

Program EVALPLOT
(Version 2021-1)

by

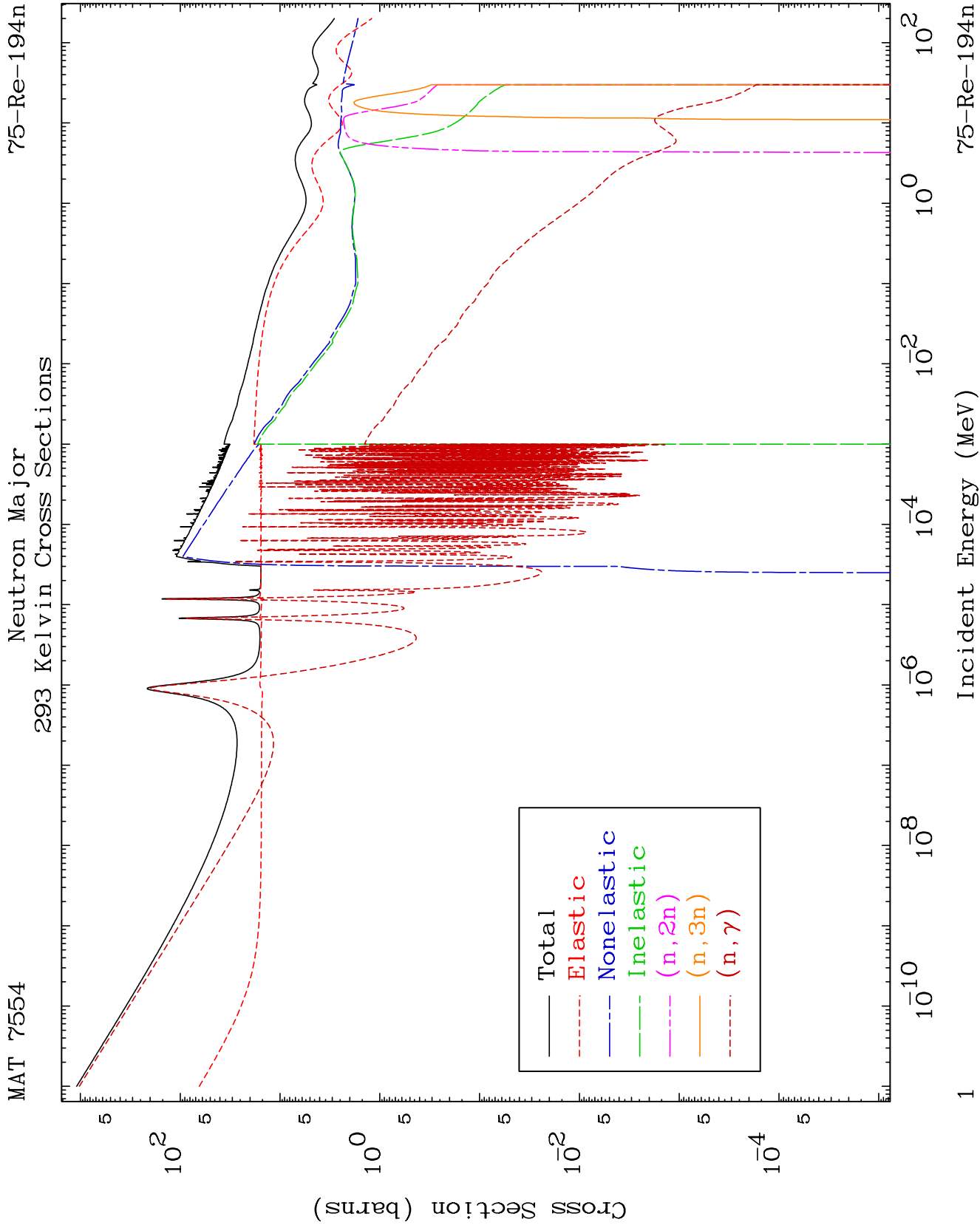
Dermott E. Cullen
(Present Contact Information)

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E.Mail: redcullen1@comcast.net
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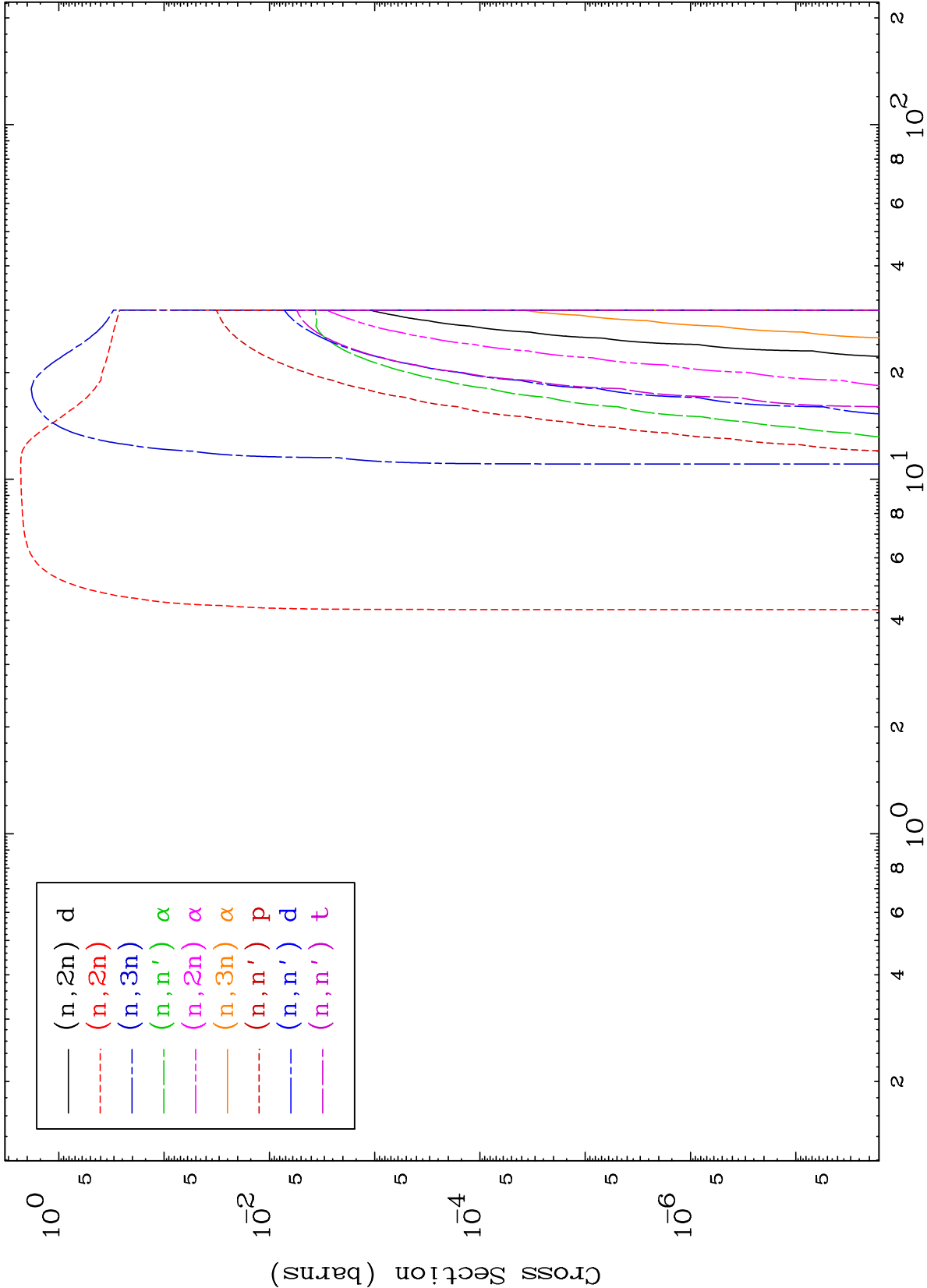
Press Mouse Button to Start

