

Program EVALPLOT
(Version 2021-1)

by

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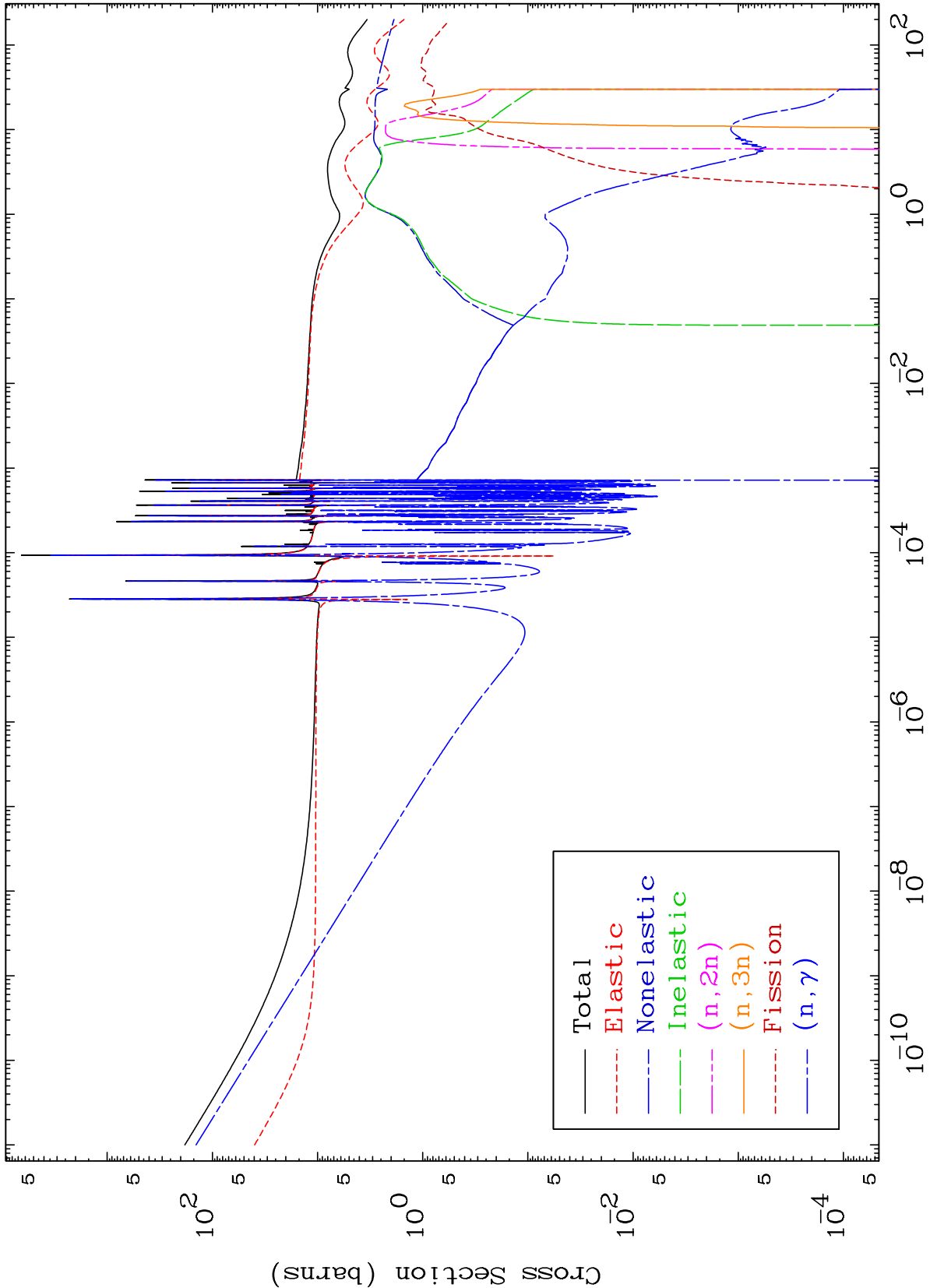
Press Mouse Button to Start

