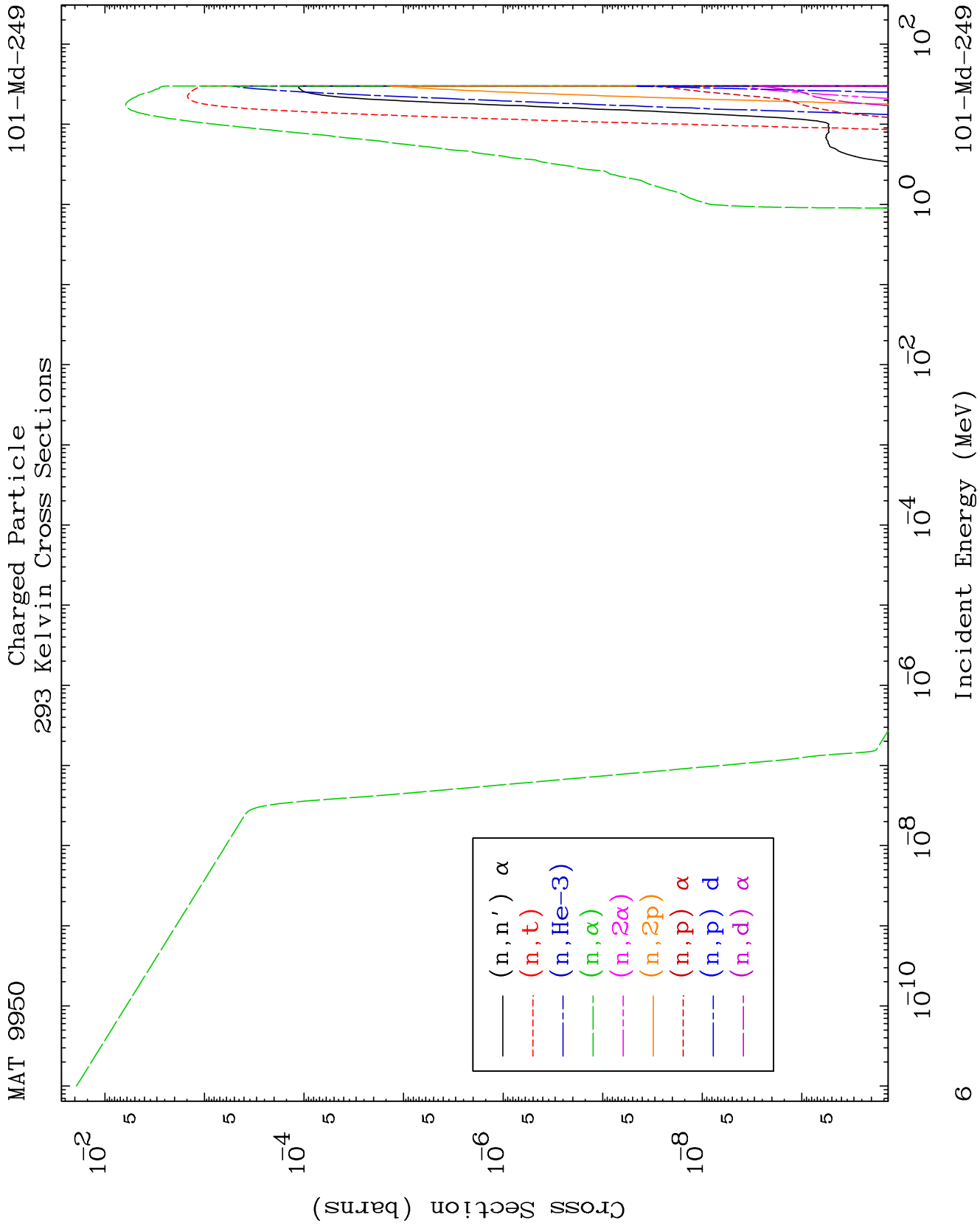
Neutron Absorption
293 Kelvin Cross Sections