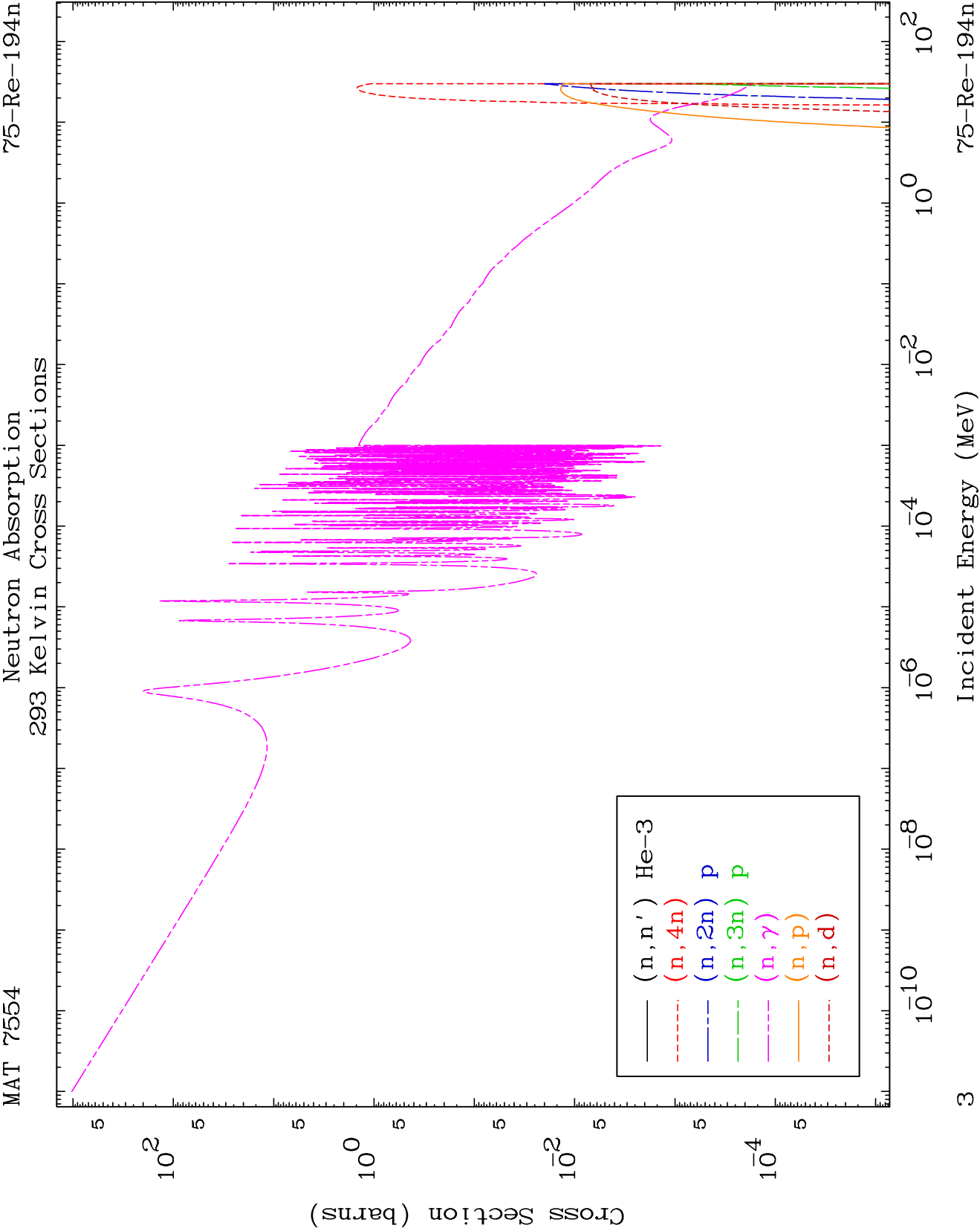


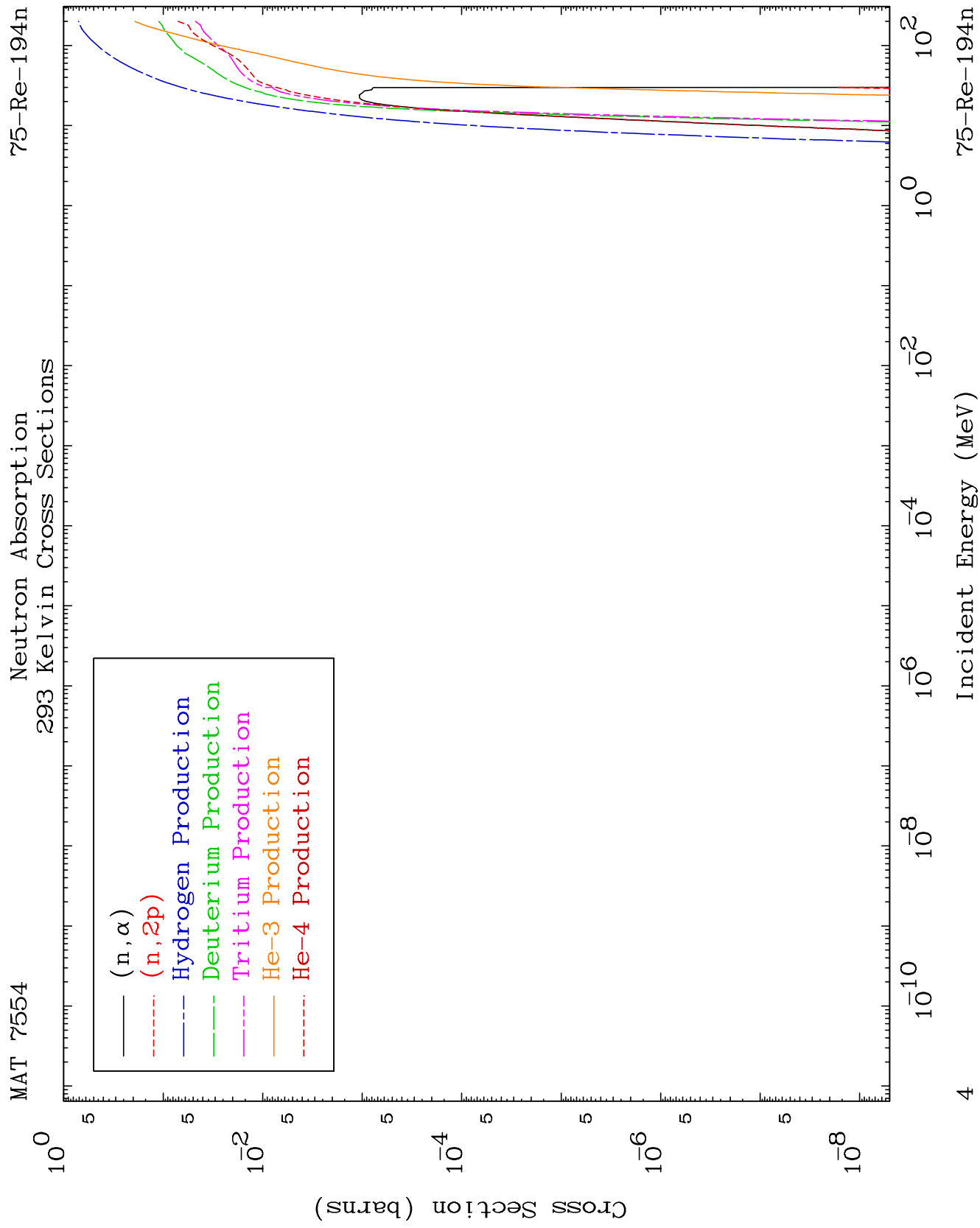
MAT 7554

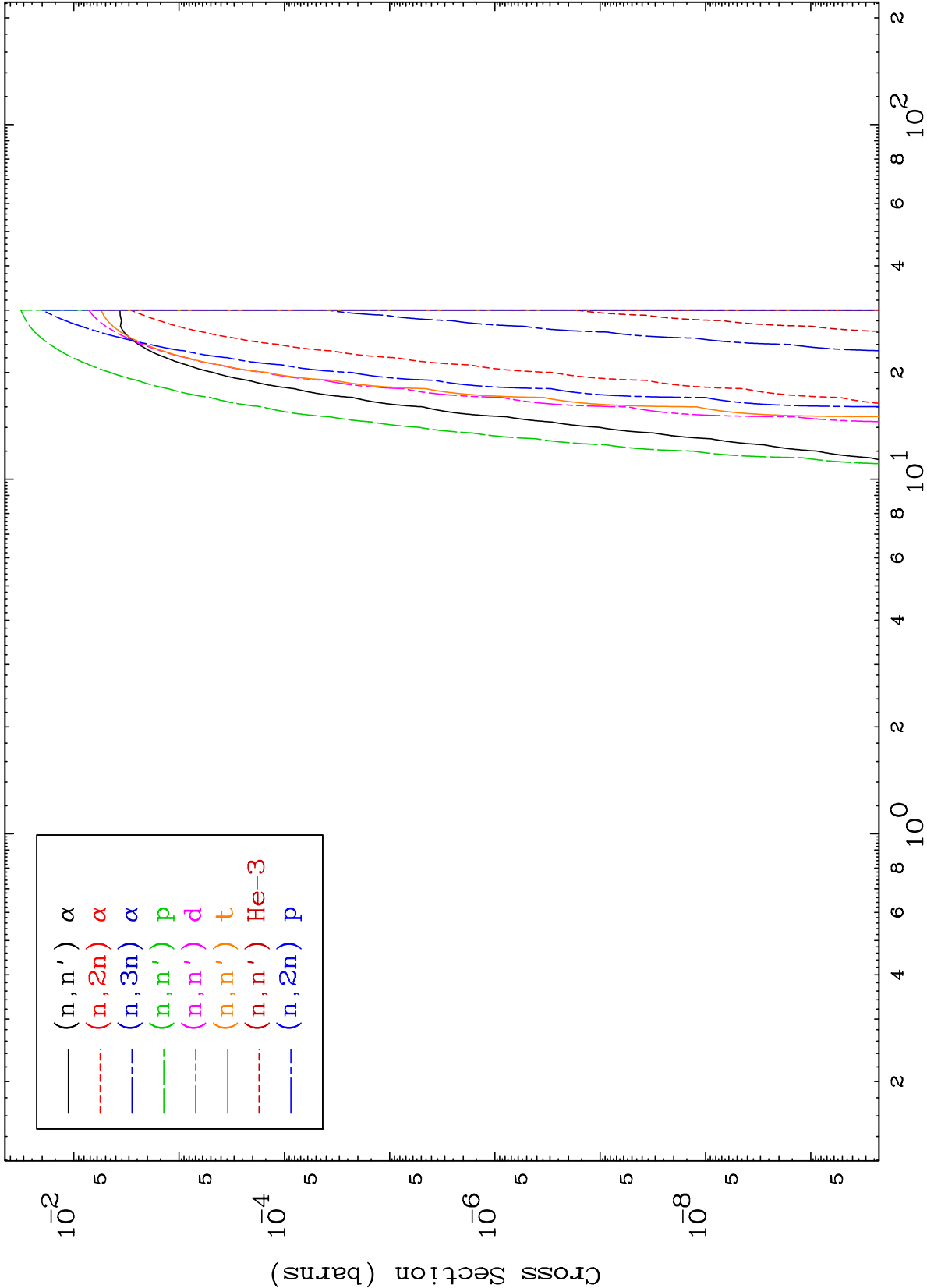
Neutron Absorption

