

Program EVALPLOT
(Version 2021-1)

by

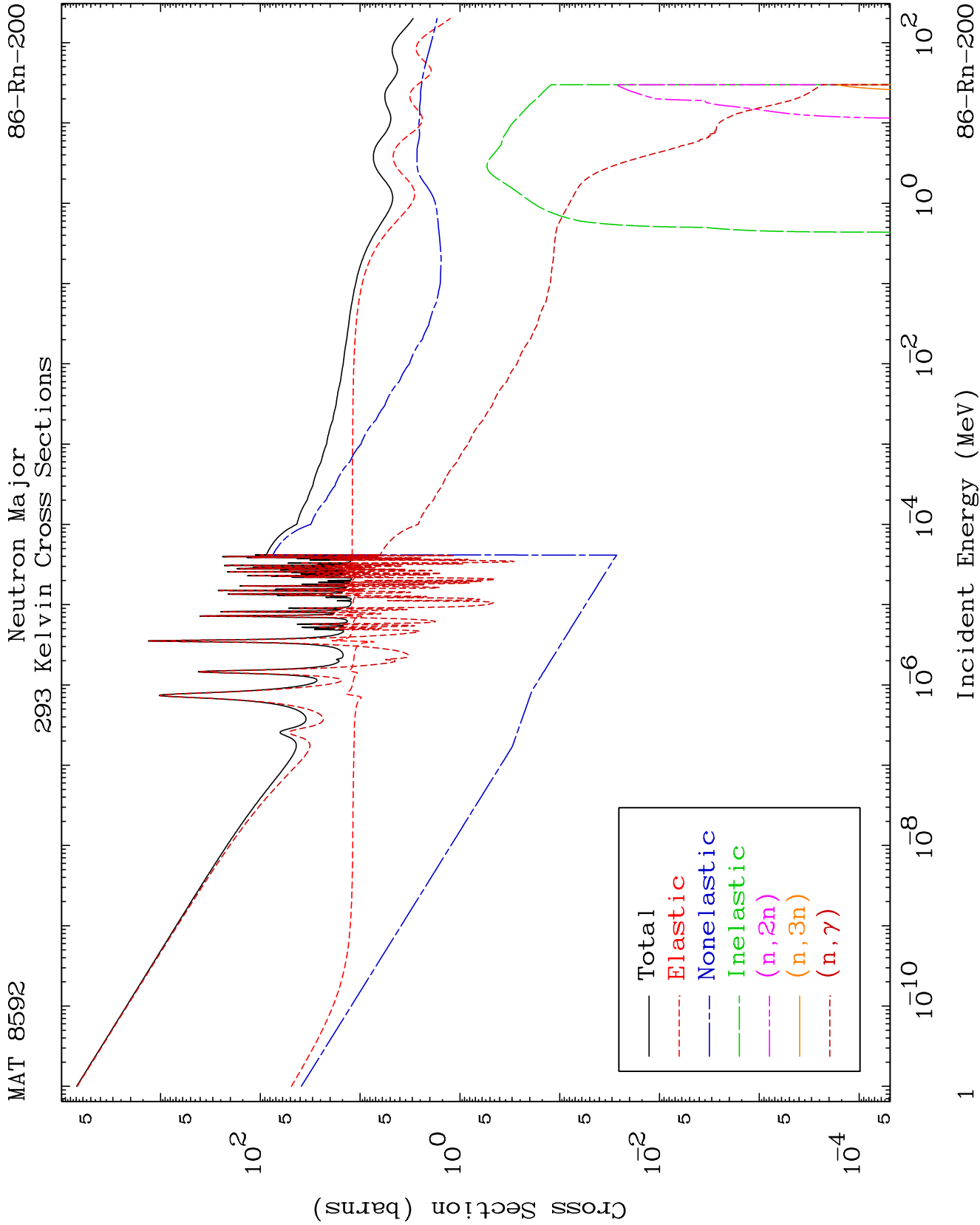
Dermott E. Cullen
(Present Contact Information)

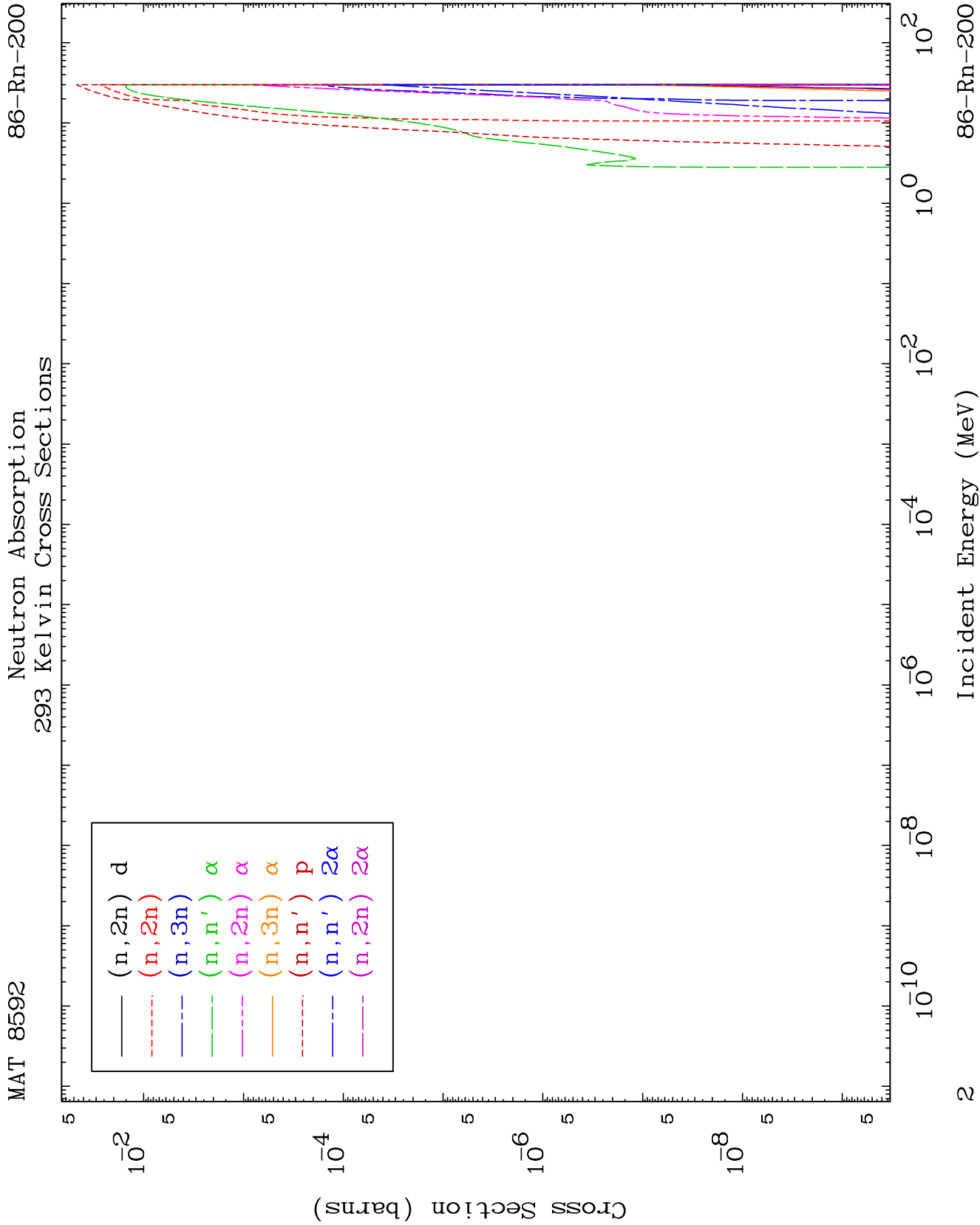
Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

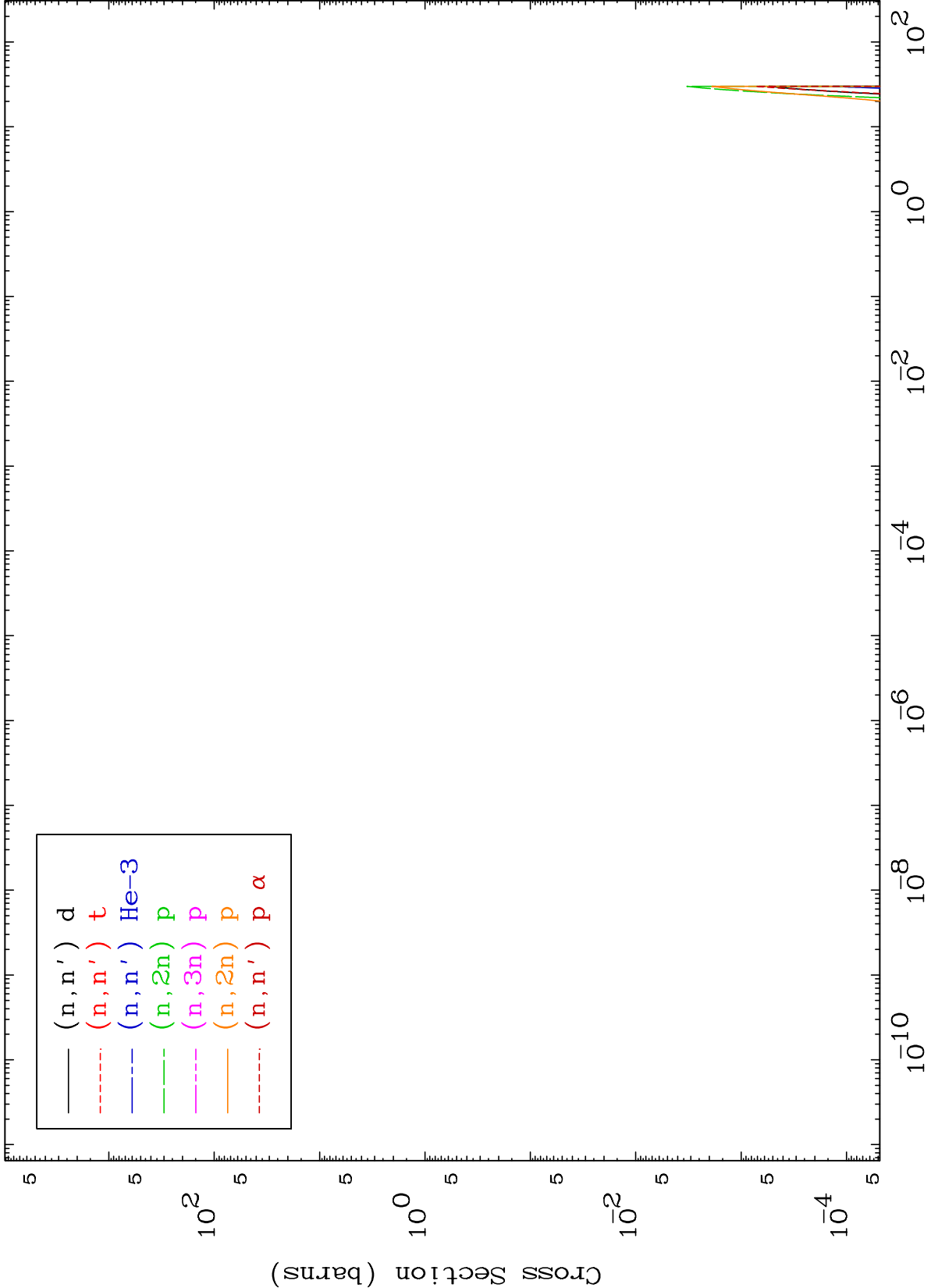


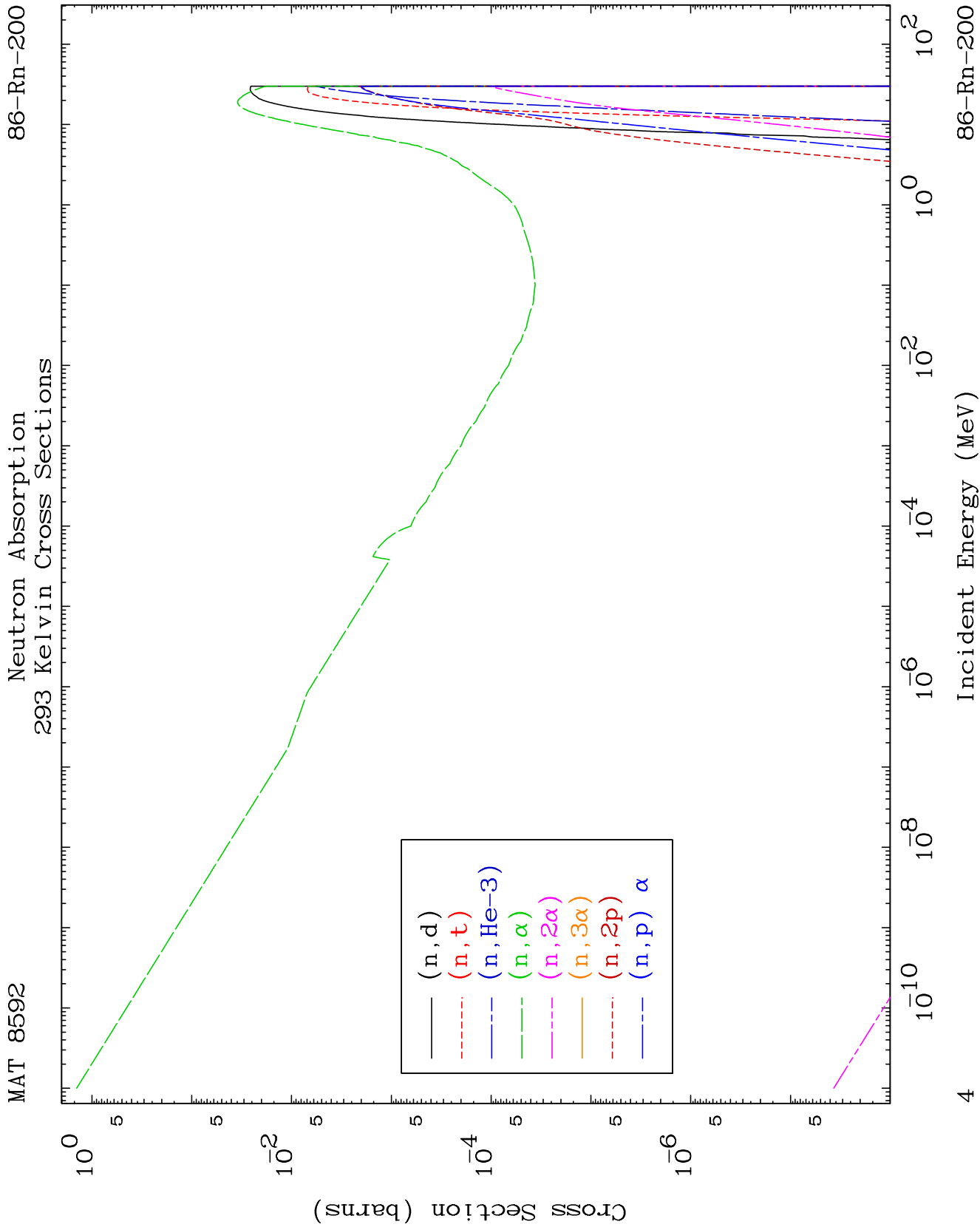


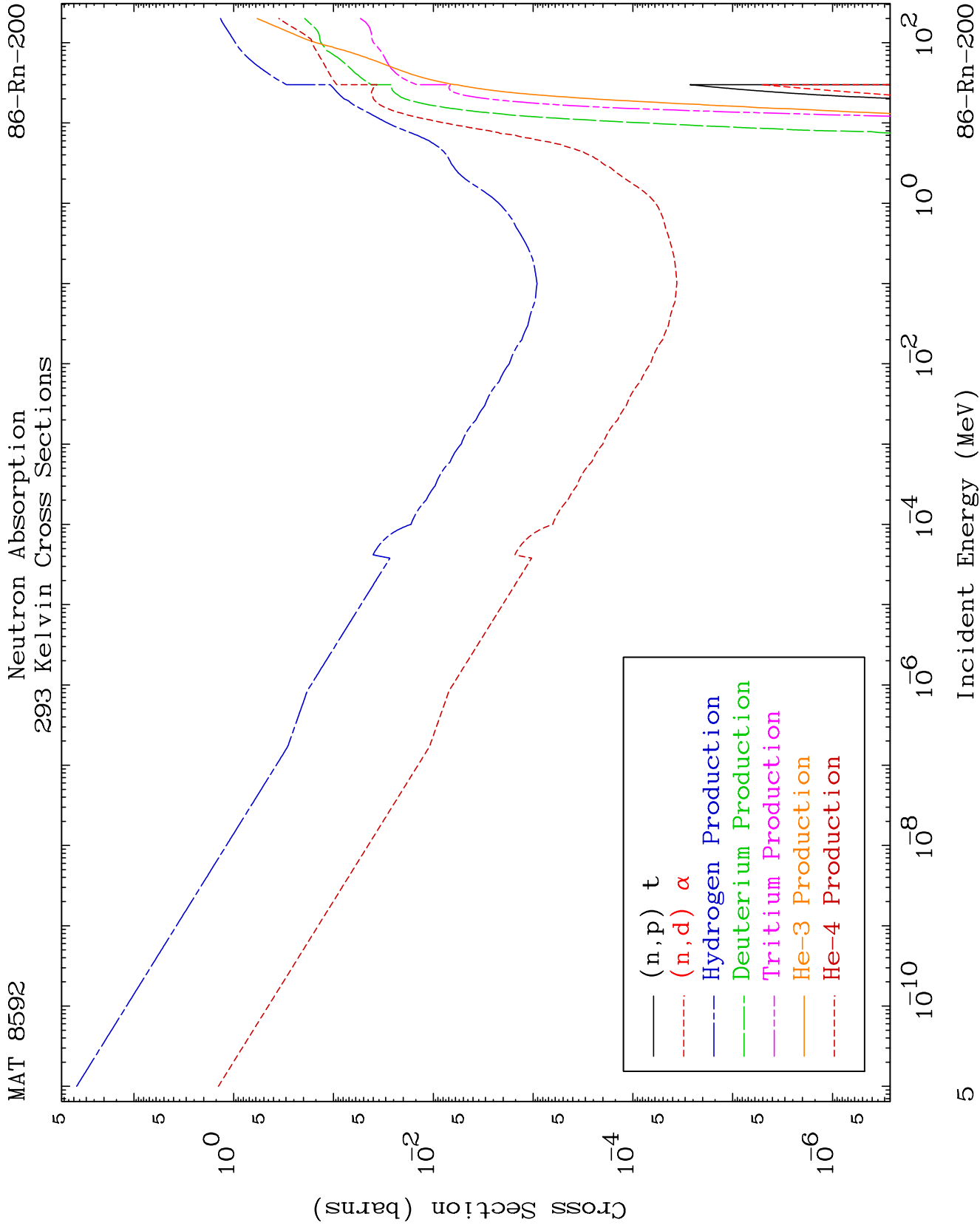
MAT 8592

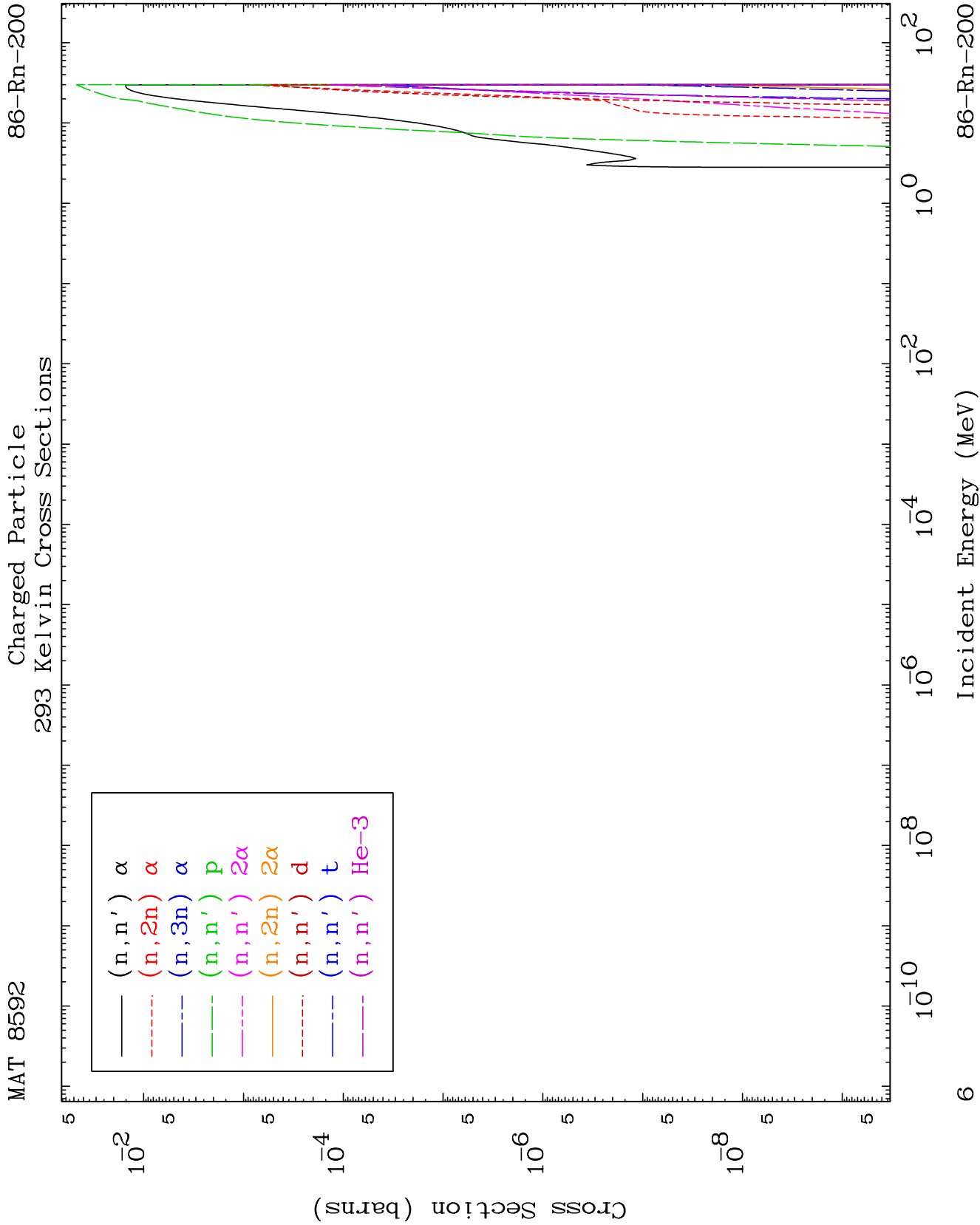
Neutron Absorption
293 Kelvin Cross Sections