MAT 9052

Neutron Major

293 Kelvin Cross Sections

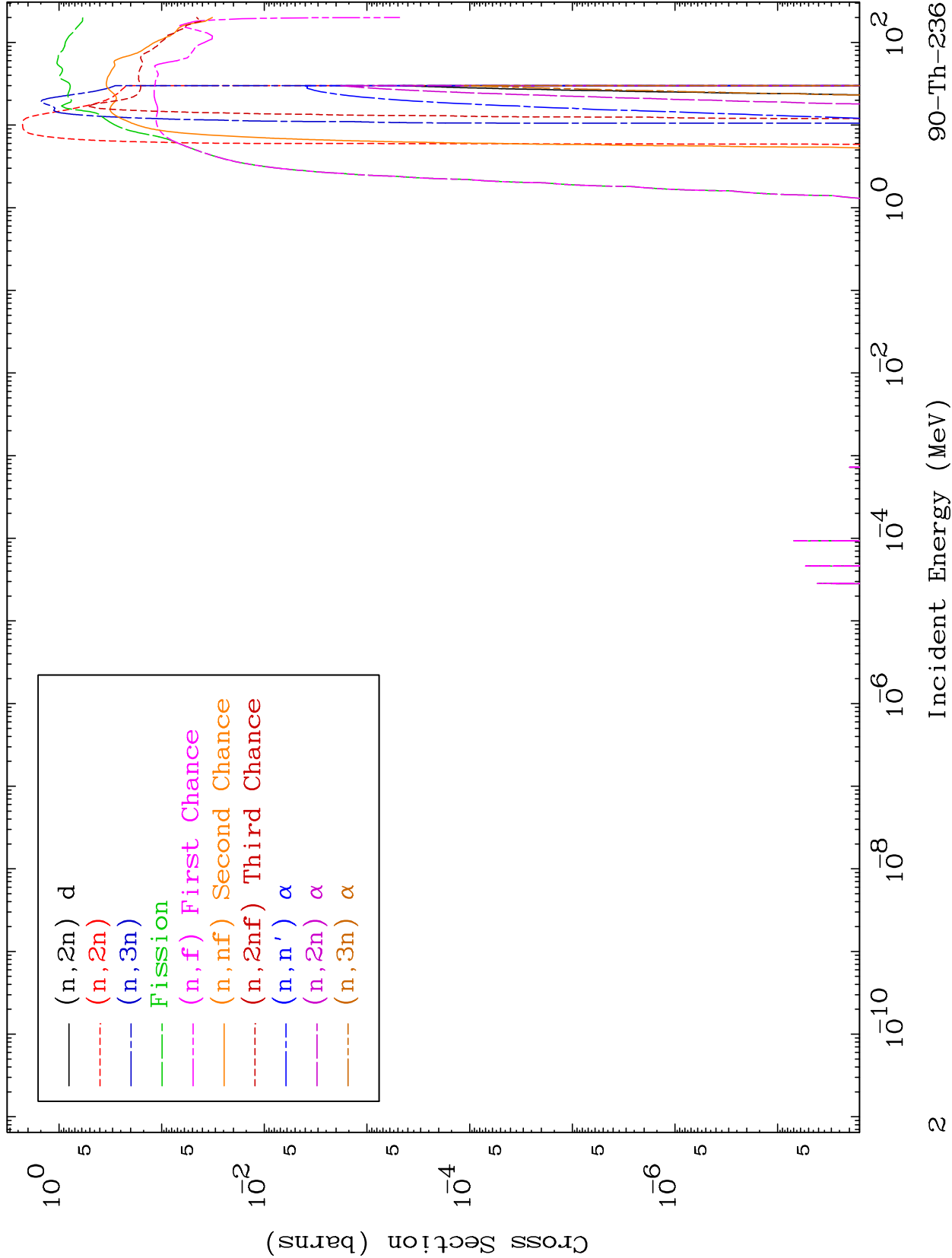
90-Th-236

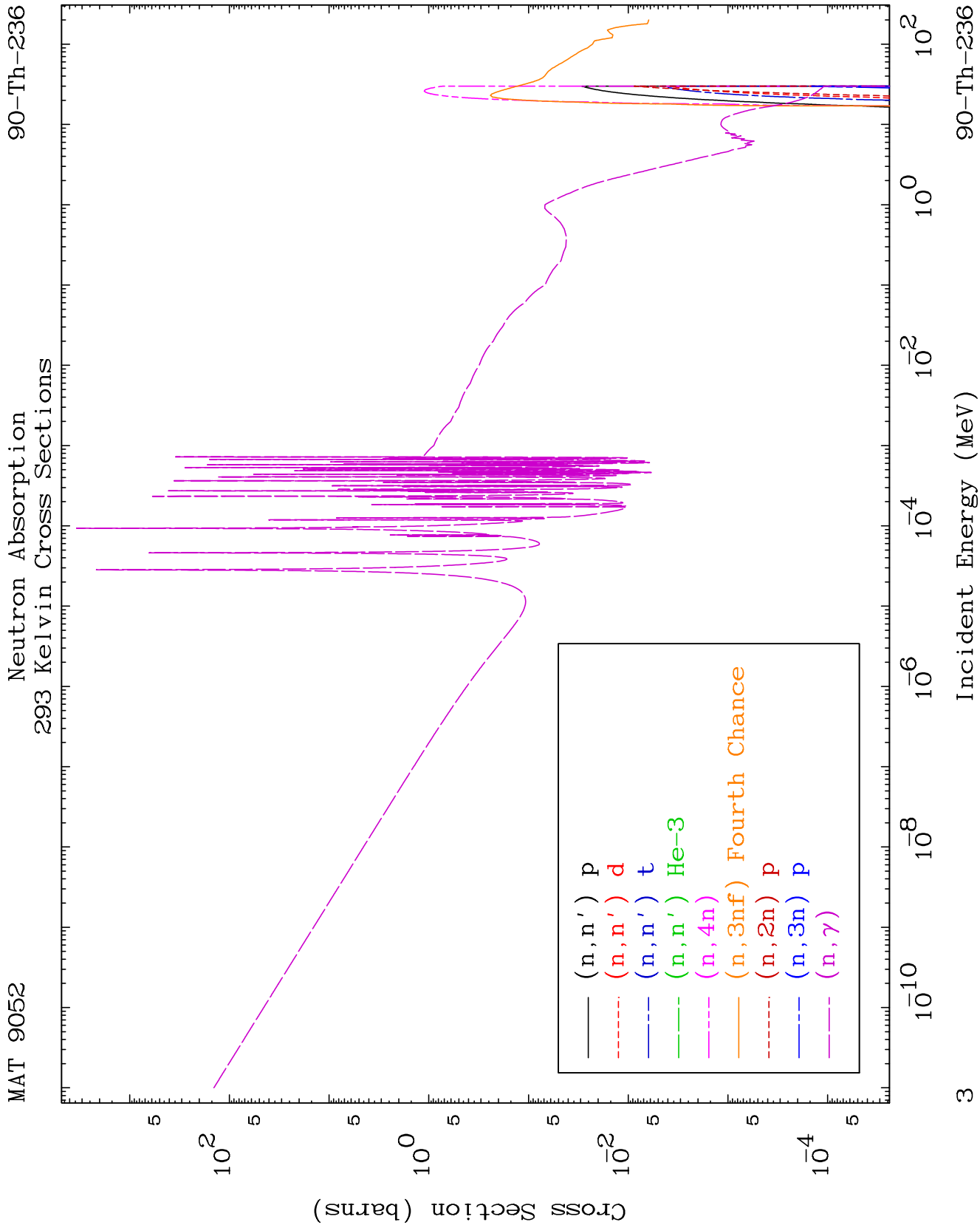


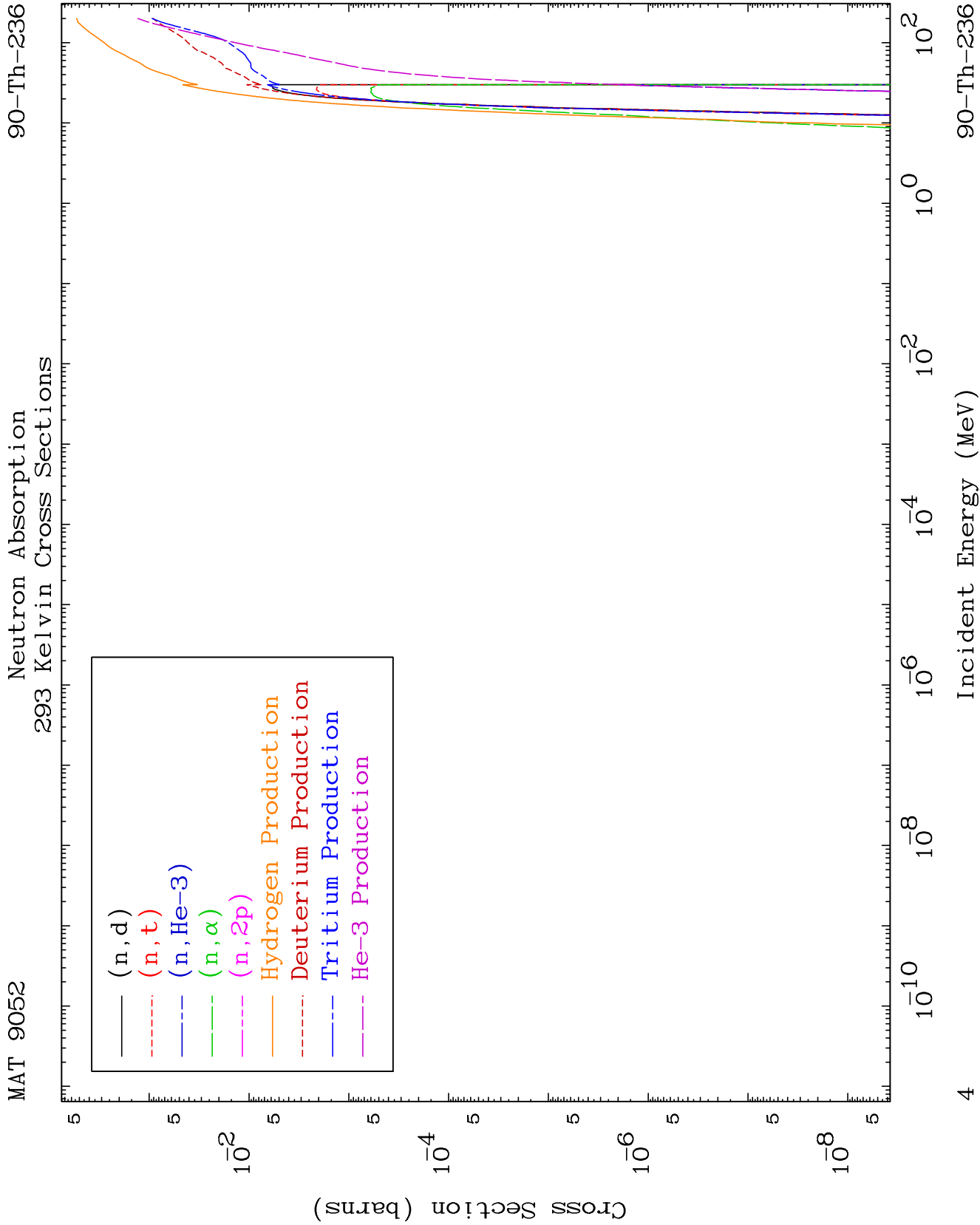
MAT 9052

Neutron Absorption
293 Kelvin Cross Sections

90-Th-236



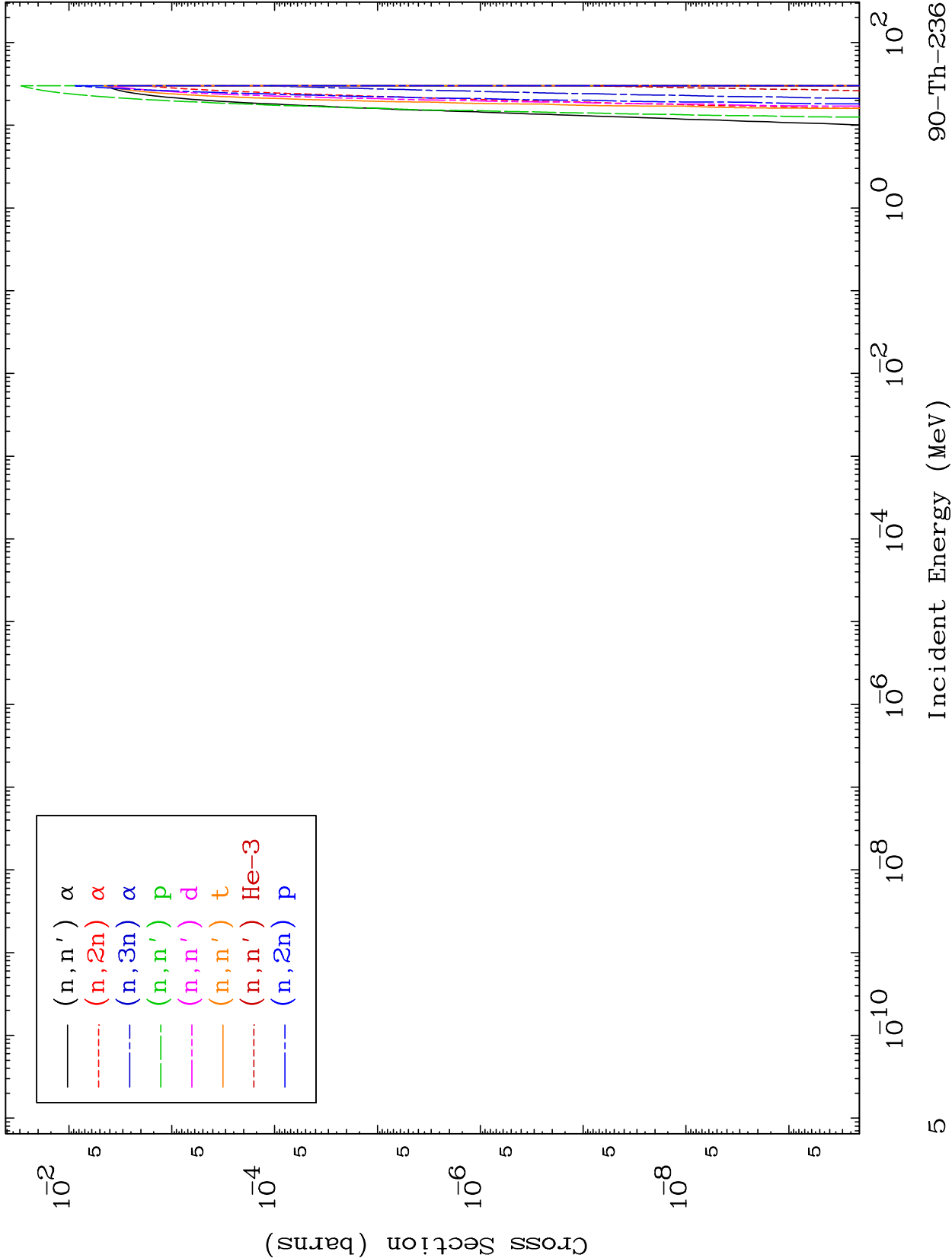


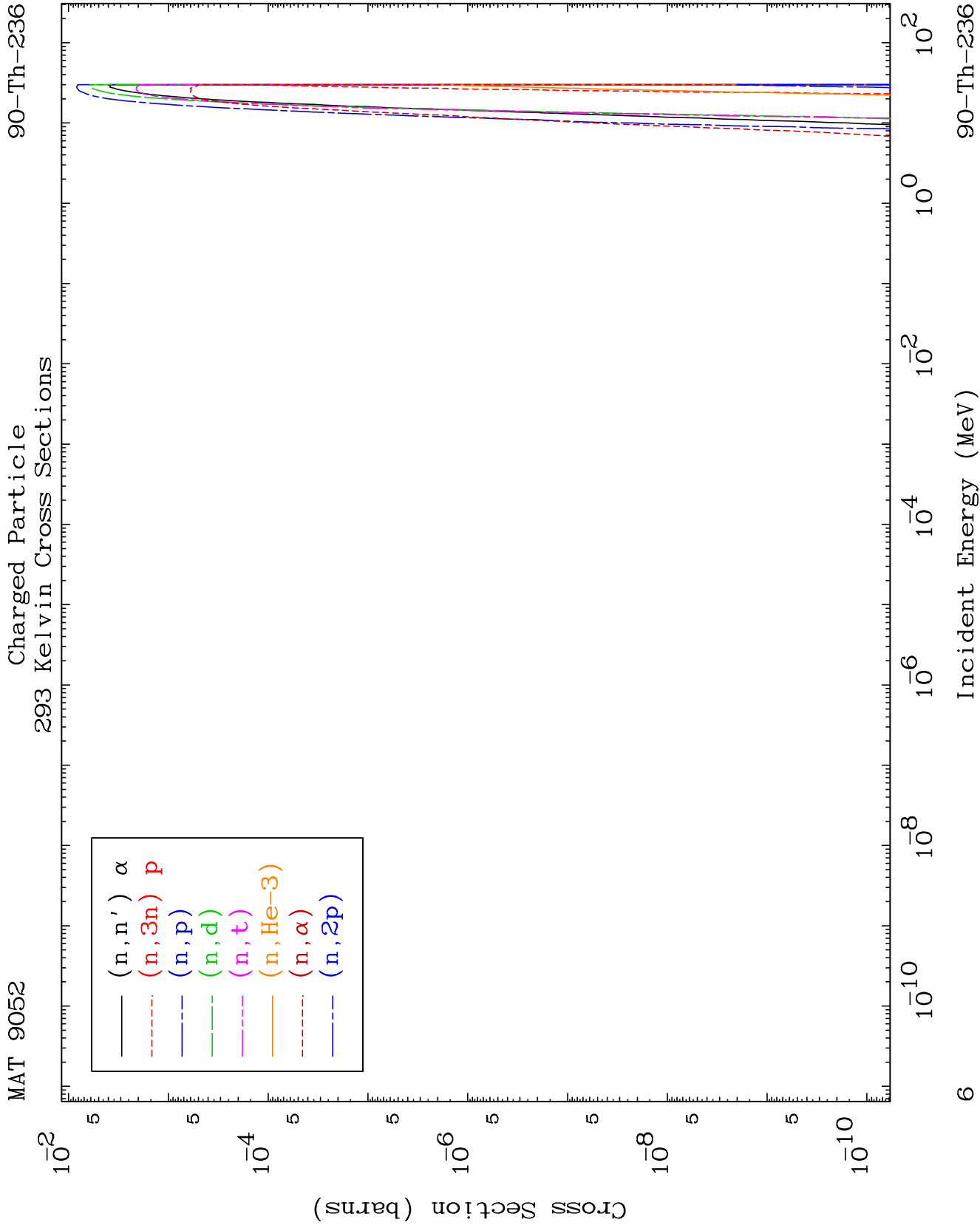


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Charged Particle
293 Kelvin Cross Sections

