

Program EVALPLOT  
(Version 2021-1)

by

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(Present Contact Information)

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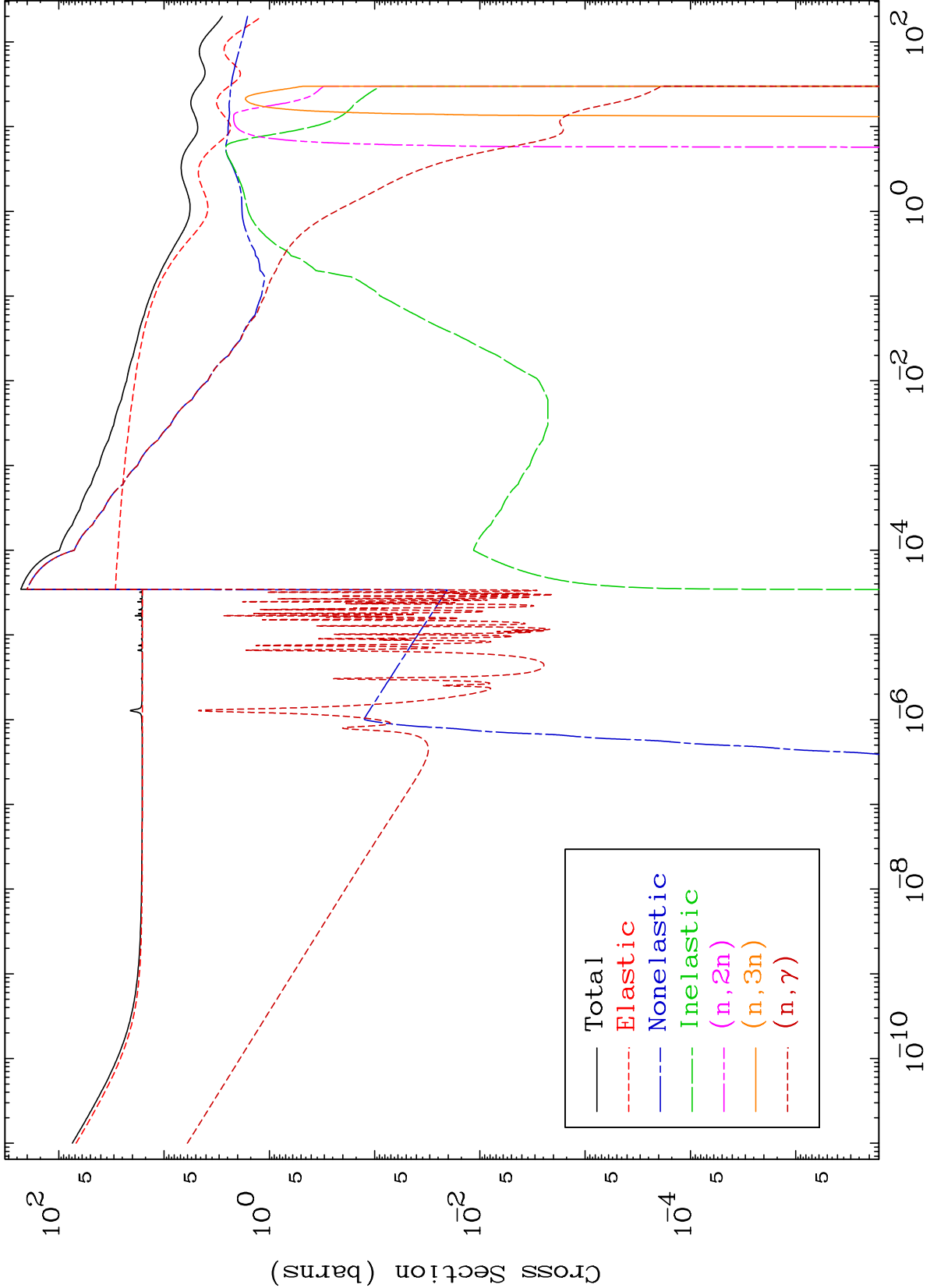
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7535