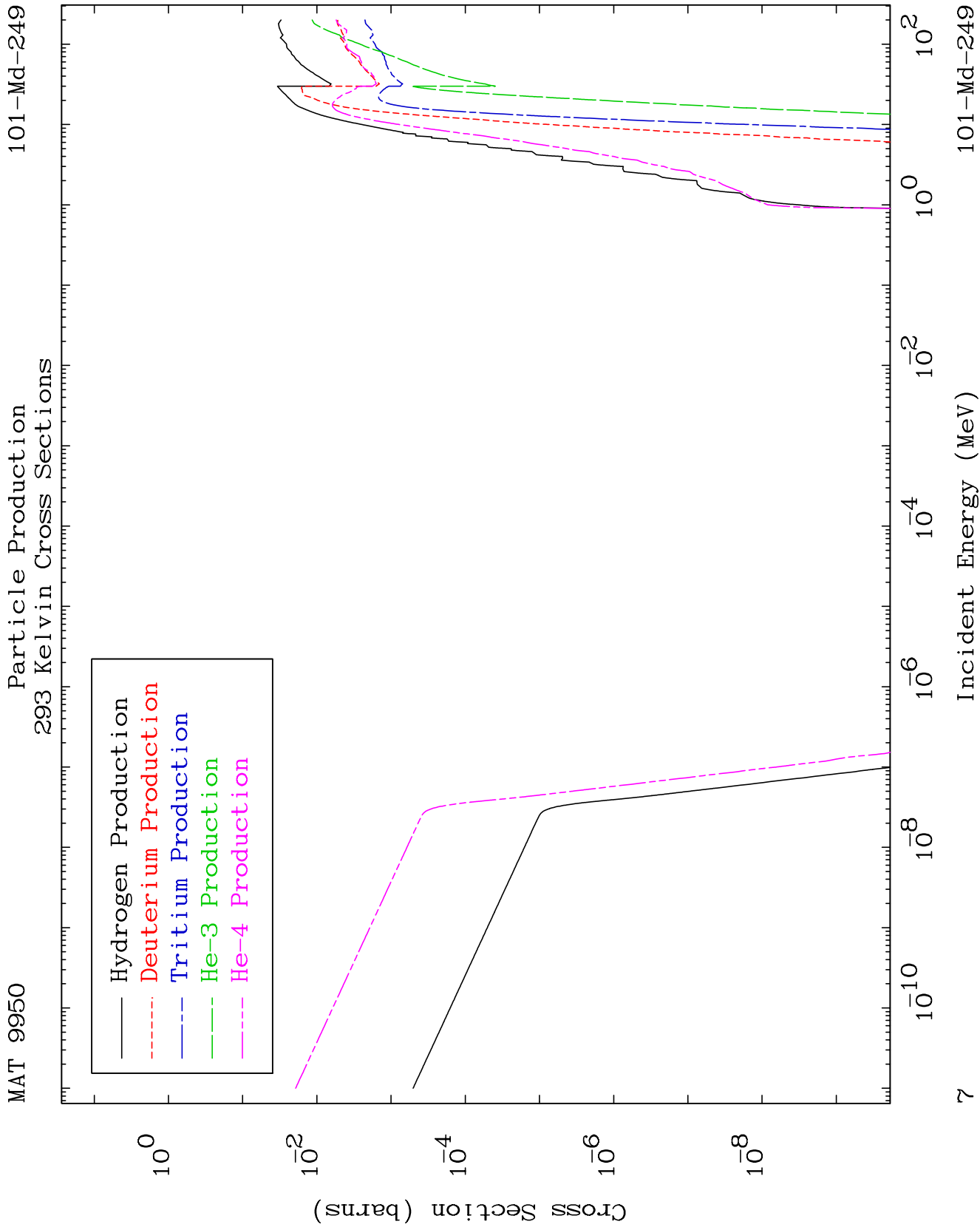
101-Md-249

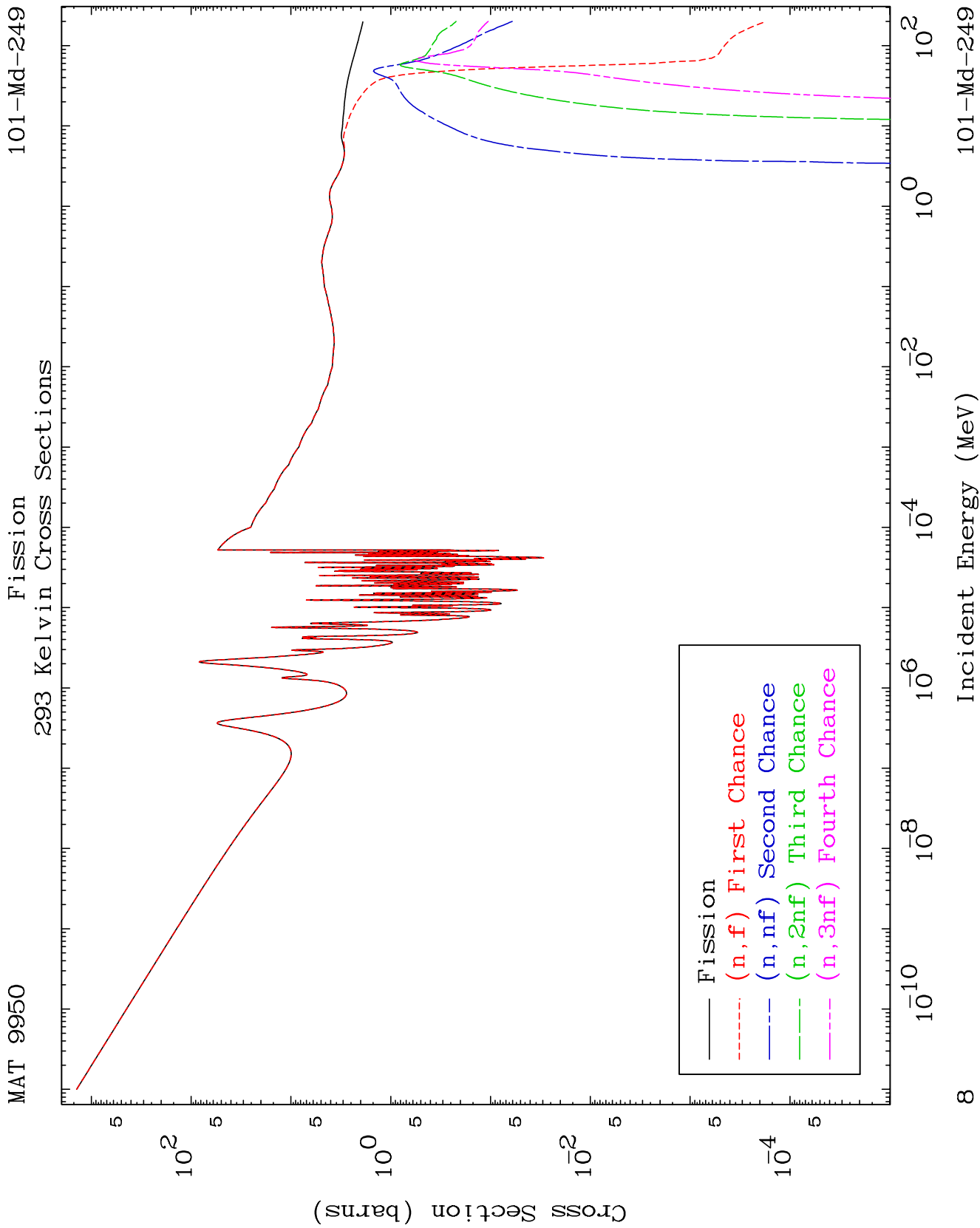


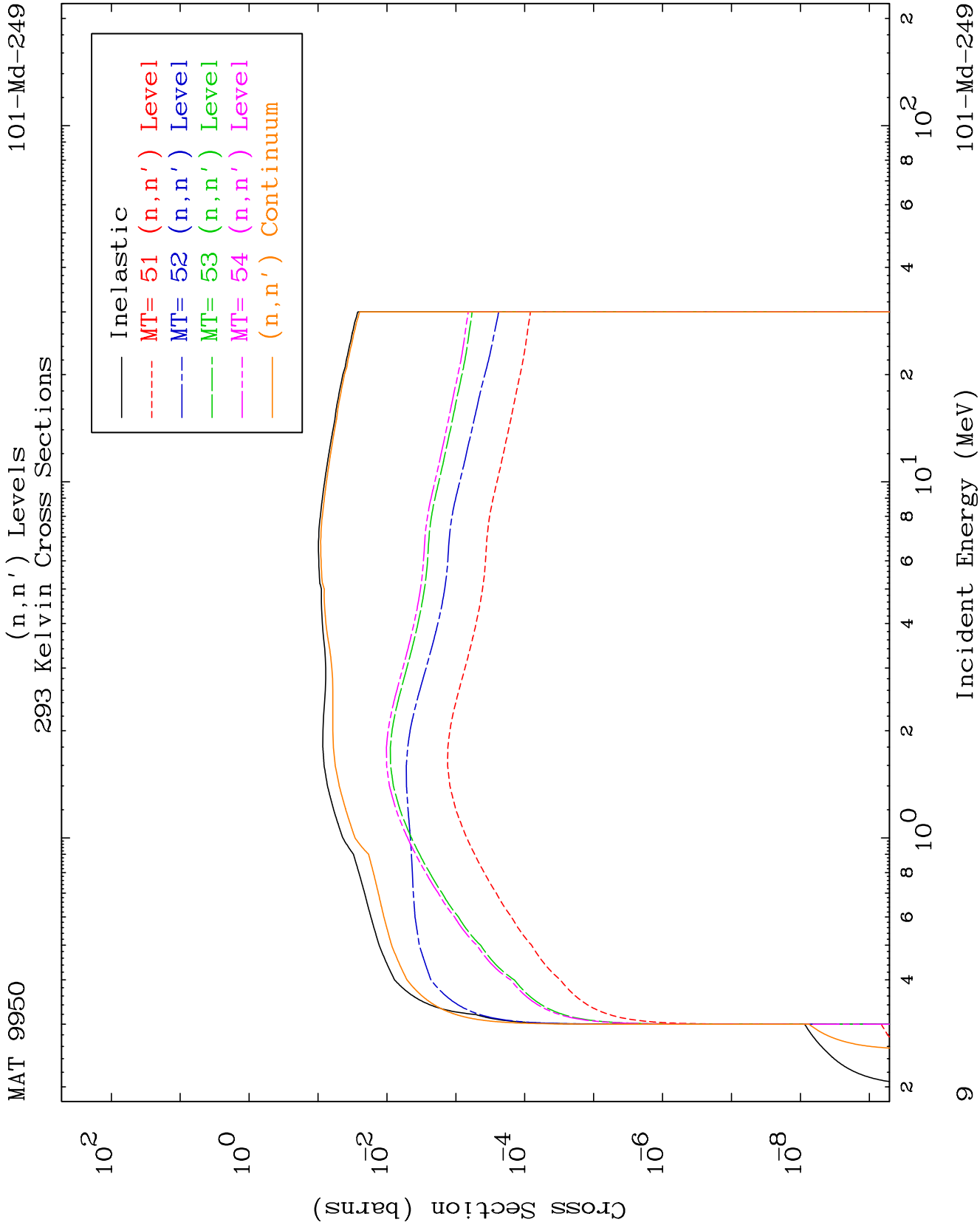


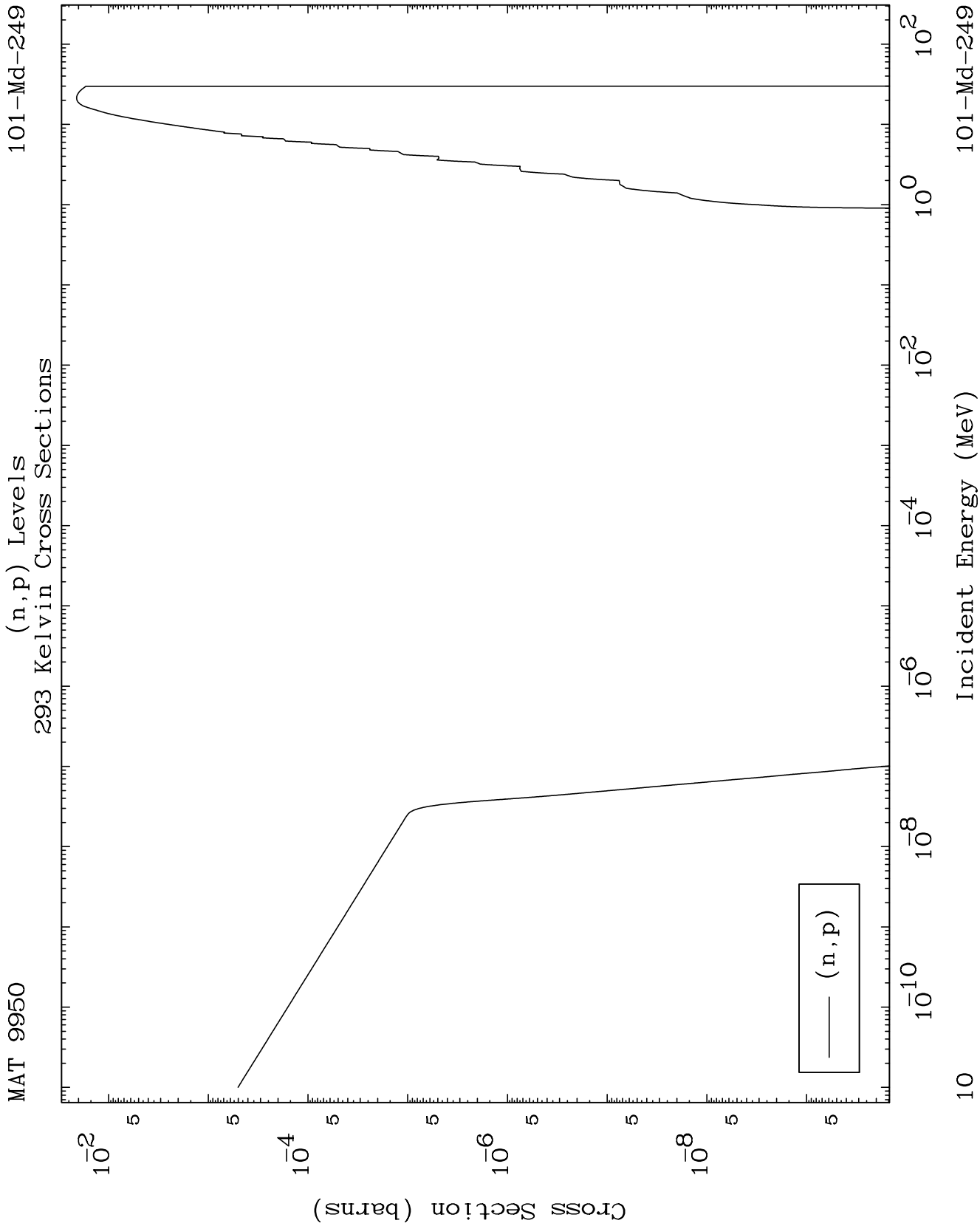








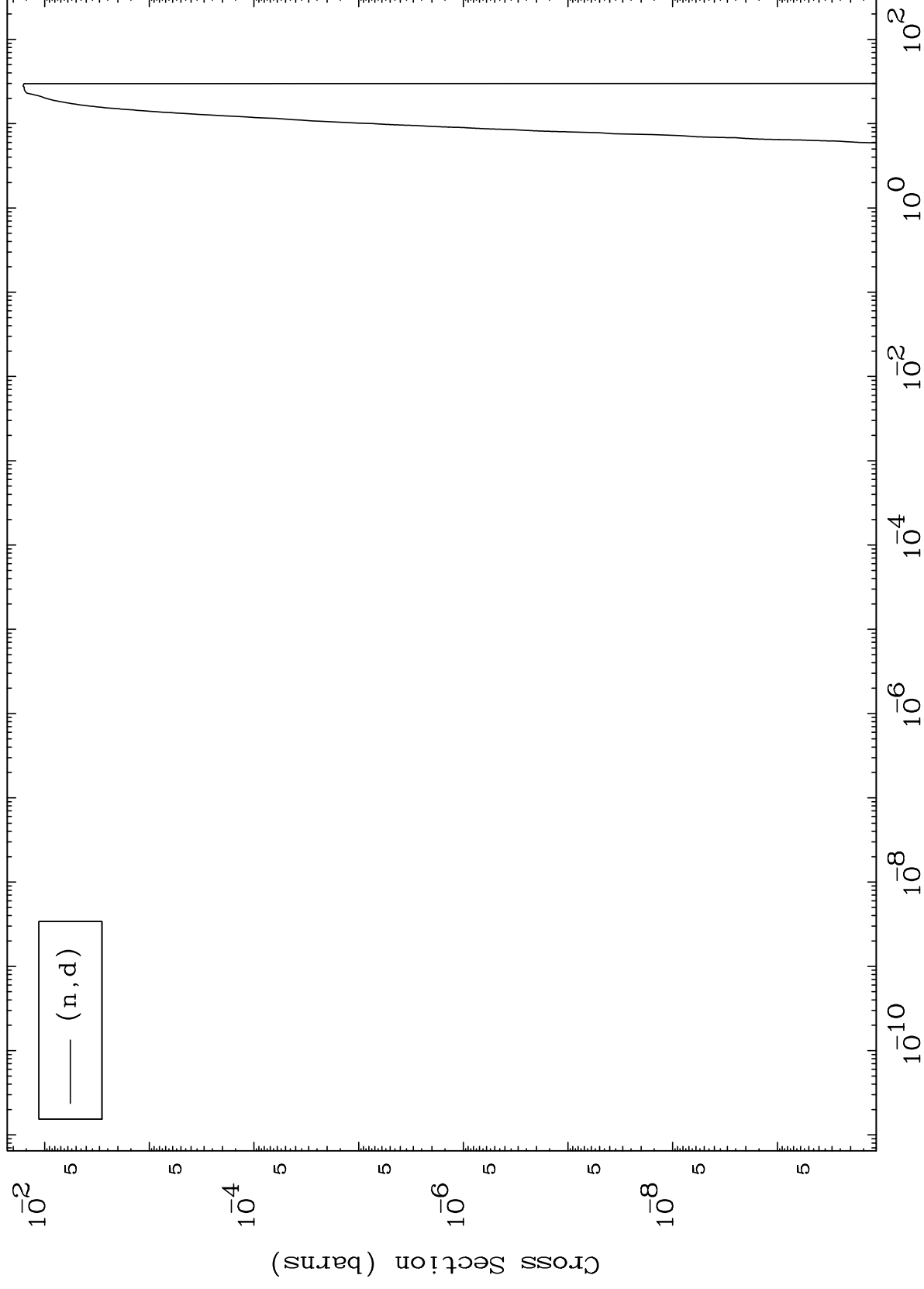




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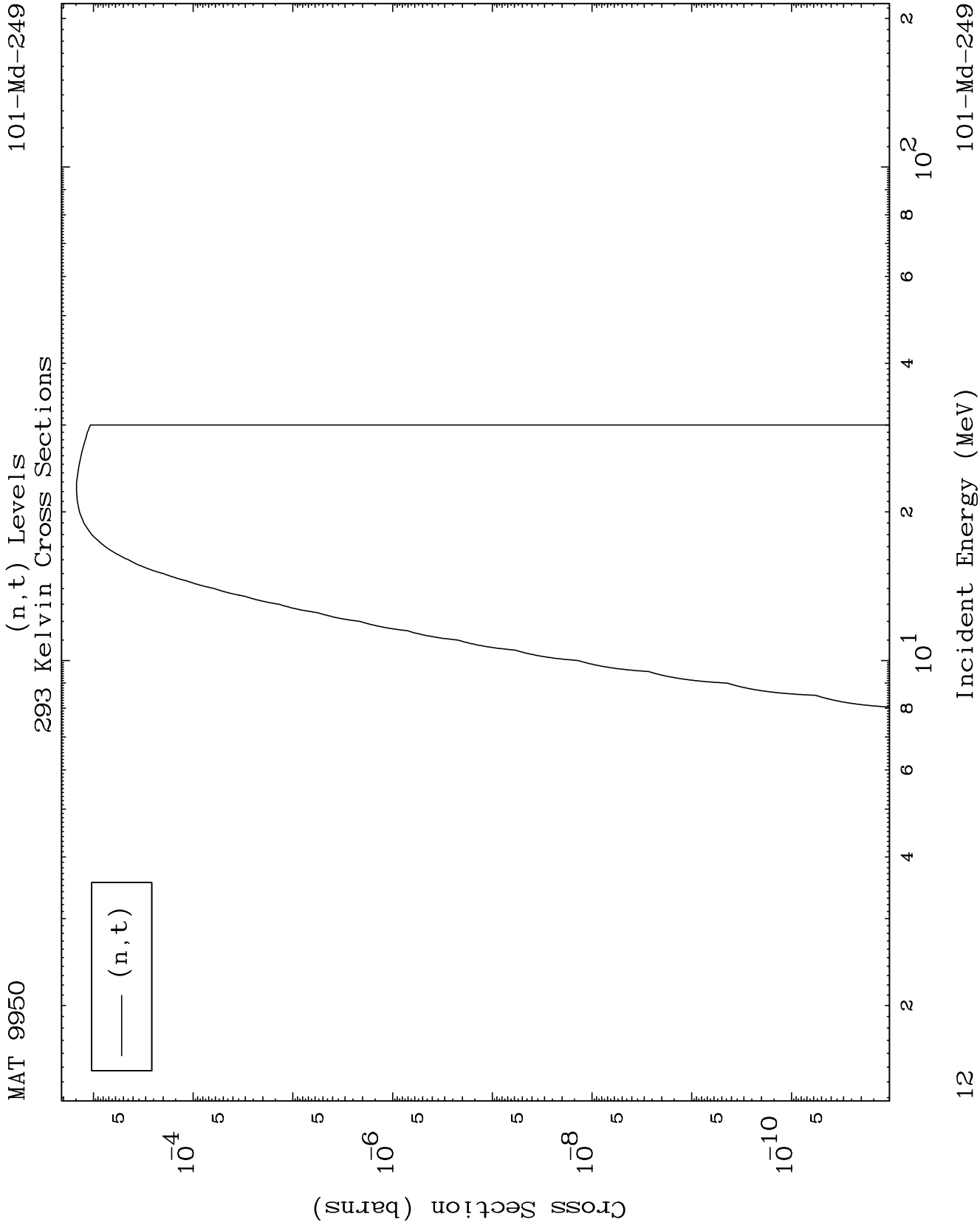
(n,d) Levels