90-Th-236

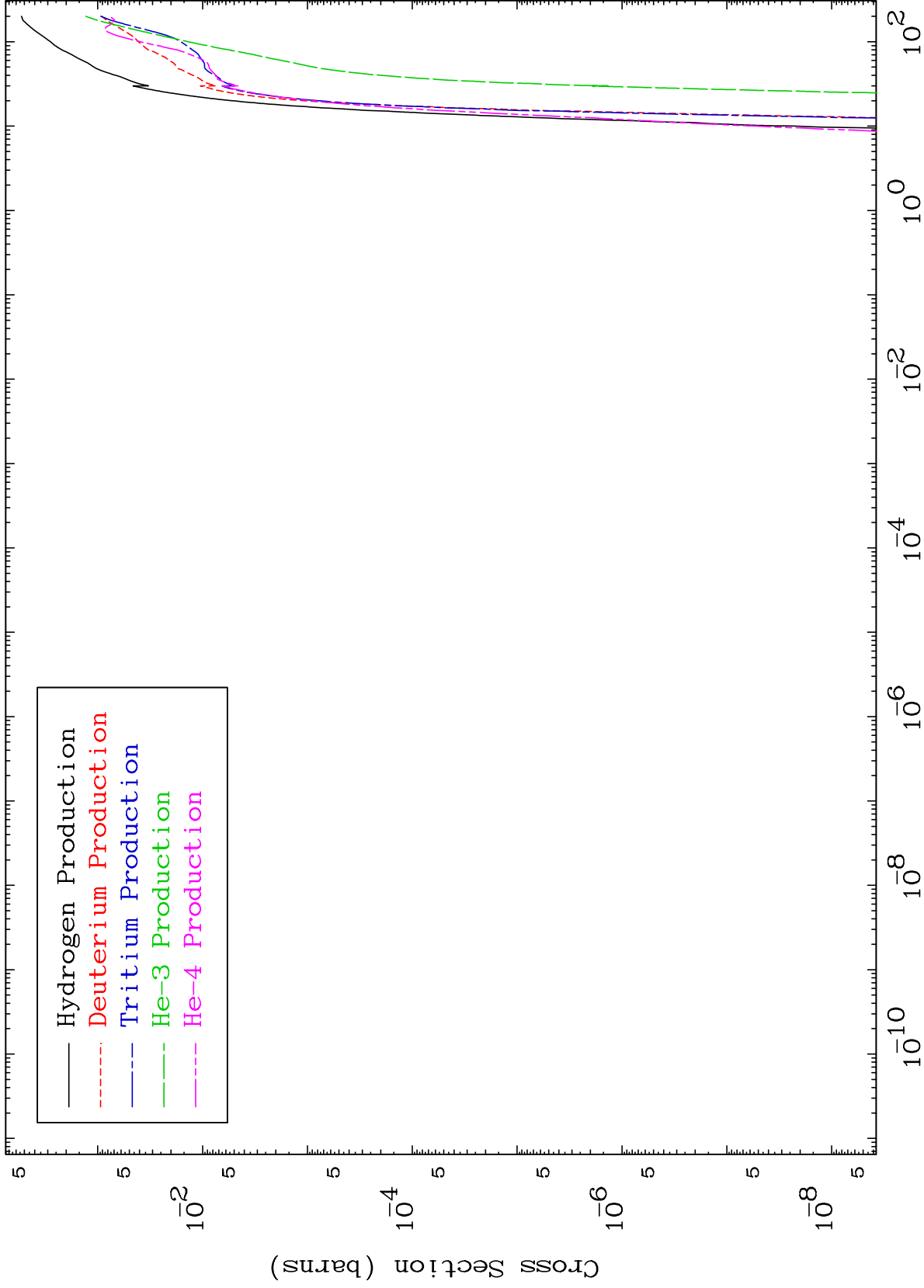


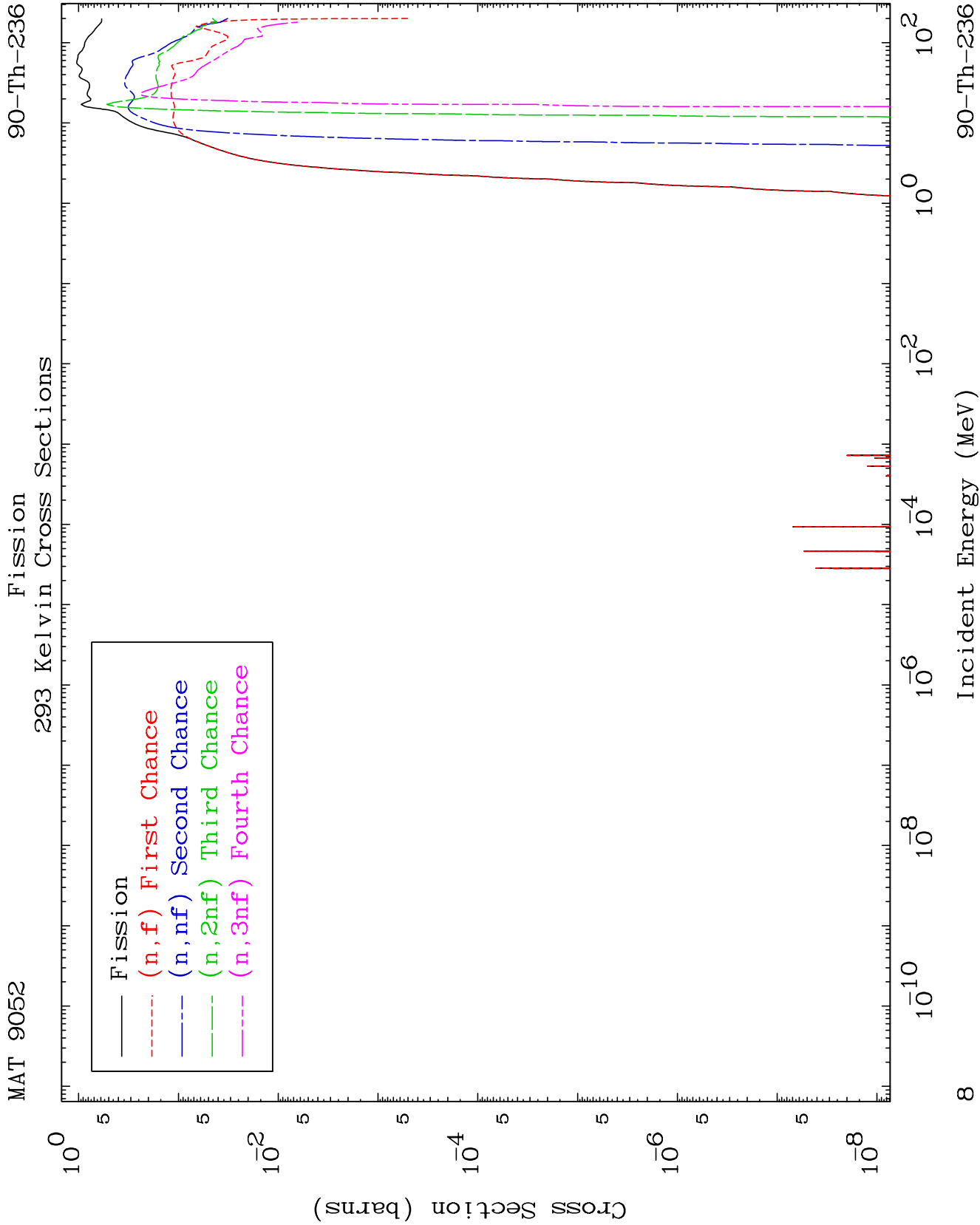


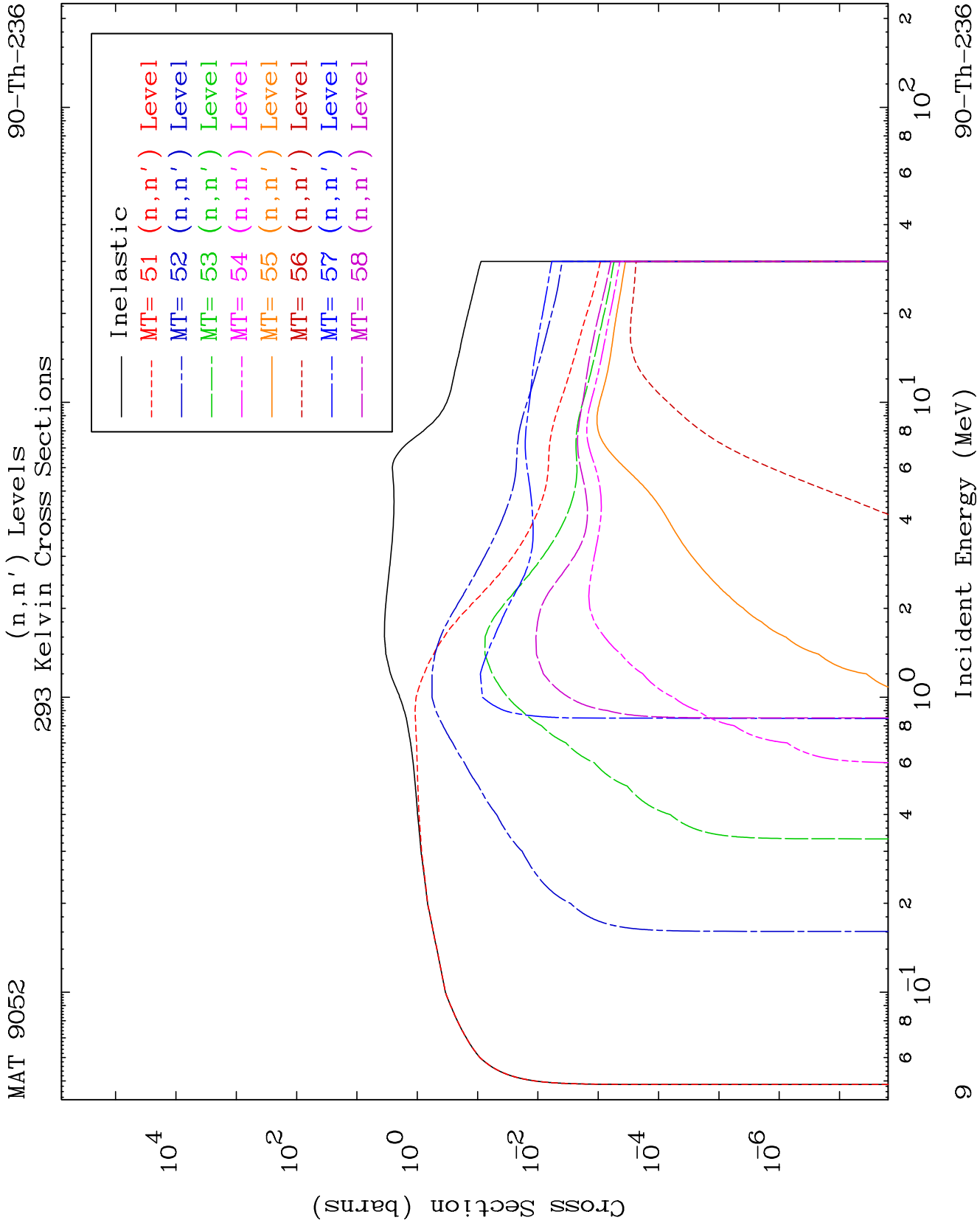
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Particle Production
293 Kelvin Cross Sections