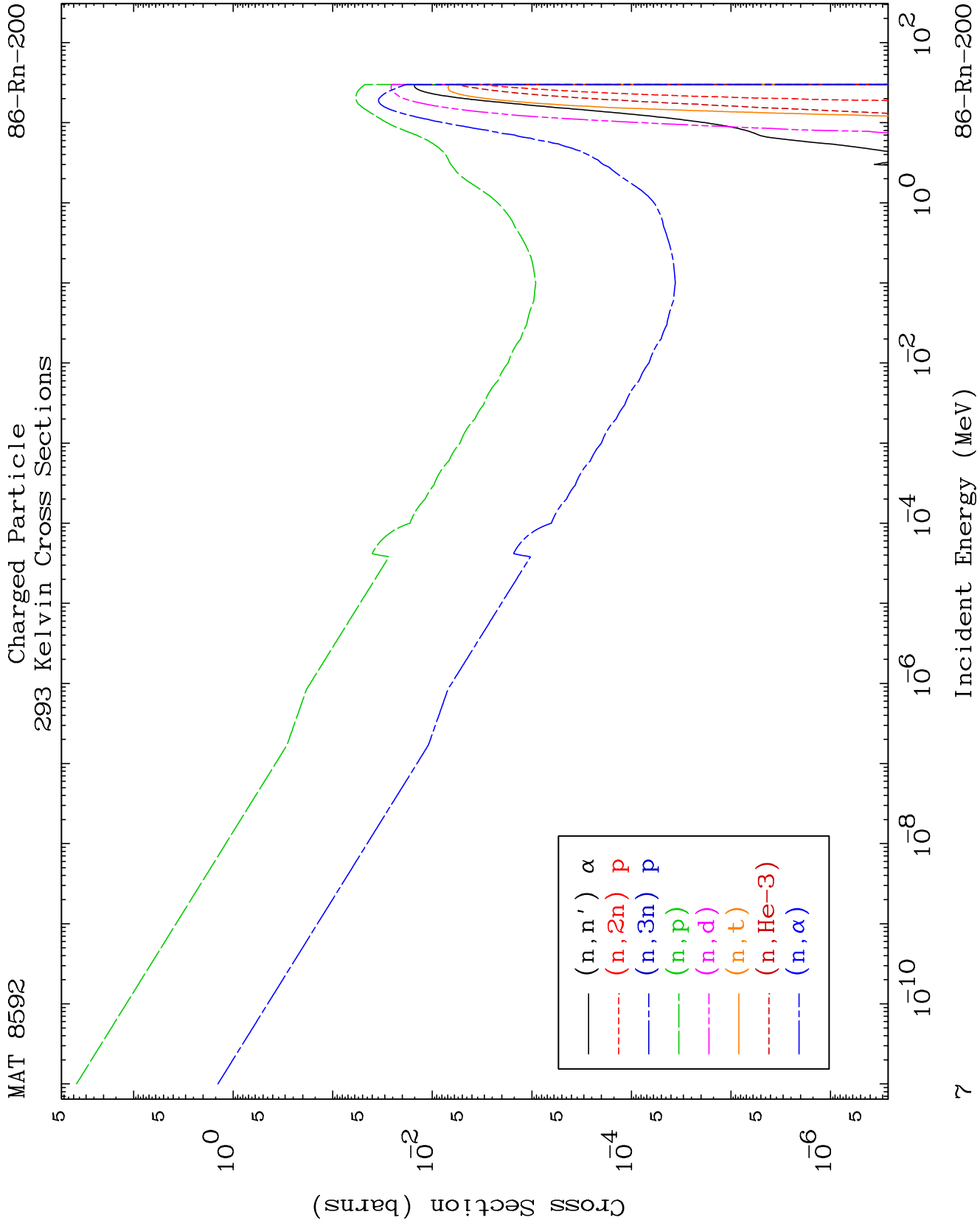
86-Rn-200

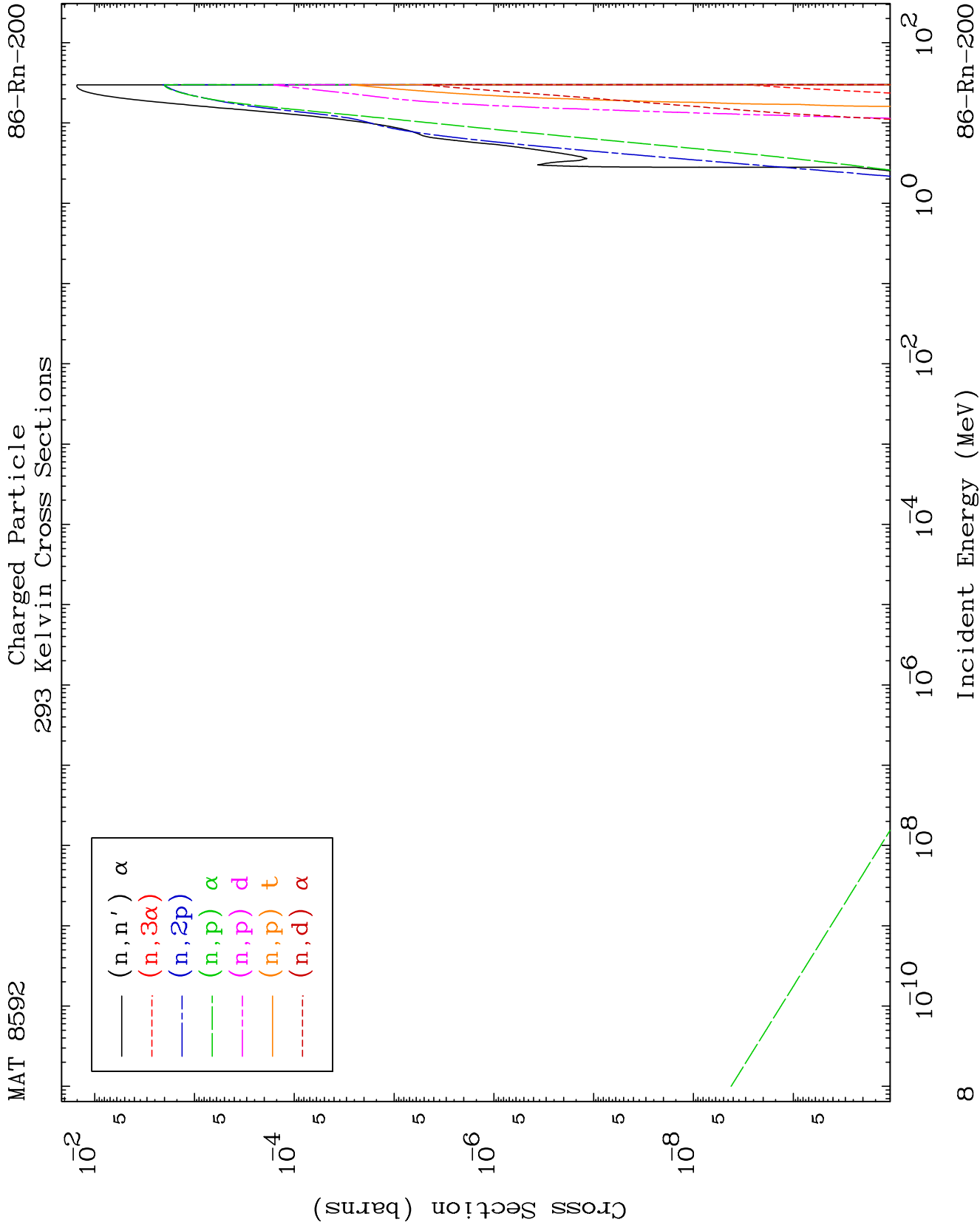


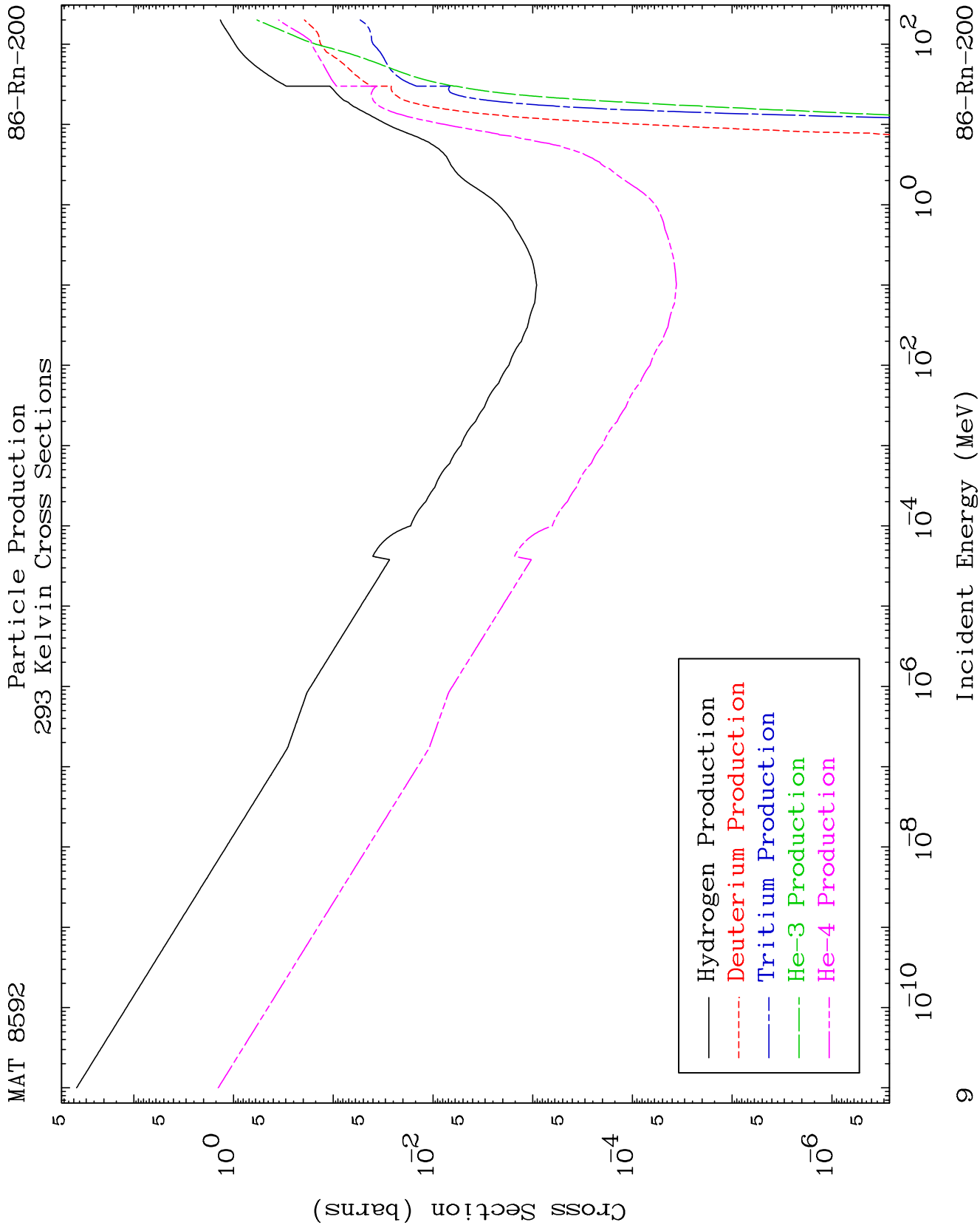








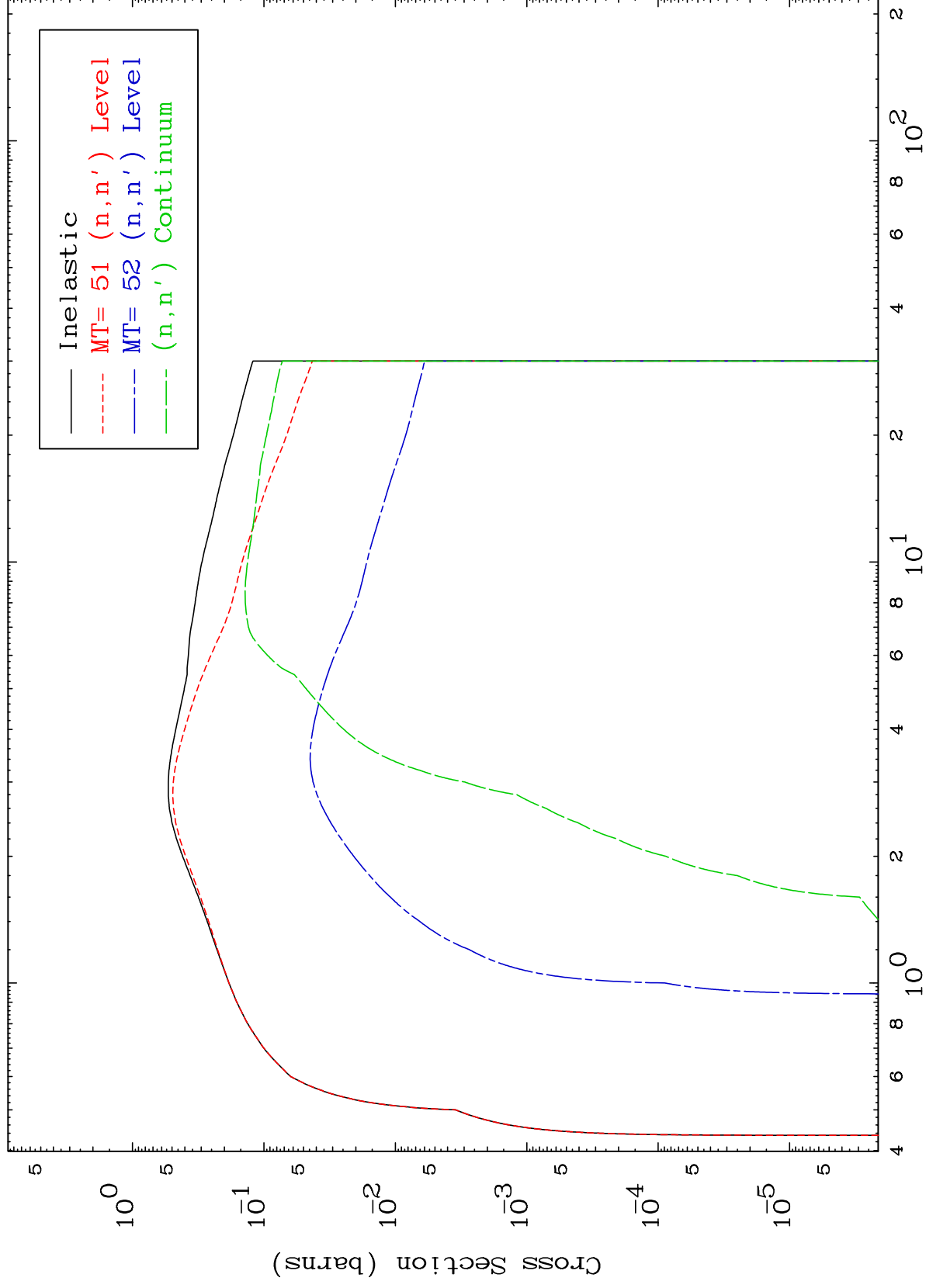




MAT 8592

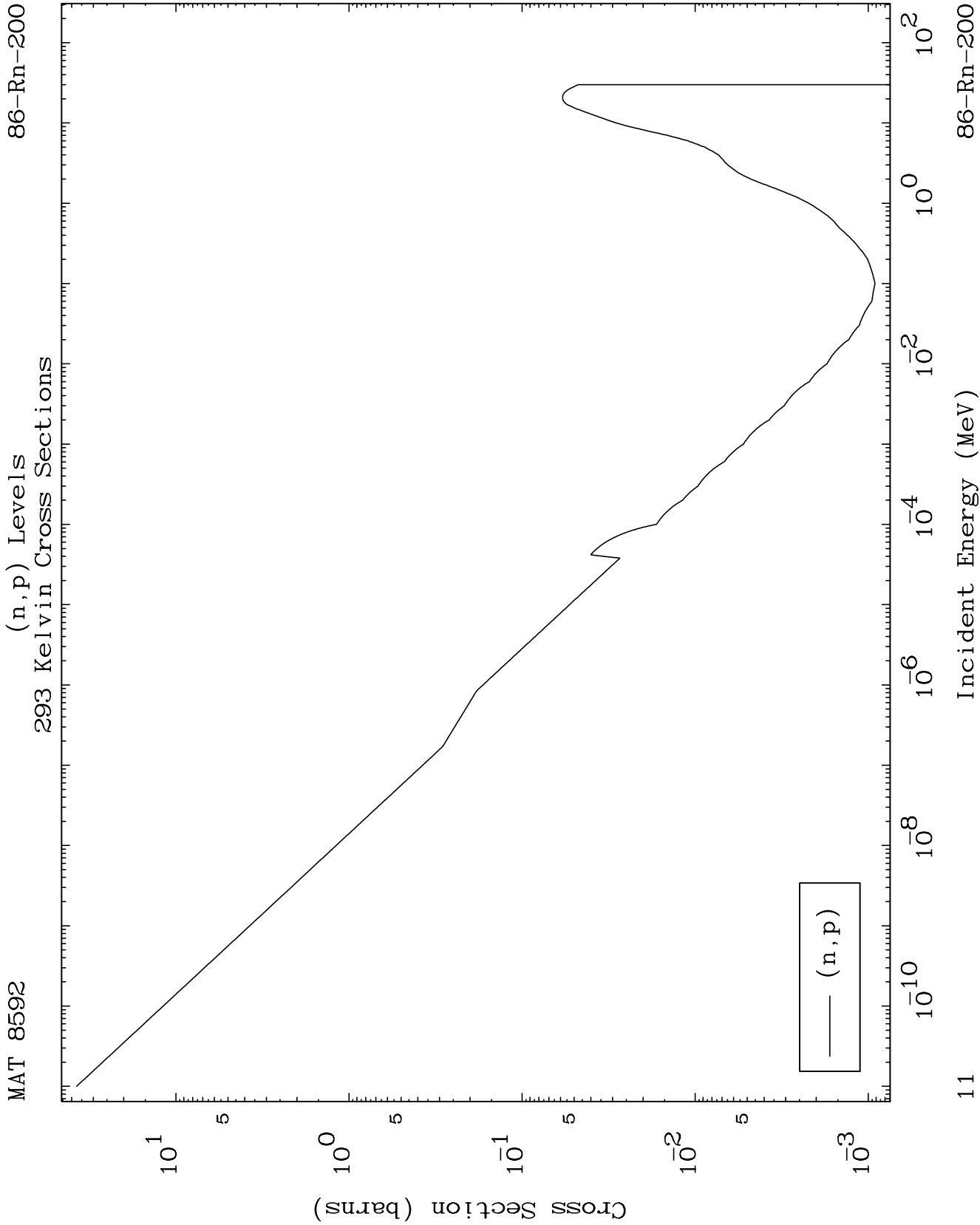
(n,n') Levels
293 Kelvin Cross Sections

86-Rn-200



Incident Energy (MeV)

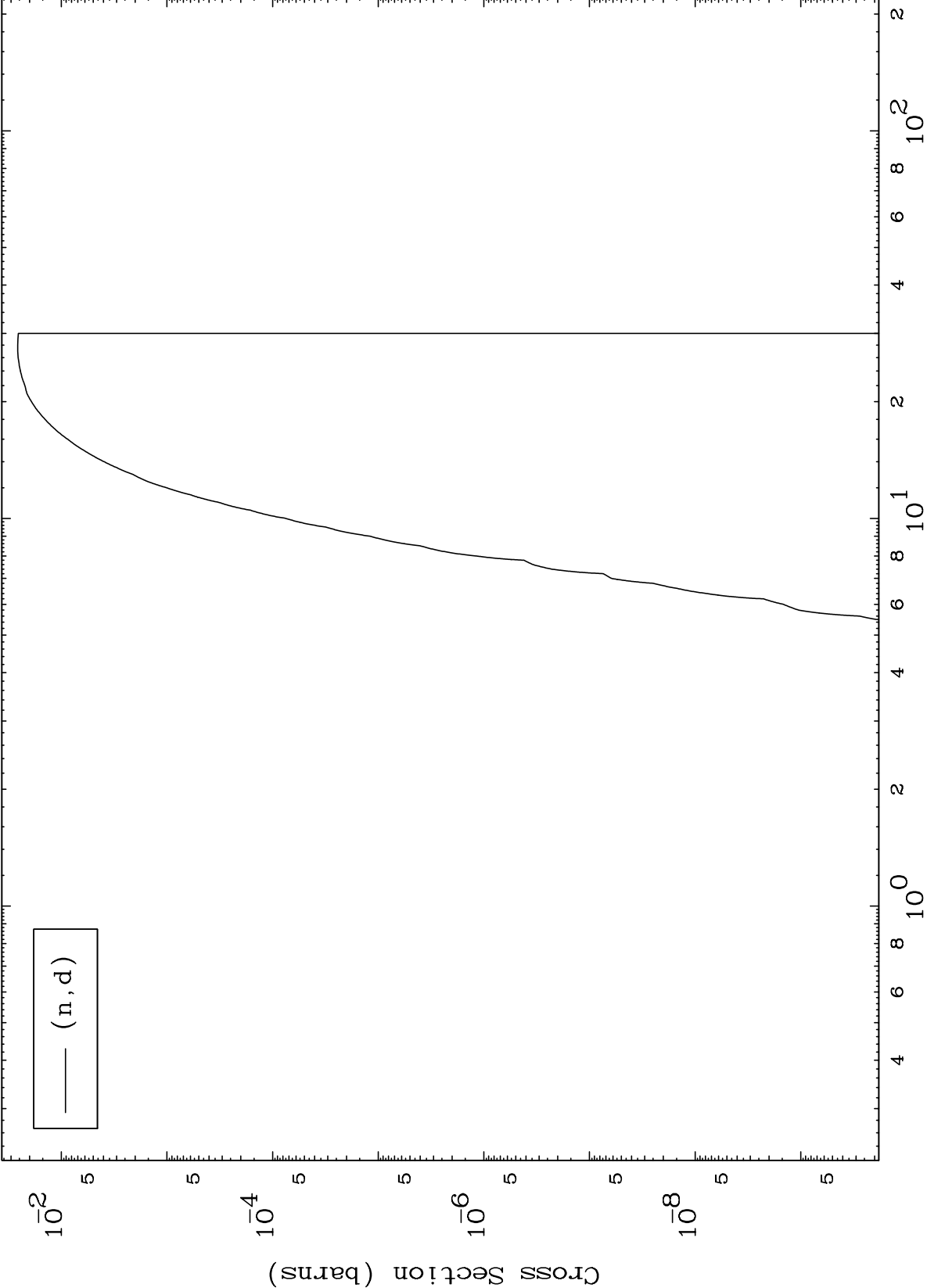
86-Rn-200



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(n,d) Levels
293 Kelvin Cross Sections