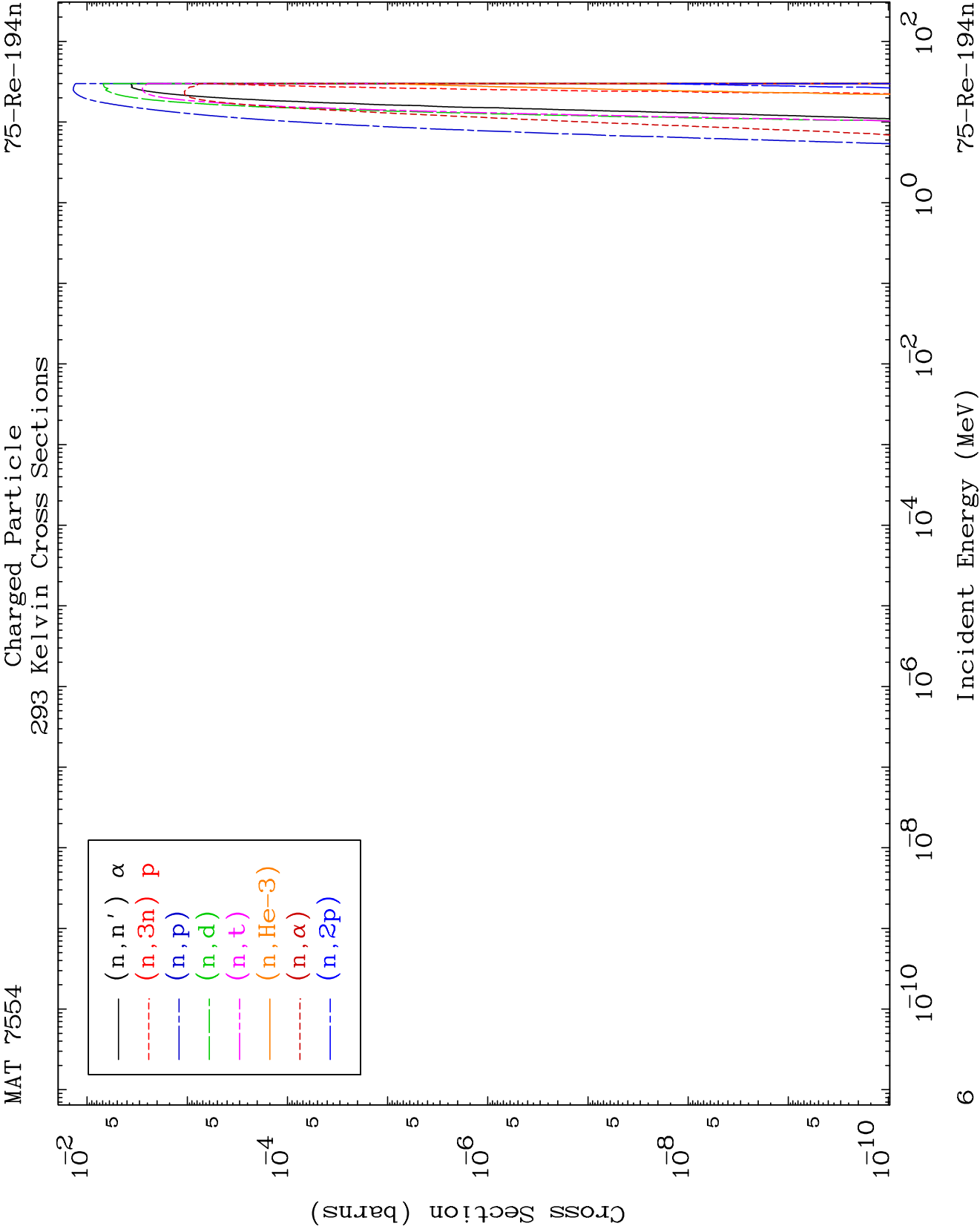
²⁹³Re-194n

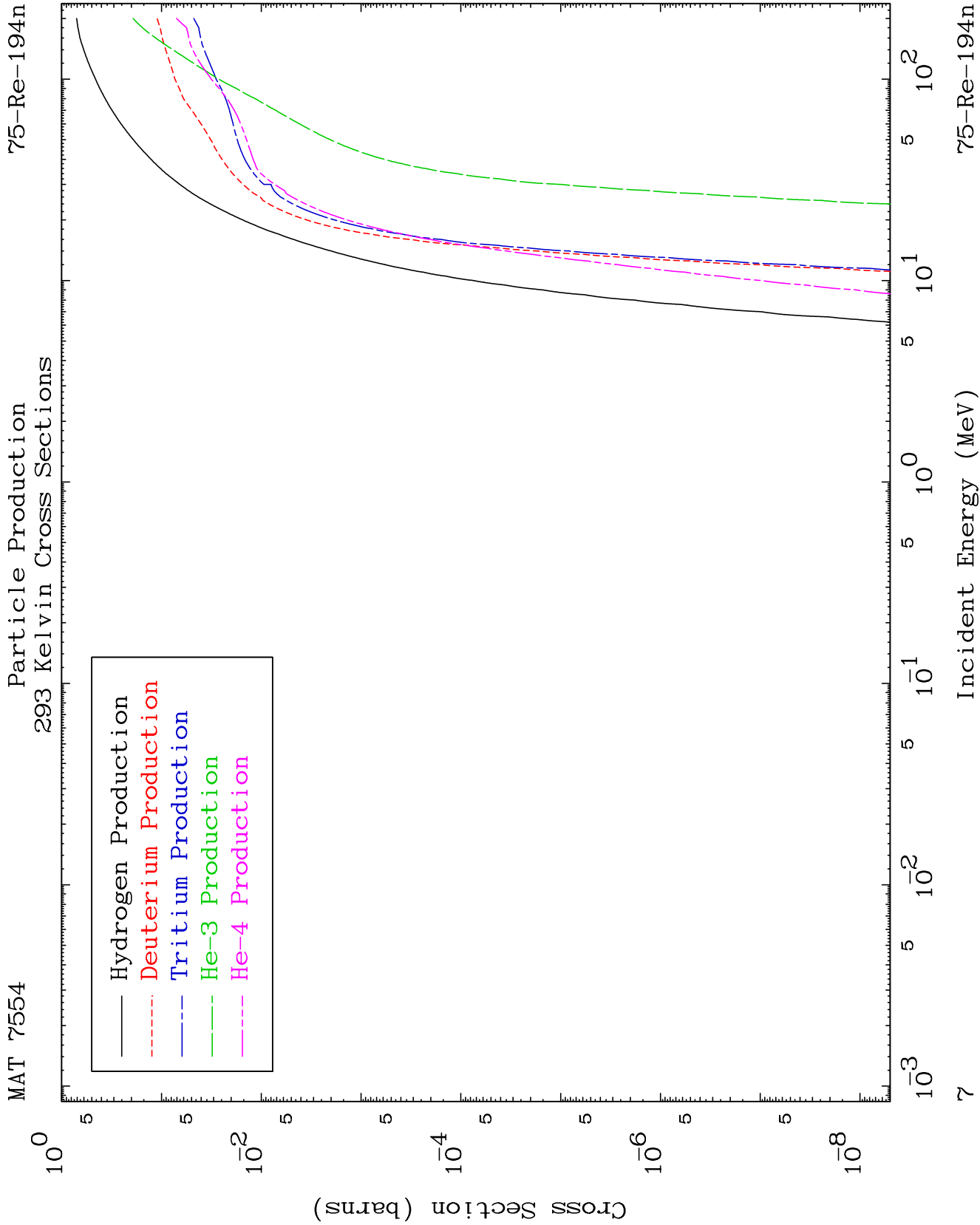


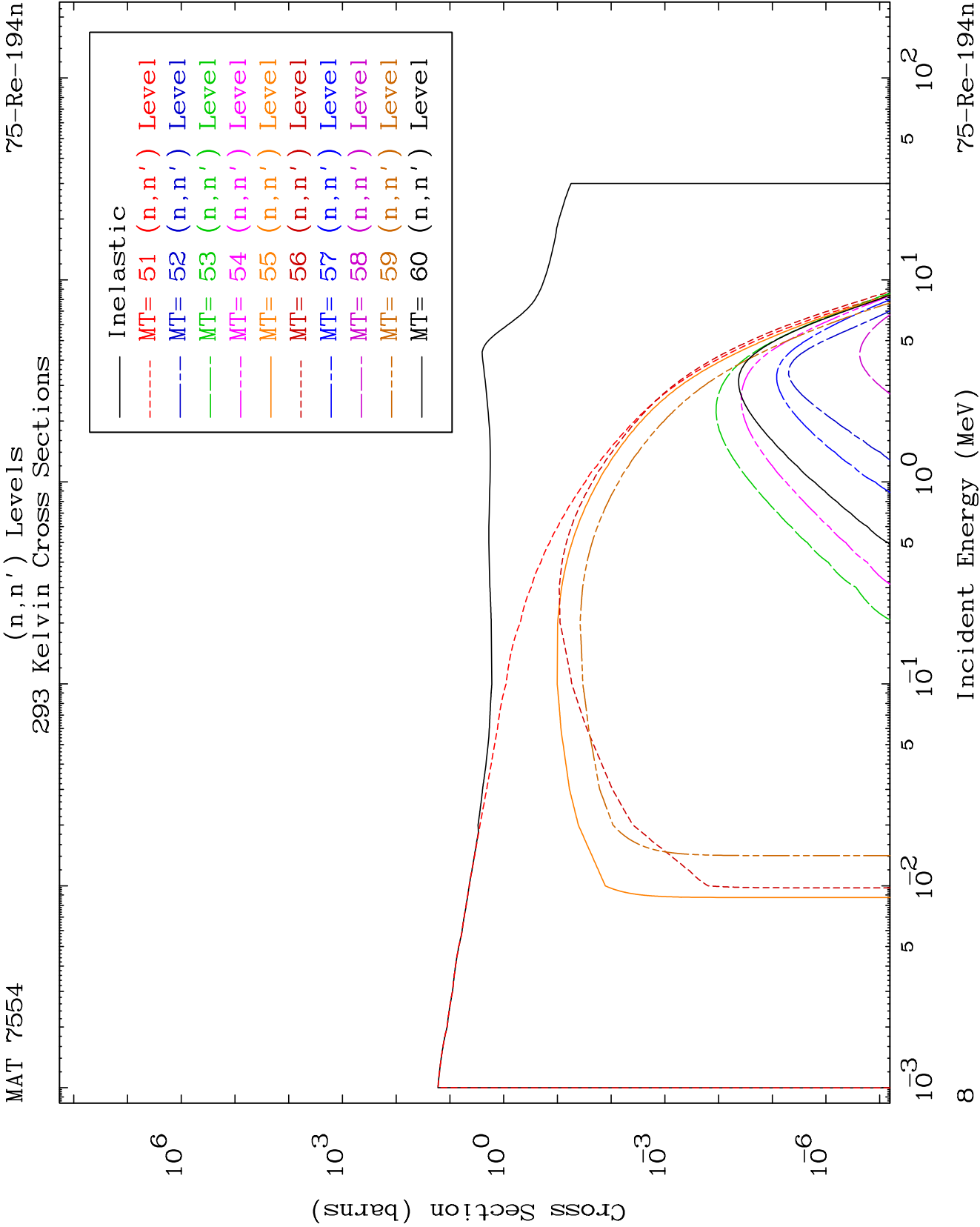