Neutron Major  
293 Kelvin Cross Sections

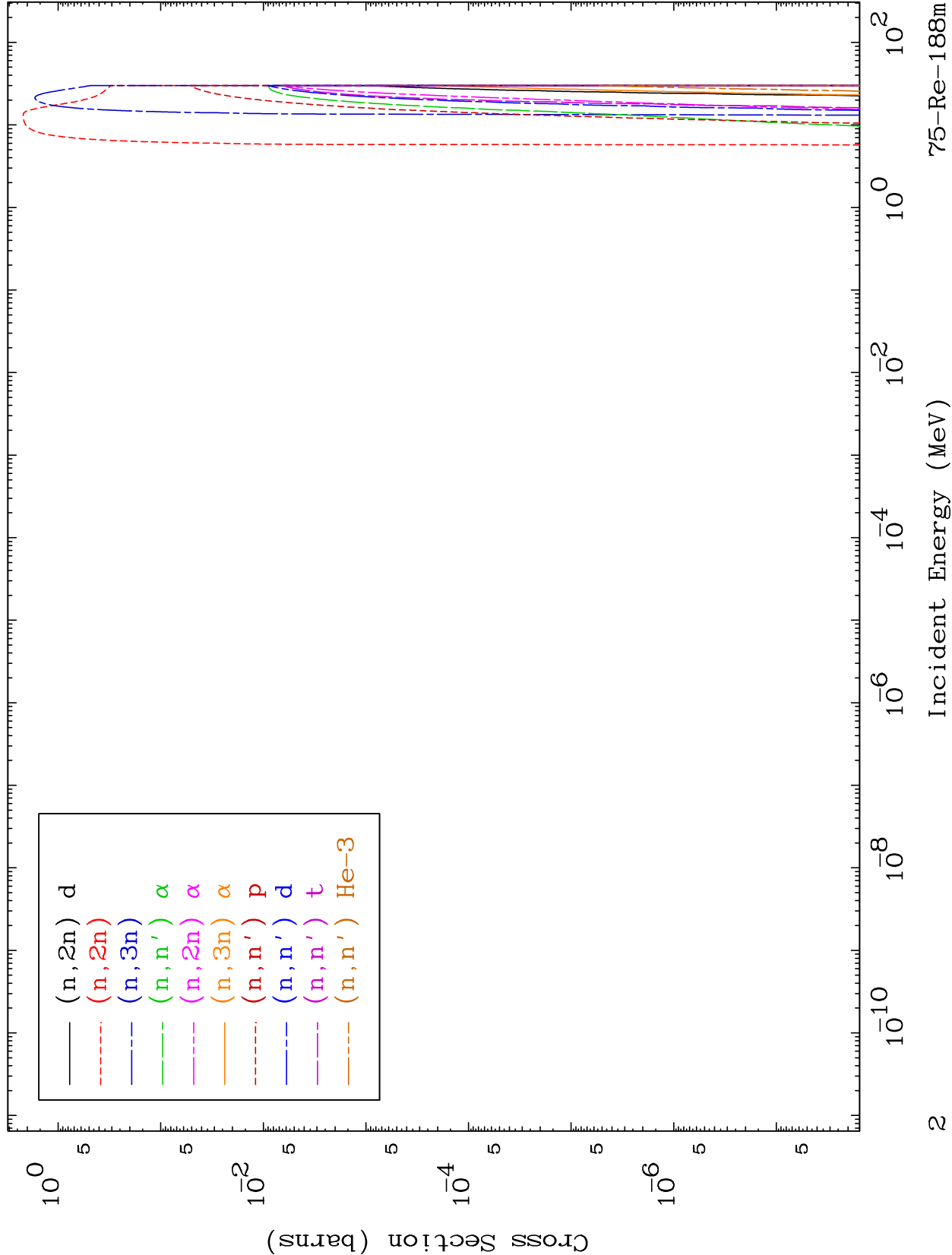
75-Re-188m

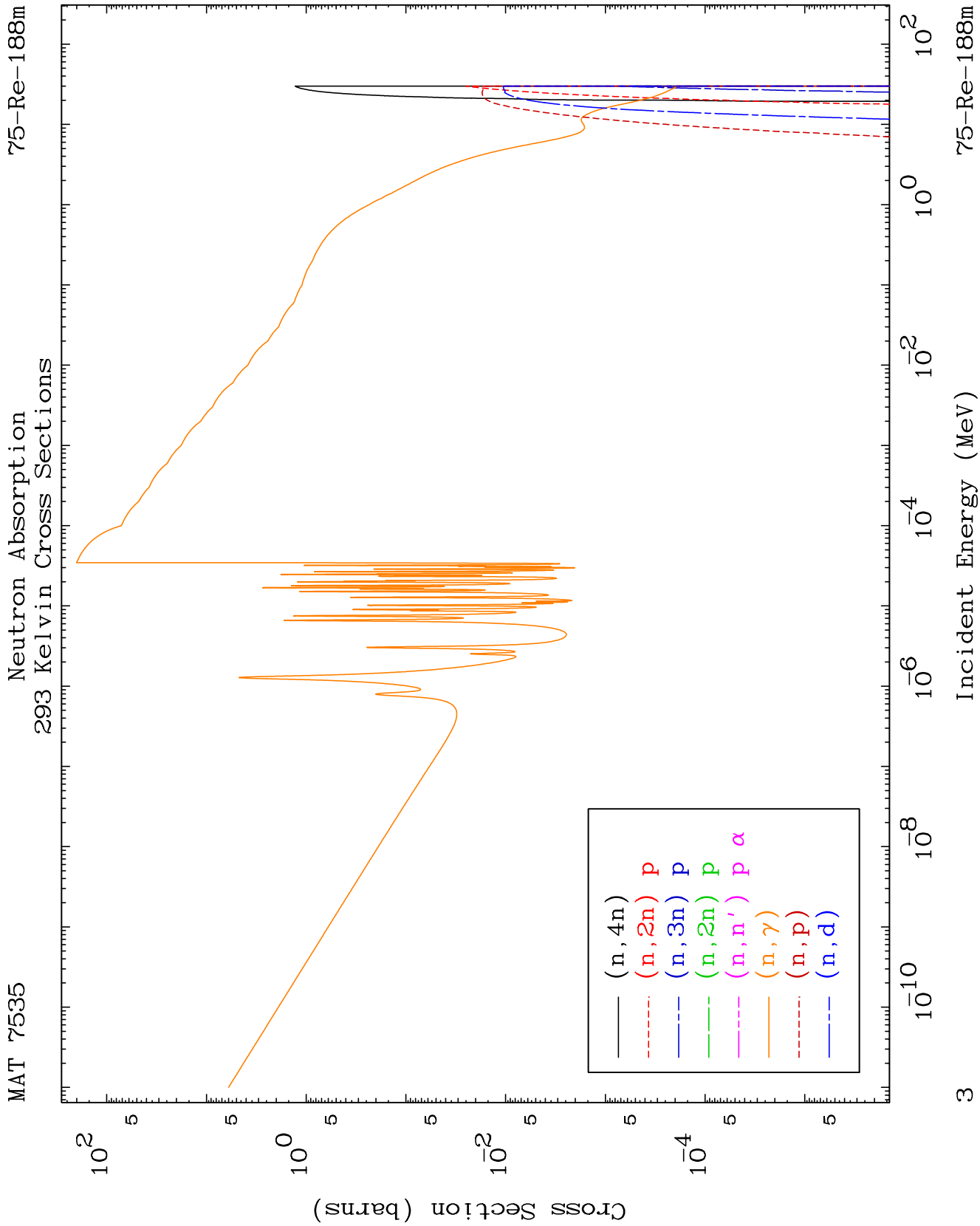


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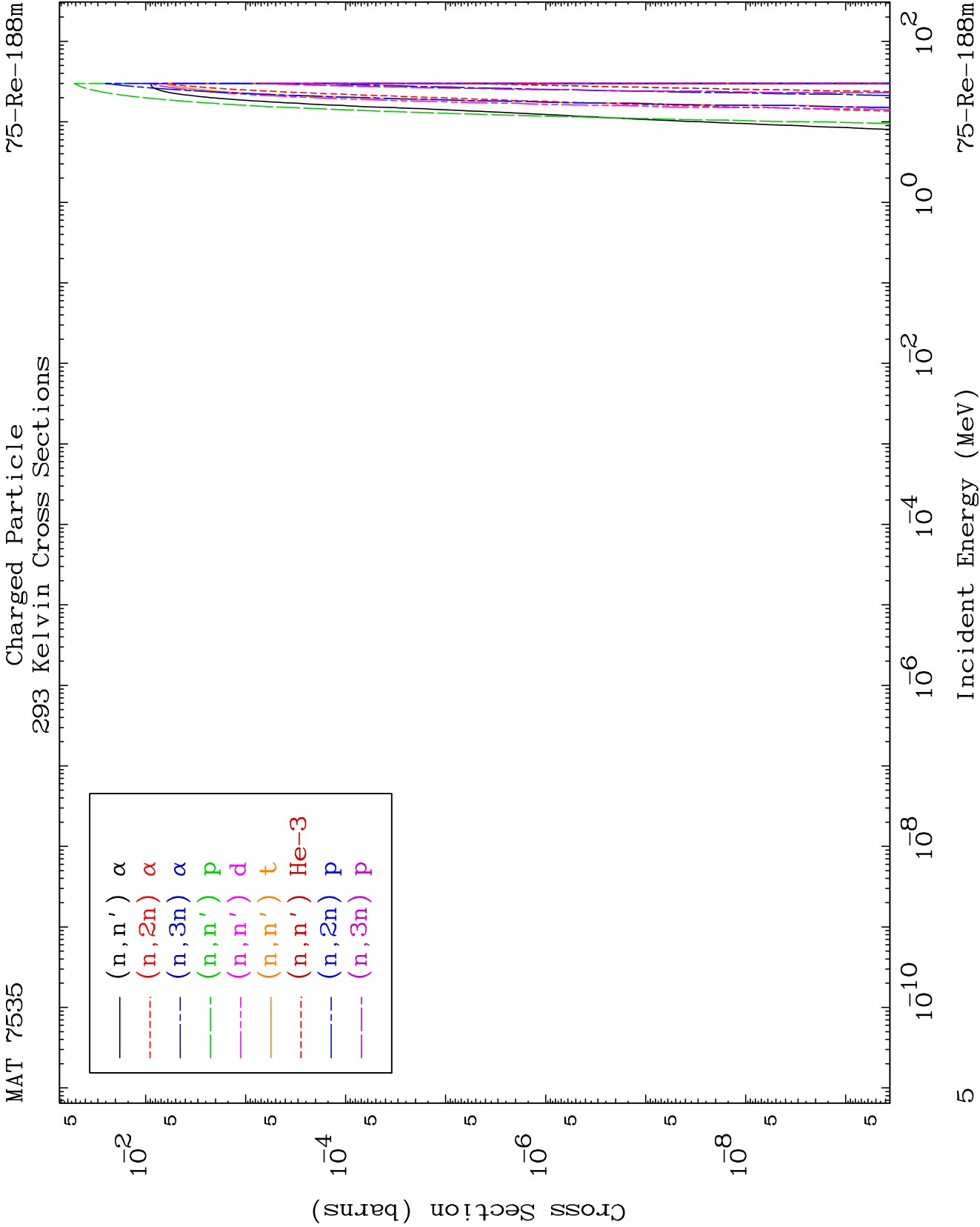
Neutron Absorption  
293 Kelvin Cross Sections