293 Kelvin Cross Sections



11

Incident Energy (MeV)

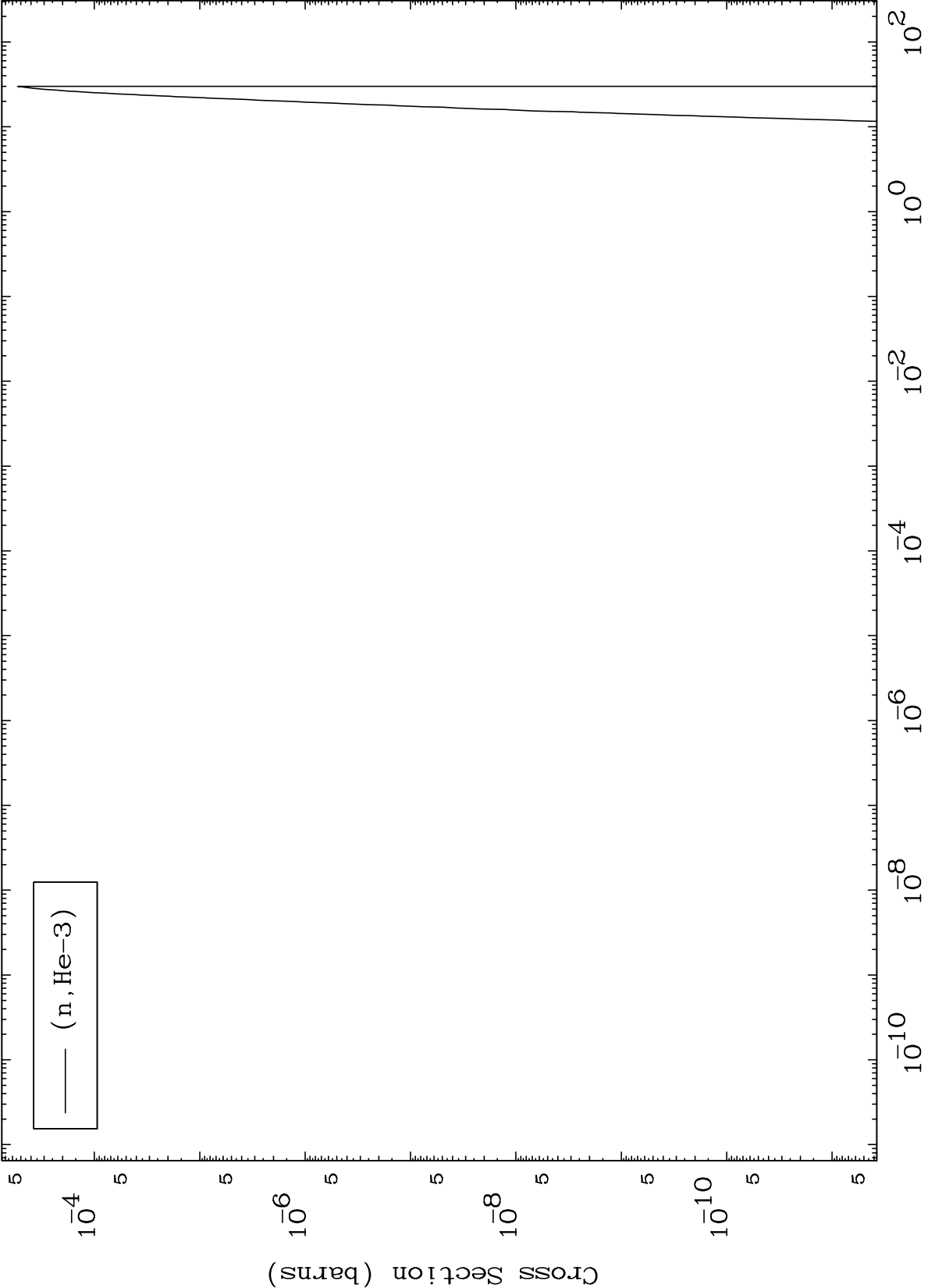
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(n,He3) Levels
293 Kelvin Cross Sections

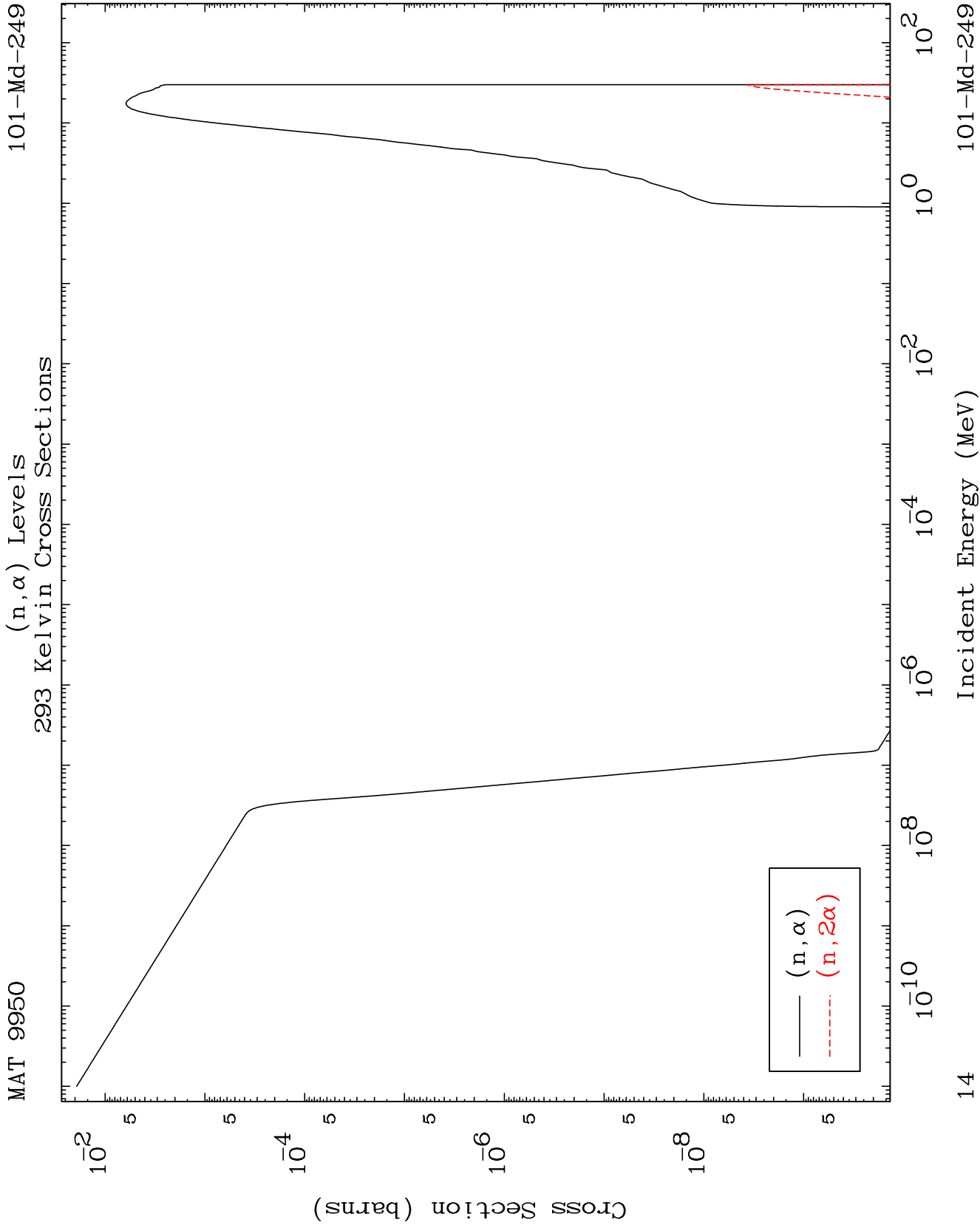
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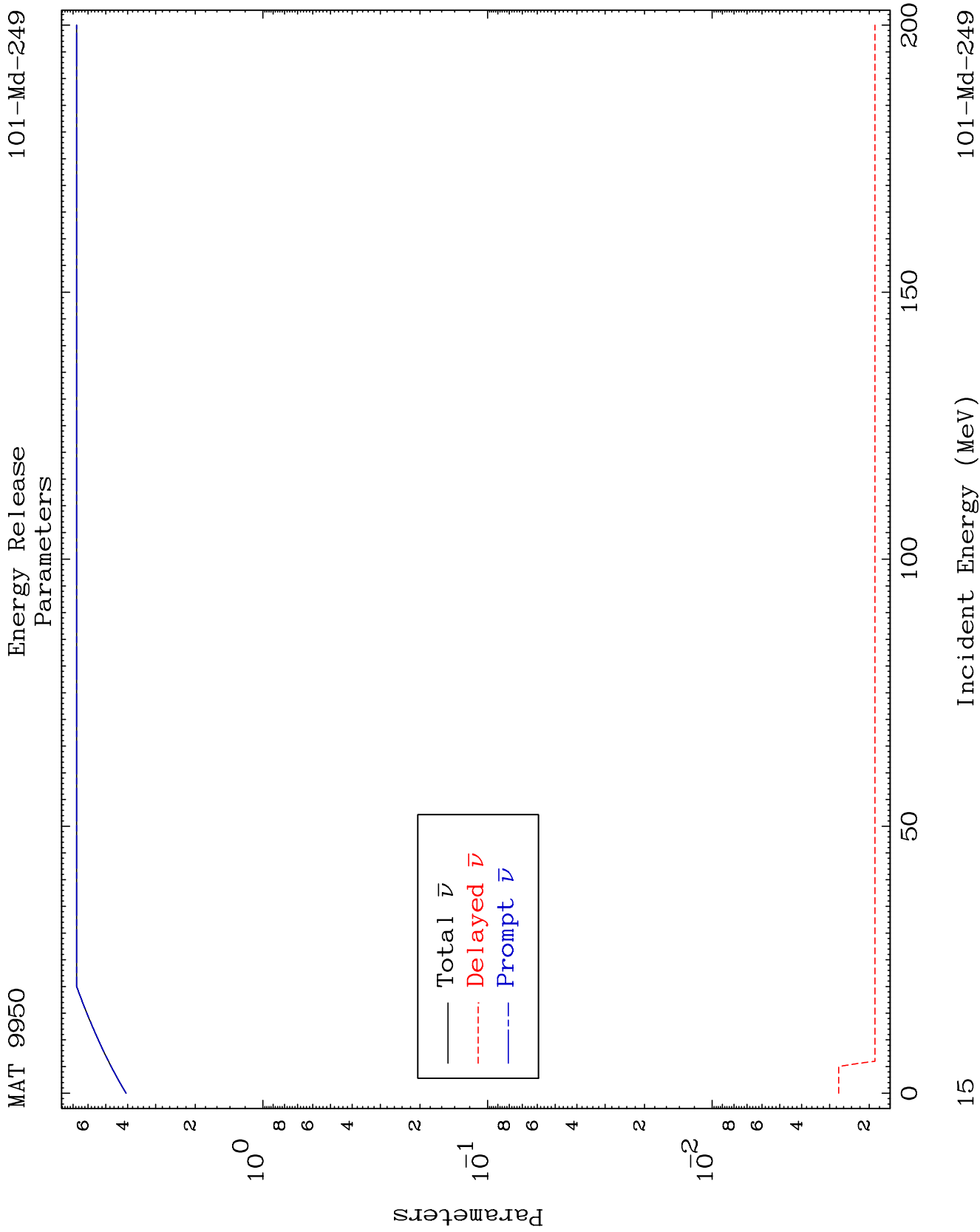