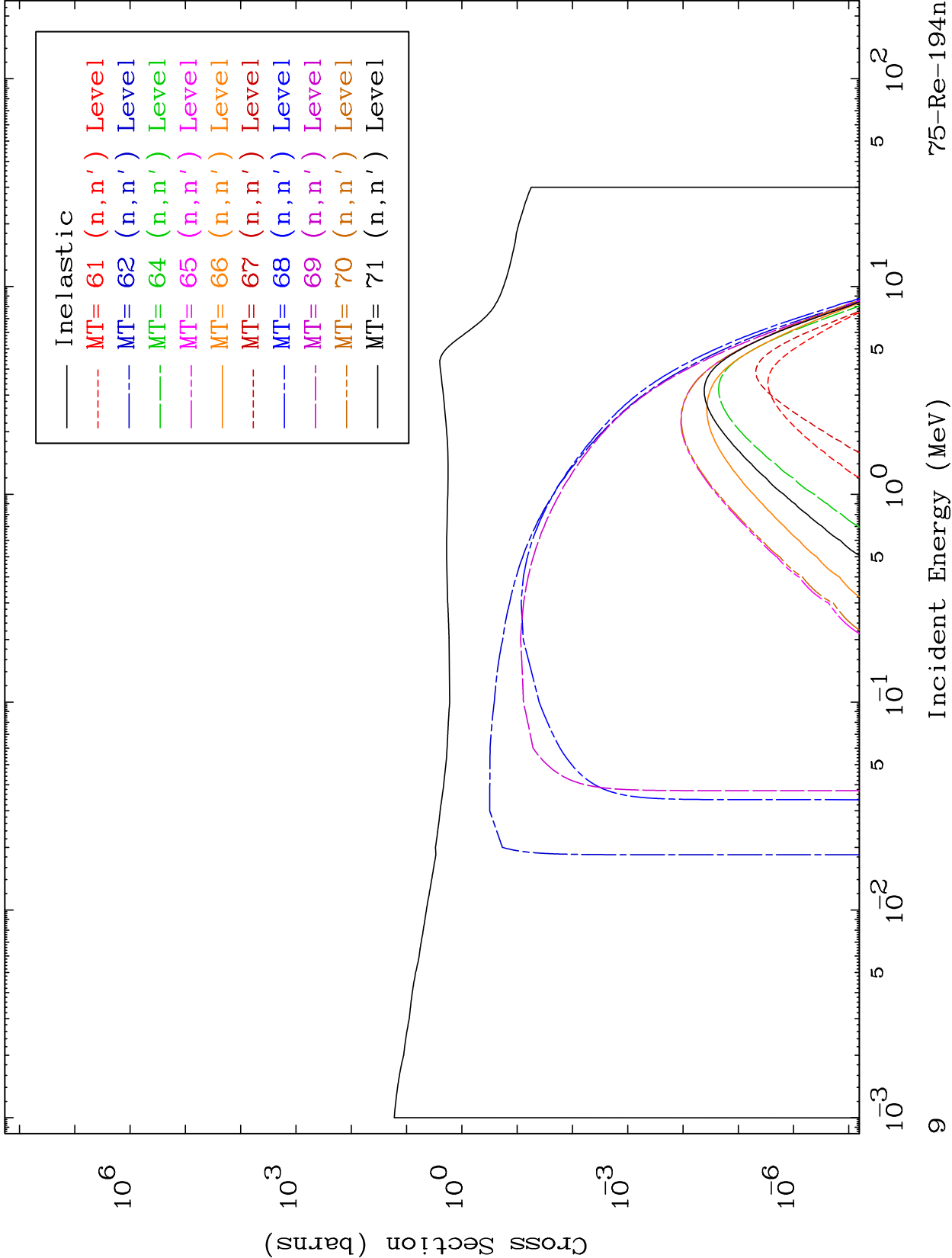


MAT 7554

(n,n') Levels

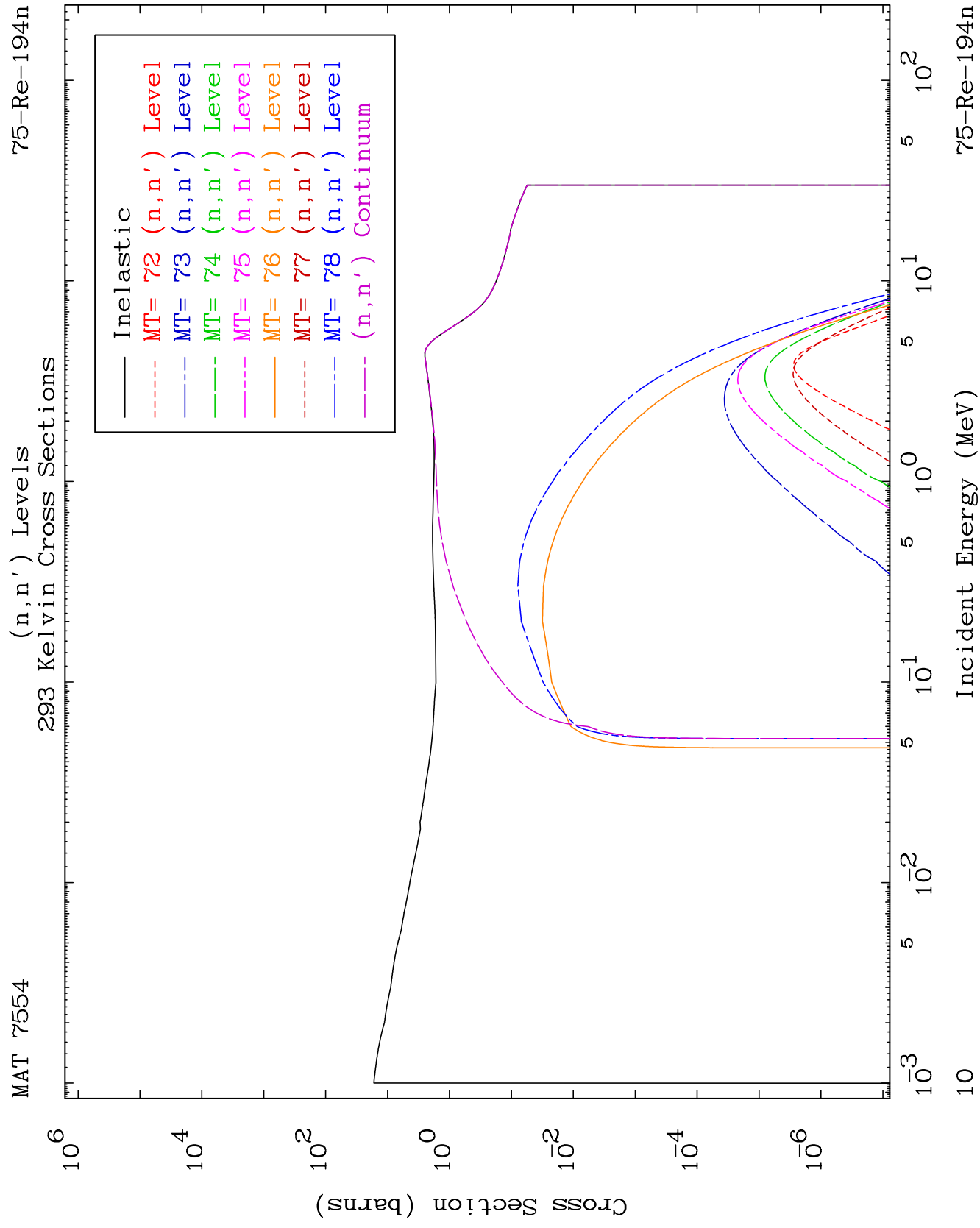
293 Kelvin Cross Sections

75-Re-194n



Incident Energy (MeV)