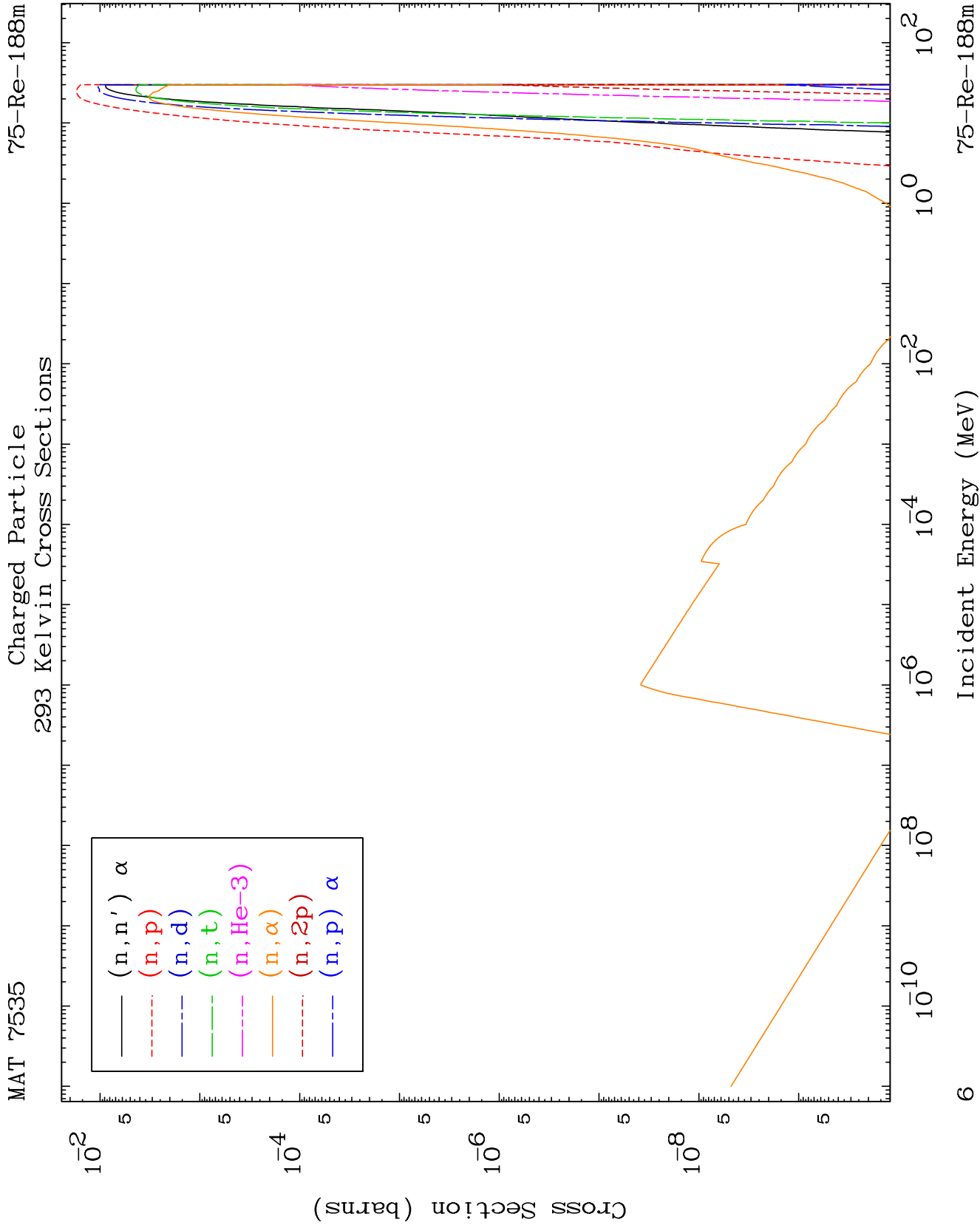
75-Re-188m

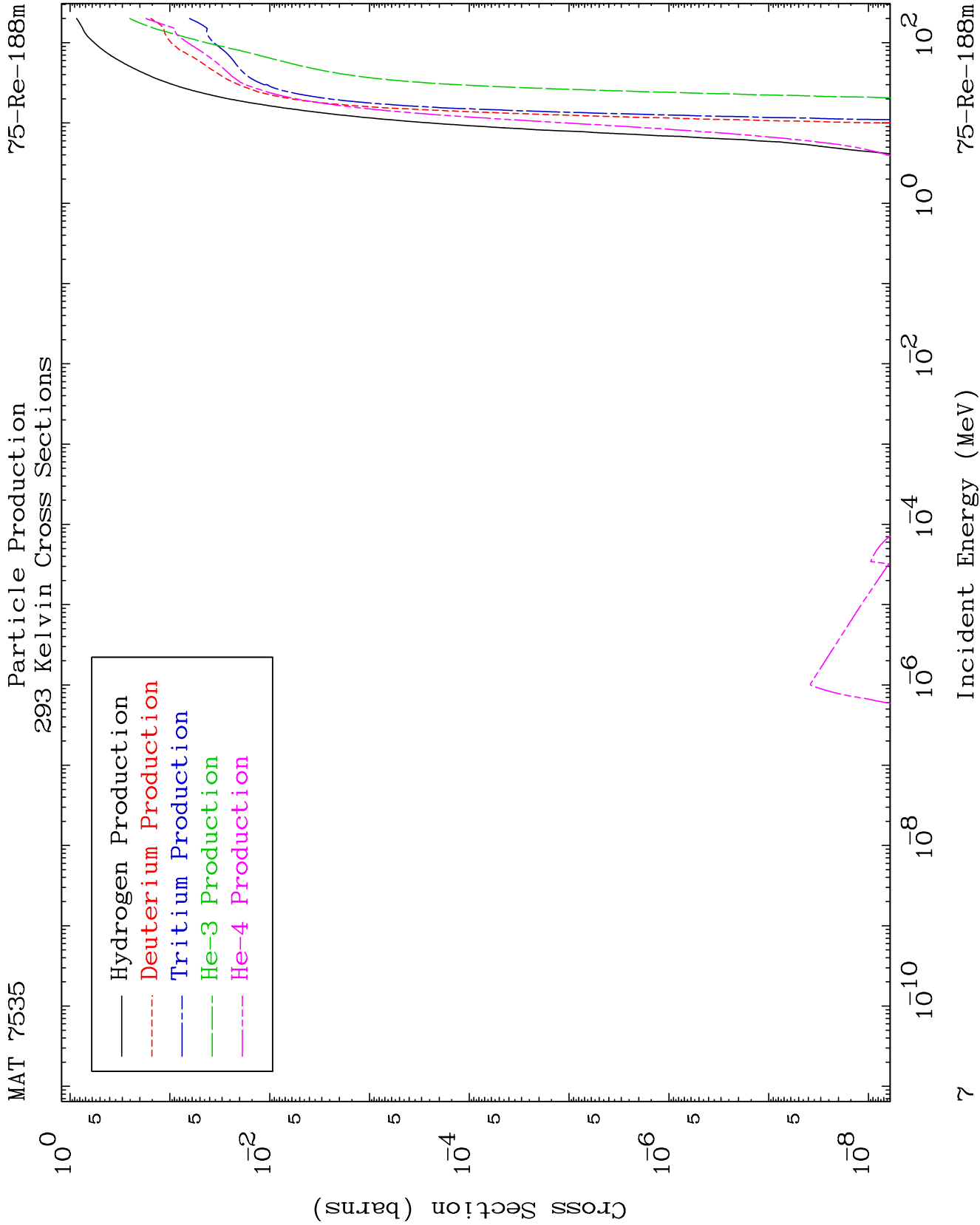




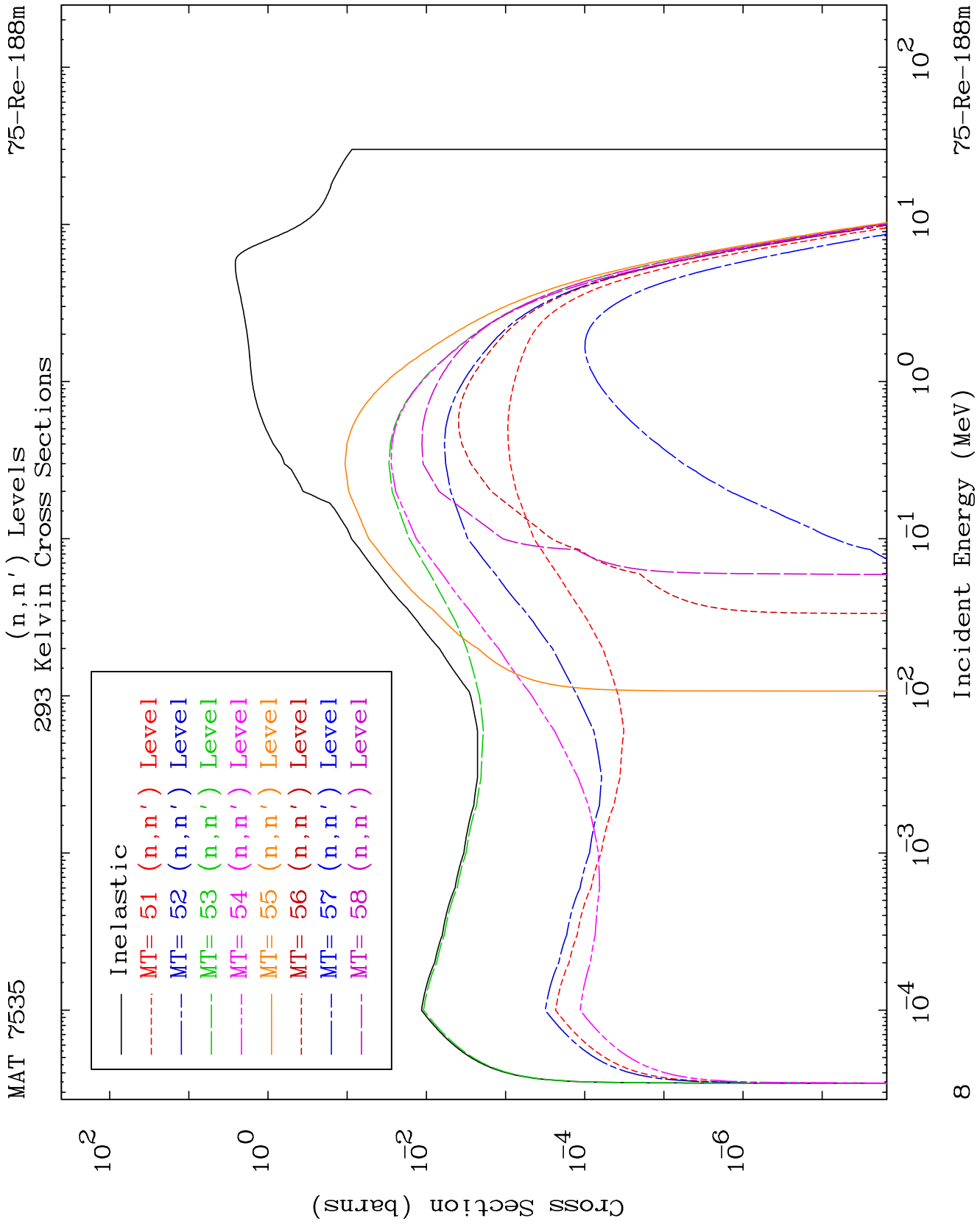


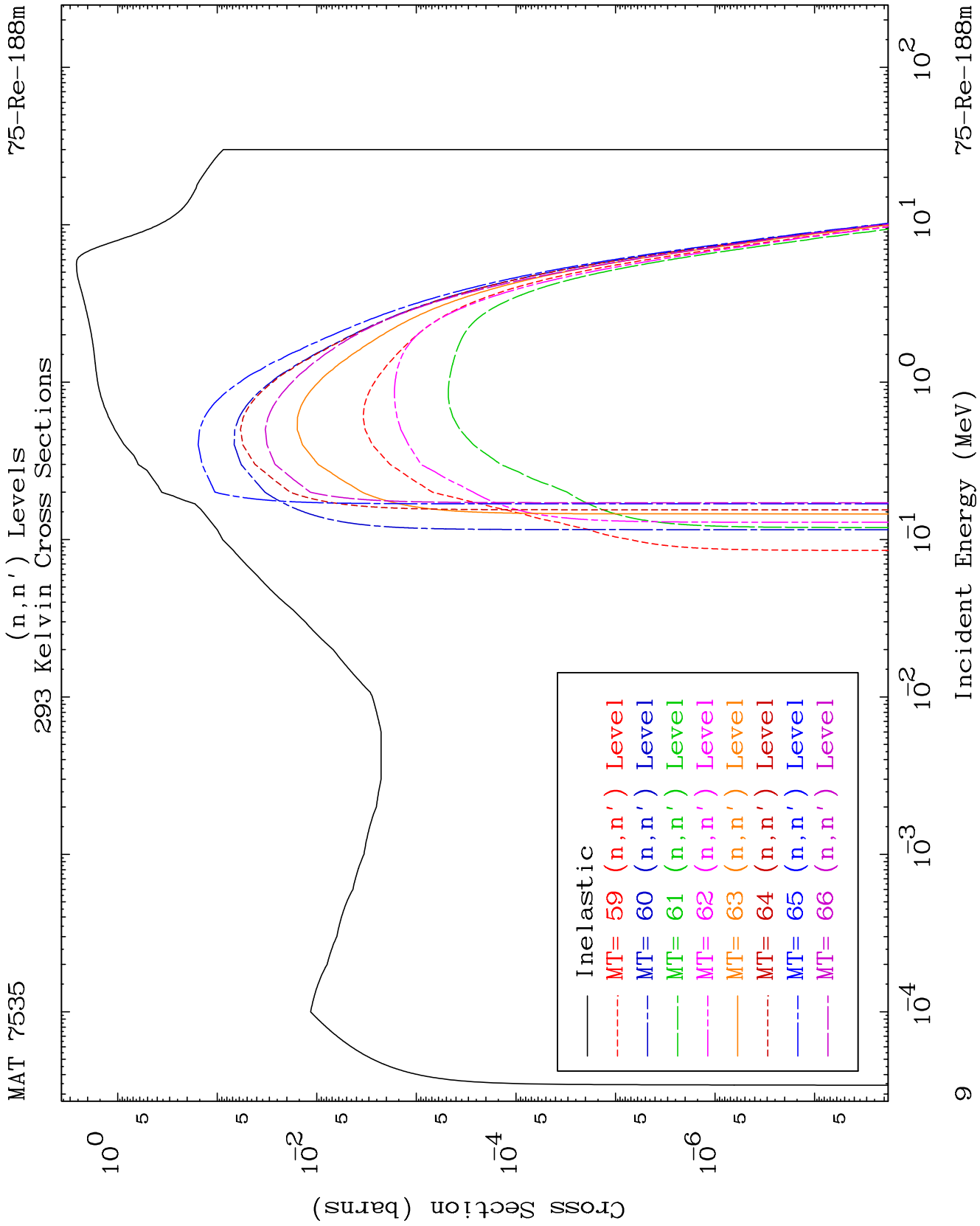


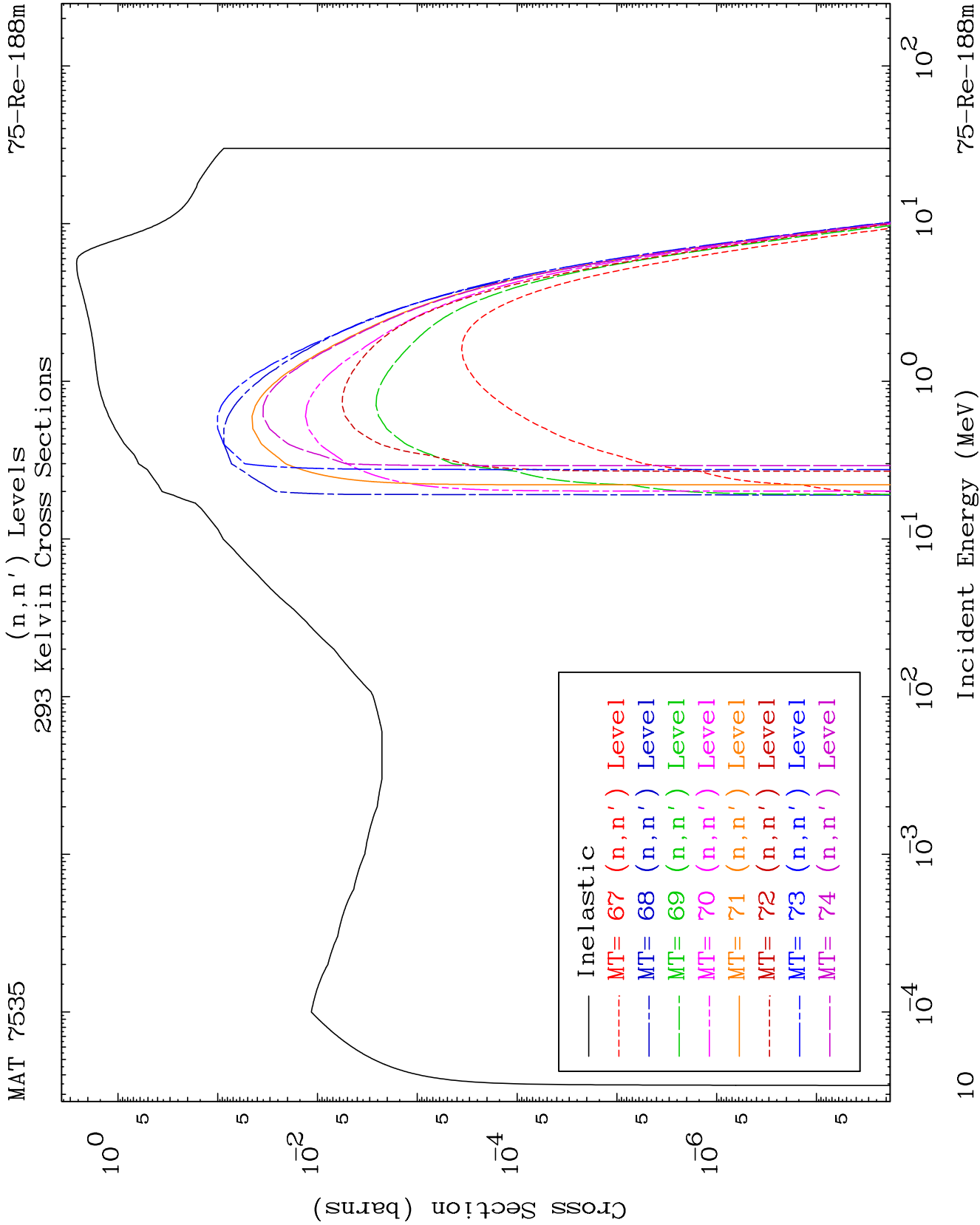


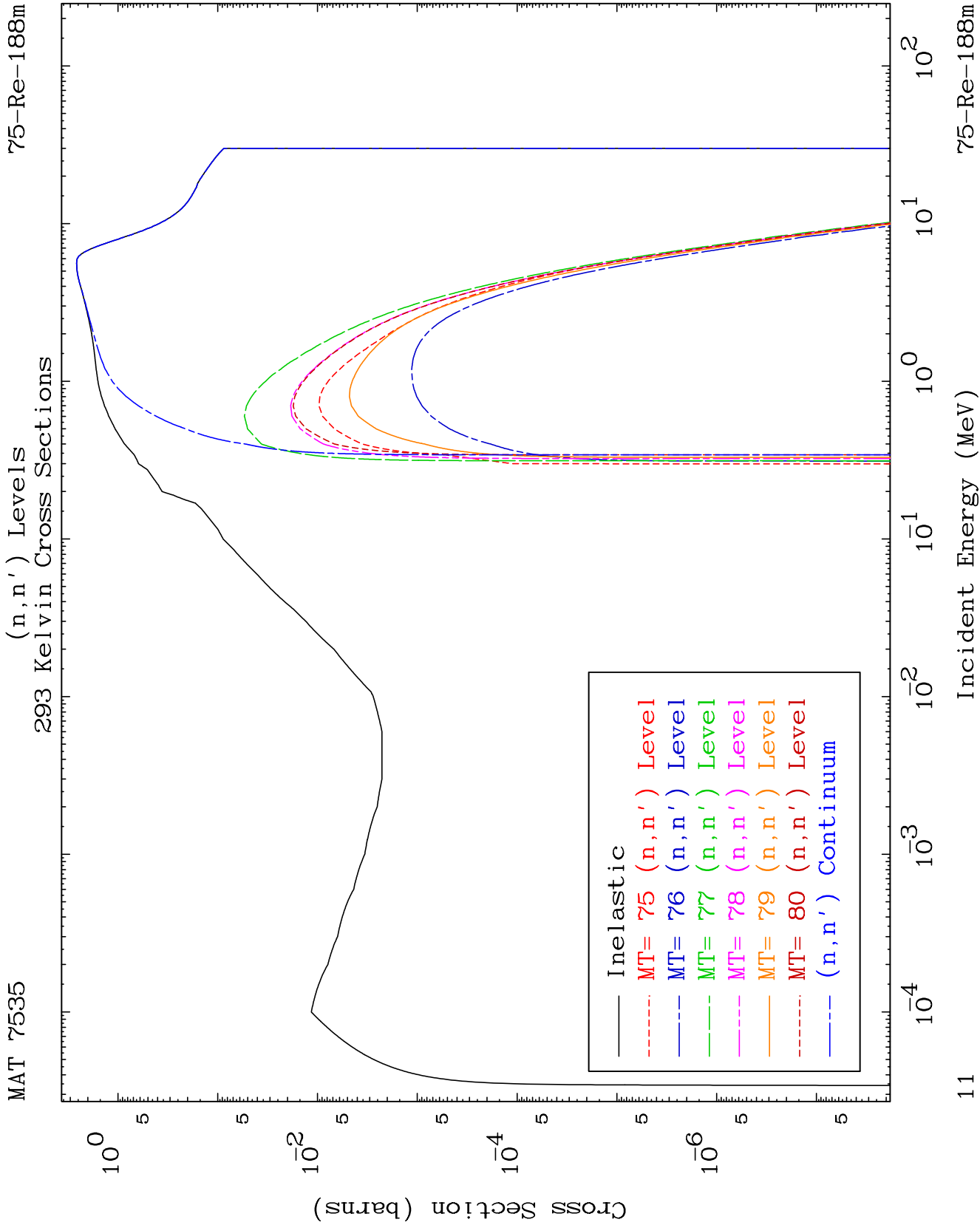








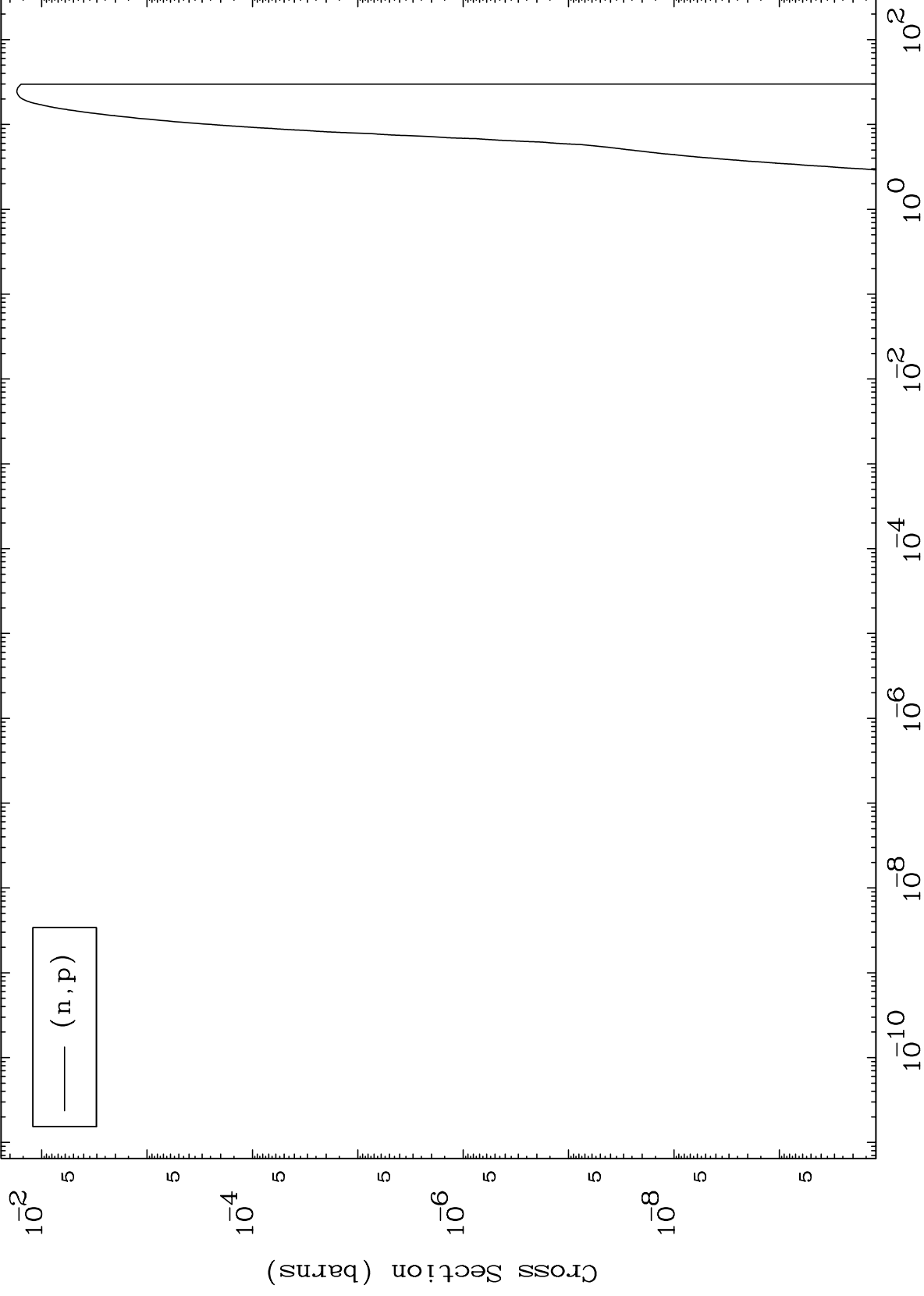




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(n,p) Levels  
293 Kelvin Cross Sections

75-Re-188m