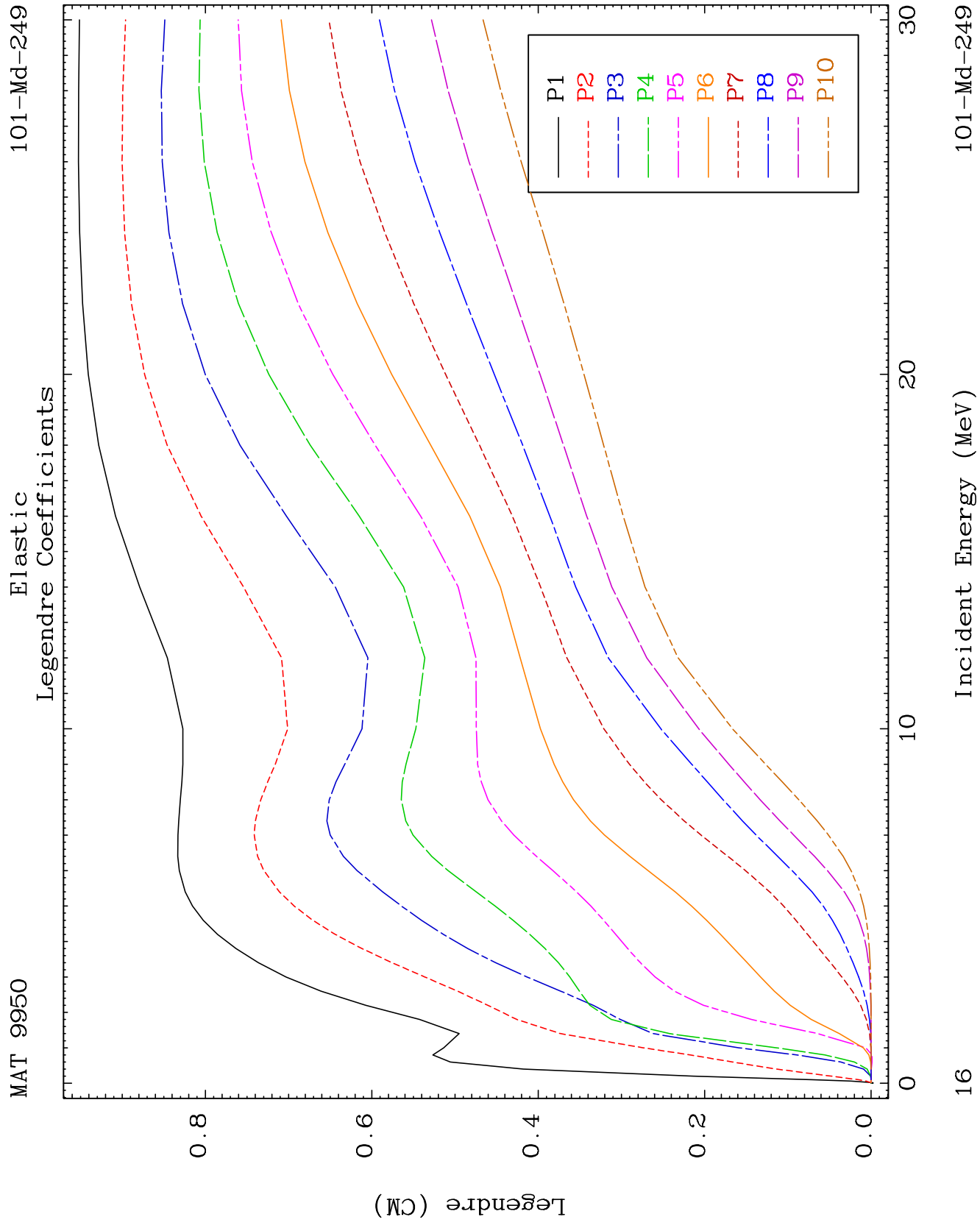
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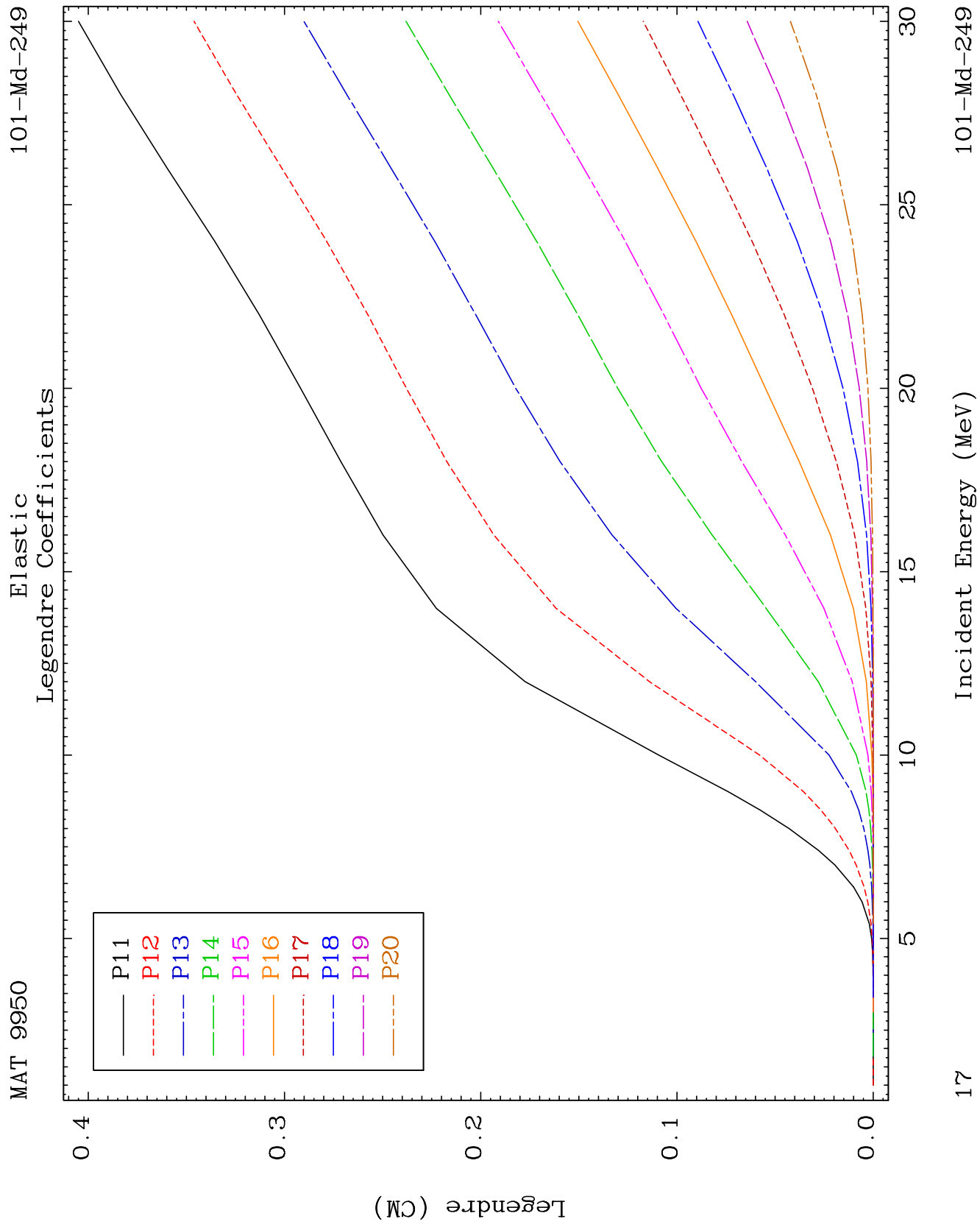
Incident Energy (MeV)