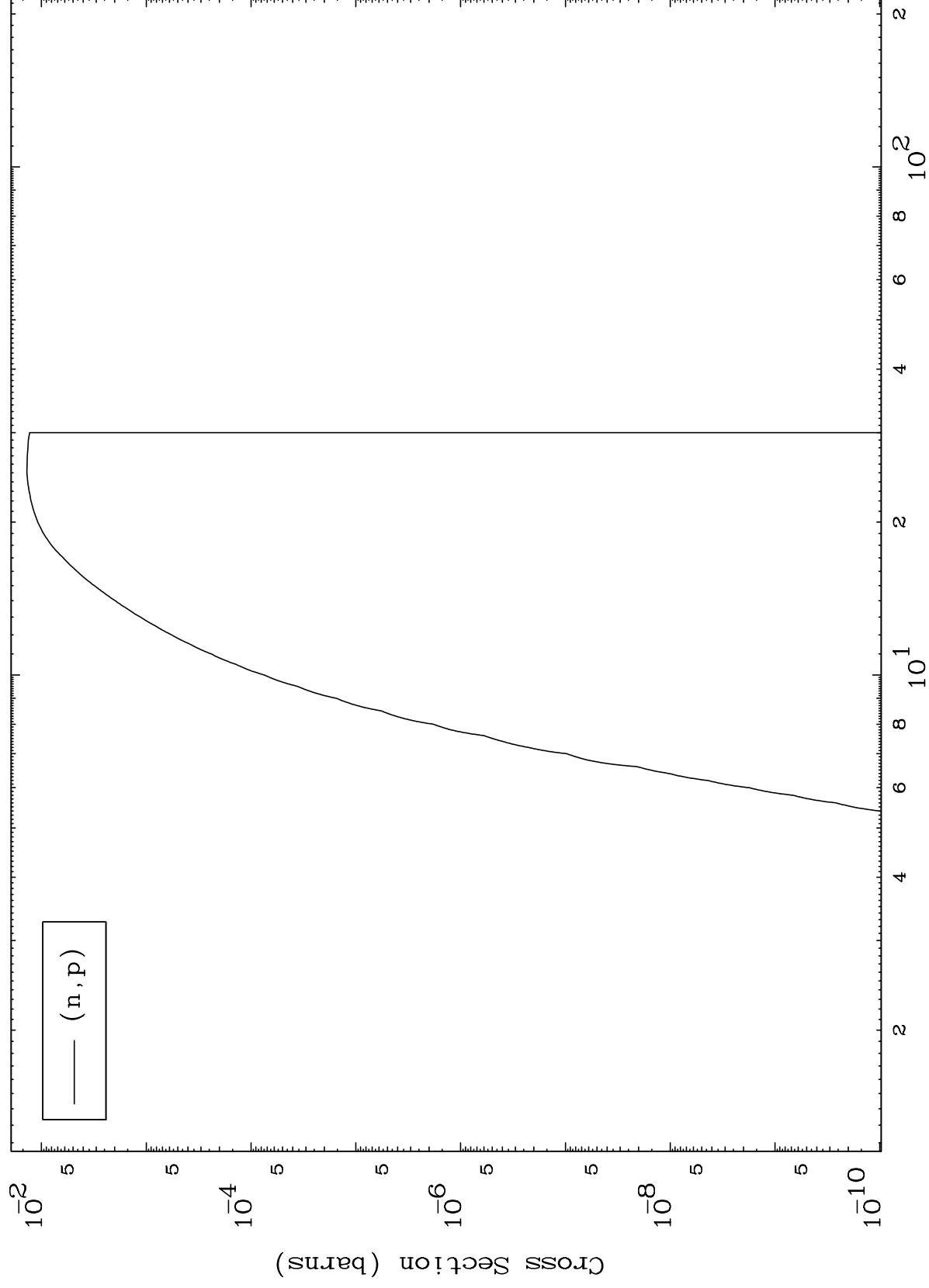
75-Re-194n



MAT 7554

(n,p) Levels
293 Kelvin Cross Sections

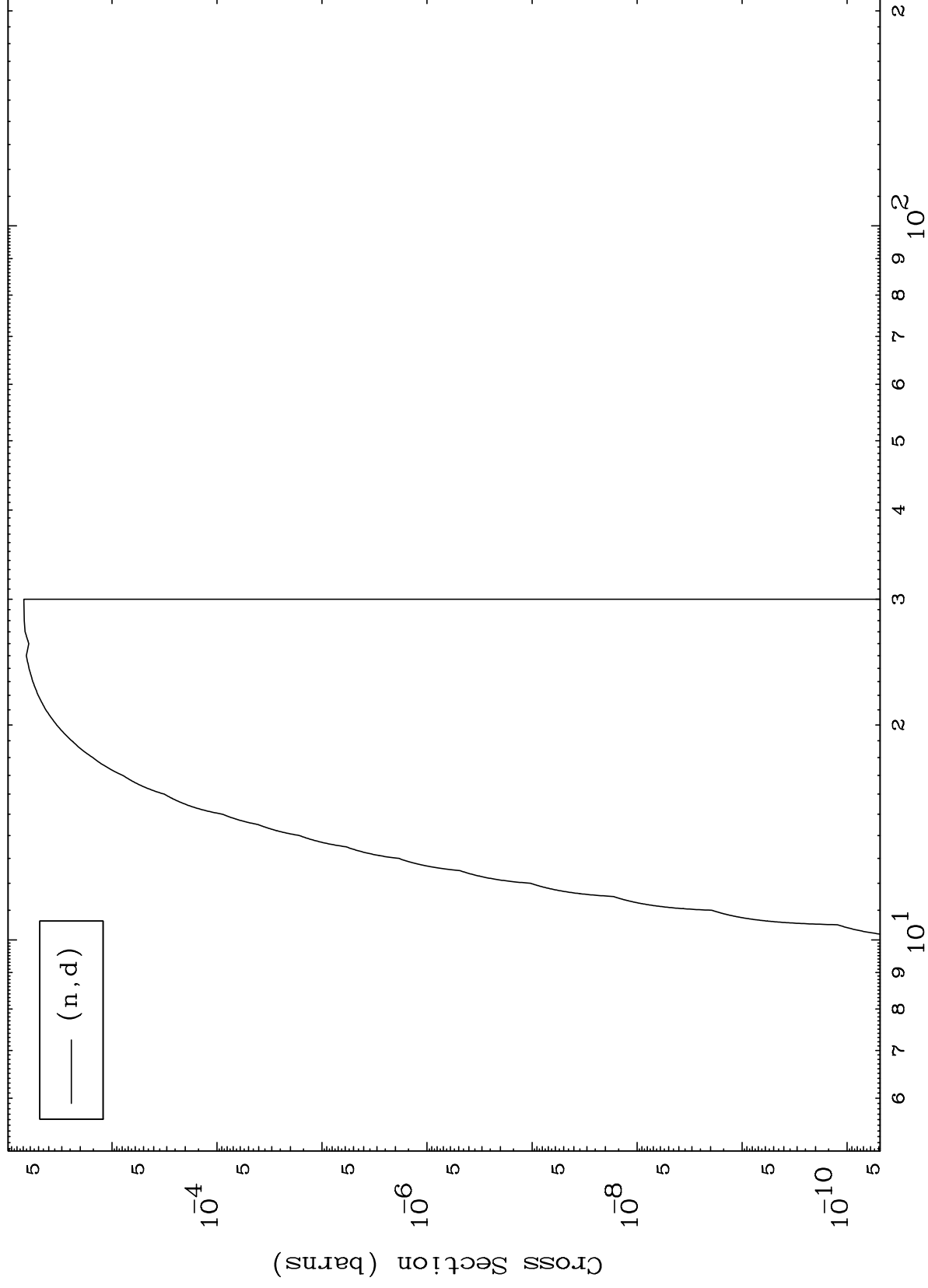
75-Re-194n



MAT 7554

(n,d) Levels
293 Kelvin Cross Sections

75-Re-194n



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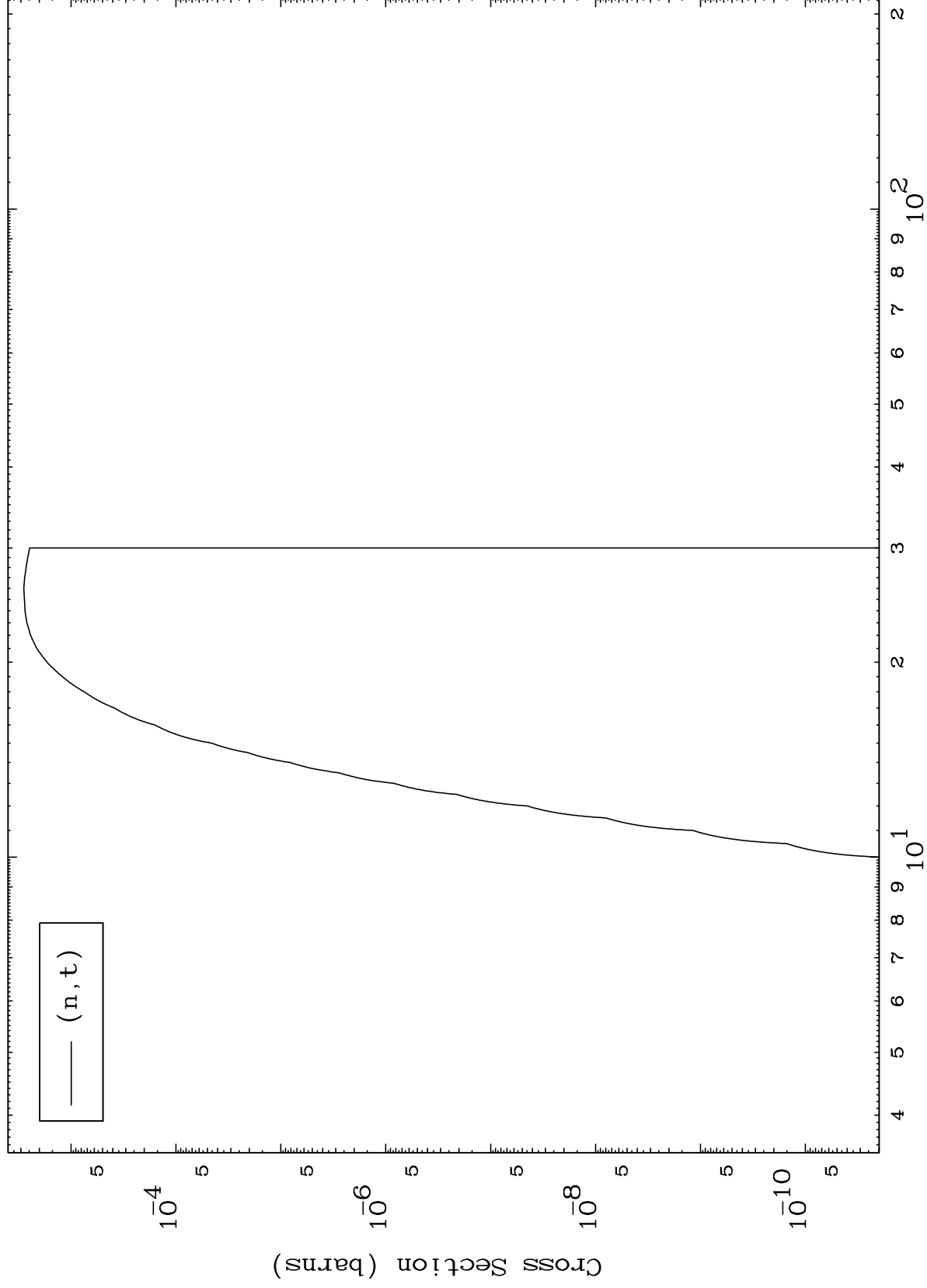
Incident Energy (MeV)