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Incident Energy (MeV)

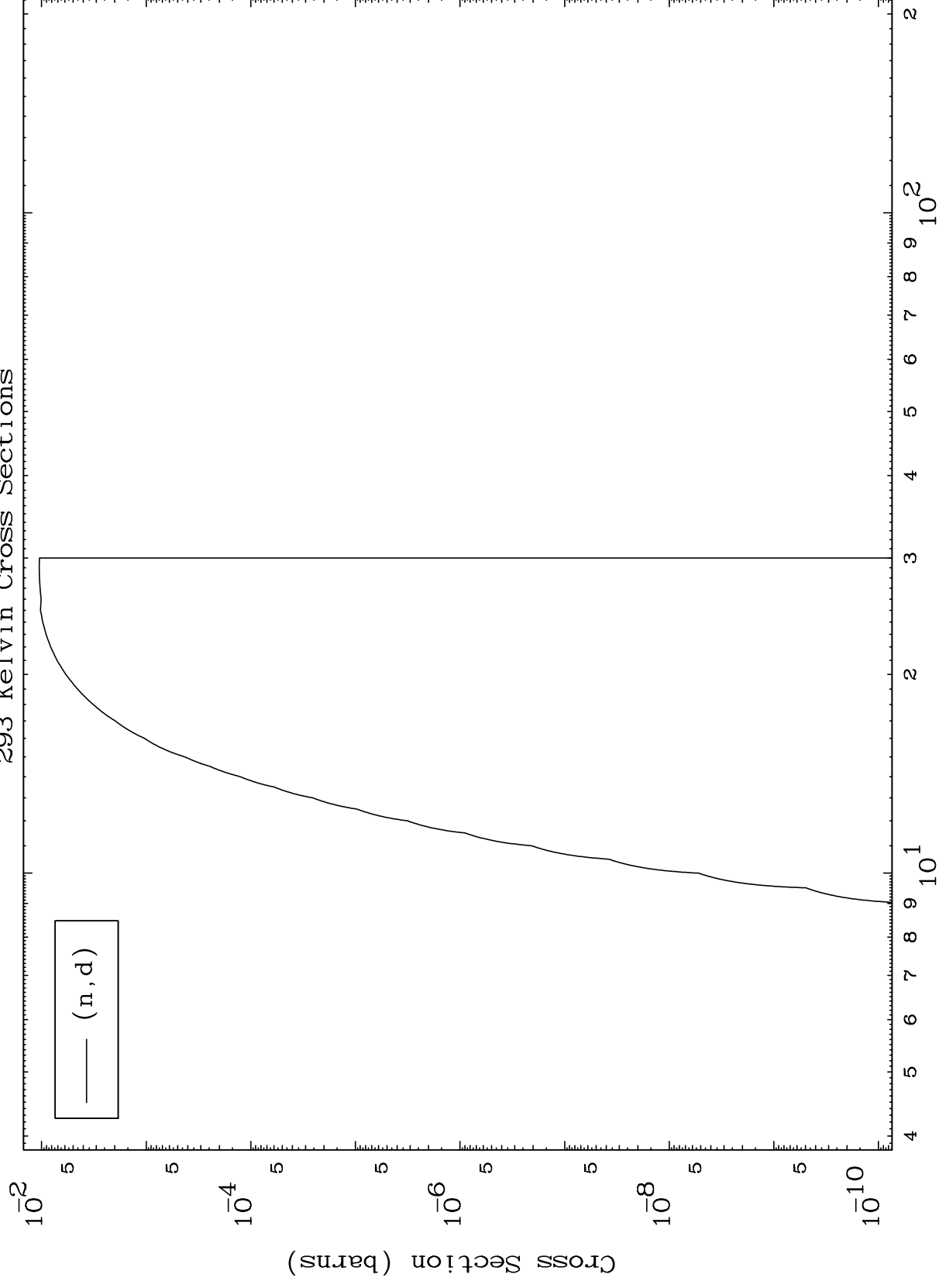
75-Re-188m

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(n,d) Levels

75-Re-188m

293 Kelvin Cross Sections



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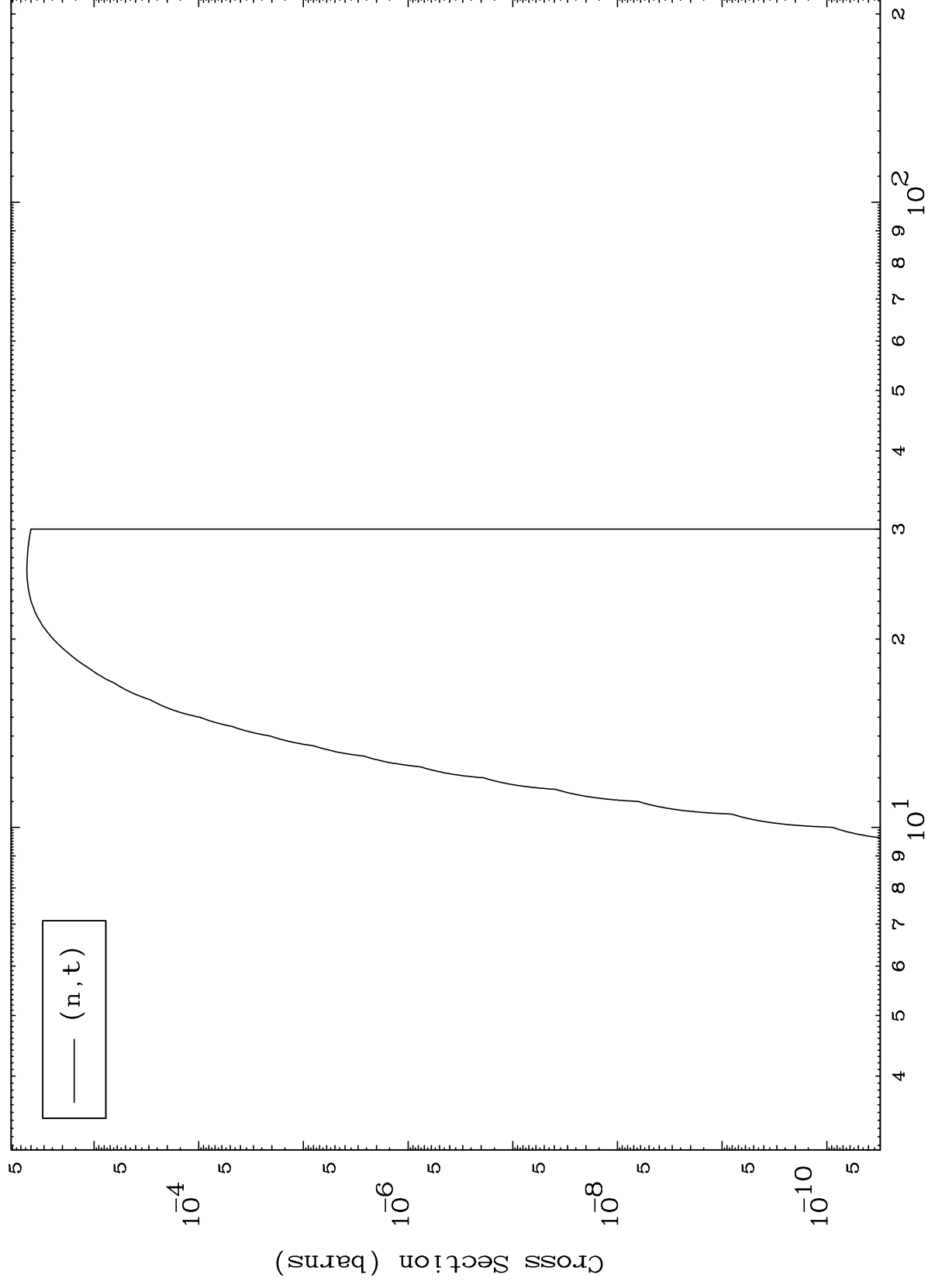
Incident Energy (MeV)

75-Re-188m

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(n, t) Levels  
293 Kelvin Cross Sections

75-Re-188m



14

Incident Energy (MeV)