90-Th-236





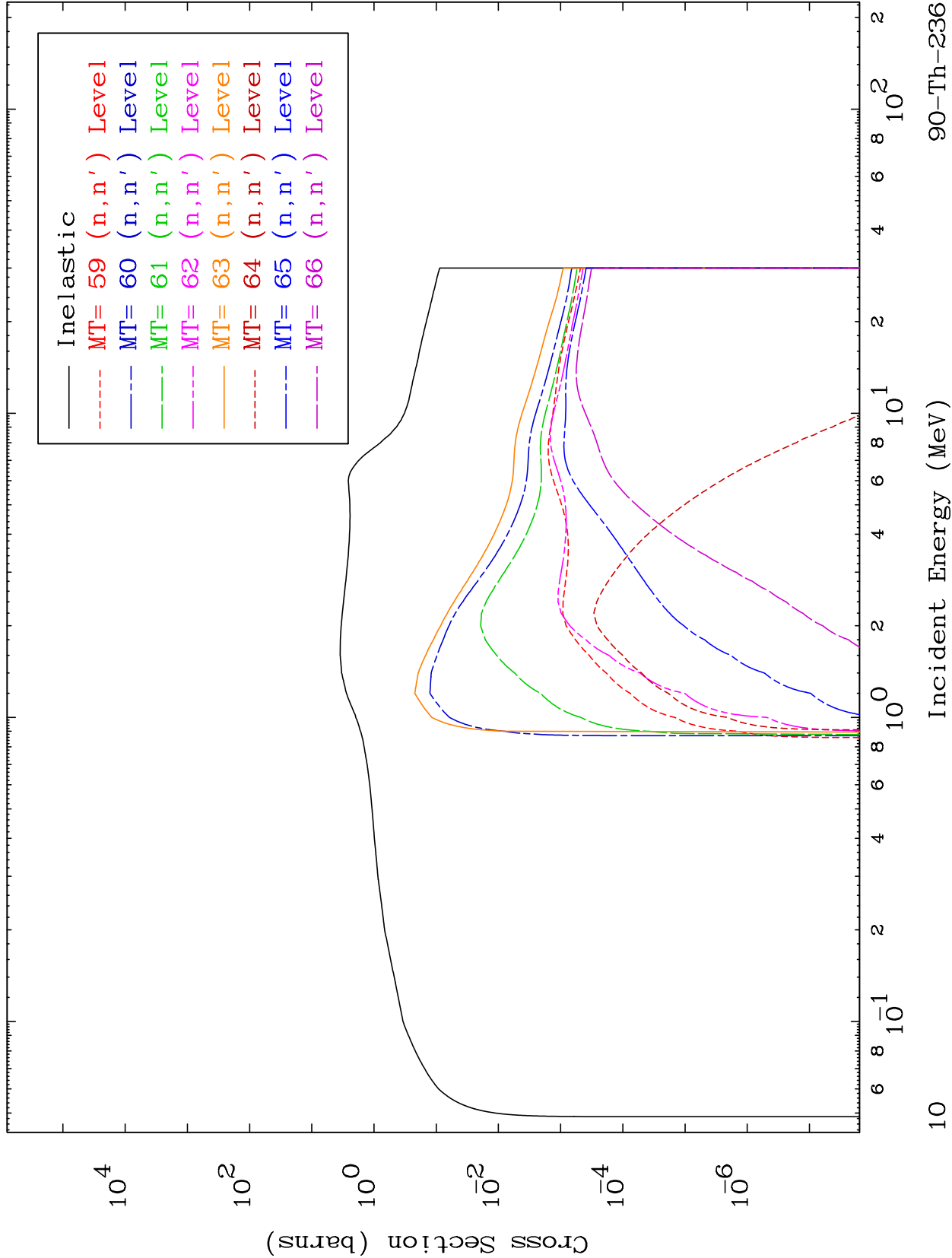


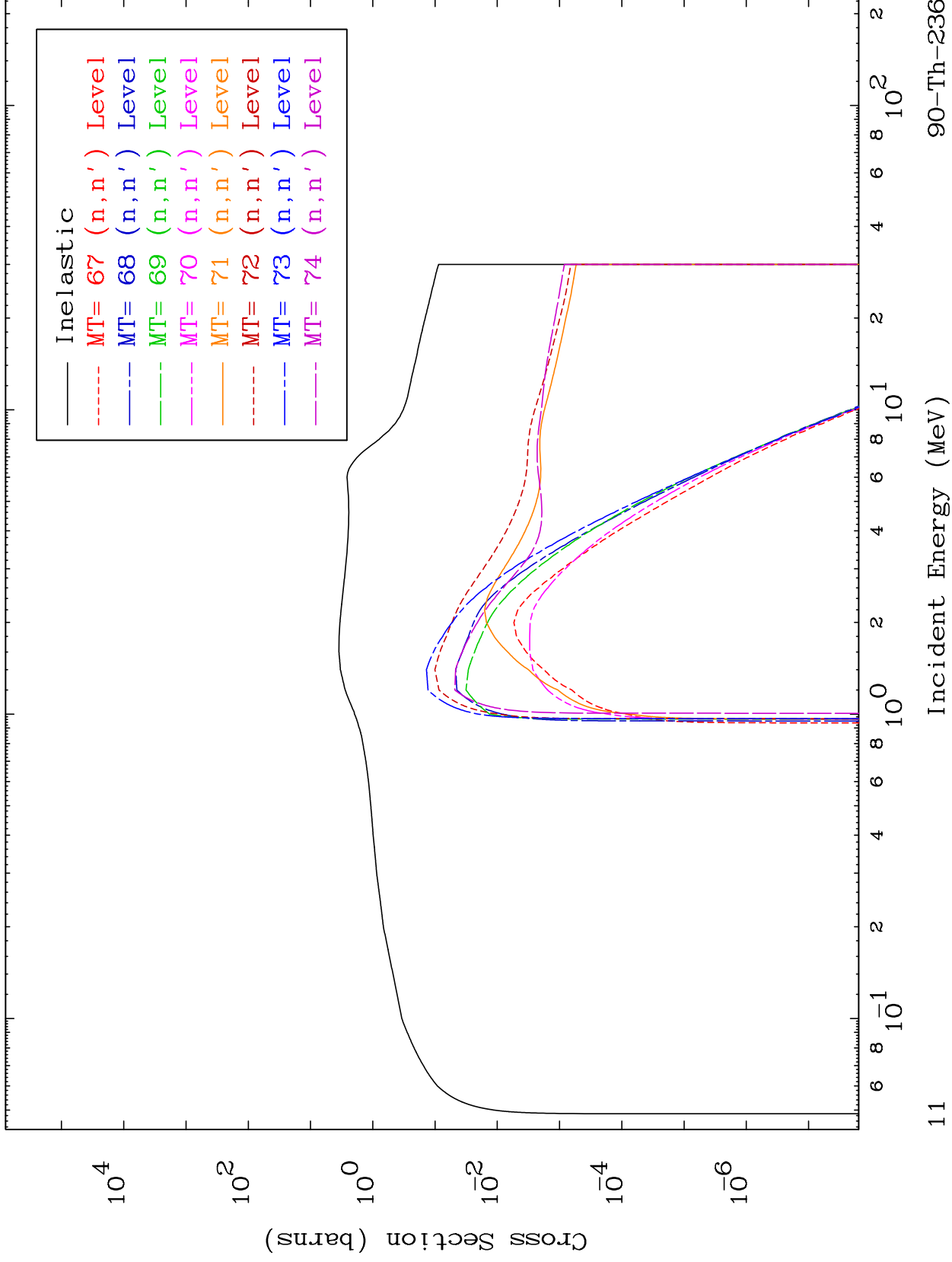
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(n,n') Levels

293 Kelvin Cross Sections

90-Th-236



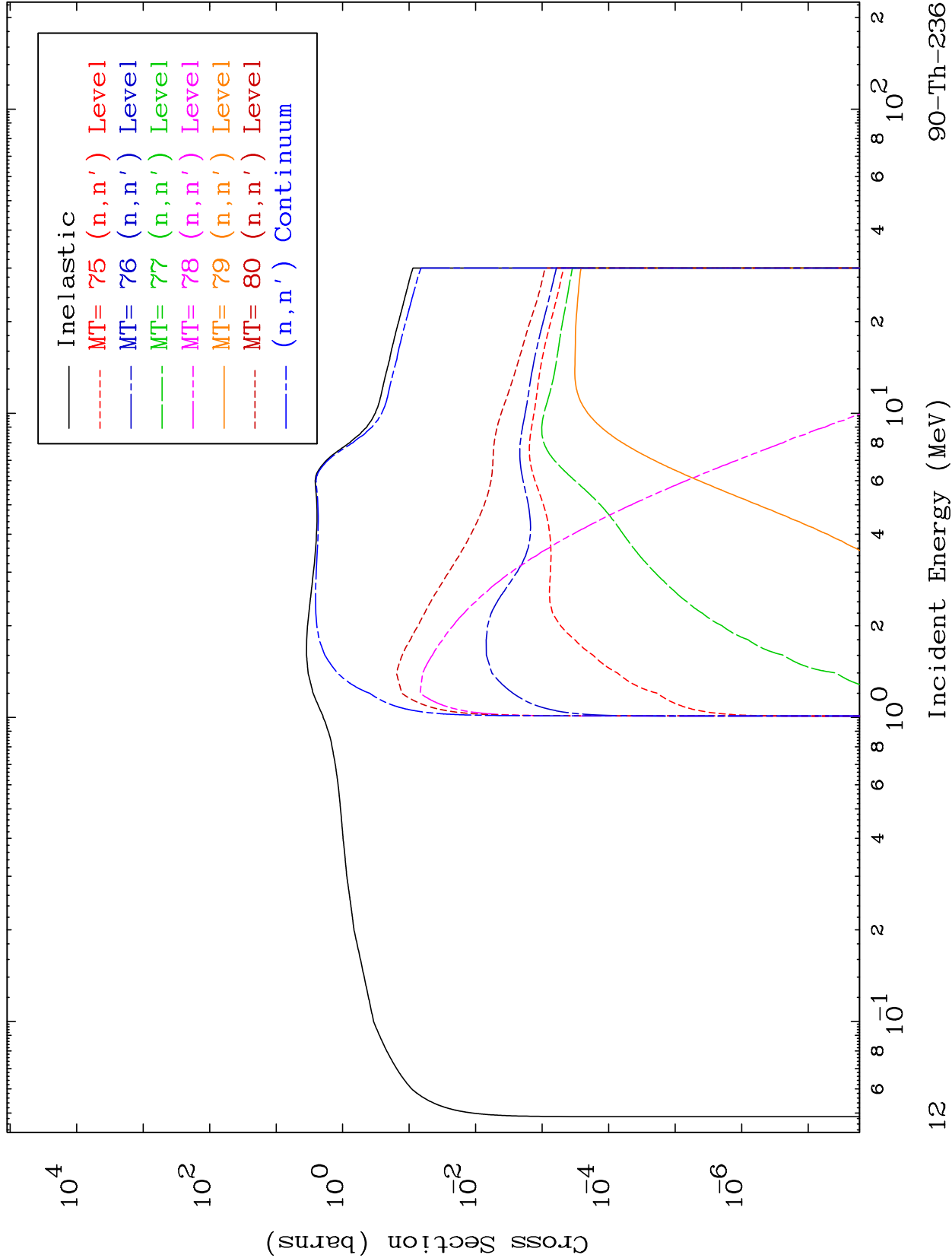


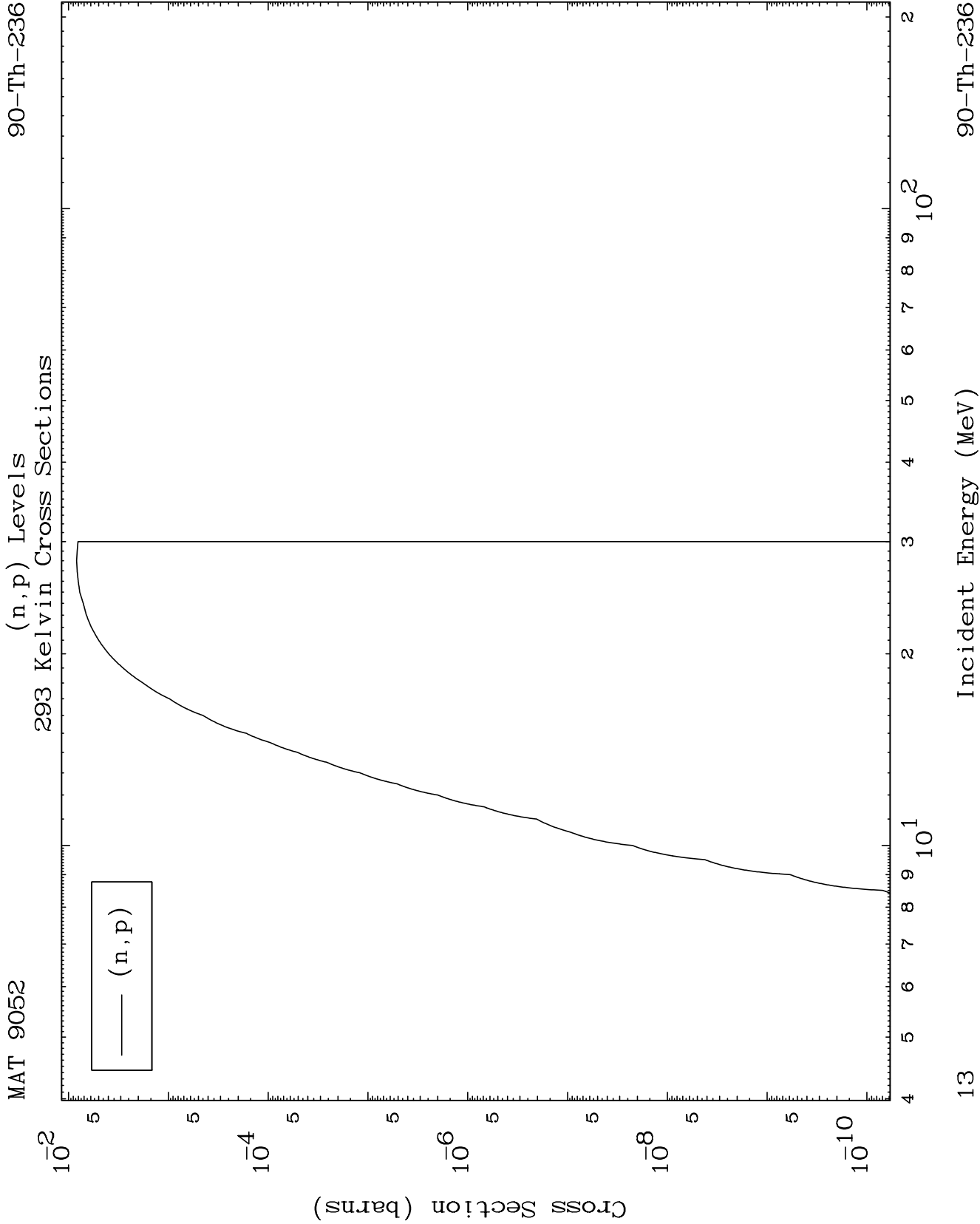
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(n,n') Levels