75-Re-194n

MAT 7554

75-Re-194n

(n,t) Levels
293 Kelvin Cross Sections



75-Re-194n

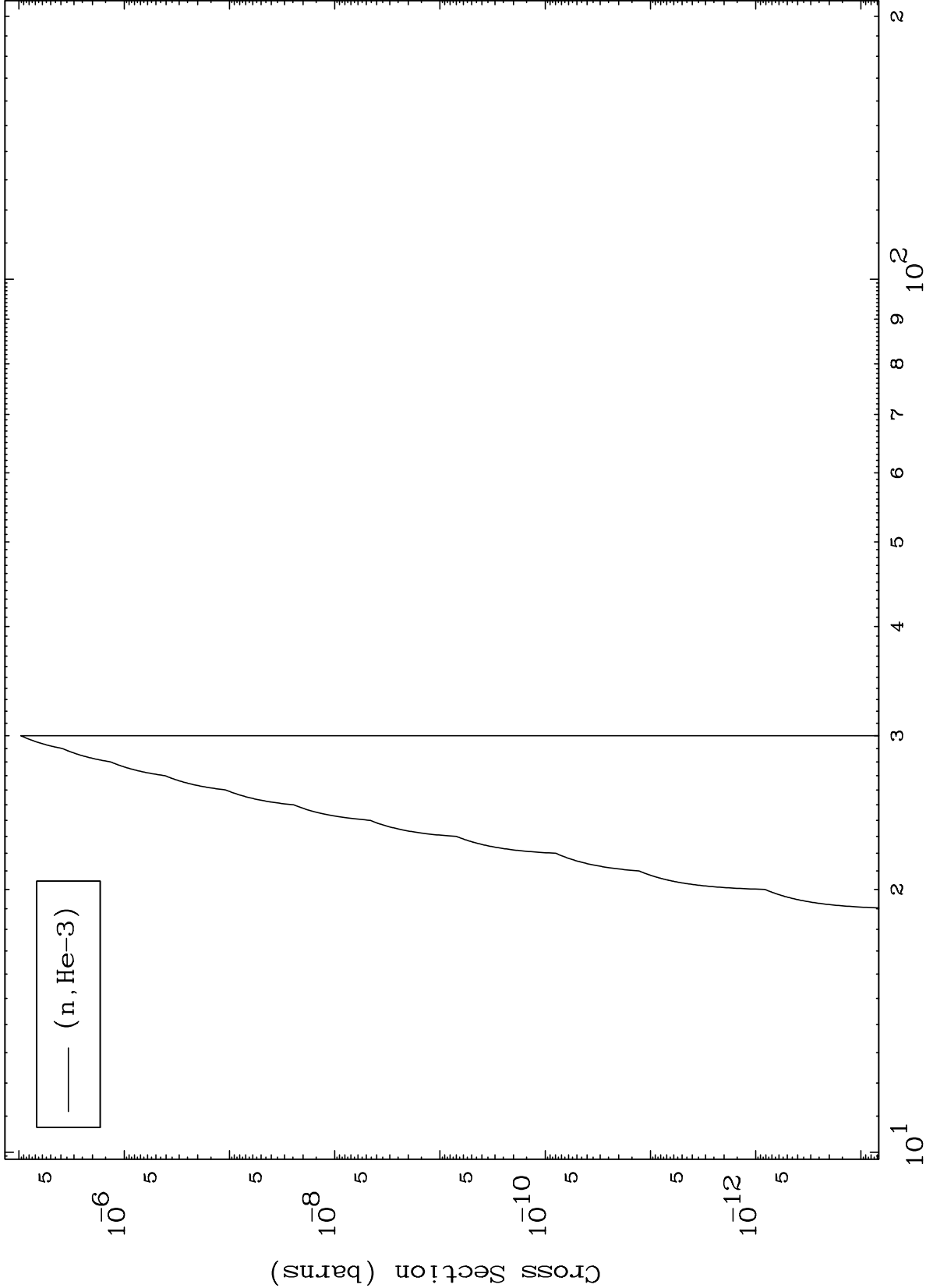
Incident Energy (MeV)

13

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(n,He3) Levels
293 Kelvin Cross Sections

⁷⁵Re-¹⁹⁴n



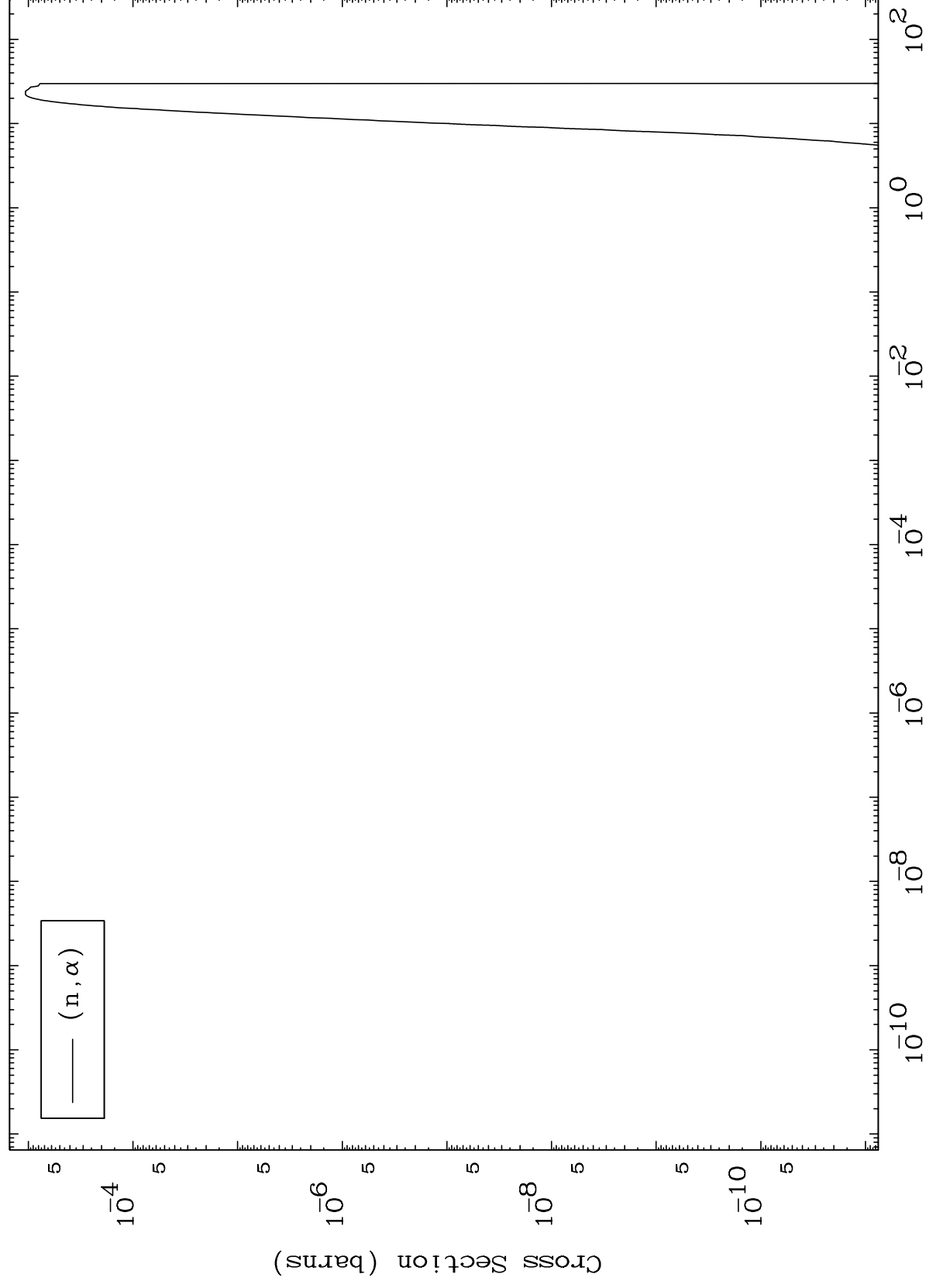
Incident Energy (MeV)

⁷⁵Re-¹⁹⁴n

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(n, α) Levels
293 Kelvin Cross Sections