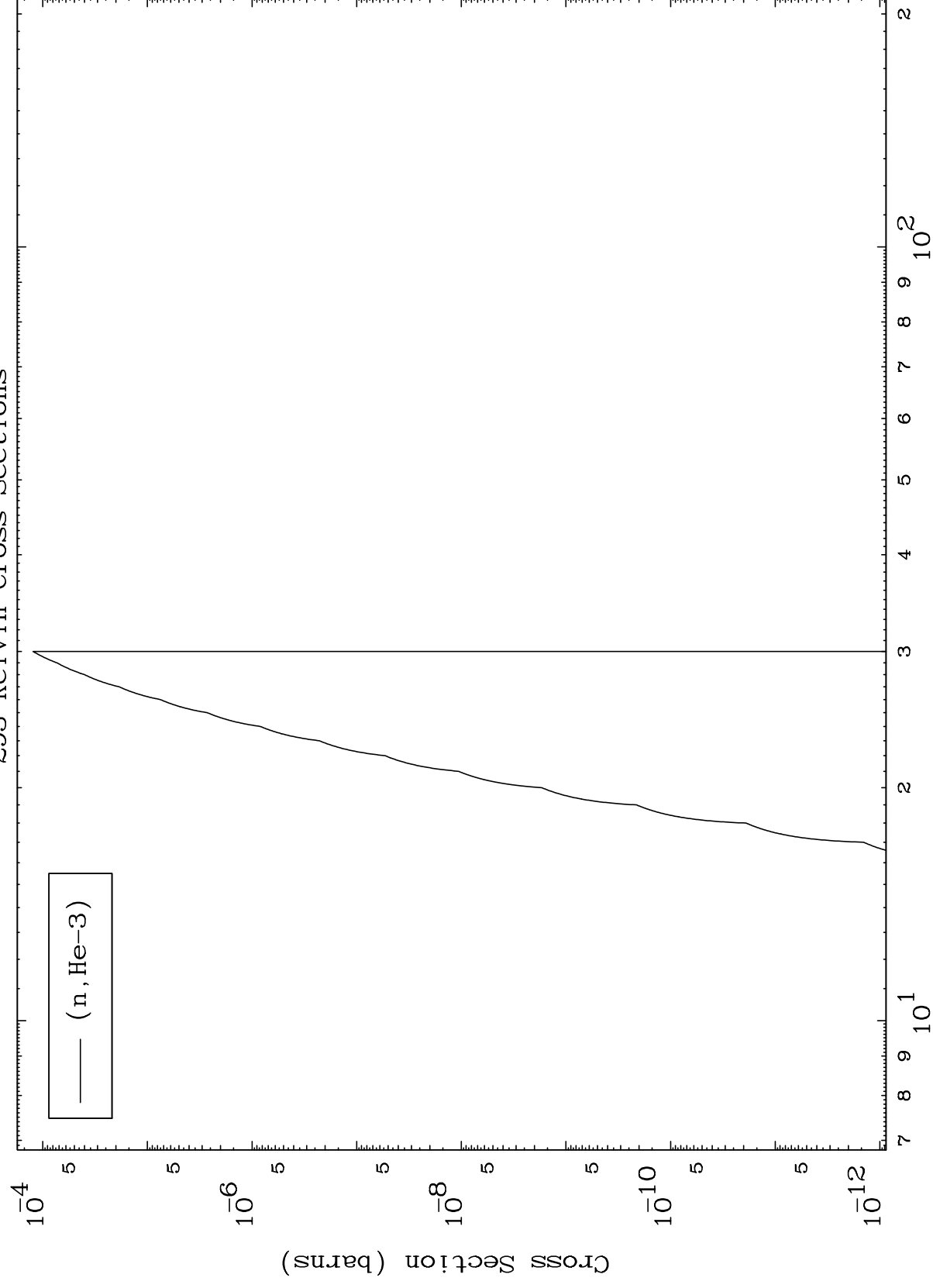
75-Re-188m

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(n, He3) Levels

75-Re-188m

293 Kelvin Cross Sections



15

Incident Energy (MeV)