293 Kelvin Cross Sections

90-Th-236

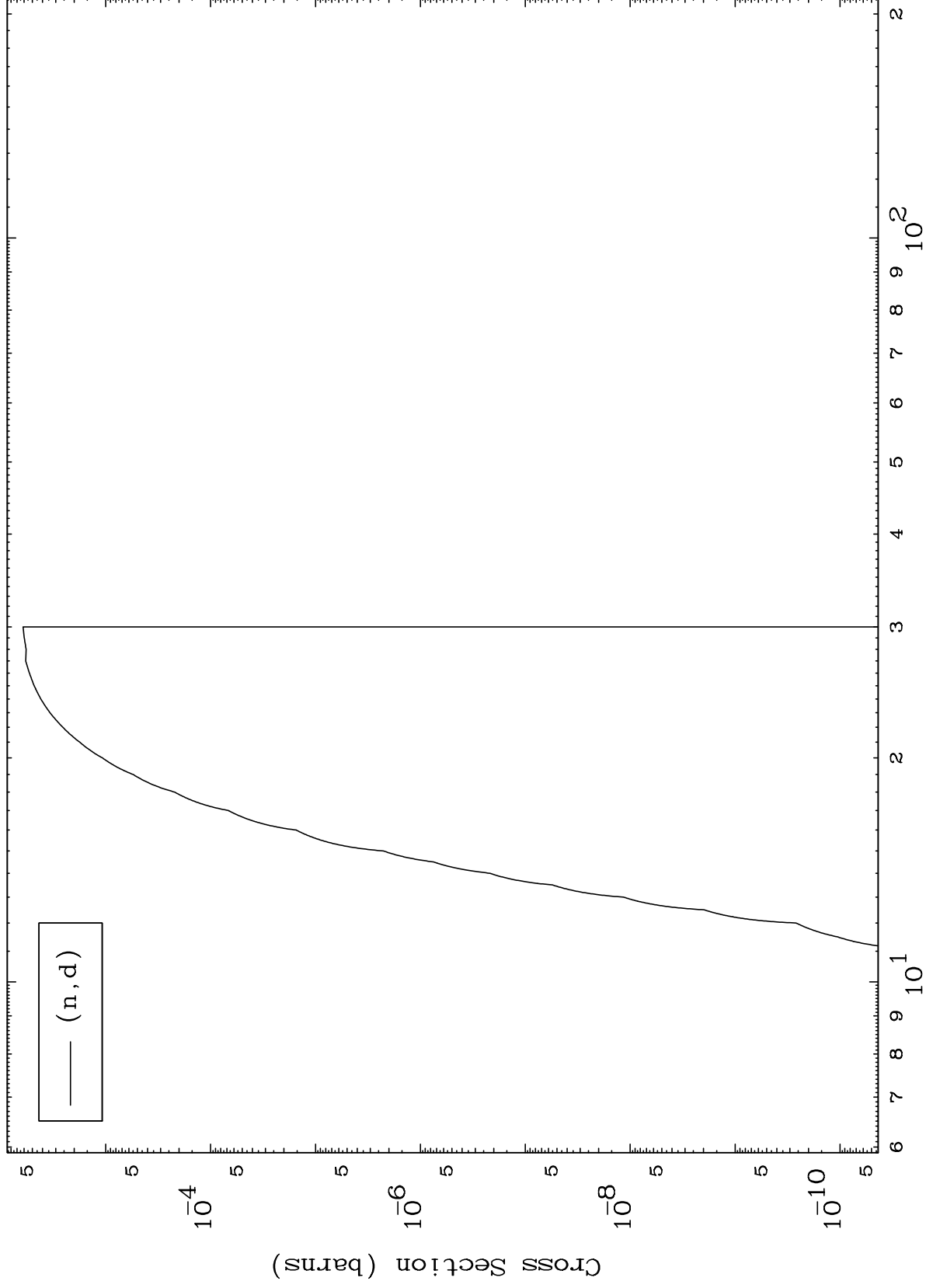




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(n,d) Levels
293 Kelvin Cross Sections



90-Th-236

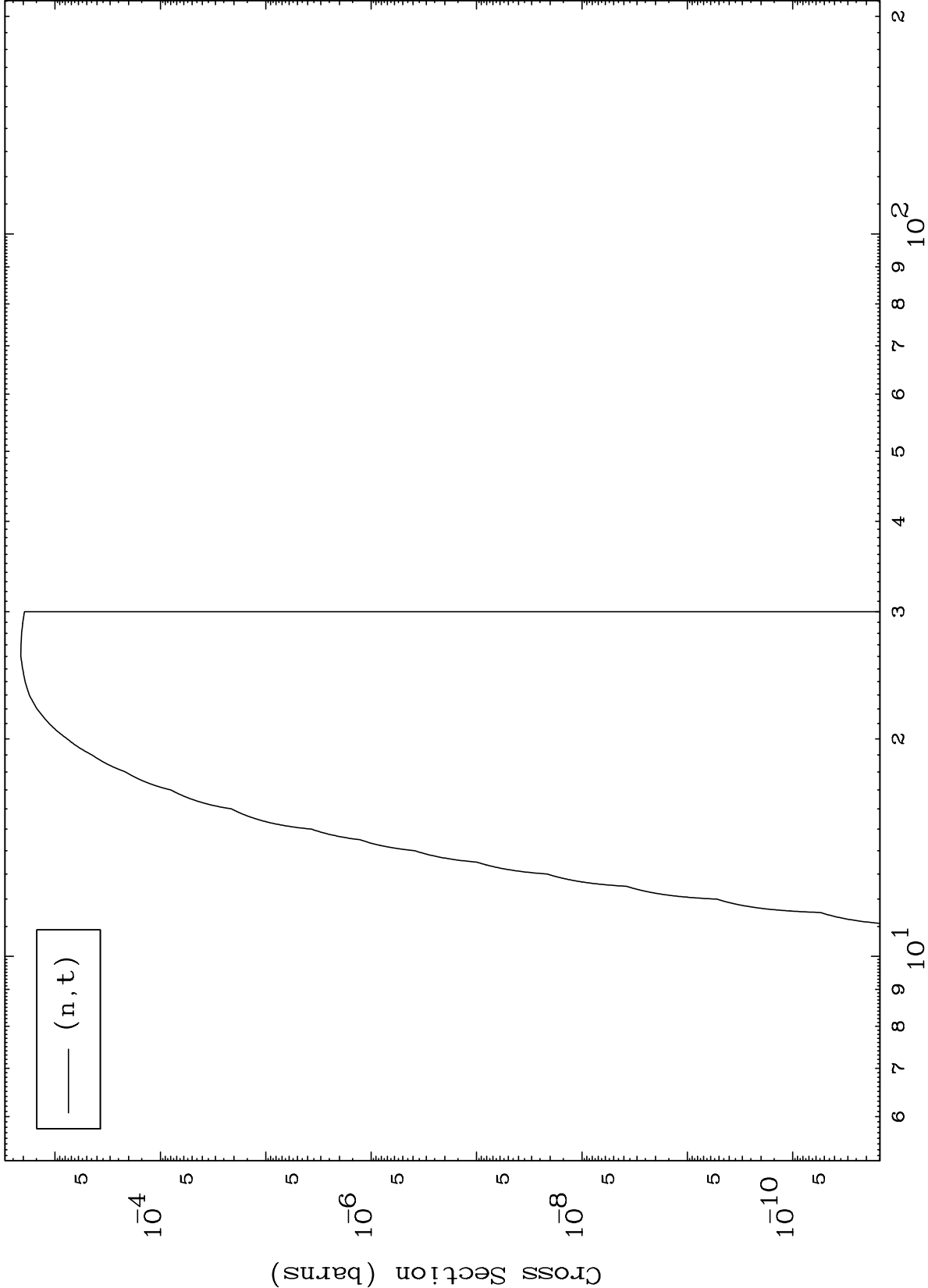
Incident Energy (MeV)

14

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(n,t) Levels
293 Kelvin Cross Sections

90-Th-236



15

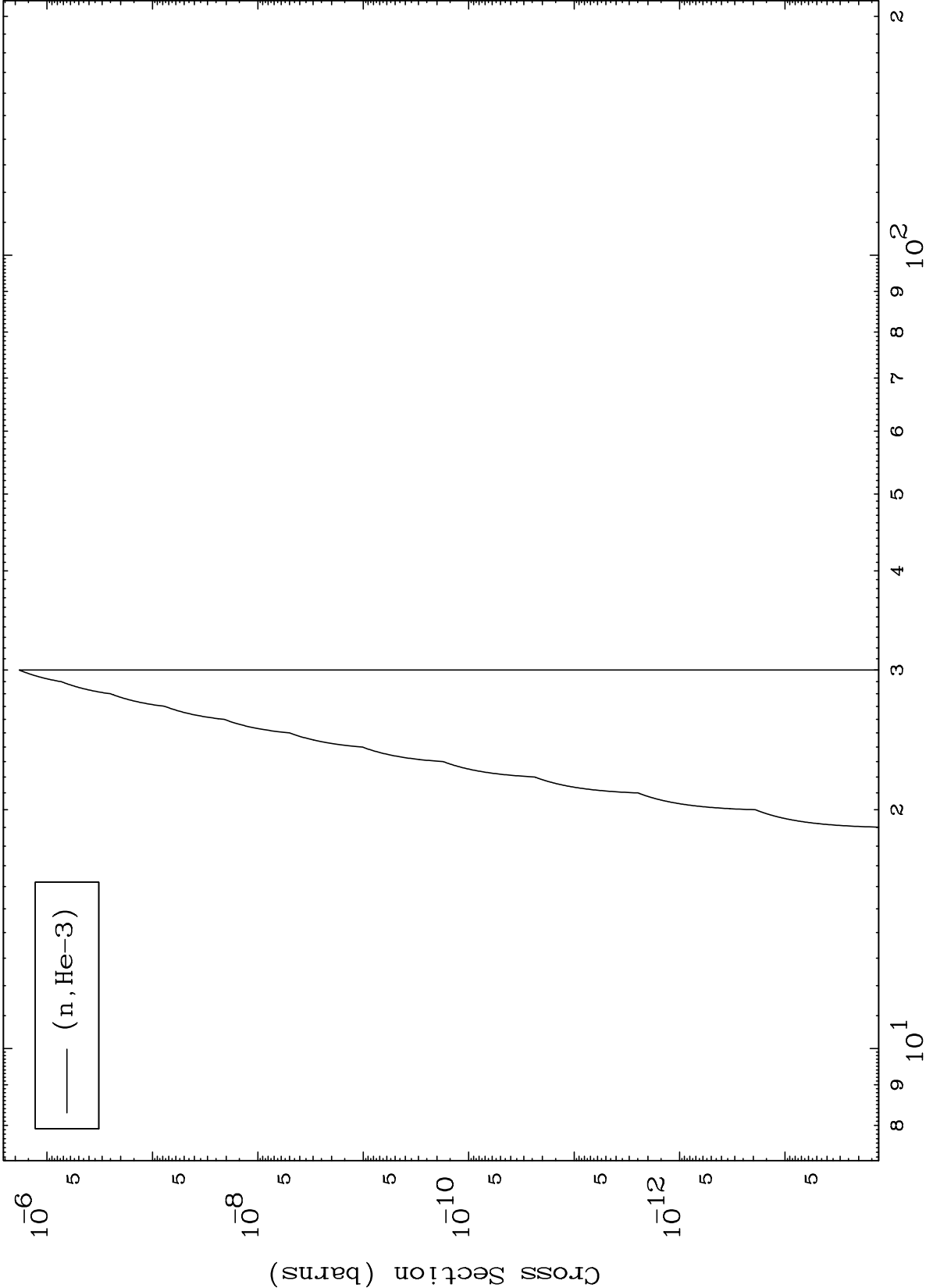
Incident Energy (MeV)