86-Rn-200



12

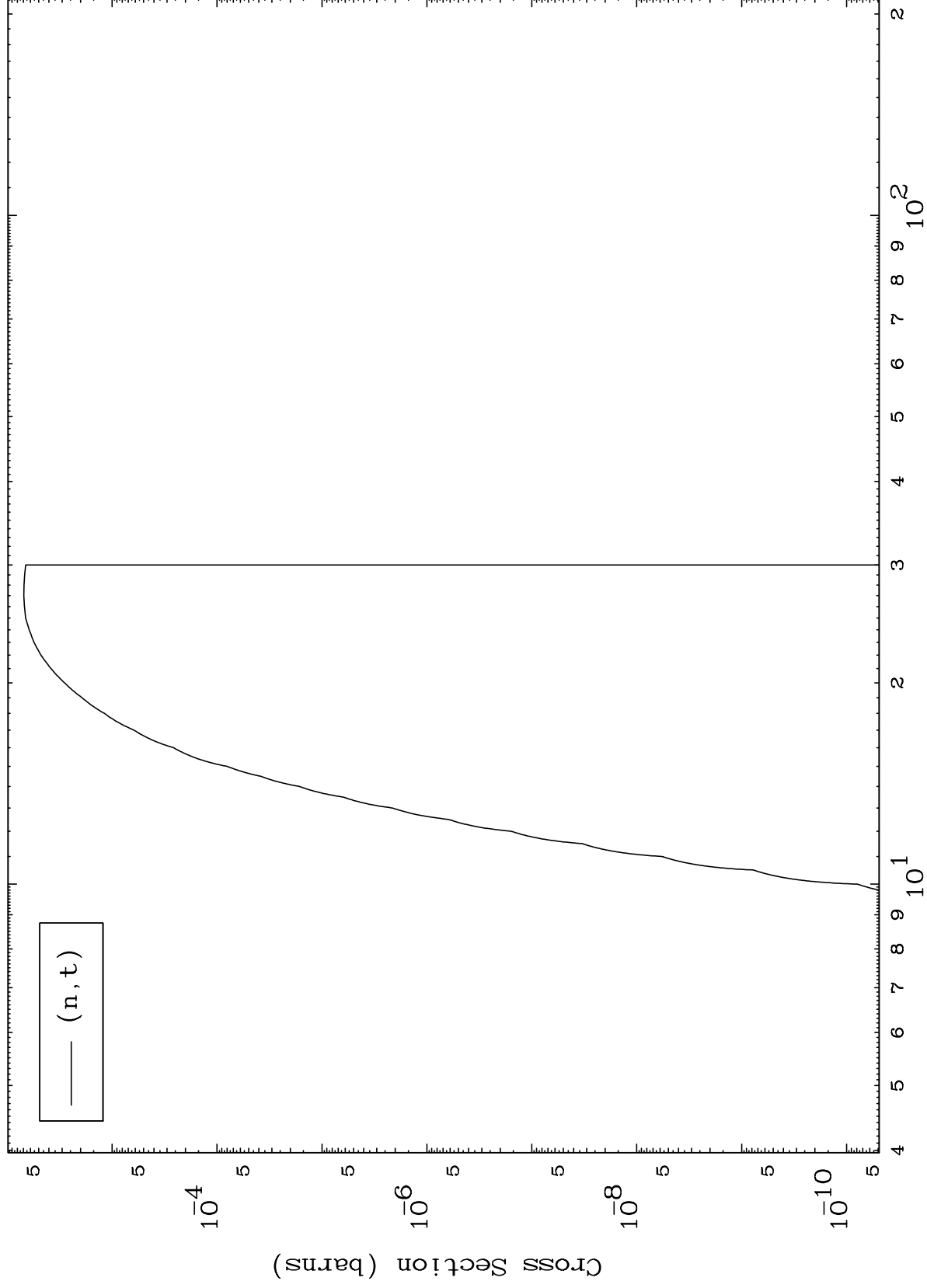
Incident Energy (MeV)

86-Rn-200

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86-Rn-200

(n,t) Levels
293 Kelvin Cross Sections



86-Rn-200

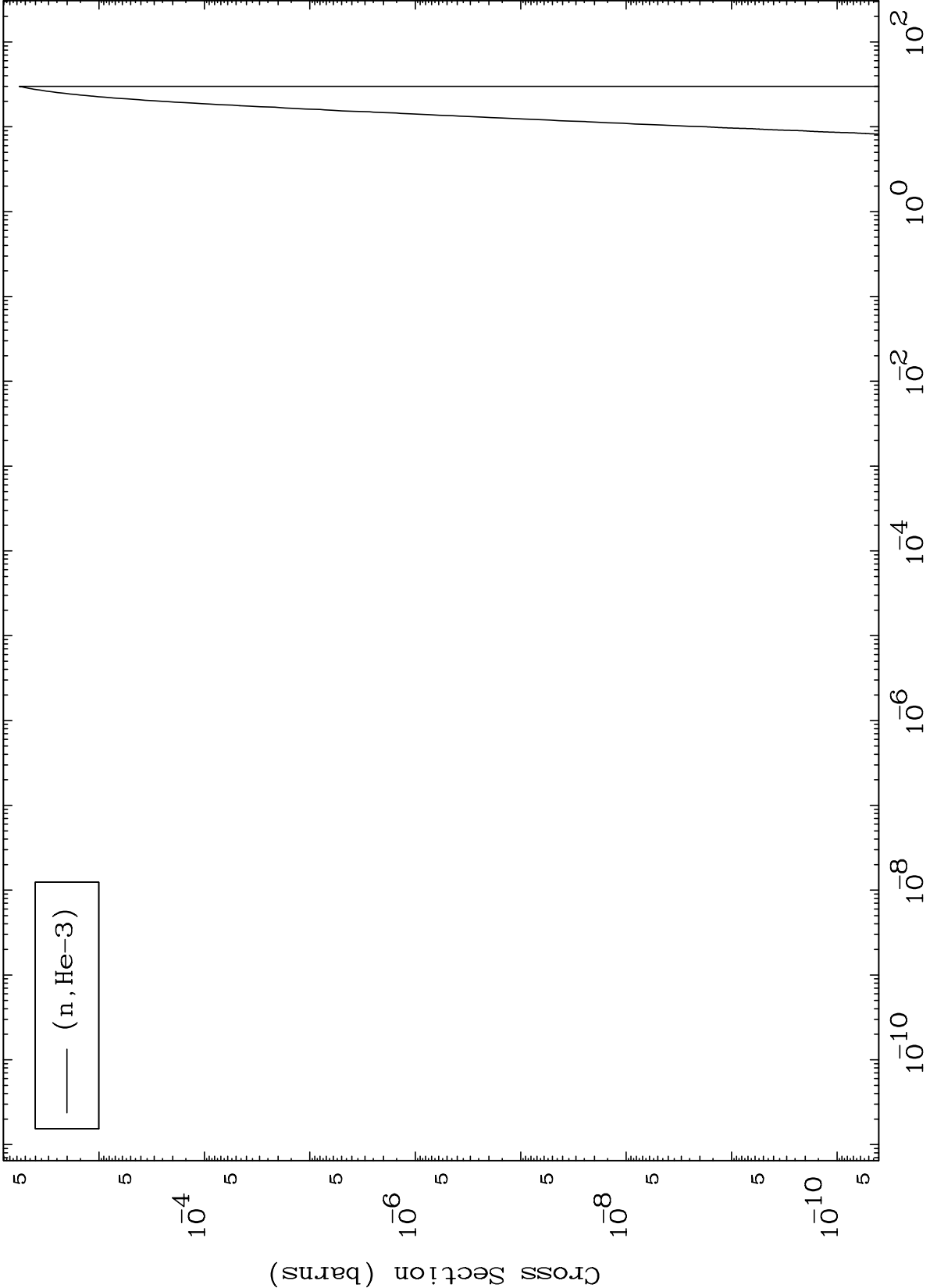
Incident Energy (MeV)

13

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(n,He3) Levels
293 Kelvin Cross Sections

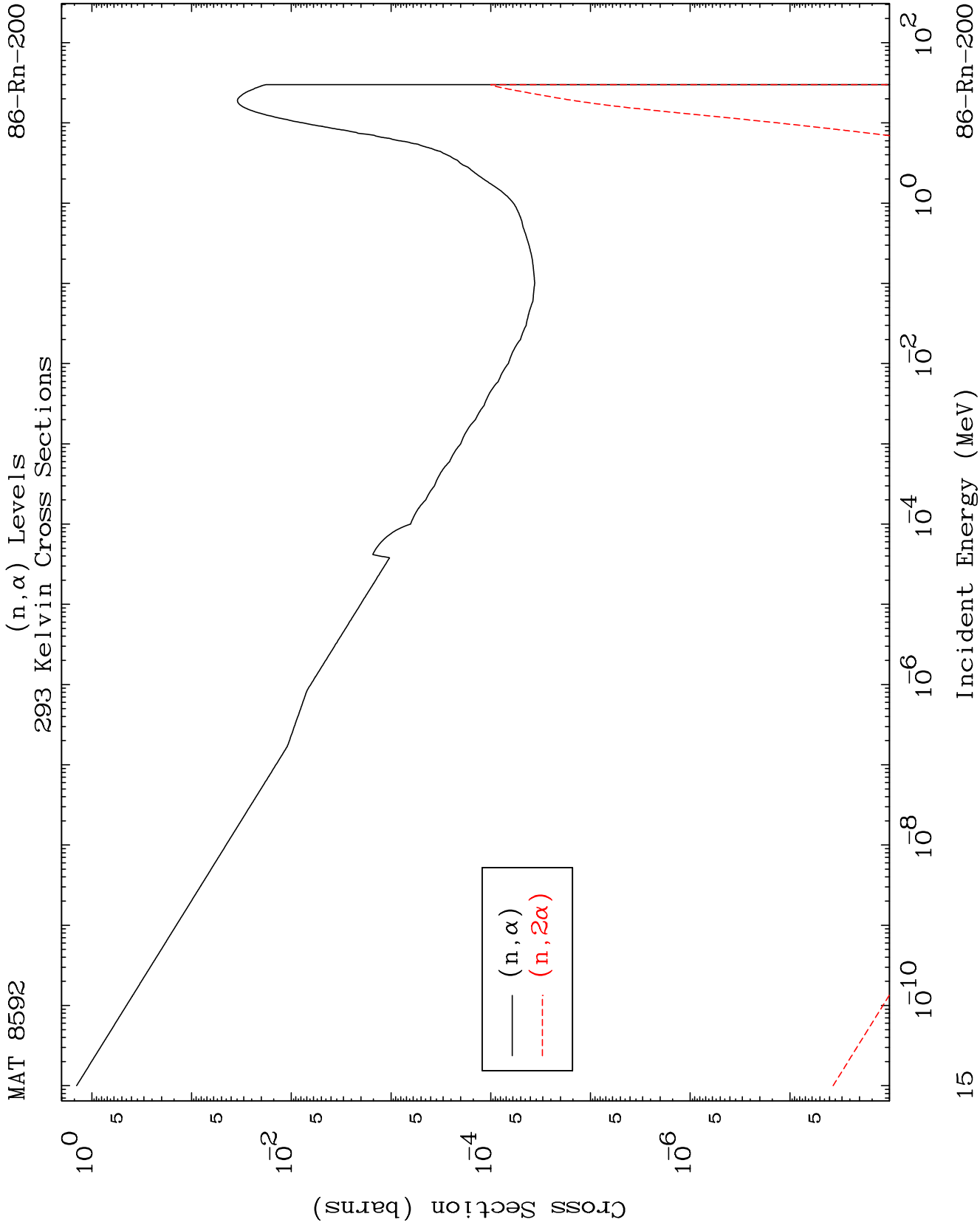
86-Rn-200

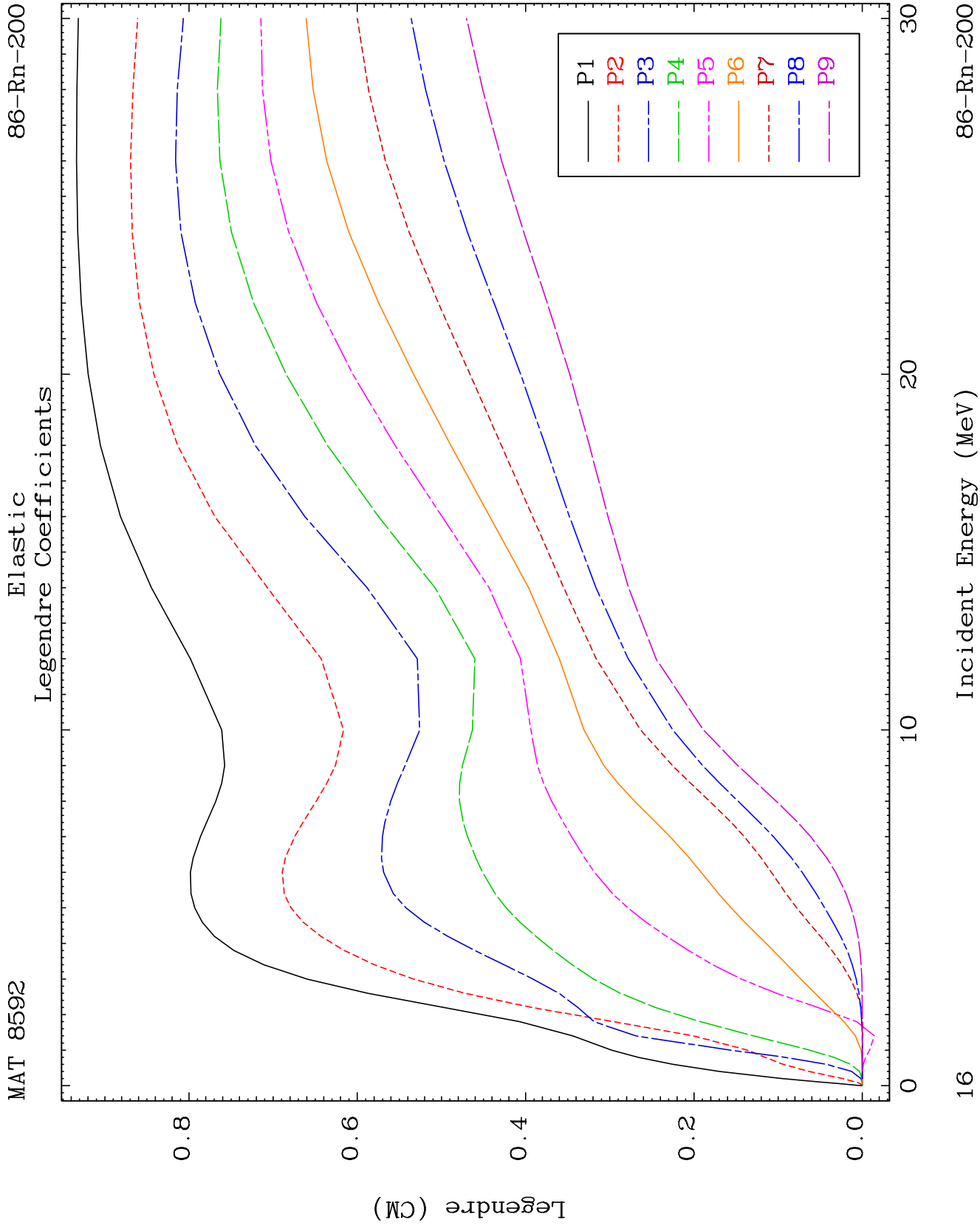


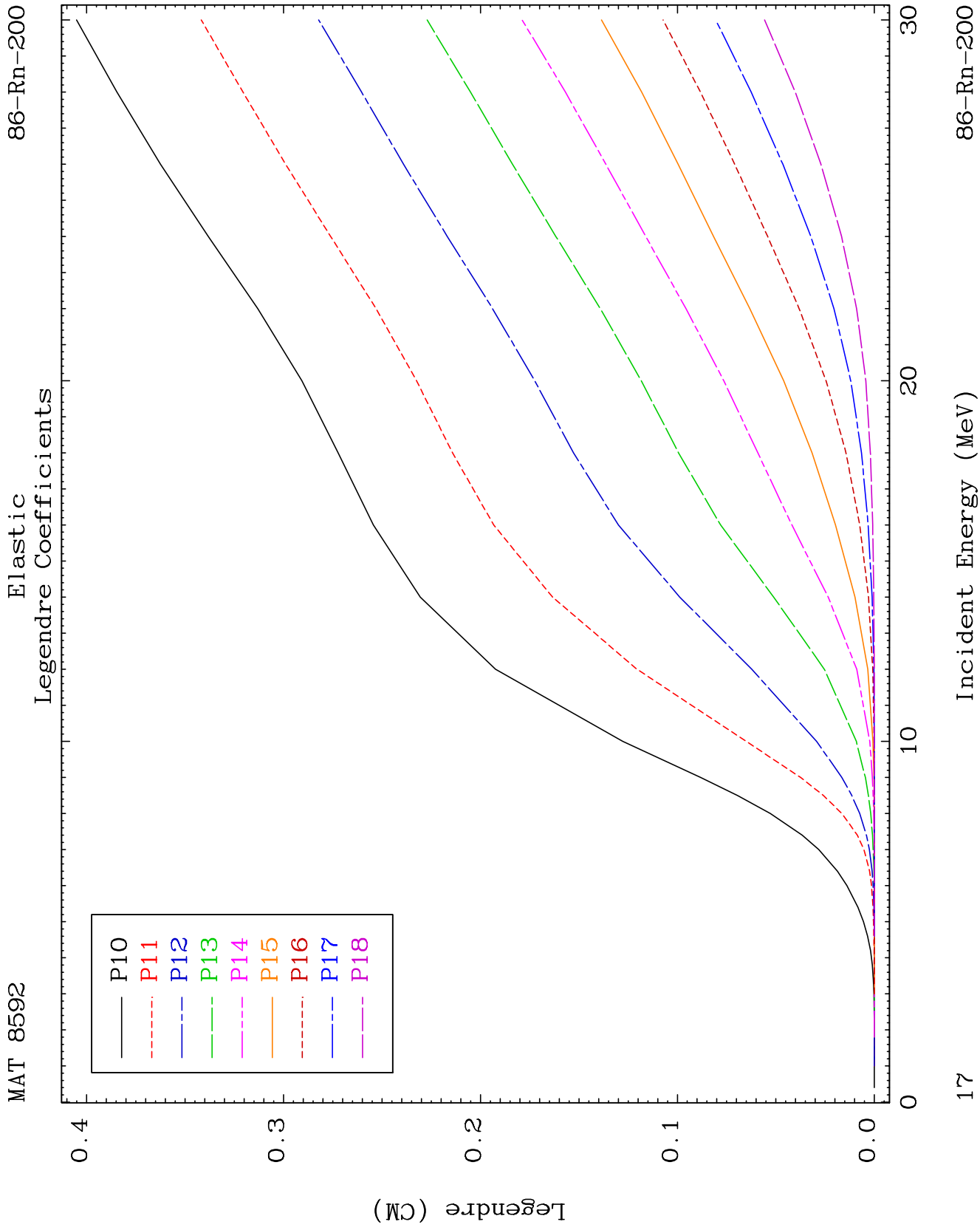
14

Incident Energy (MeV)