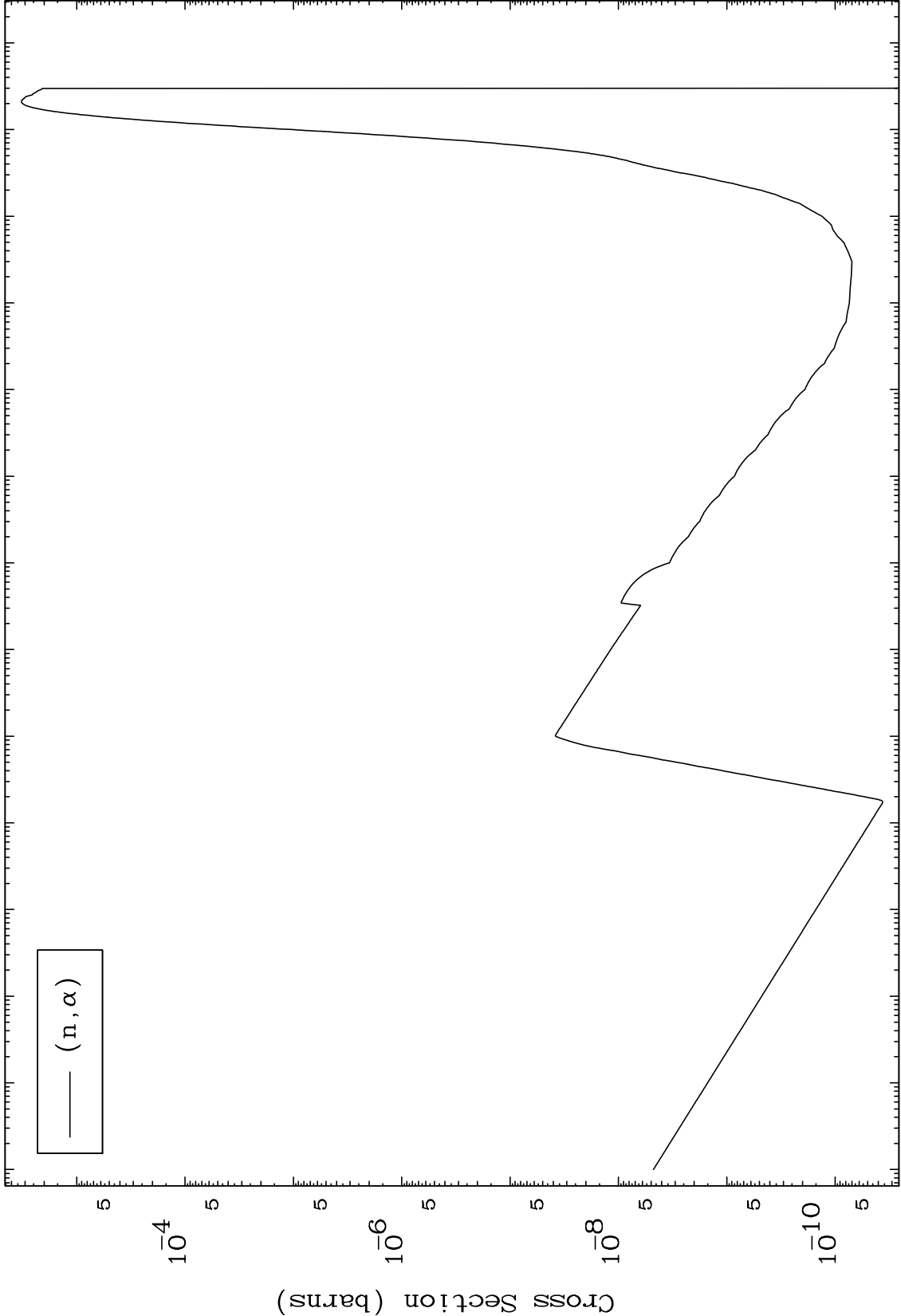
75-Re-188m

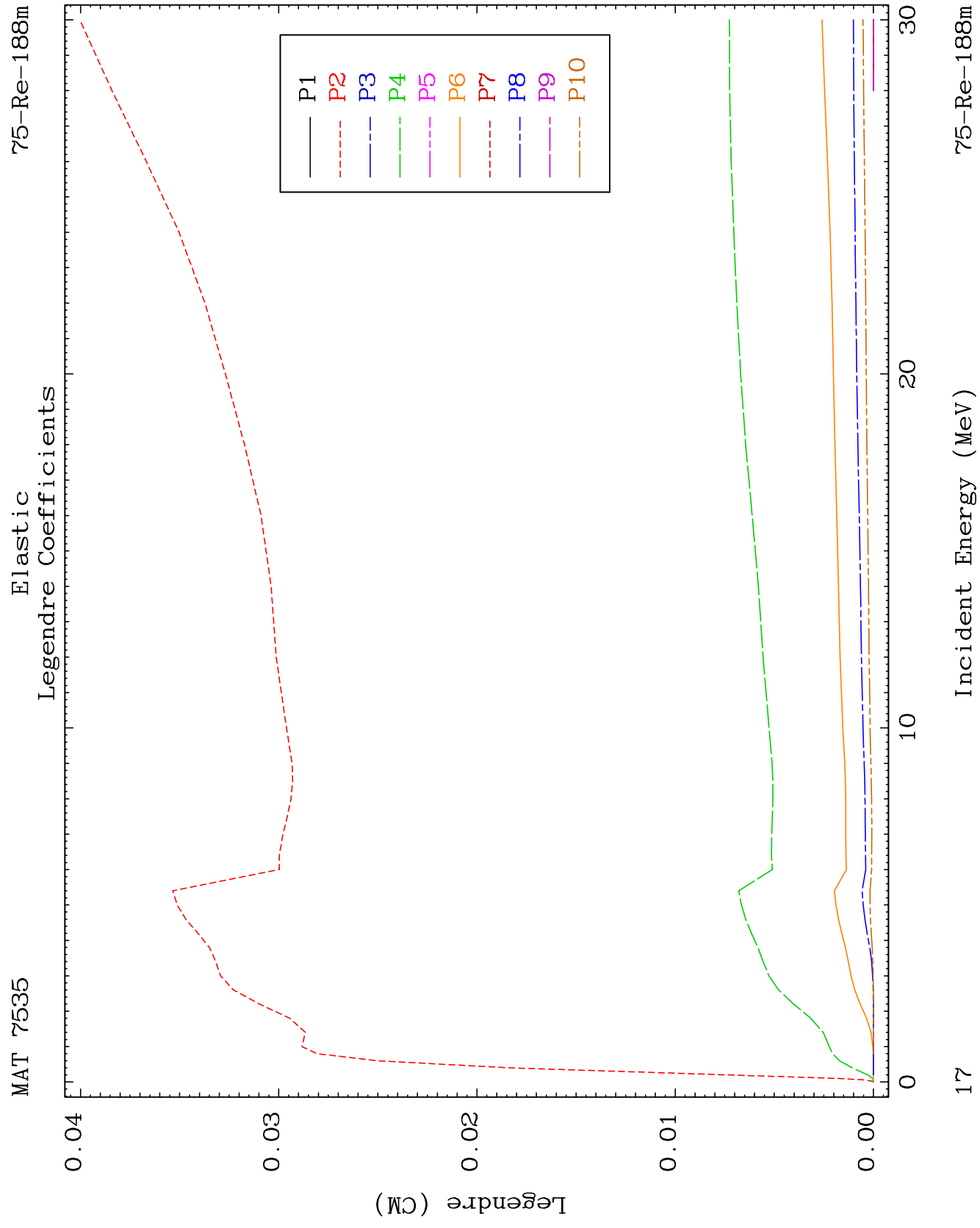


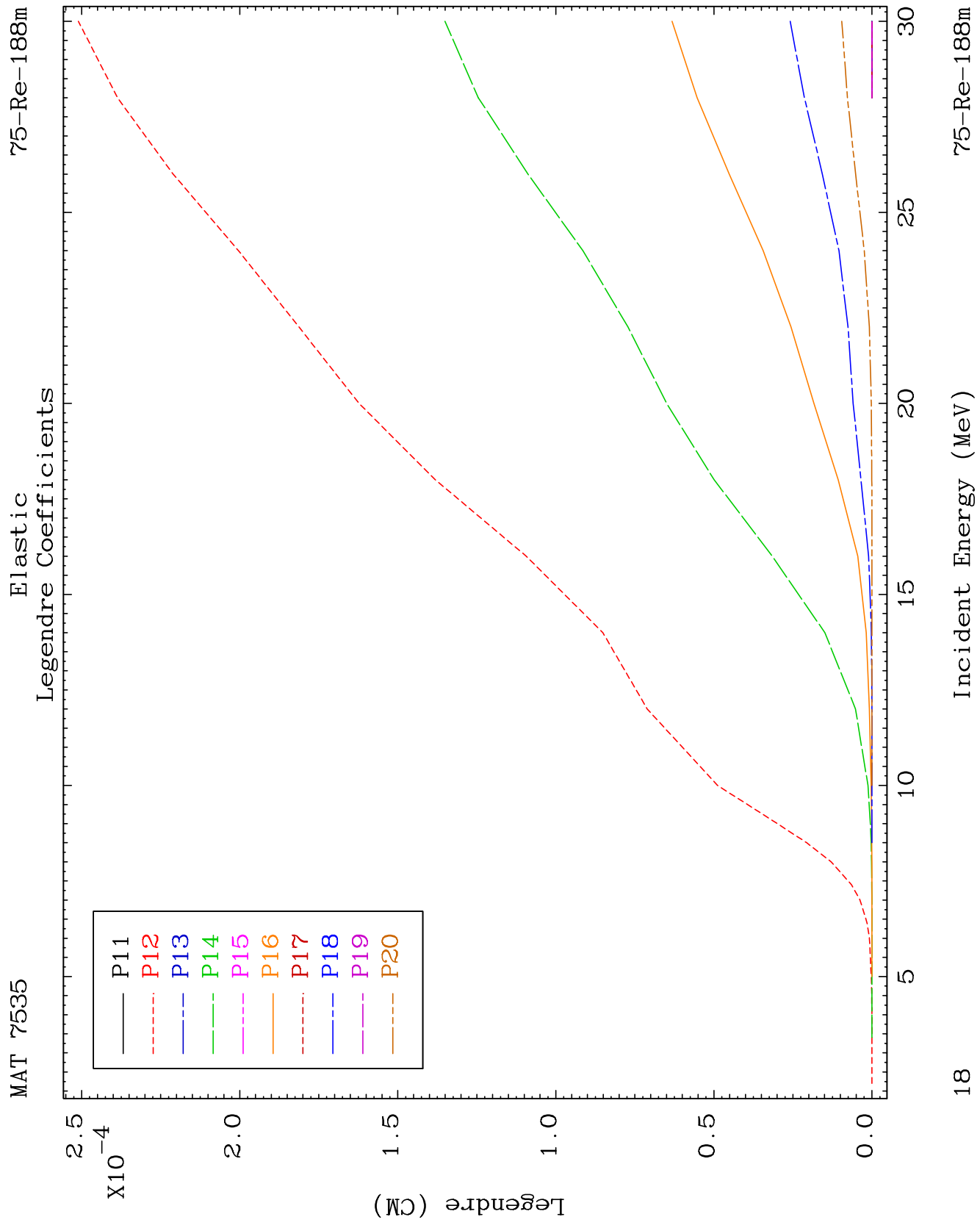
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(n, $\alpha$ ) Levels  
293 Kelvin Cross Sections

75-Re-188m



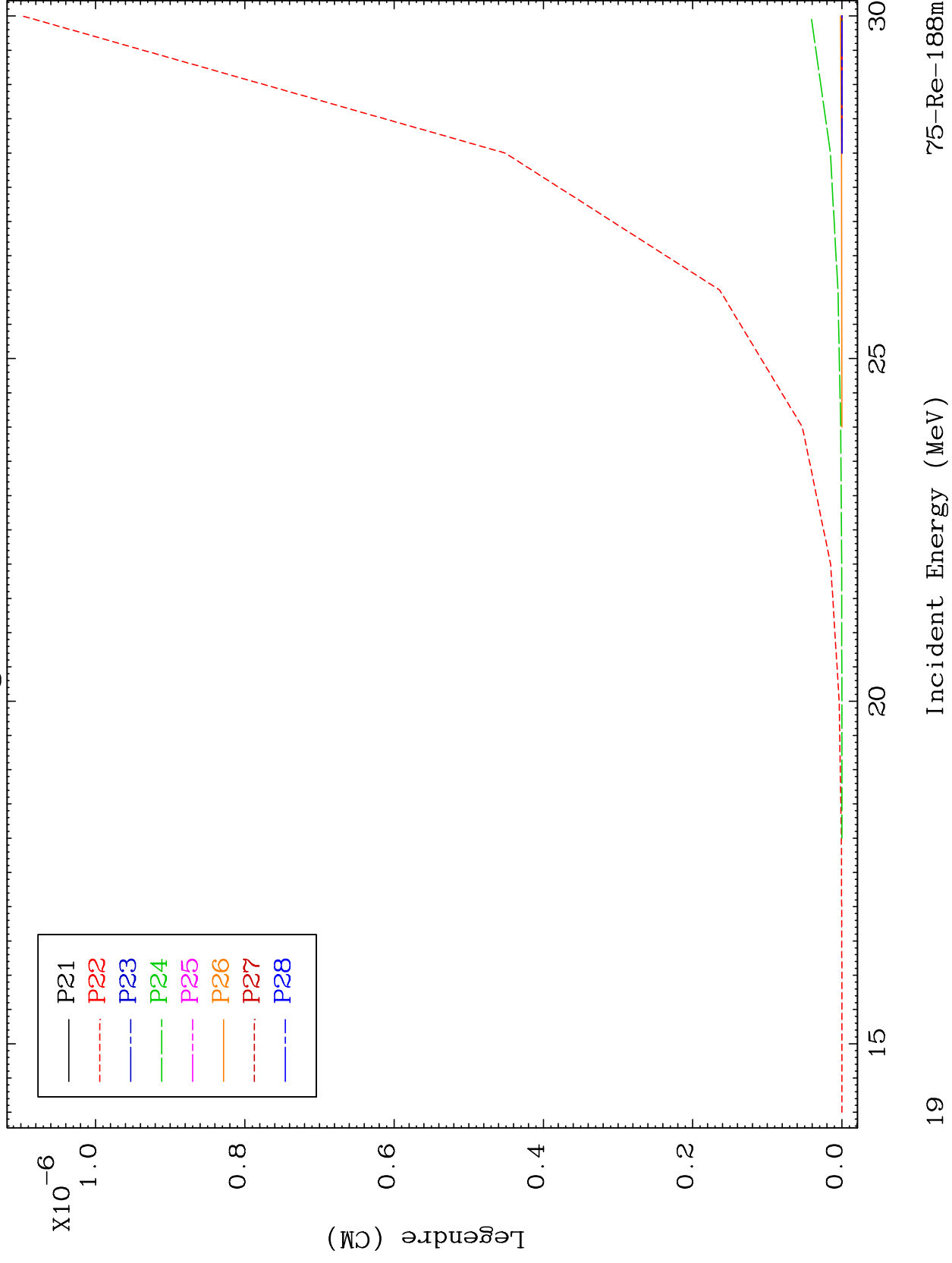




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## Elastic Legendre Coefficients

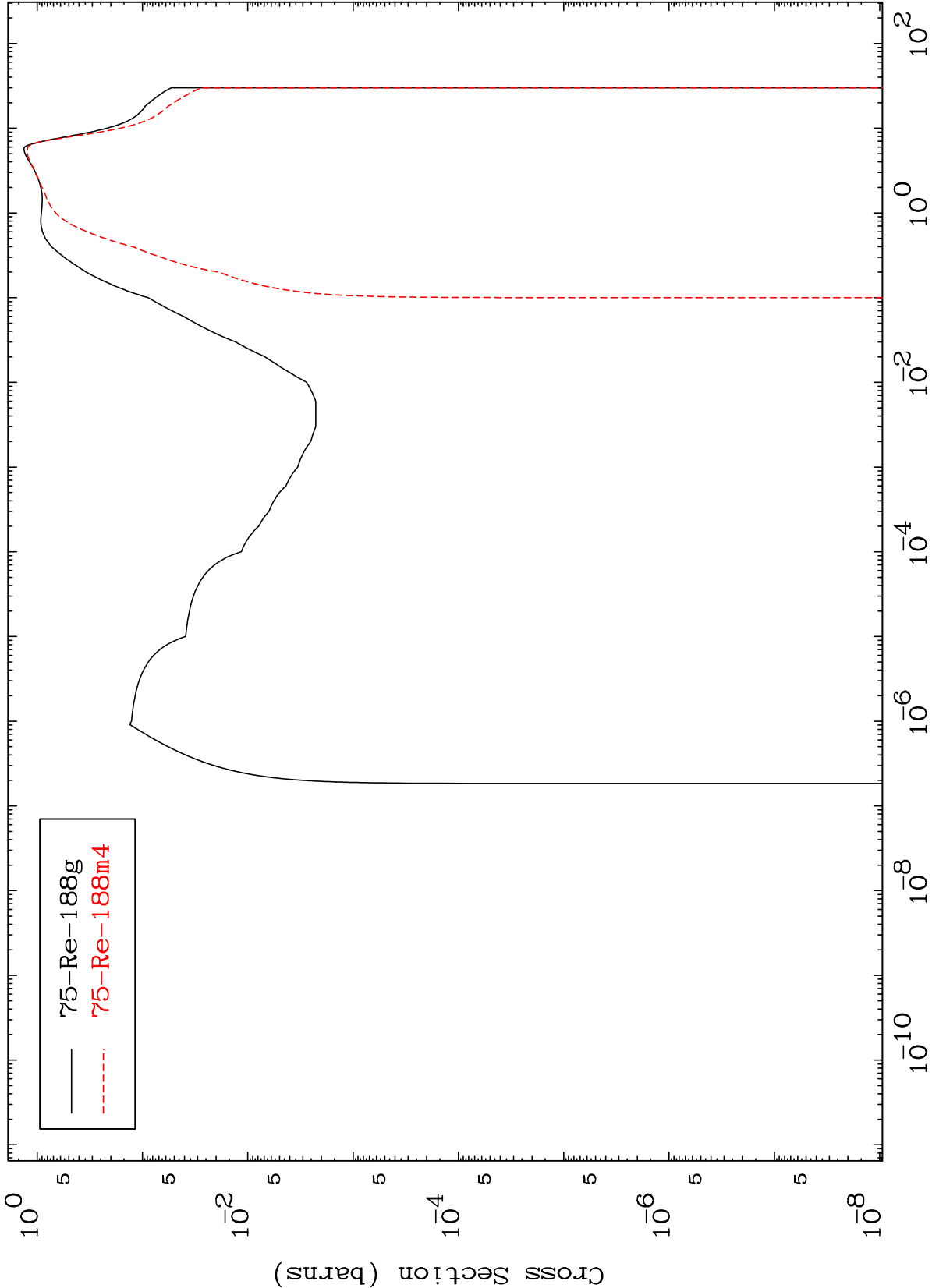
75-Re-188m



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<sup>75</sup>Re-188m

Inelastic  
Radionuclide Production Cross Section



Incident Energy (MeV)