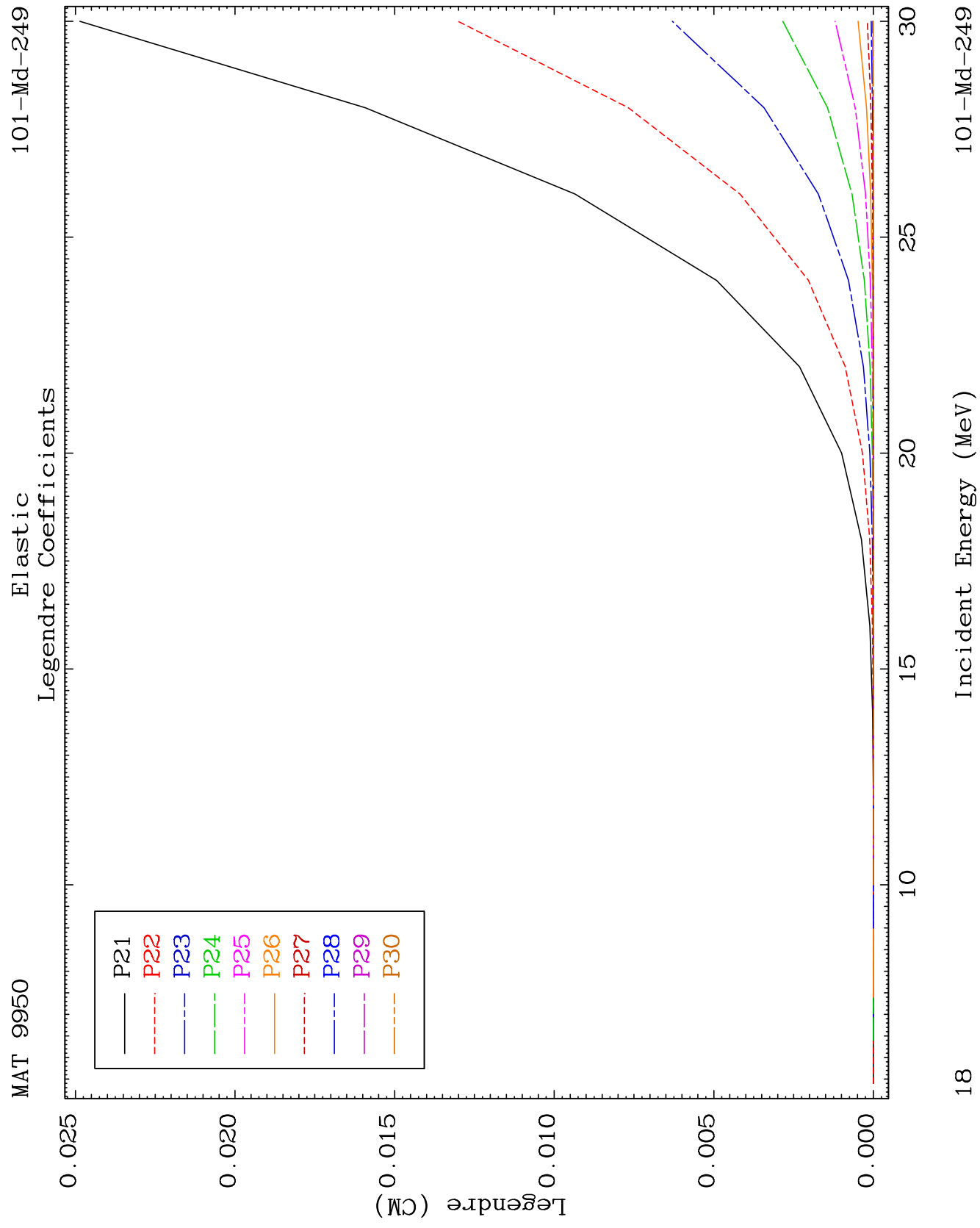
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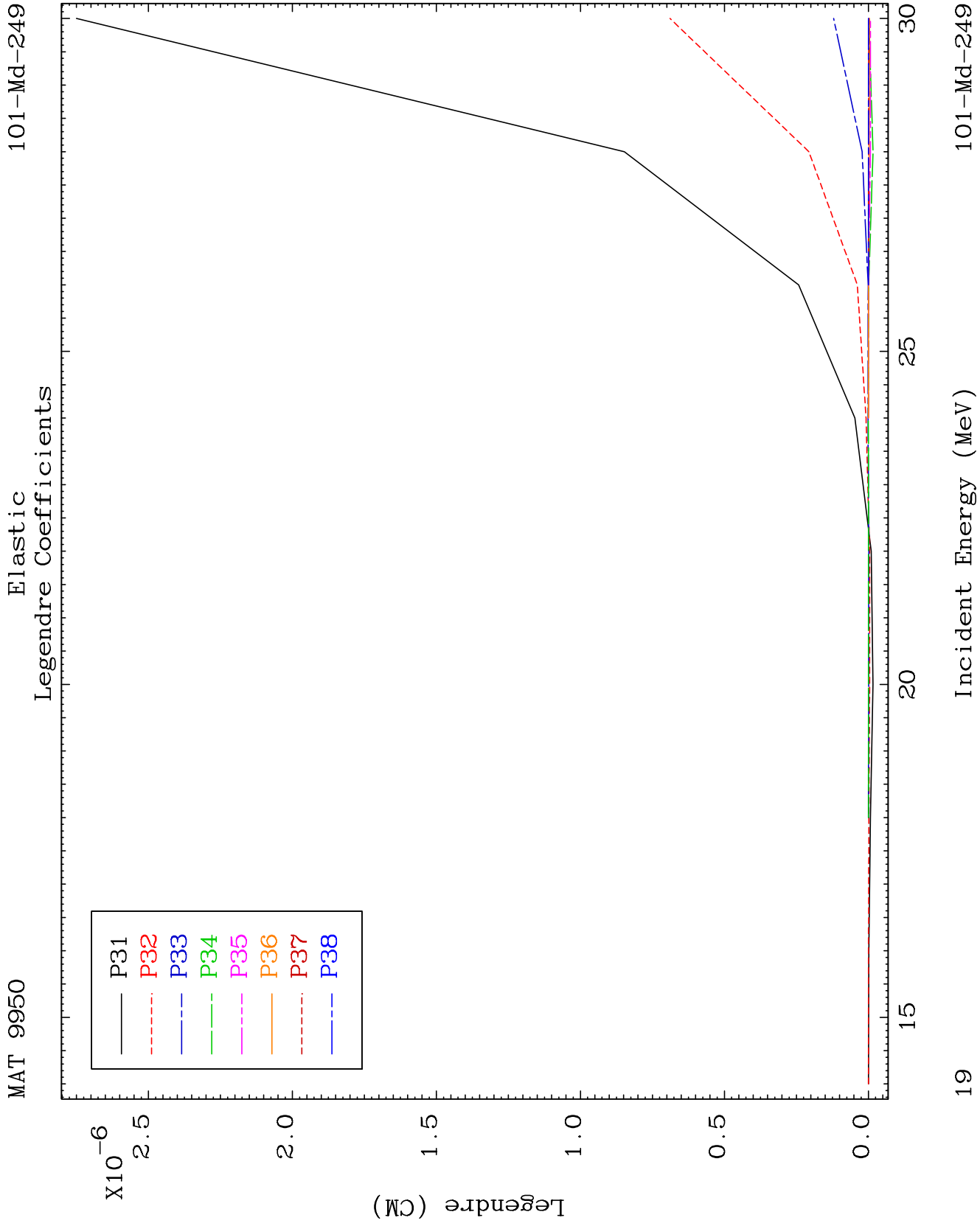