86-Rn-200



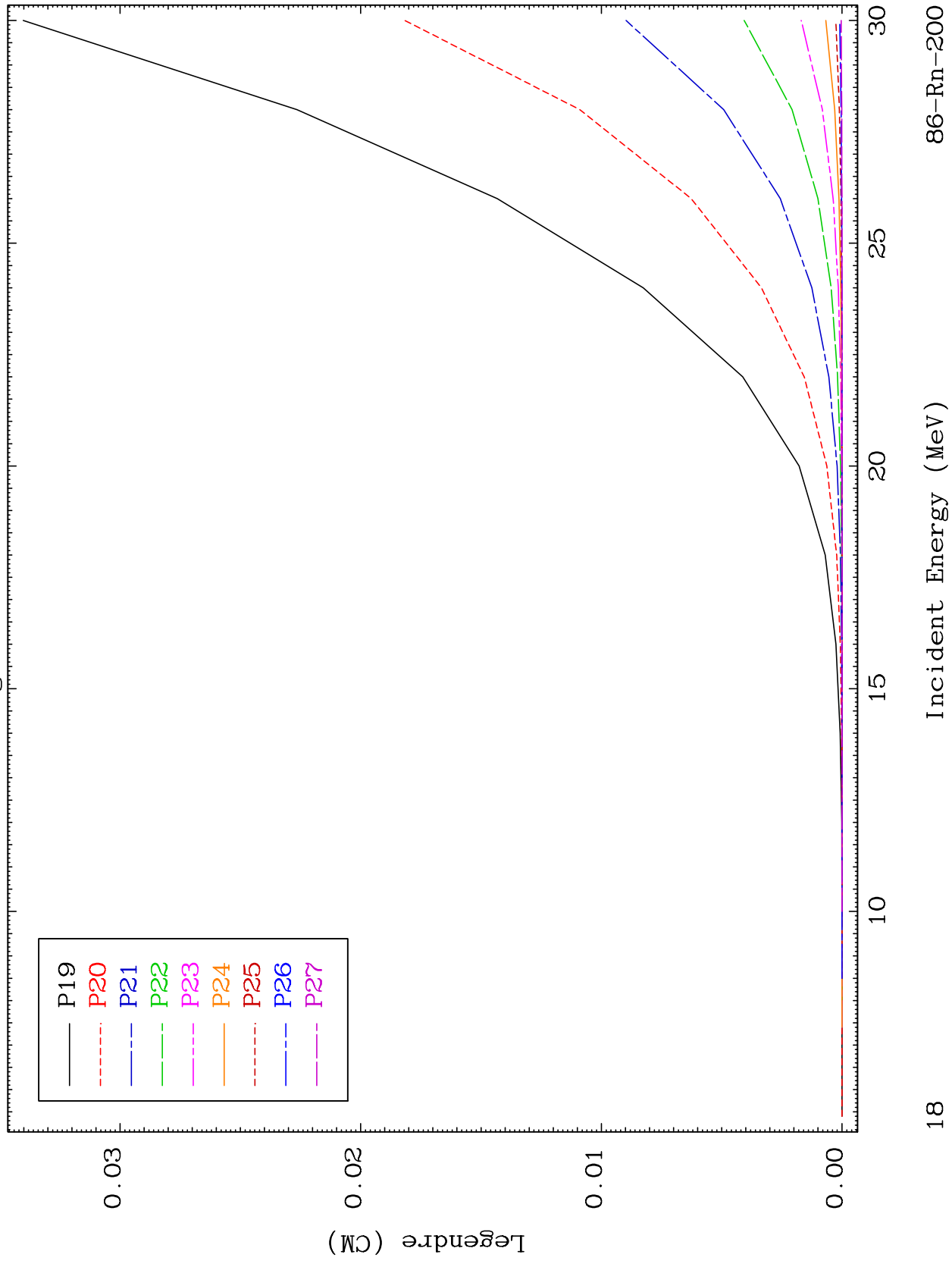


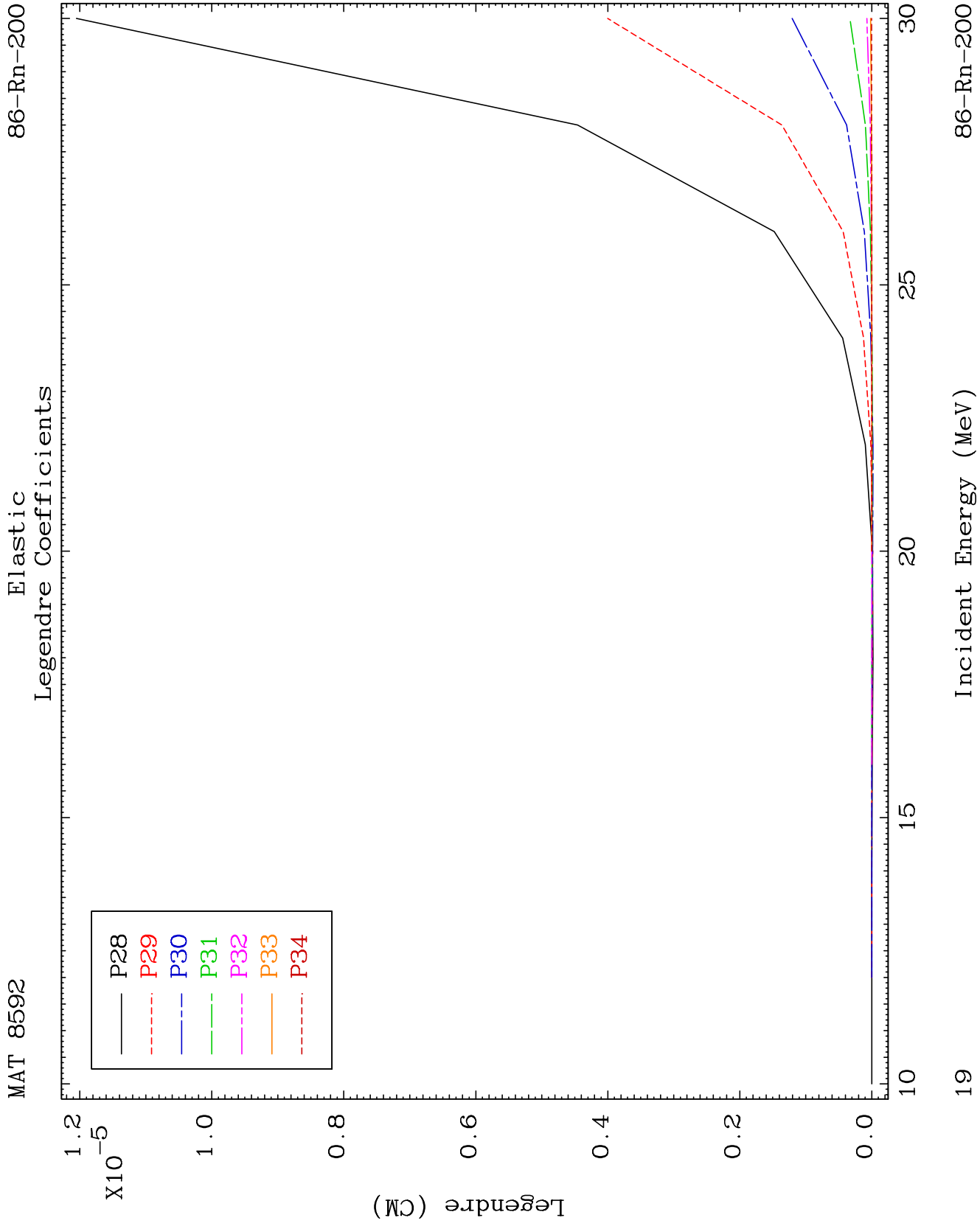


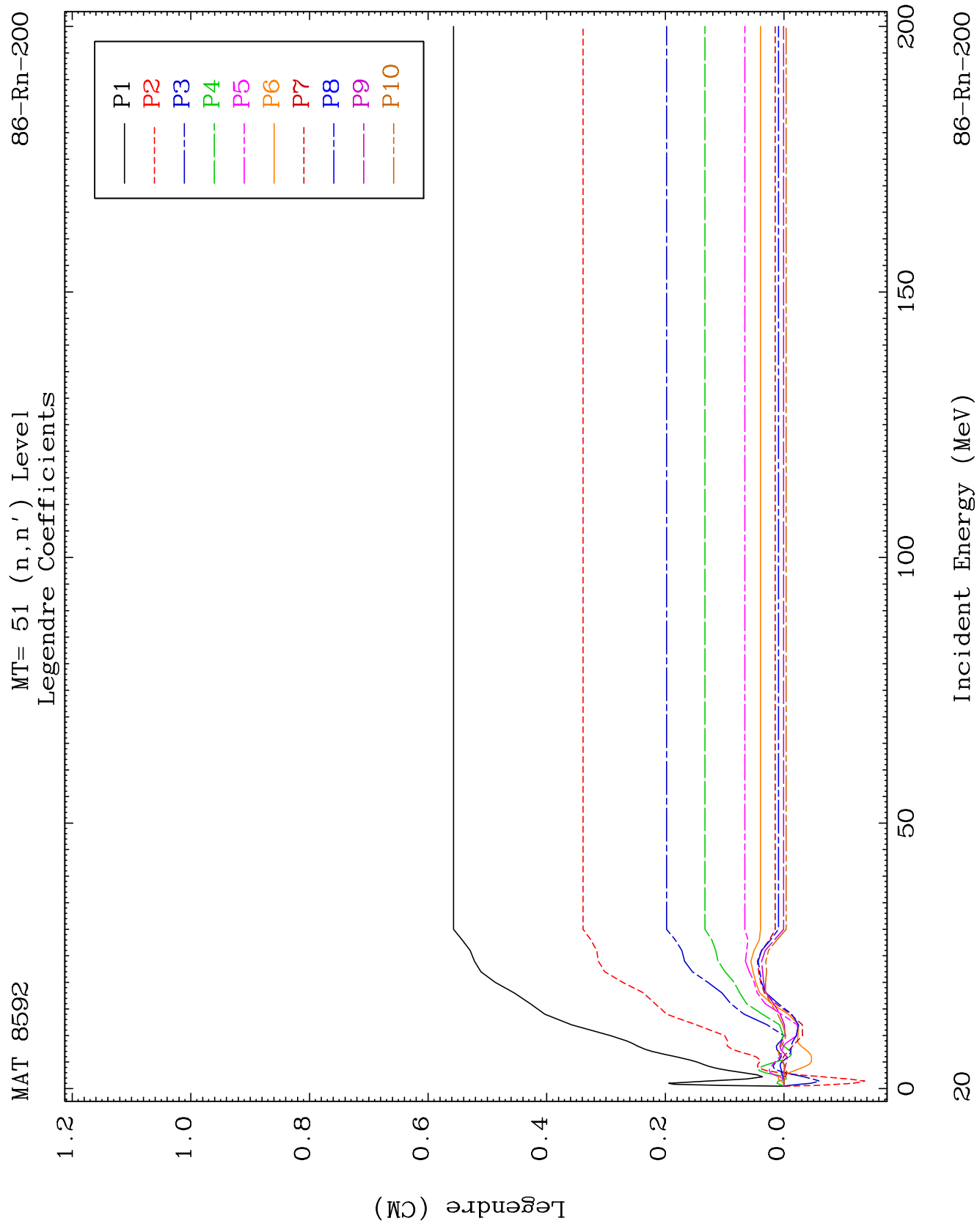
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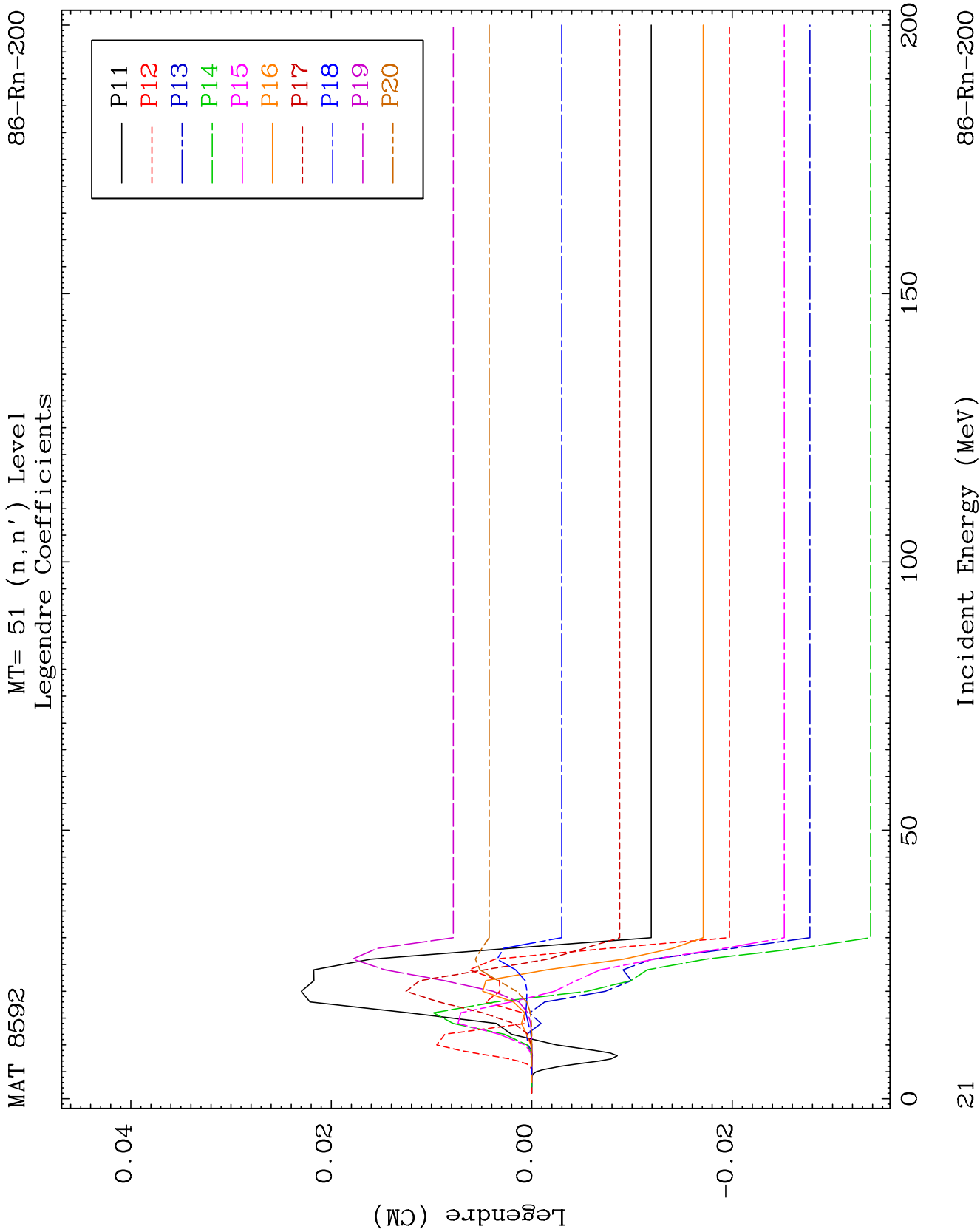
Elastic Legendre Coefficients