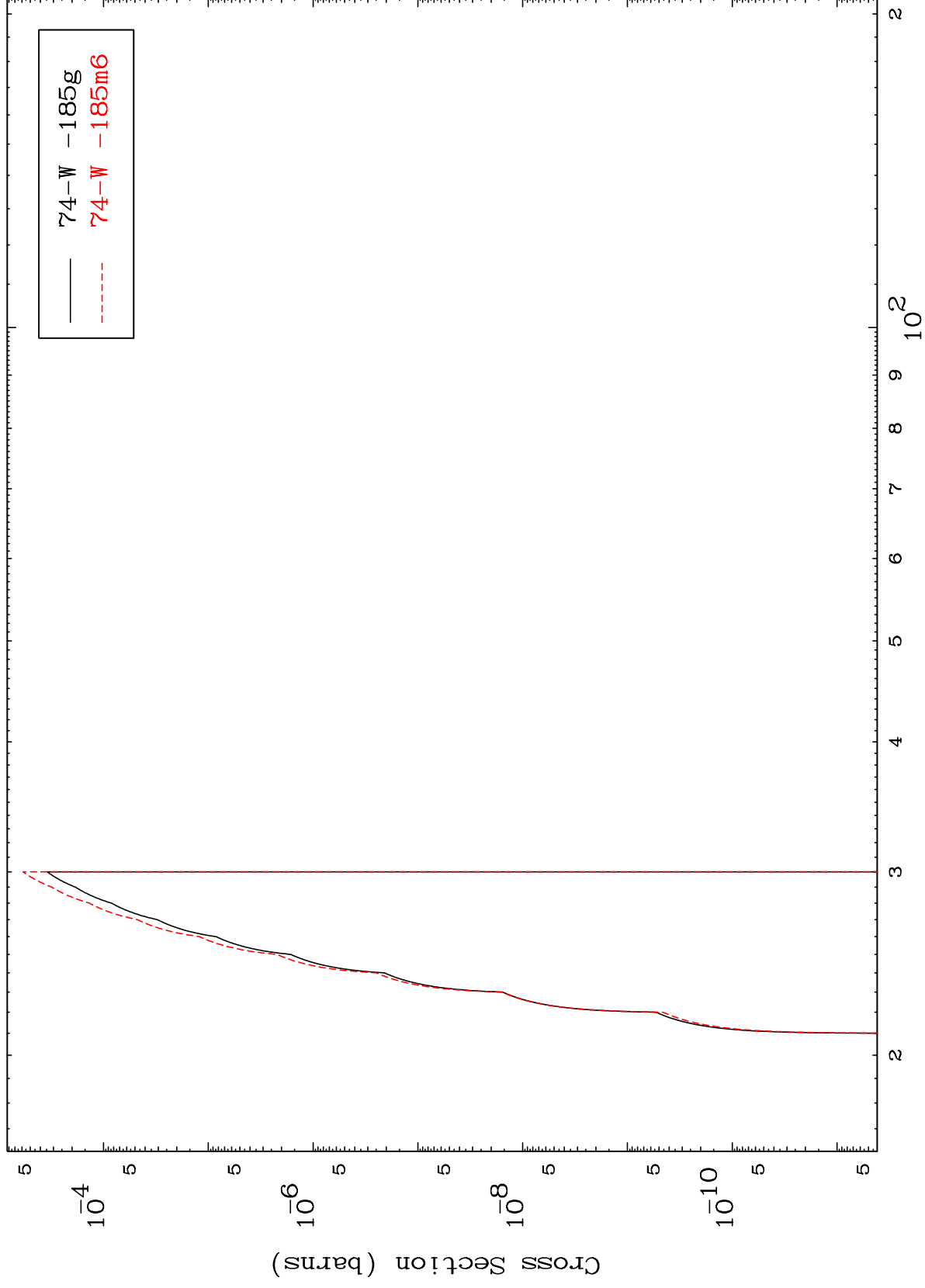
<sup>75</sup>Re-188m

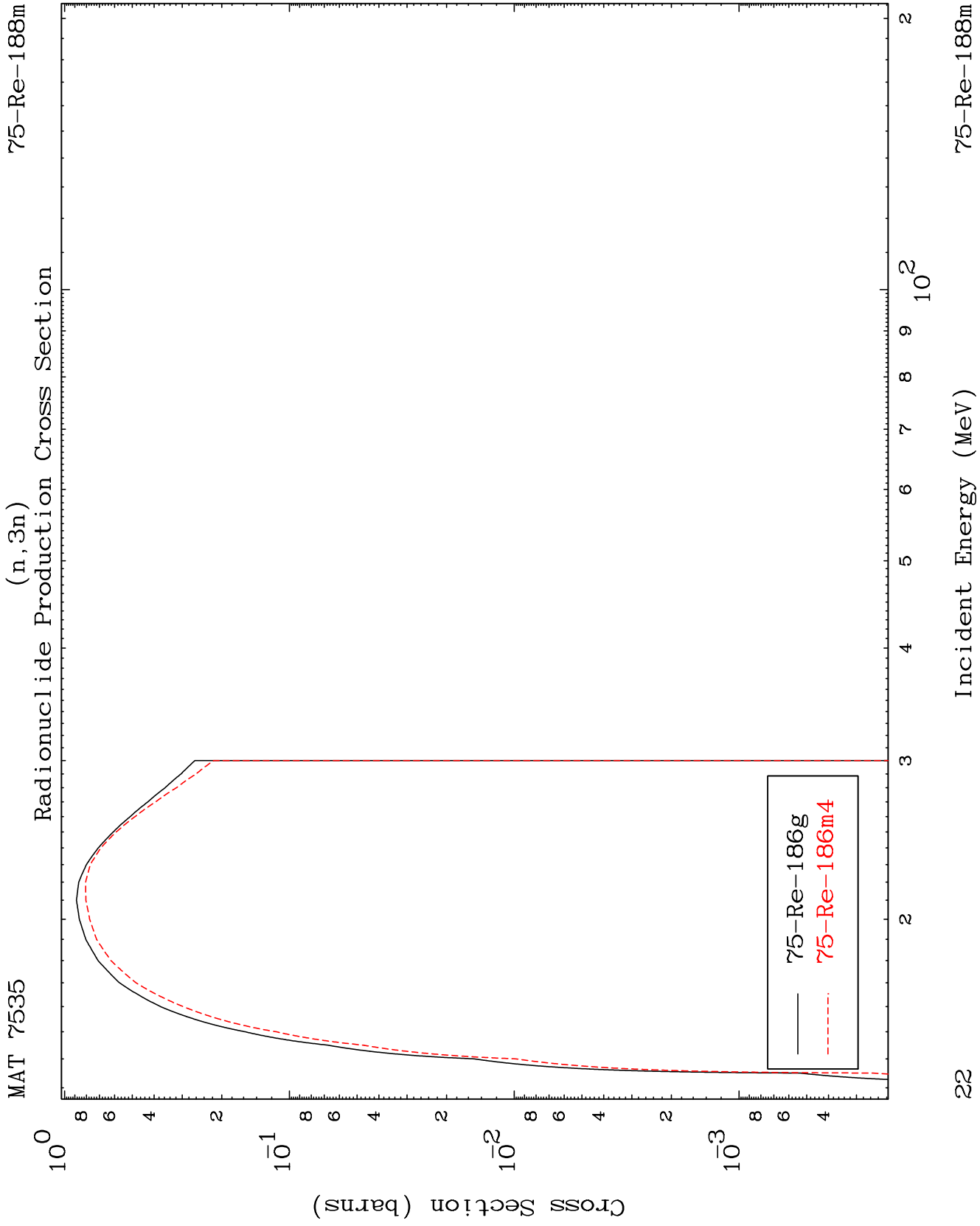
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(n,2n) d