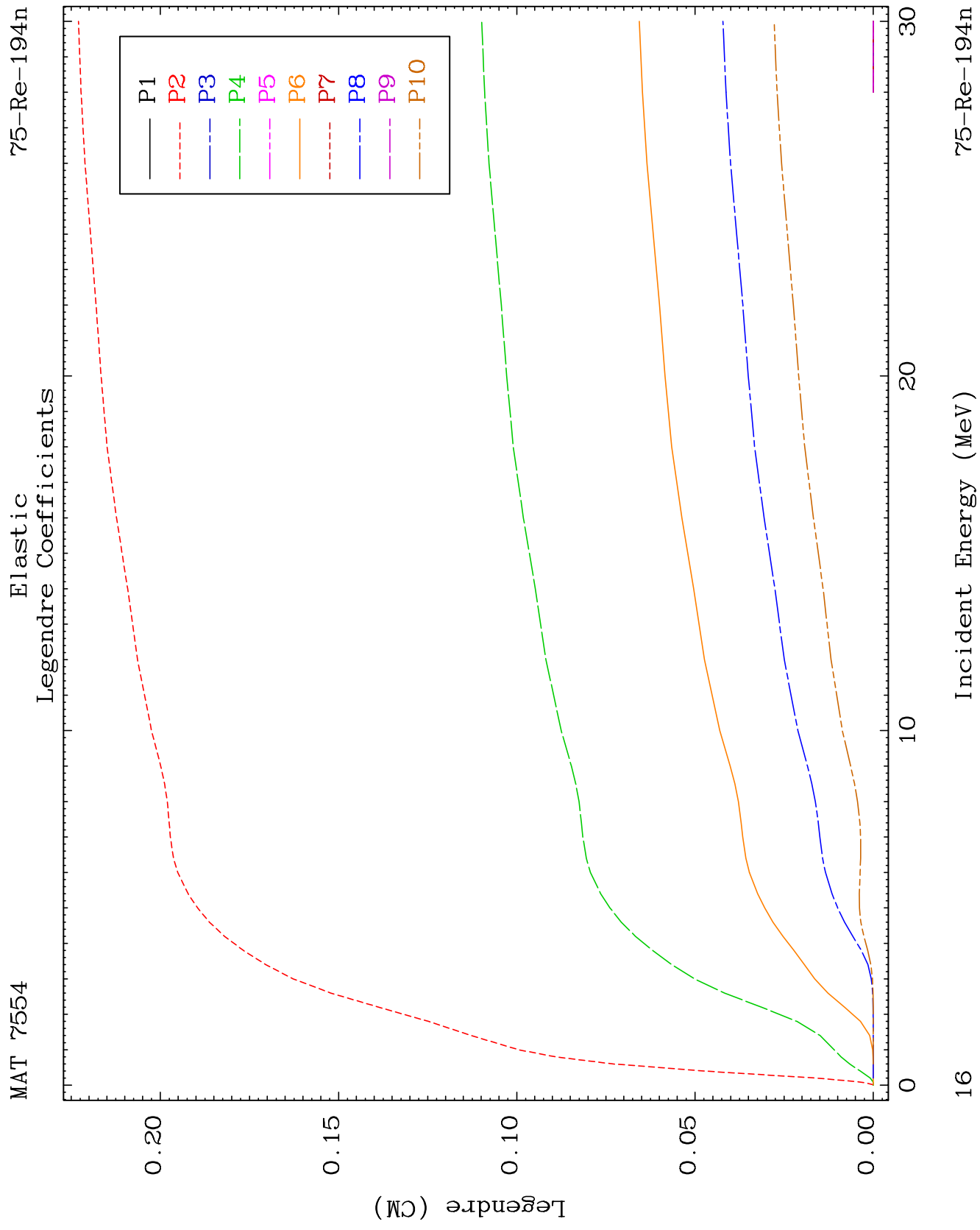
⁷⁵Re-194n

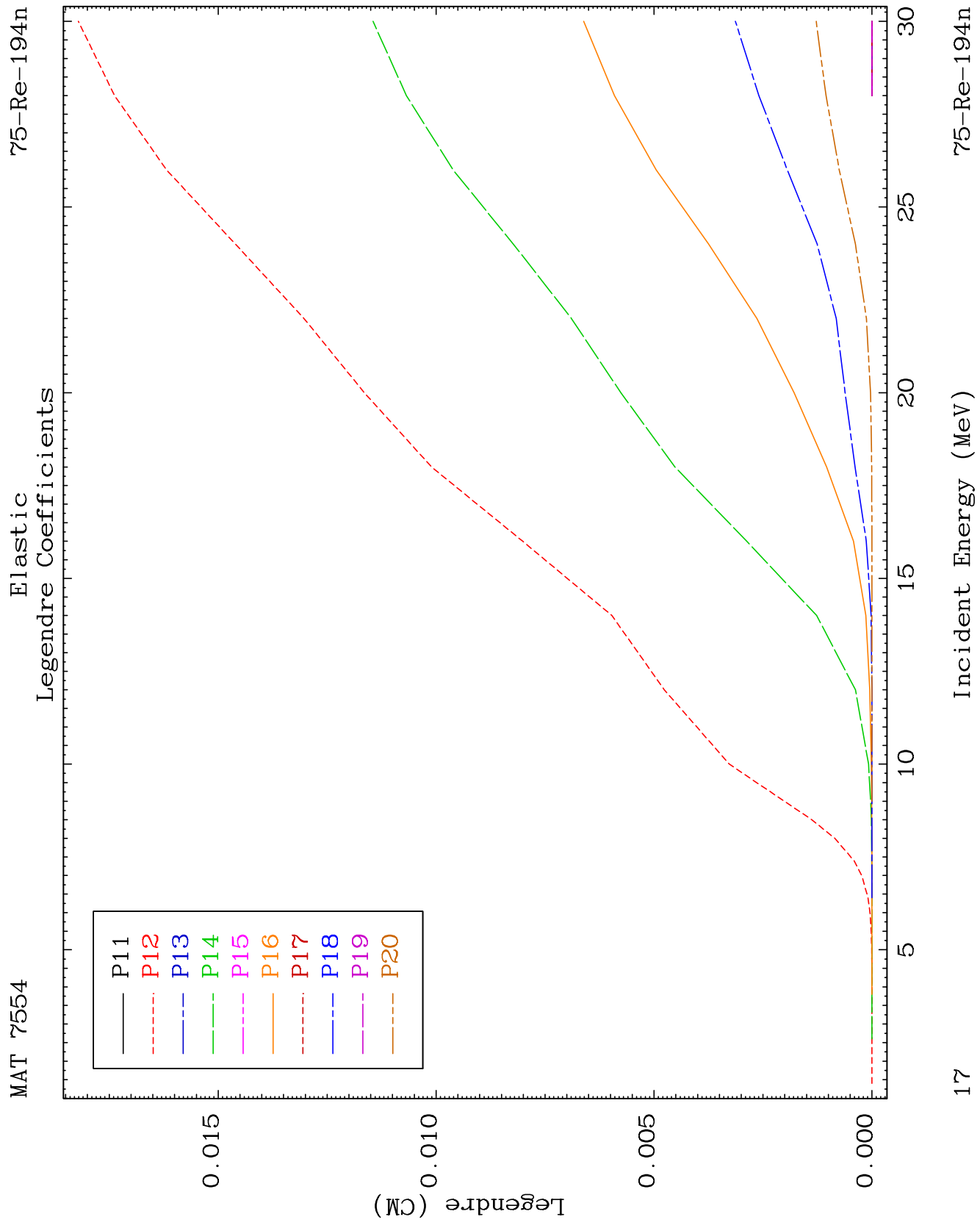


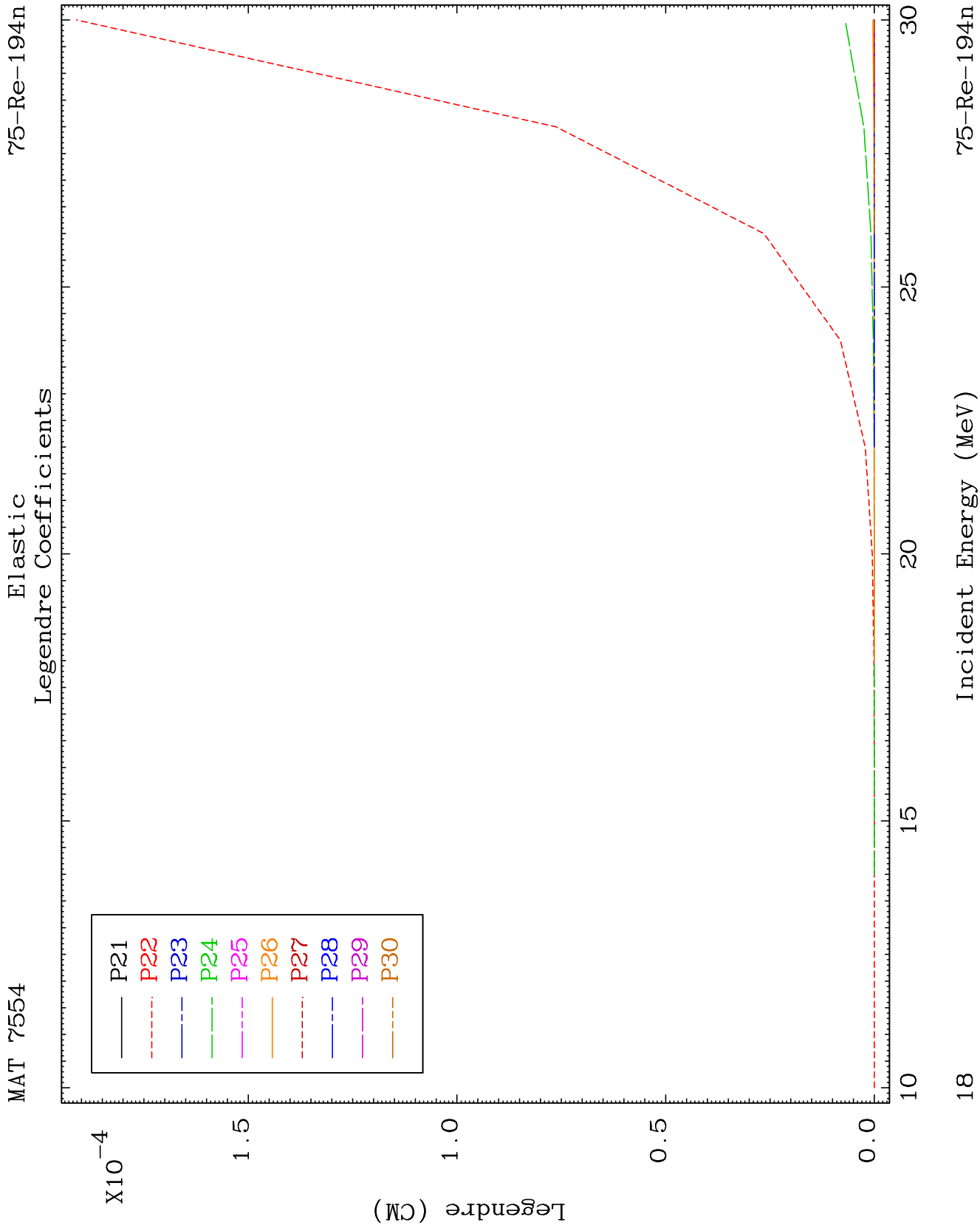
15

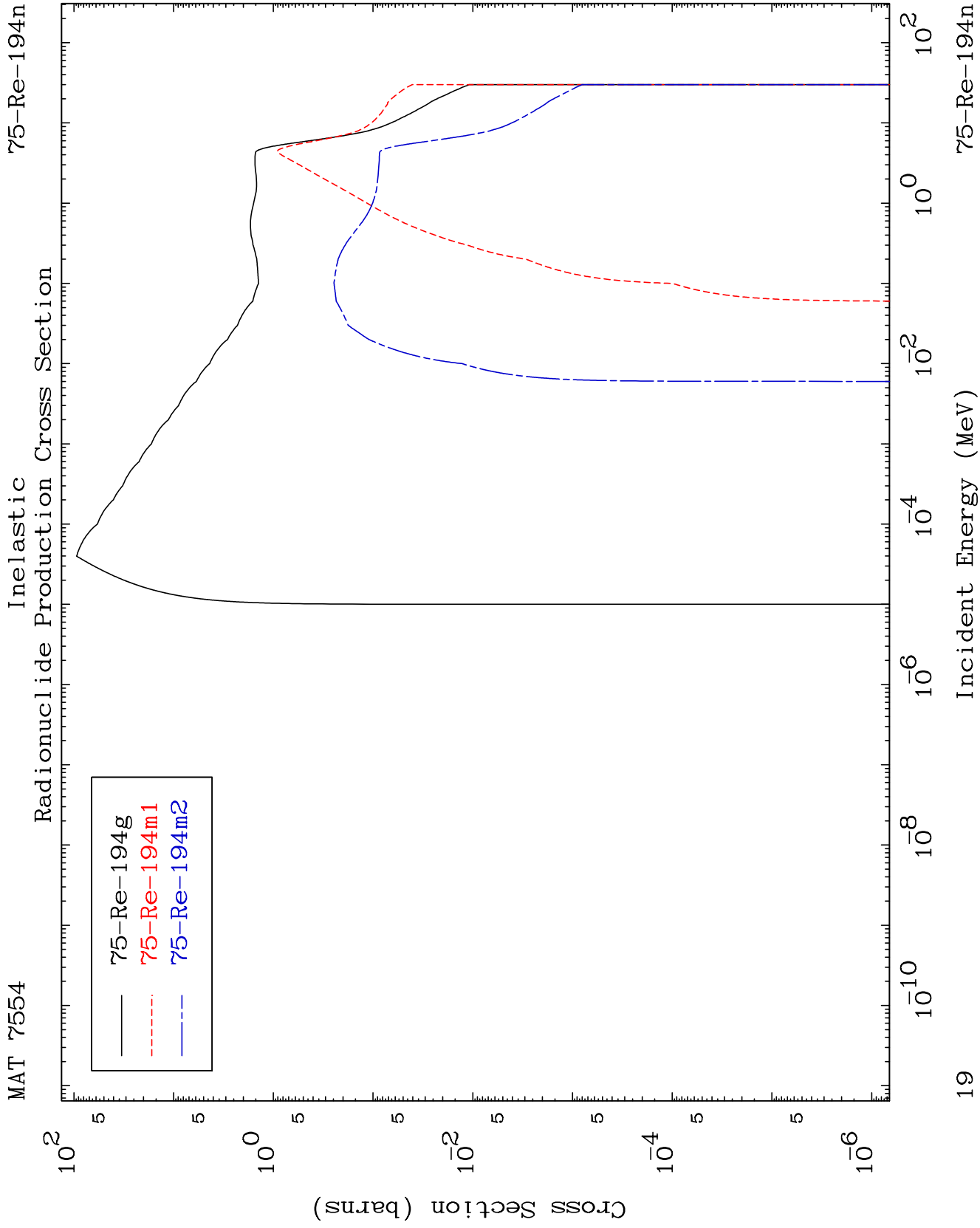
Incident Energy (MeV)