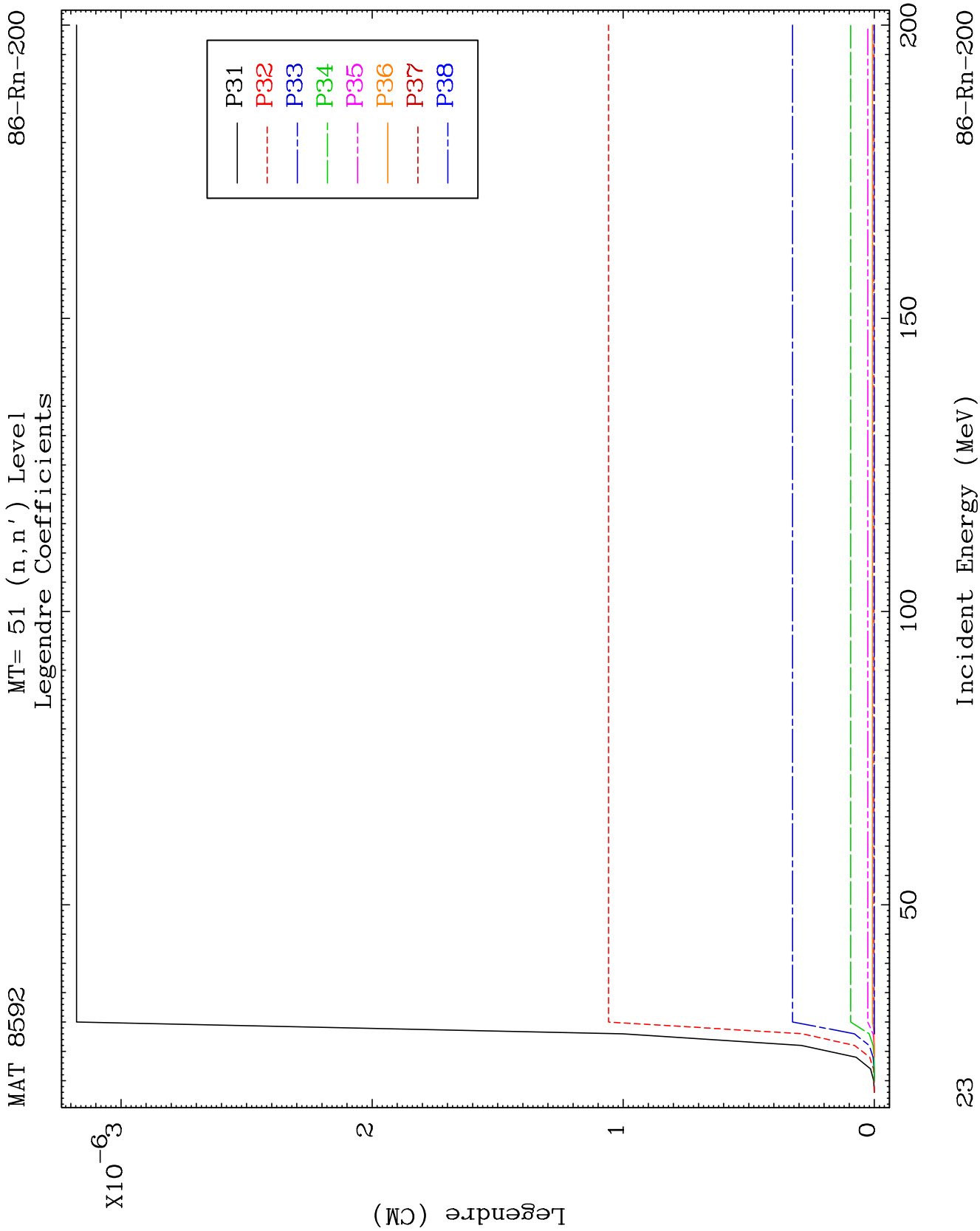
86-Rn-200

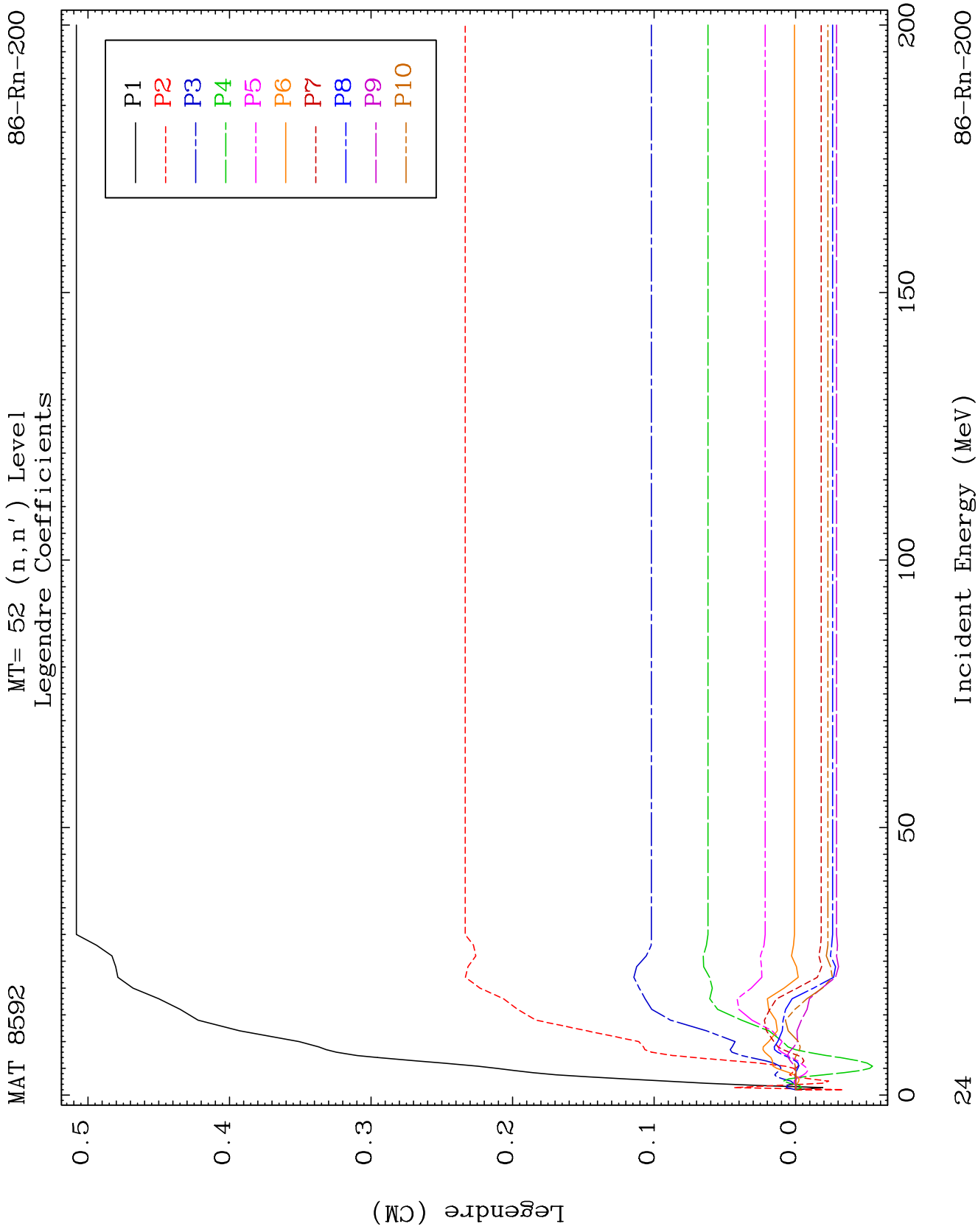


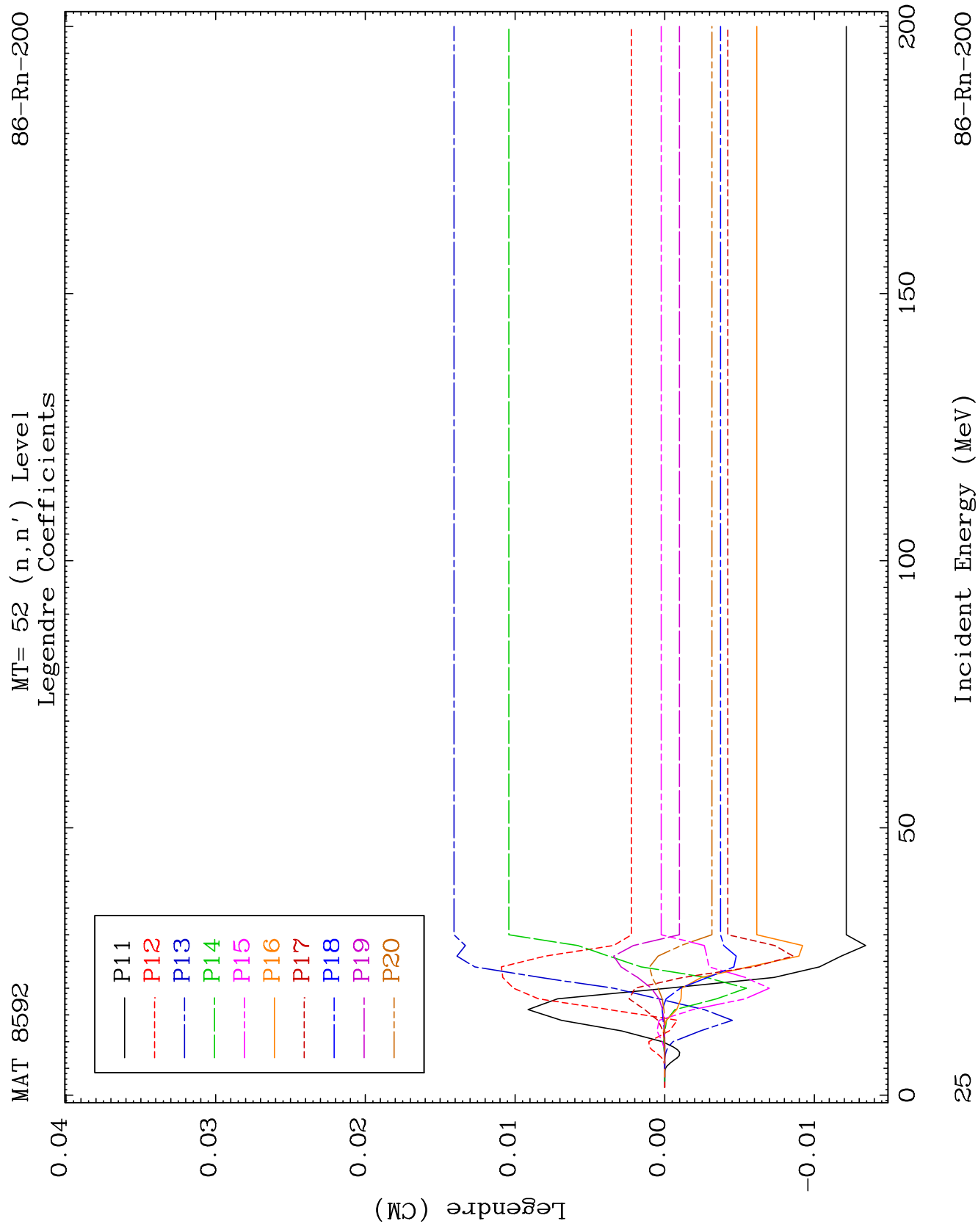








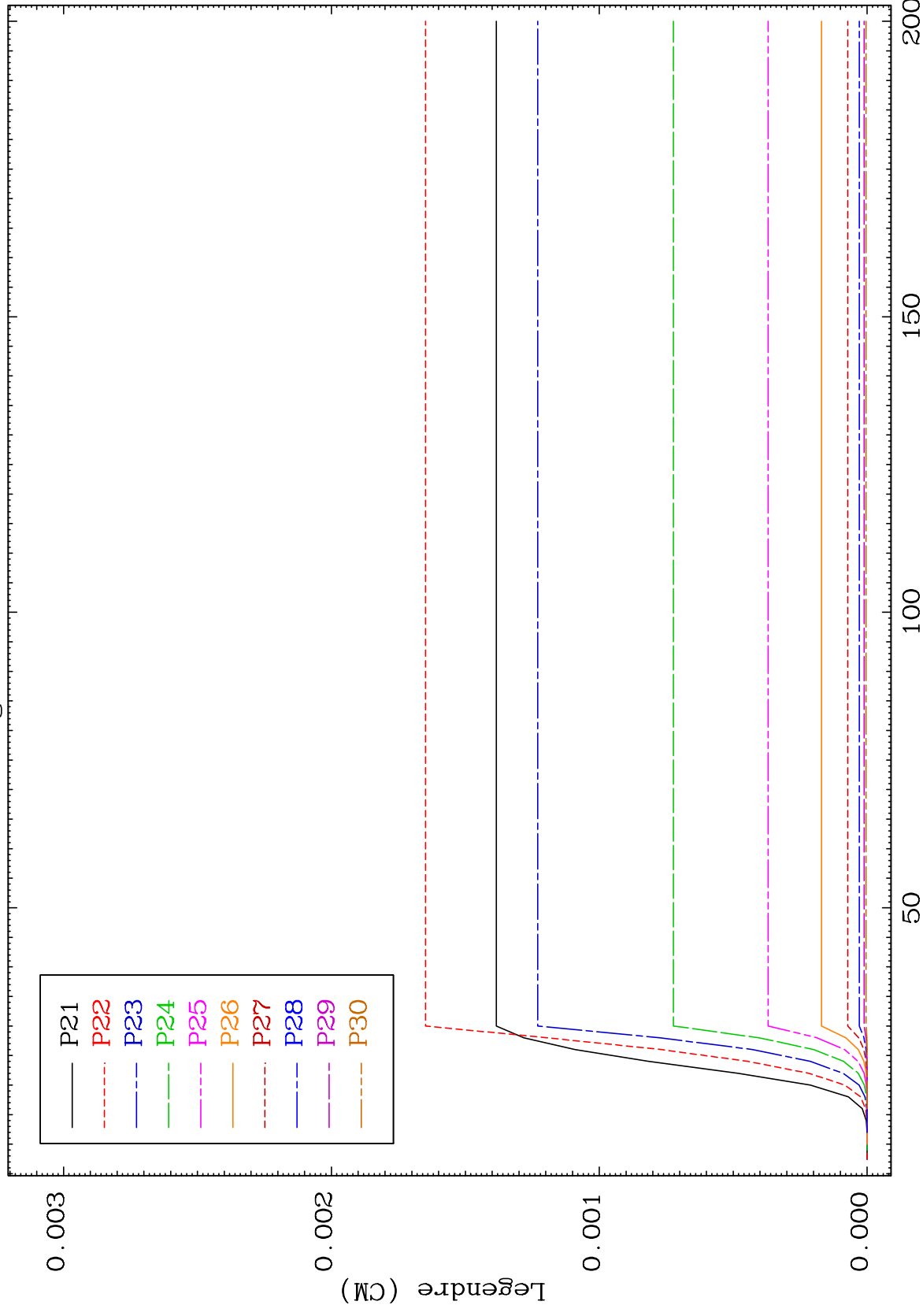




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MT= 52 (n, n') Level
Legendre Coefficients

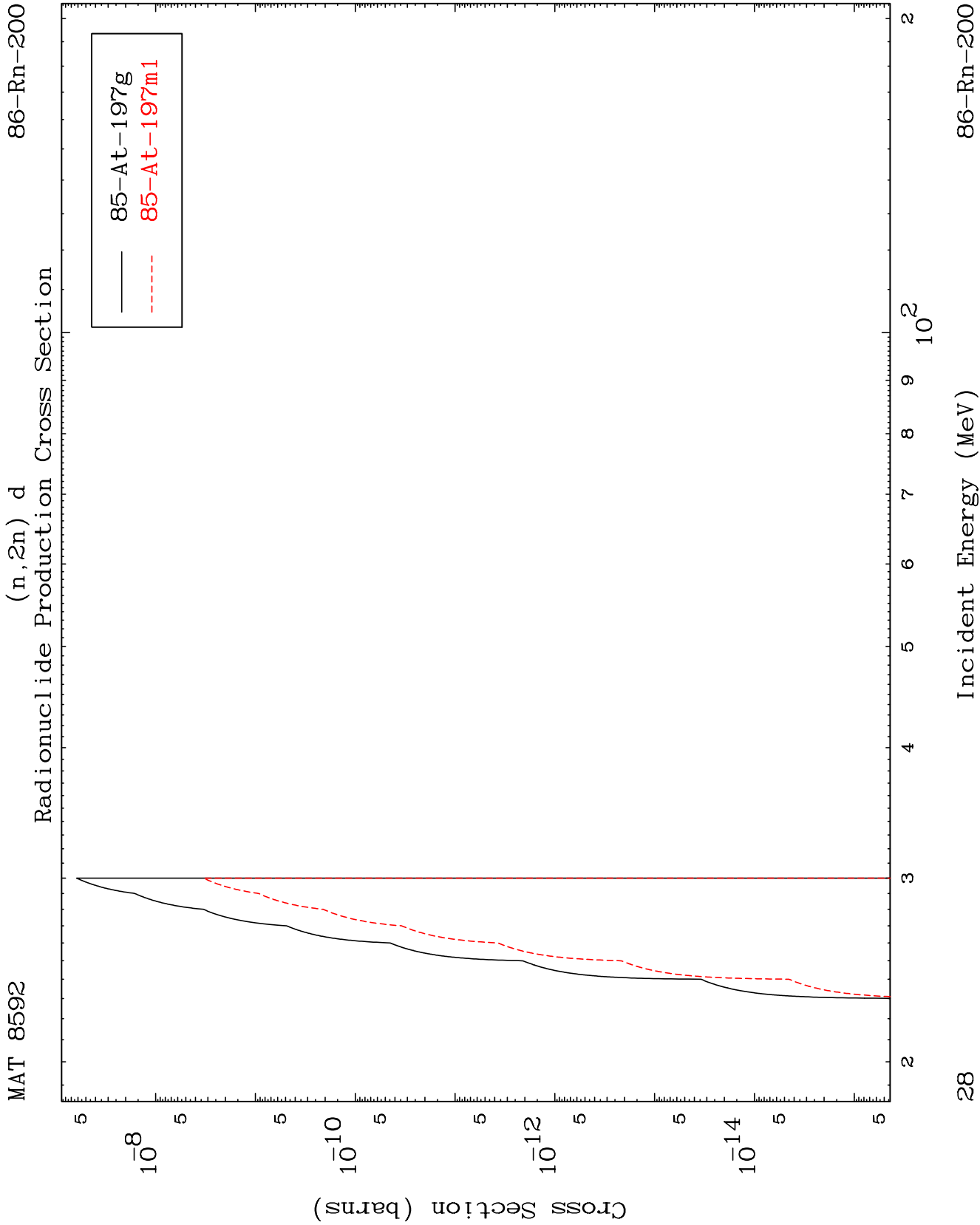
86-Rn-200



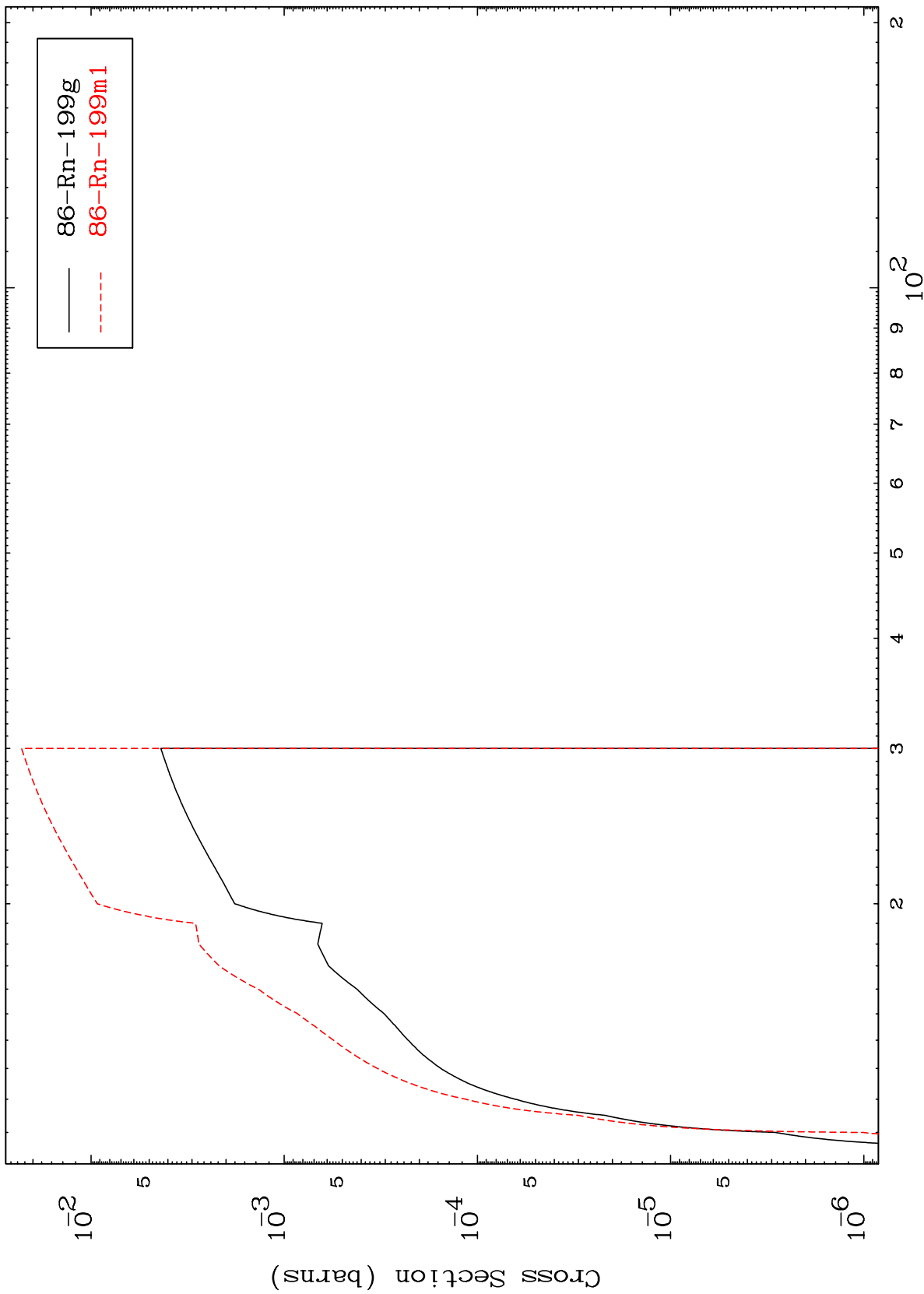
26

Incident Energy (MeV)

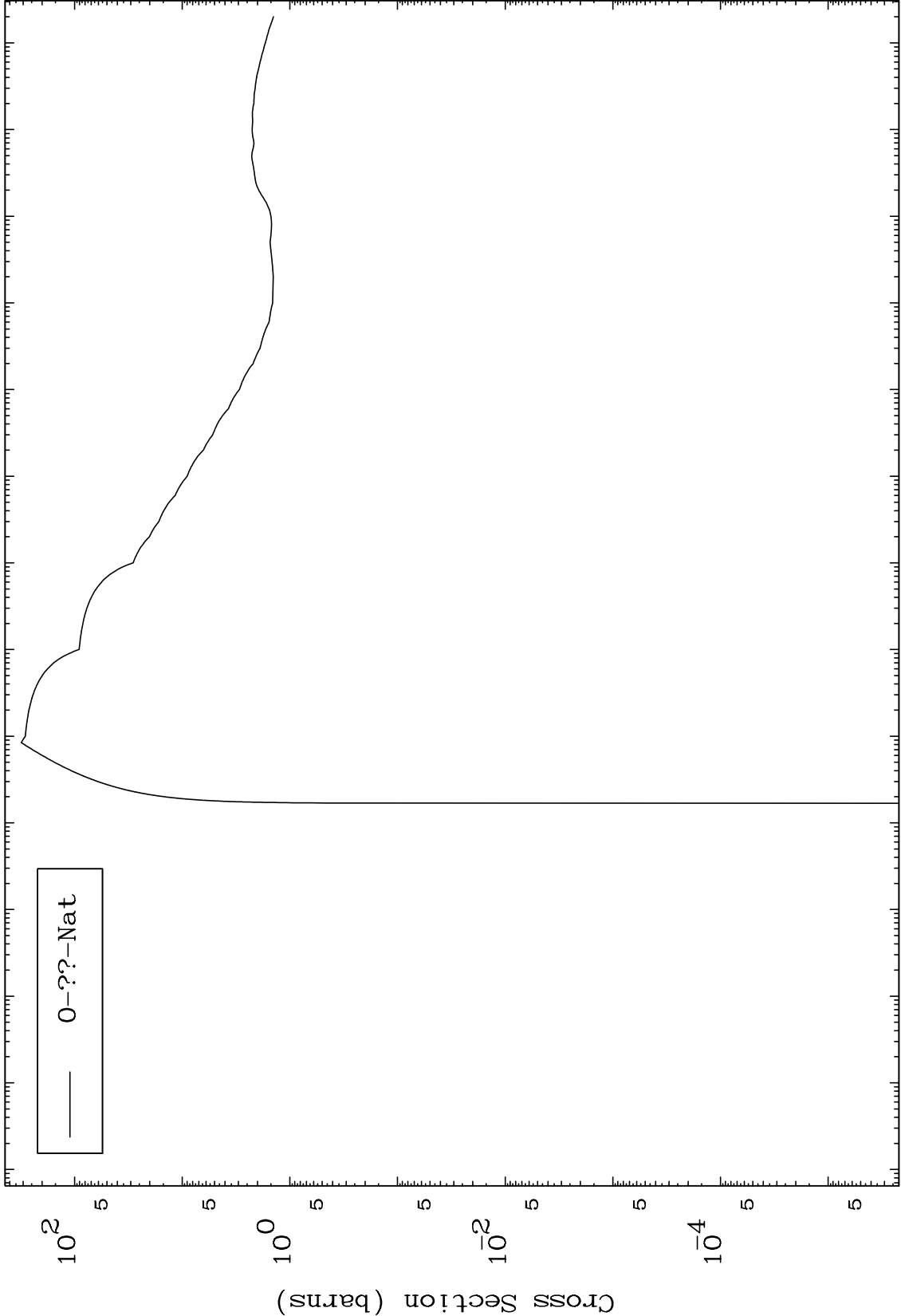
 86-Rn-200



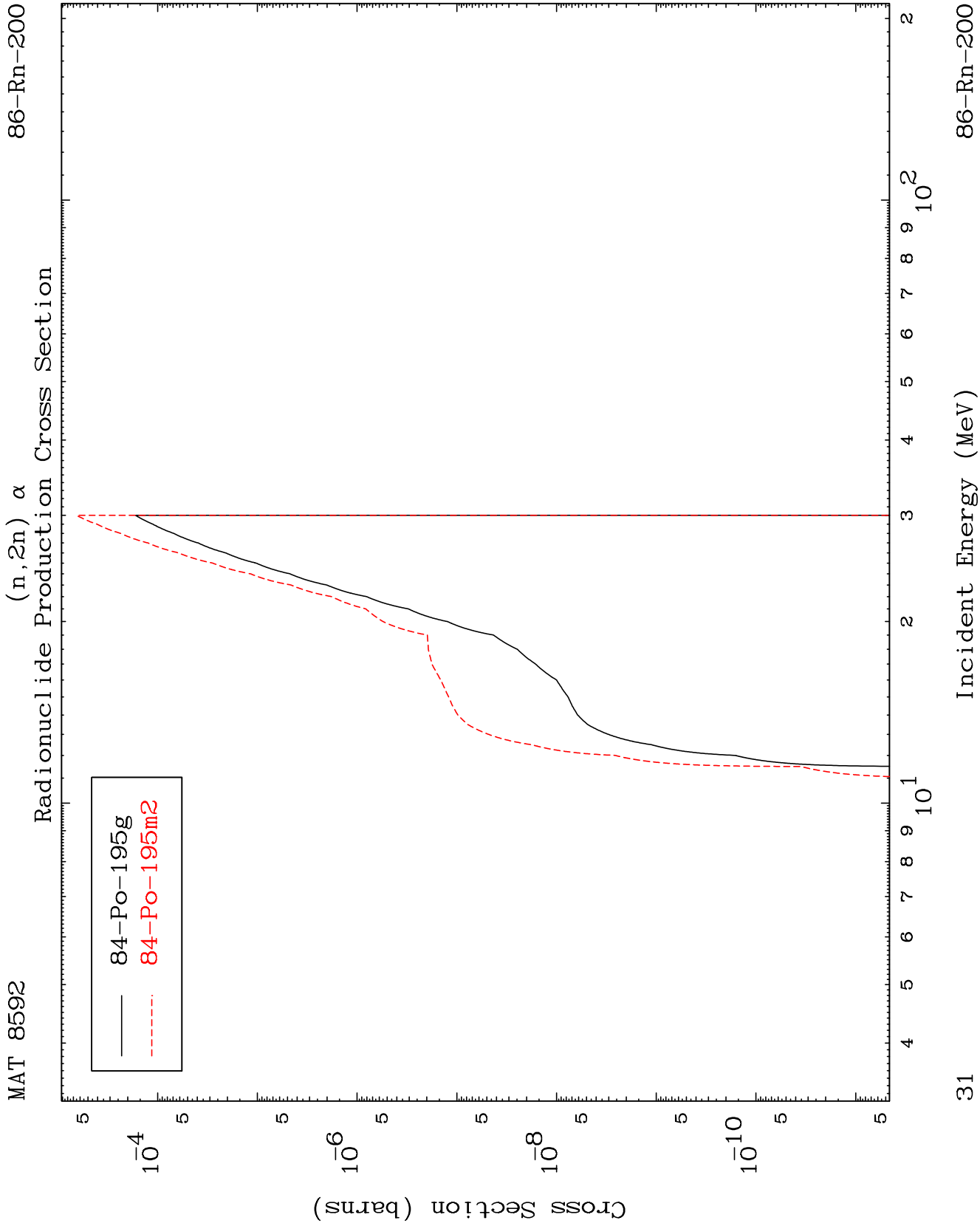
(n,2n)
Radionuclide Production Cross Section