<sup>75</sup>Re-188m

Radionuclide Production Cross Section



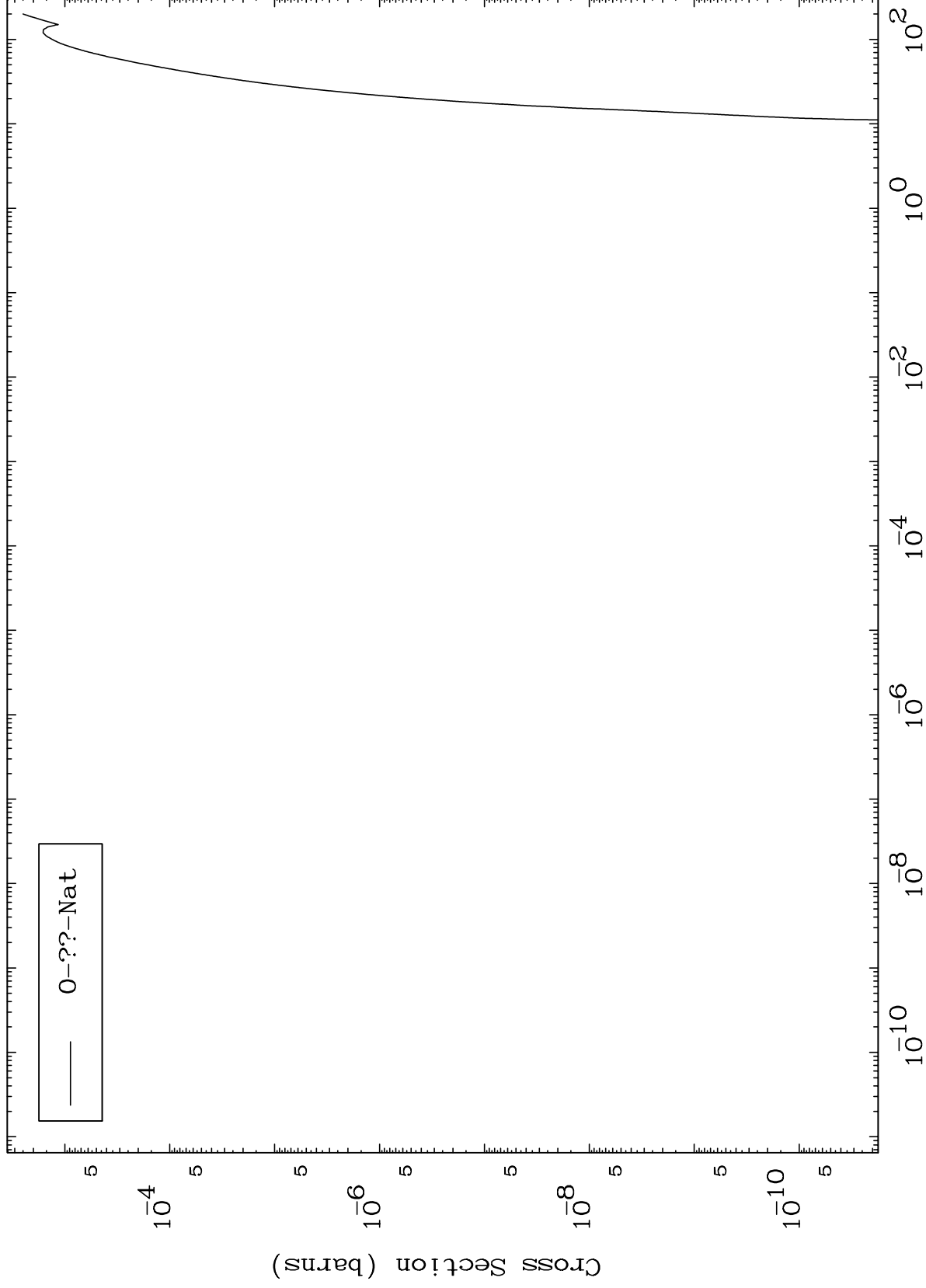


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Fission

<sup>75</sup>Re-188m

Radionuclide Production Cross Section

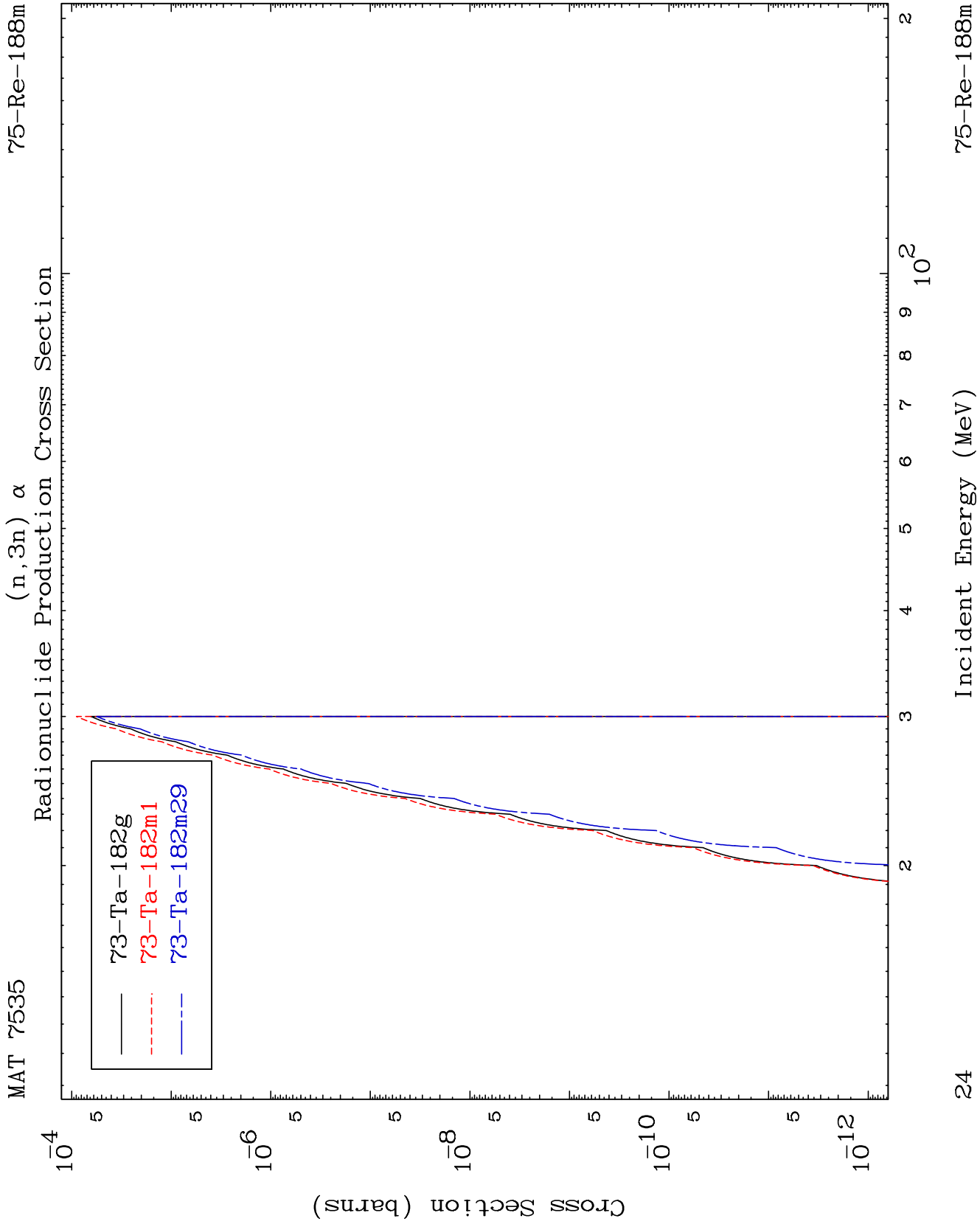


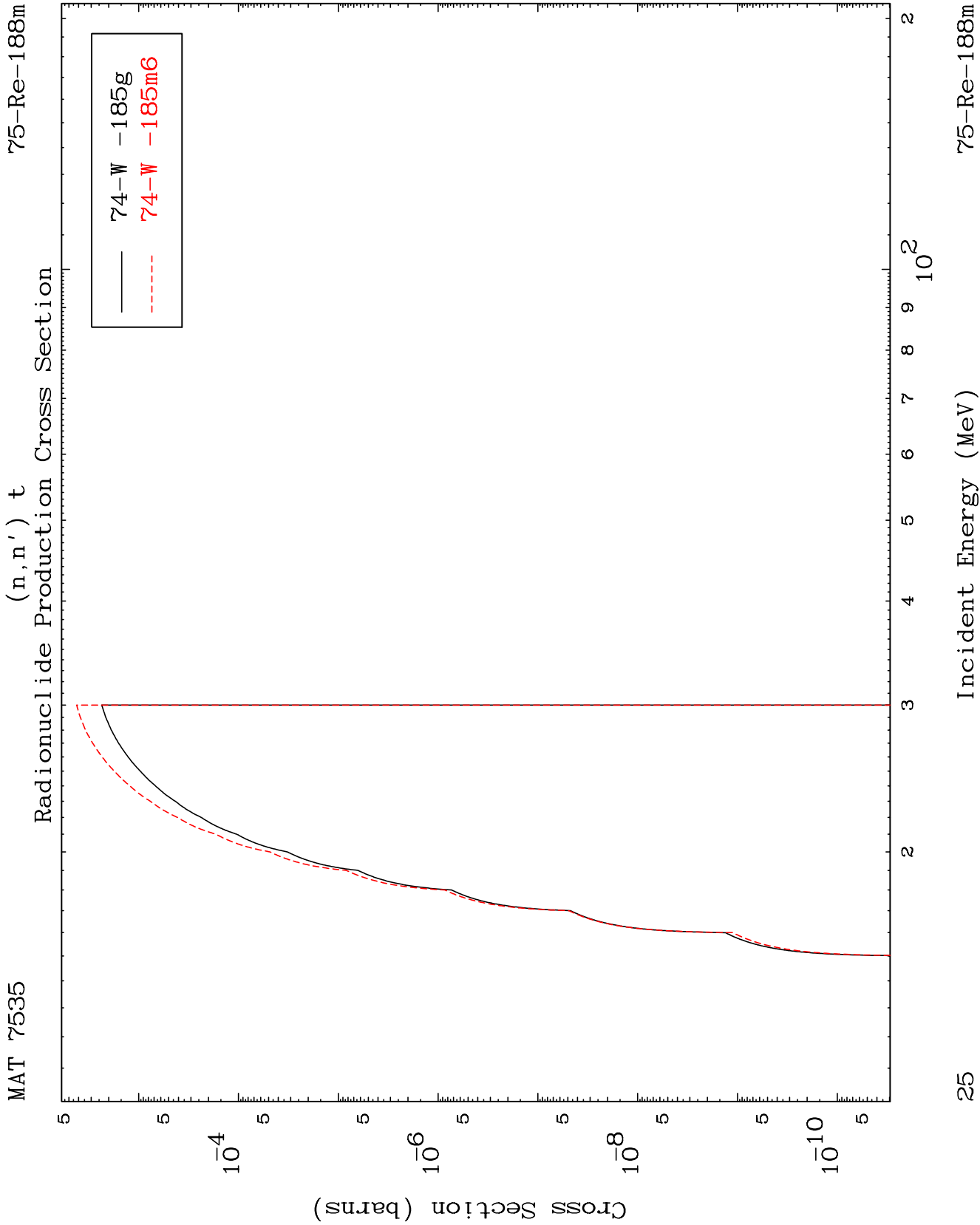
23

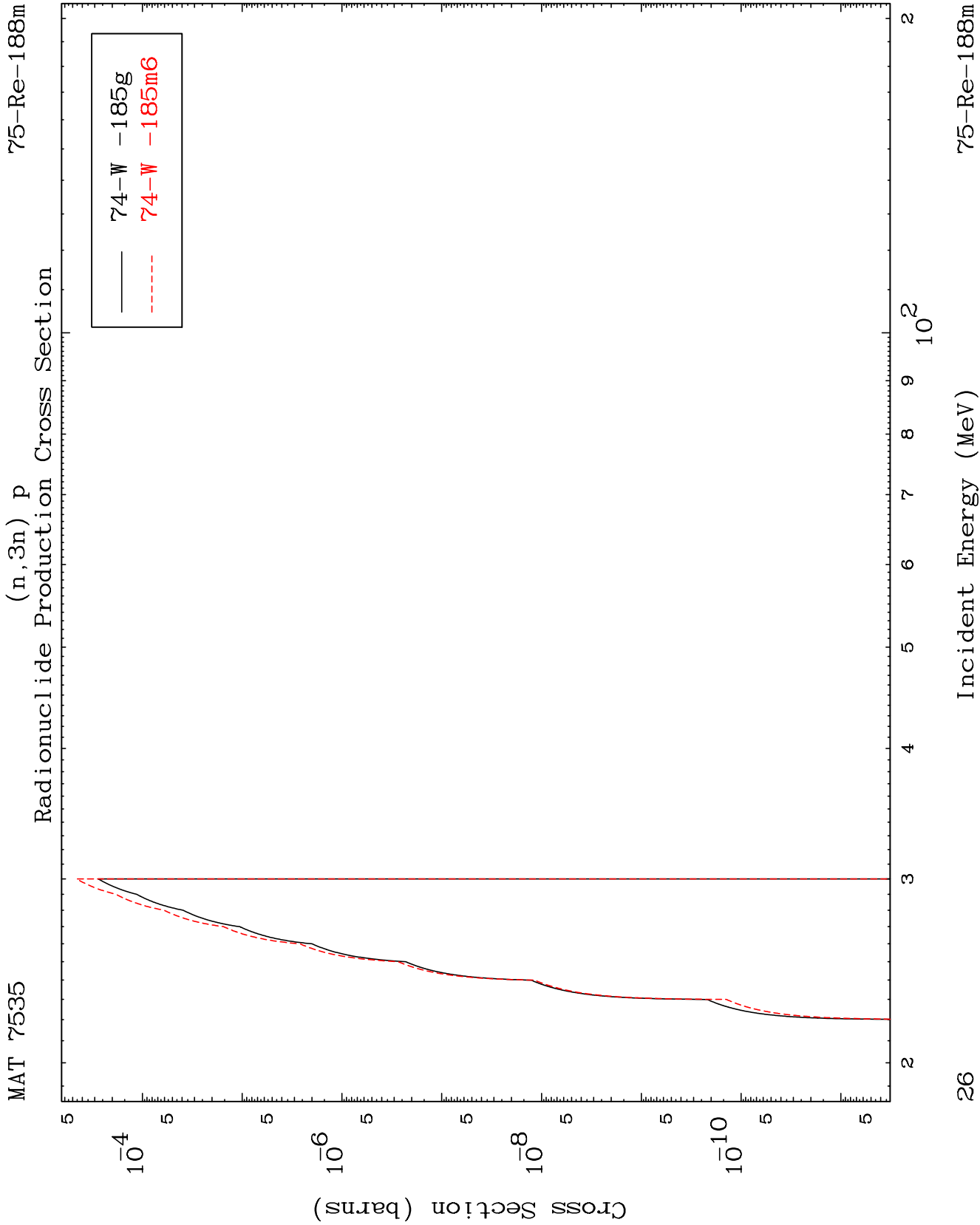
Incident Energy (MeV)

<sup>75</sup>Re-188m







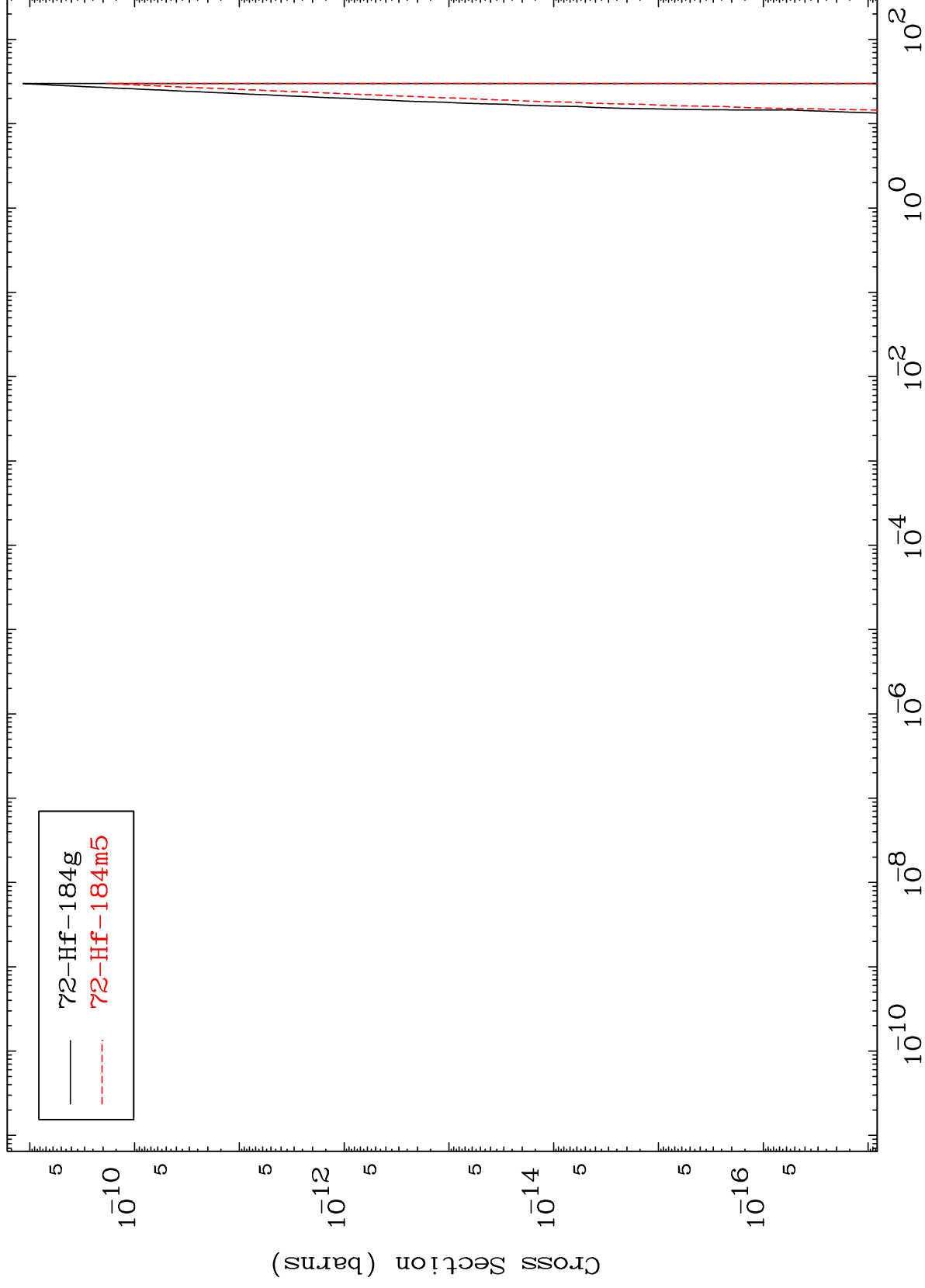


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(n,p)  $\alpha$

$^{75}\text{Re-188m}$

Radionuclide Production Cross Section



27

Incident Energy (MeV)

$^{75}\text{Re-188m}$