90-Th-236

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(n,He3) Levels
293 Kelvin Cross Sections

90-Th-236



16

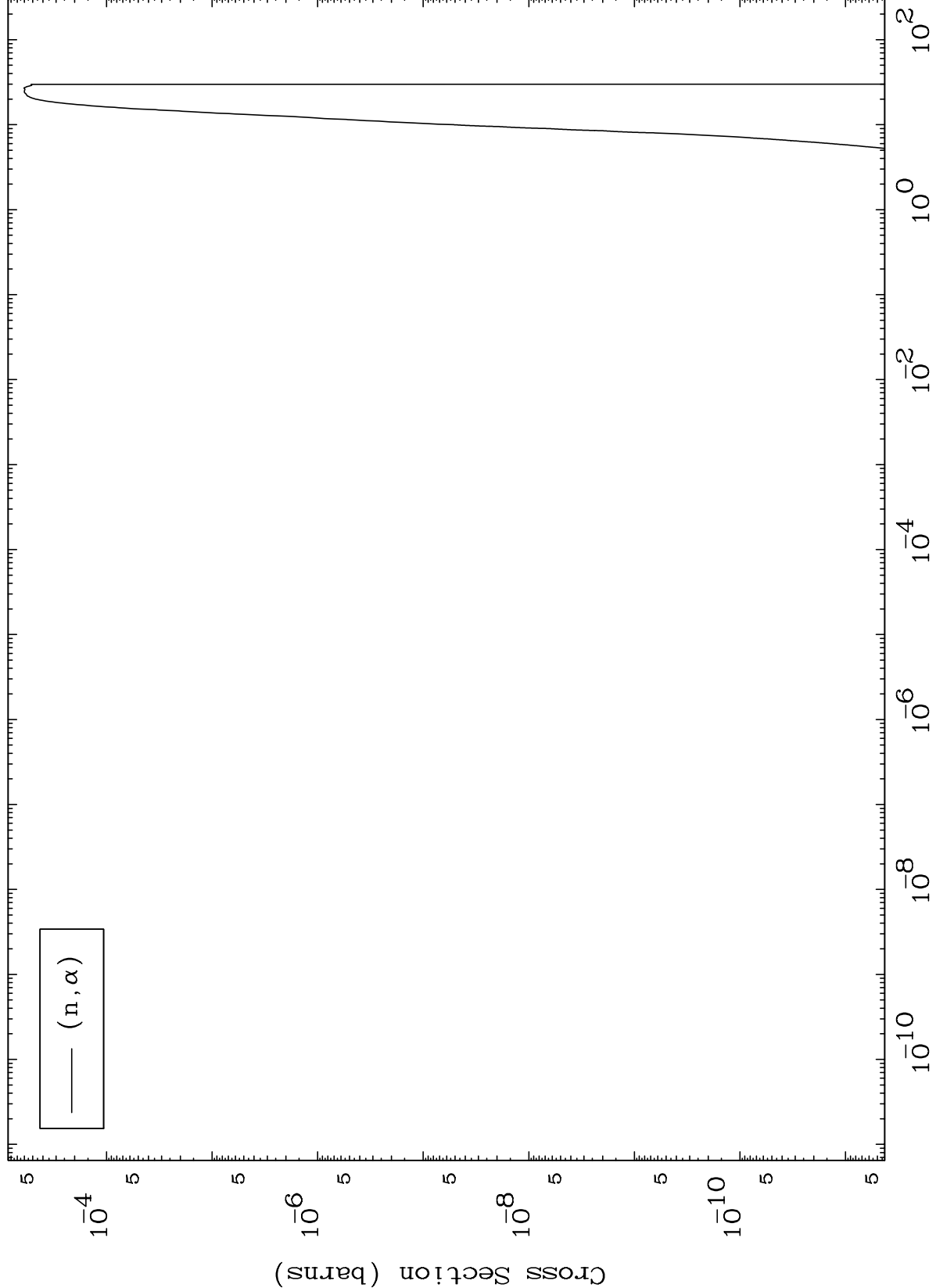
Incident Energy (MeV)

90-Th-236

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(n, α) Levels
293 Kelvin Cross Sections

90-Th-236

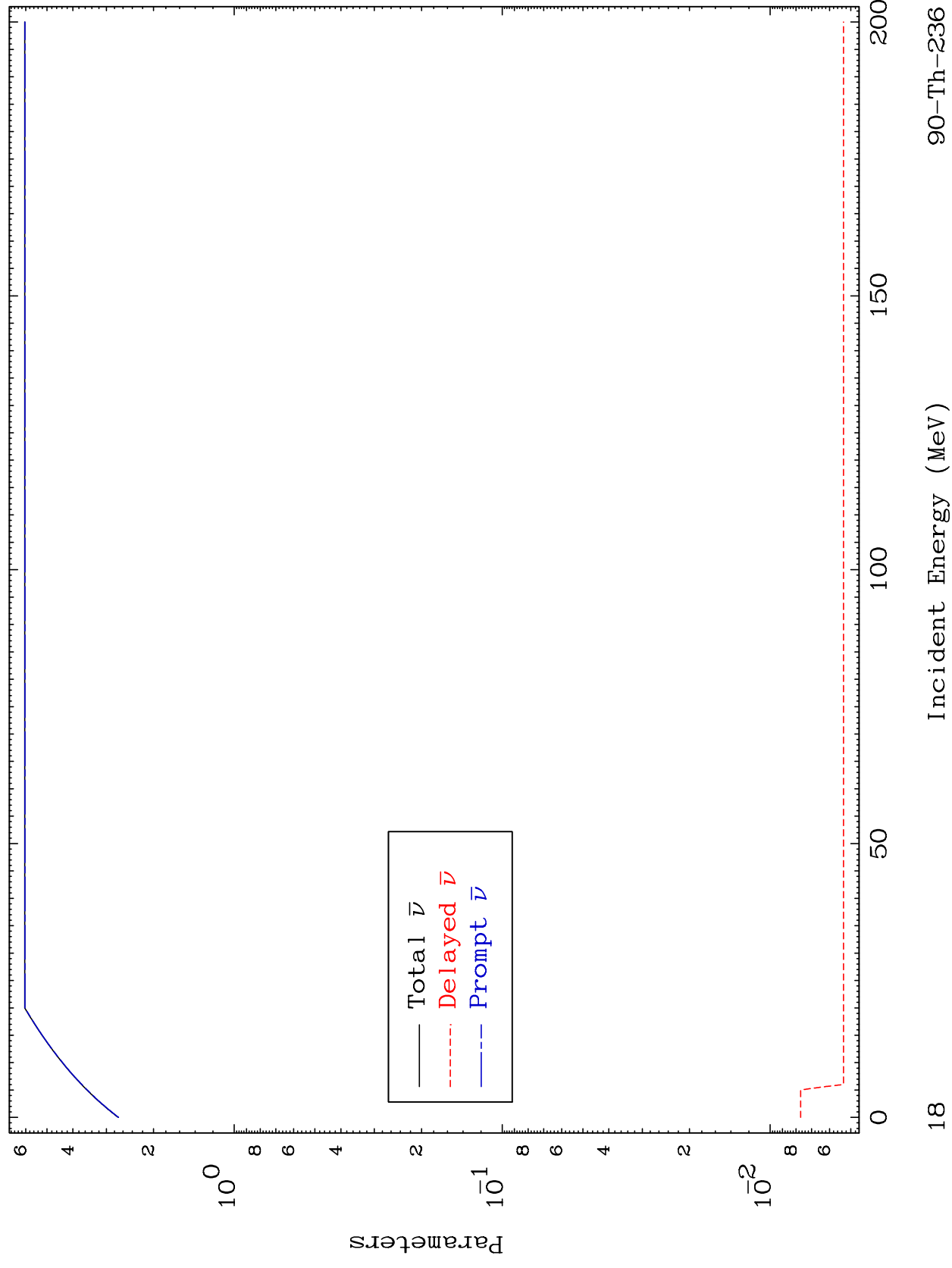


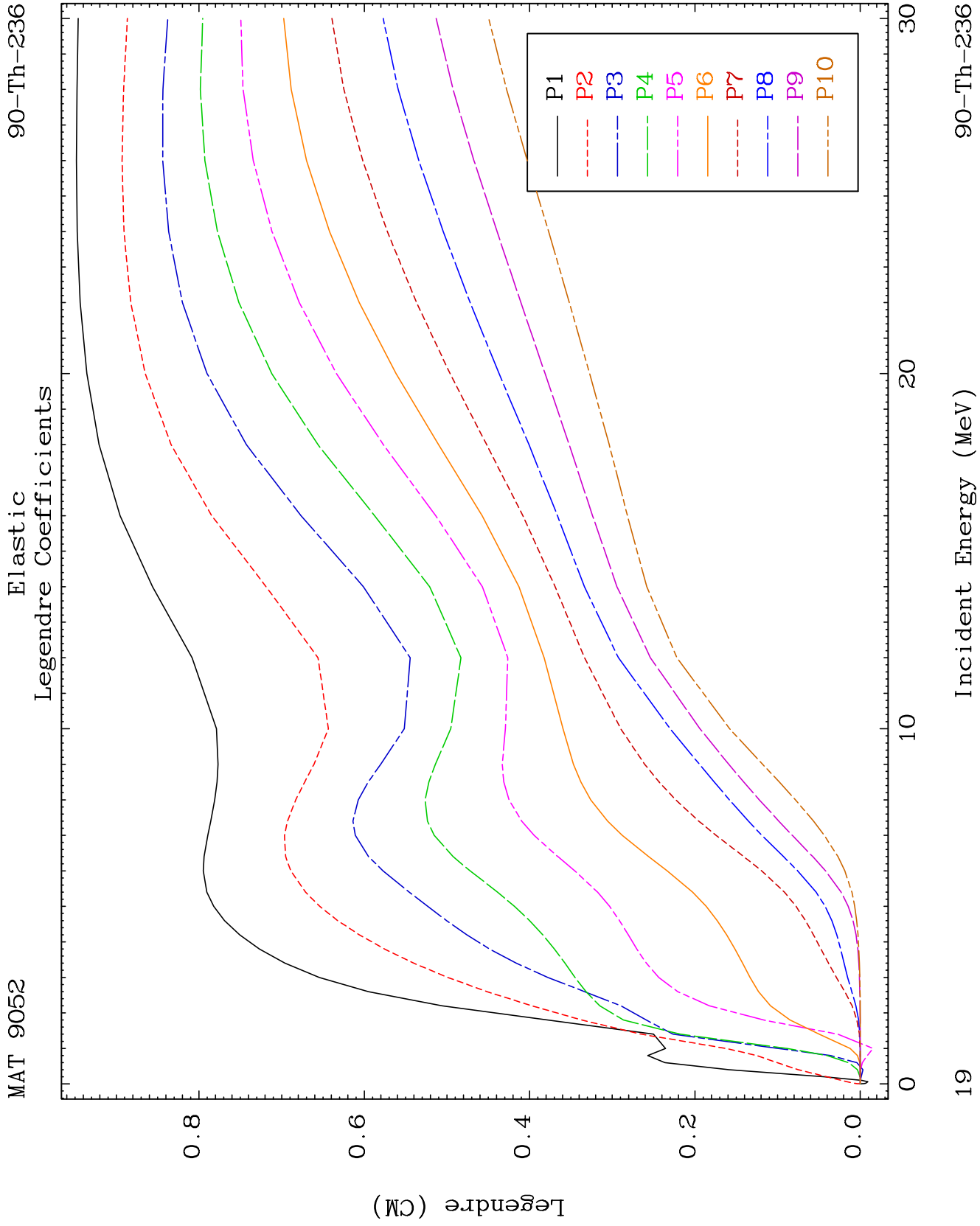
— (n, α)