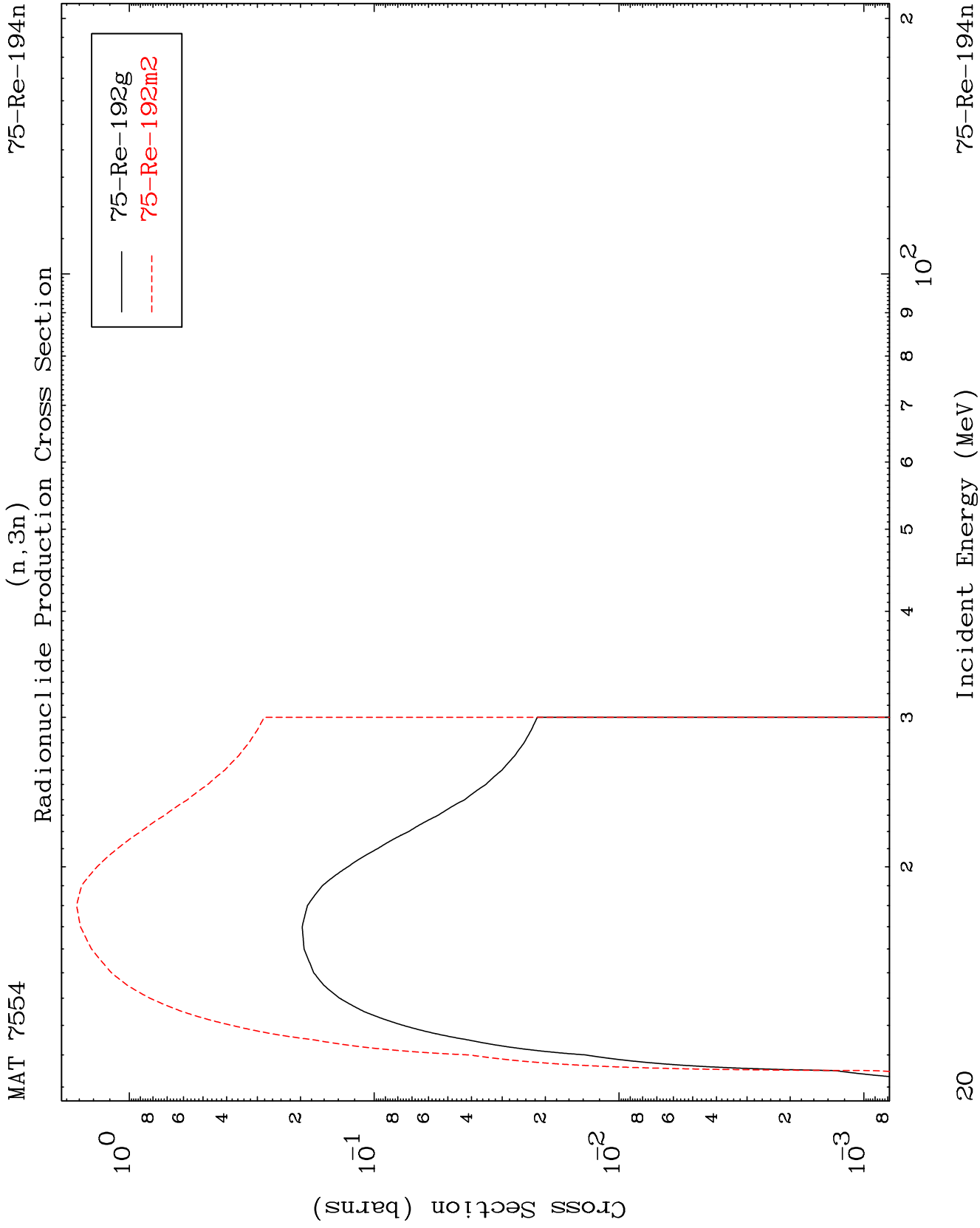
⁷⁵Re-194n









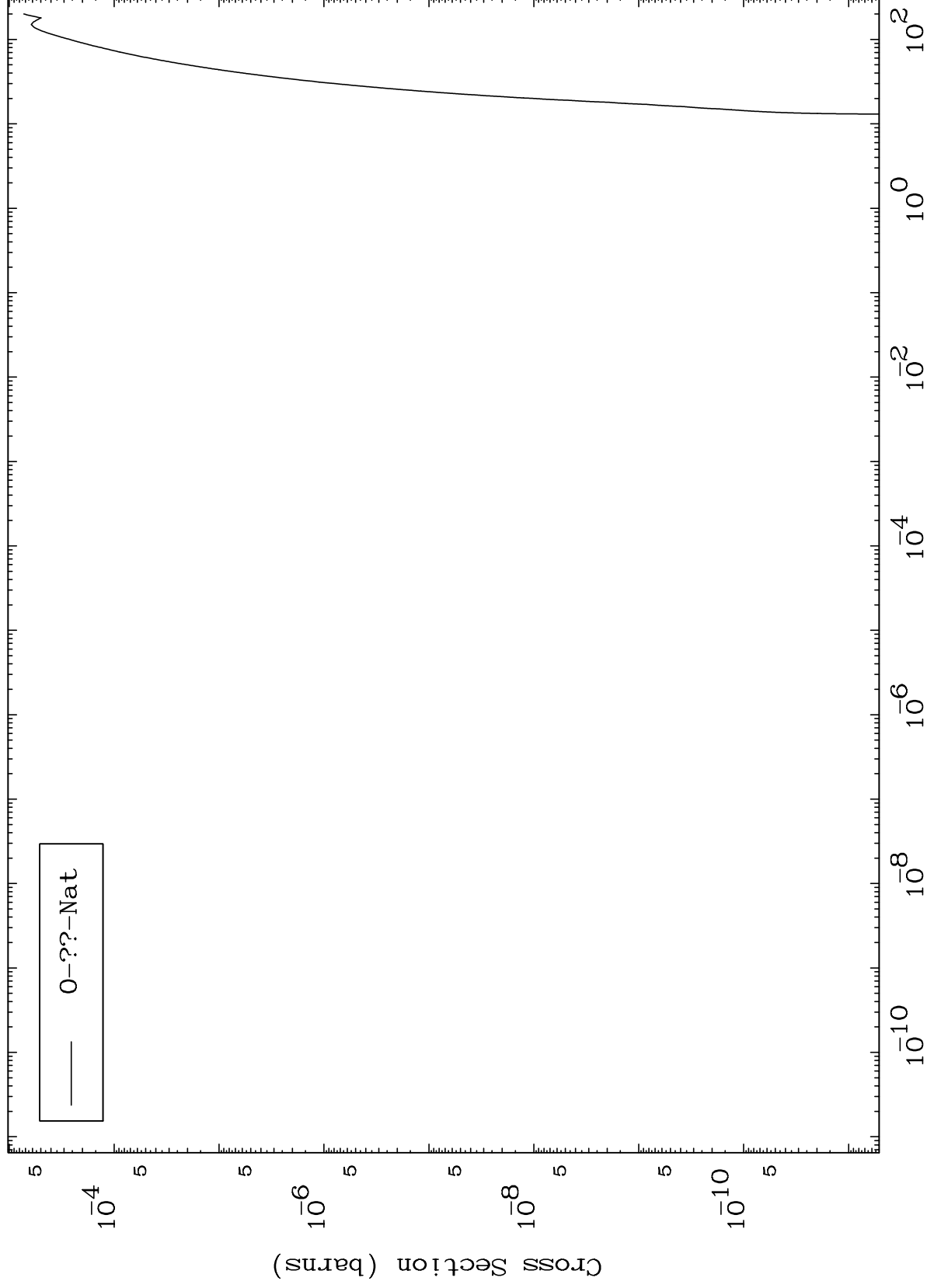


MAT 7554

Fission

⁷⁵Re-¹⁹⁴n

Radionuclide Production Cross Section



0-??-Nat

Radionuclide Production Cross Section