Fission
Radionuclide Production Cross Section



— 0-??-Nat

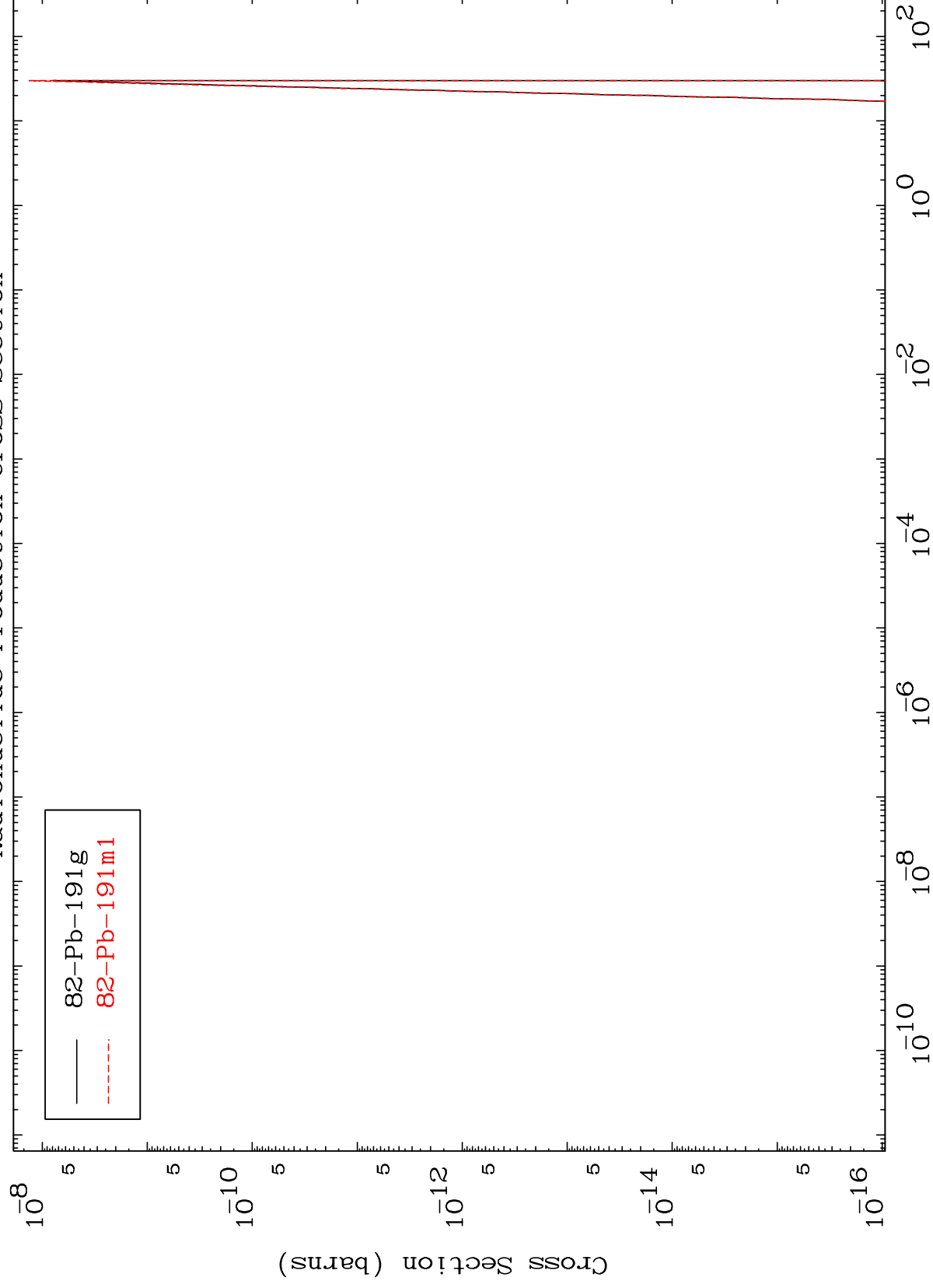


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(n,2n) 2α

86-Rn-200

Radionuclide Production Cross Section

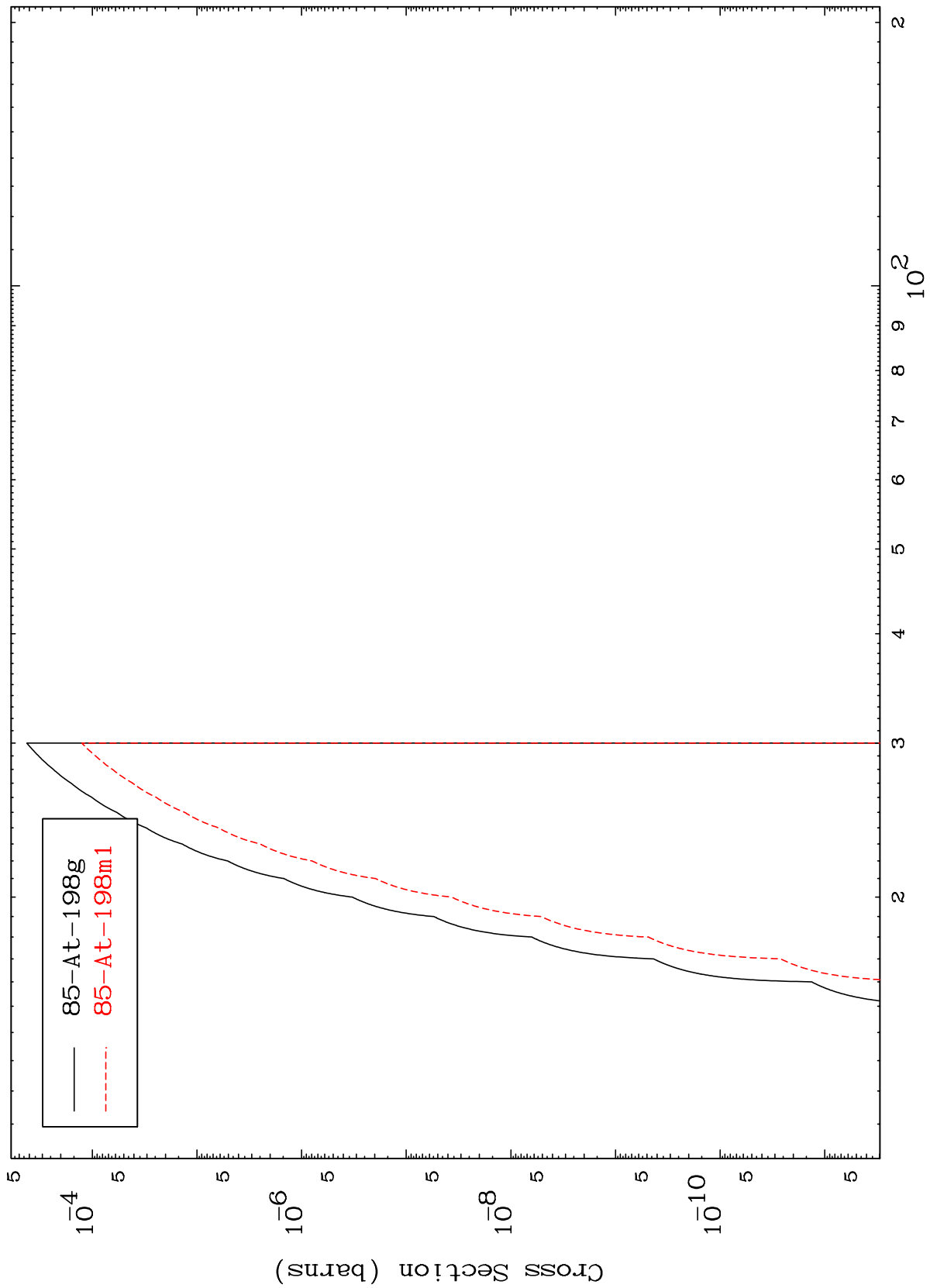


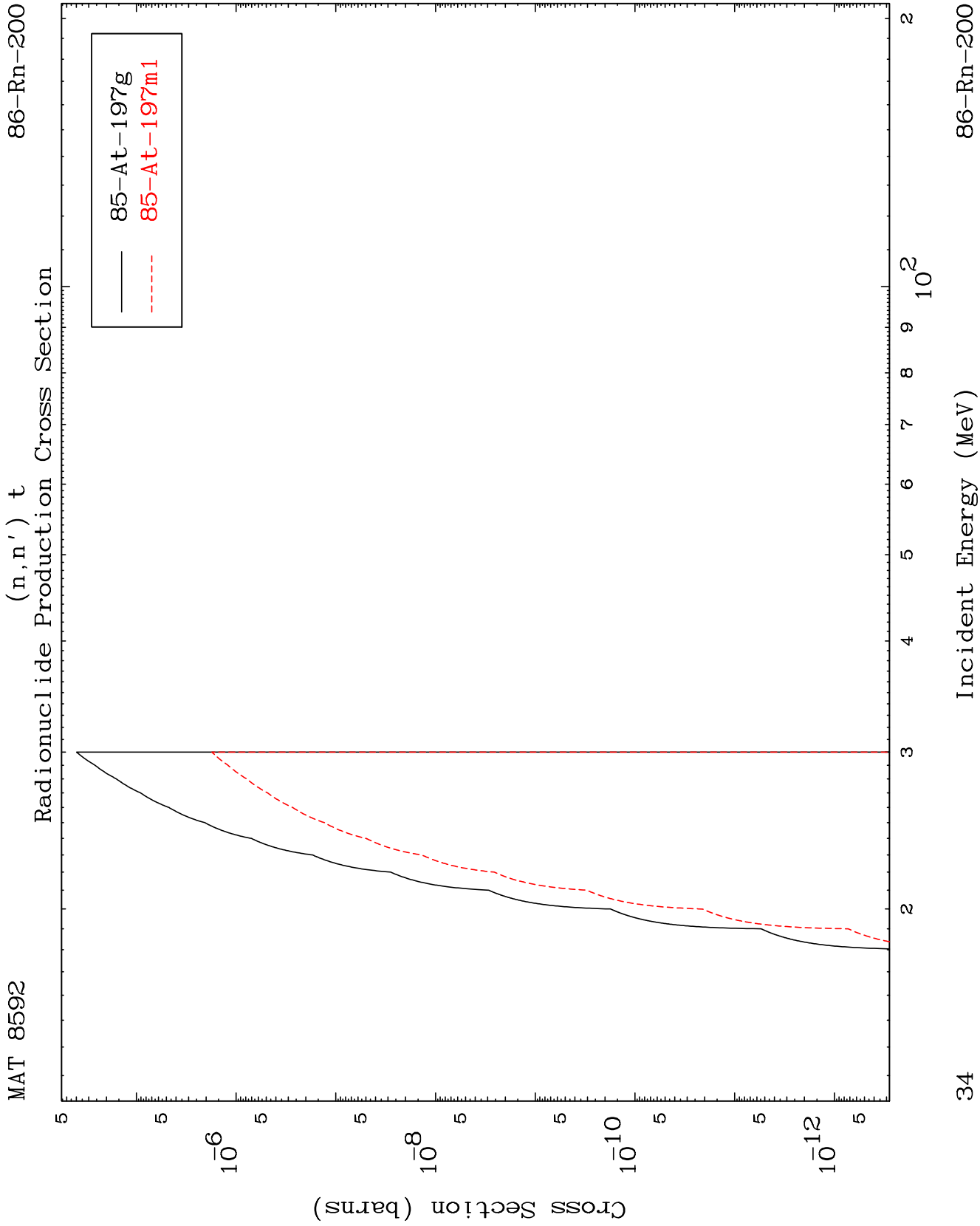
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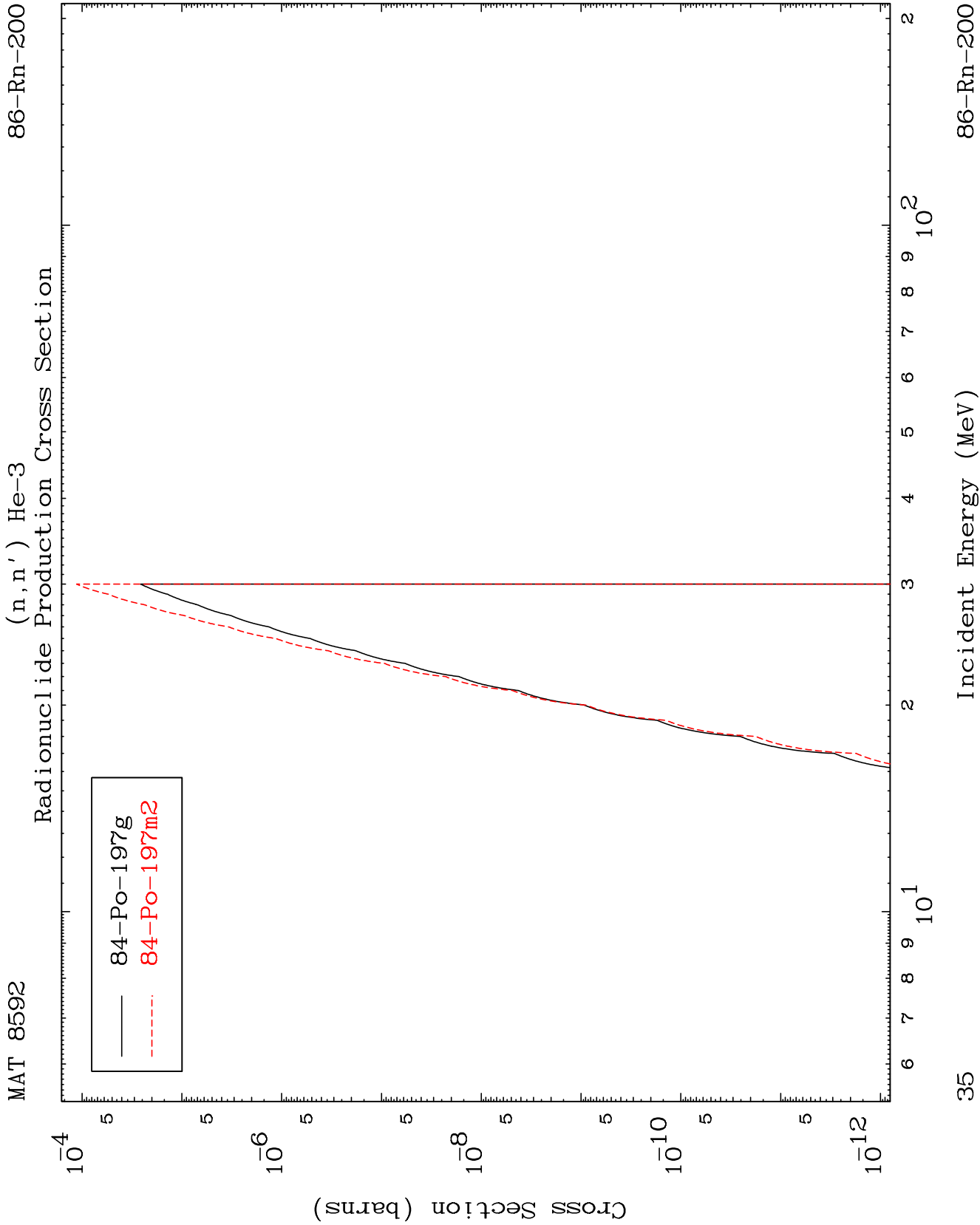
Incident Energy (MeV)

86-Rn-200

Radionuclide Production Cross Section
(n,n') d



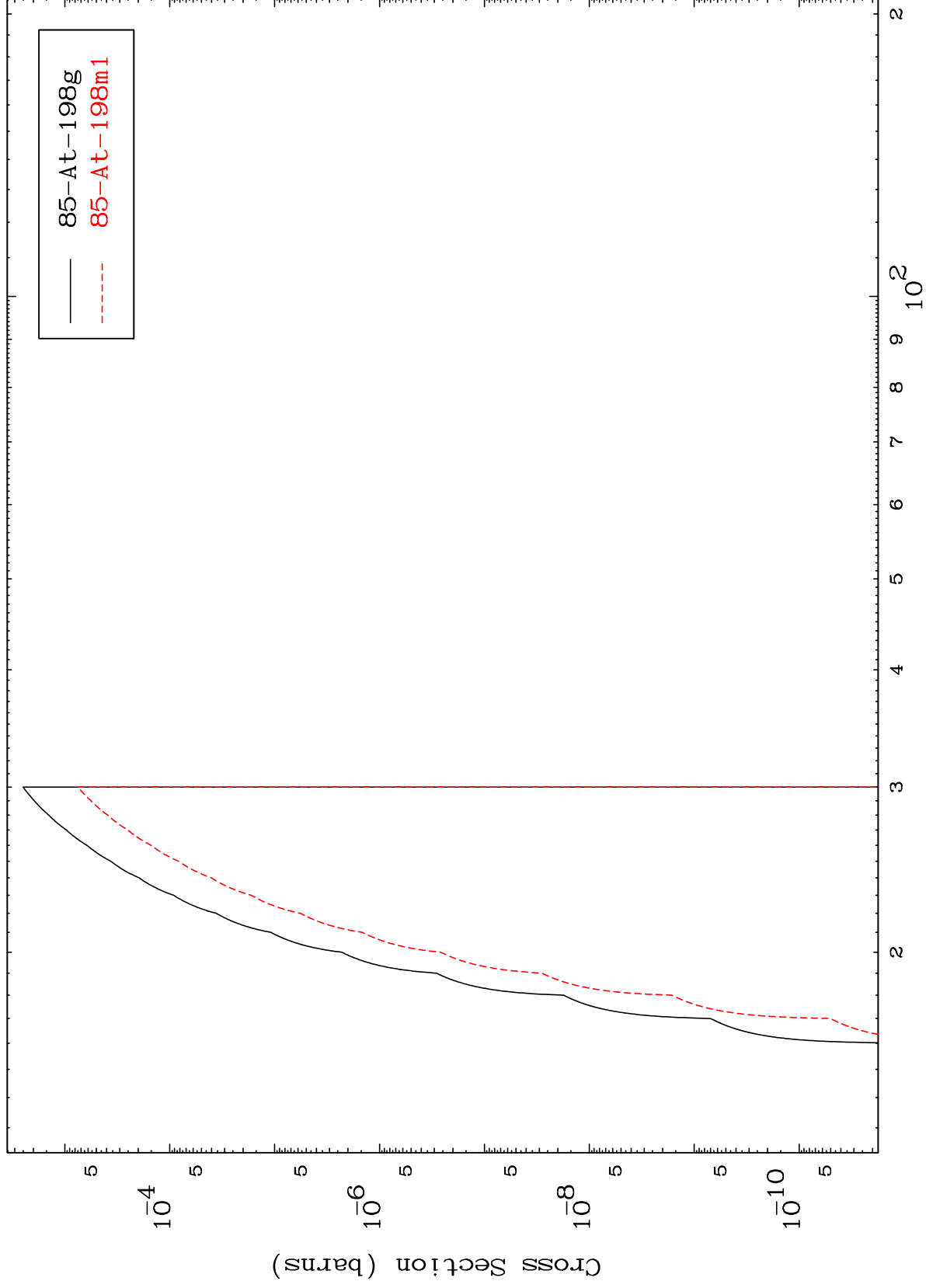




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86-Rn-200

(n,2n) p
Radionuclide Production Cross Section

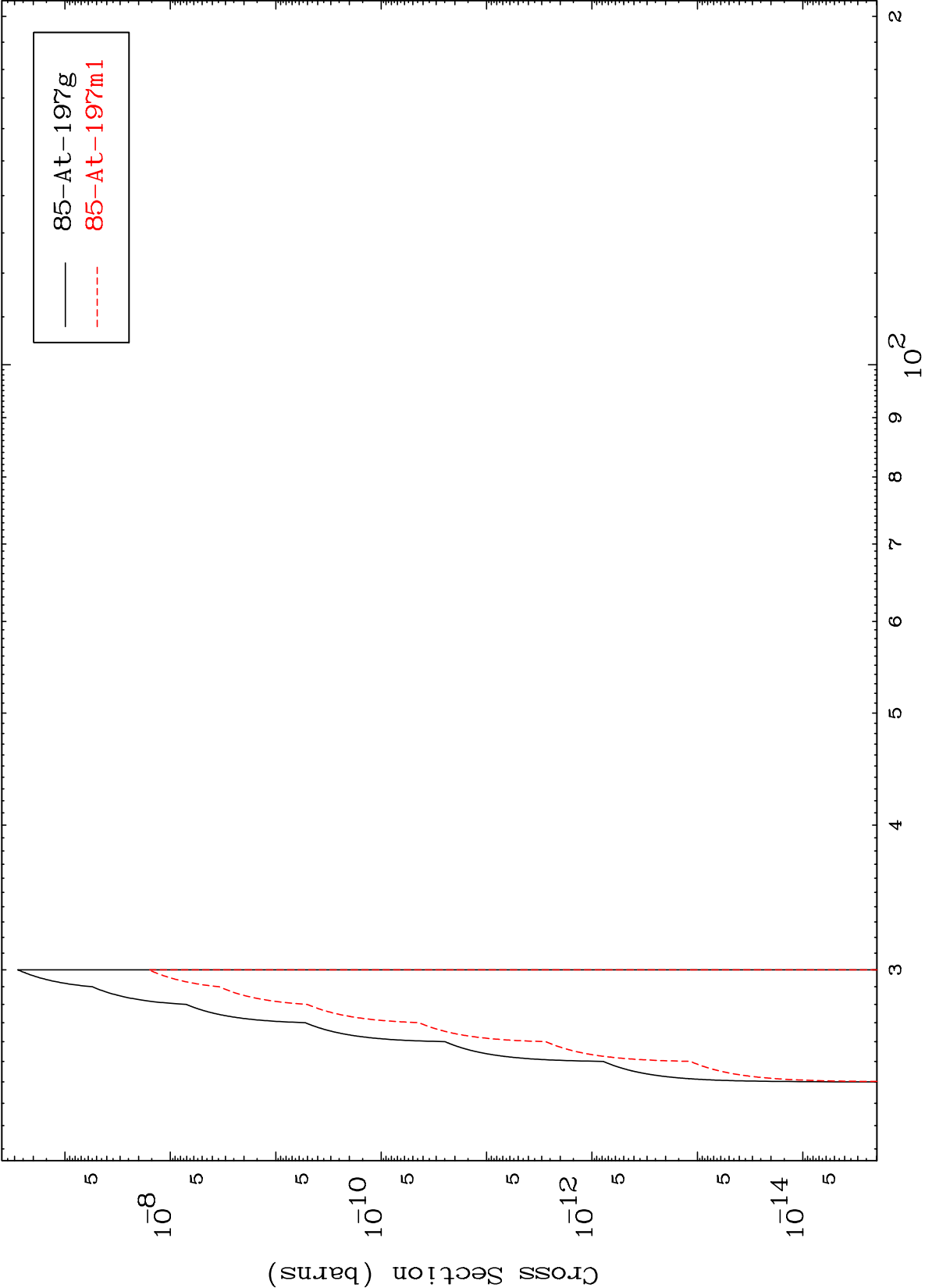


36

Incident Energy (MeV)