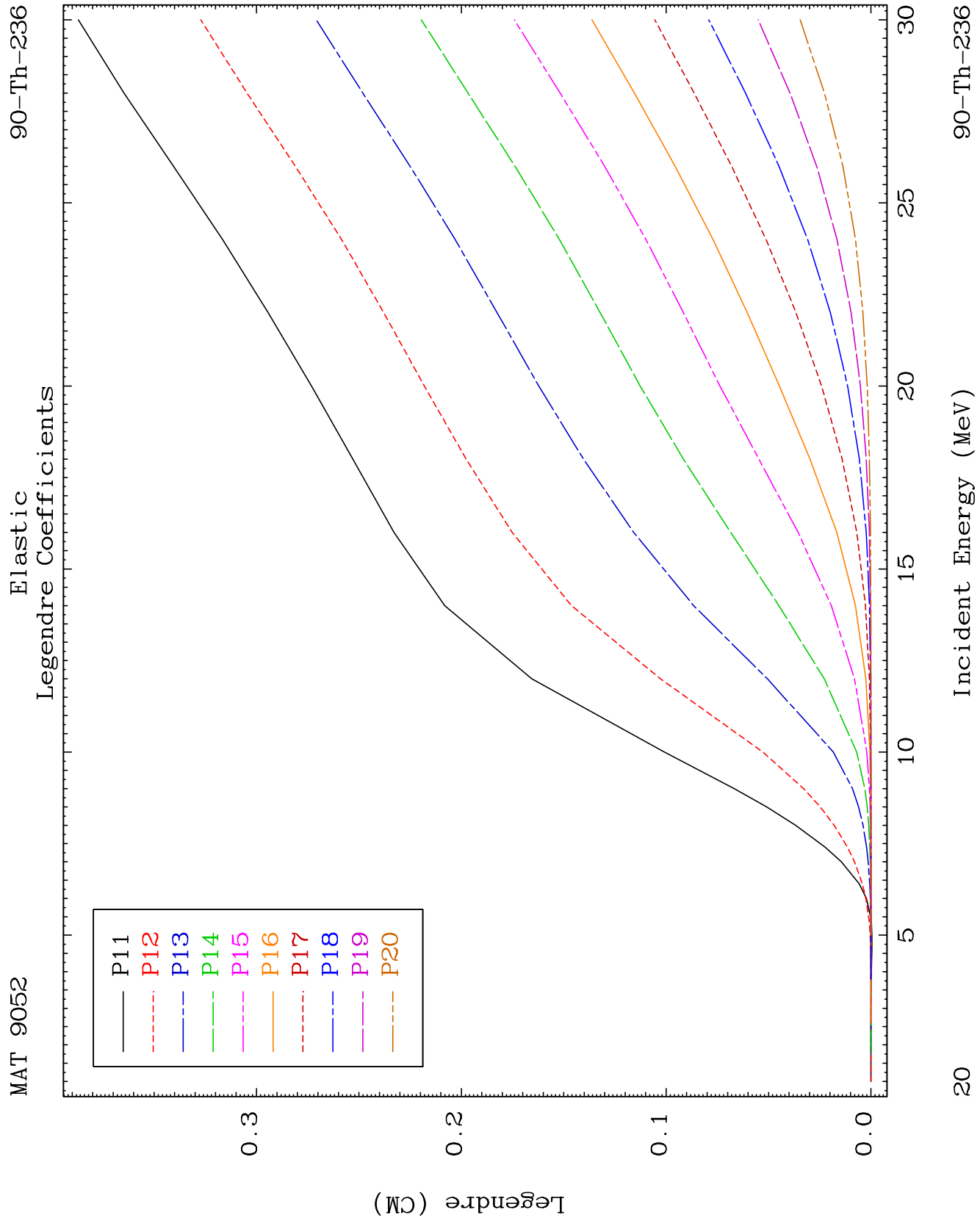
MAT 9052

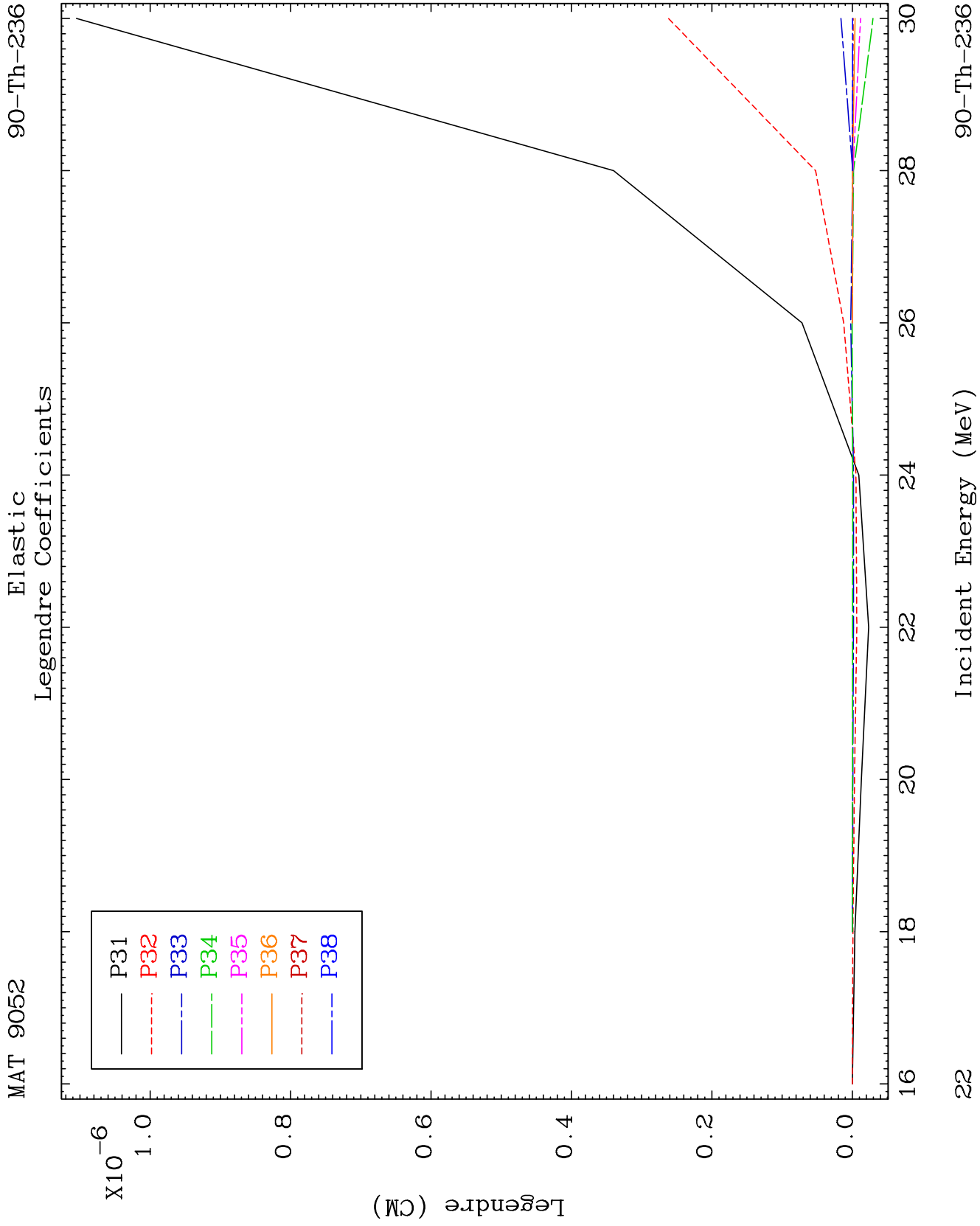
Energy Release Parameters

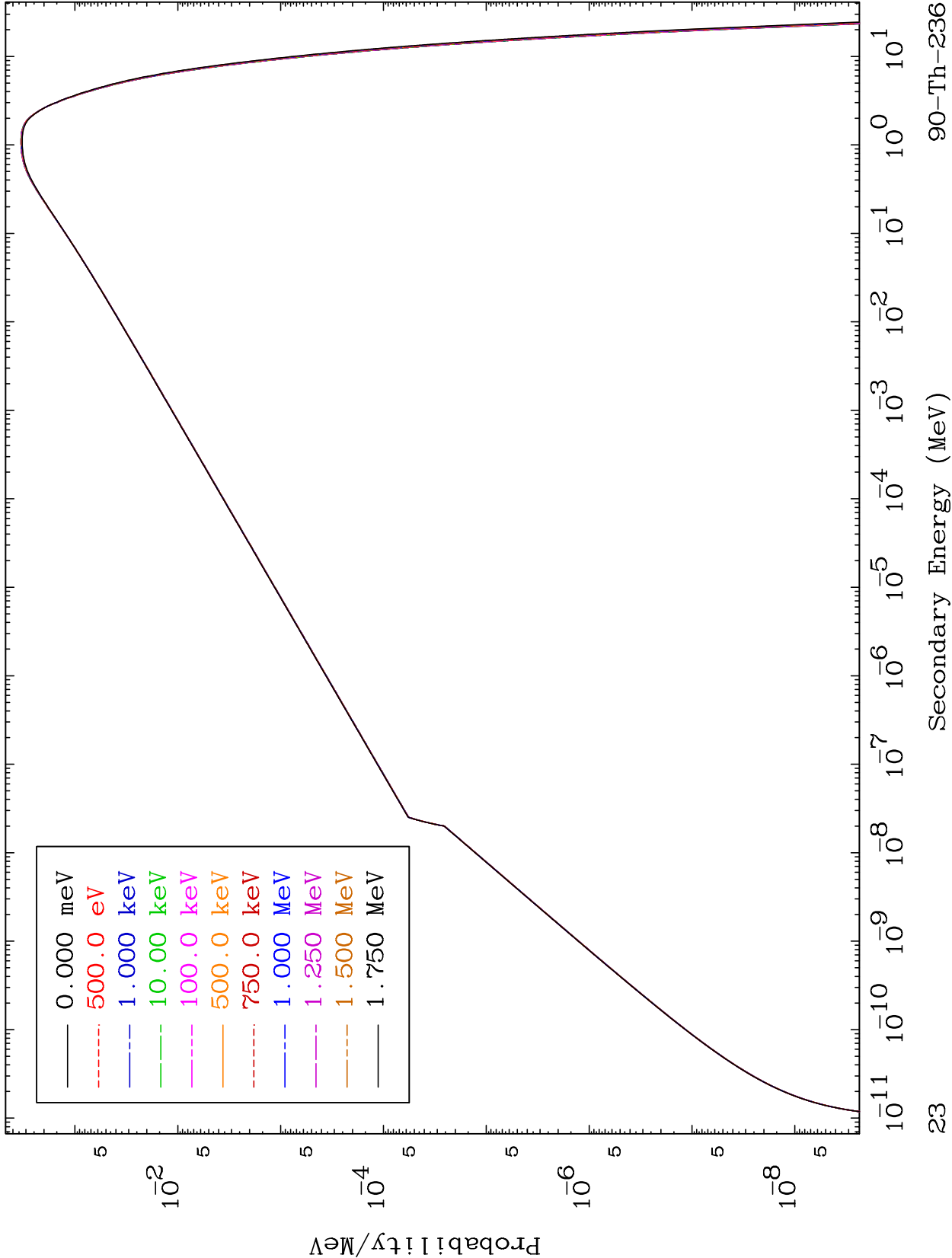
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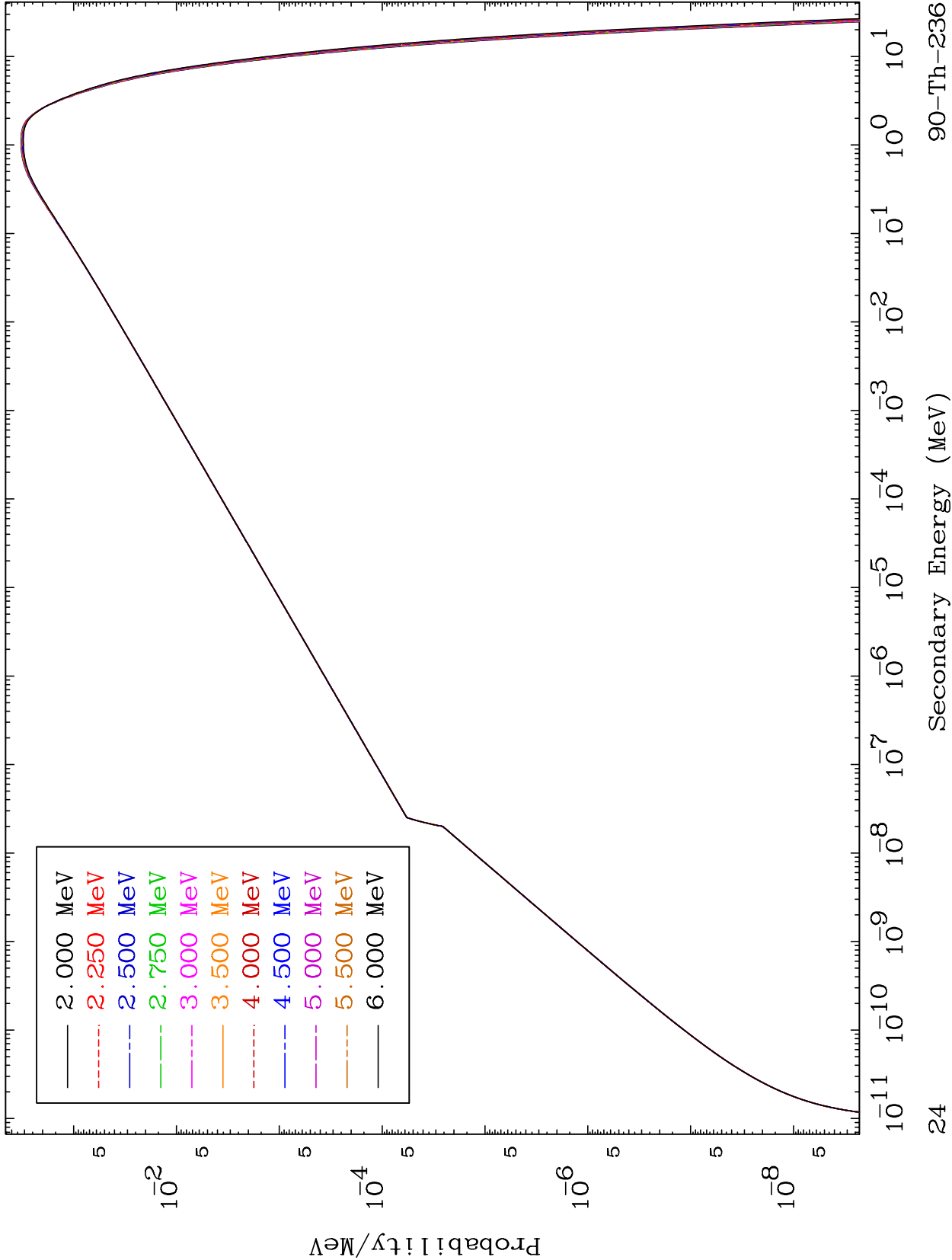