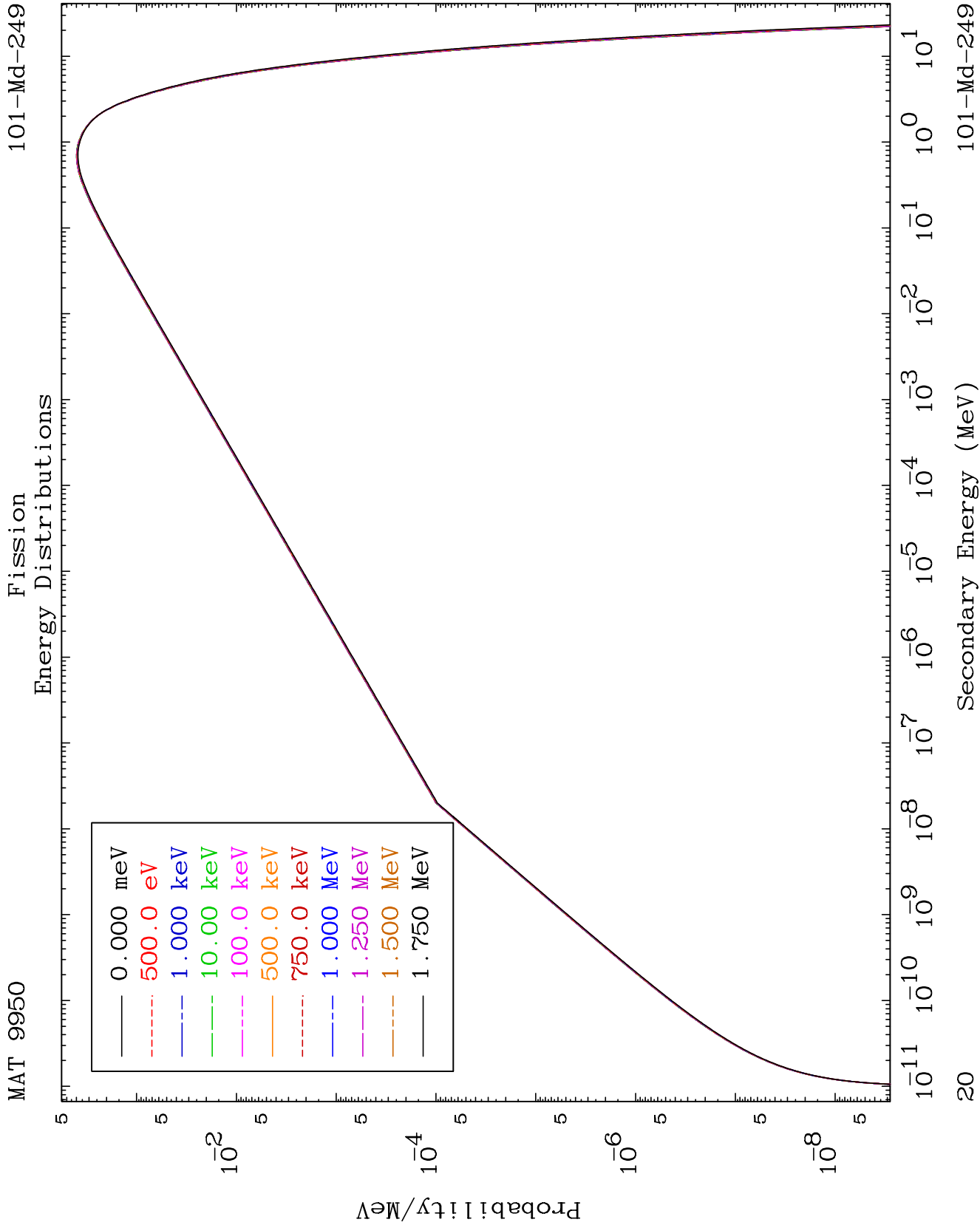


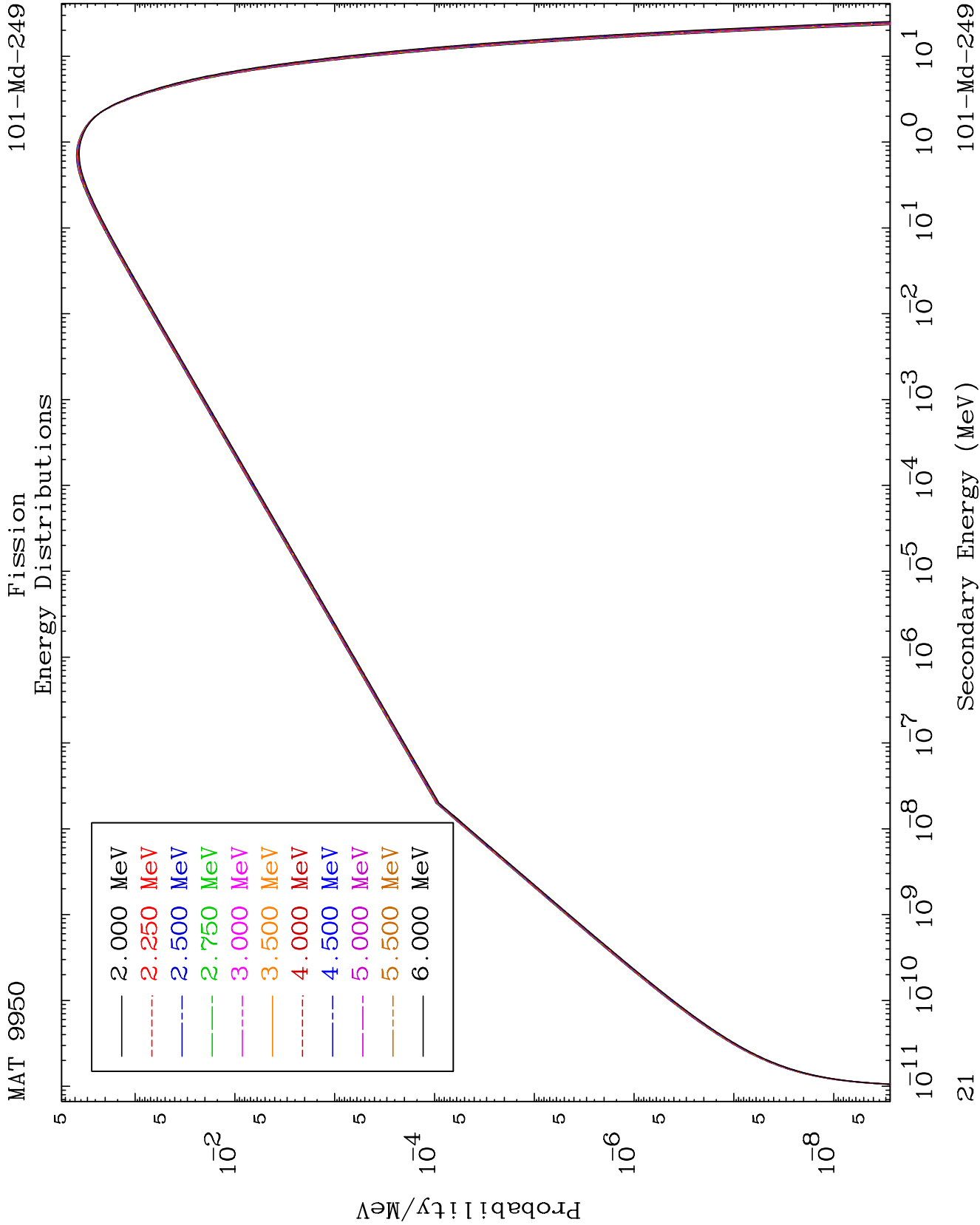


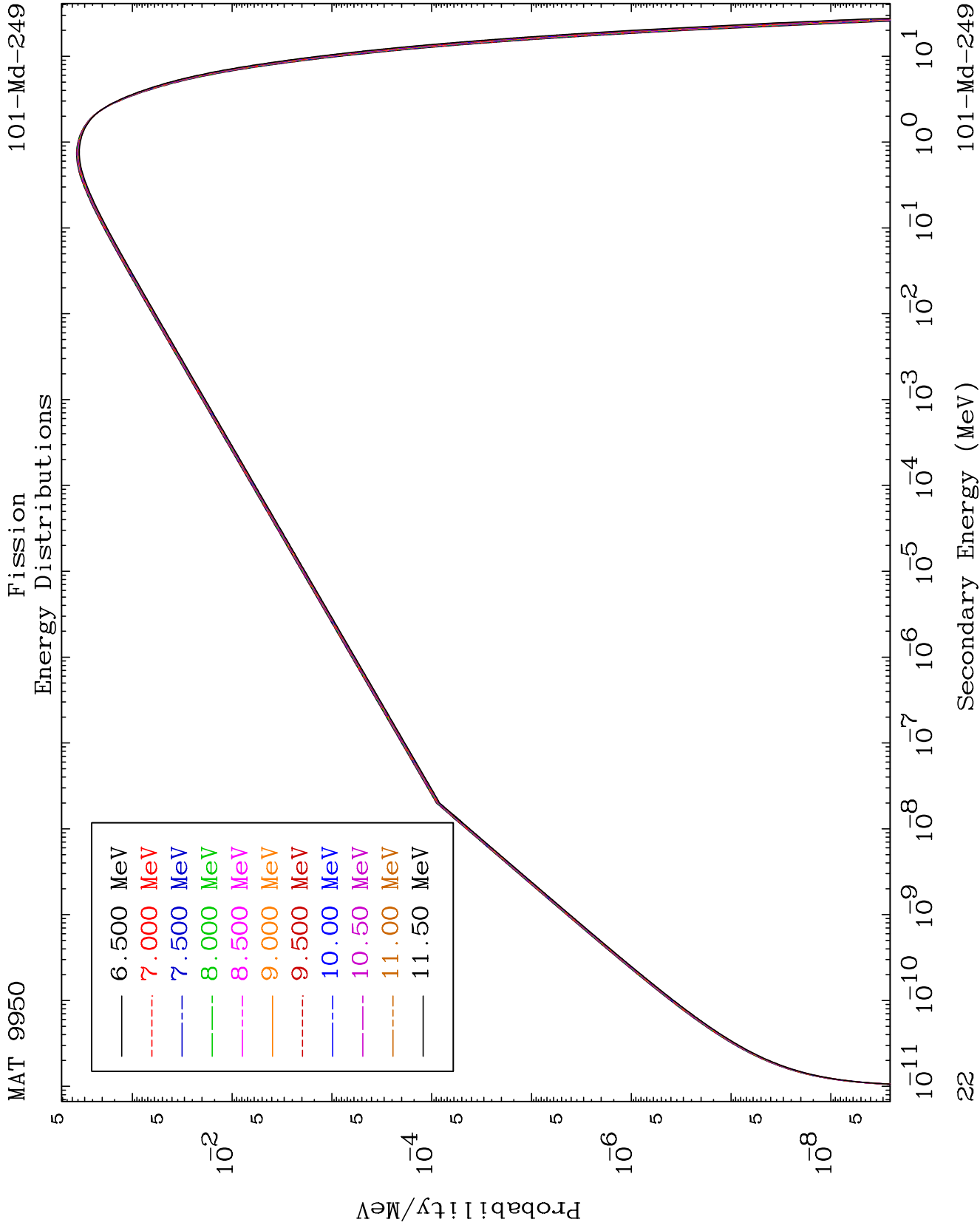








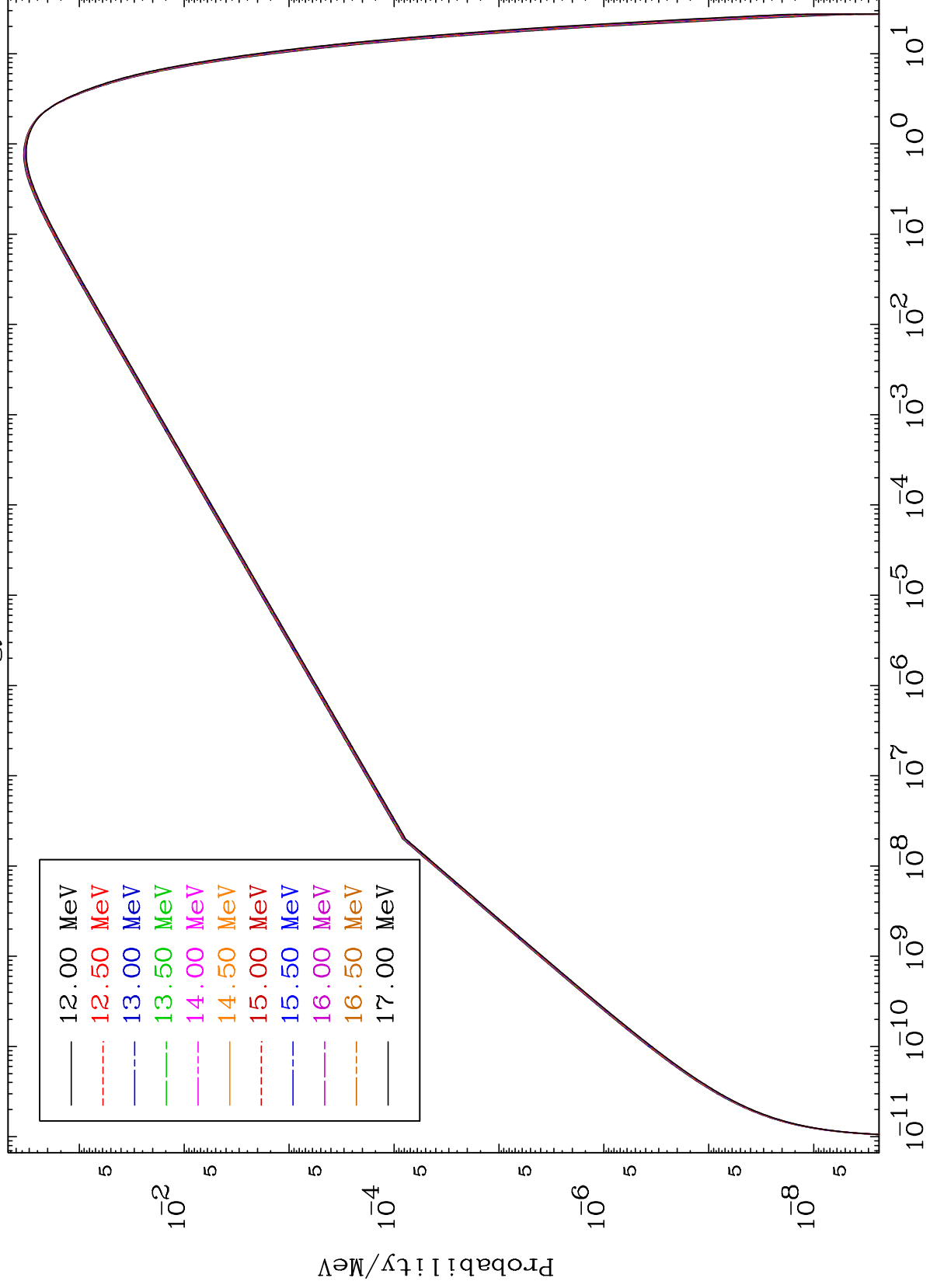




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Fission
Energy Distributions