86-Rn-200

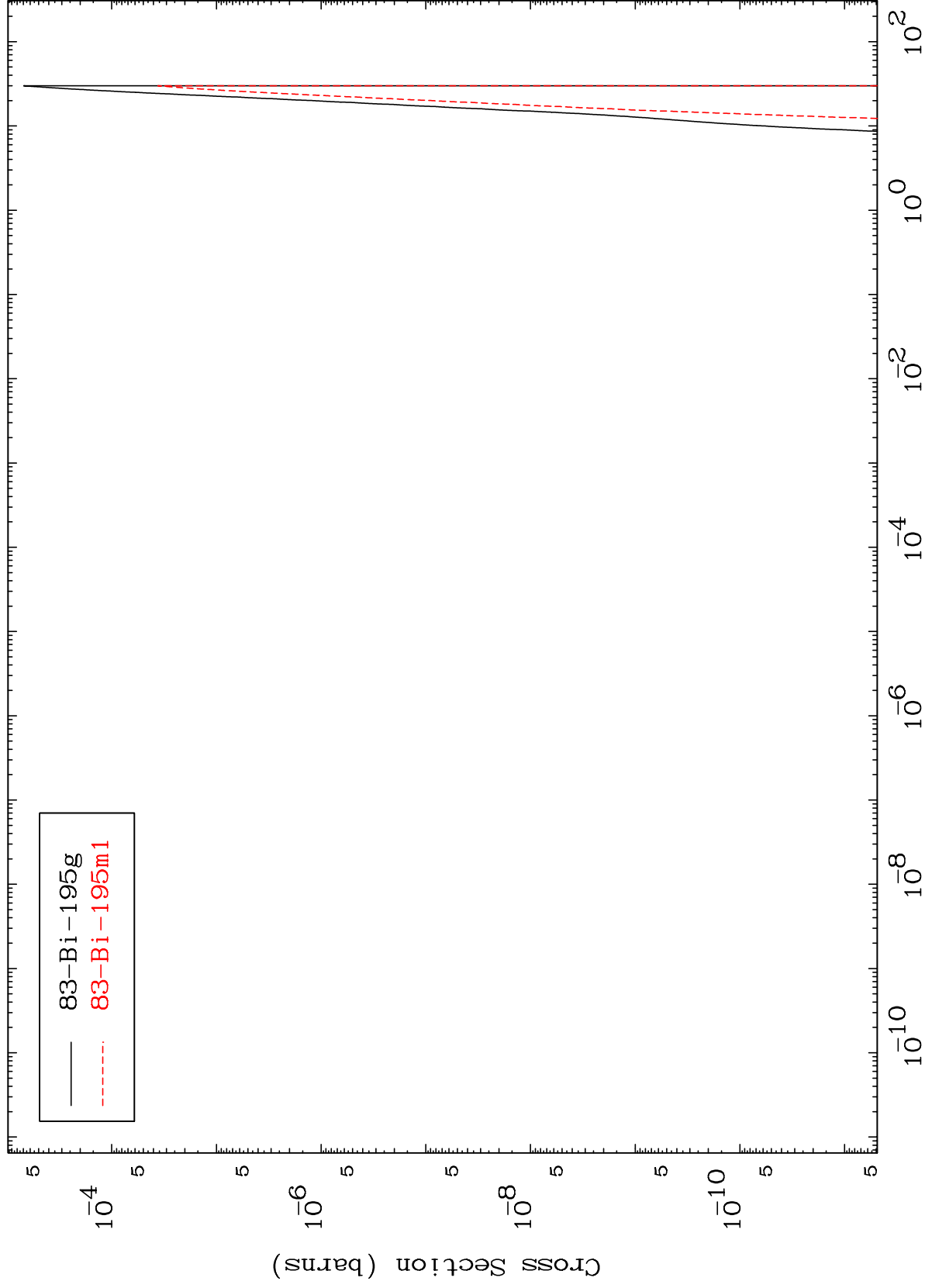
Radionuclide Production Cross Section



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86-Rn-200

(n,n') p α
Radionuclide Production Cross Section



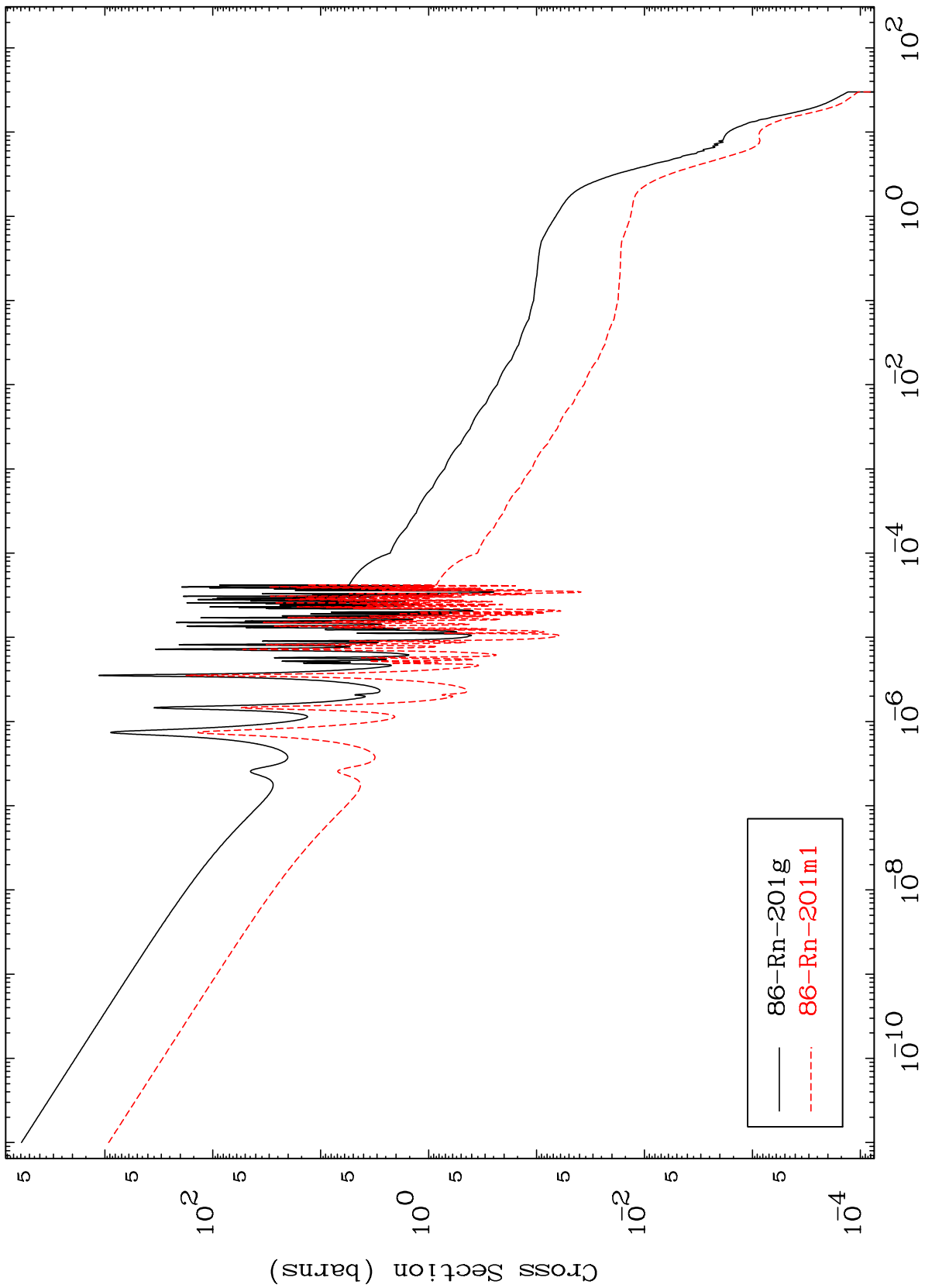
38

86-Rn-200

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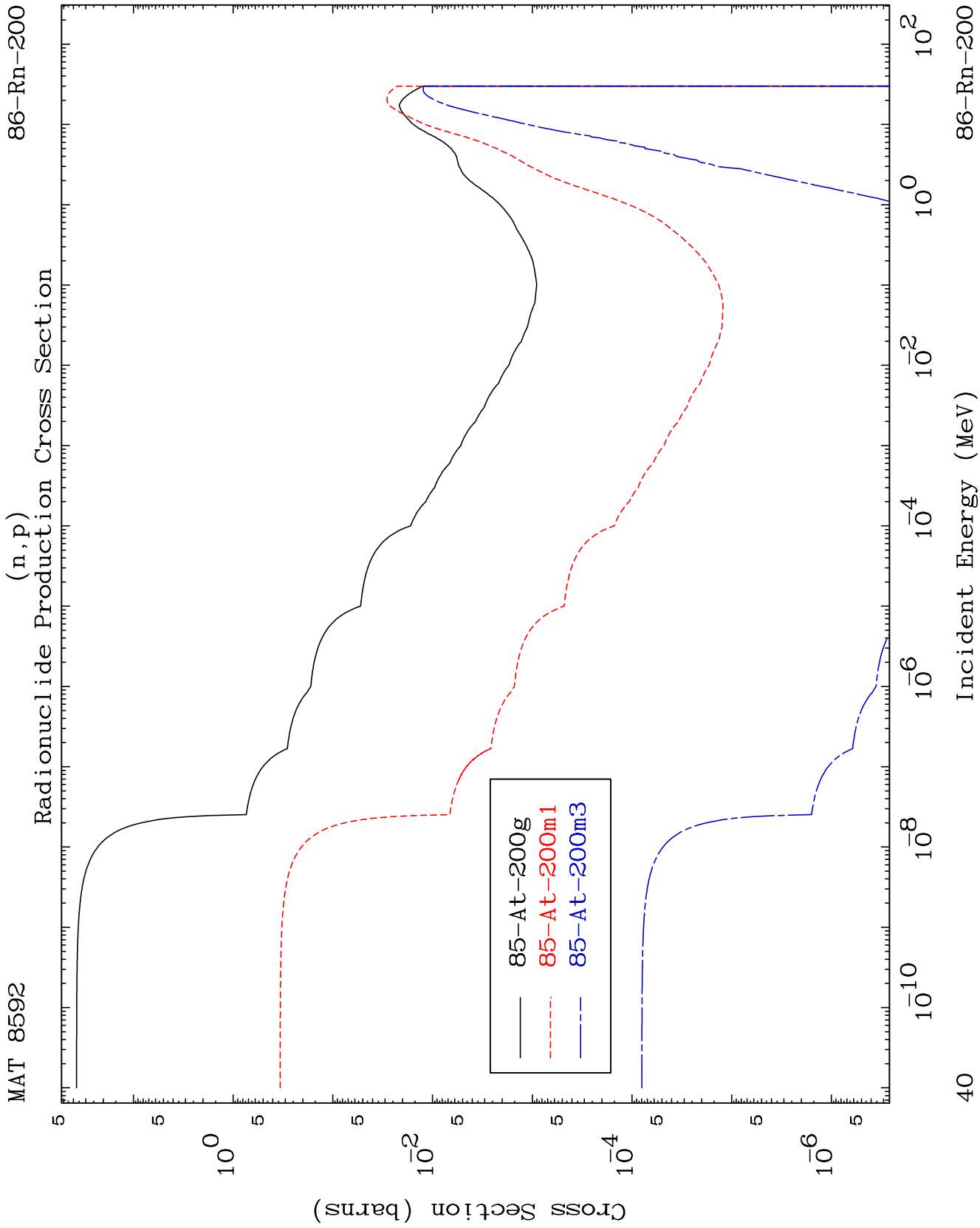
86-Rn-200

Radionuclide Production Cross Section
(n, γ)



39

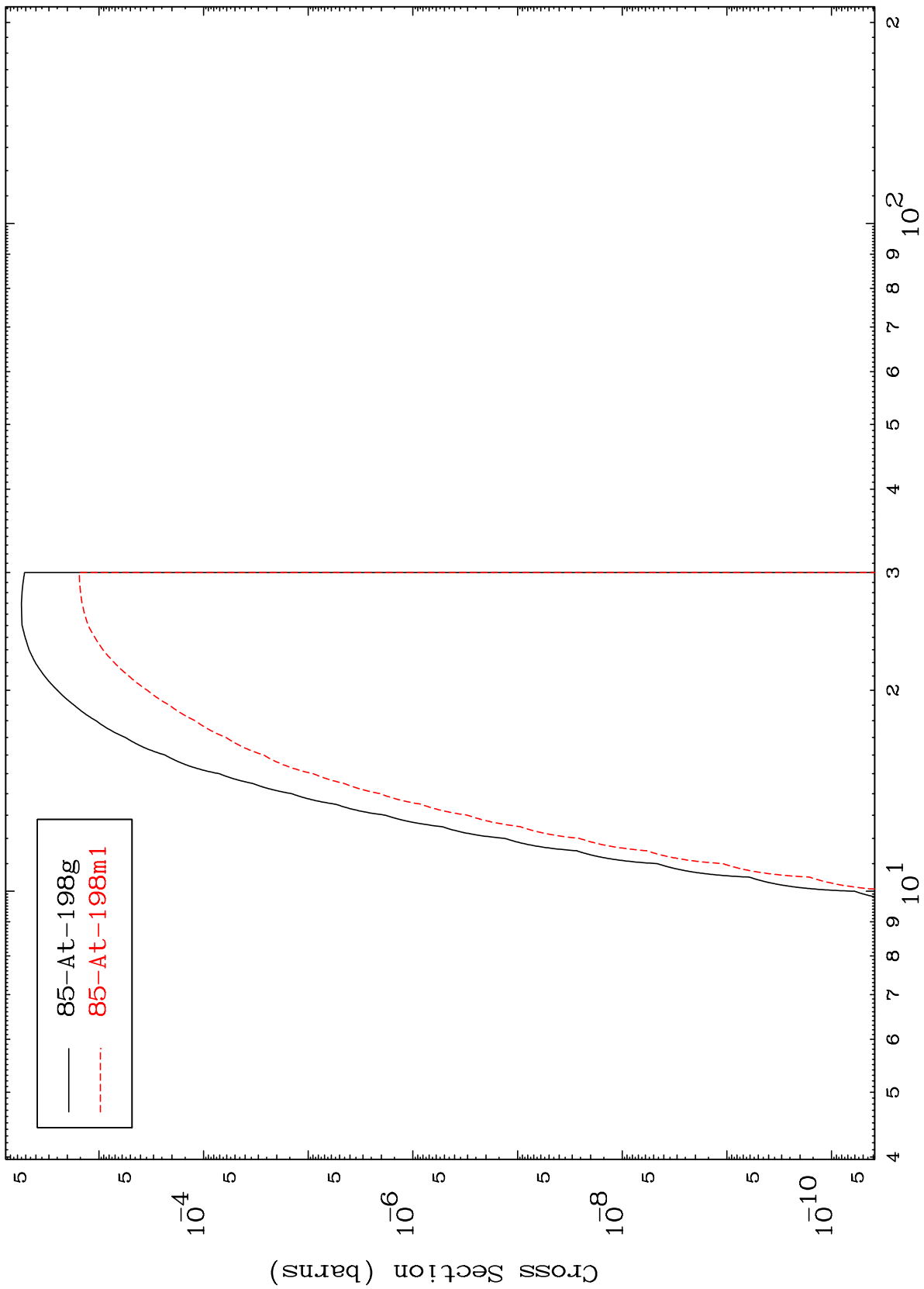
86-Rn-200



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86-Rn-200

Radionuclide Production Cross Section
(n,t)



Incident Energy (MeV)

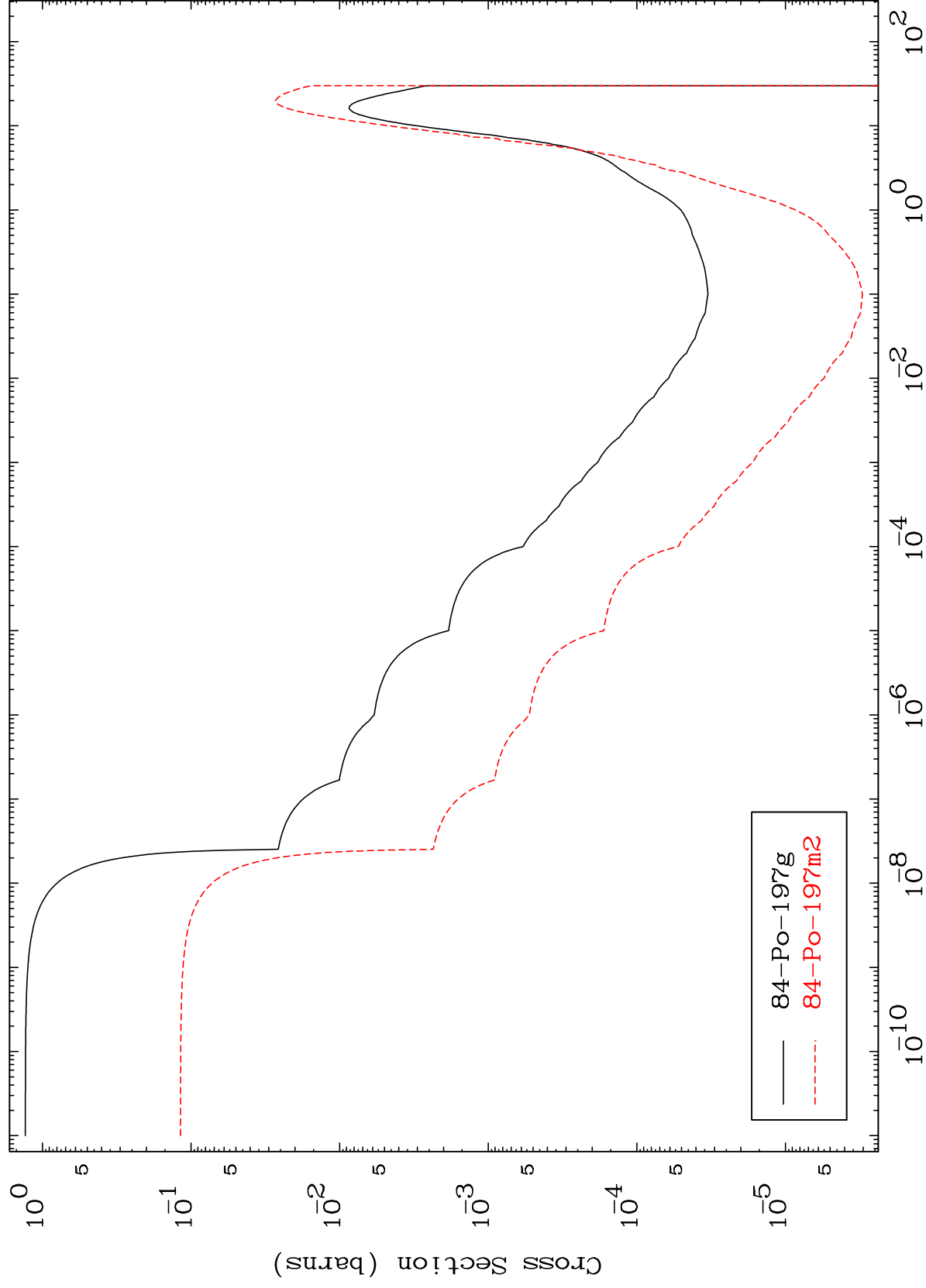
86-Rn-200

41

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86-Rn-200

Radionuclide Production Cross Section
(n, α)