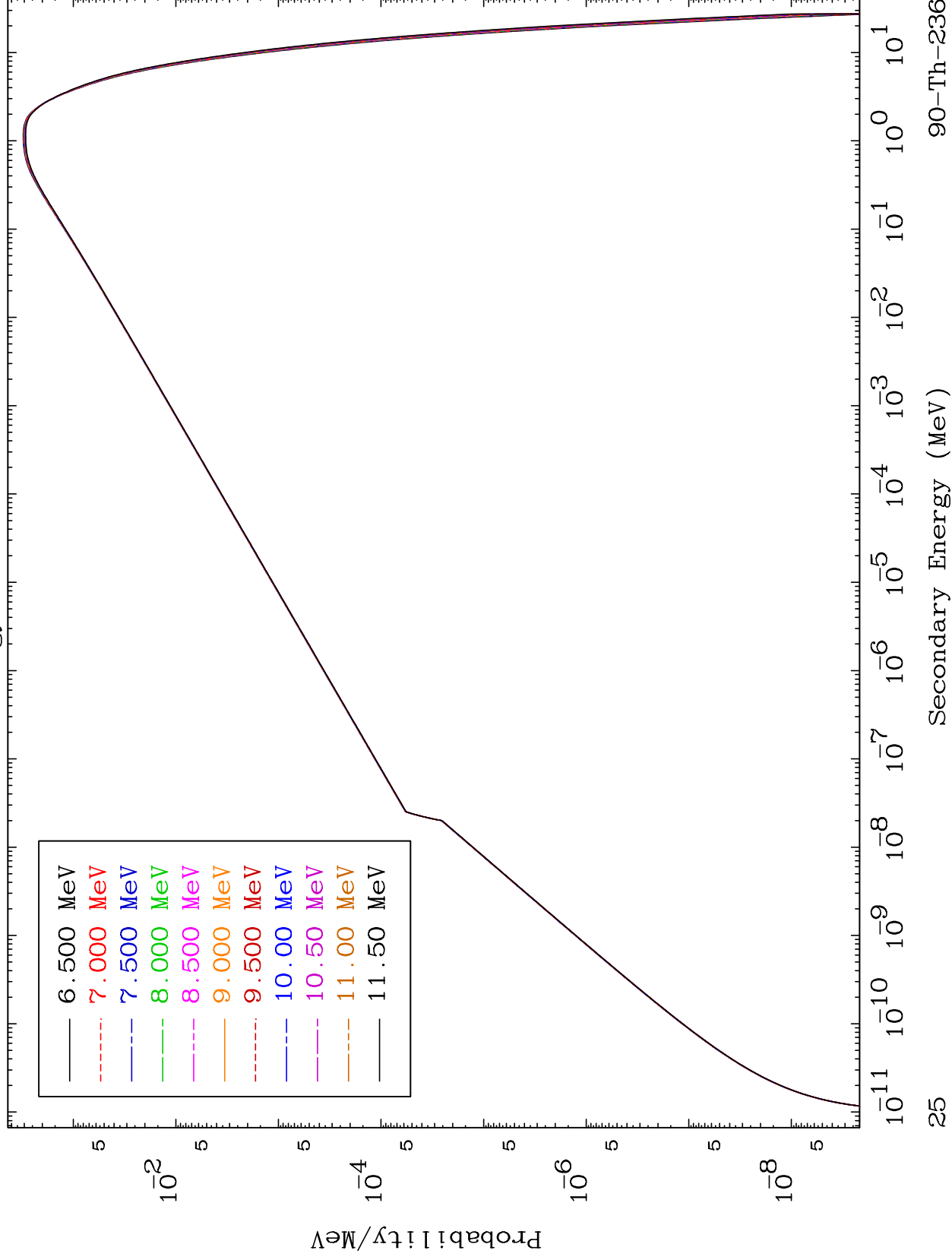






Fission
Energy Distributions

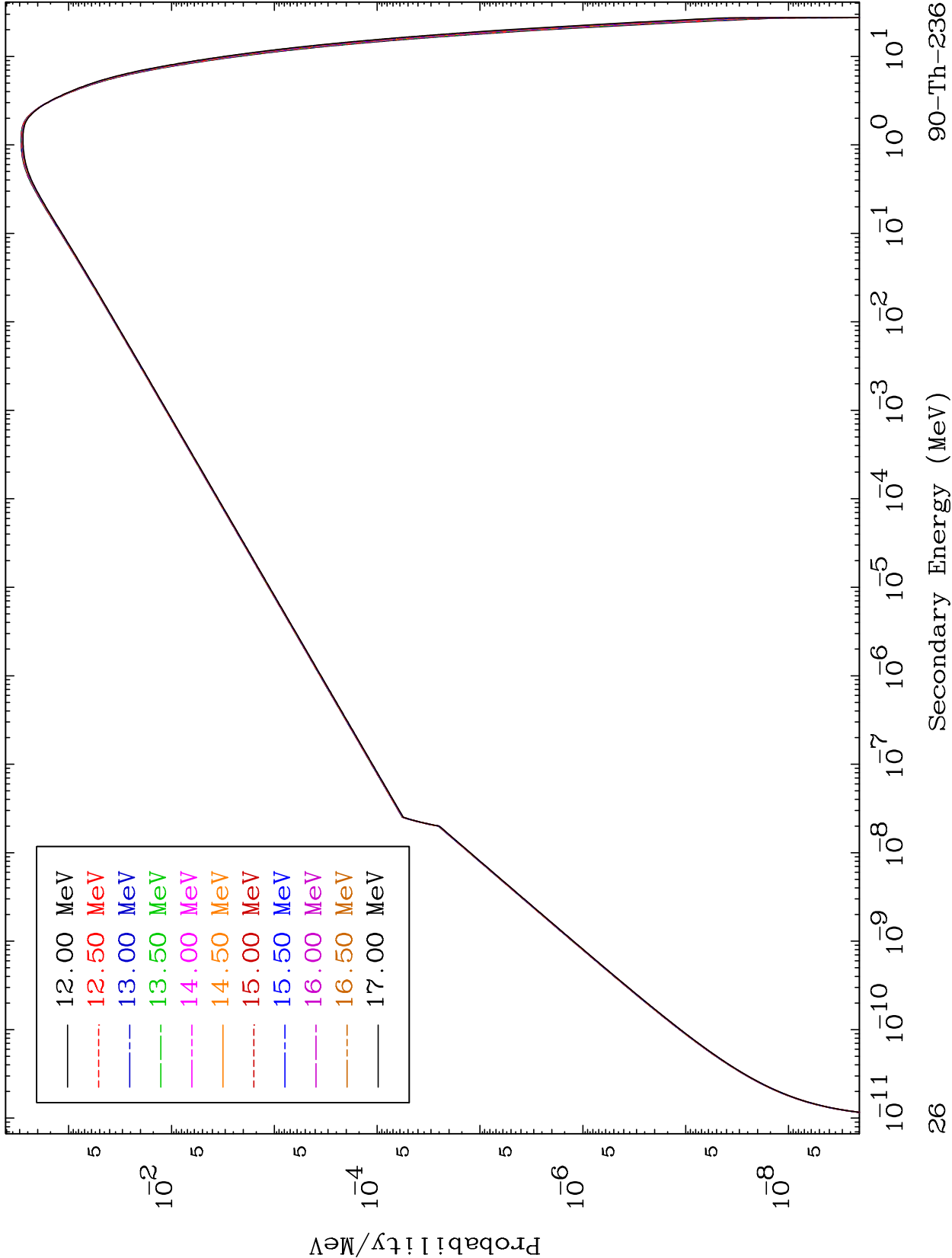




MAT 9052

Fission
Energy Distributions

90-Th-236



MAT 9052

Fission
Energy Distributions

90-Th-236