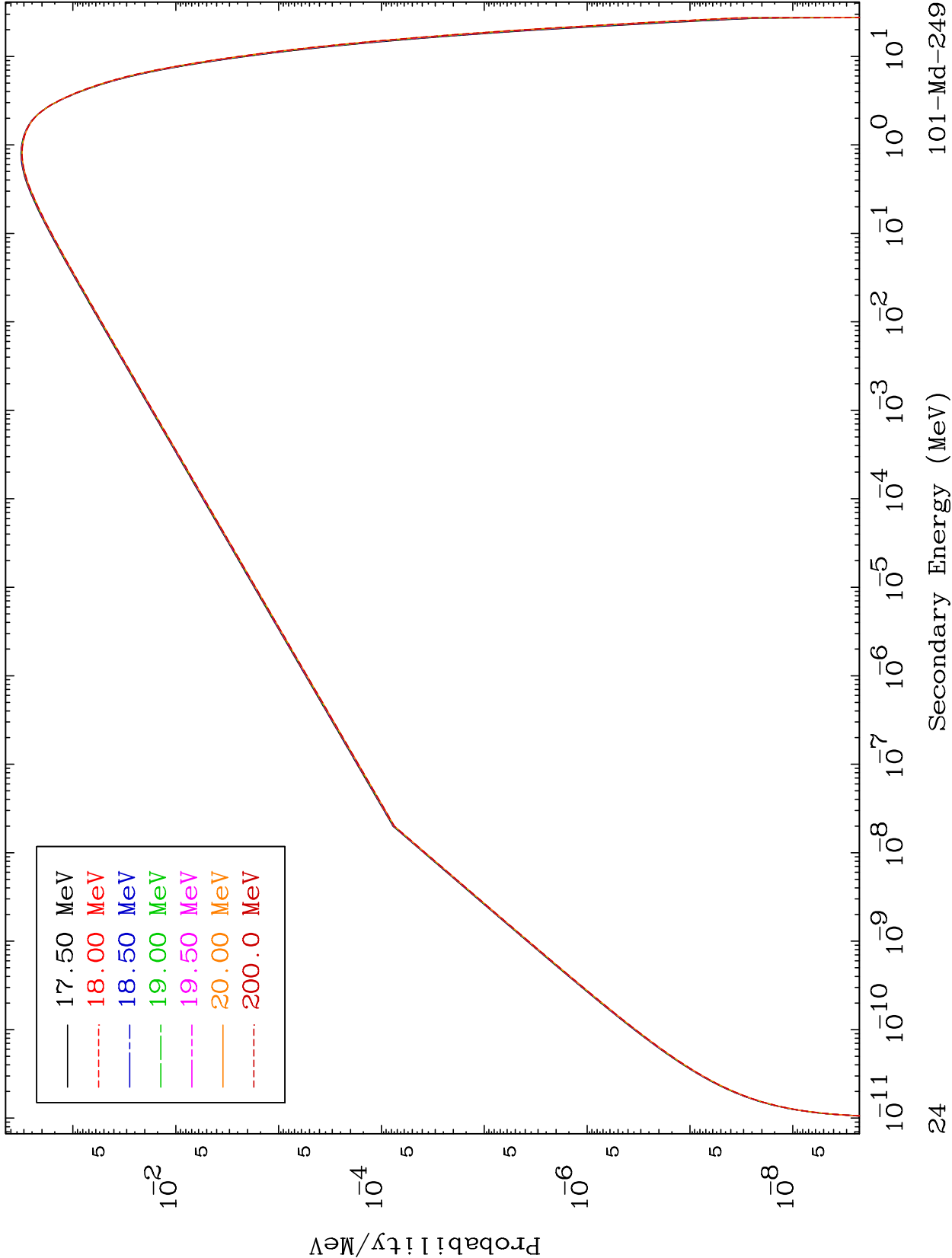
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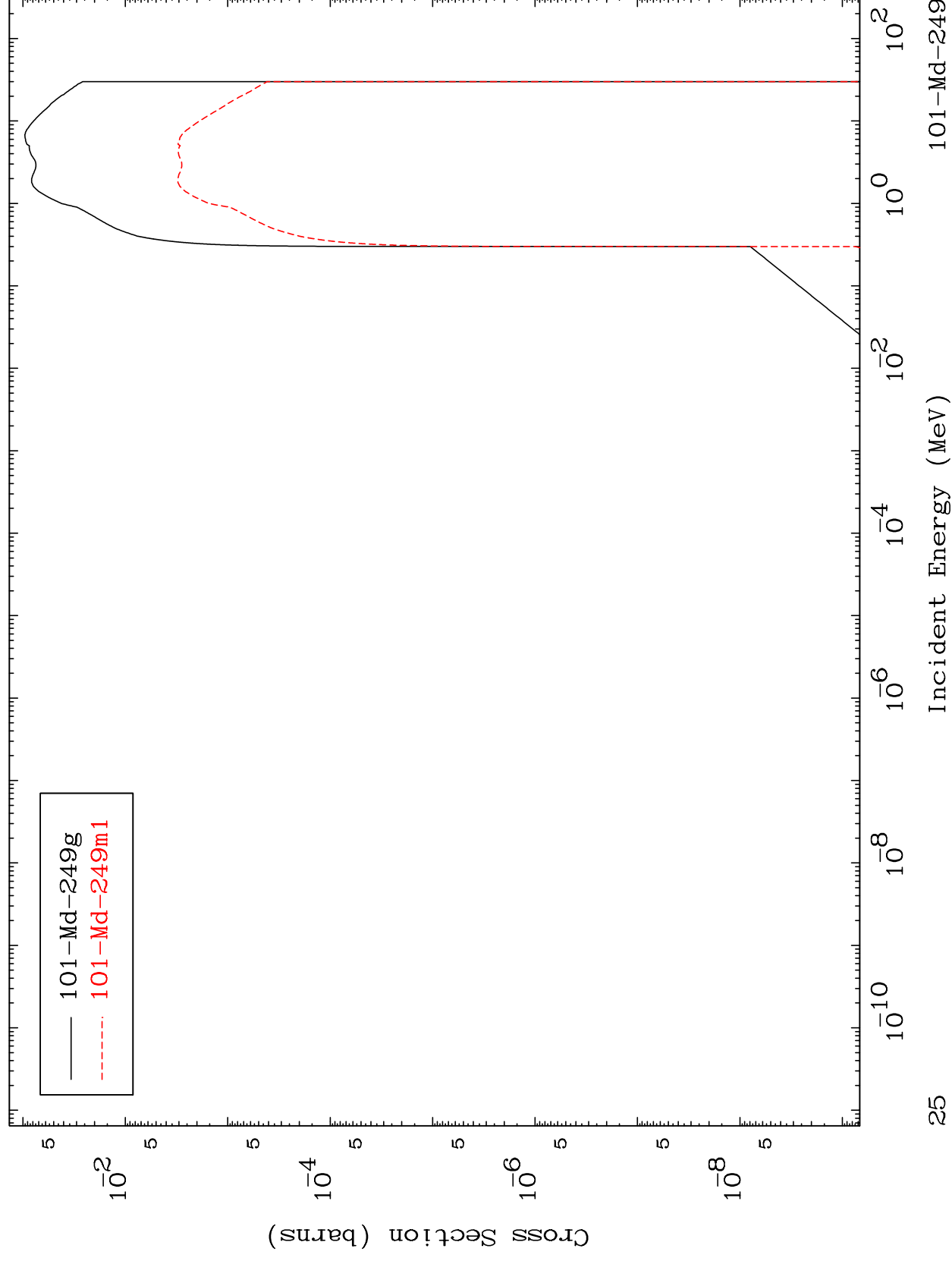
MAT 9950