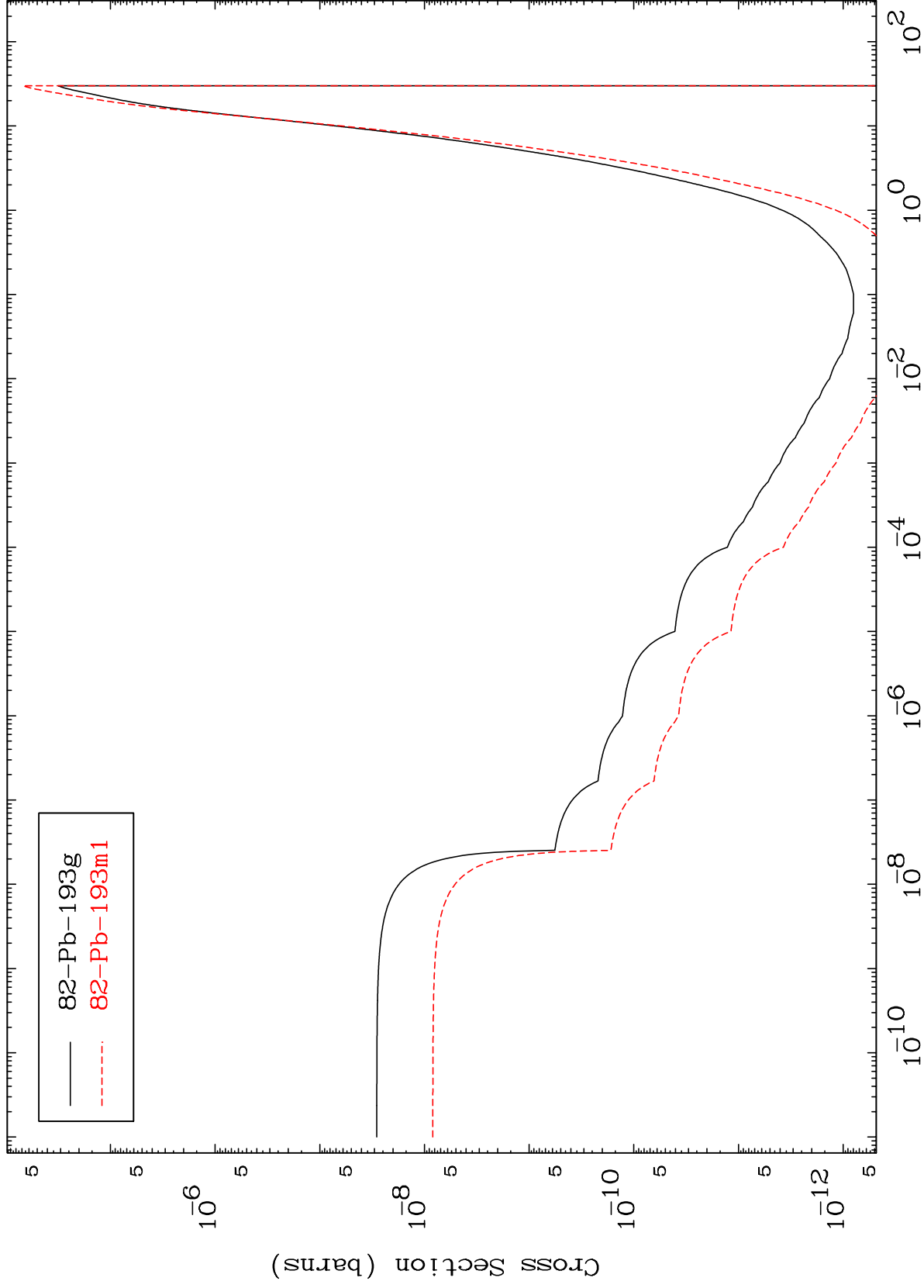
42

86-Rn-200

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86-Rn-200

Radionuclide Production Cross Section
(n,2α)



43

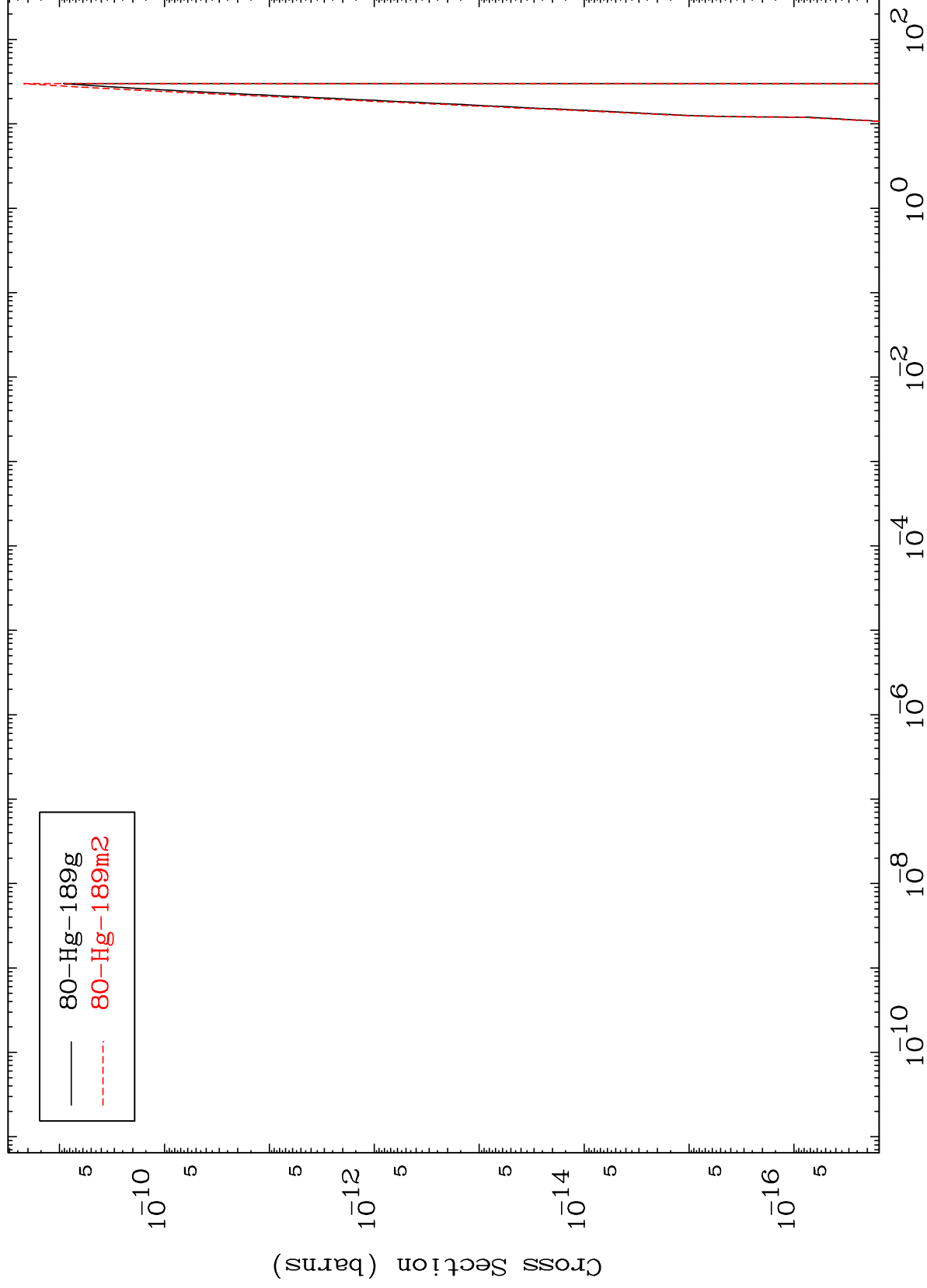
Incident Energy (MeV)

86-Rn-200

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86-Rn-200

(n,3α)
Radionuclide Production Cross Section



86-Rn-200

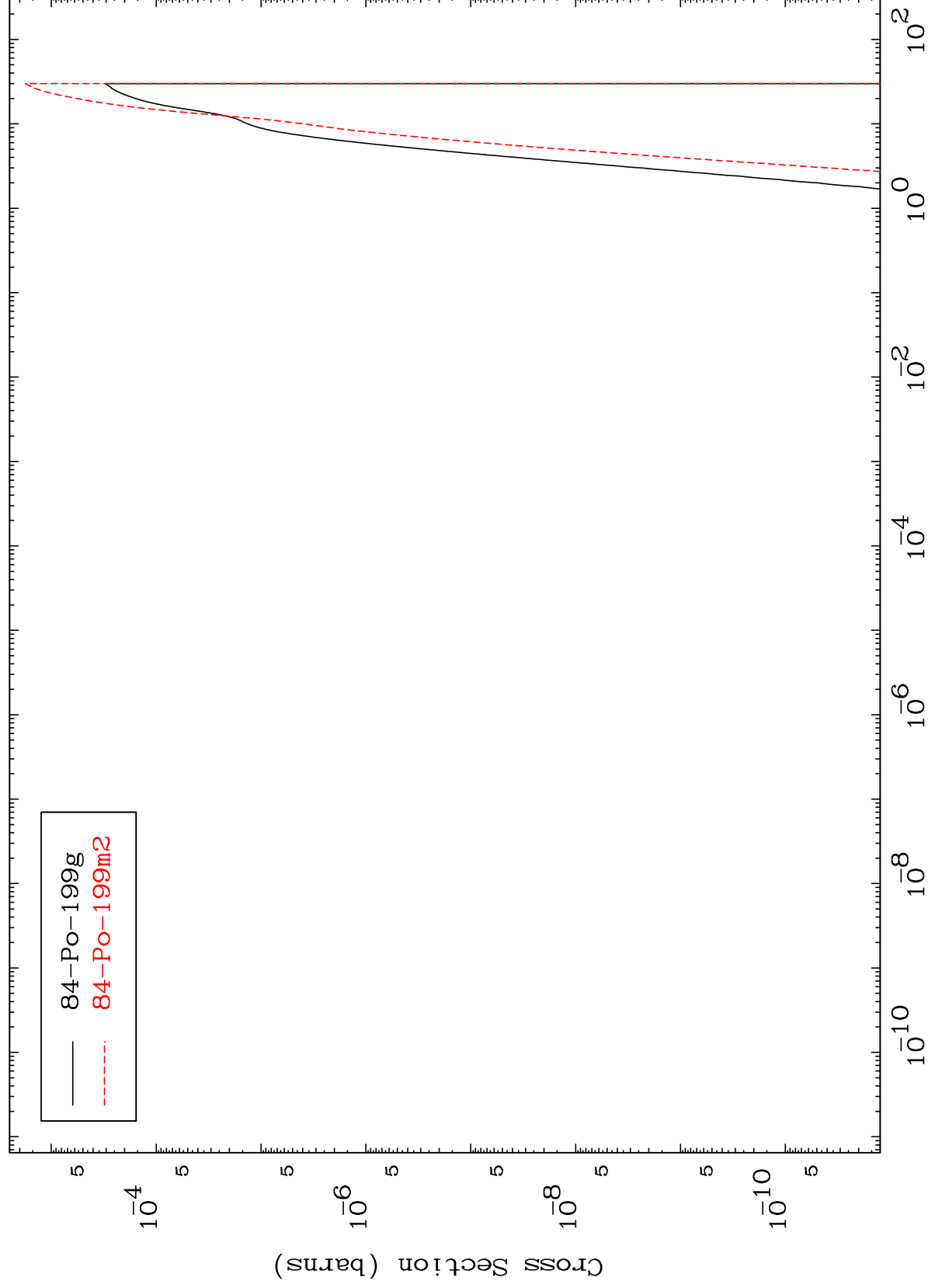
Incident Energy (MeV)

44

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86-Rn-200

Radionuclide Production Cross Section
(n,2p)



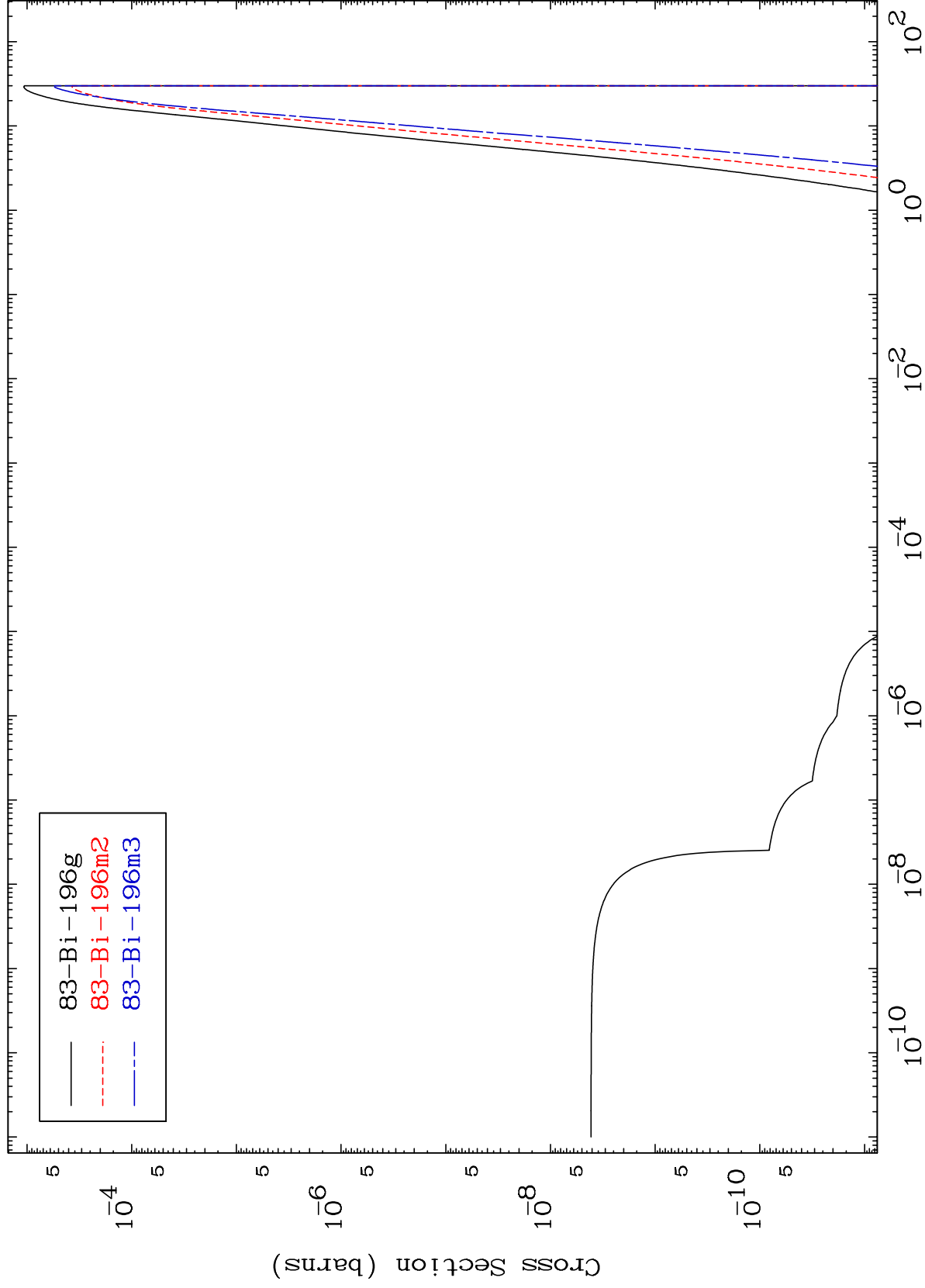
45

86-Rn-200

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86-Rn-200

(n,p) α
Radionuclide Production Cross Section



46

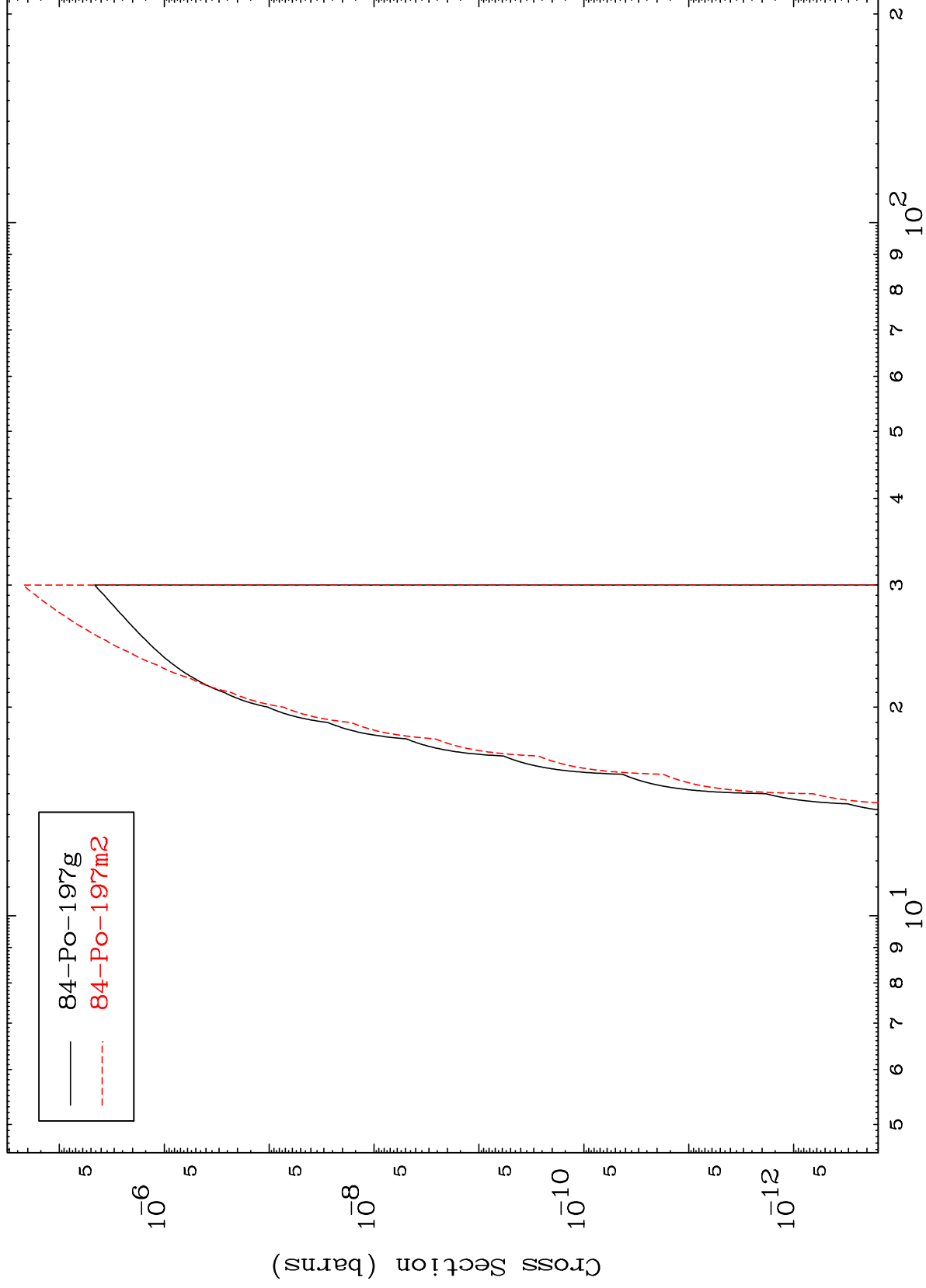
86-Rn-200

MAT 8592

(n,p) t

86-Rn-200

Radionuclide Production Cross Section



47

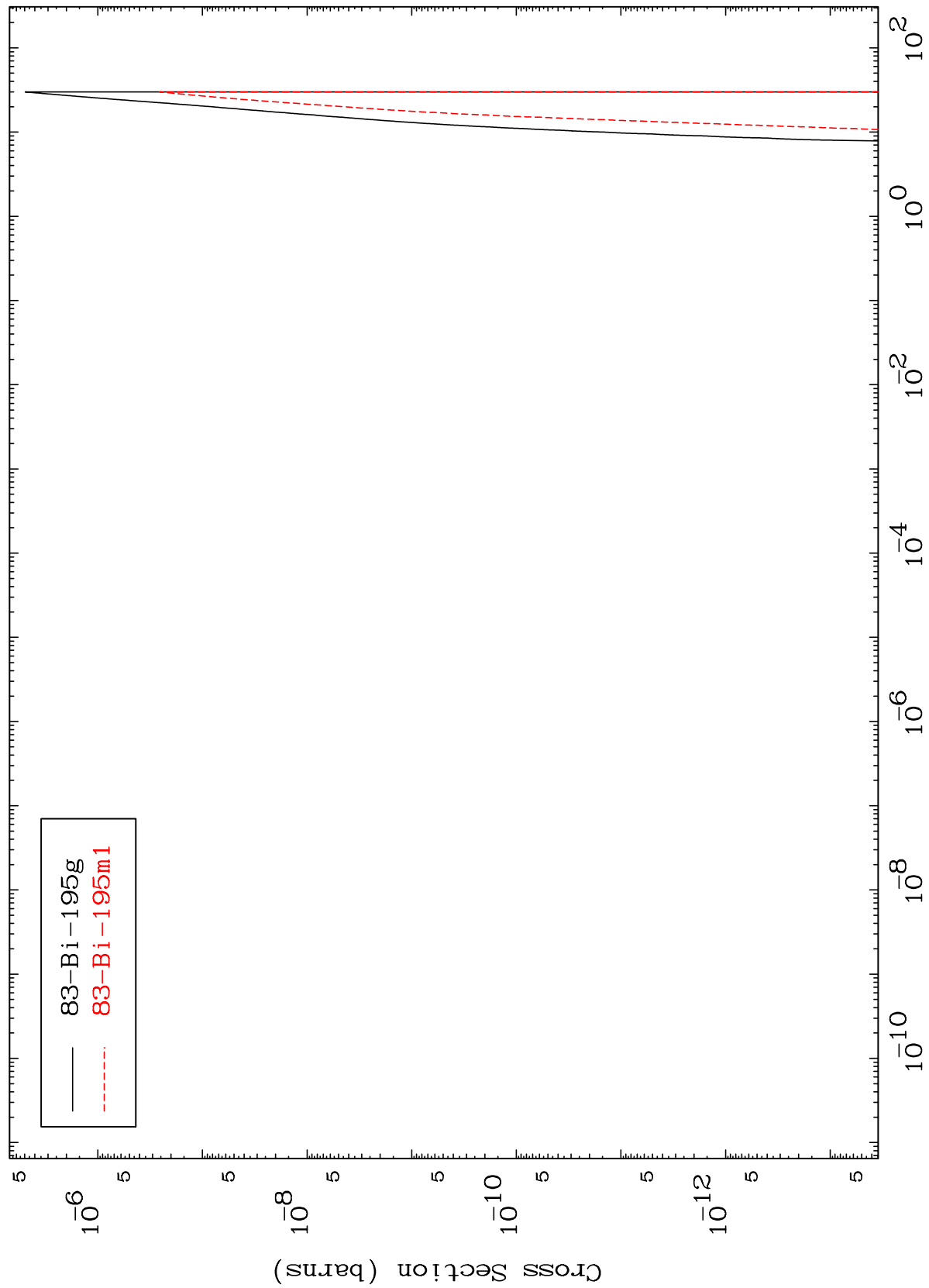
Incident Energy (MeV)

86-Rn-200

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86-Rn-200

(n,d) α
Radionuclide Production Cross Section



48

86-Rn-200

Incident Energy (MeV)