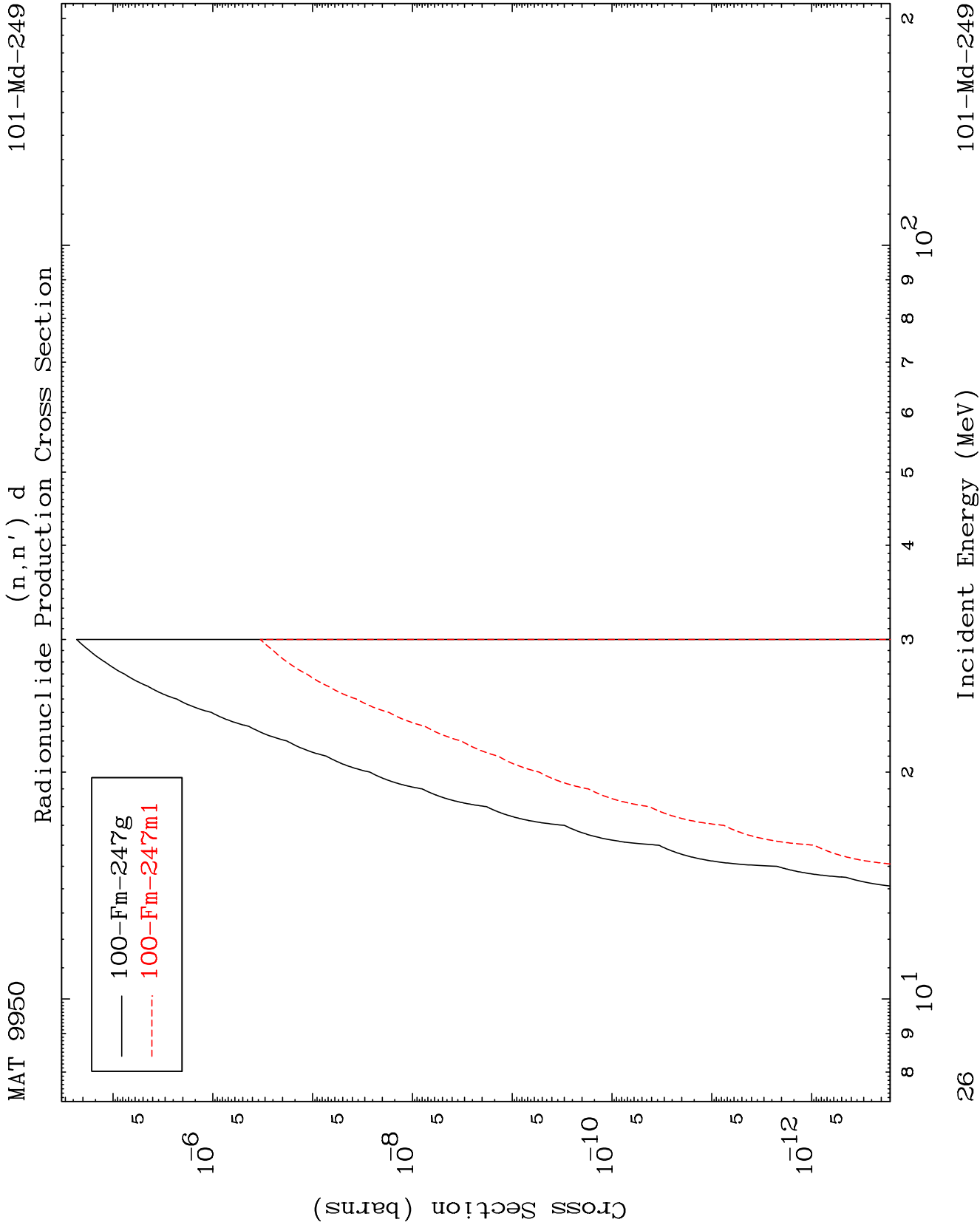
Fission
Energy Distributions

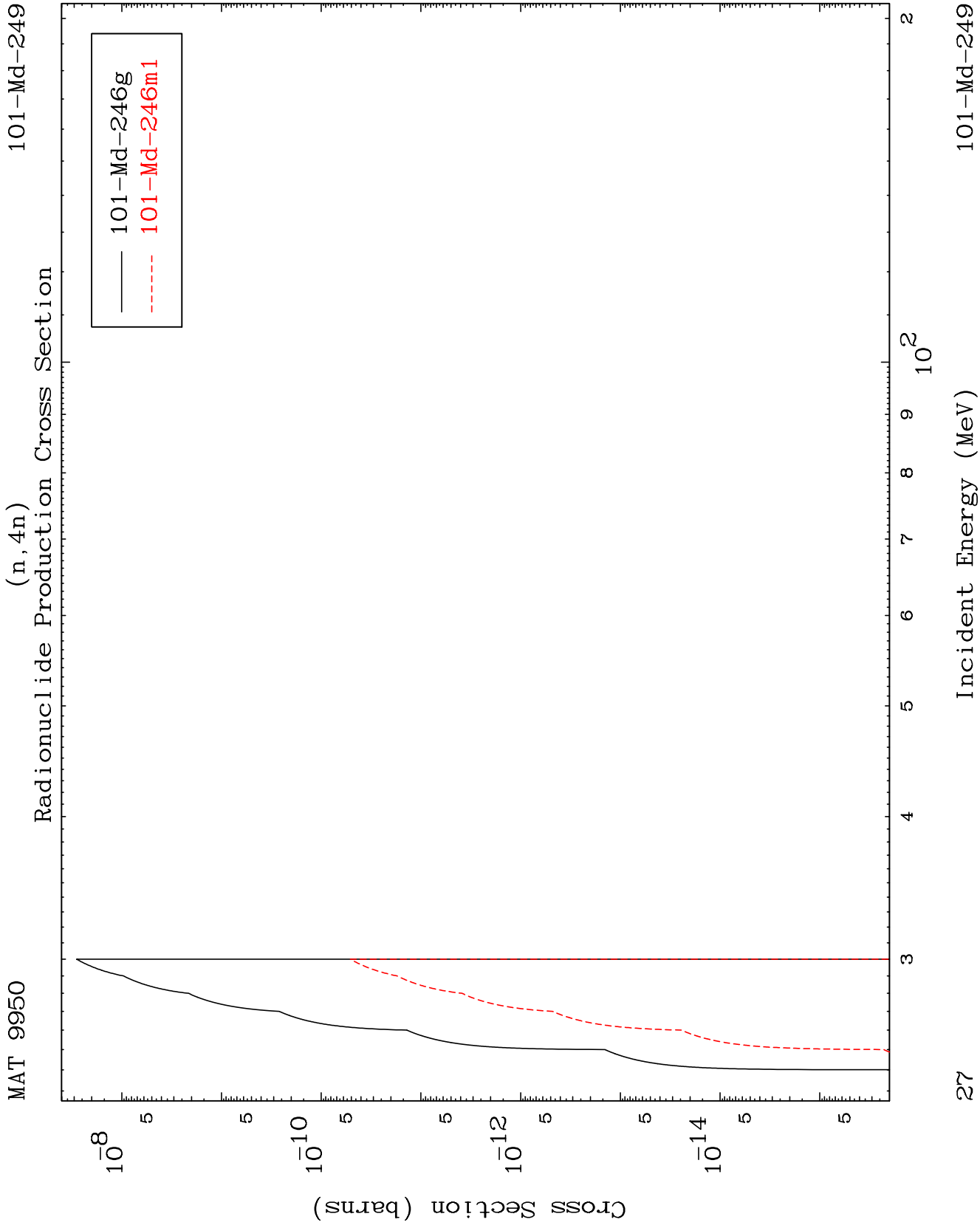
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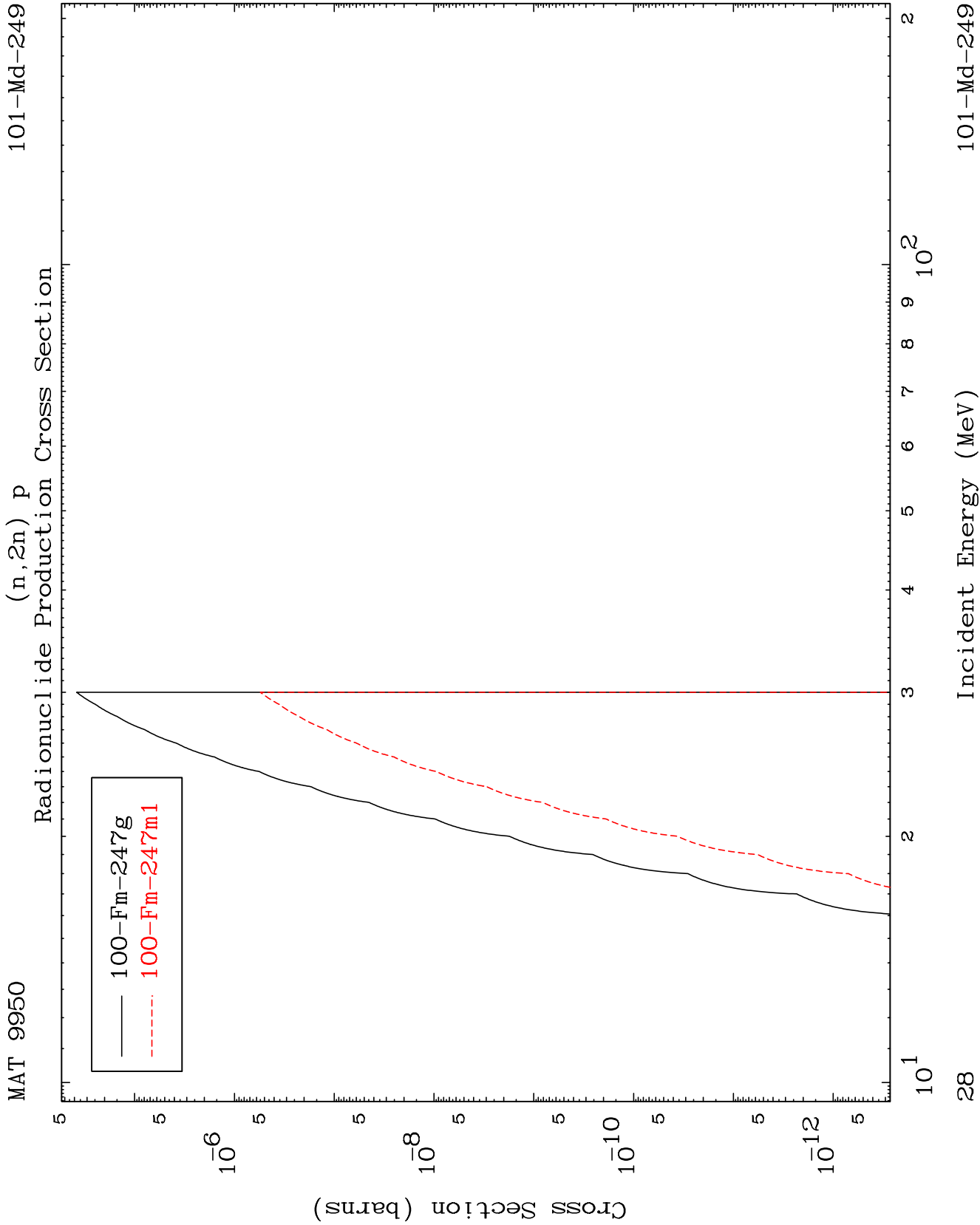


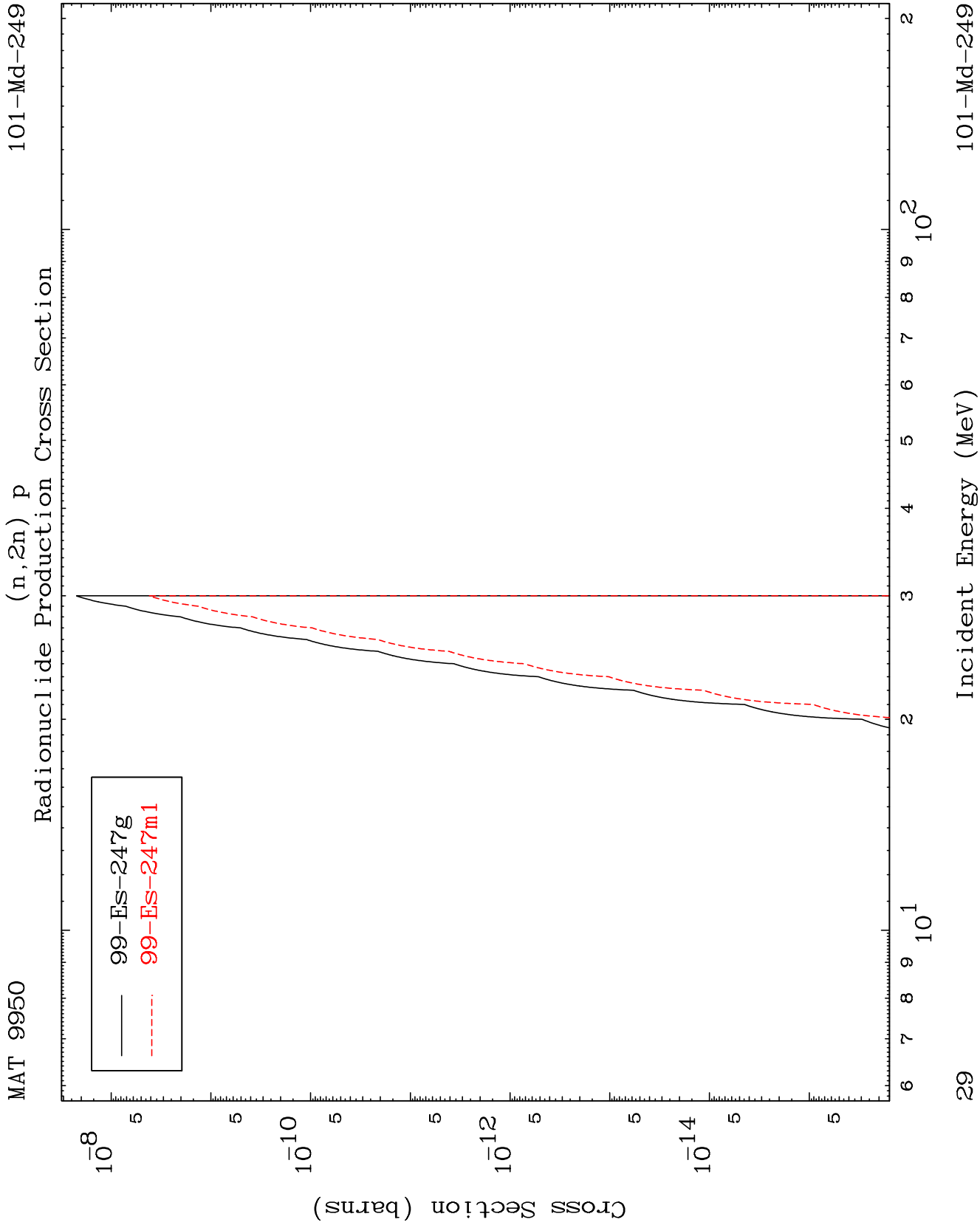
Inelastic Radionuclide Production Cross Section





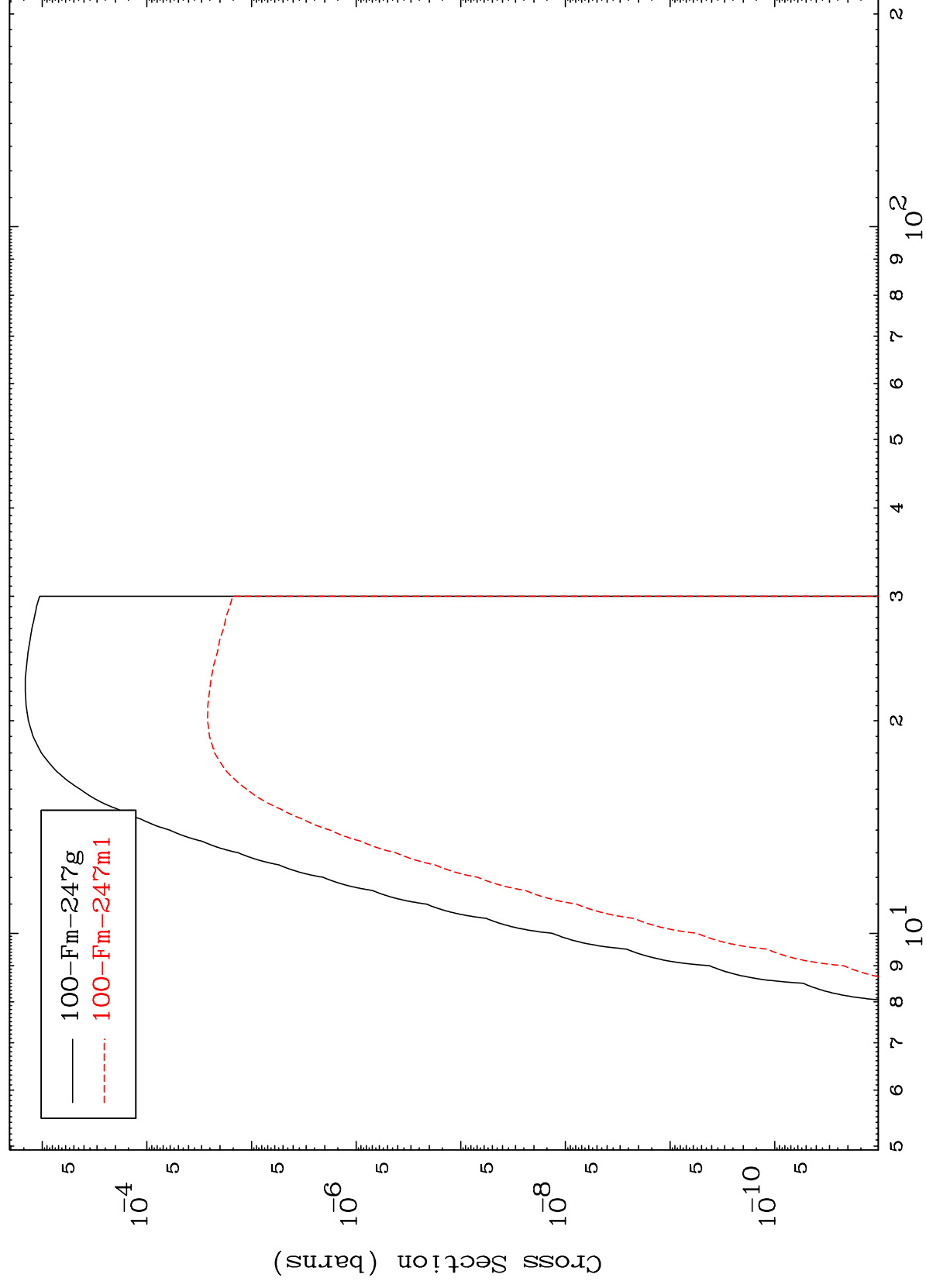






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Radionuclide Production Cross Section
(n, t)

101-Md-249

Incident Energy (MeV)

30

